

Program EVALPLOT  
(Version 2021-1)

by

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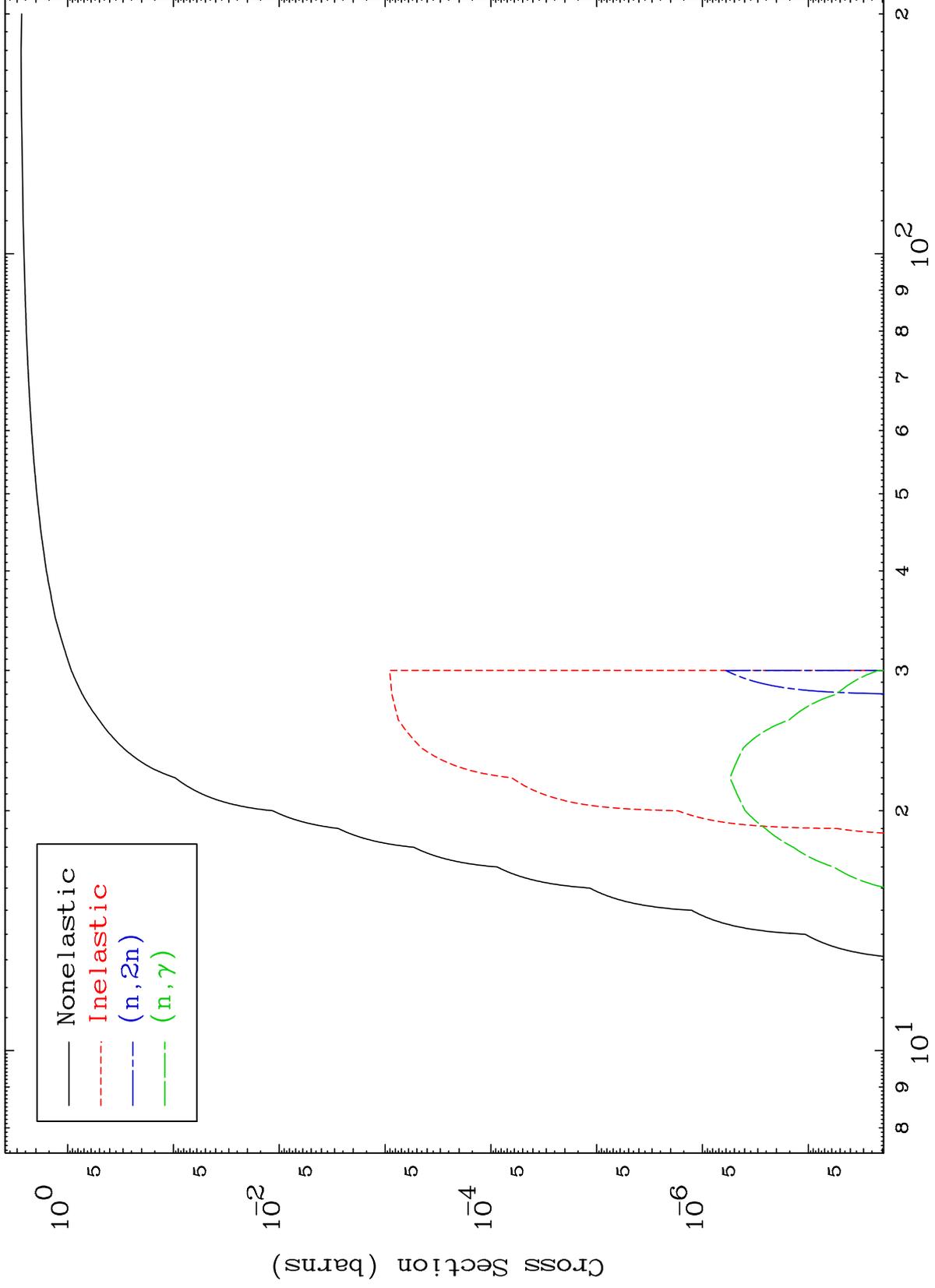
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8592

