

Program EVALPLOT  
(Version 2021-1)

by

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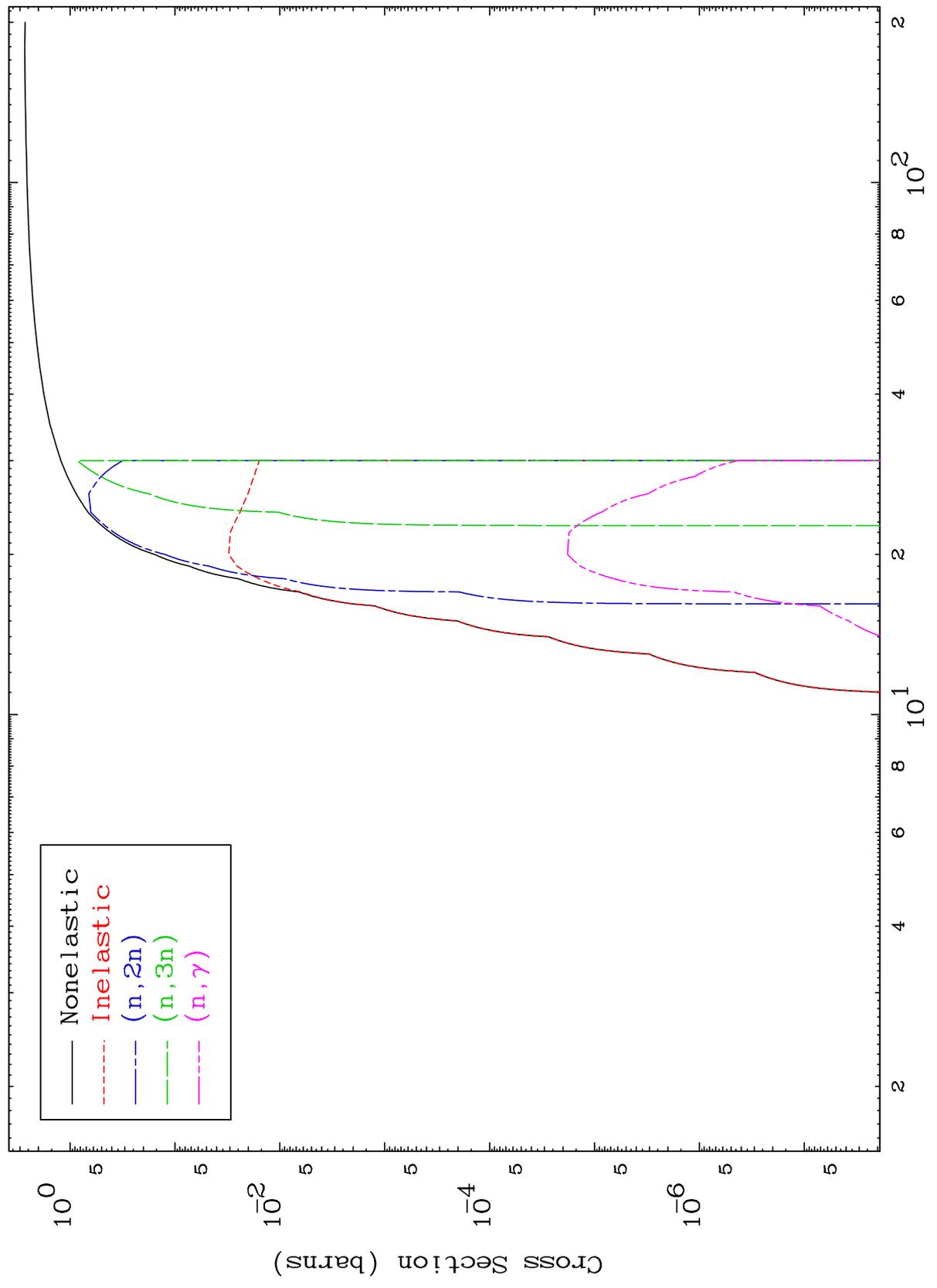
Press Mouse Button to Start

