

Program EVALPLOT
(Version 2021-1)

by

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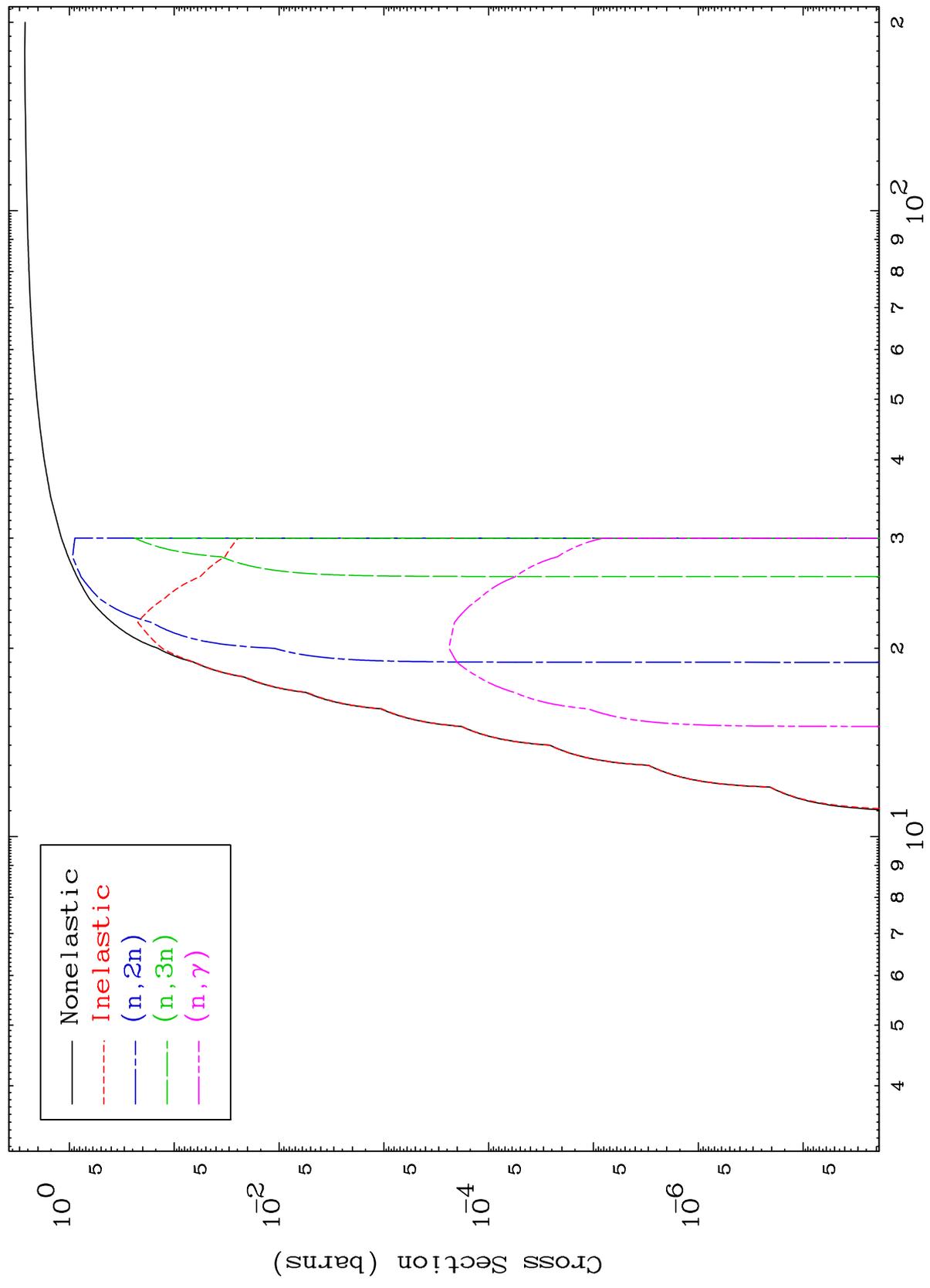
Press Mouse Button to Start

