

Program EVALPLOT
(Version 2021-1)

by

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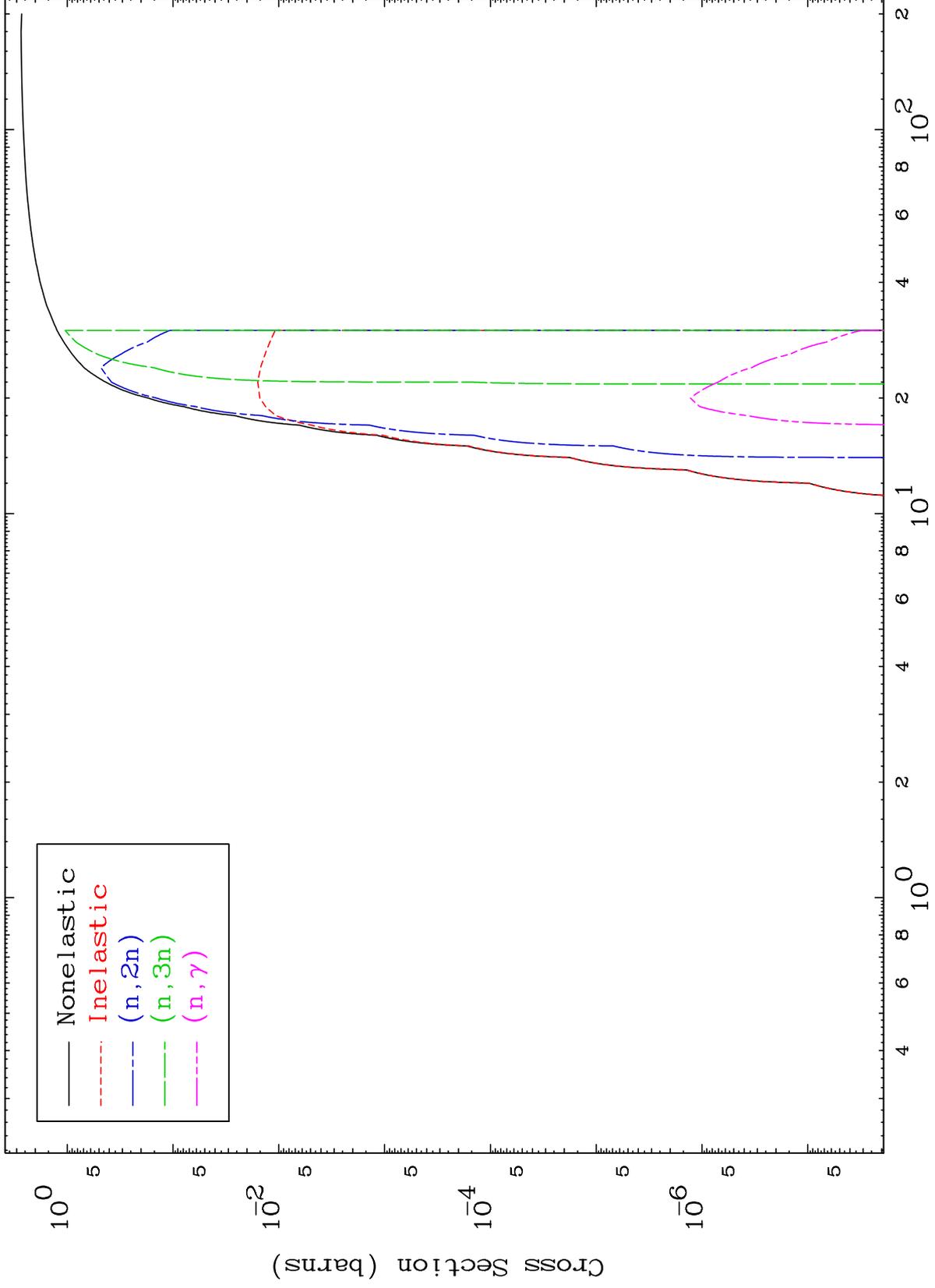
Press Mouse Button to Start

MAT 7543

α Major

75-Re-191

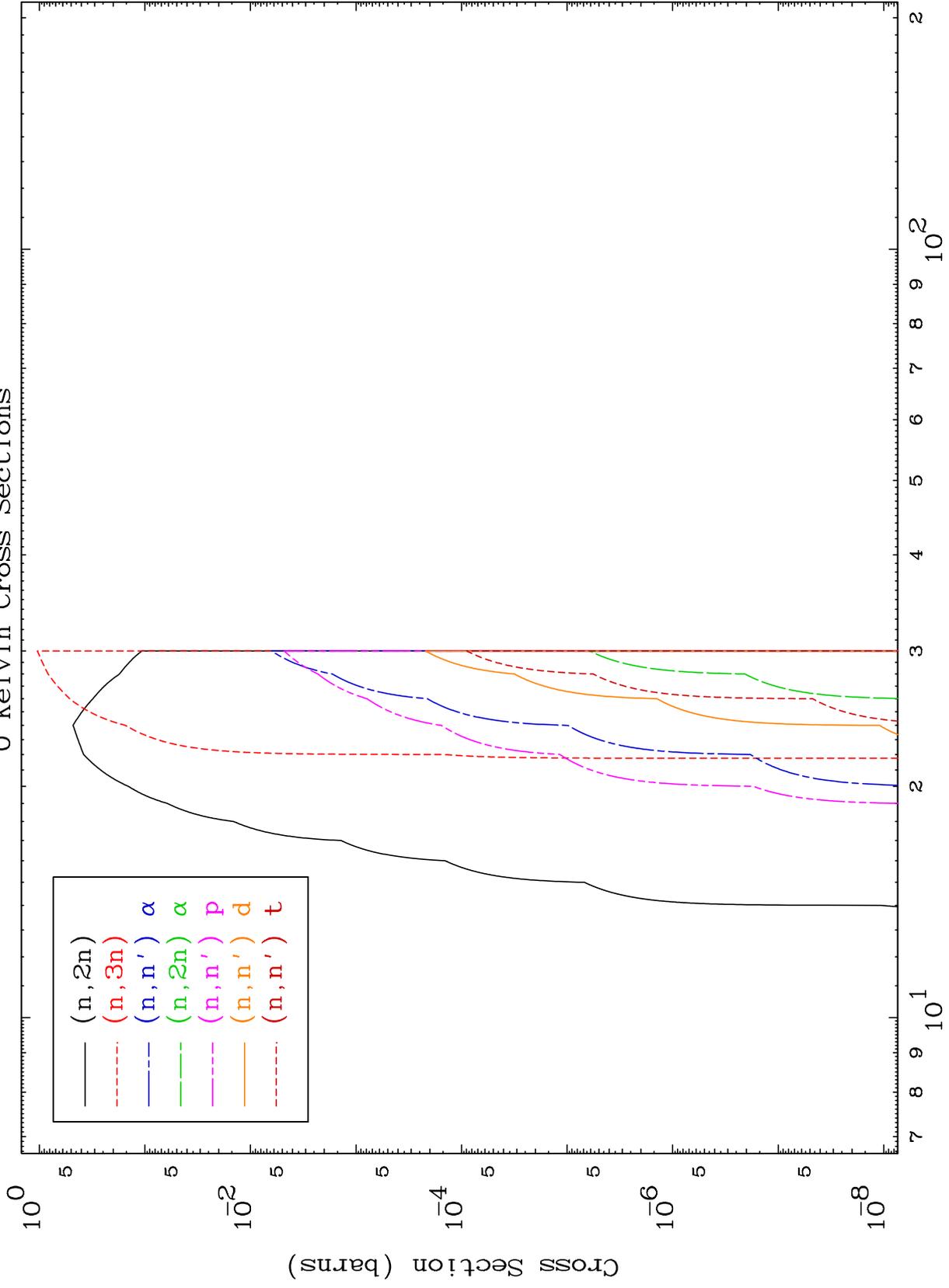
0 Kelvin Cross Sections



MAT 7543

α Neutron Absorption
0 Kelvin Cross Sections

75-Re-191



2

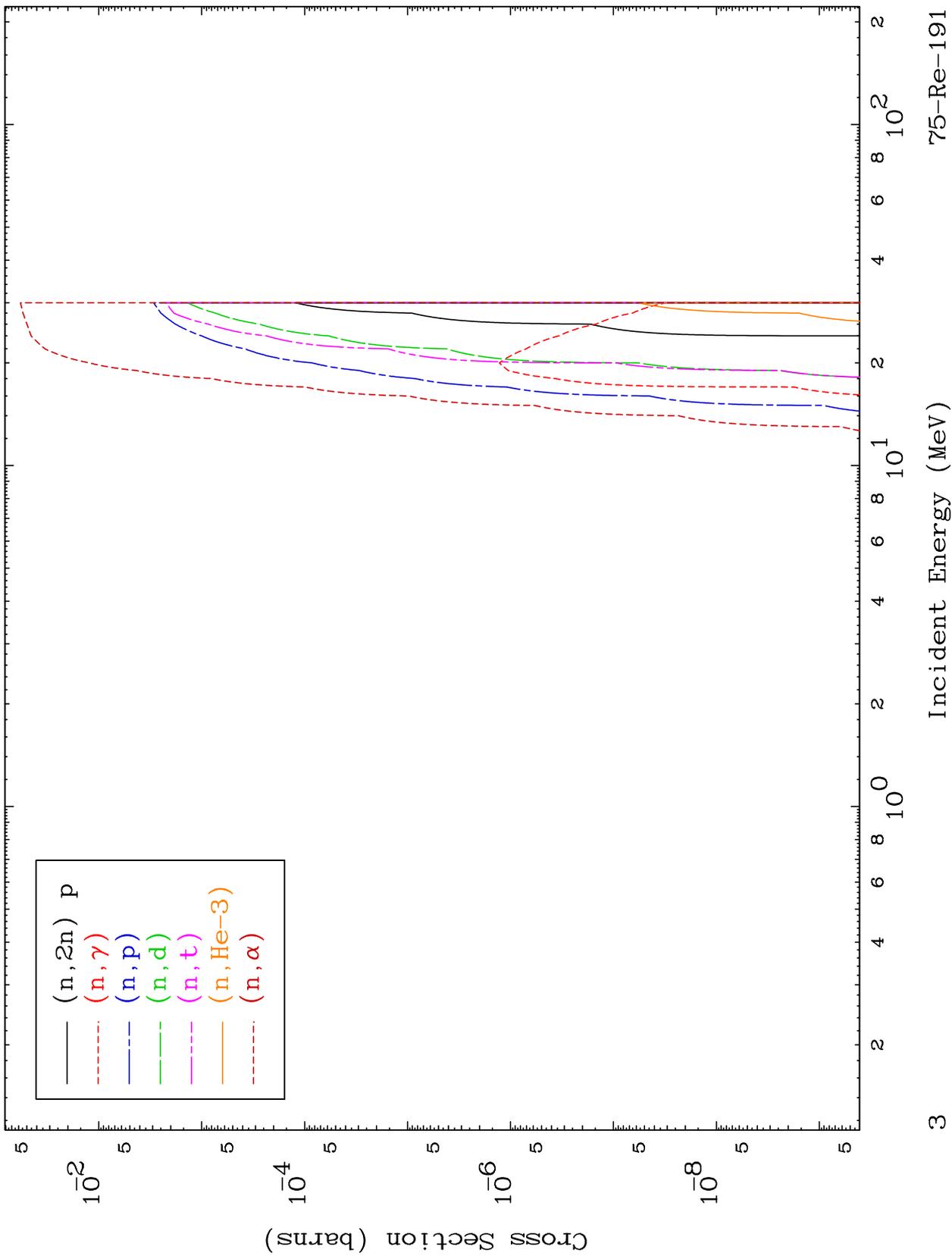
Incident Energy (MeV)