MAT 7535

0 Kelvin

$\alpha$  Major Cross Sections

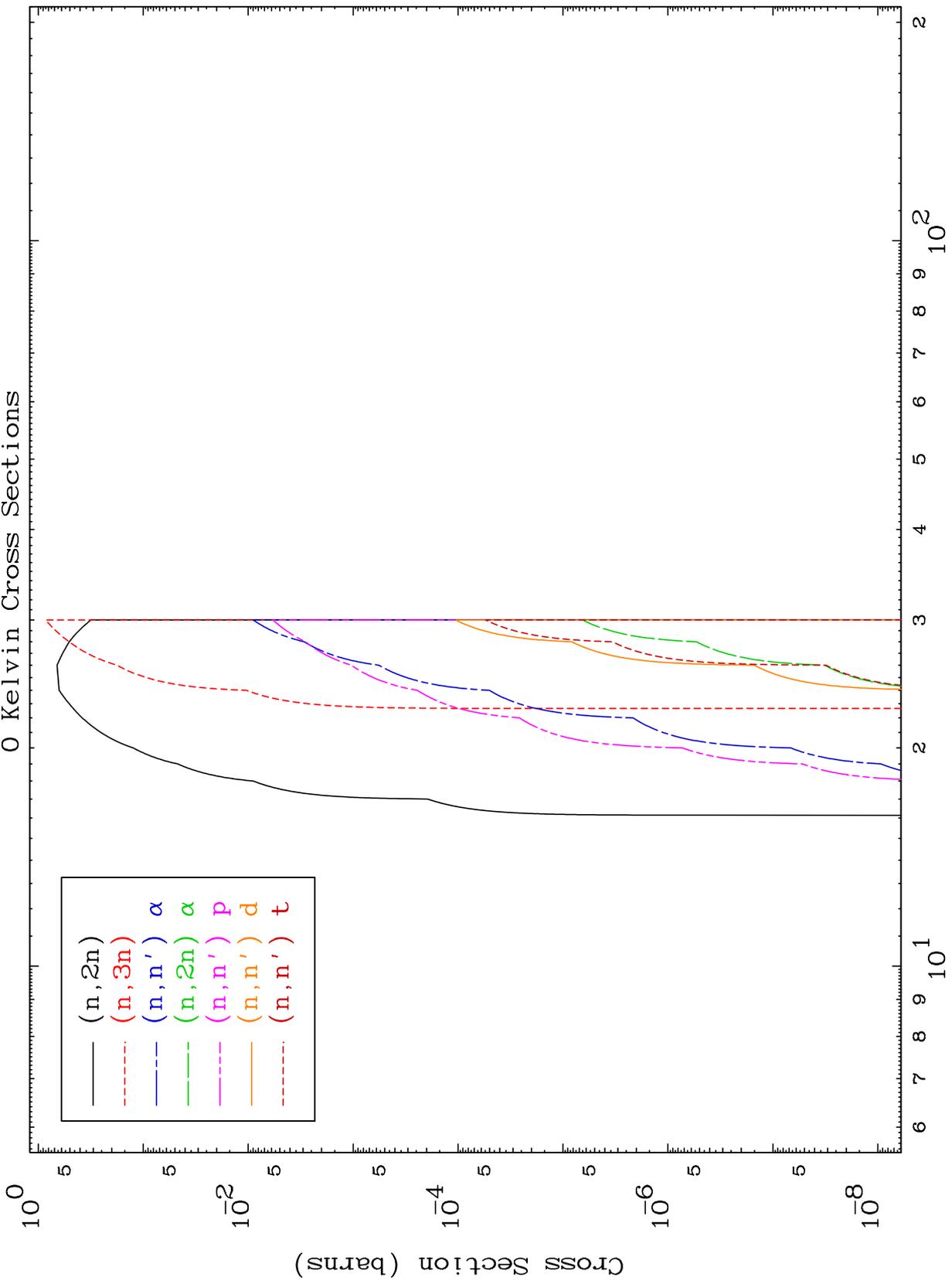
<sup>75</sup>Re-188m



MAT 7535

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

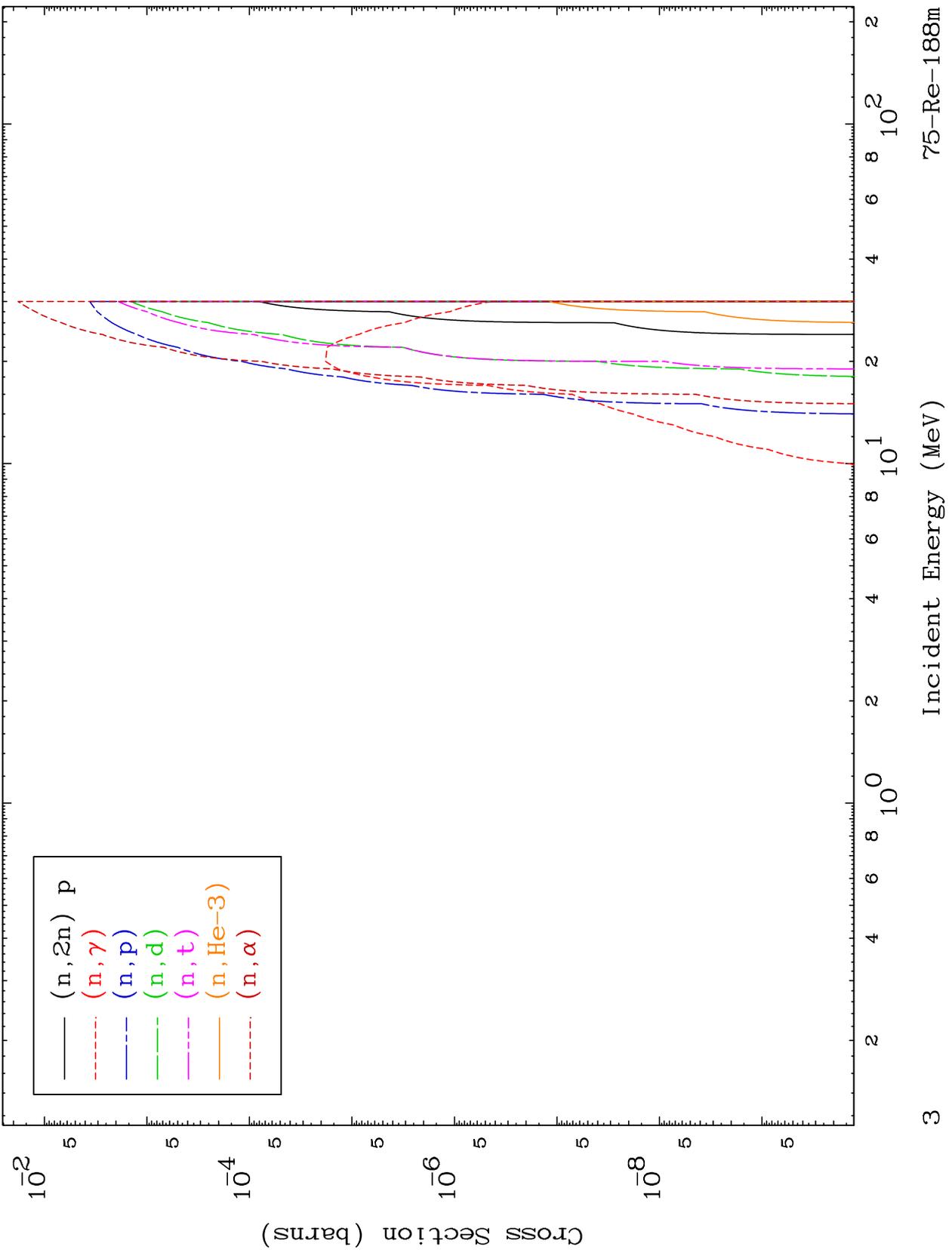
75-Re-188m



MAT 7535

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

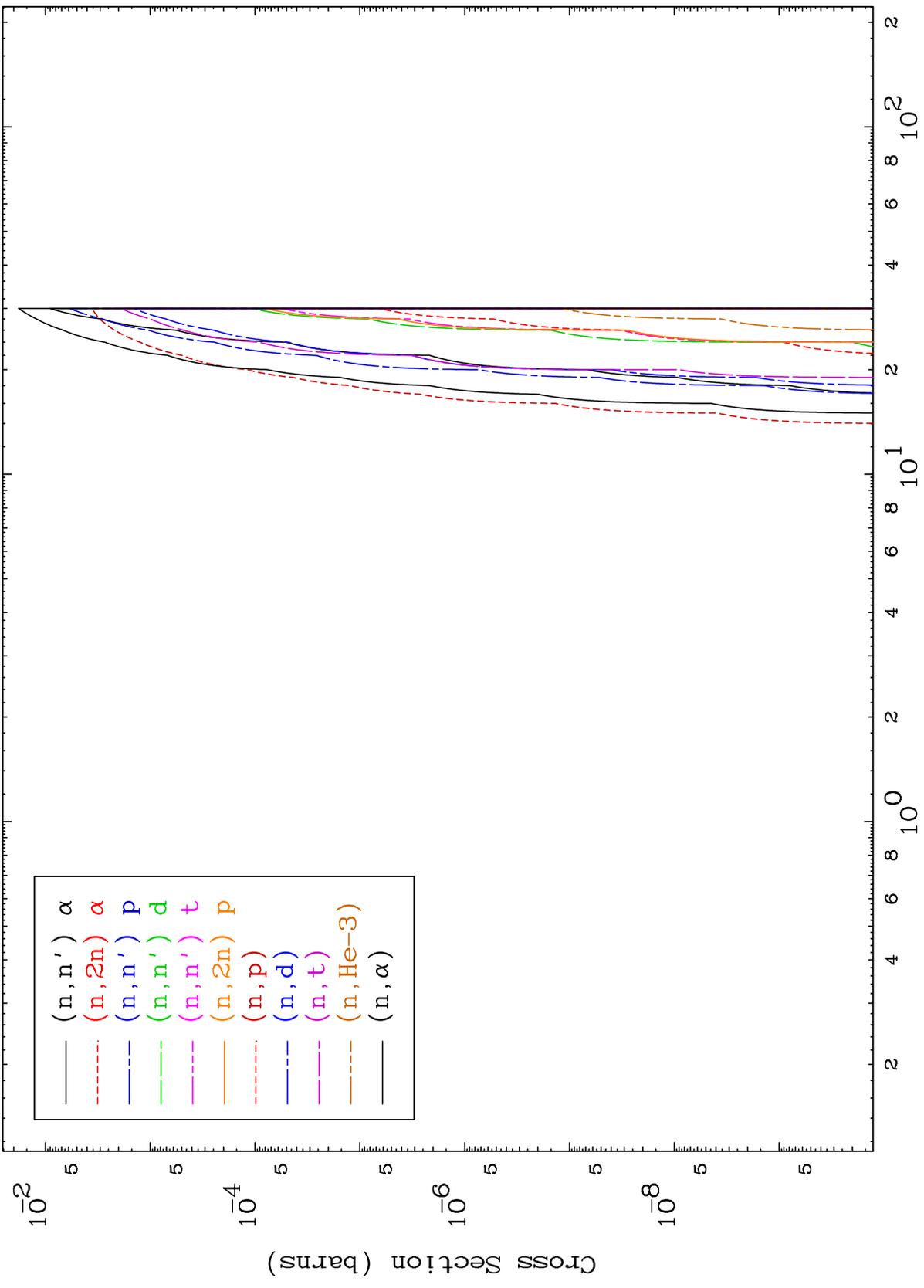
<sup>75</sup>Re-188m



MAT 7535

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

75-Re-188m

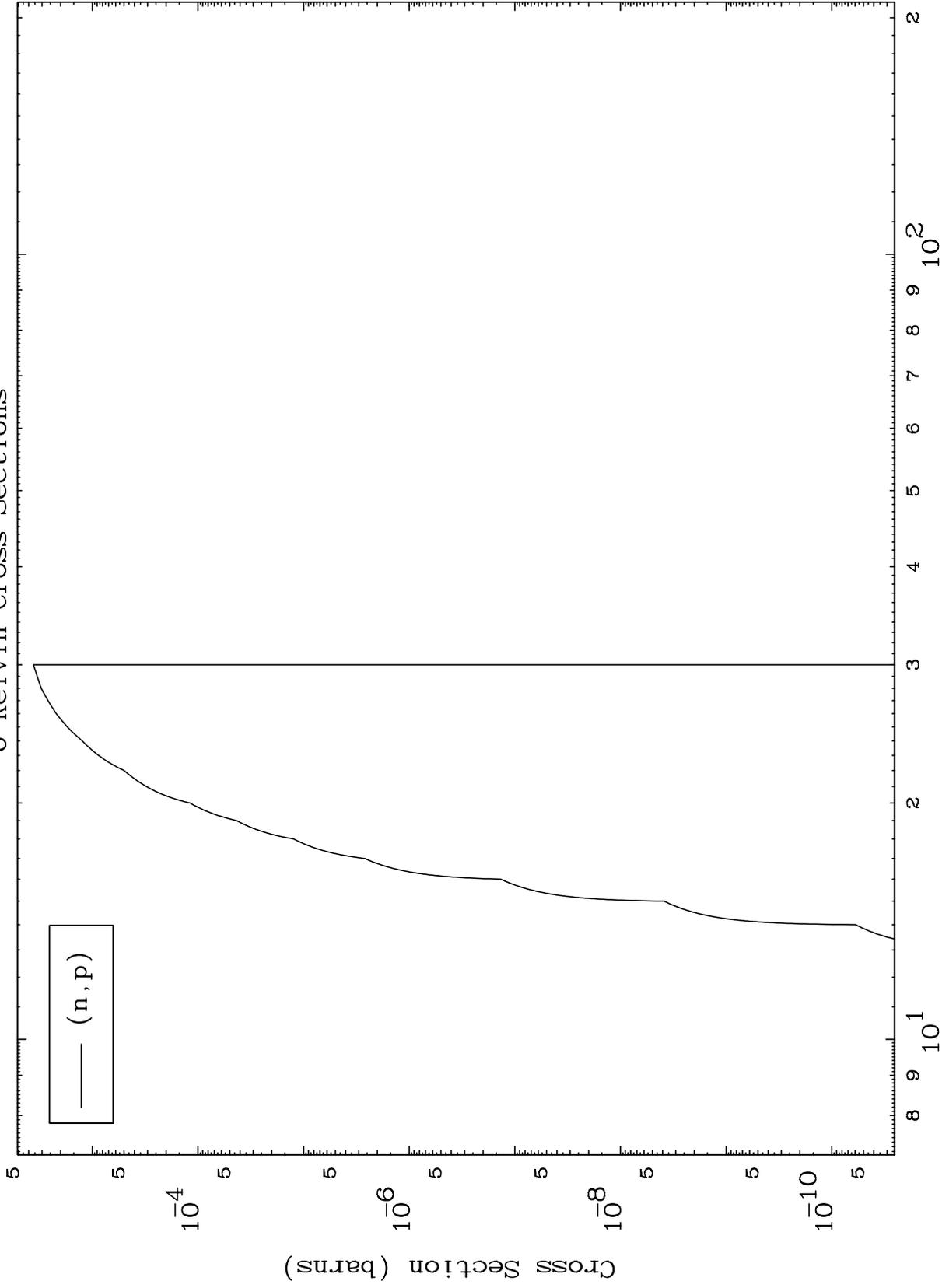


MAT 7535

( $\alpha, p$ ) Levels

75-Re-188m

0 Kelvin Cross Sections



5

Incident Energy (MeV)

