

Program EVALPLOT  
(Version 2021-1)

by

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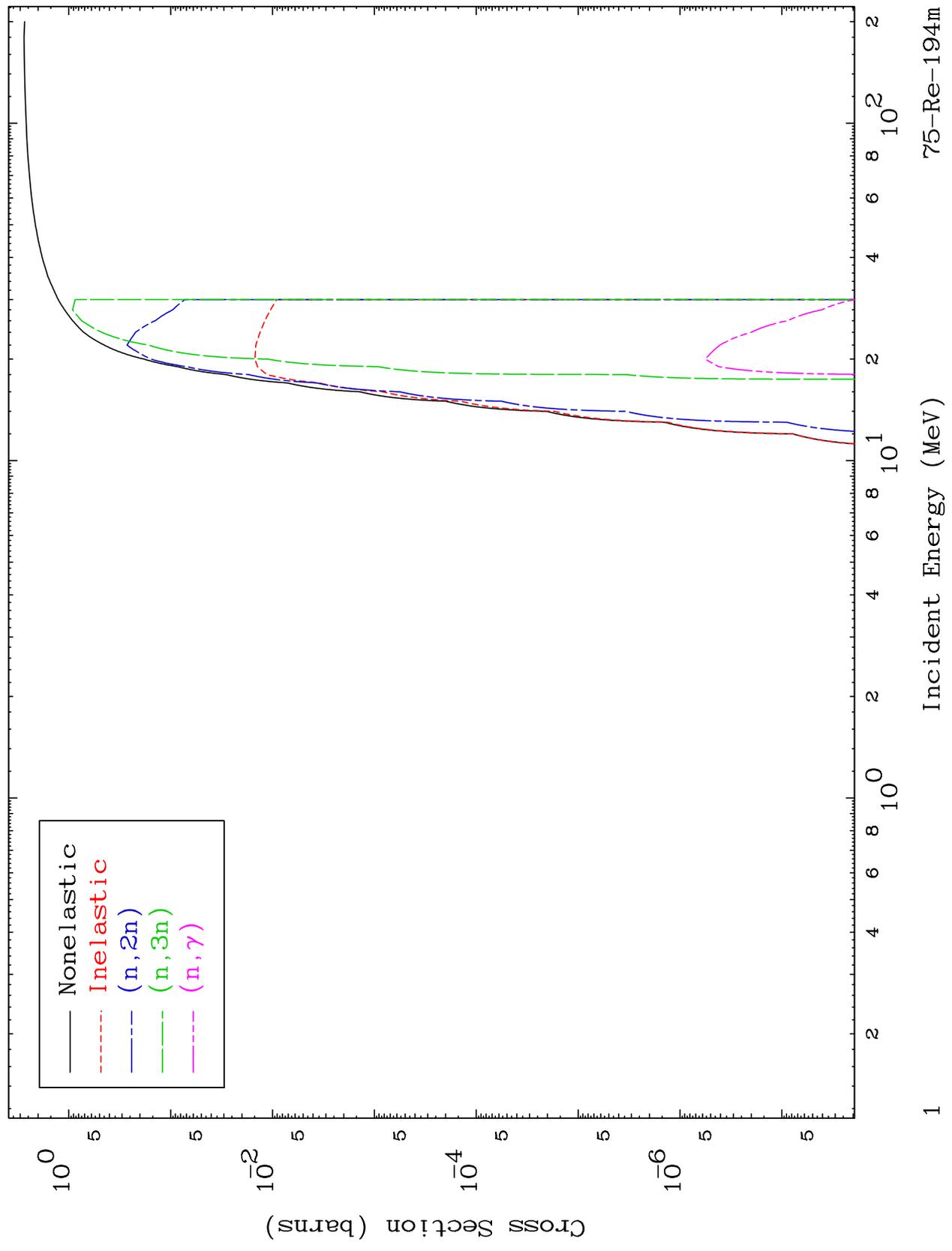
Press Mouse Button to Start

MAT 7553

0 Kelvin

$\alpha$  Major Cross Sections

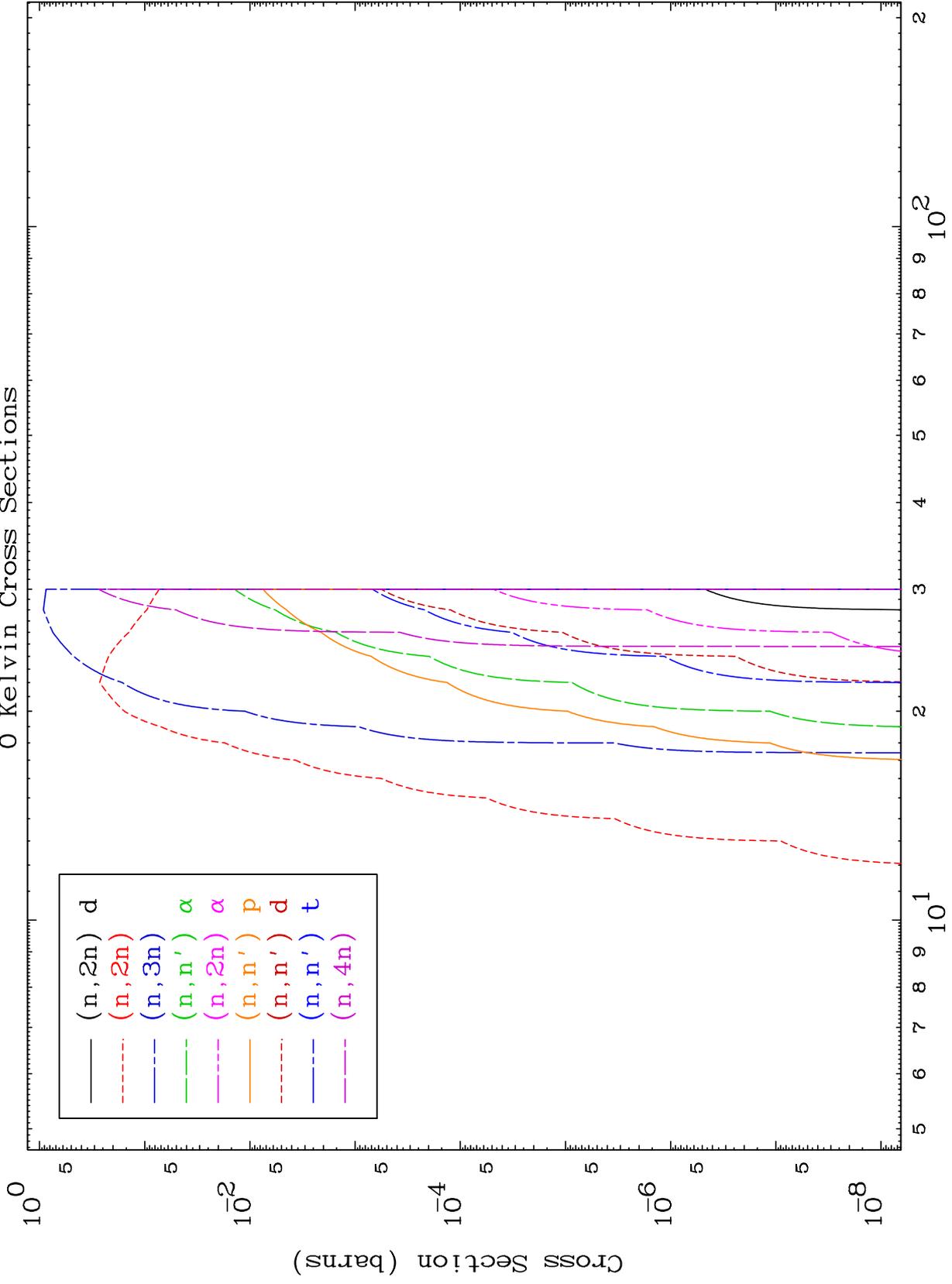
75-Re-194m



MAT 7553

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

75-Re-194m



Incident Energy (MeV)