75-Re-191

MAT 7543

α Neutron Absorption
0 Kelvin Cross Sections

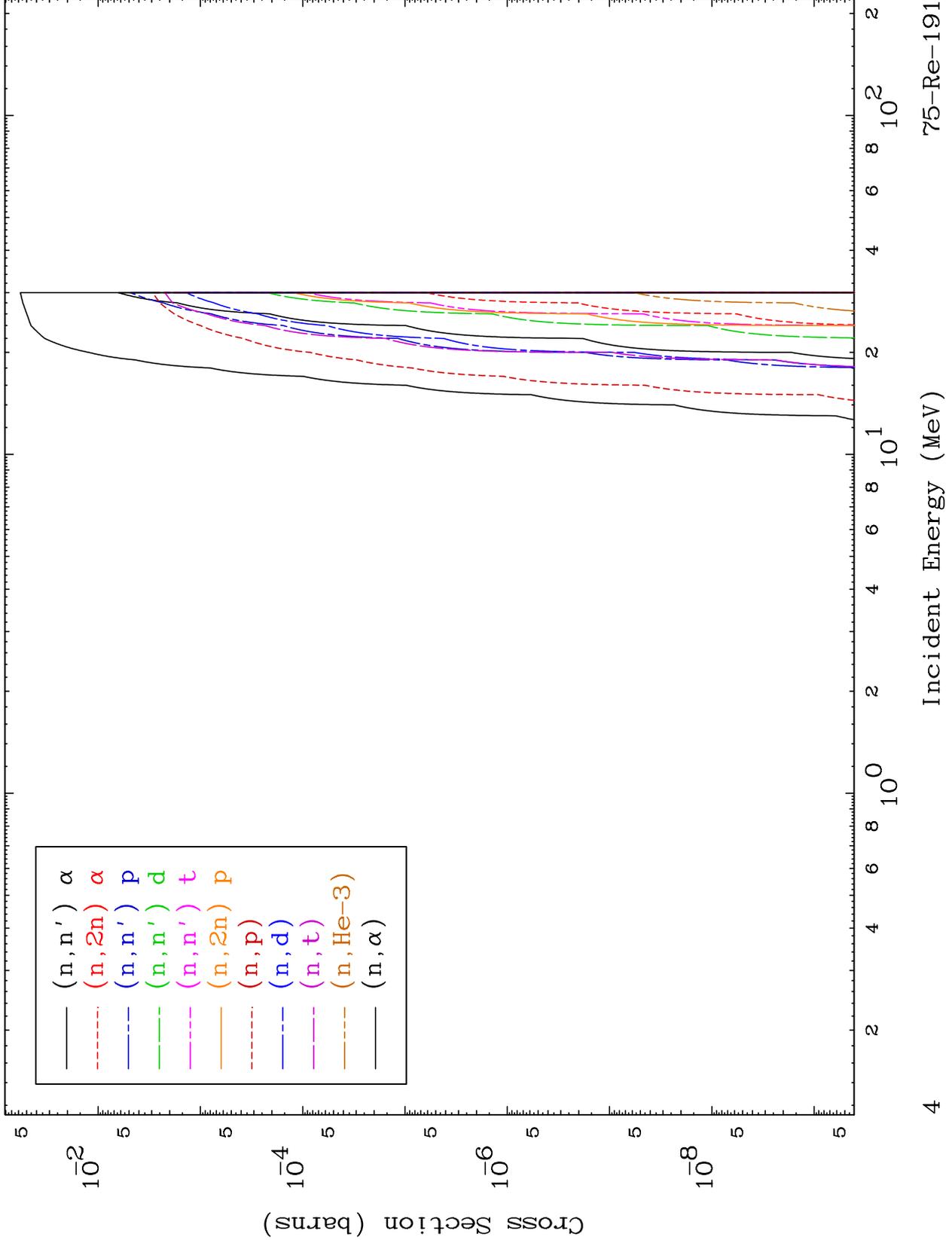
75-Re-191



MAT 7543

α Charged Particle
0 Kelvin Cross Sections

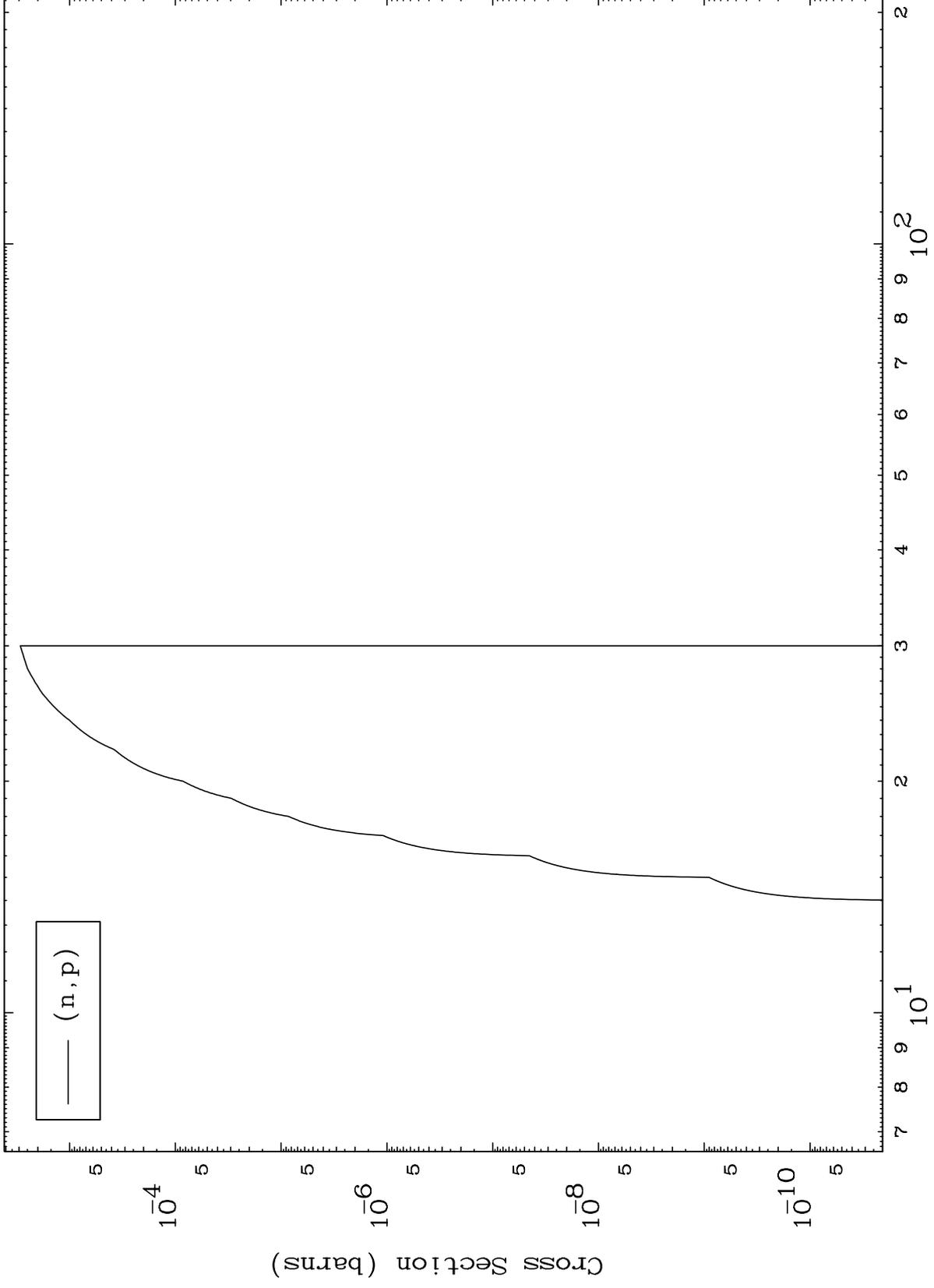