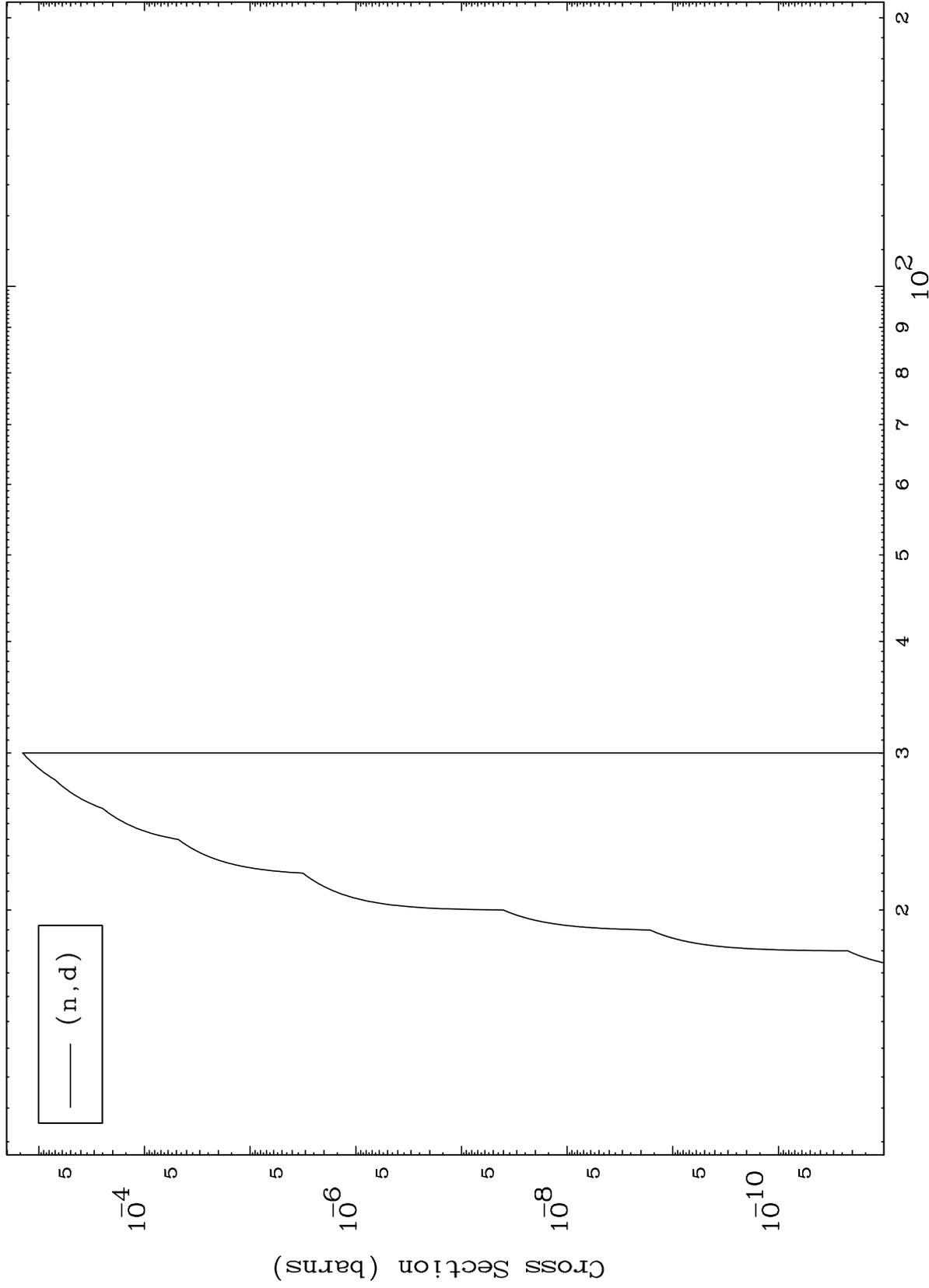
75-Re-188m

MAT 7535

( $\alpha, d$ ) Levels

75-Re-188m

0 Kelvin Cross Sections



6

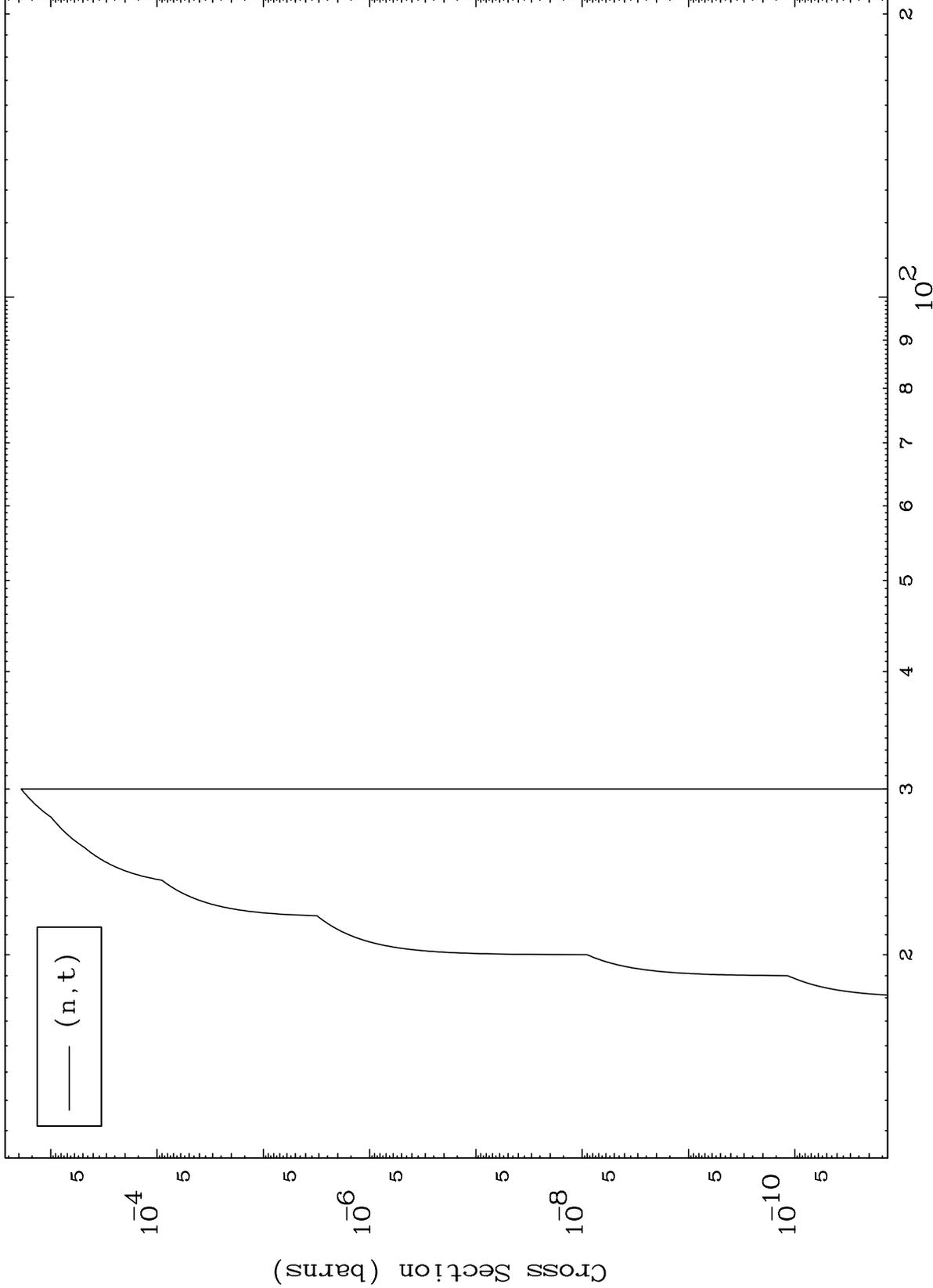
Incident Energy (MeV)