MAT 7523

0 Kelvin

α Major Cross Sections

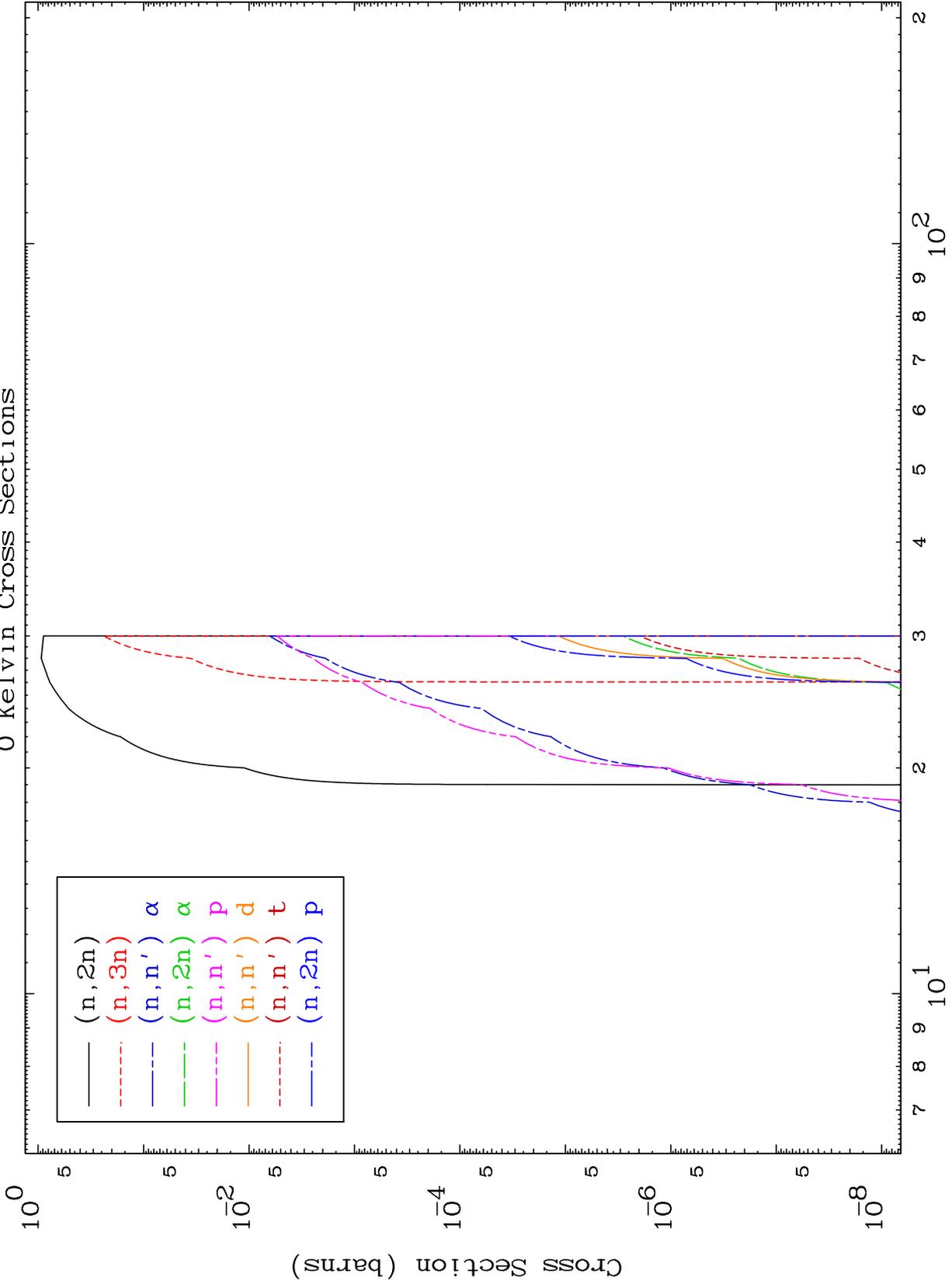
75-Re-184m



MAT 7523

α Neutron Absorption
0 Kelvin Cross Sections

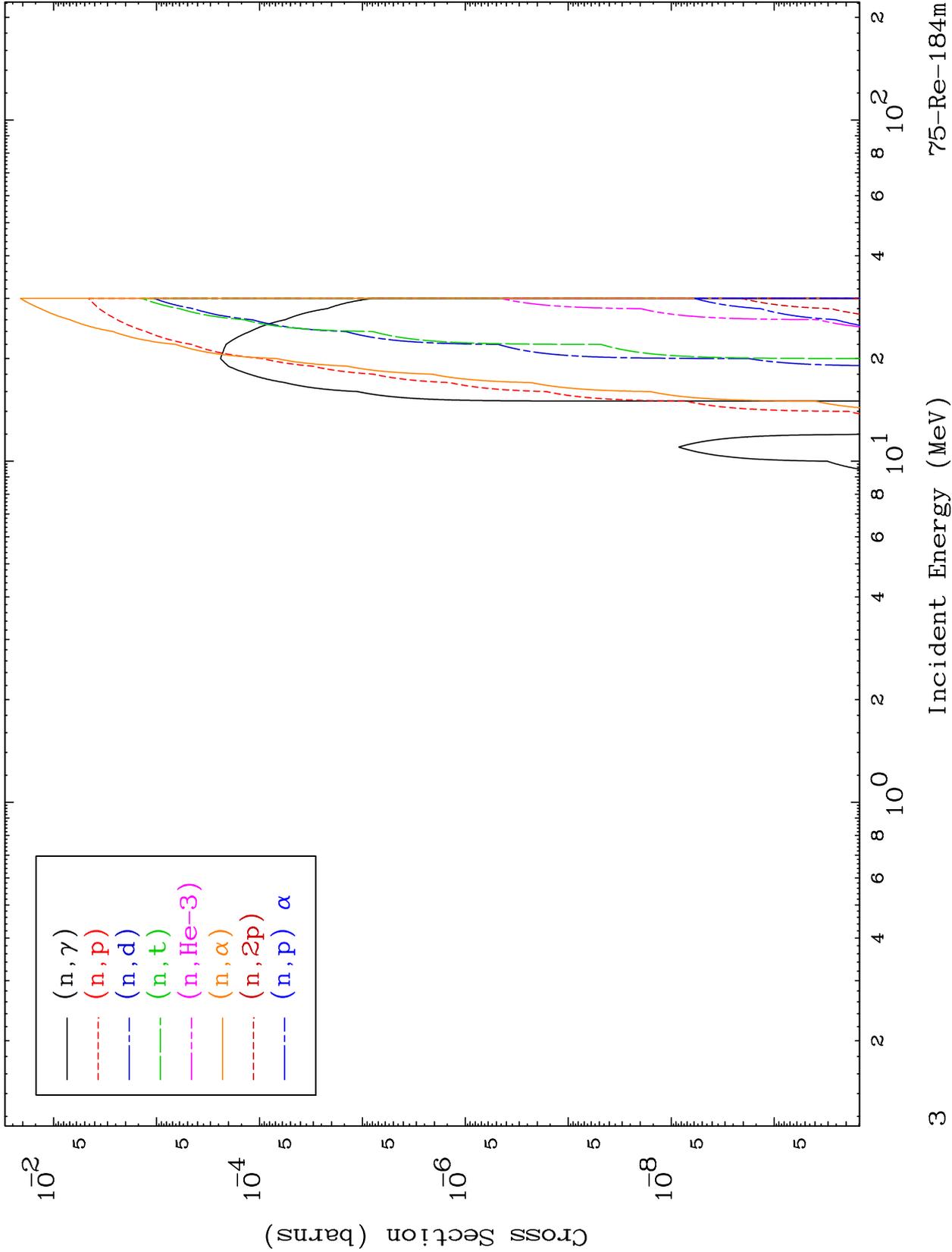