75-Re-191



MAT 7543

75-Re-191

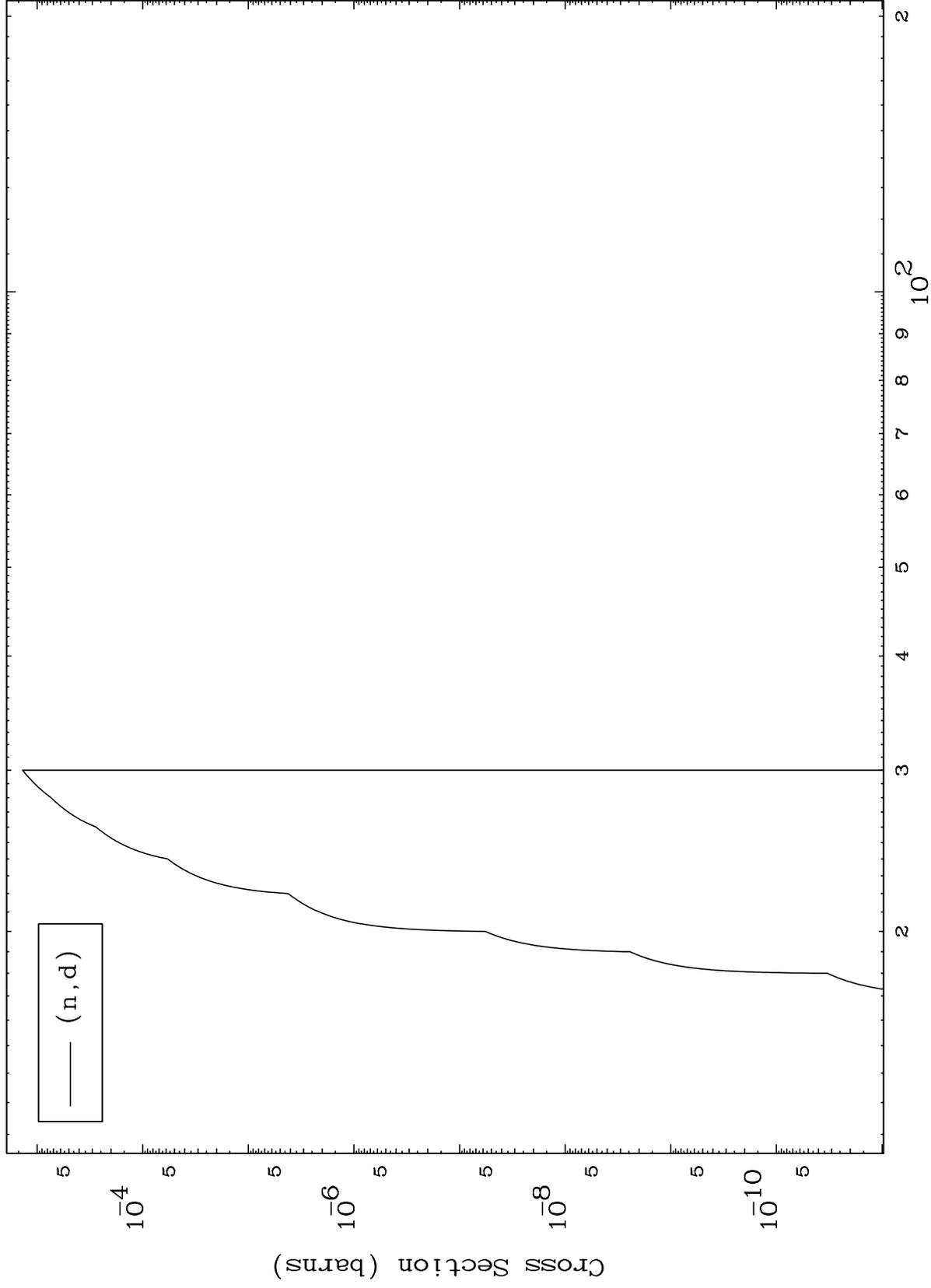
(α, p) Levels
0 Kelvin Cross Sections

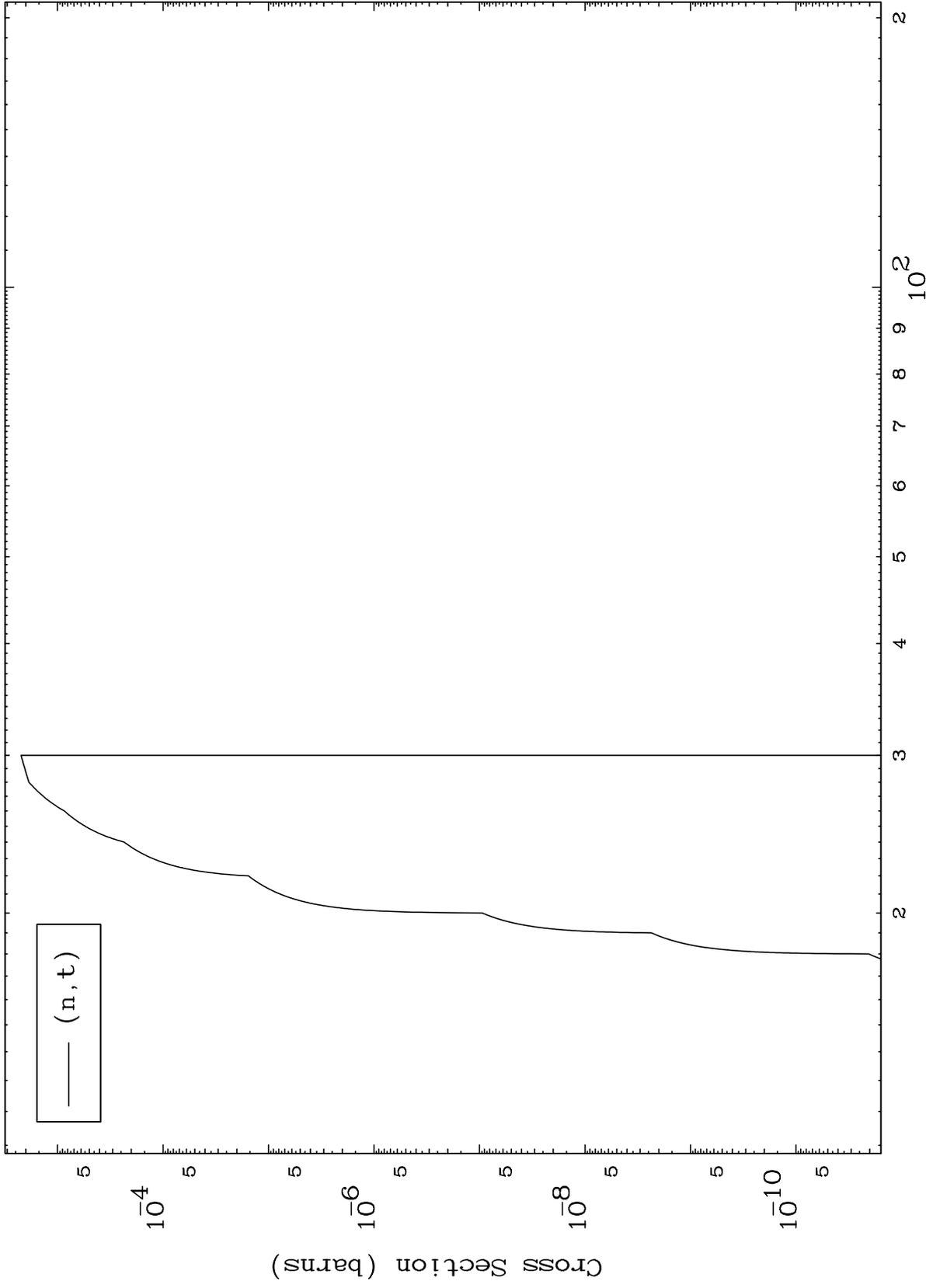


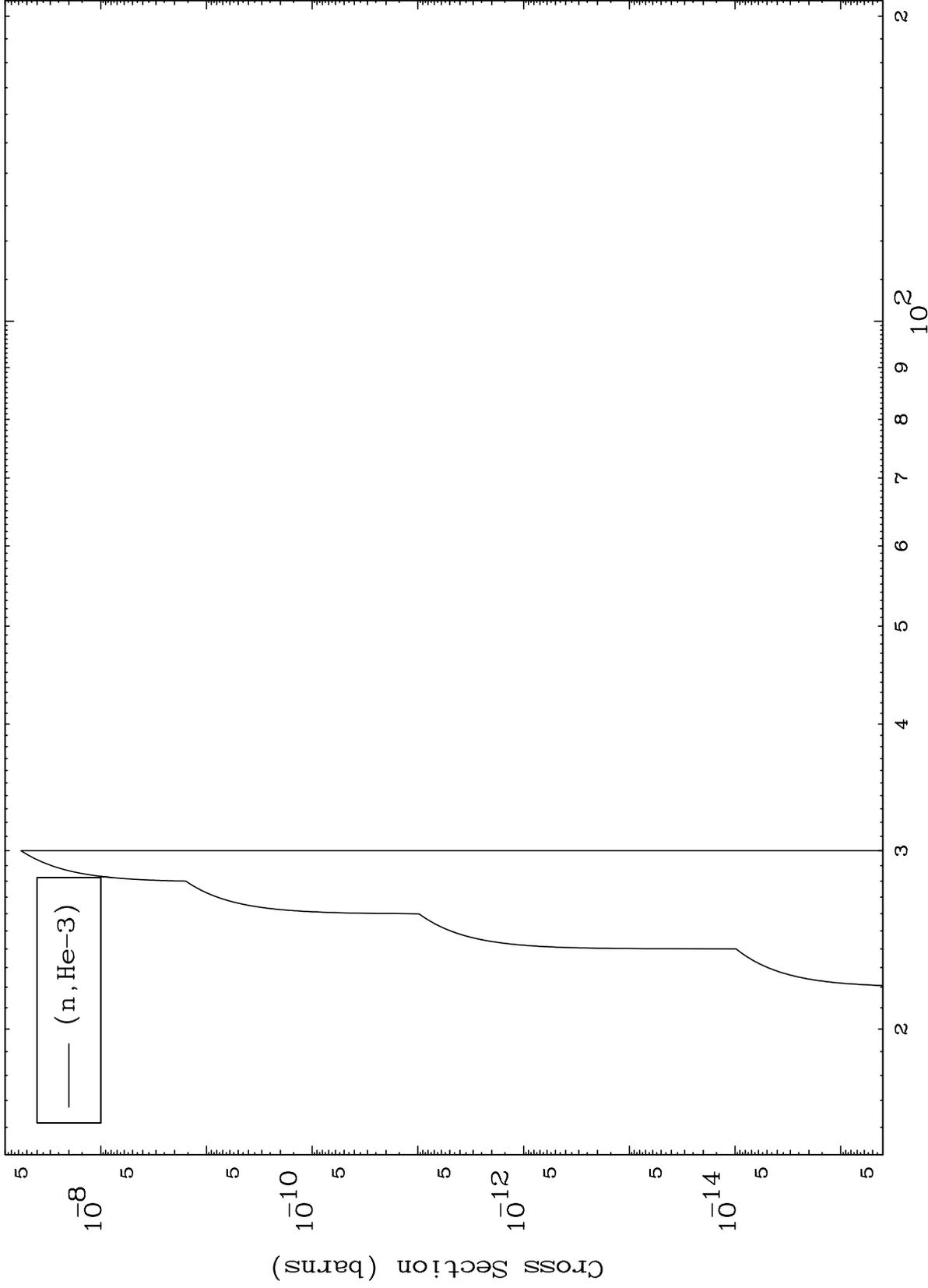
5

Incident Energy (MeV)