75-Re-188m

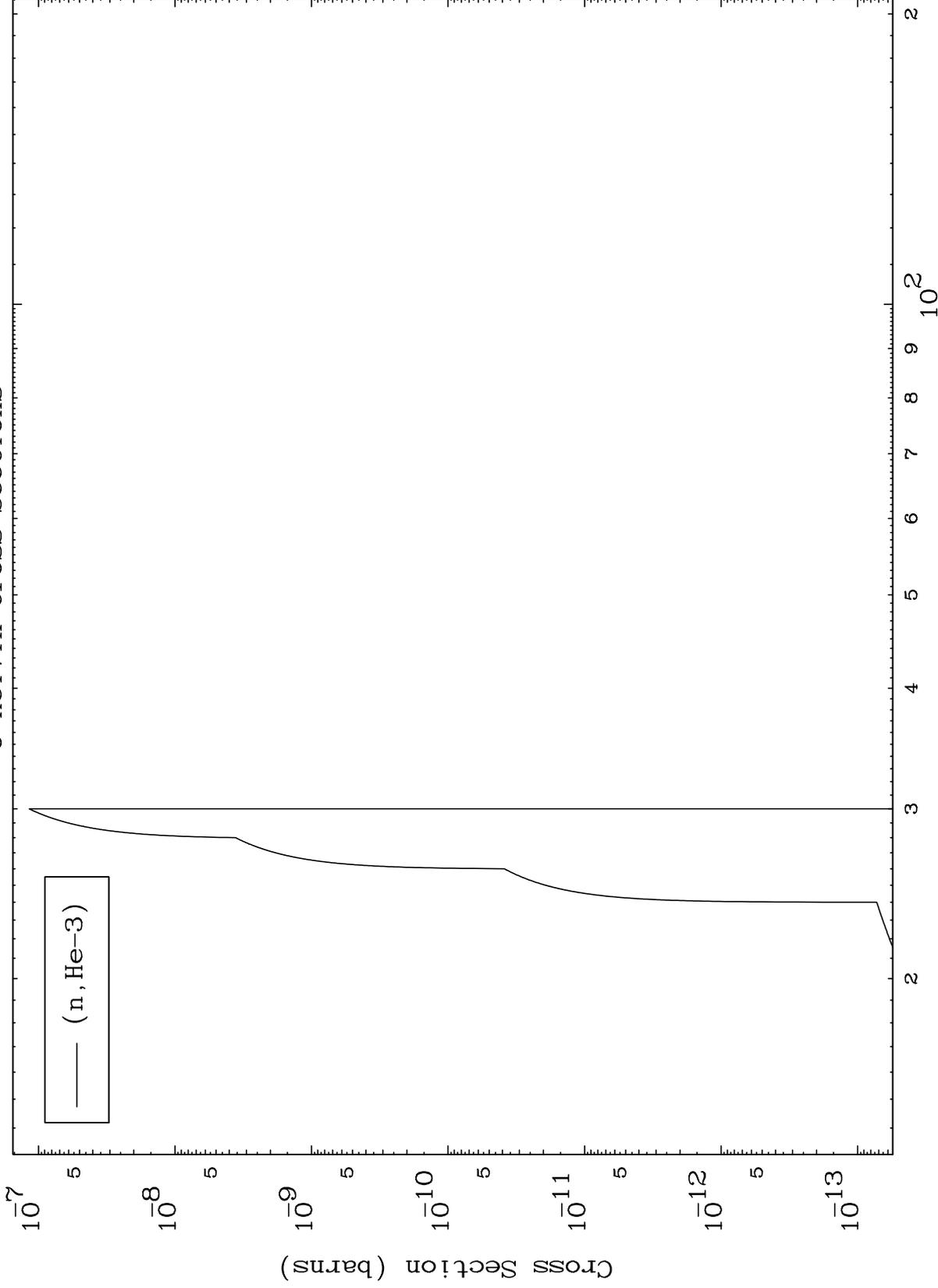
MAT 7535

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

<sup>75</sup>Re-188m



0 Kelvin Cross Sections

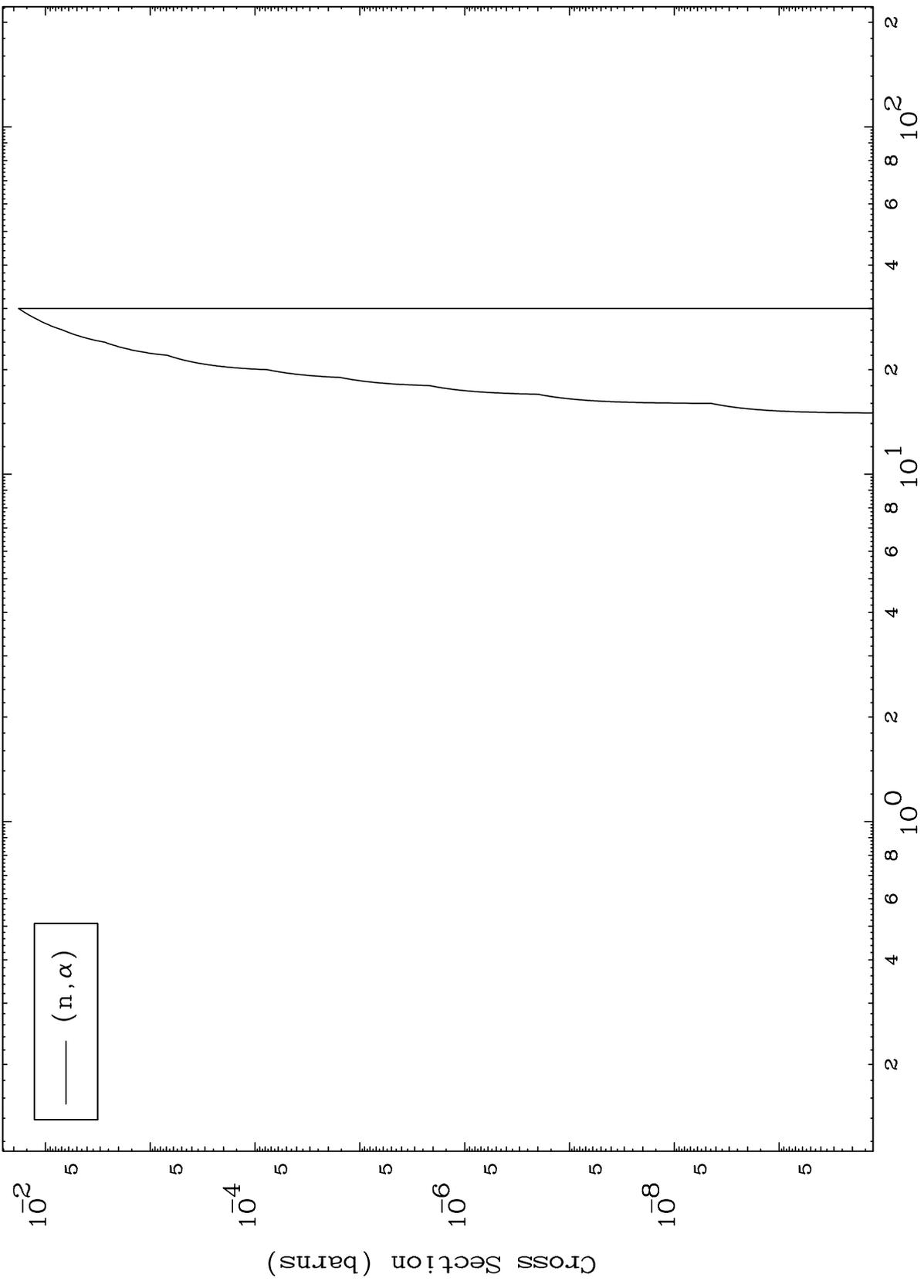


MAT 7535

( $\alpha, \alpha$ ) Levels

75-Re-188m

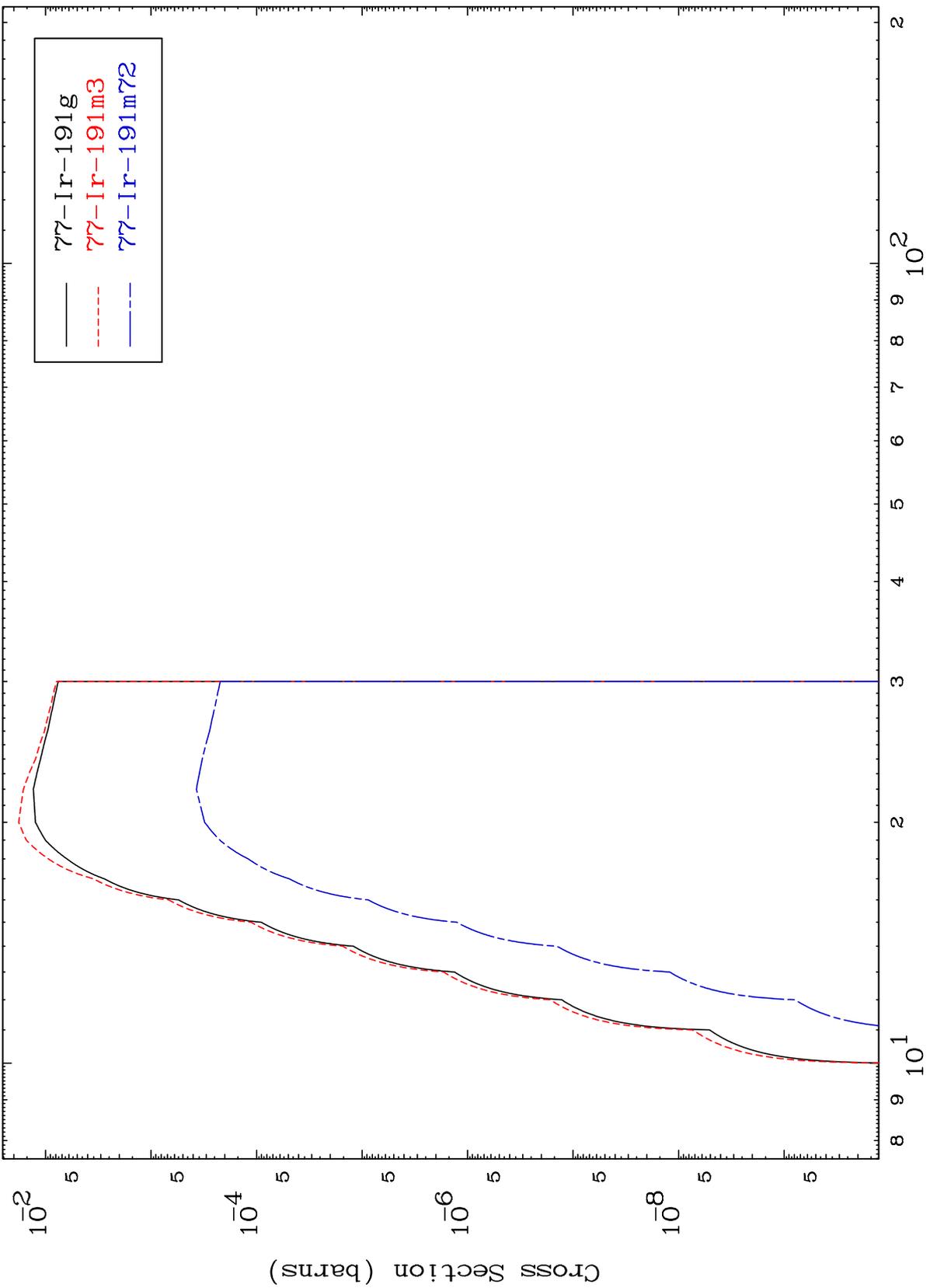
0 Kelvin Cross Sections



MAT 7535

75-Re-188m

Inelastic  
Radionuclide Production Cross Section



75-Re-188m

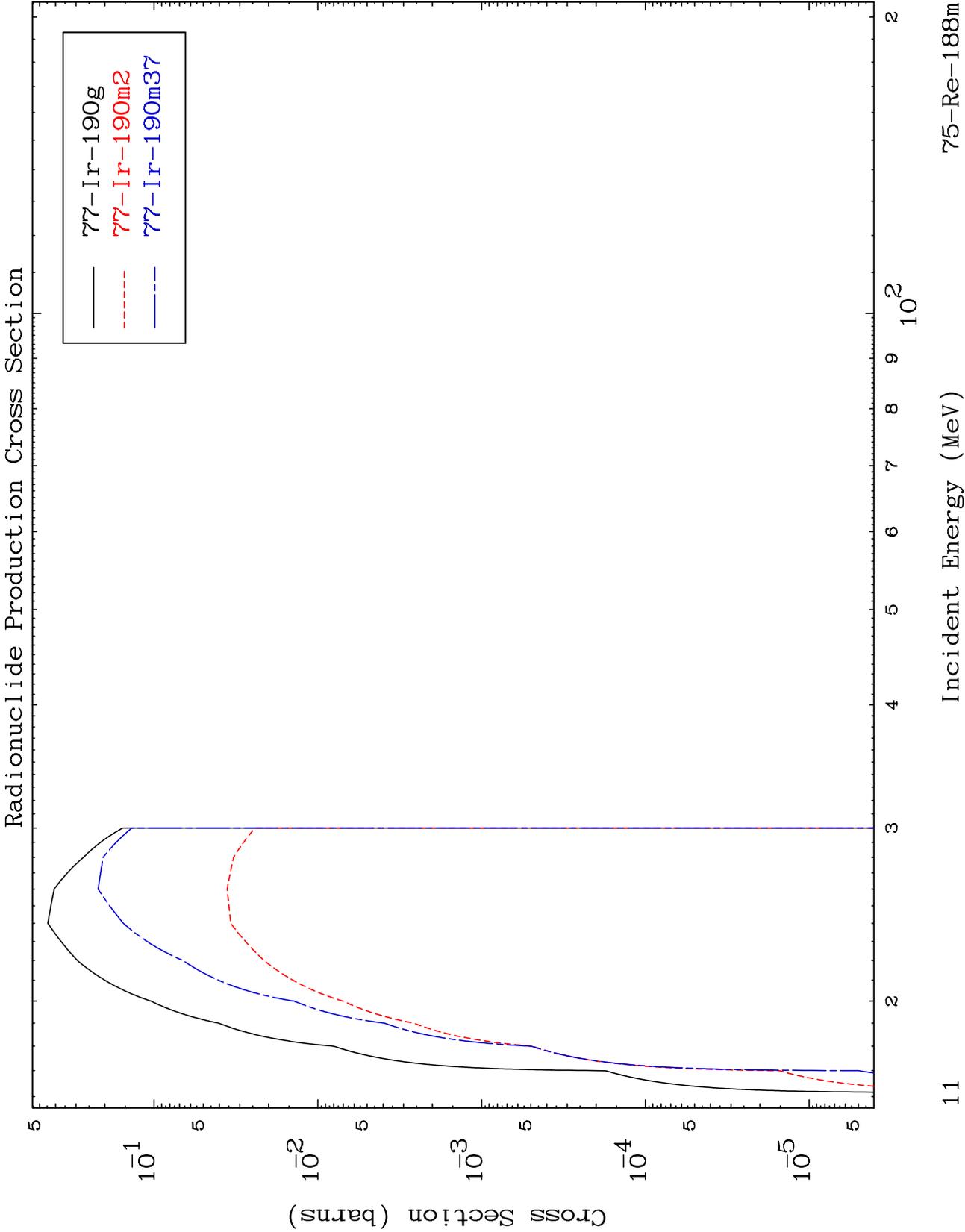
Incident Energy (MeV)