75-Re-184m

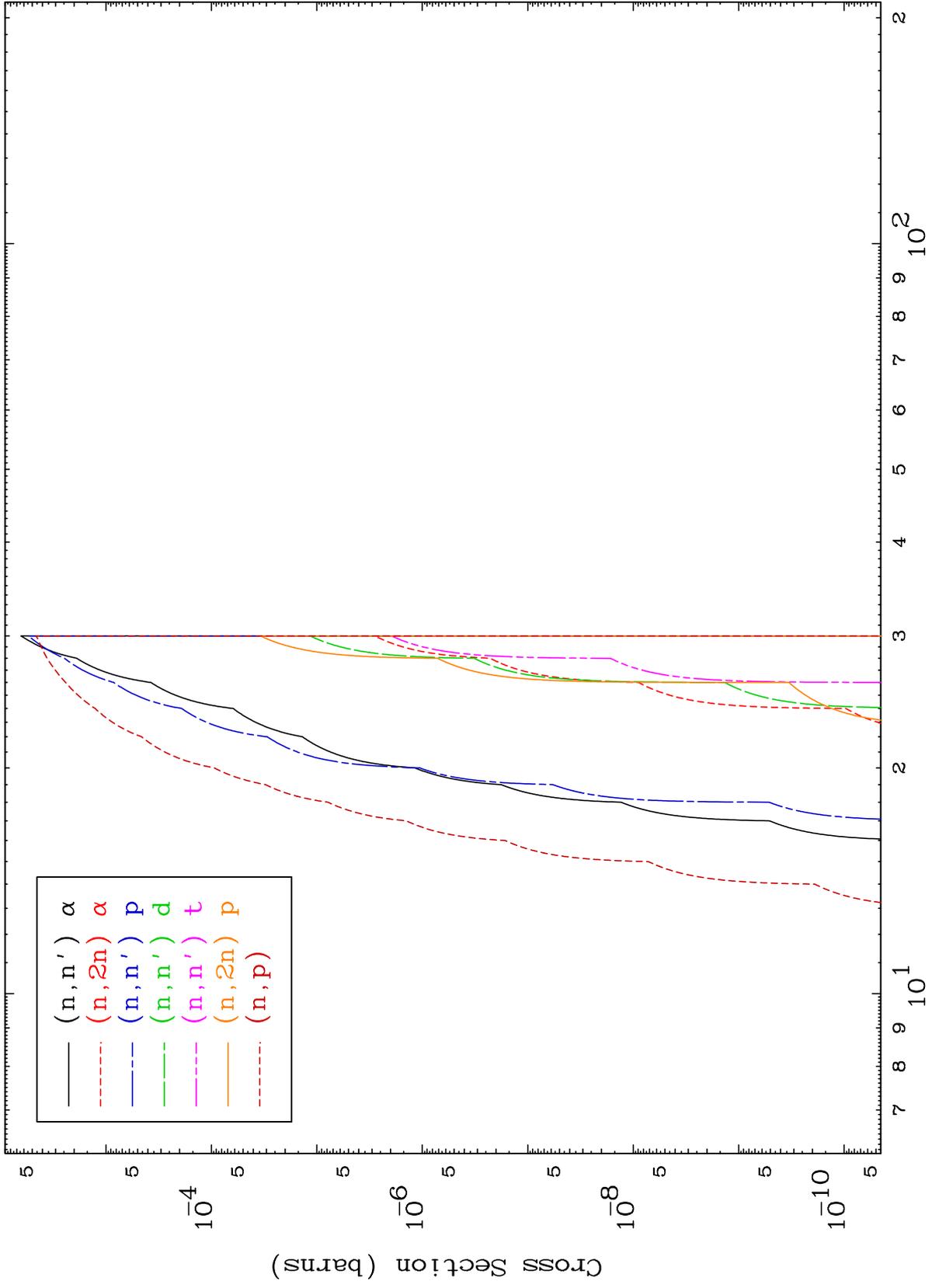


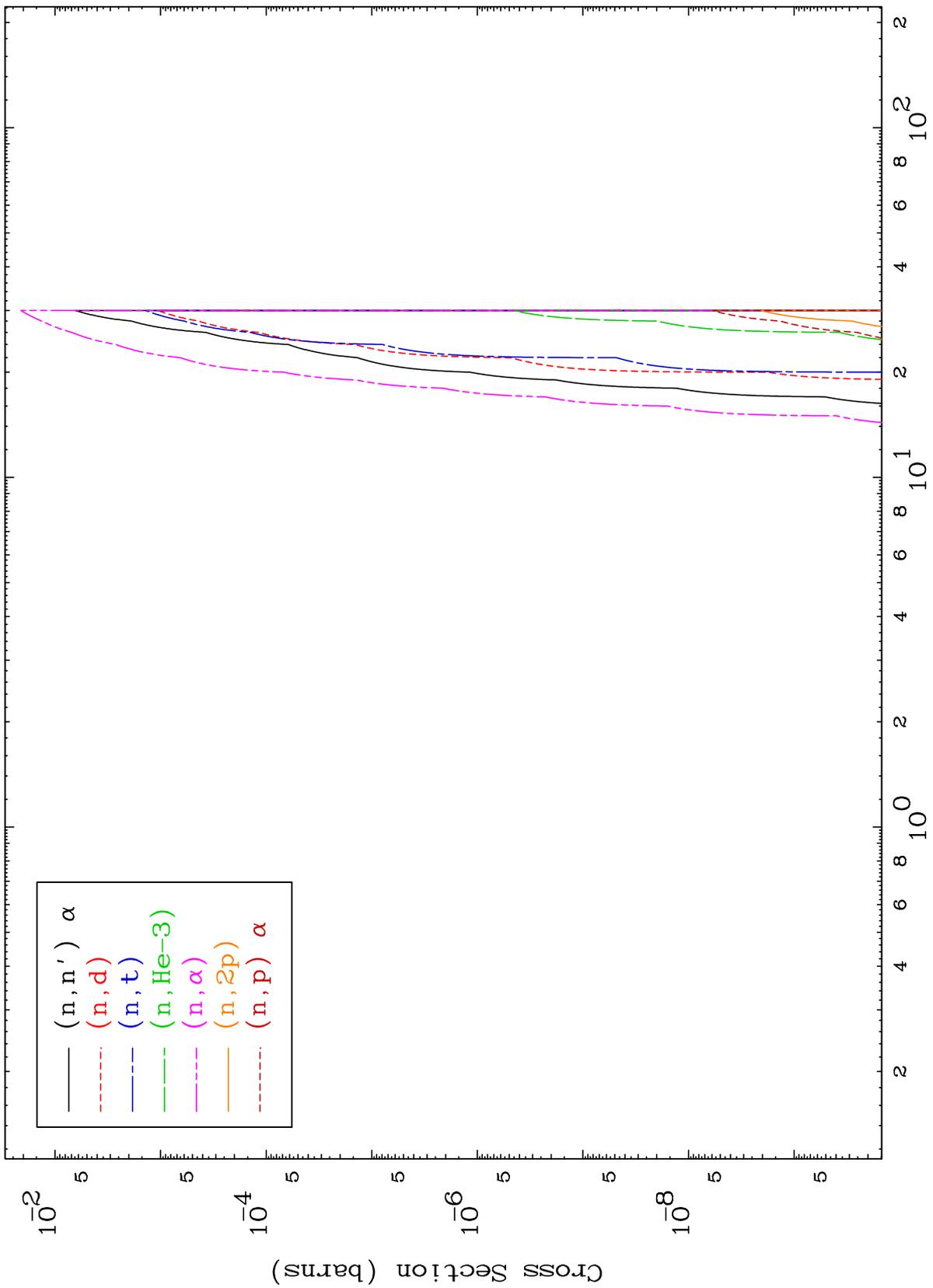
MAT 7523

α Neutron Absorption
0 Kelvin Cross Sections

⁷⁵Re-184m



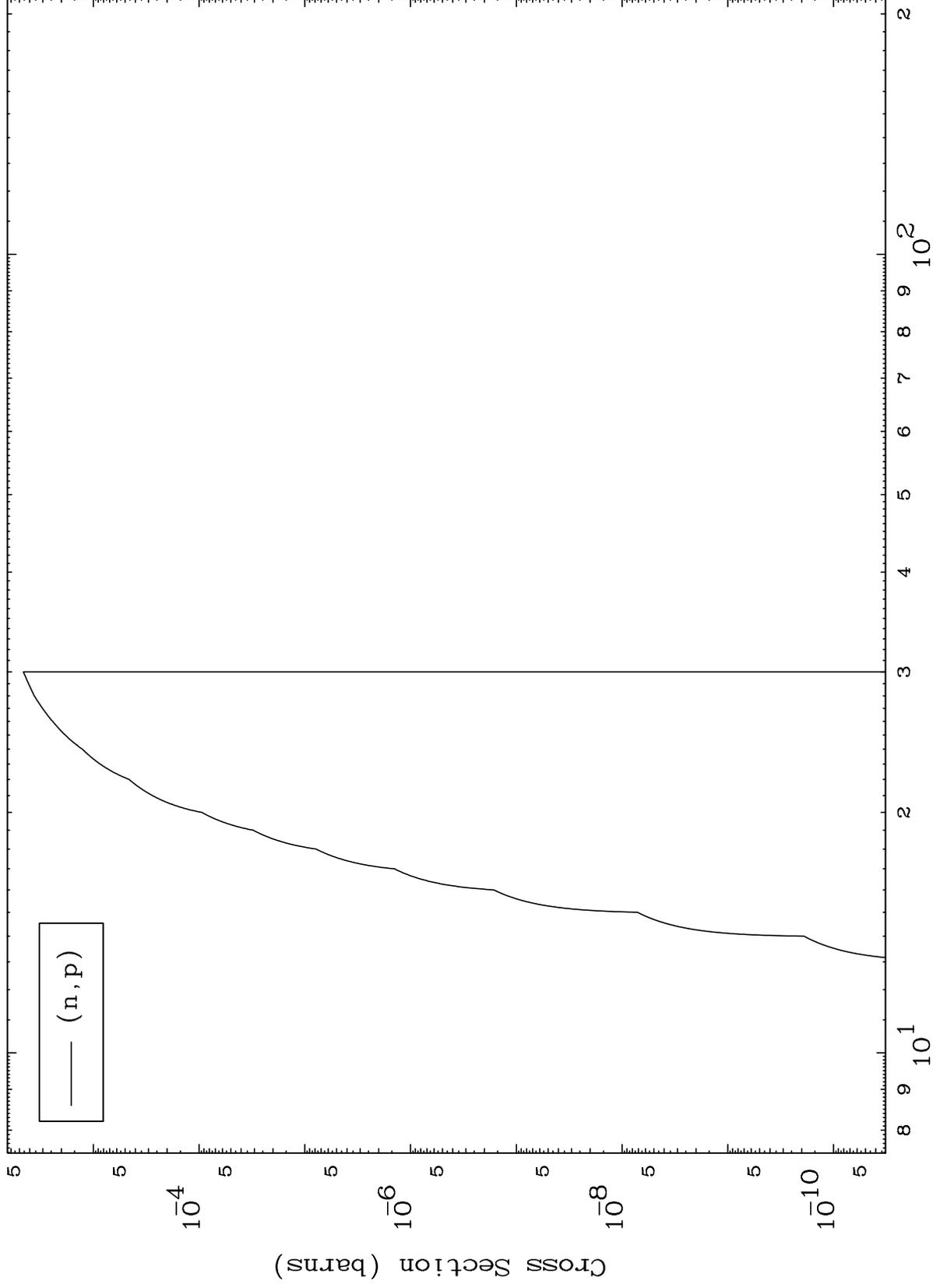