75-Re-191





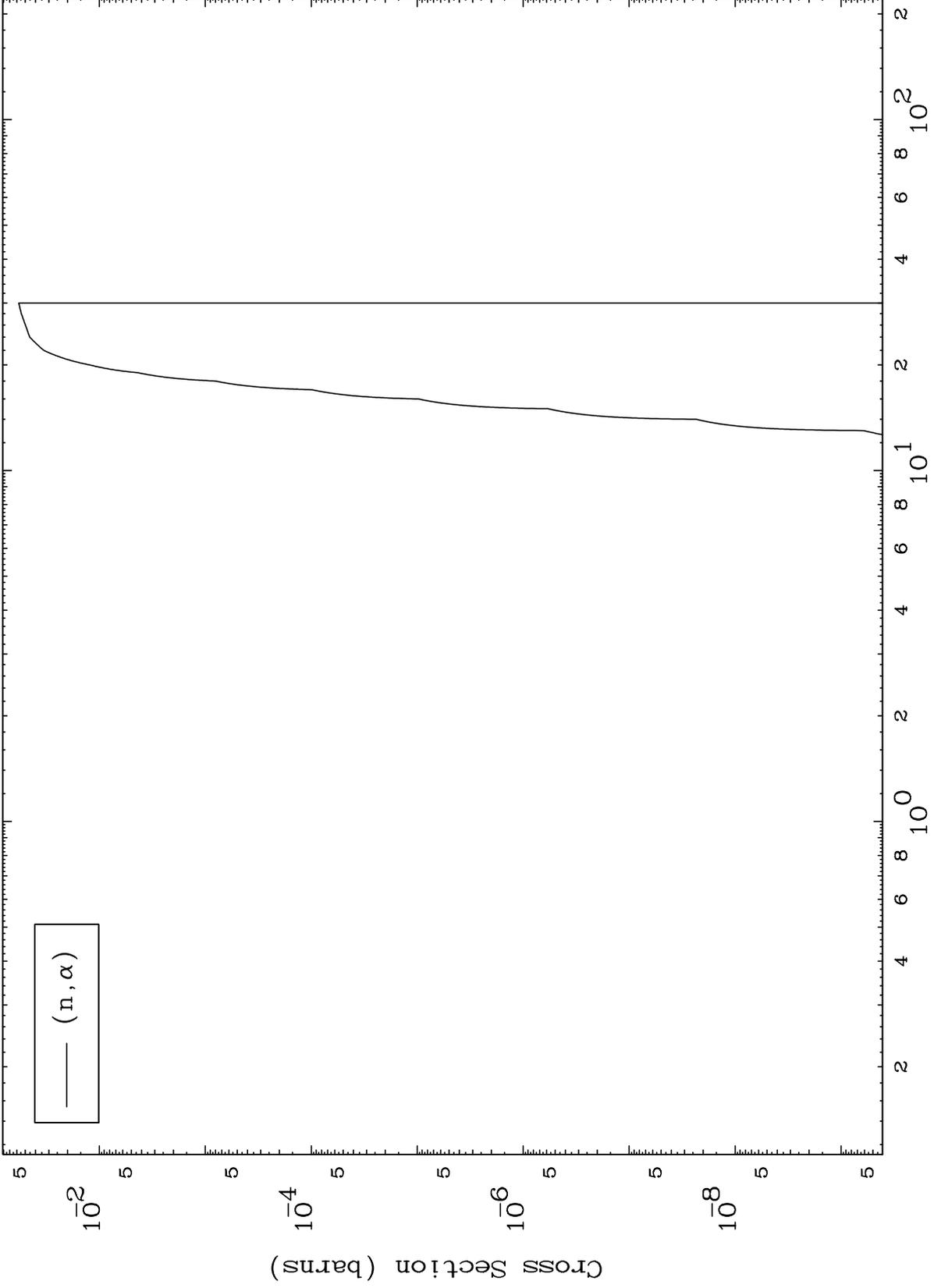


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(α, α) Levels

75-Re-191

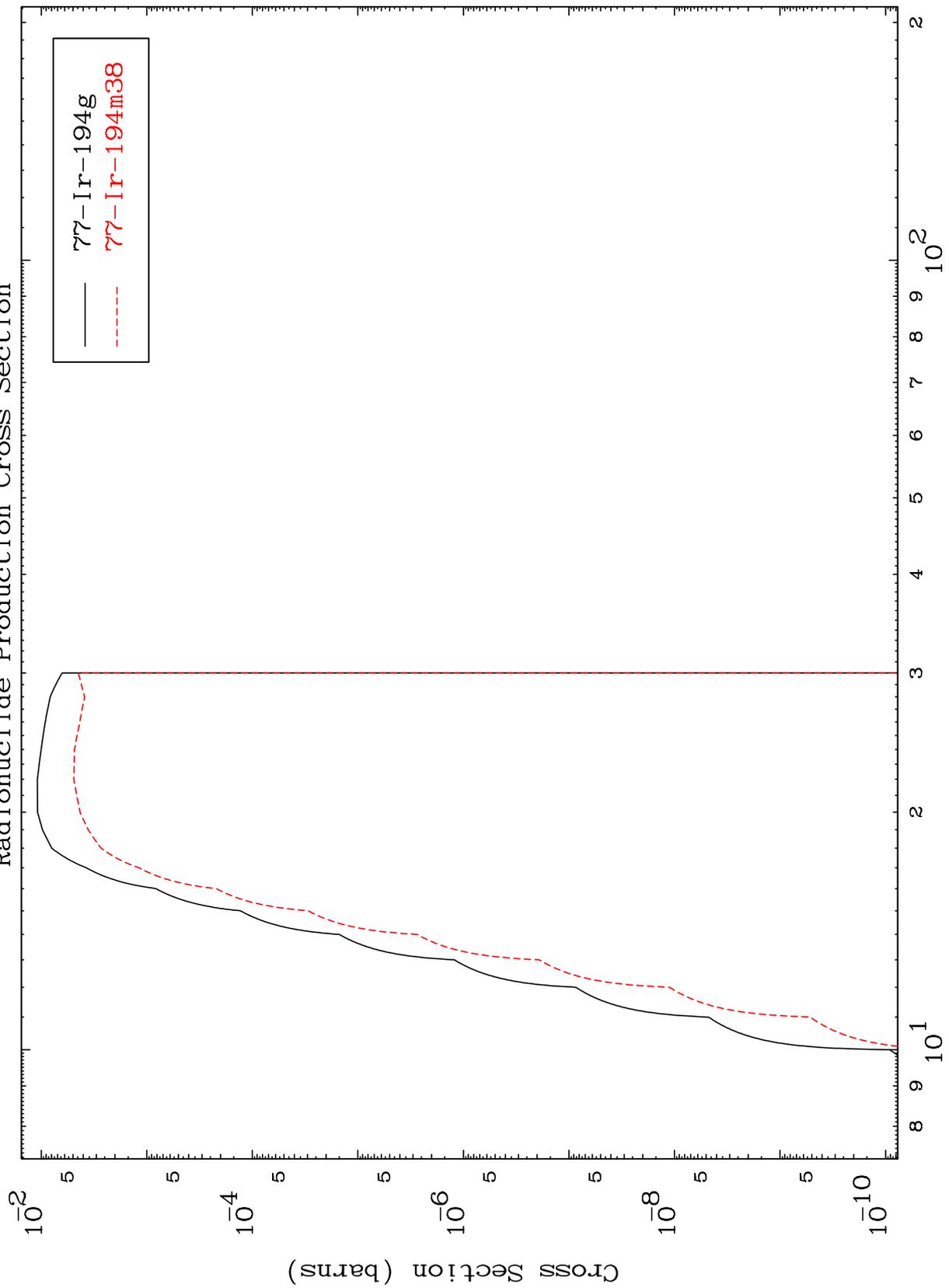
0 Kelvin Cross Sections



MAT 7543

75-Re-191

Inelastic
Radionuclide Production Cross Section



75-Re-191