0 Kelvin  
 $\alpha$  Major  
Cross Sections

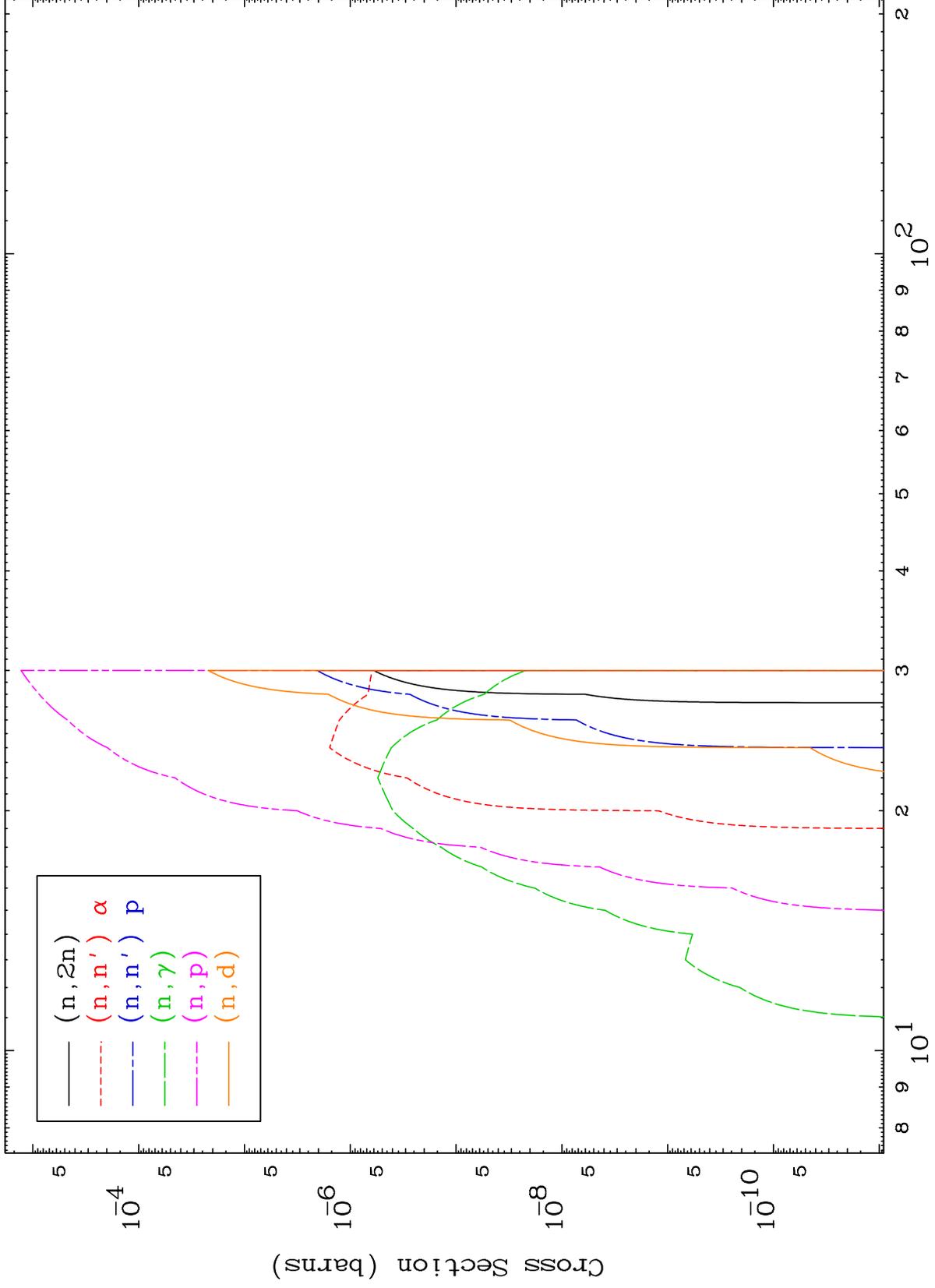
86-Rn-200



1

Incident Energy (MeV)