MAT 7523

(α, p) Levels
0 Kelvin Cross Sections

⁷⁵Re-184m



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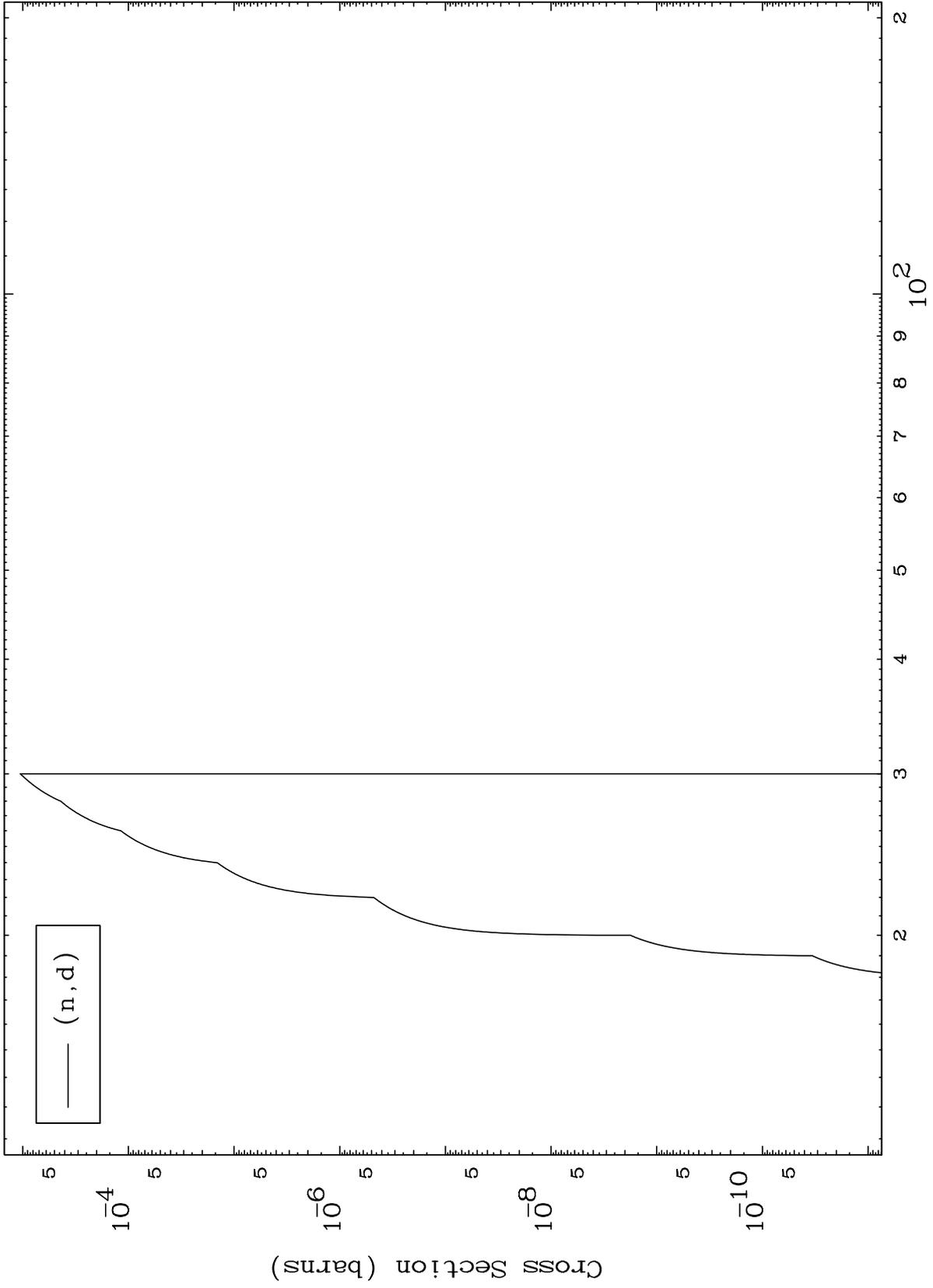
Incident Energy (MeV)