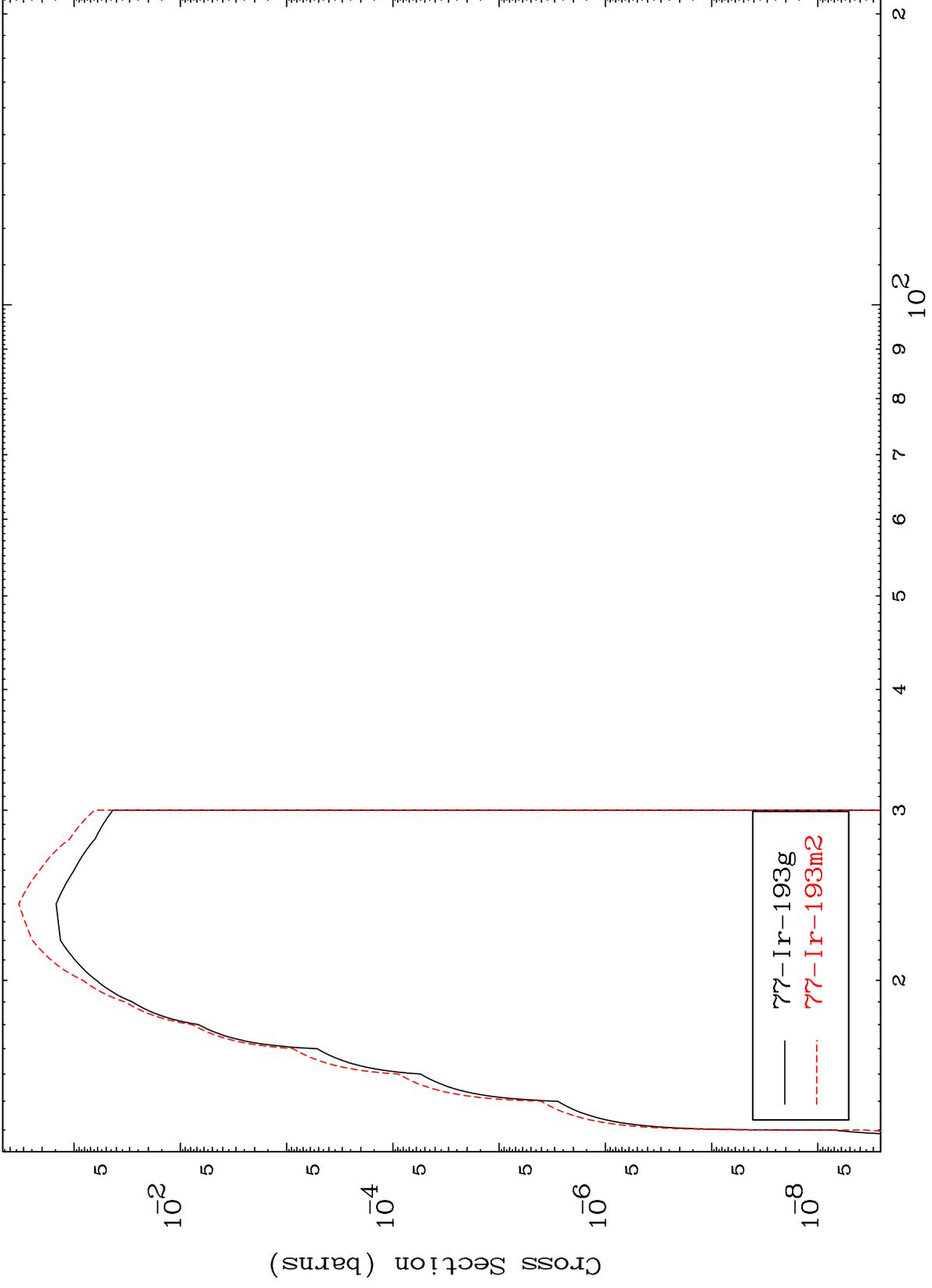
Incident Energy (MeV)

10

MAT 7543

75-Re-191

(n,2n)
Radionuclide Production Cross Section

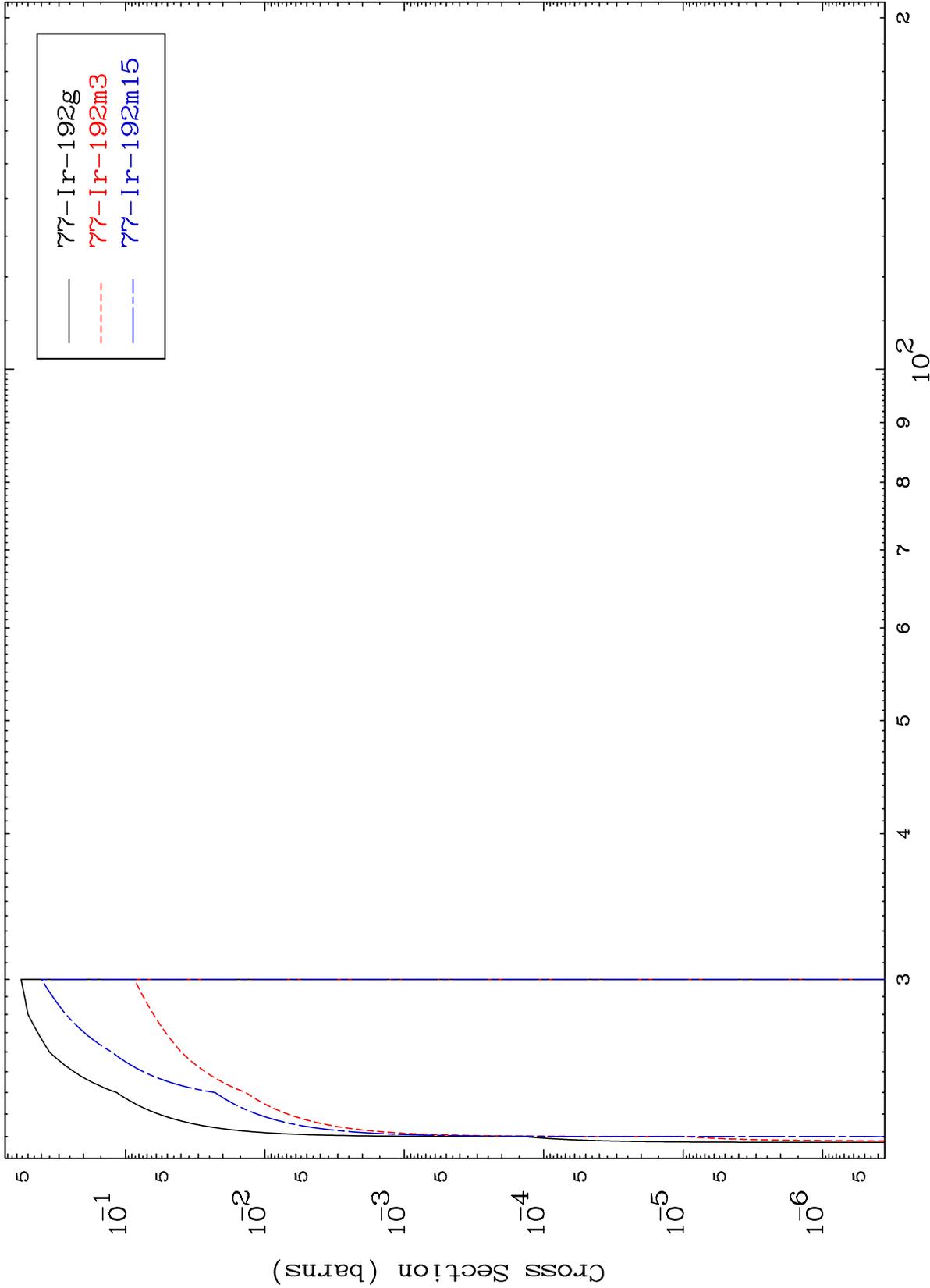


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(n,3n)

75-Re-191

Radionuclide Production Cross Section



12

Incident Energy (MeV)