86-Rn-200

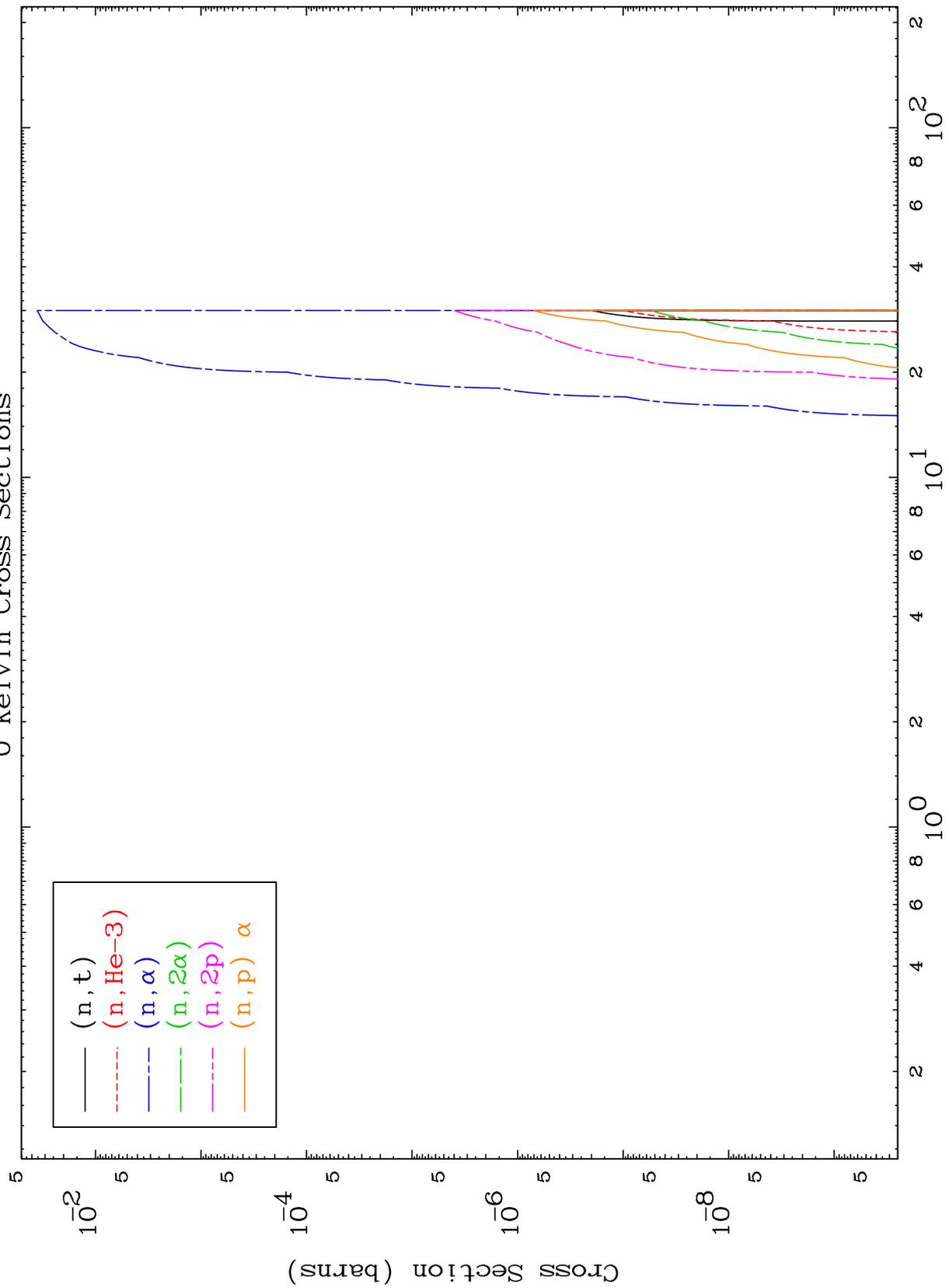


MAT 8592

$\alpha$  Neutron Absorption

86-Rn-200

0 Kelvin Cross Sections



Incident Energy (MeV)