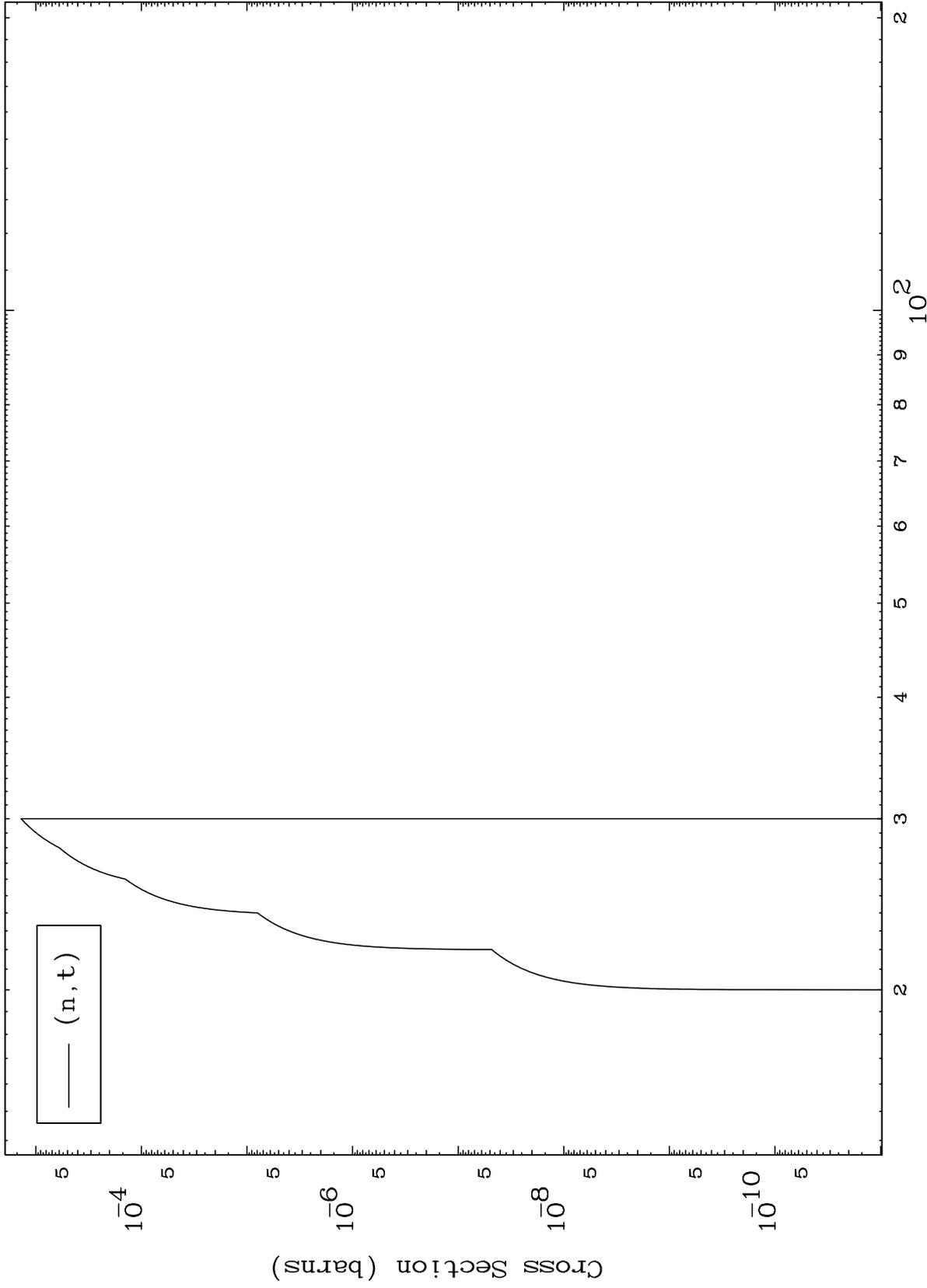
⁷⁵Re-184m

MAT 7523

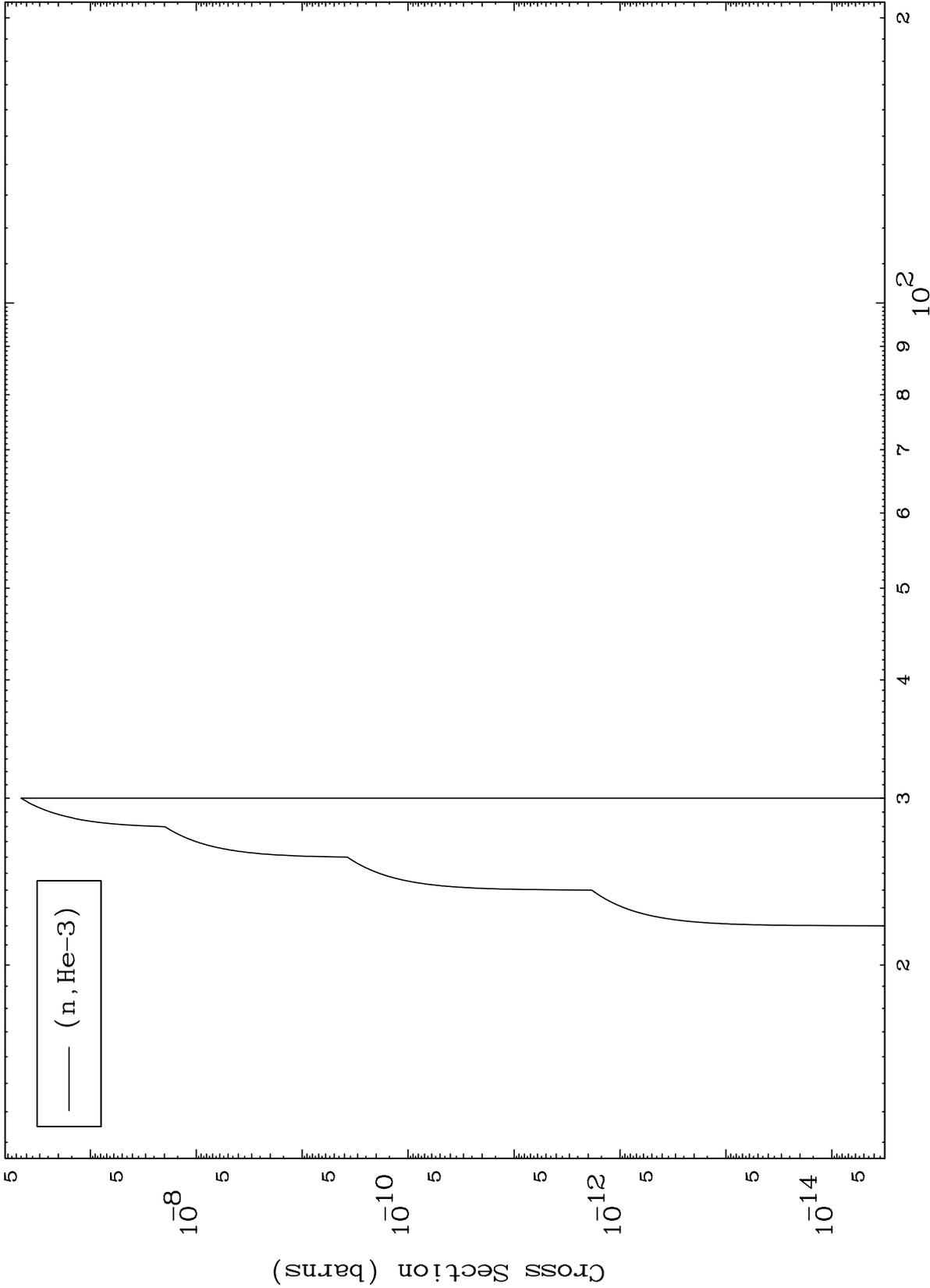
(α, d) Levels
0 Kelvin Cross Sections

⁷⁵Re-184m





0 Kelvin Cross Sections

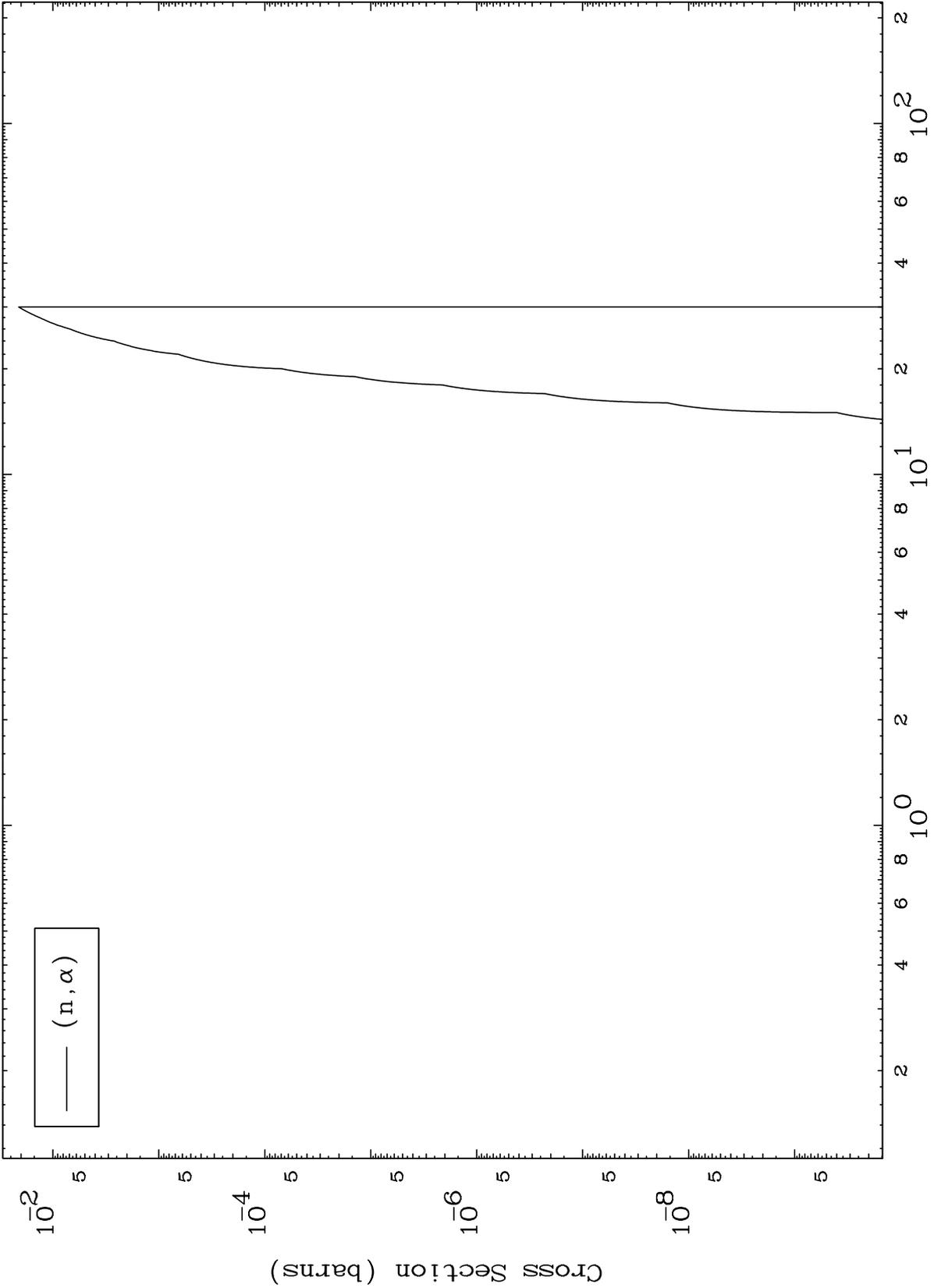


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(α, α) Levels

75-Re-184m

0 Kelvin Cross Sections



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Incident Energy (MeV)