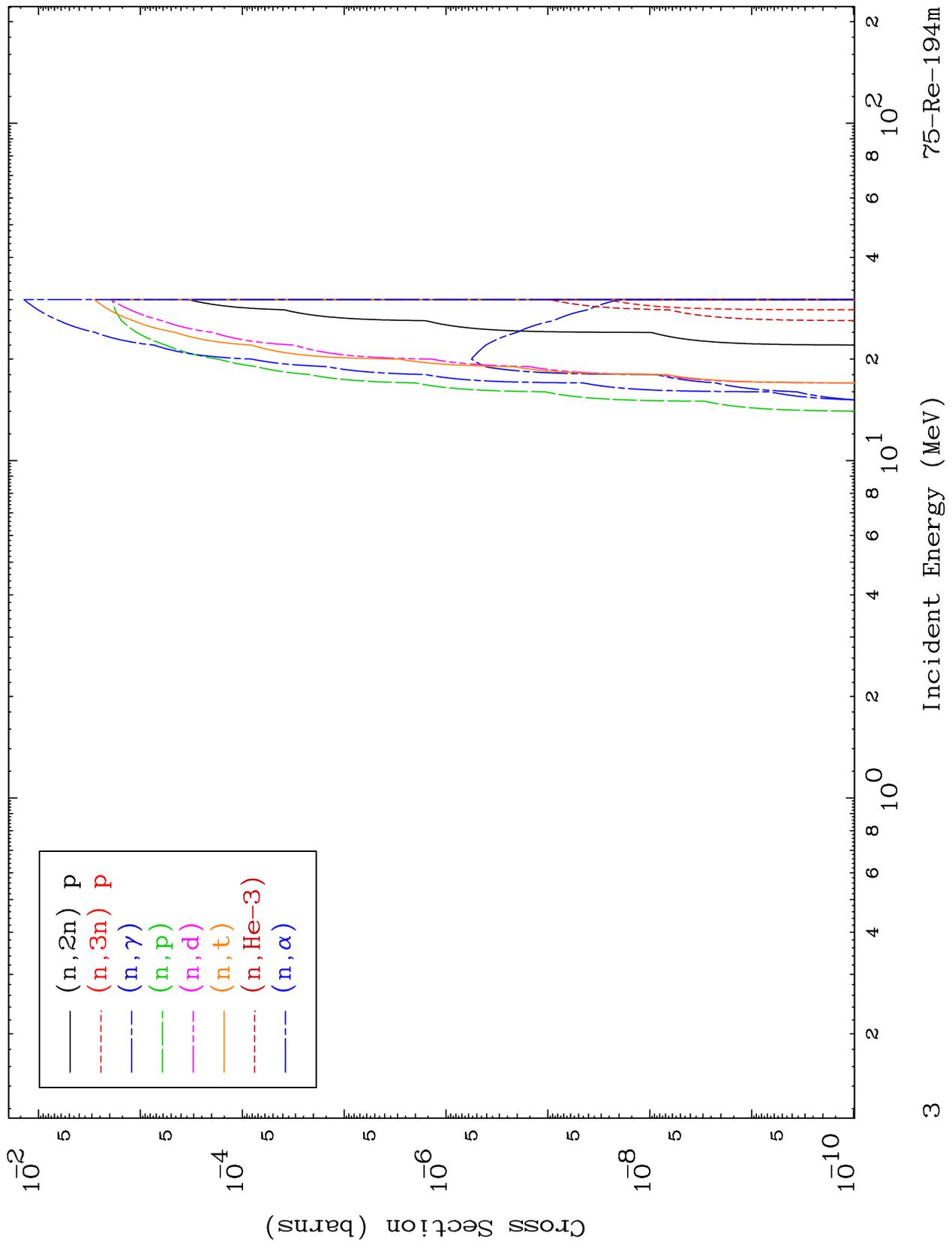
75-Re-194m

2

MAT 7553

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

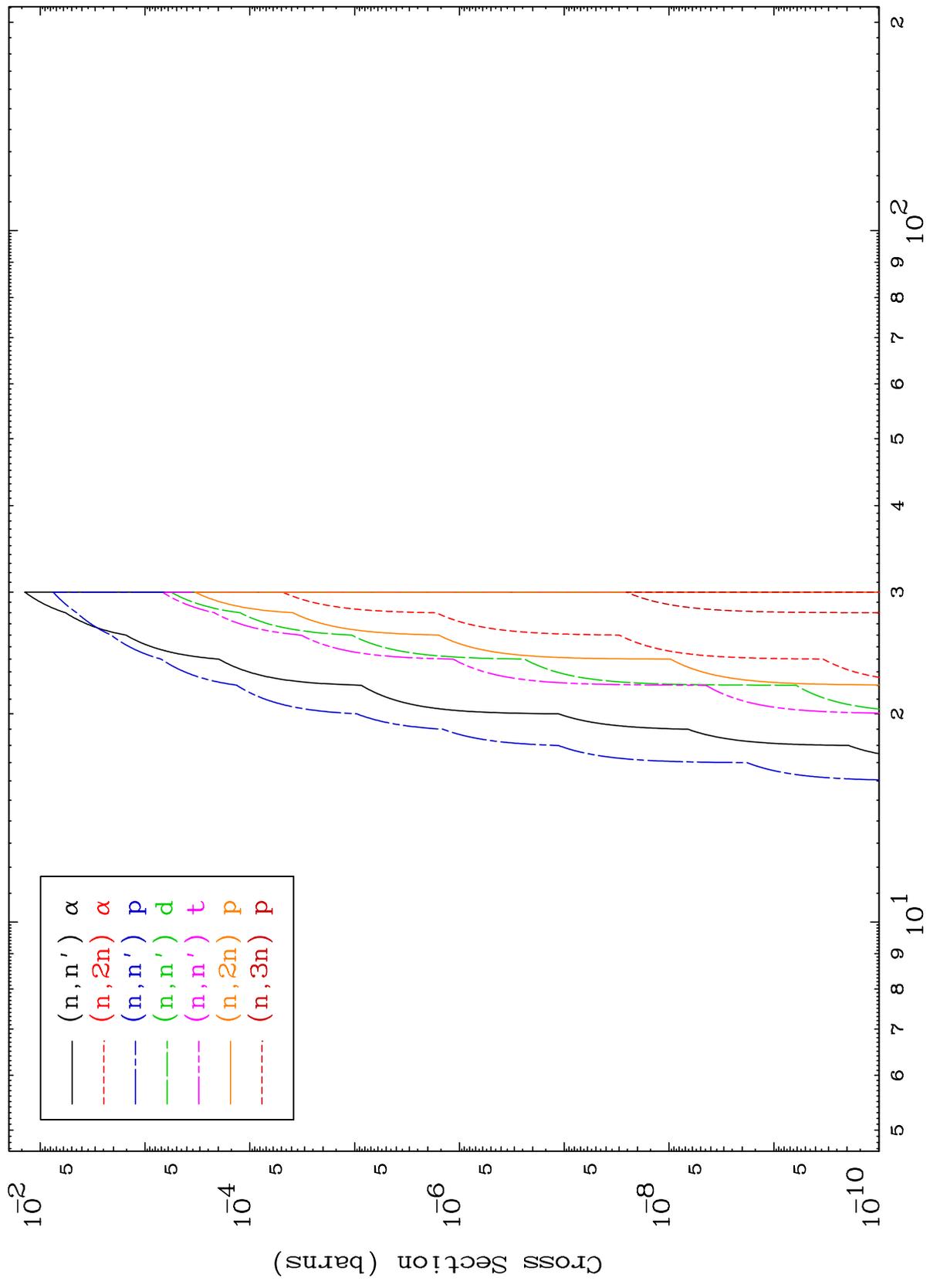
75-Re-194m



MAT 7553

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

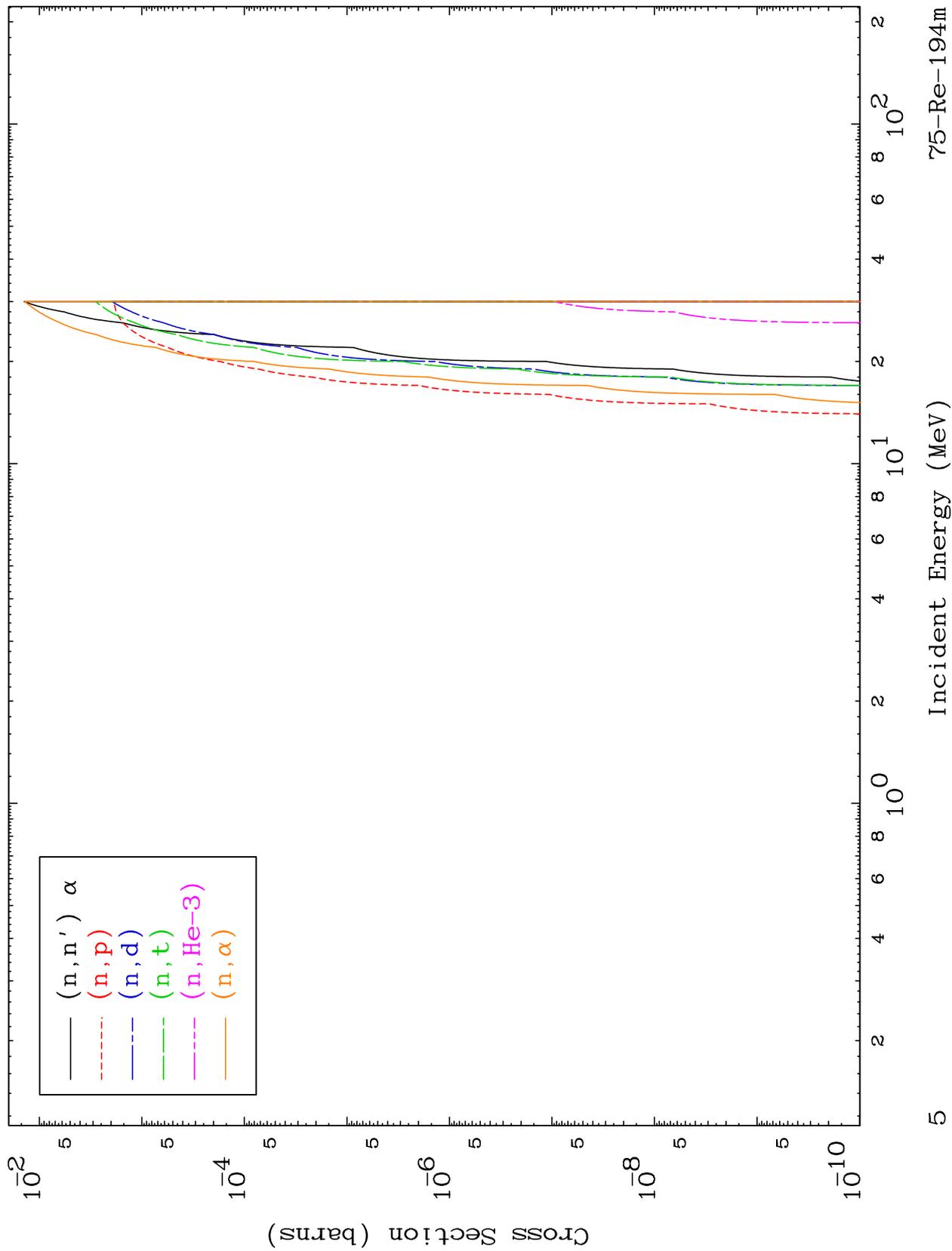