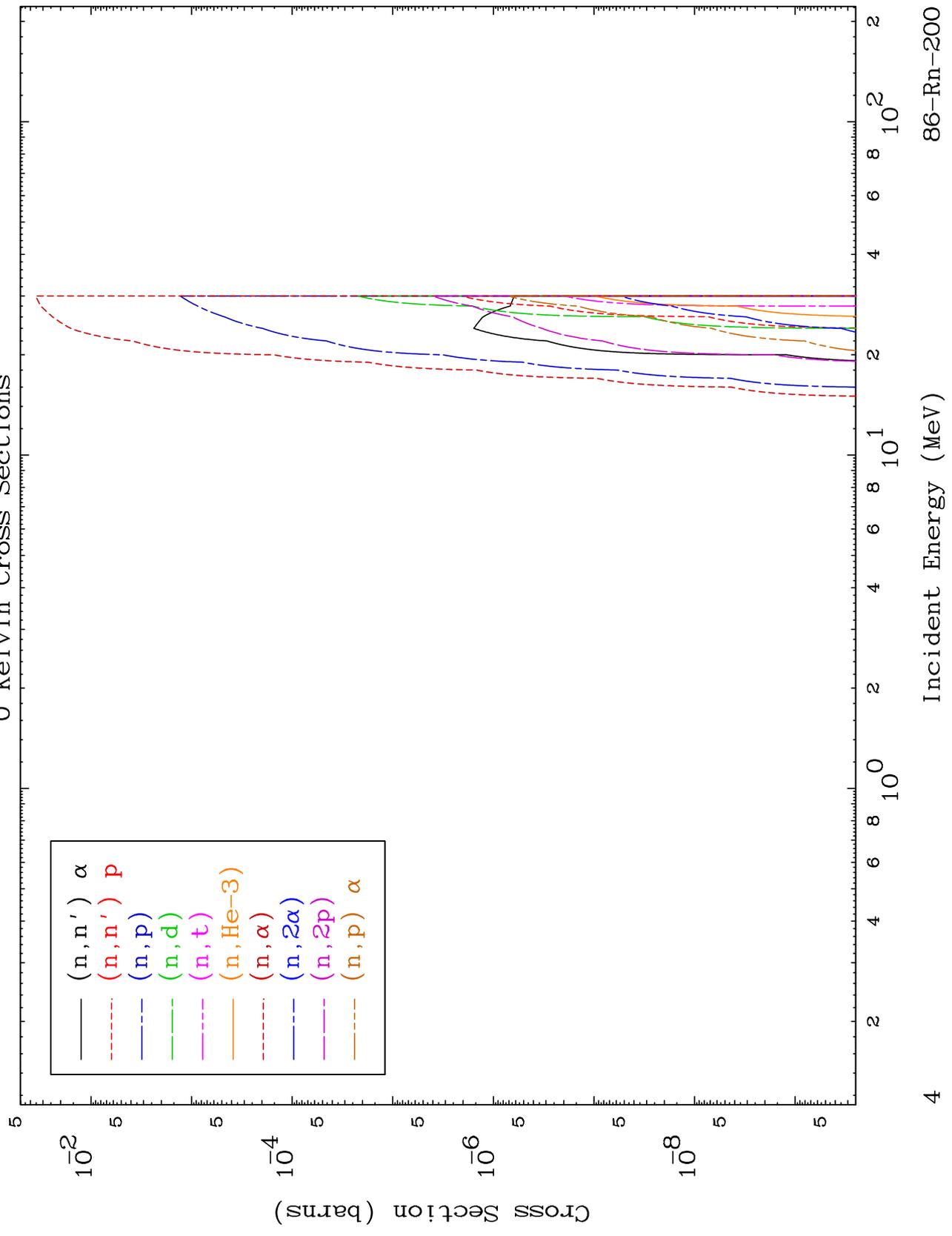
86-Rn-200

3

MAT 8592

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

86-Rn-200

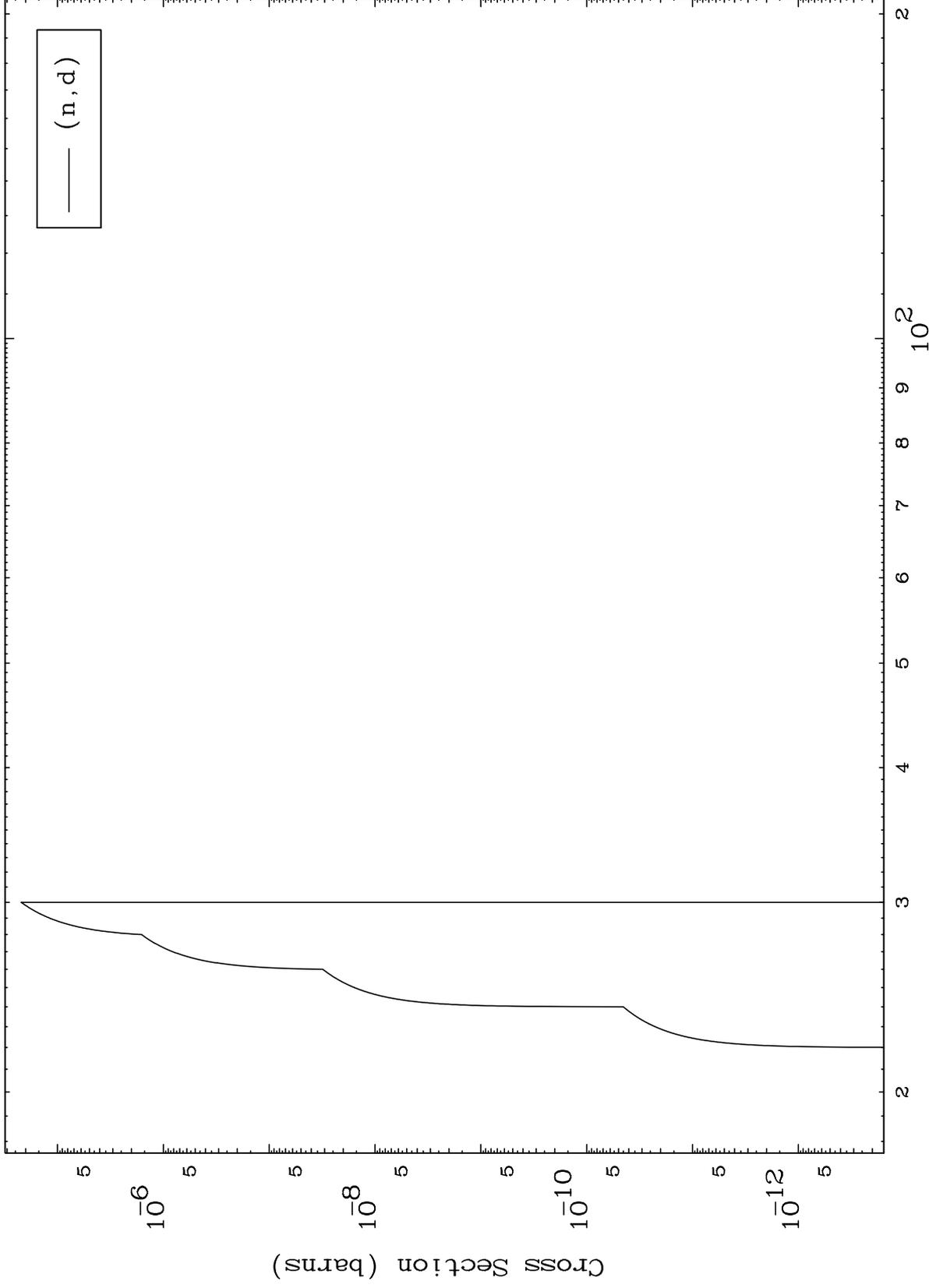