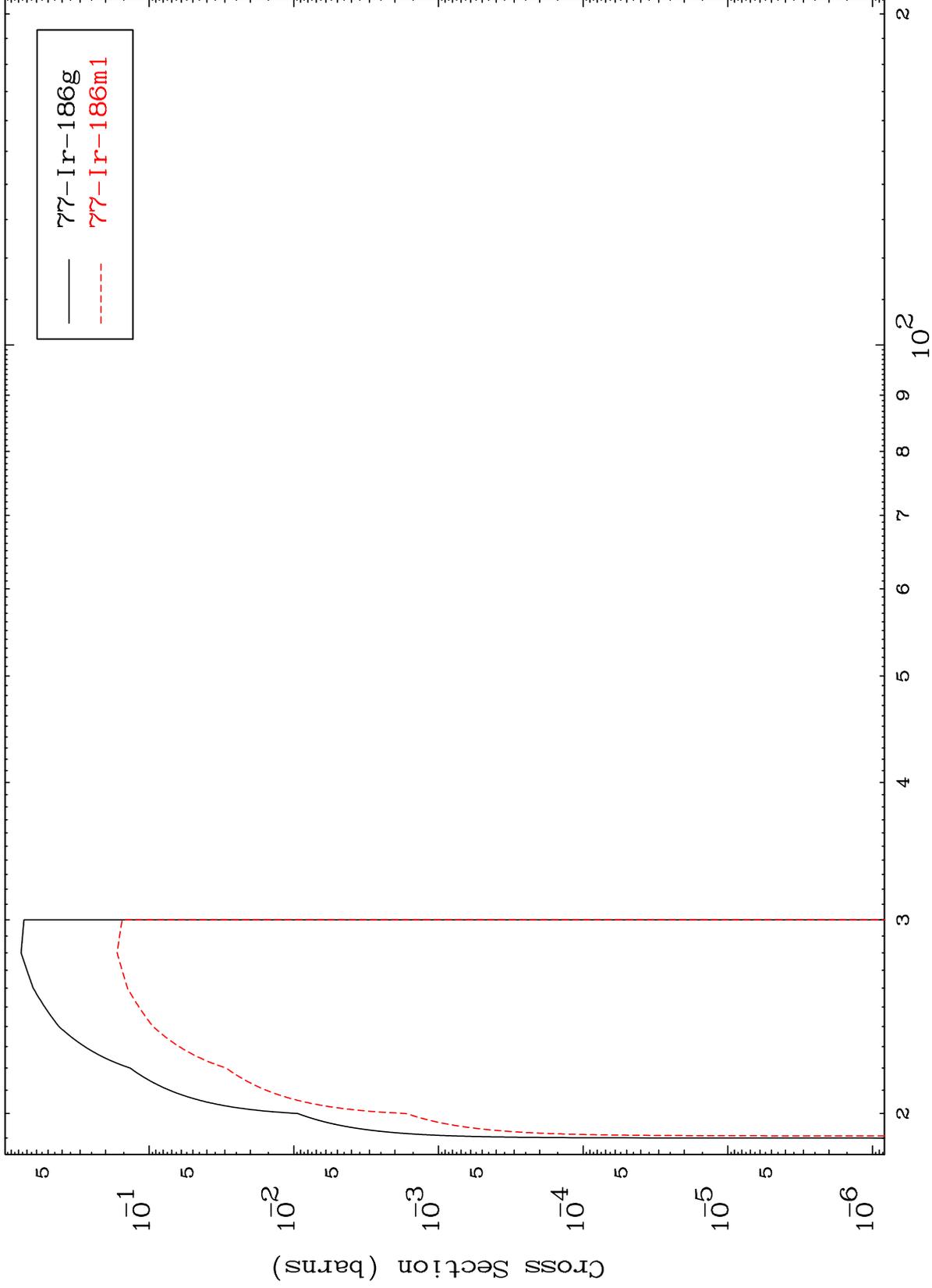
75-Re-184m

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(n,2n)

75-Re-184m

Radionuclide Production Cross Section



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Incident Energy (MeV)