75-Re-194m



MAT 7553

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

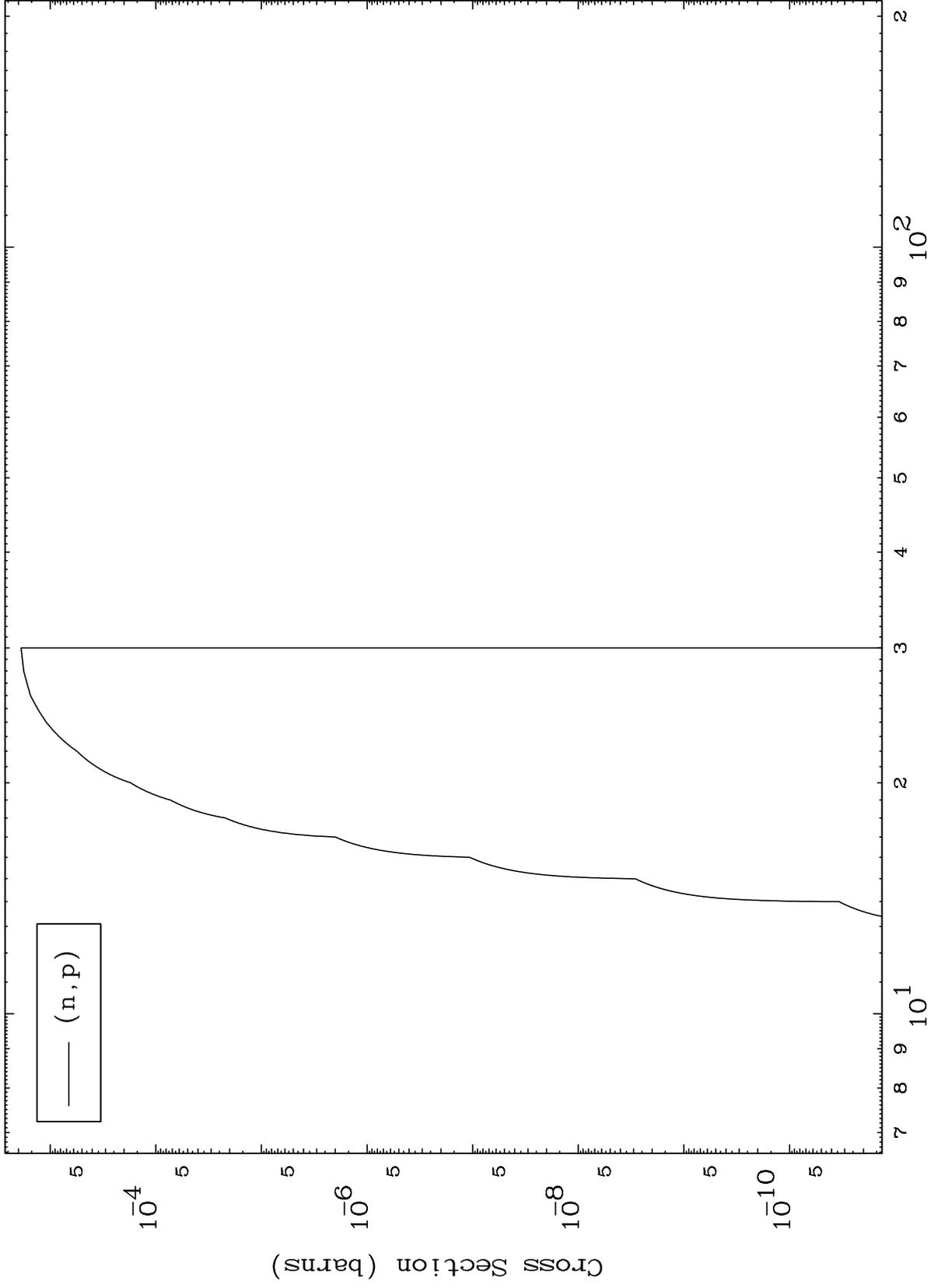
75-Re-194m



MAT 7553

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

75-Re-194m



6

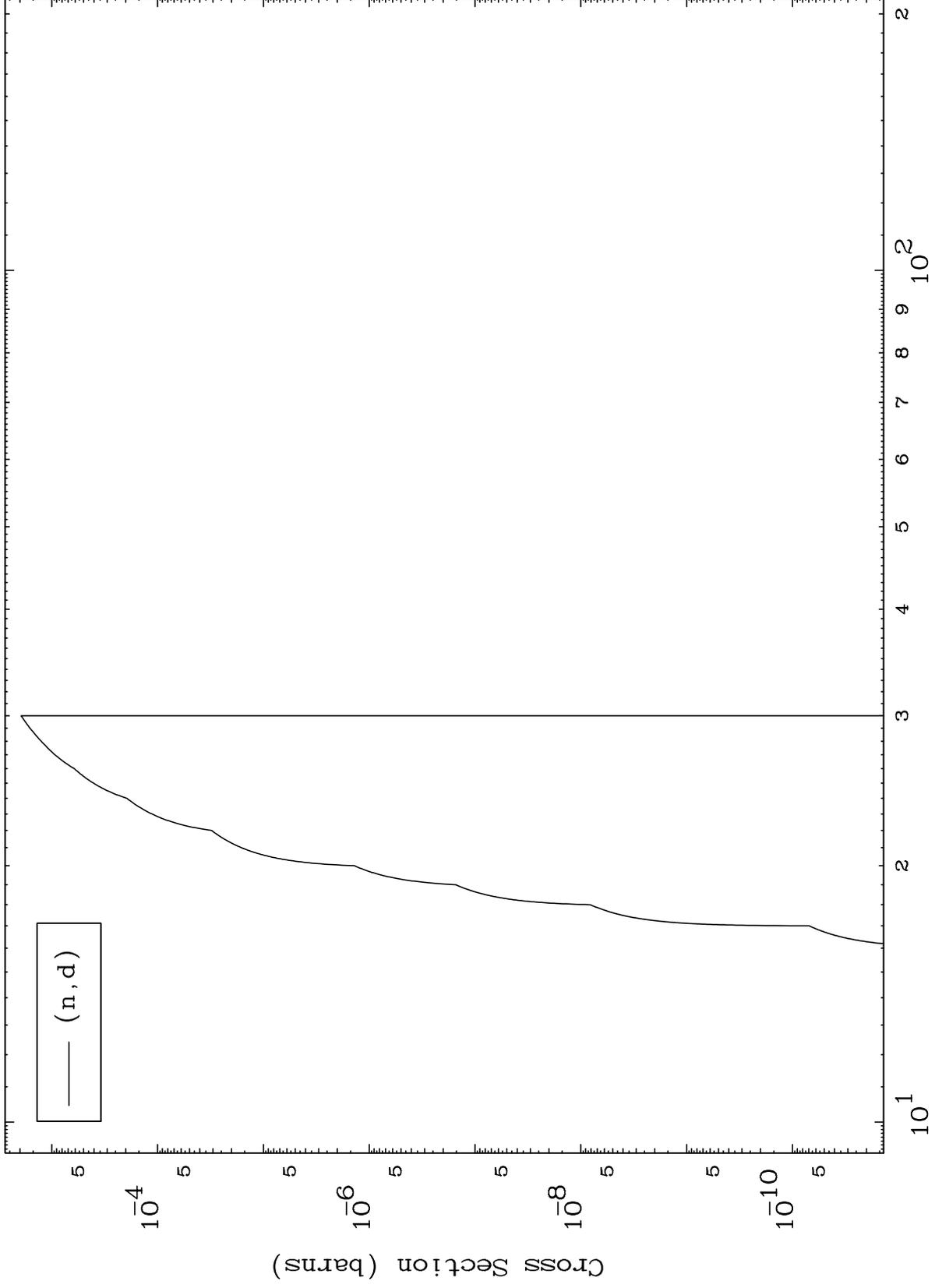
Incident Energy (MeV)