10

MAT 7535

(n,2n)

75-Re-188m

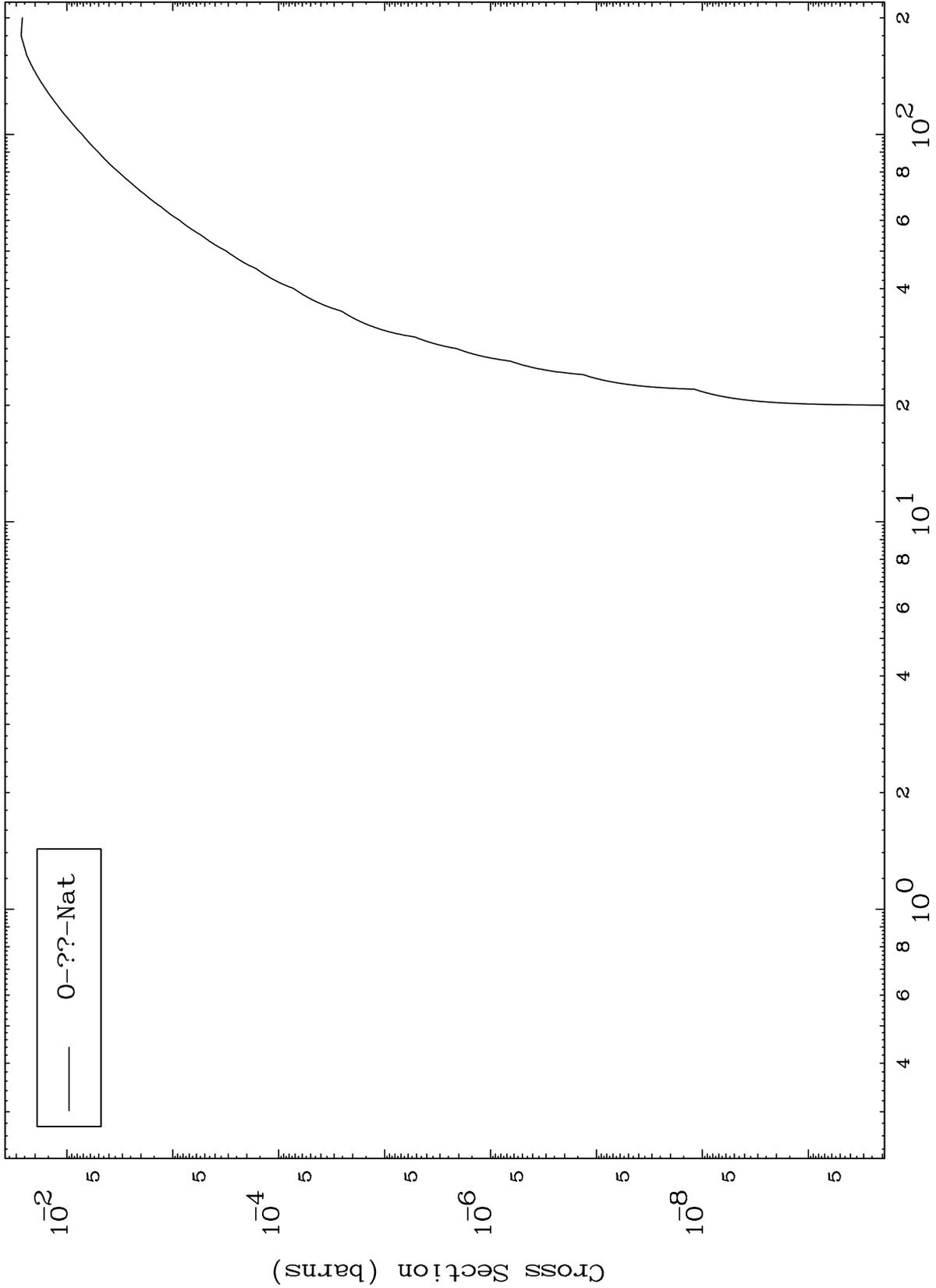


MAT 7535

Fission

<sup>75</sup>Re-188m

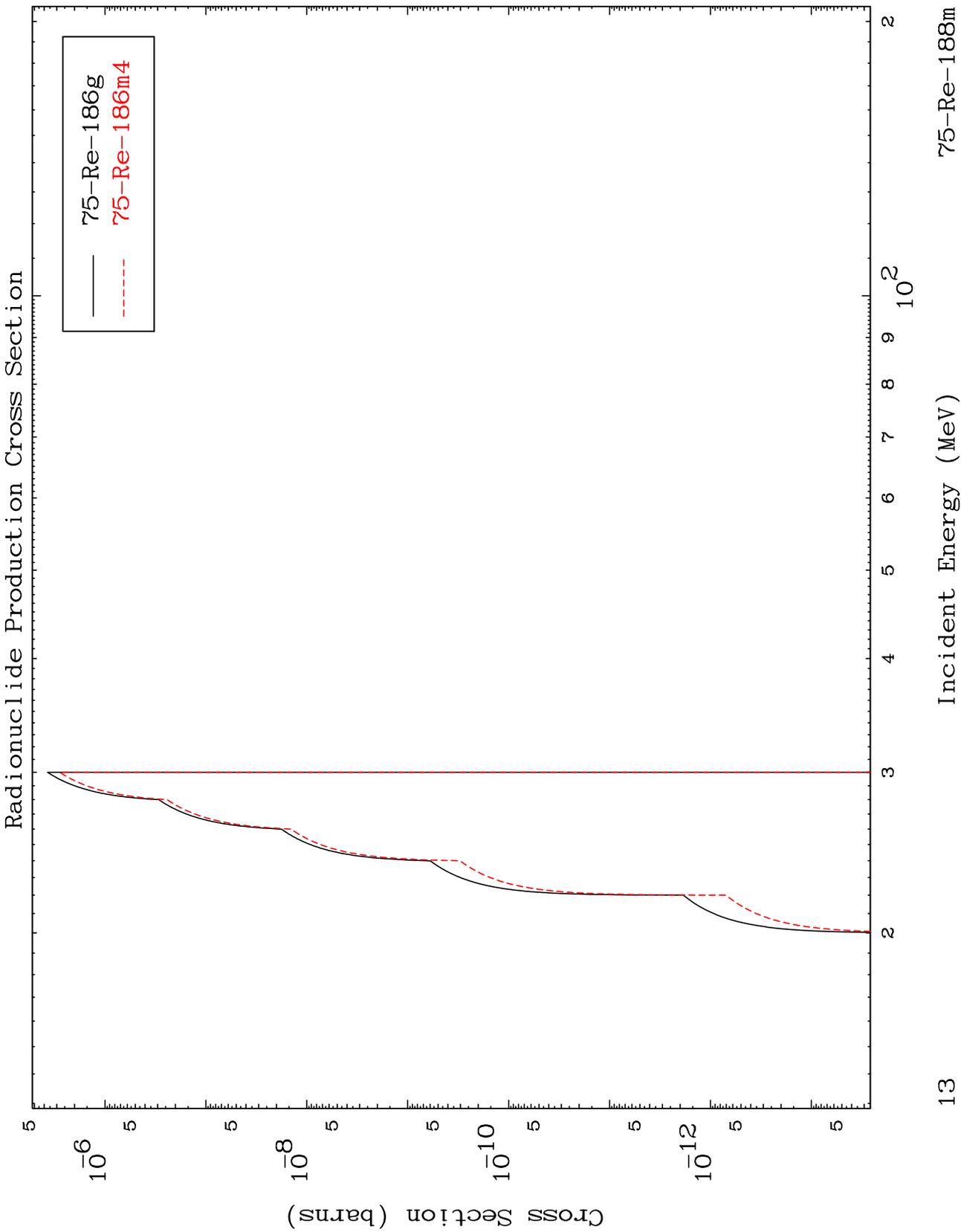
Radionuclide Production Cross Section



MAT 7535

$(n,2n) \alpha$

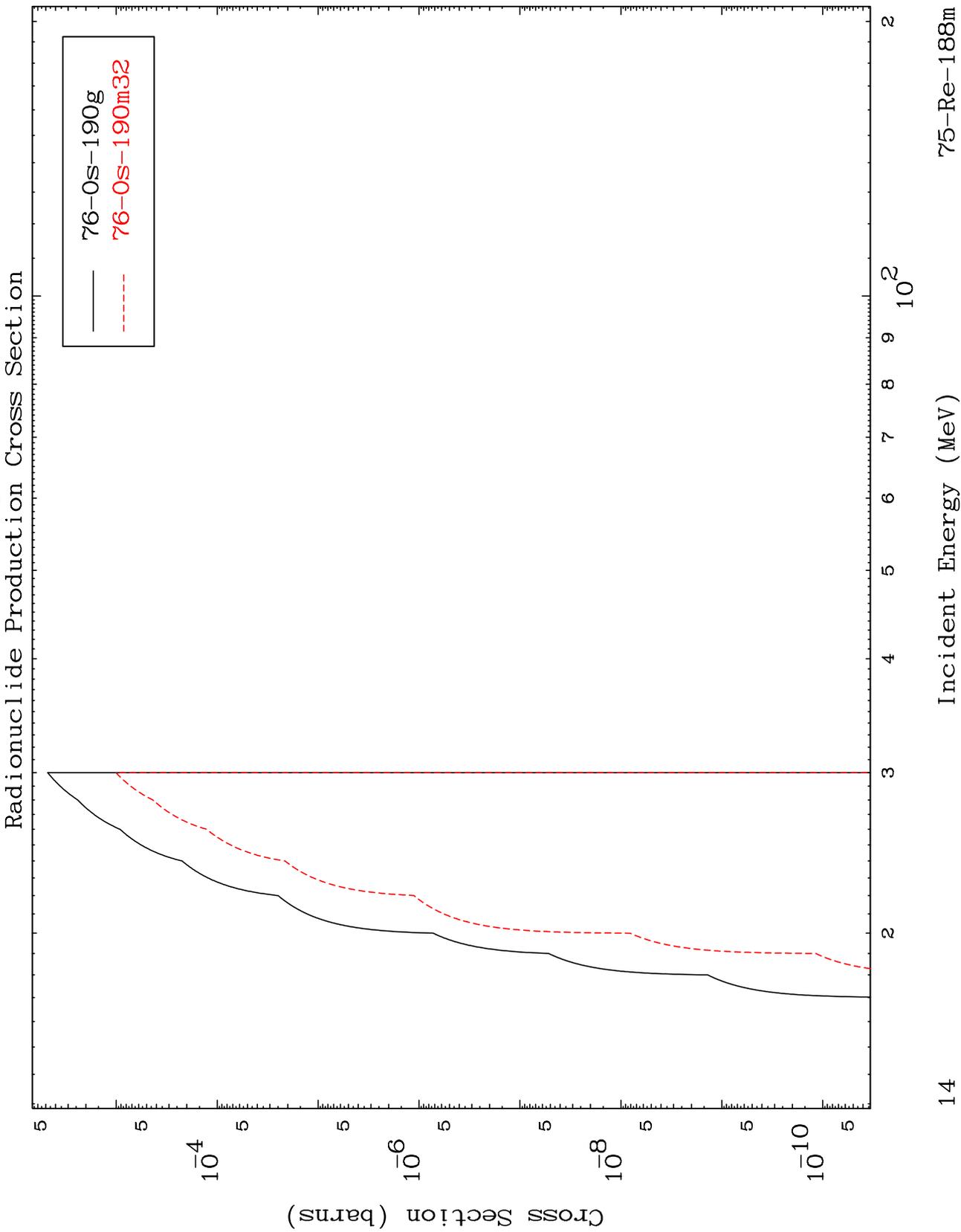