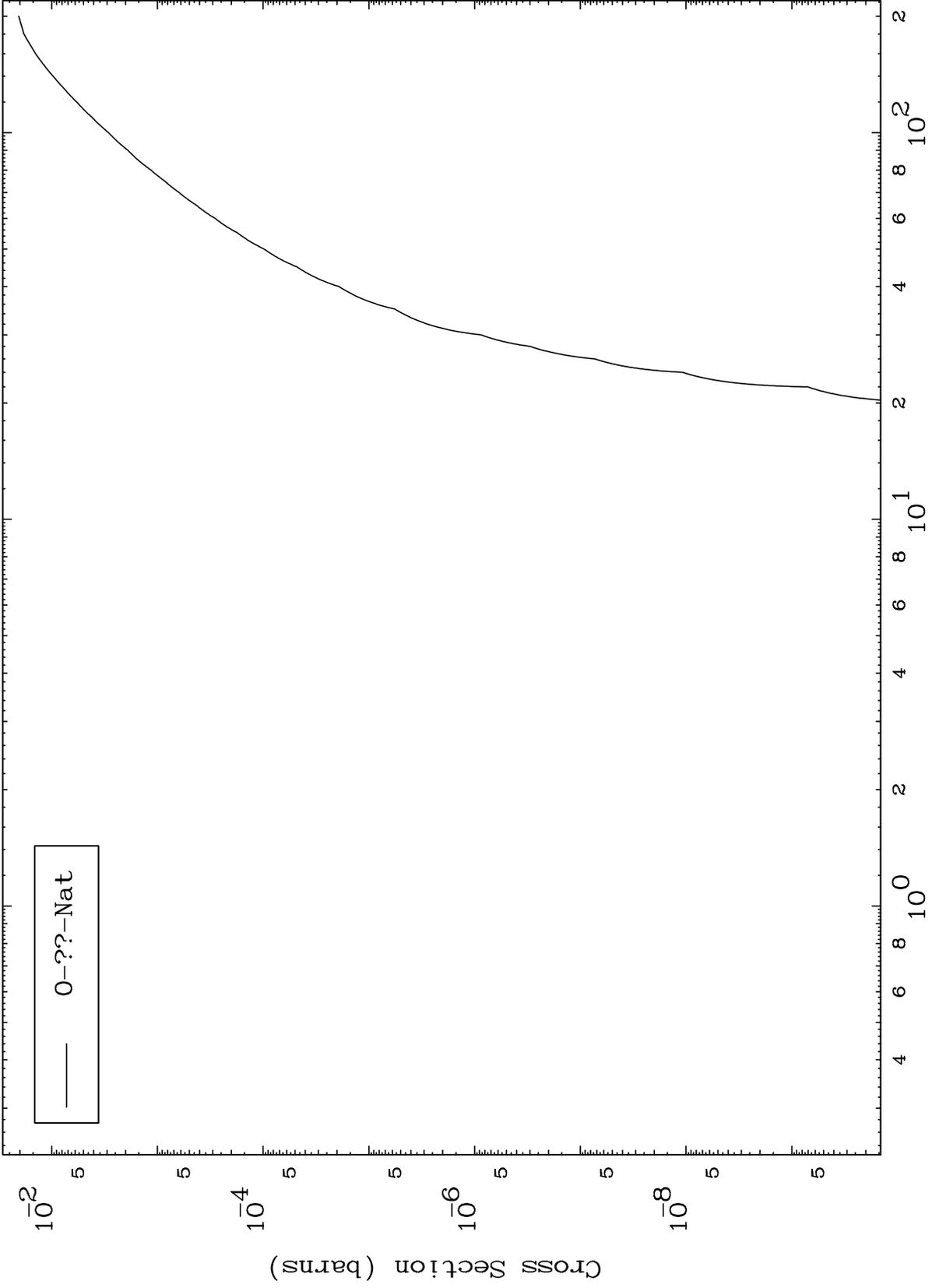
75-Re-191

MAT 7543

Fission

⁷⁵Re-191

Radionuclide Production Cross Section



13

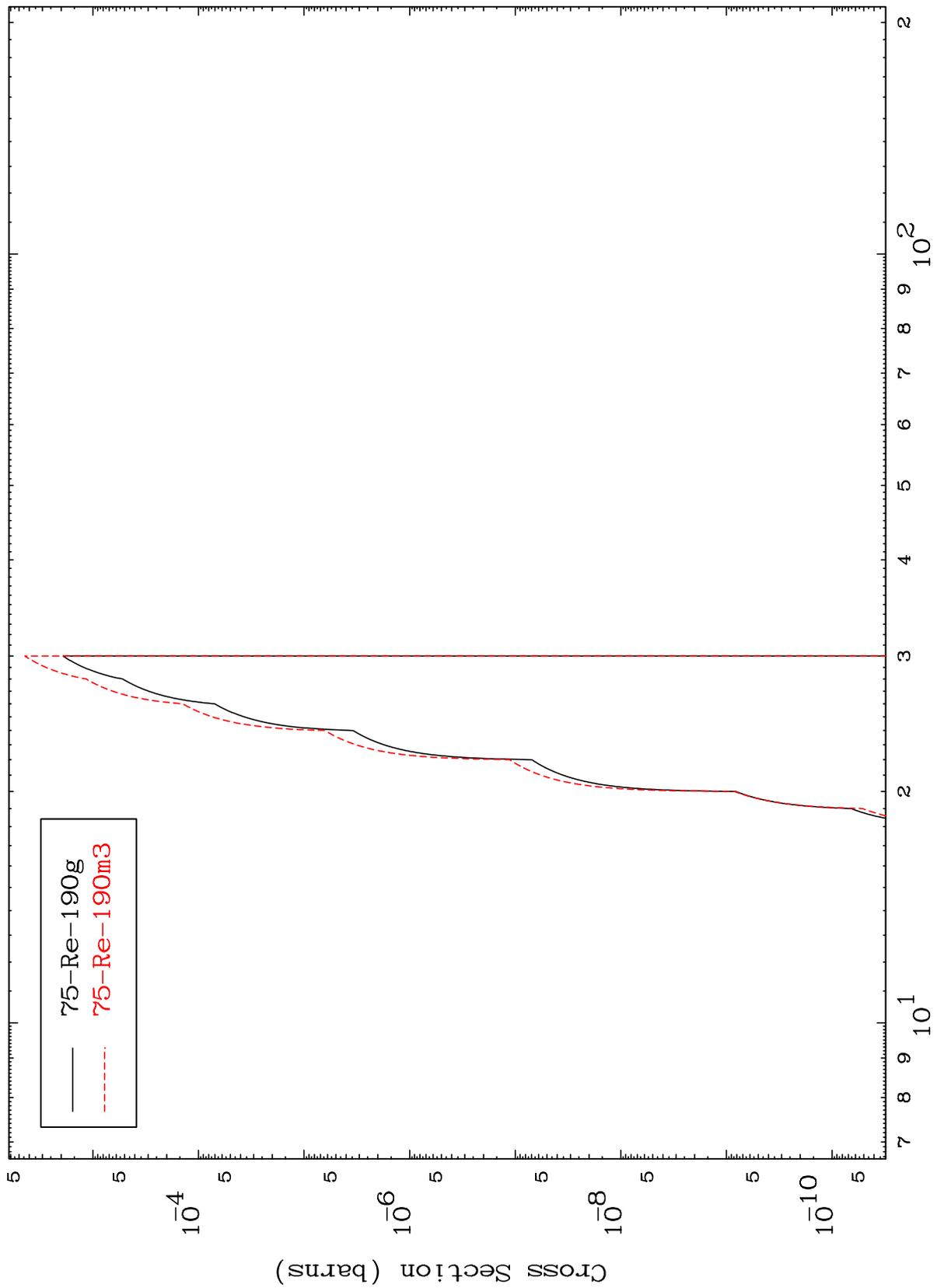
Incident Energy (MeV)

⁷⁵Re-191

MAT 7543

75-Re-191

$(n, n') \alpha$
Radionuclide Production Cross Section



75-Re-190g
75-Re-190m3

75-Re-191

Incident Energy (MeV)

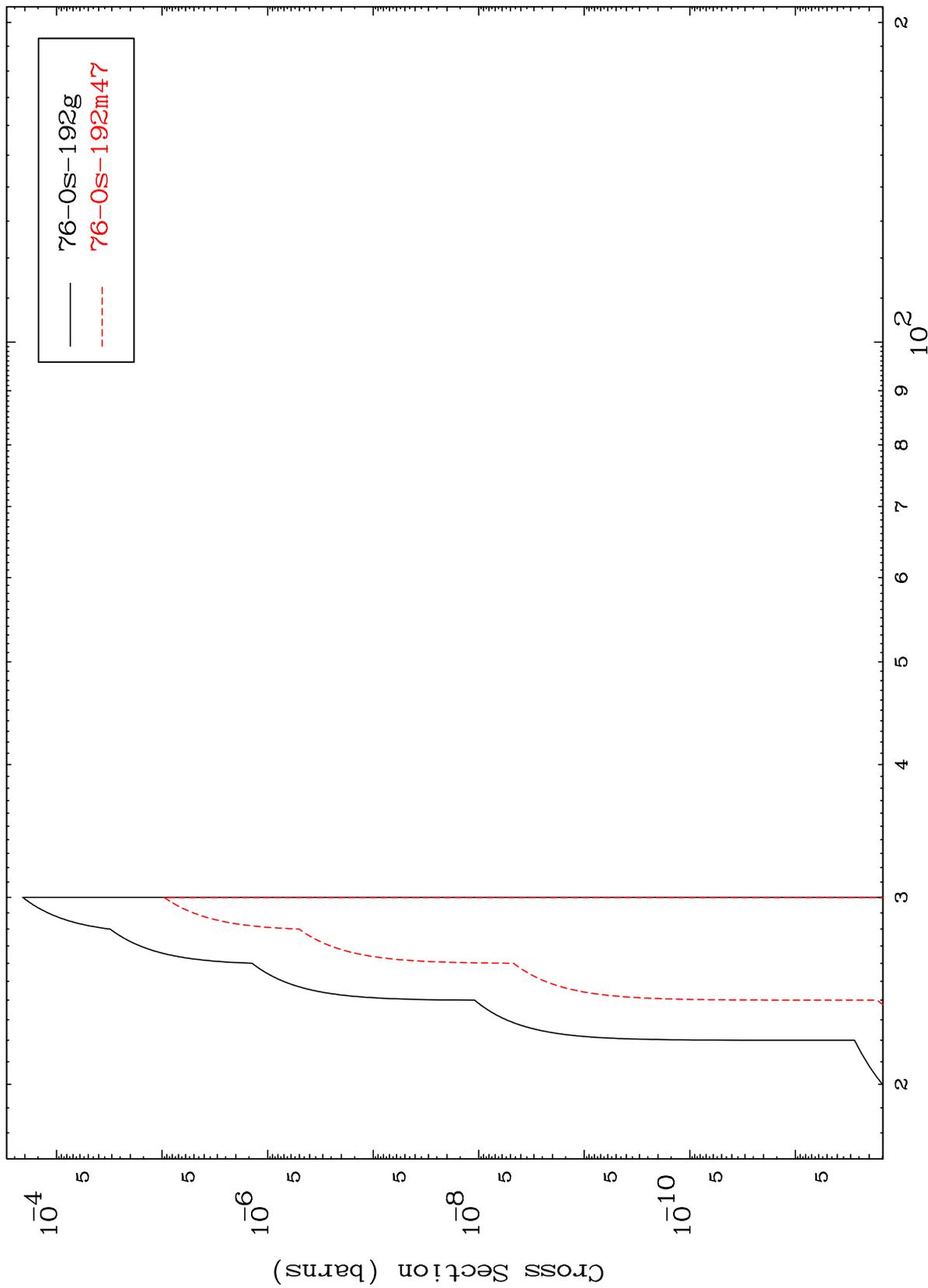
14

MAT 7543

(n,n') d

75-Re-191

Radionuclide Production Cross Section



15

Incident Energy (MeV)