75-Re-194m

MAT 7553

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

75-Re-194m



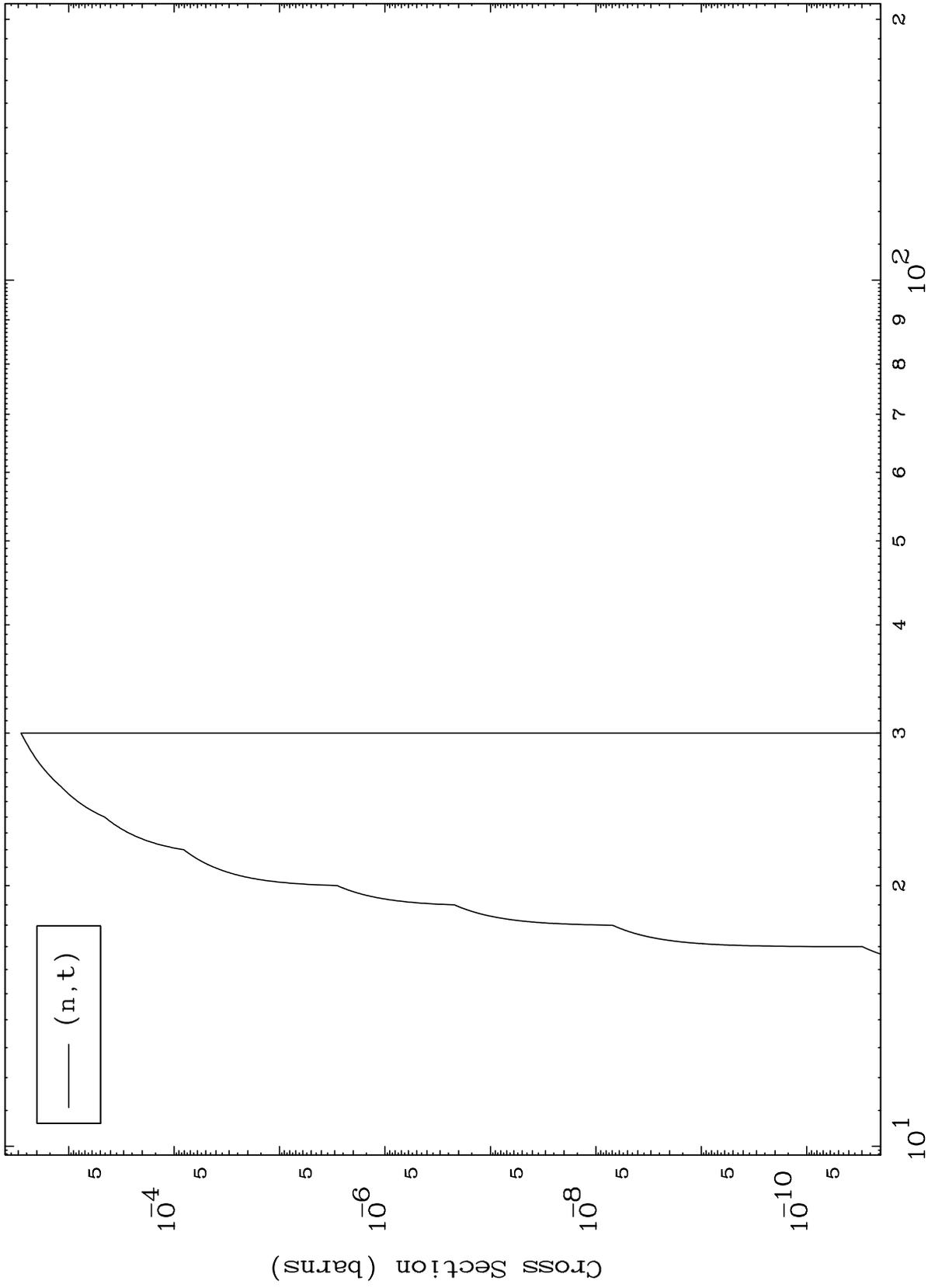
Incident Energy (MeV)

75-Re-194m

MAT 7553

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

75-Re-194m



75-Re-194m

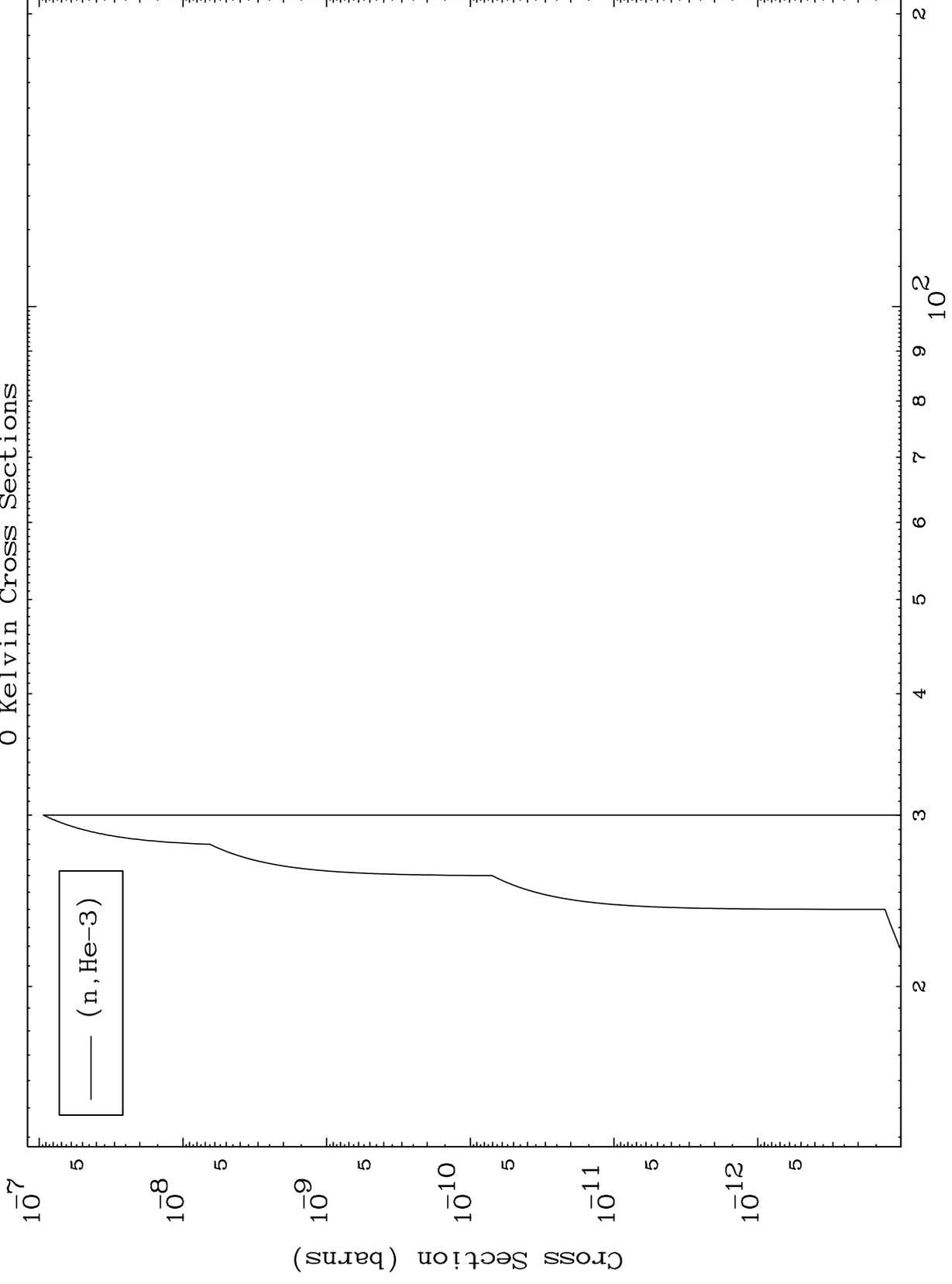
Incident Energy (MeV)