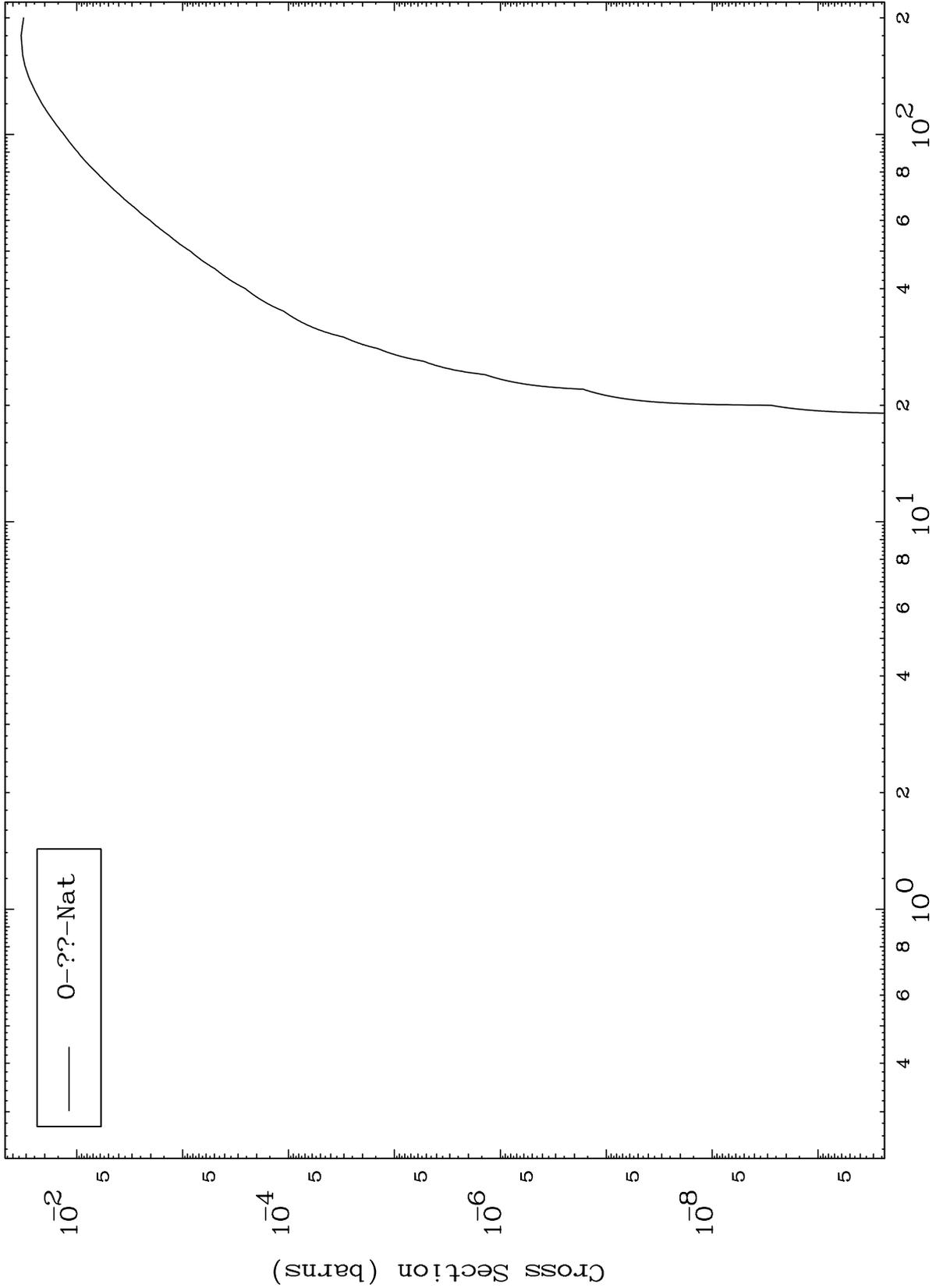
75-Re-184m

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Fission

⁷⁵Re-184m

Radionuclide Production Cross Section



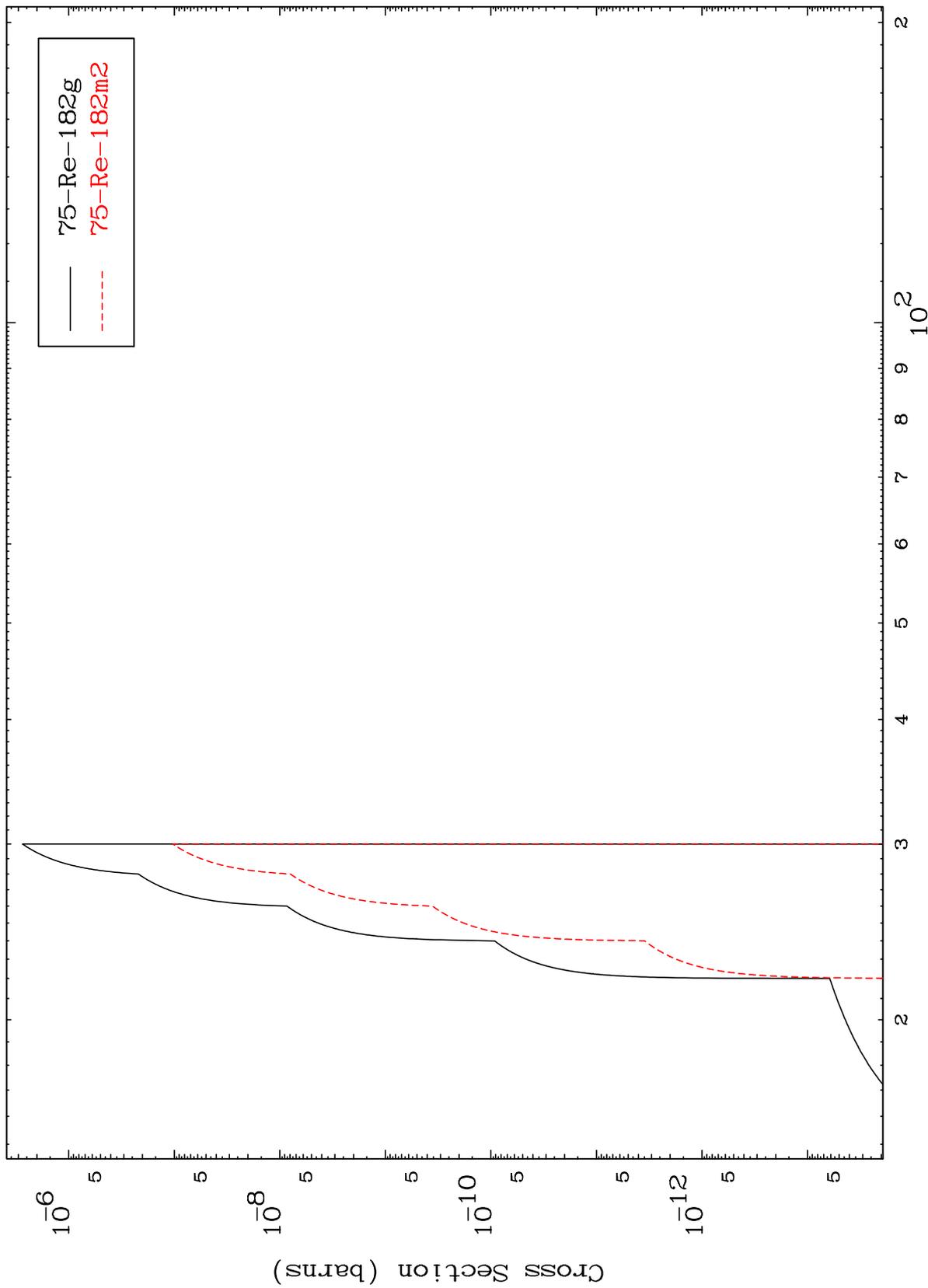
— 0-??-Nat

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$(n,2n) \alpha$

⁷⁵Re-184m

Radionuclide Production Cross Section



13

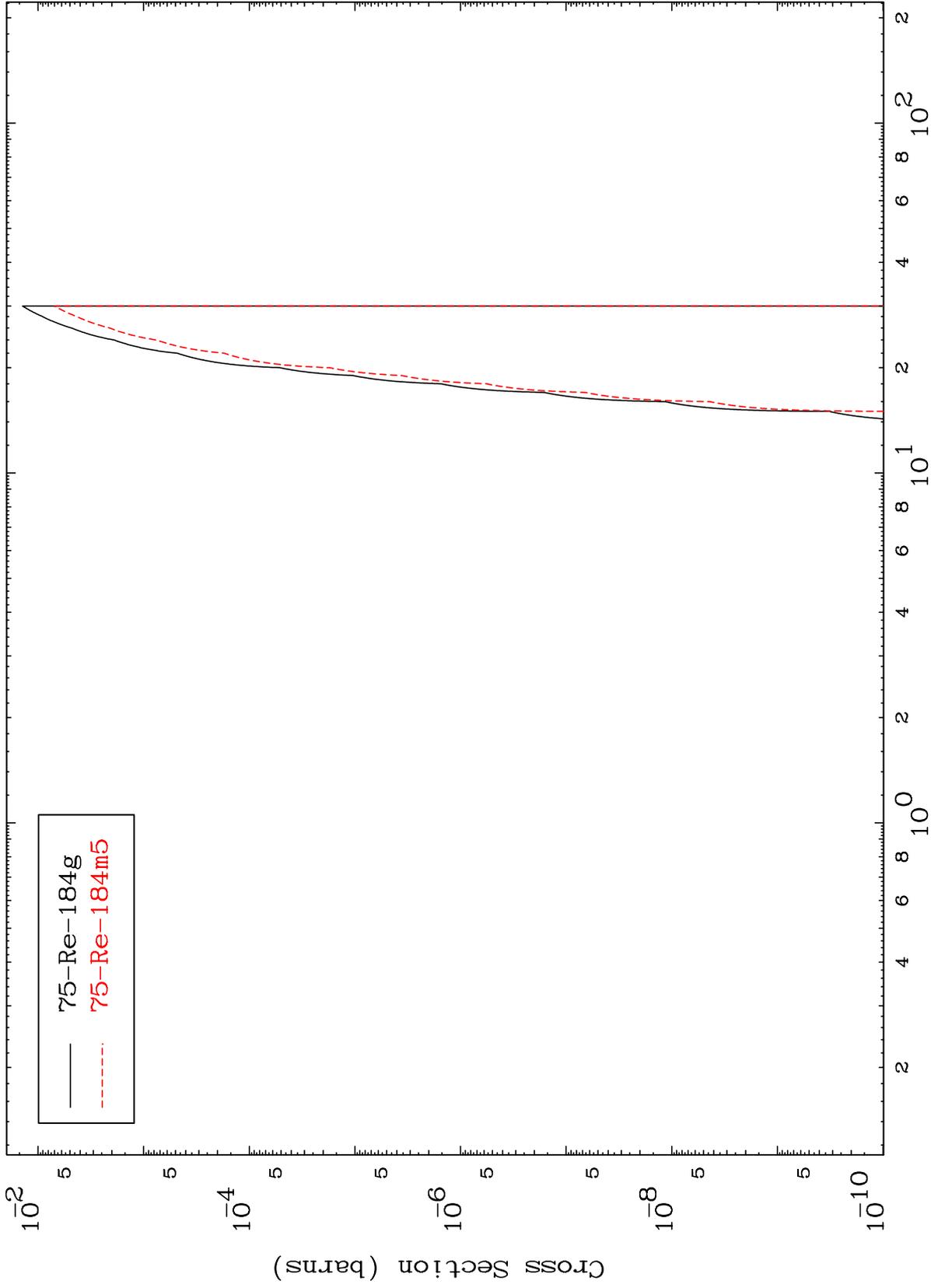
Incident Energy (MeV)