MAT 8592

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

86-Rn-200



6

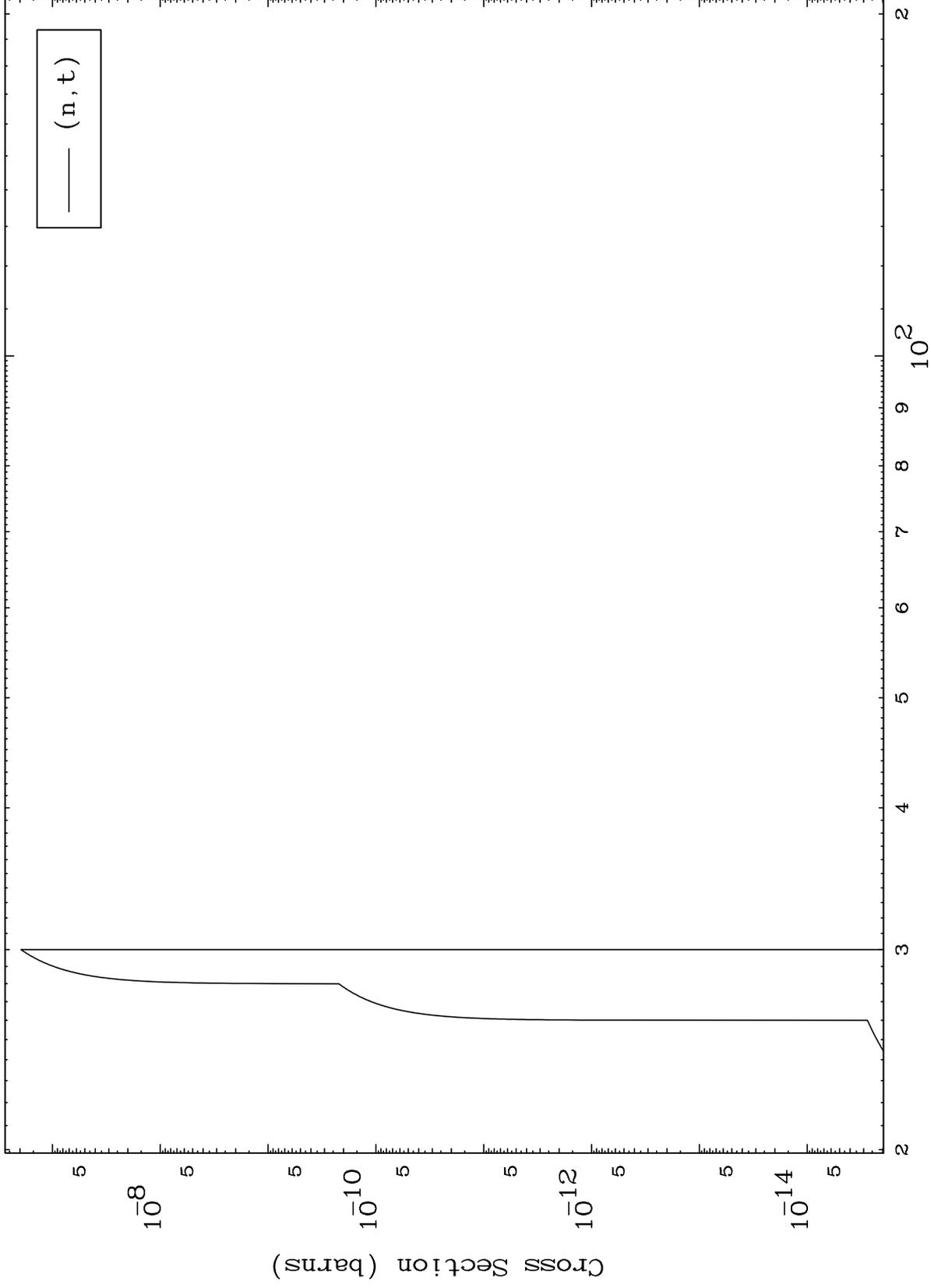
Incident Energy (MeV)