MAT 7553

( $\alpha, \text{He}3$ ) Levels

75-Re-194m

0 Kelvin Cross Sections

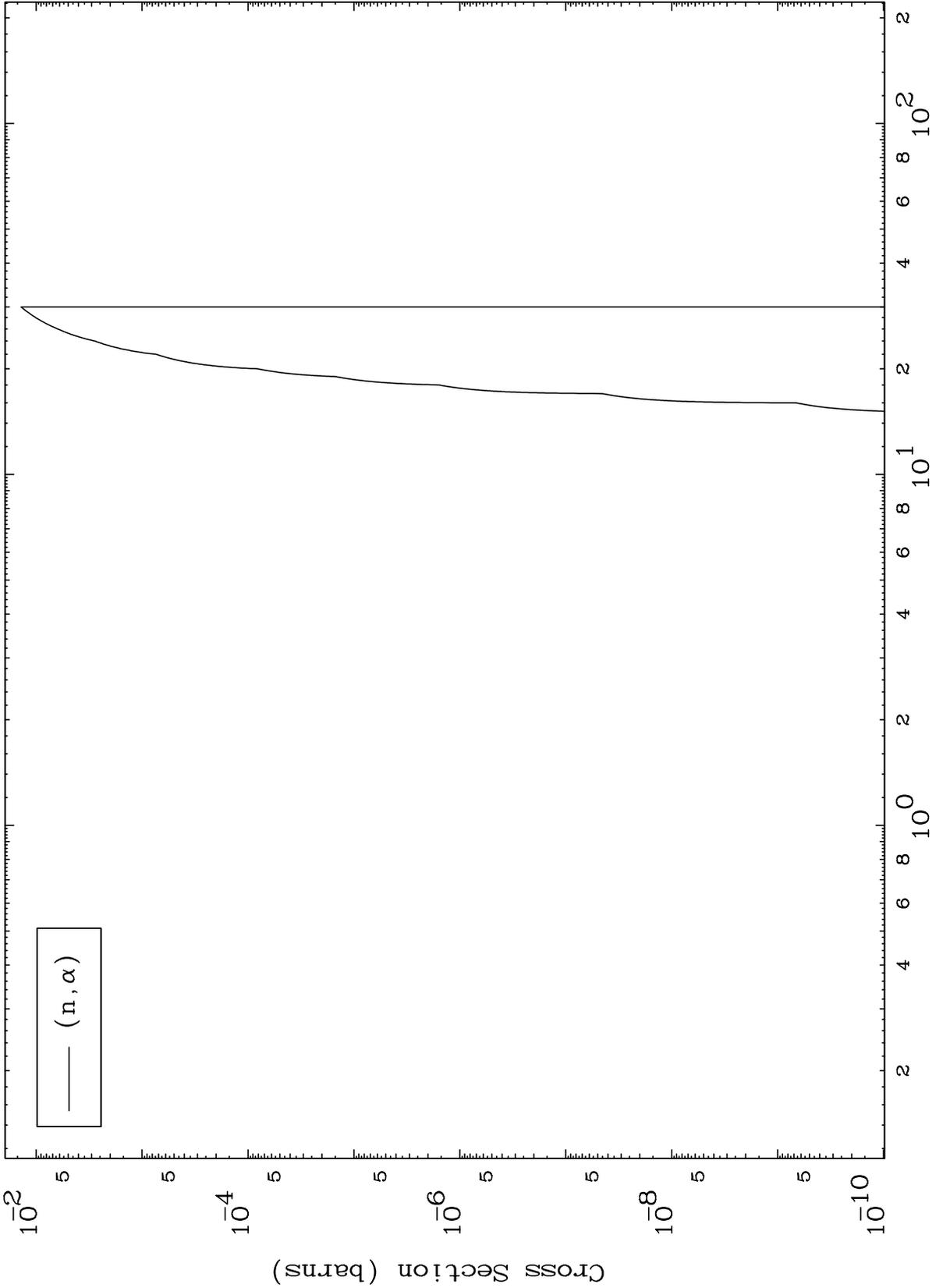


MAT 7553

( $\alpha, \alpha$ ) Levels

75-Re-194m

0 Kelvin Cross Sections



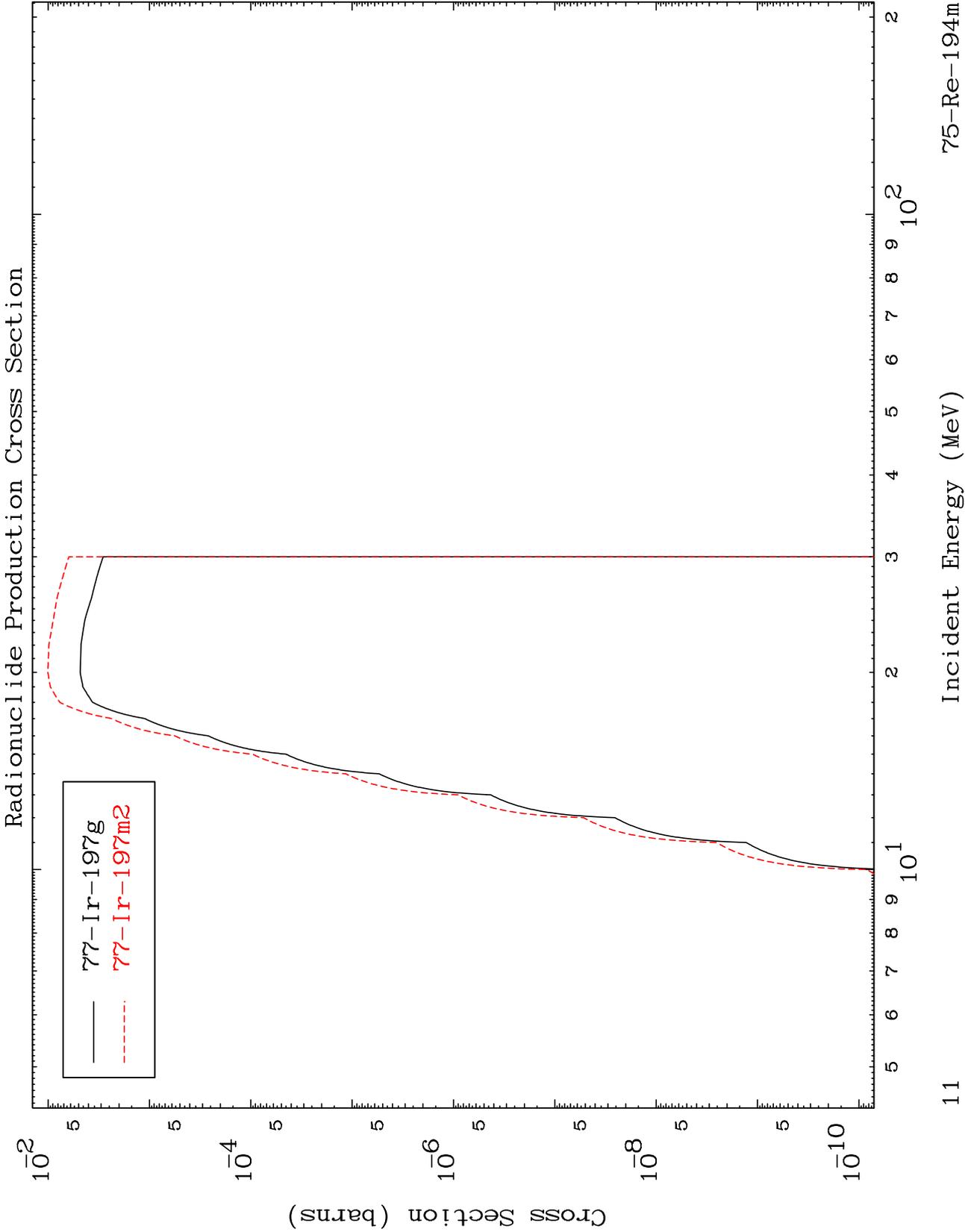
10

Incident Energy (MeV)

75-Re-194m

MAT 7553

75-Re-194m

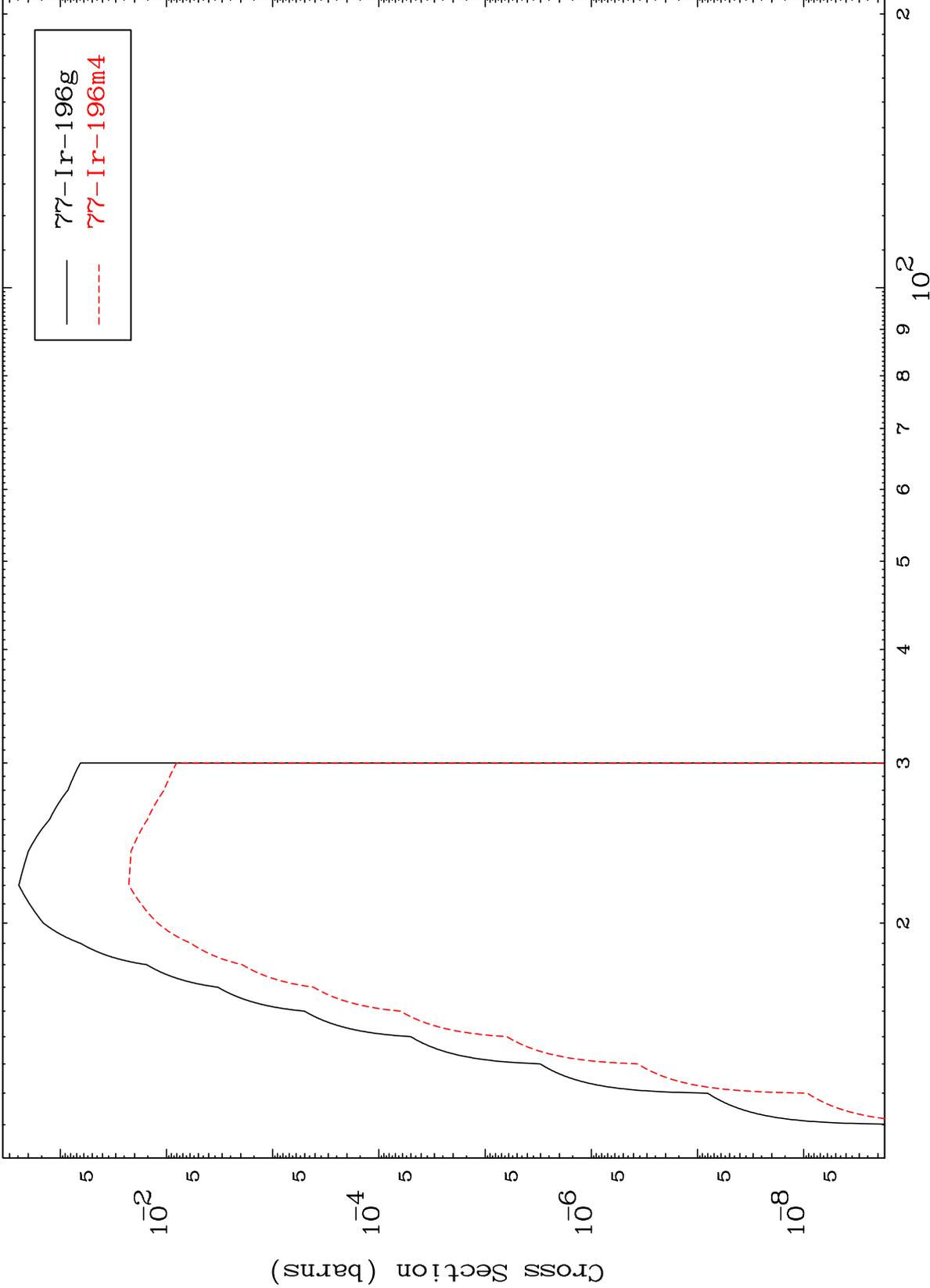


MAT 7553

(n,2n)