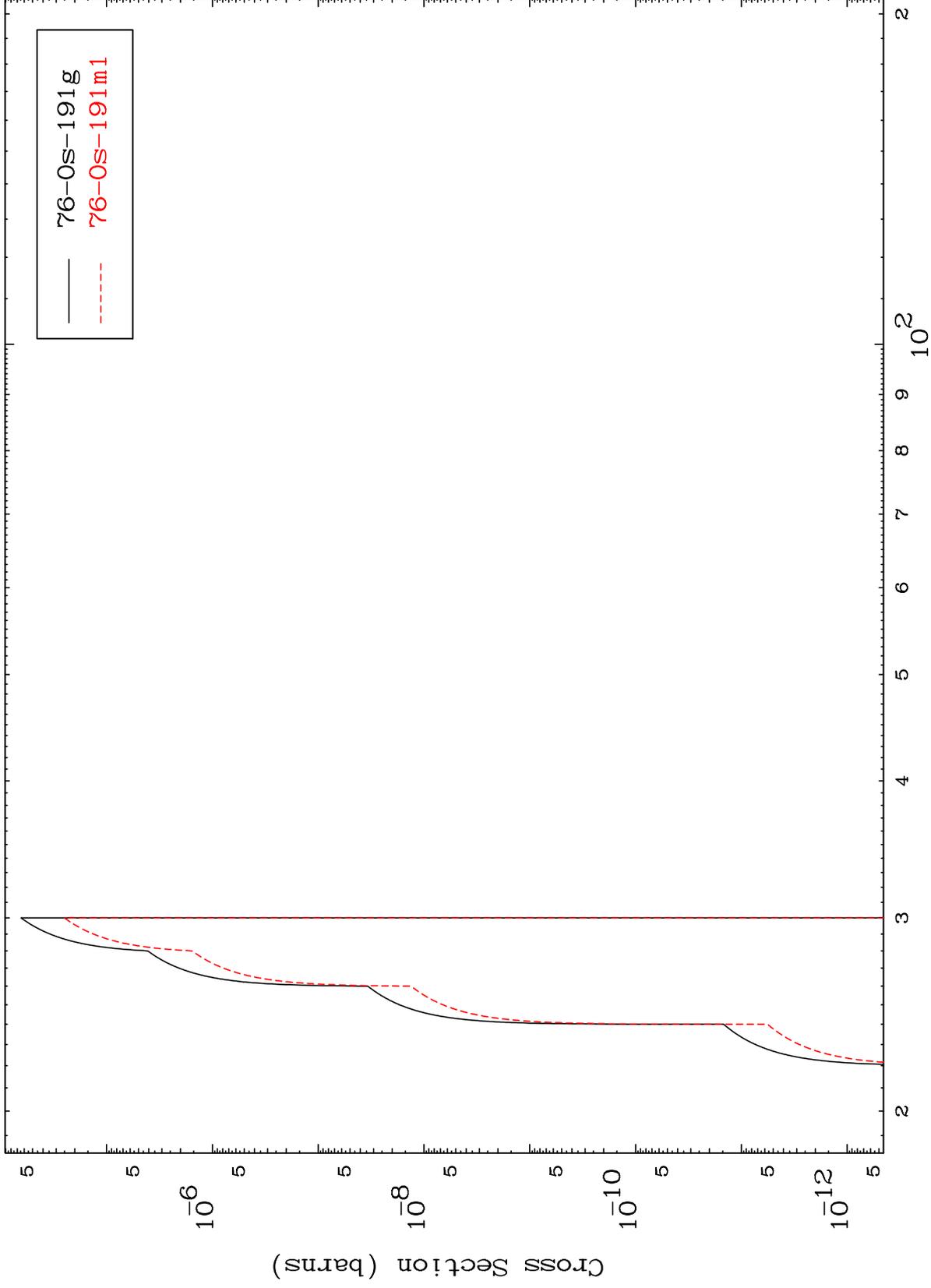
75-Re-191

MAT 7543

(n,n') t

75-Re-191

Radionuclide Production Cross Section



16

Incident Energy (MeV)

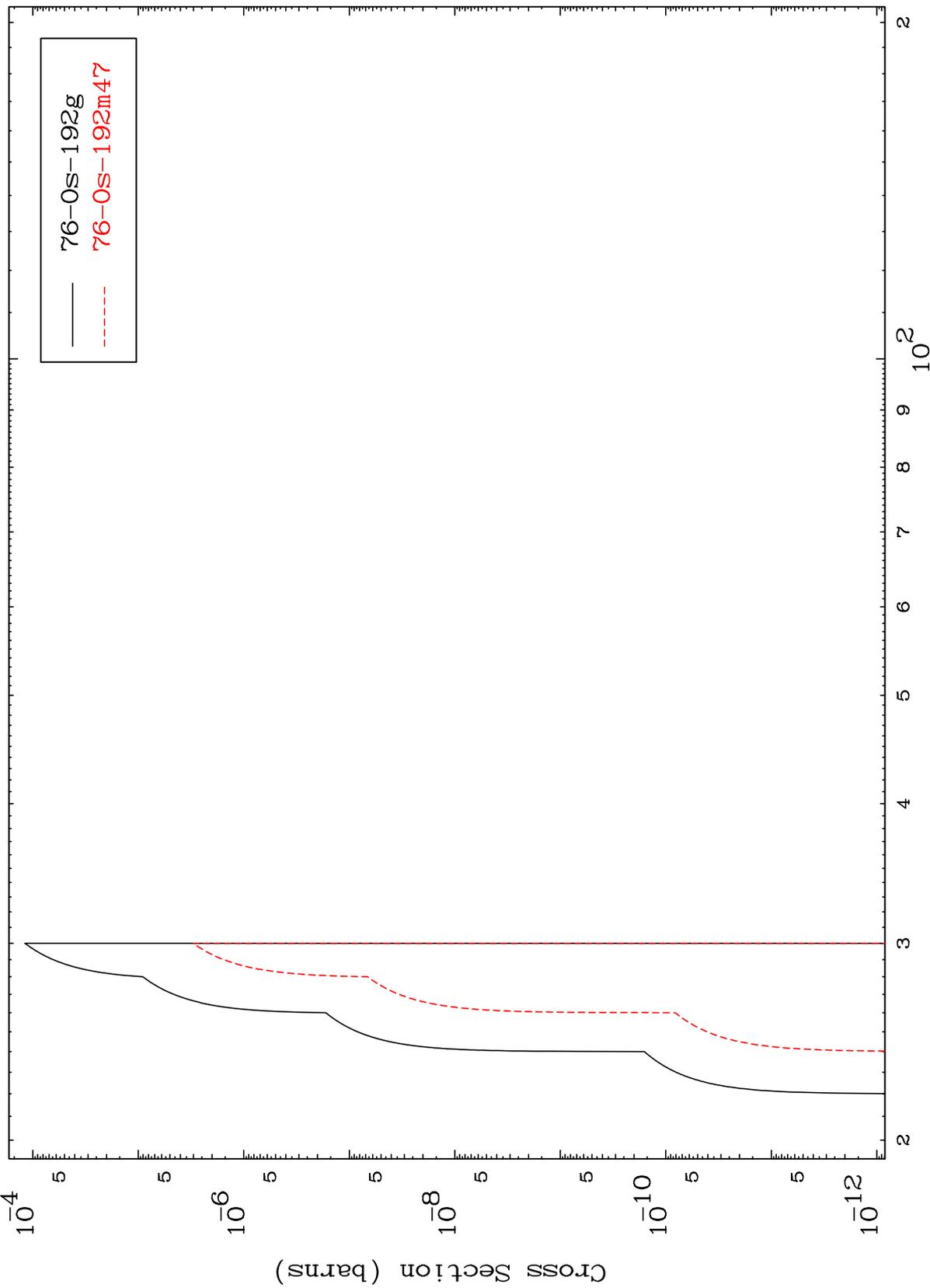
75-Re-191

MAT 7543

(n,2n) p

75-Re-191

Radionuclide Production Cross Section



17

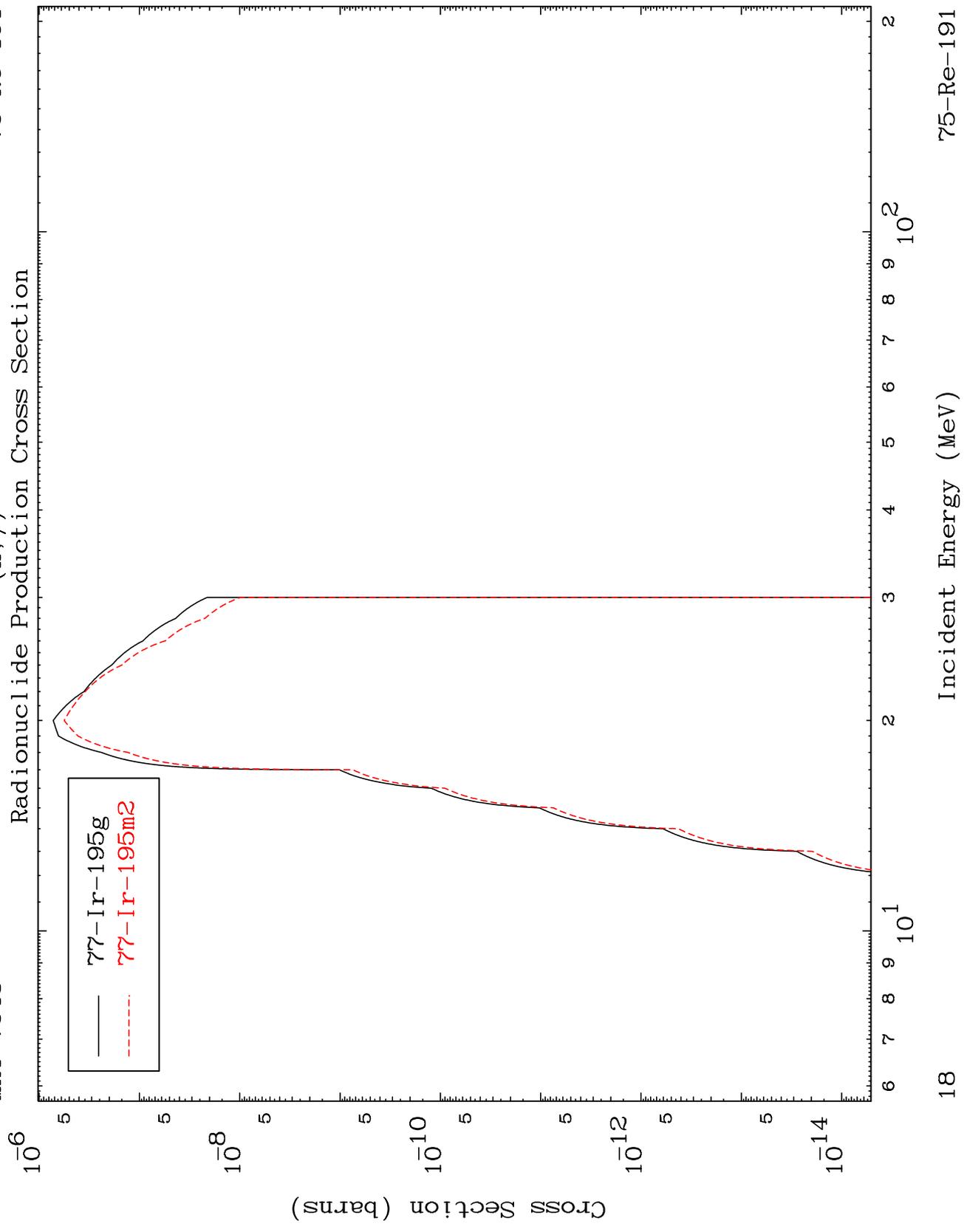
Incident Energy (MeV)