<sup>75</sup>Re-188m



MAT 7535

(n,n') p

75-Re-188m



14

Incident Energy (MeV)

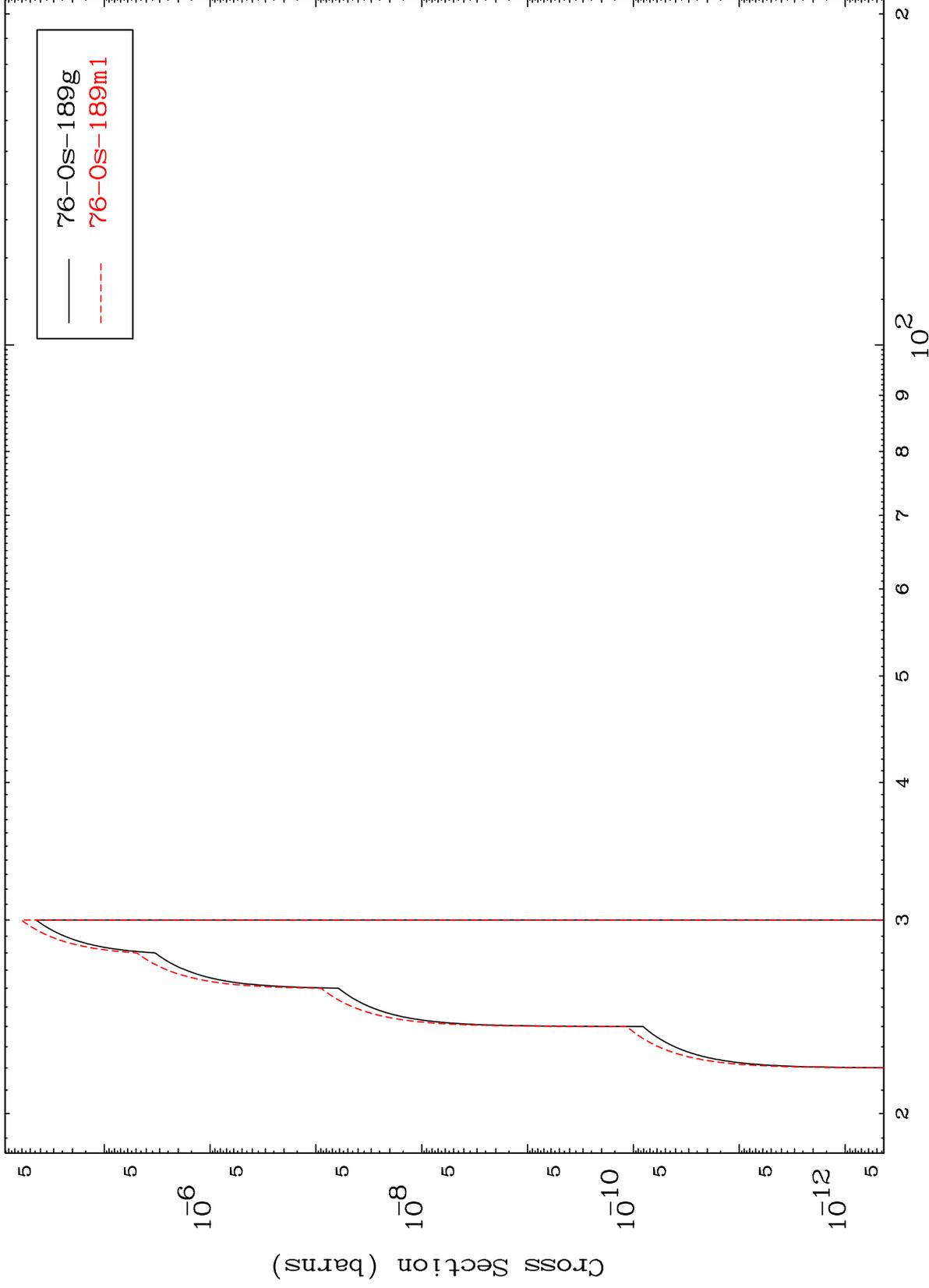
75-Re-188m

MAT 7535

(n,n') d

75-Re-188m

Radionuclide Production Cross Section



15

Incident Energy (MeV)