75-Re-194m

Radionuclide Production Cross Section



12

Incident Energy (MeV)

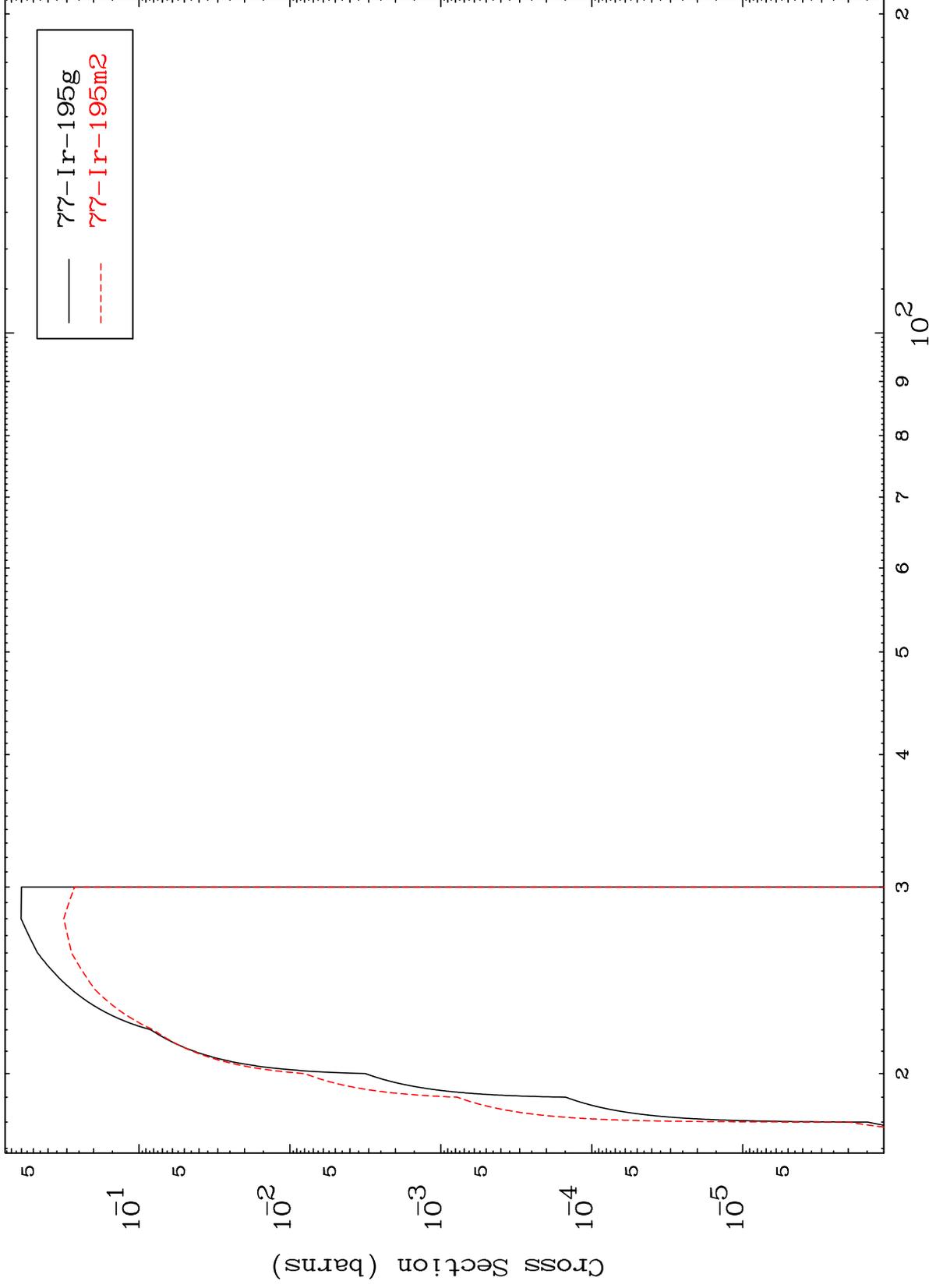
75-Re-194m

MAT 7553

(n,3n)