86-Rn-200

MAT 8592

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

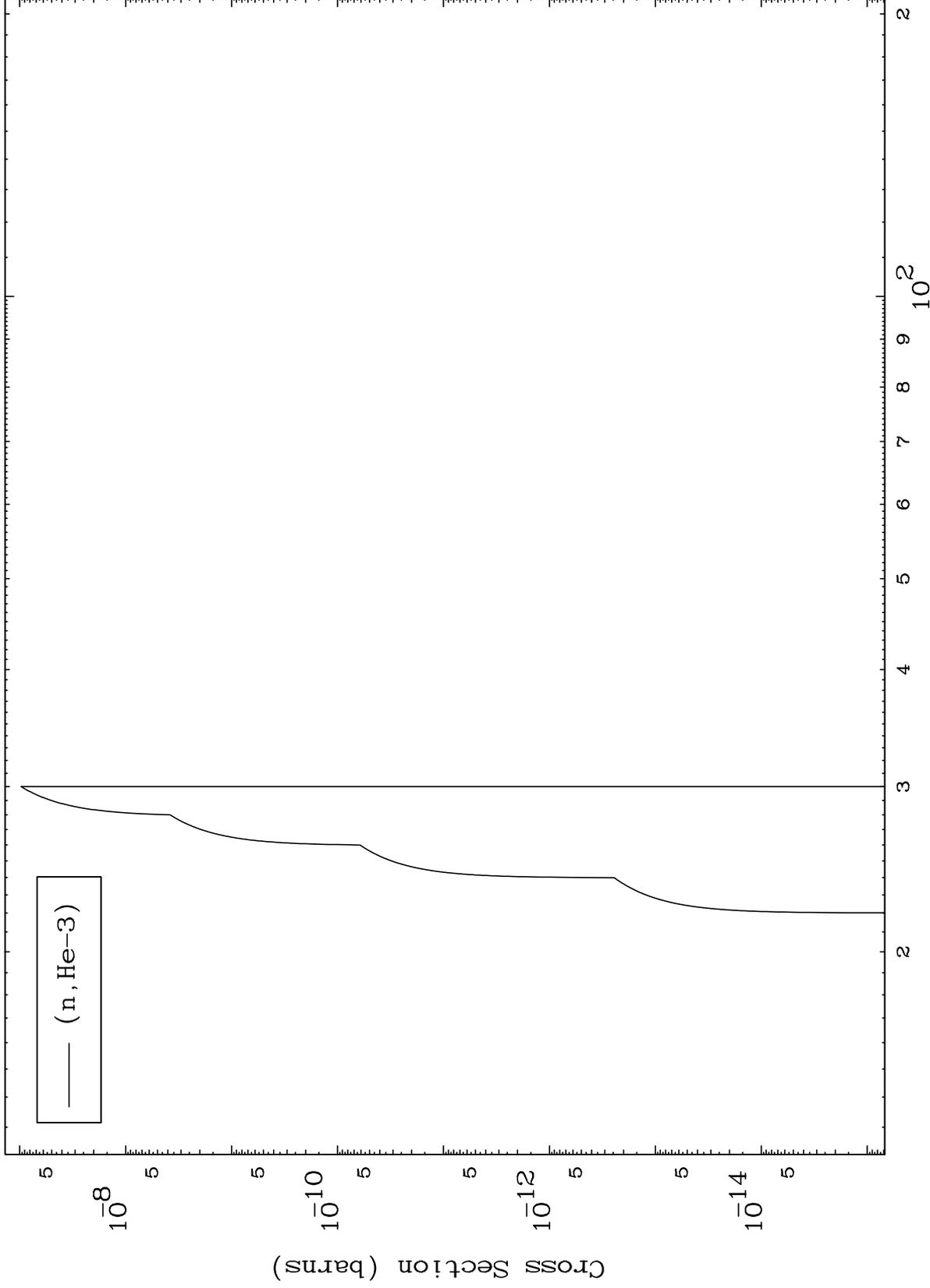
86-Rn-200



7

Incident Energy (MeV)