75-Re-191

MAT 7543

75-Re-191

Radionuclide Production Cross Section (n,γ)



18

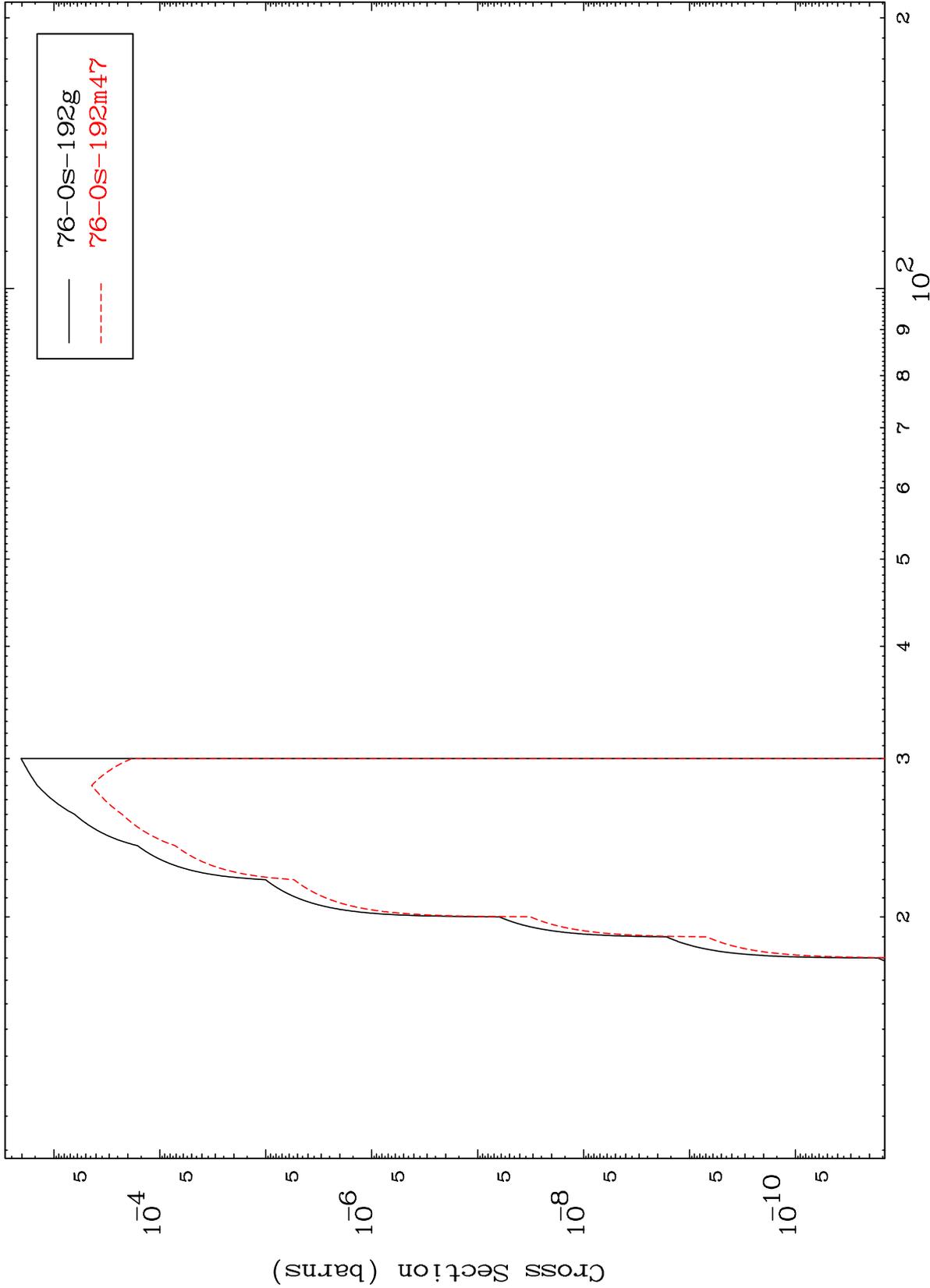
Incident Energy (MeV)

75-Re-191

MAT 7543

75-Re-191

(n, t)
Radionuclide Production Cross Section



19

Incident Energy (MeV)

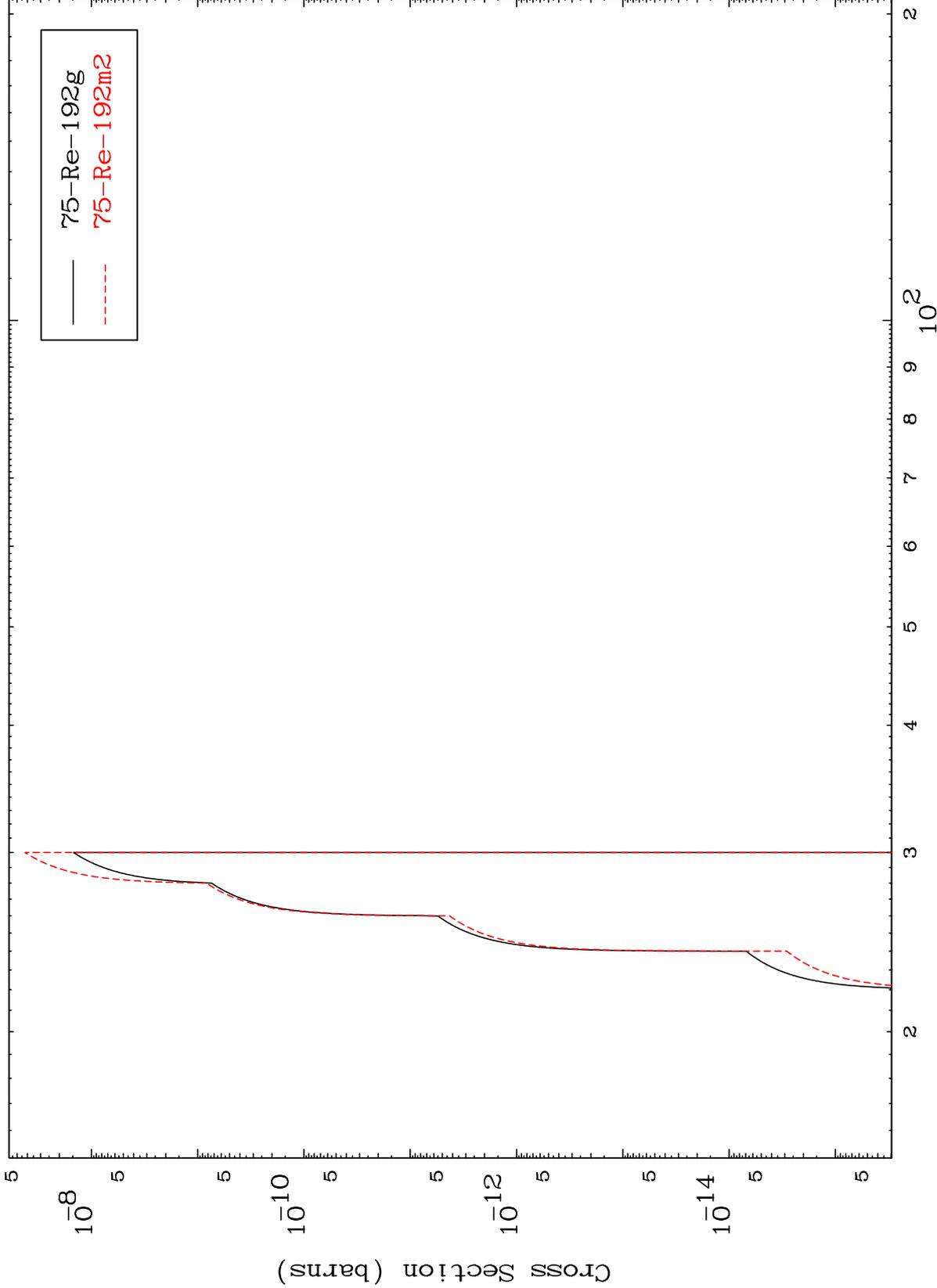
75-Re-191

MAT 7543

(n,He-3)

75-Re-191

Radionuclide Production Cross Section



20

Incident Energy (MeV)

75-Re-191