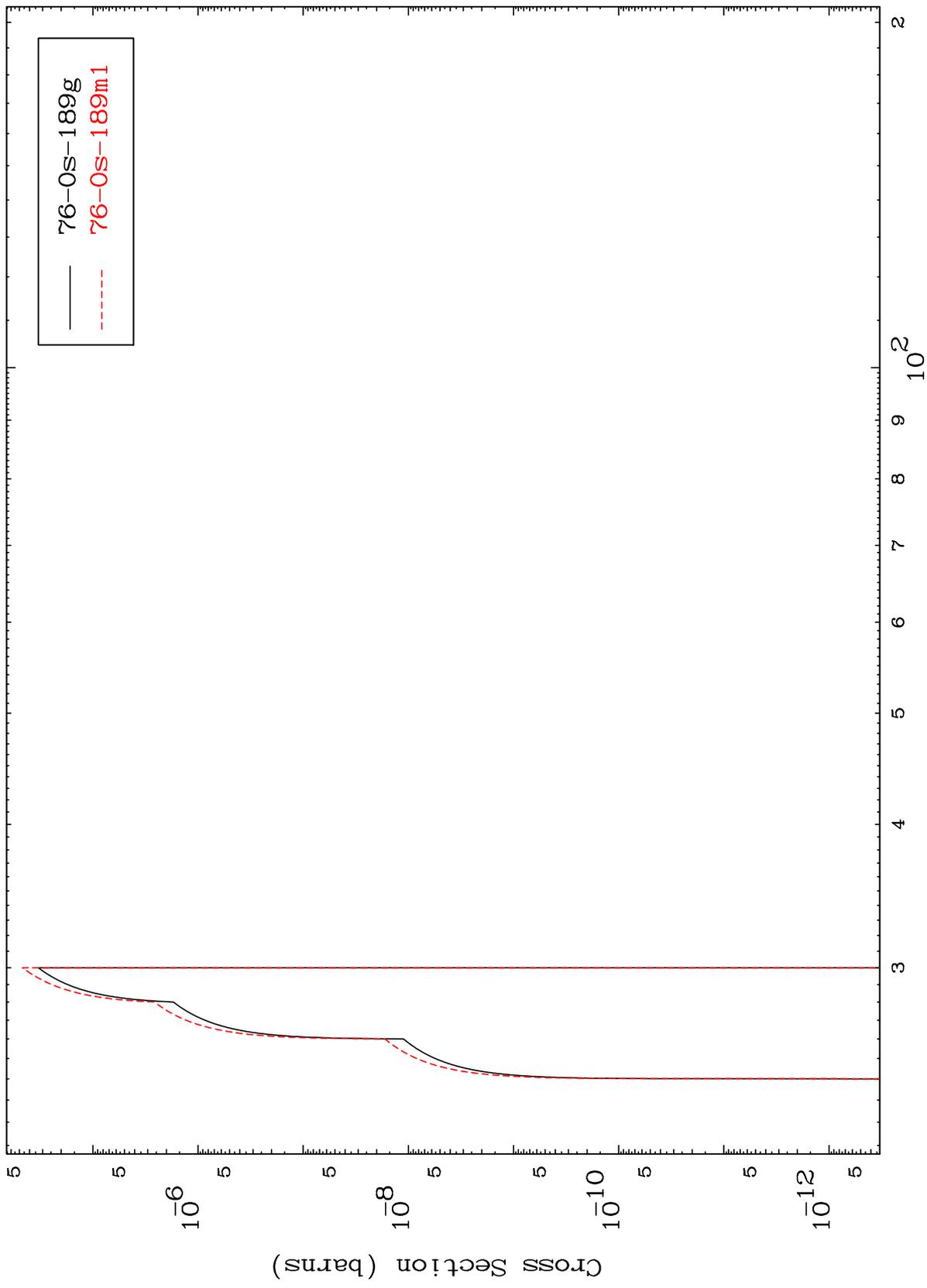
75-Re-188m

MAT 7535

(n,2n) p

75-Re-188m

Radionuclide Production Cross Section



16

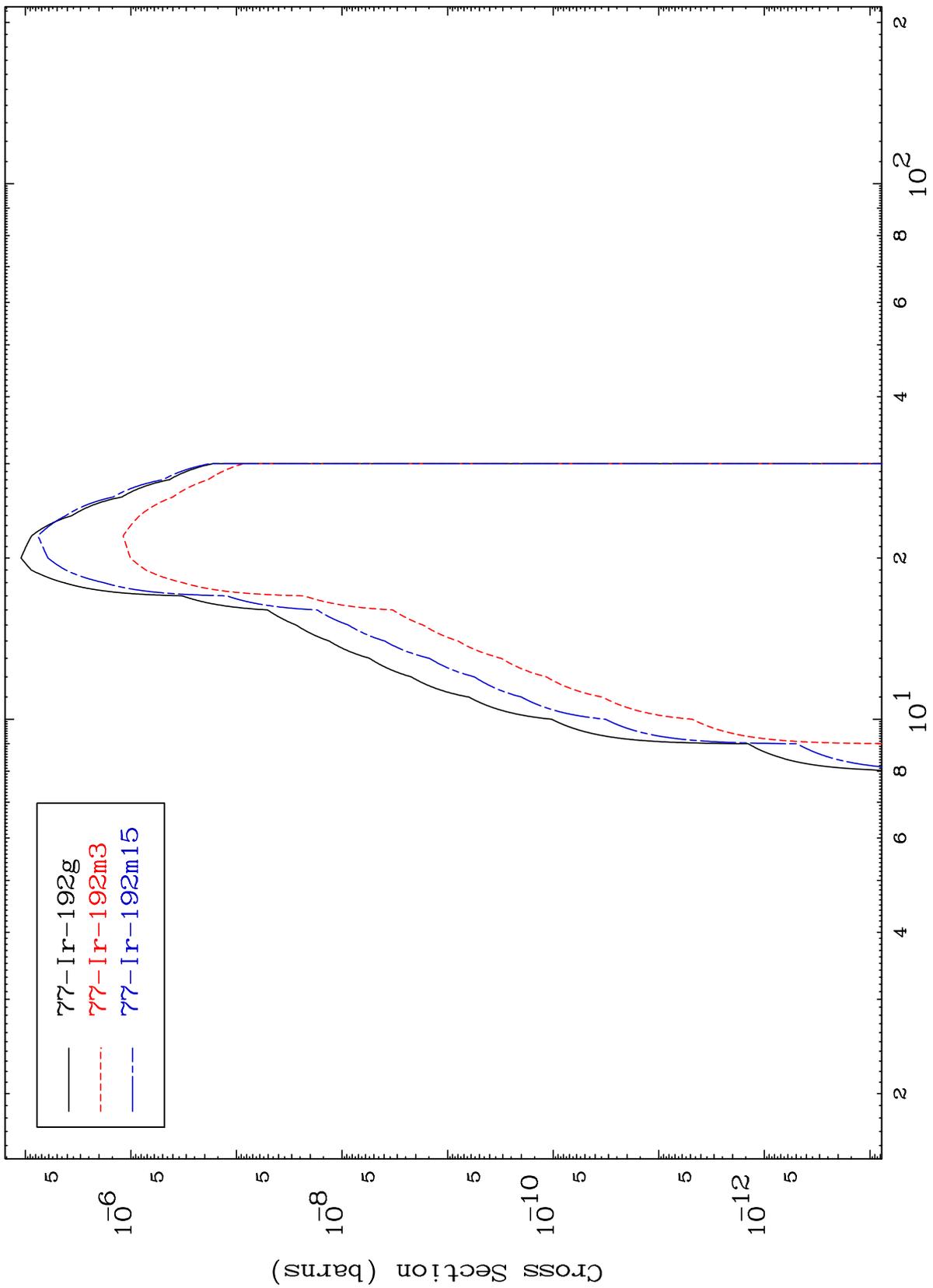
Incident Energy (MeV)

75-Re-188m

MAT 7535

<sup>75</sup>Re-188m

(n,  $\gamma$ )  
Radionuclide Production Cross Section



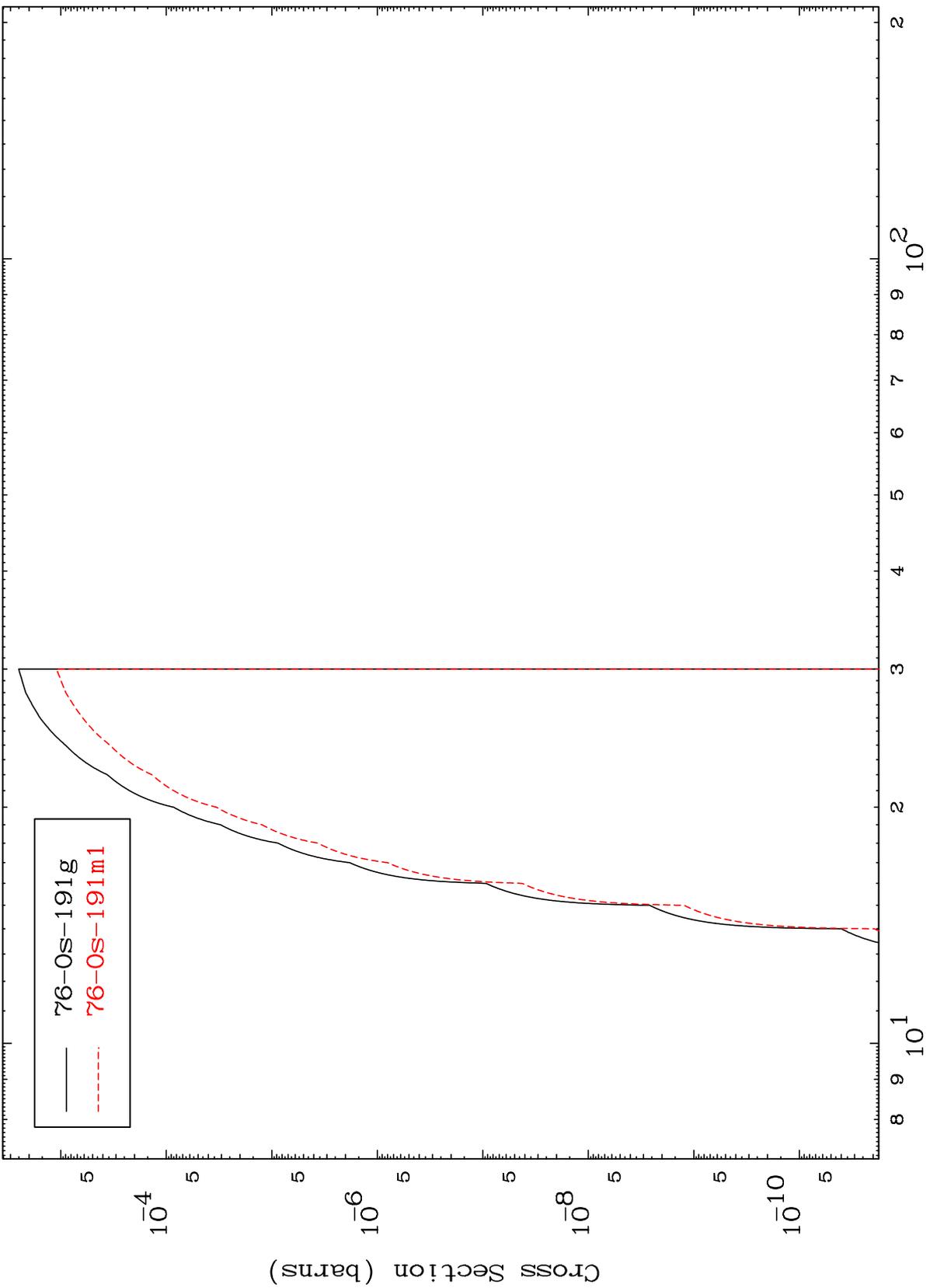
17

<sup>75</sup>Re-188m

MAT 7535

<sup>75</sup>Re-188m

Radionuclide Production Cross Section (n,p)