86-Rn-200

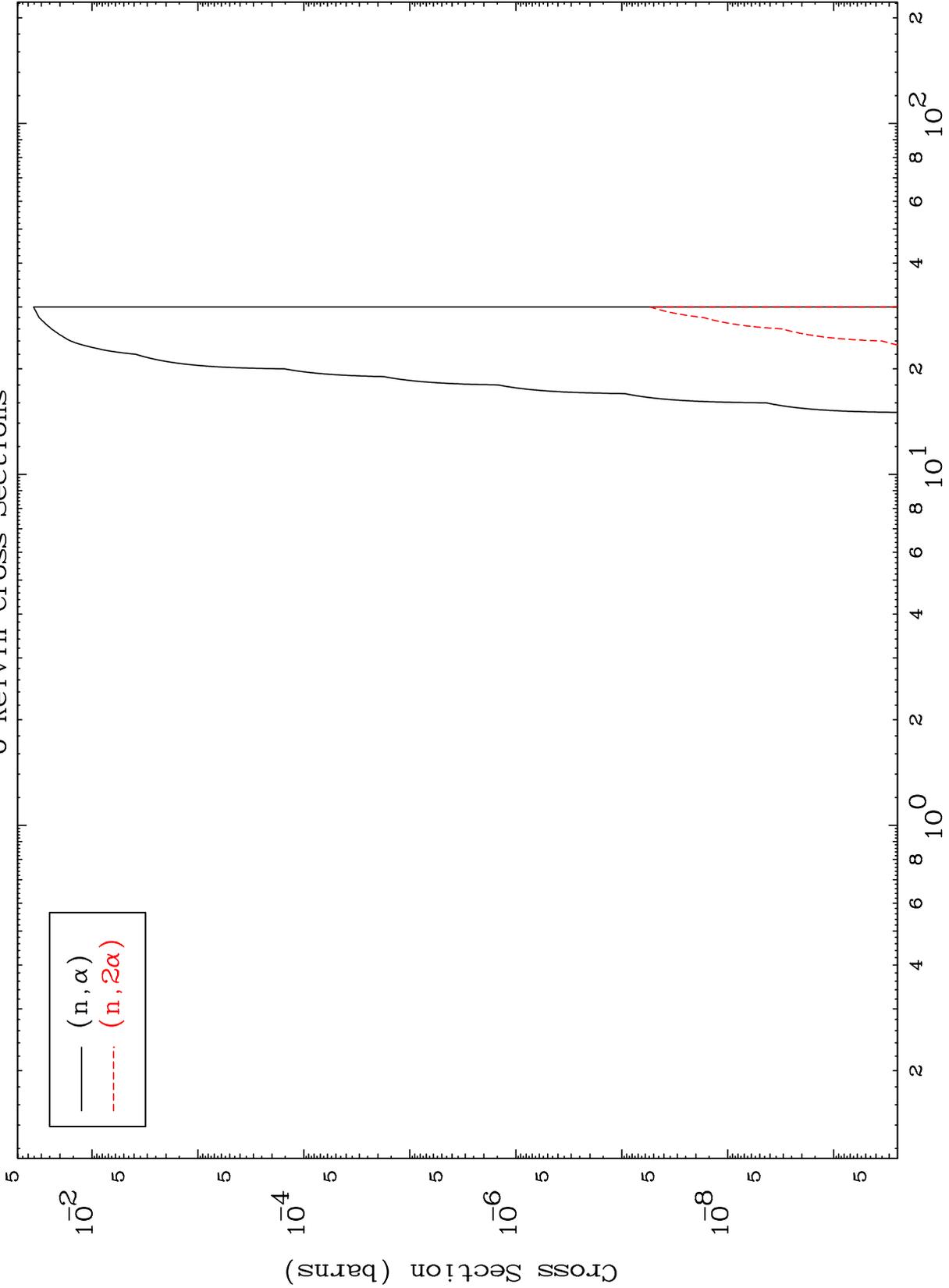


MAT 8592

( $\alpha, \alpha$ ) Levels

86-Rn-200

0 Kelvin Cross Sections



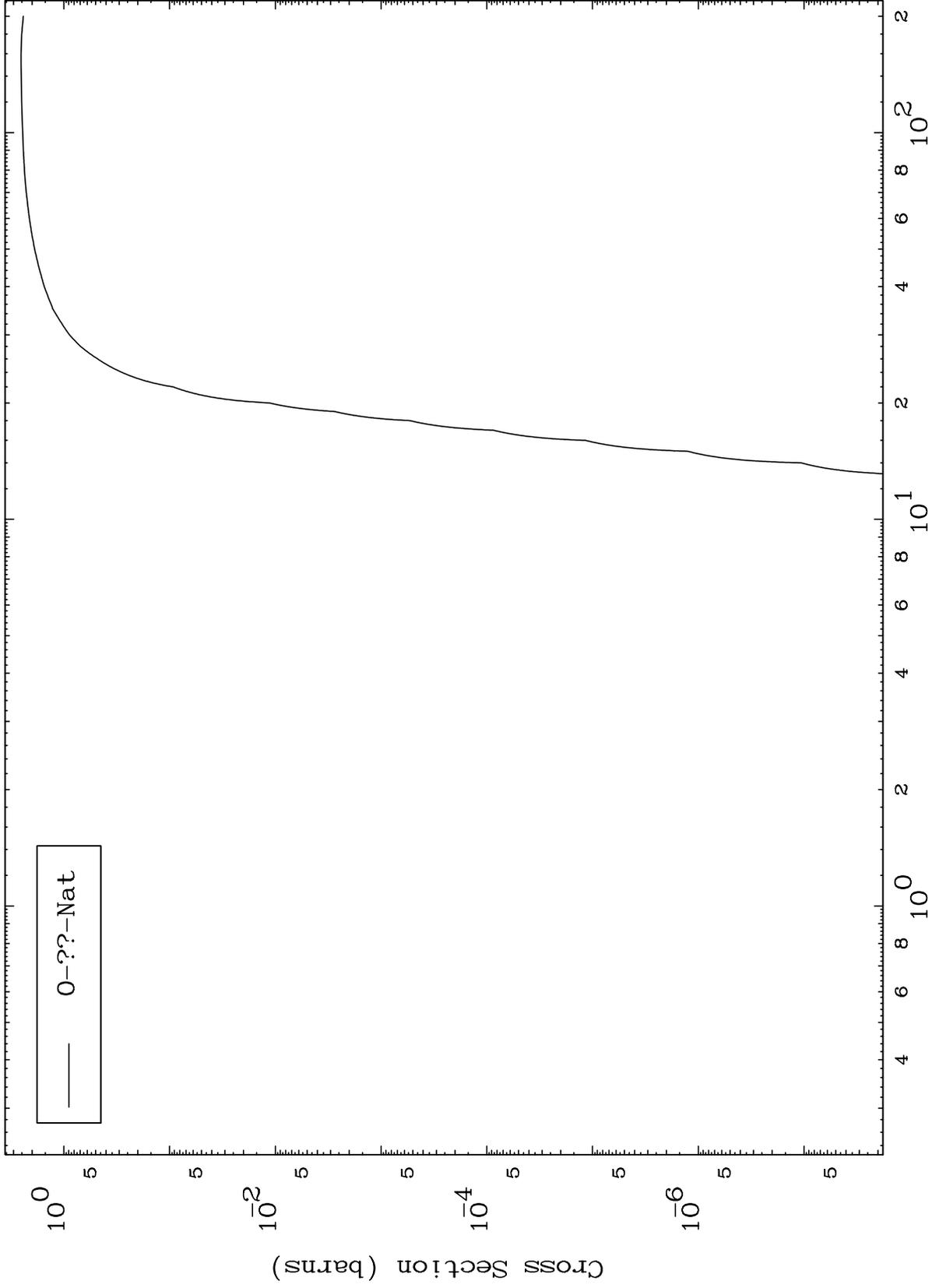
— (n,  $\alpha$ )  
- - - (n,  $2\alpha$ )

MAT 8592

Fission

86-Rn-200

Radionuclide Production Cross Section



10

Incident Energy (MeV)