75-Re-194m

Radionuclide Production Cross Section

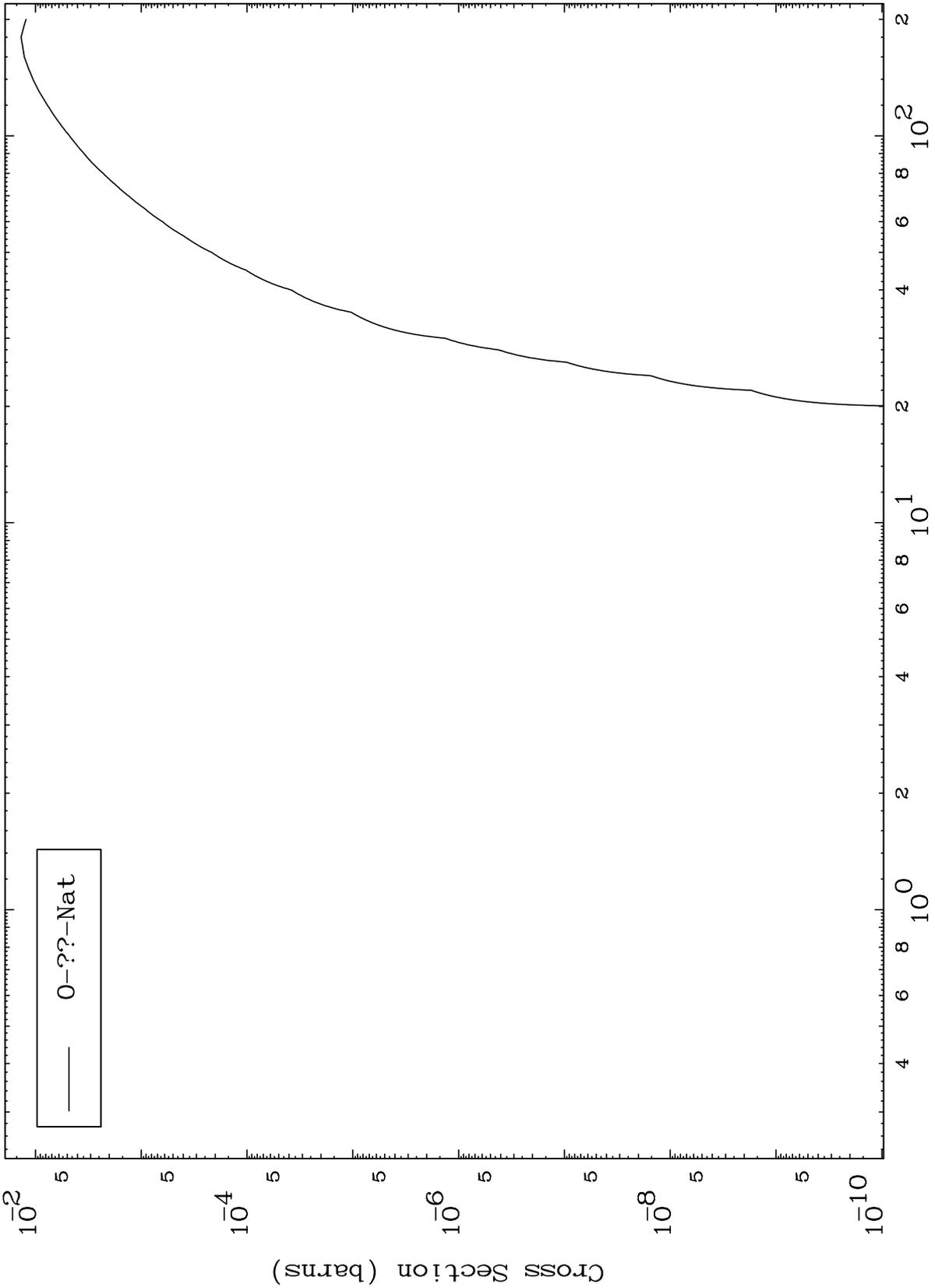


MAT 7553

Fission

75-Re-194m

Radionuclide Production Cross Section

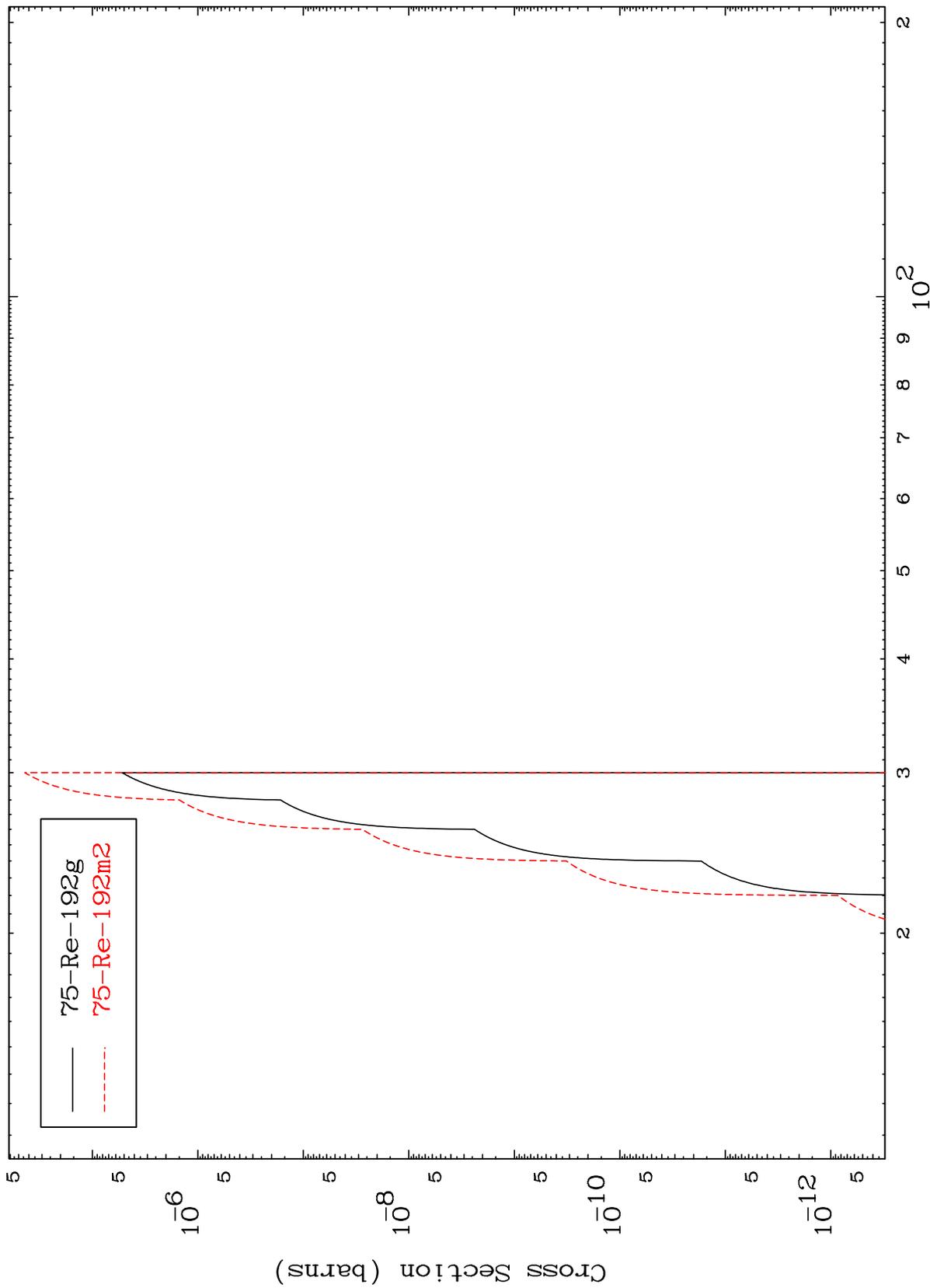


MAT 7553

(n,2n)  $\alpha$

75-Re-194m

Radionuclide Production Cross Section



15

Incident Energy (MeV)

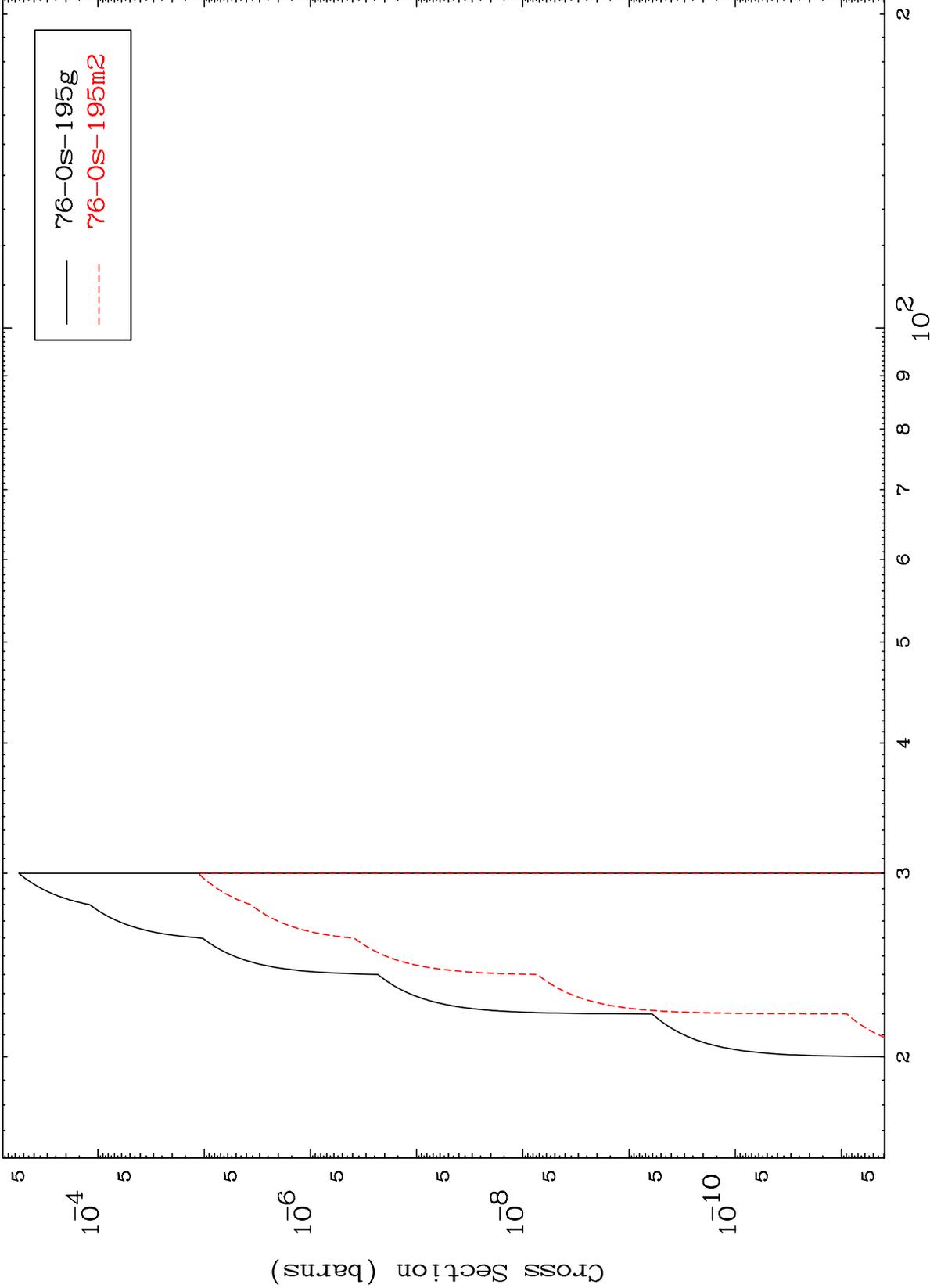
75-Re-194m

MAT 7553

(n,n') d

75-Re-194m

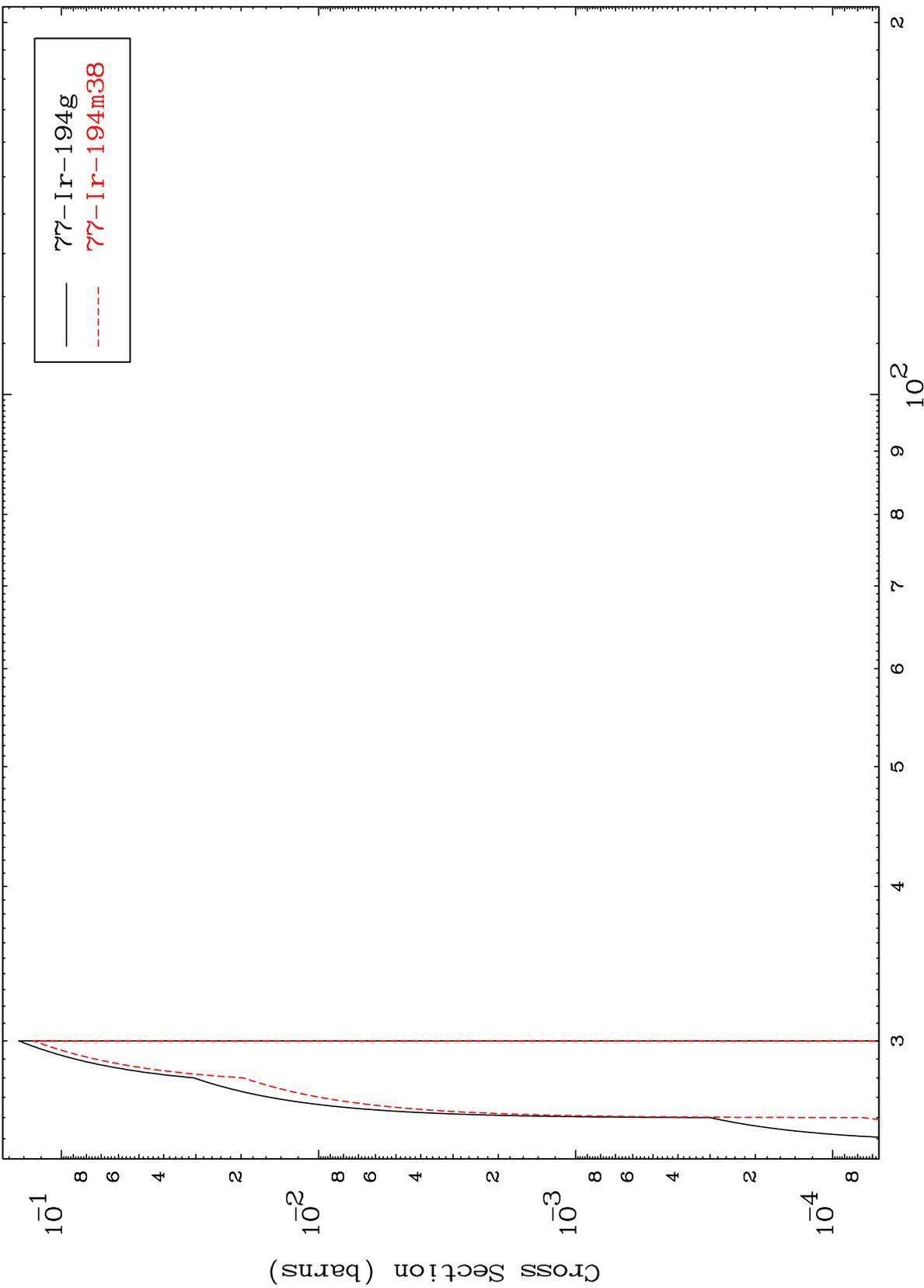
Radionuclide Production Cross Section



MAT 7553

75-Re-194m

(n,4n)  
Radionuclide Production Cross Section



17

Incident Energy (MeV)