— 76-Os-191g  
- - - 76-Os-191m1

18

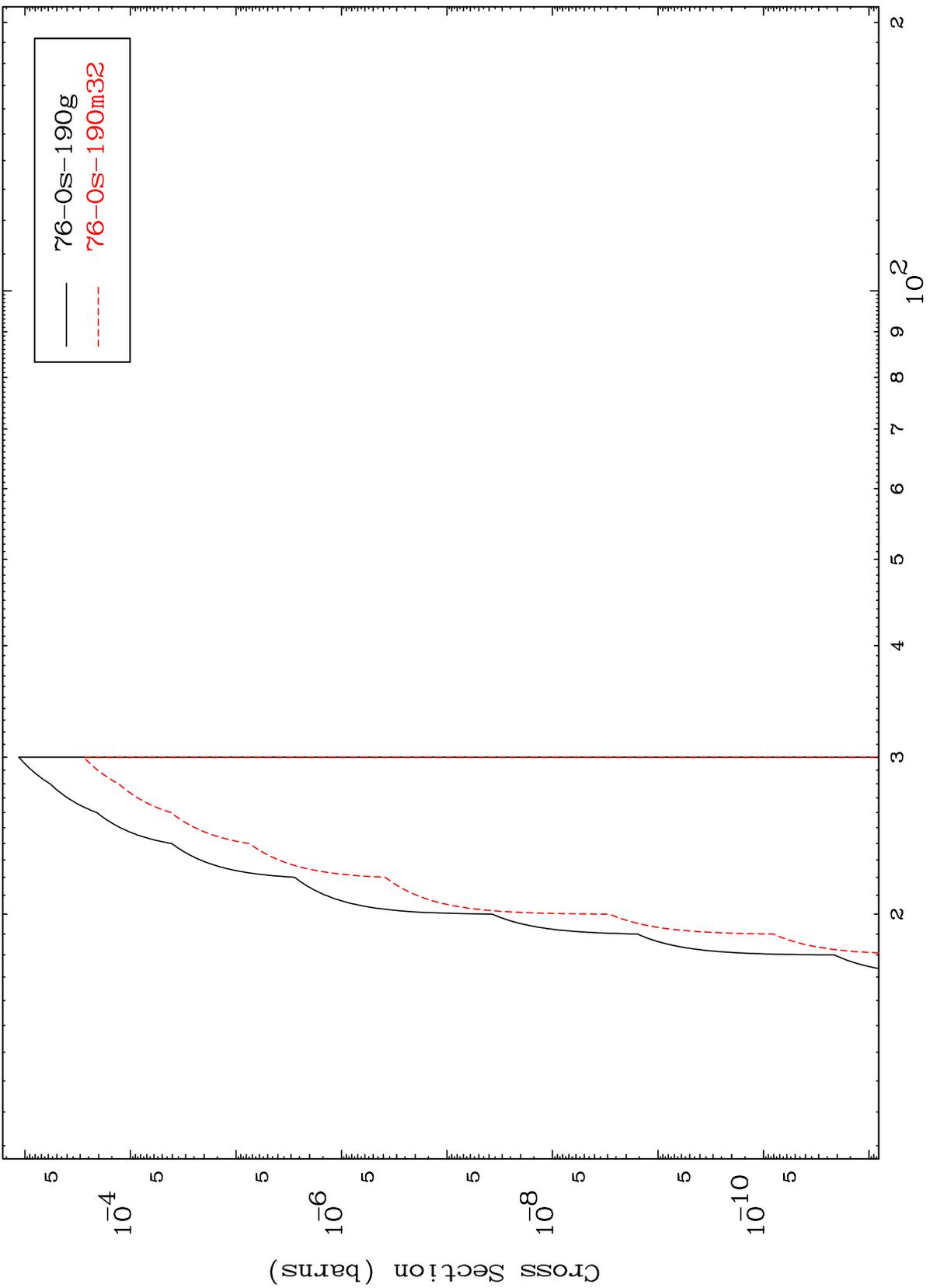
Incident Energy (MeV)

<sup>75</sup>Re-188m

MAT 7535

75-Re-188m

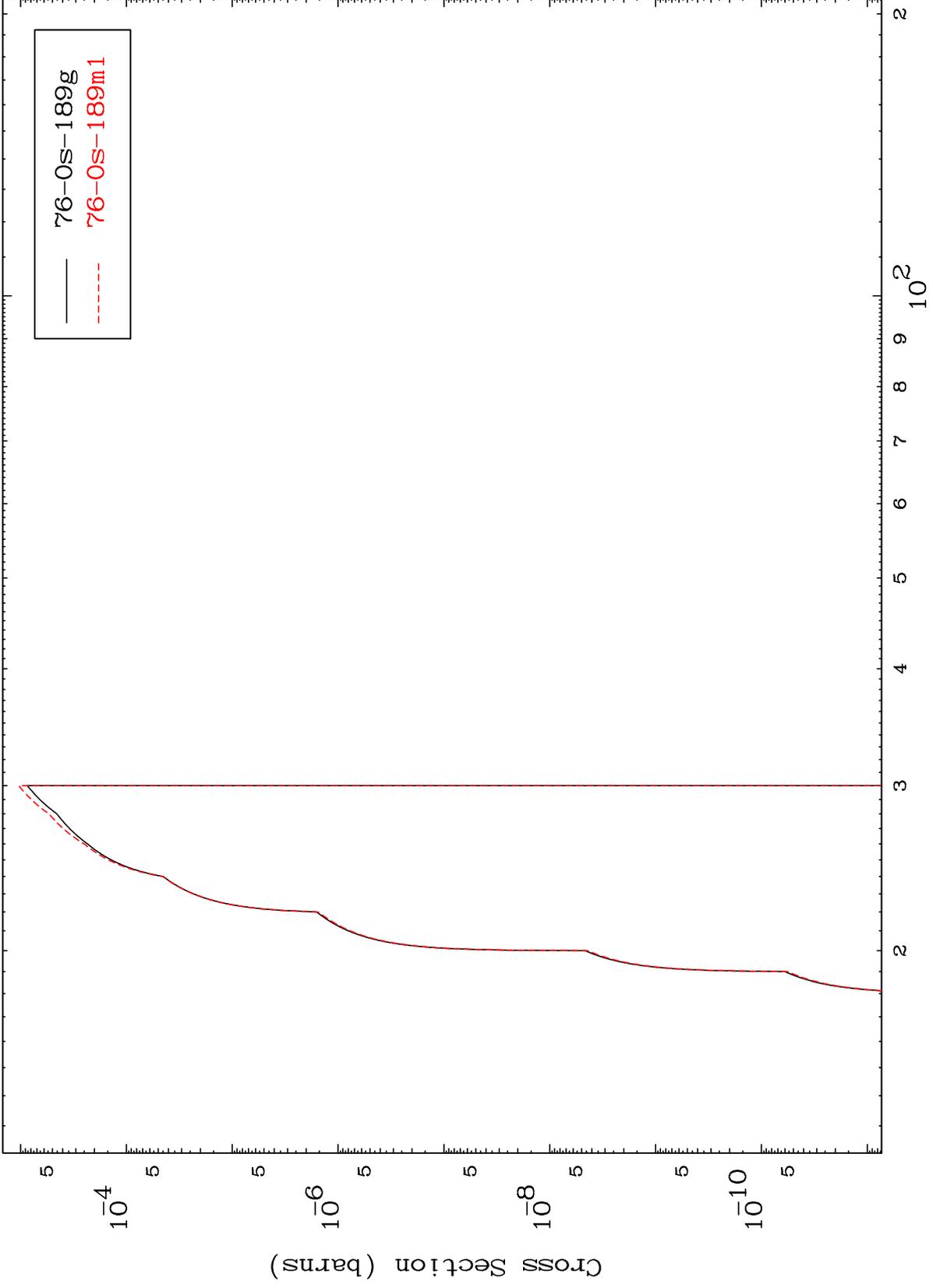
(n,d)  
Radionuclide Production Cross Section



MAT 7535

75-Re-188m

(n,t)  
Radionuclide Production Cross Section



20

Incident Energy (MeV)

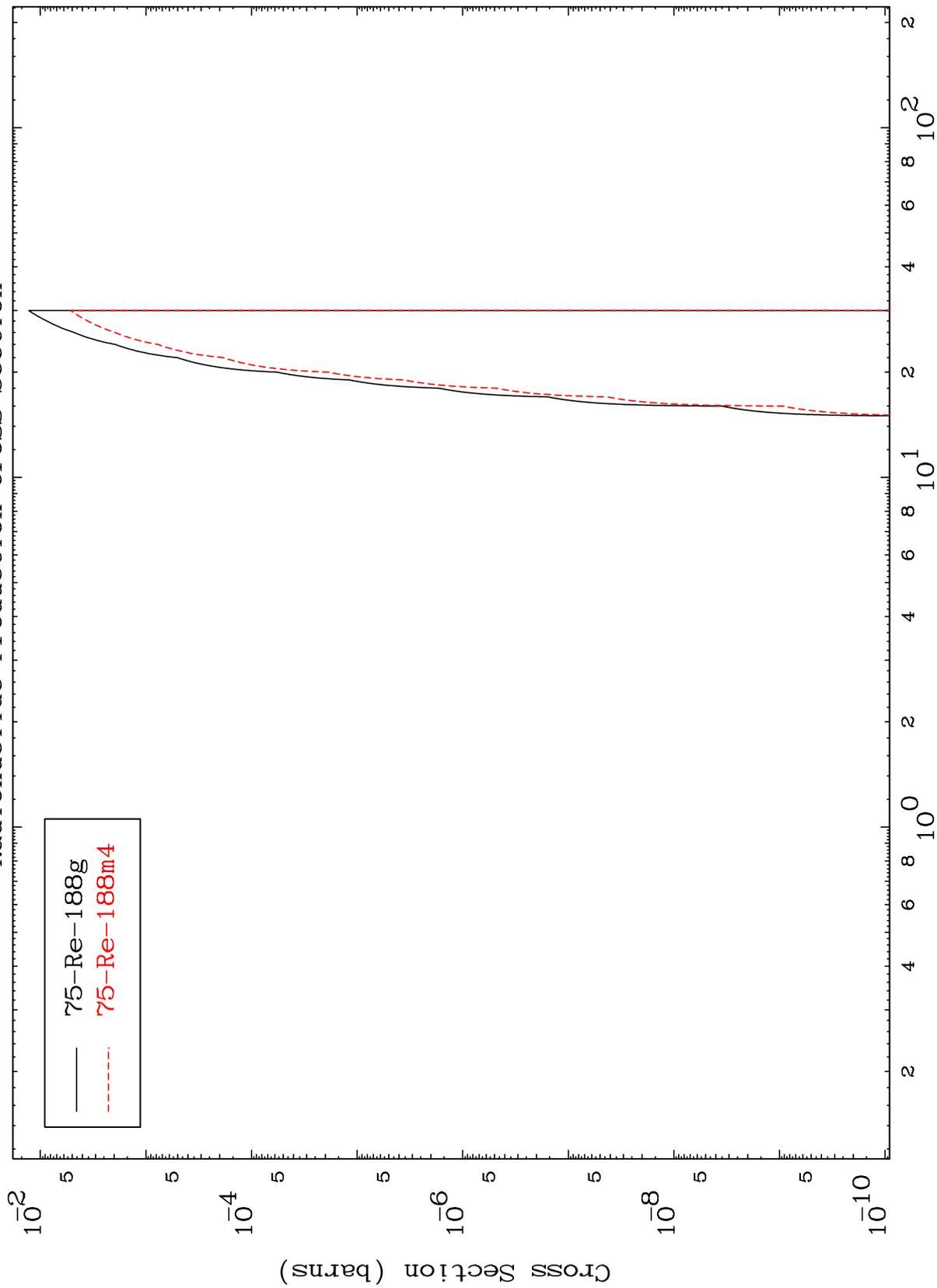
75-Re-188m

MAT 7535

(n,  $\alpha$ )

<sup>75</sup>Re-188m

Radionuclide Production Cross Section



75-Re-188g  
75-Re-188m4

Incident Energy (MeV)

<sup>75</sup>Re-188m