⁷⁵Re-184m

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⁷⁵Re-184m

Radionuclide Production Cross Section



14

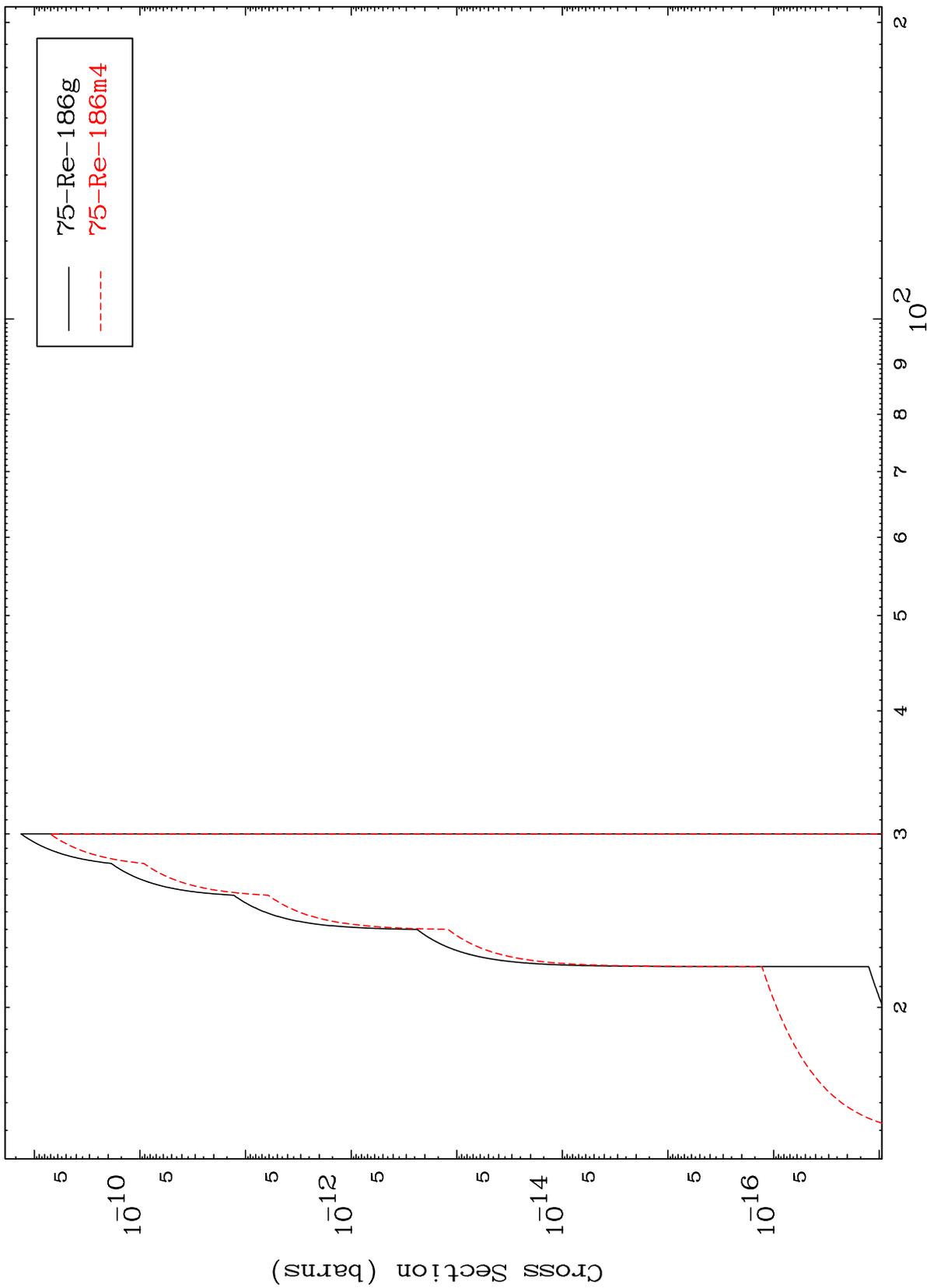
Incident Energy (MeV)

⁷⁵Re-184m

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⁷⁵Re-184m

(n,2p)
Radionuclide Production Cross Section



⁷⁵Re-184m

Incident Energy (MeV)

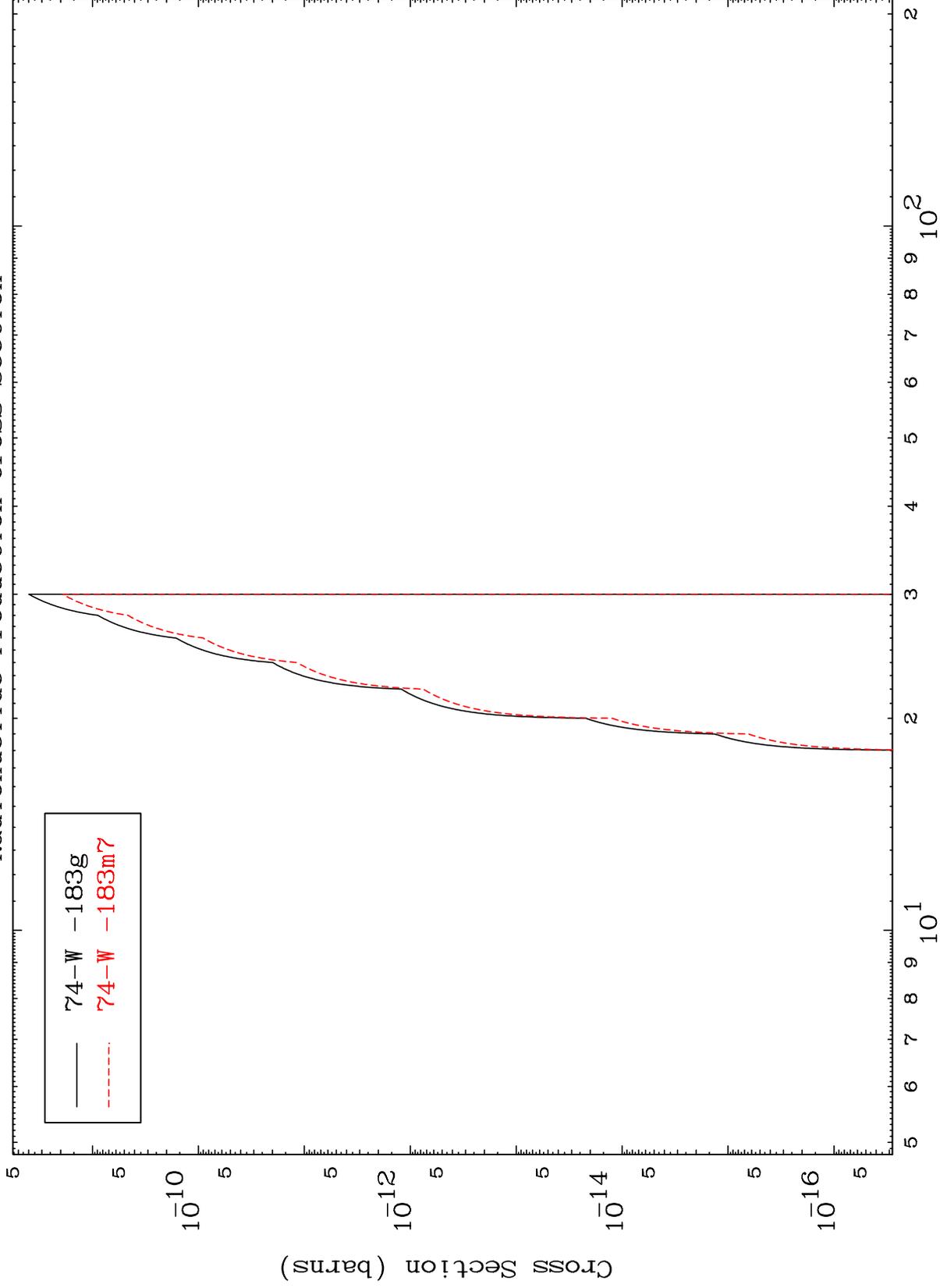
15

MAT 7523

(n,p) α

⁷⁵Re-184m

Radionuclide Production Cross Section



16

Incident Energy (MeV)

⁷⁵Re-184m