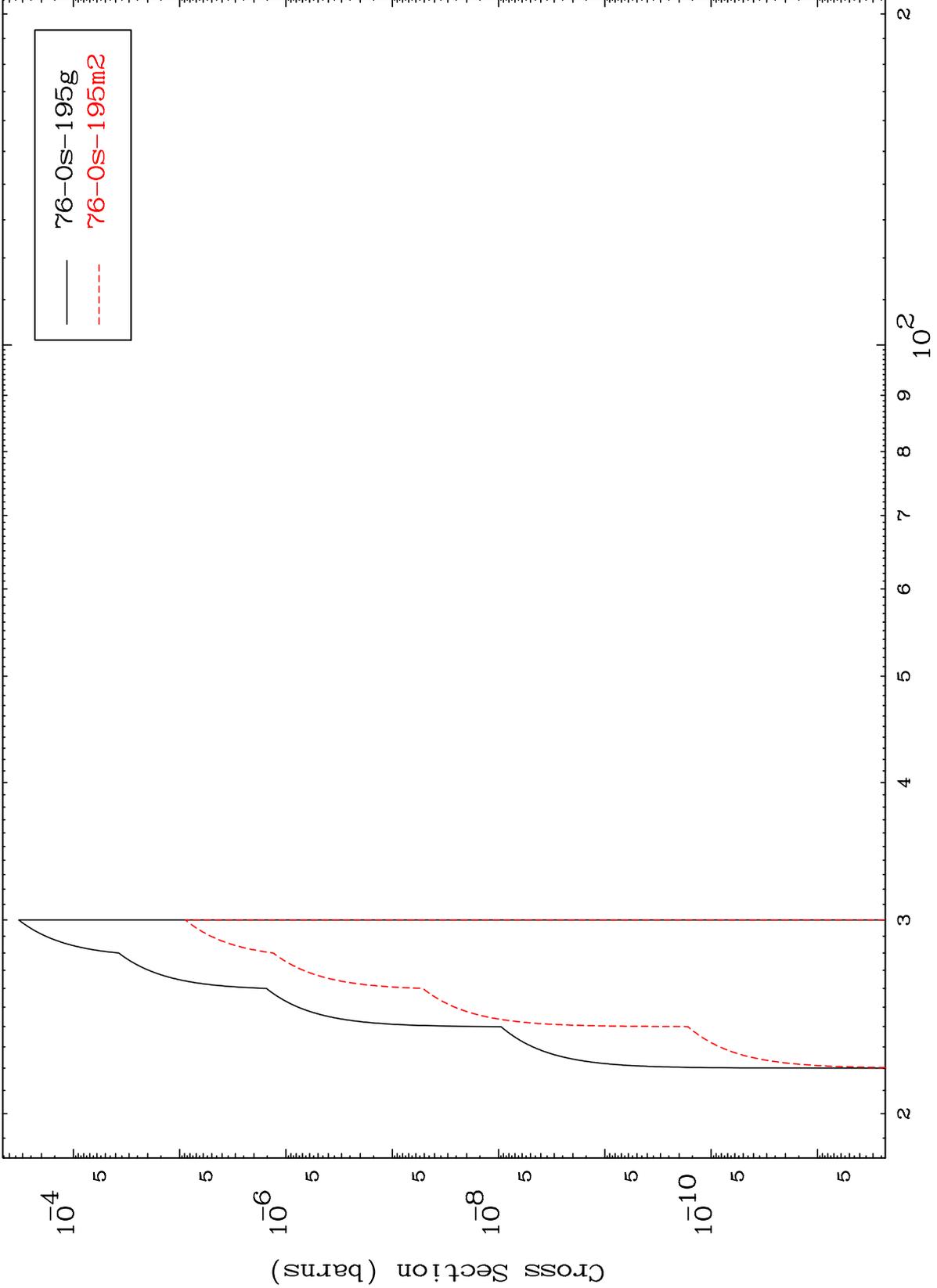
75-Re-194m

MAT 7553

(n,2n) p

75-Re-194m

Radionuclide Production Cross Section



18

Incident Energy (MeV)

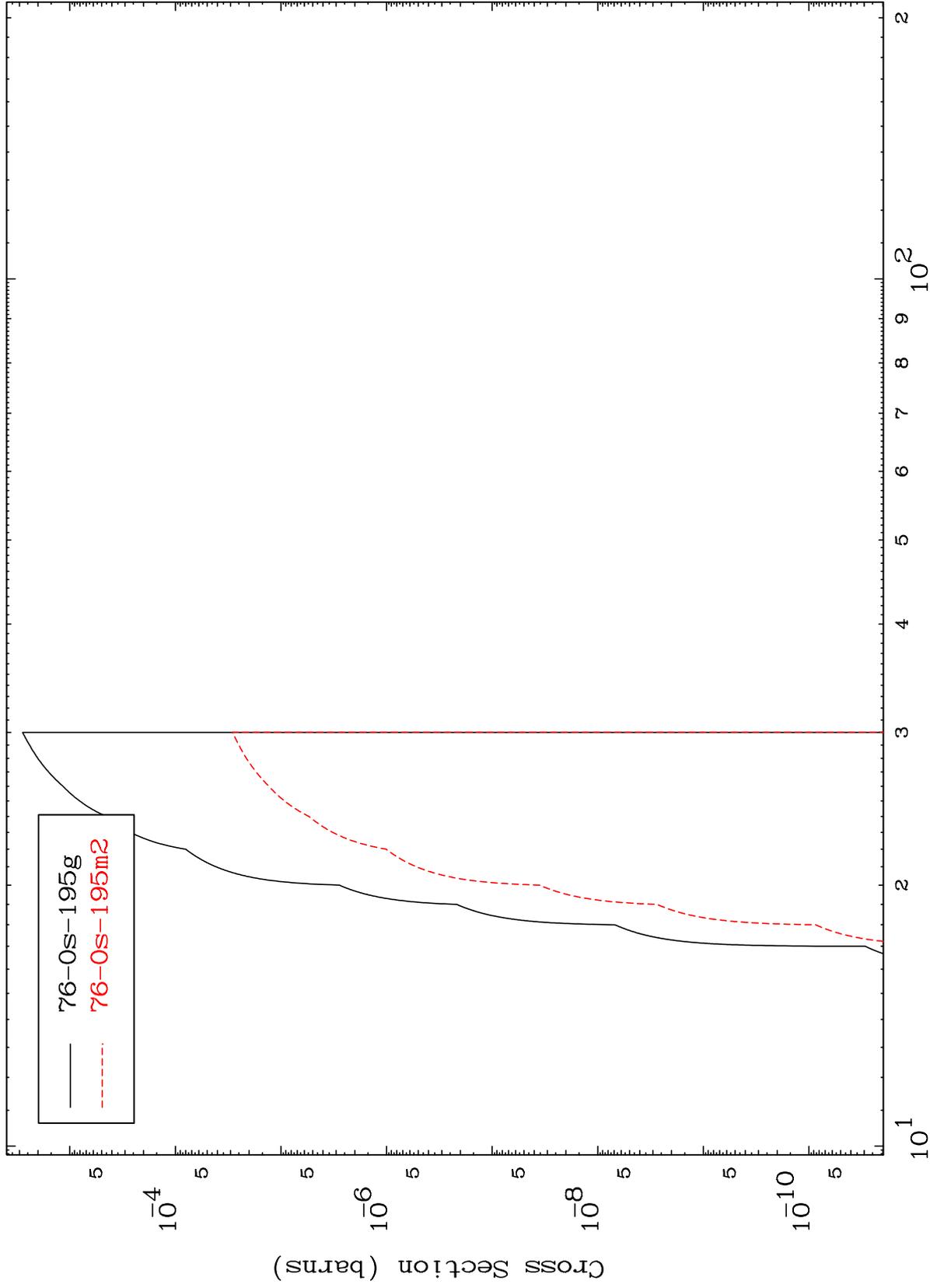
75-Re-194m

MAT 7553

(n, t)

75-Re-194m

Radionuclide Production Cross Section



Incident Energy (MeV)

75-Re-194m

MAT 7553

75-Re-194m