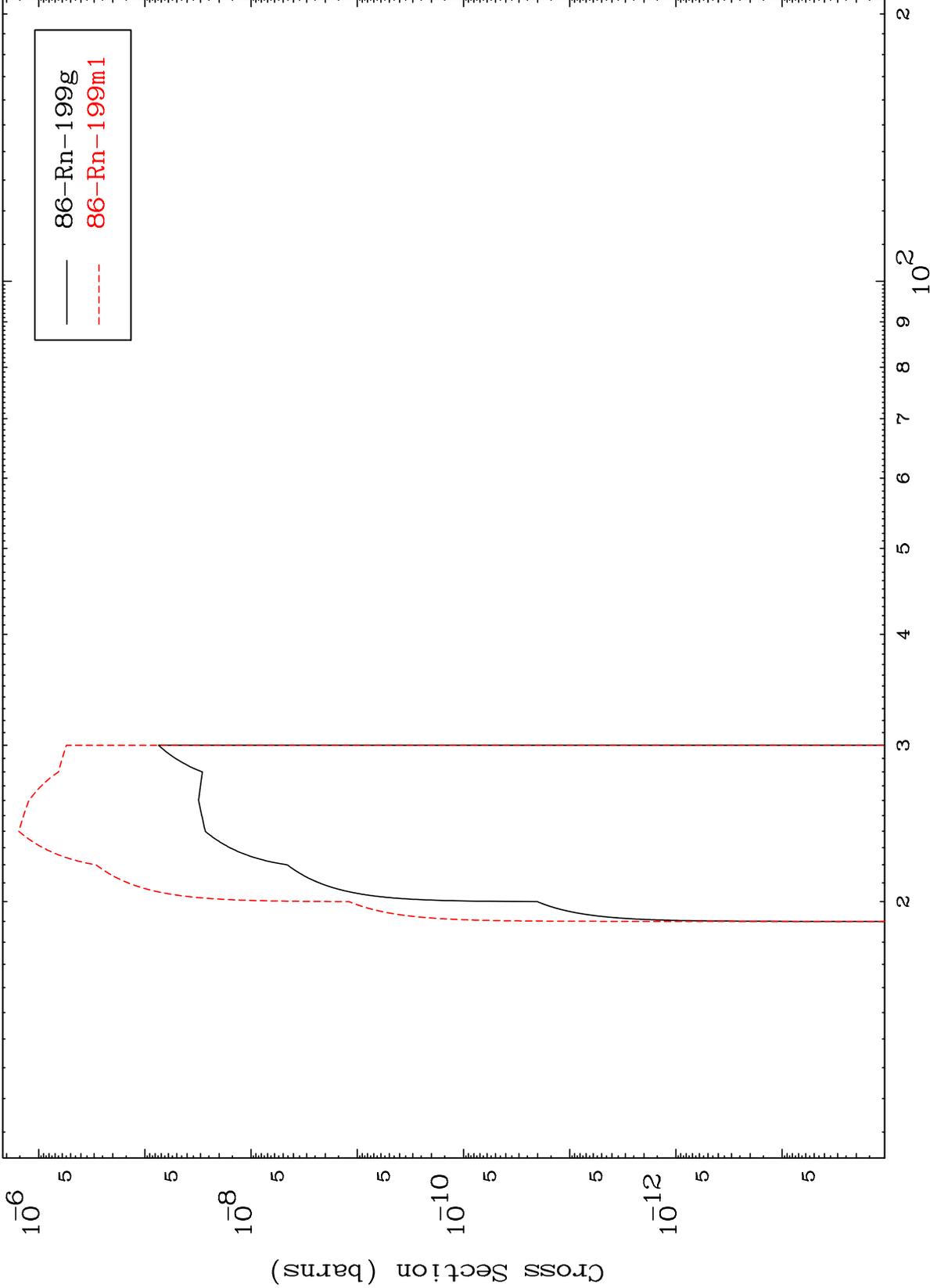
86-Rn-200

MAT 8592

$(n, n') \alpha$

86-Rn-200

Radionuclide Production Cross Section

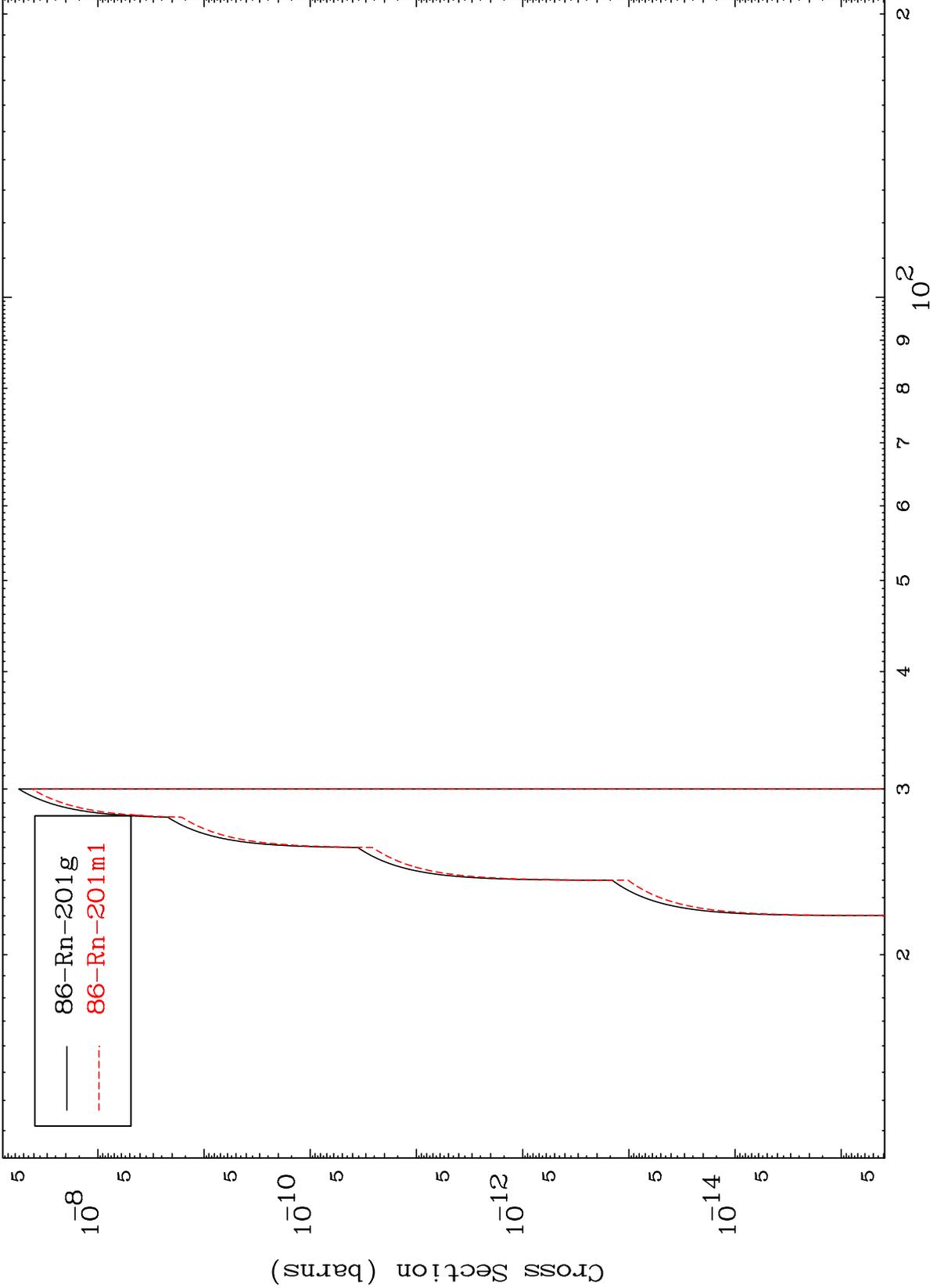


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(n,He-3)

86-Rn-200

Radionuclide Production Cross Section



12

Incident Energy (MeV)

86-Rn-200