

Program EVALPLOT  
(Version 2021-1)

by

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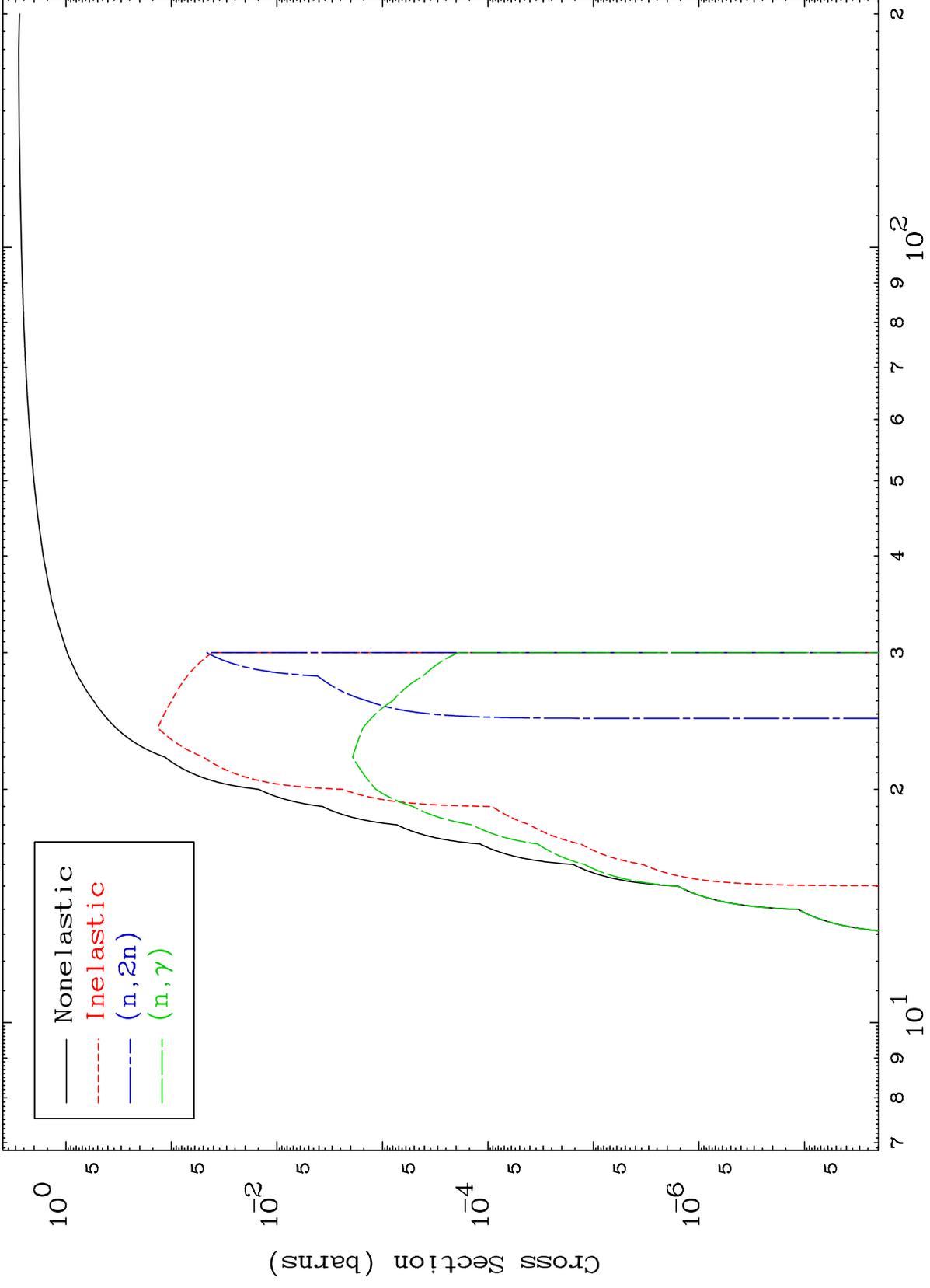
Press Mouse Button to Start

MAT 8613

$\alpha$  Major

86-Rn-207

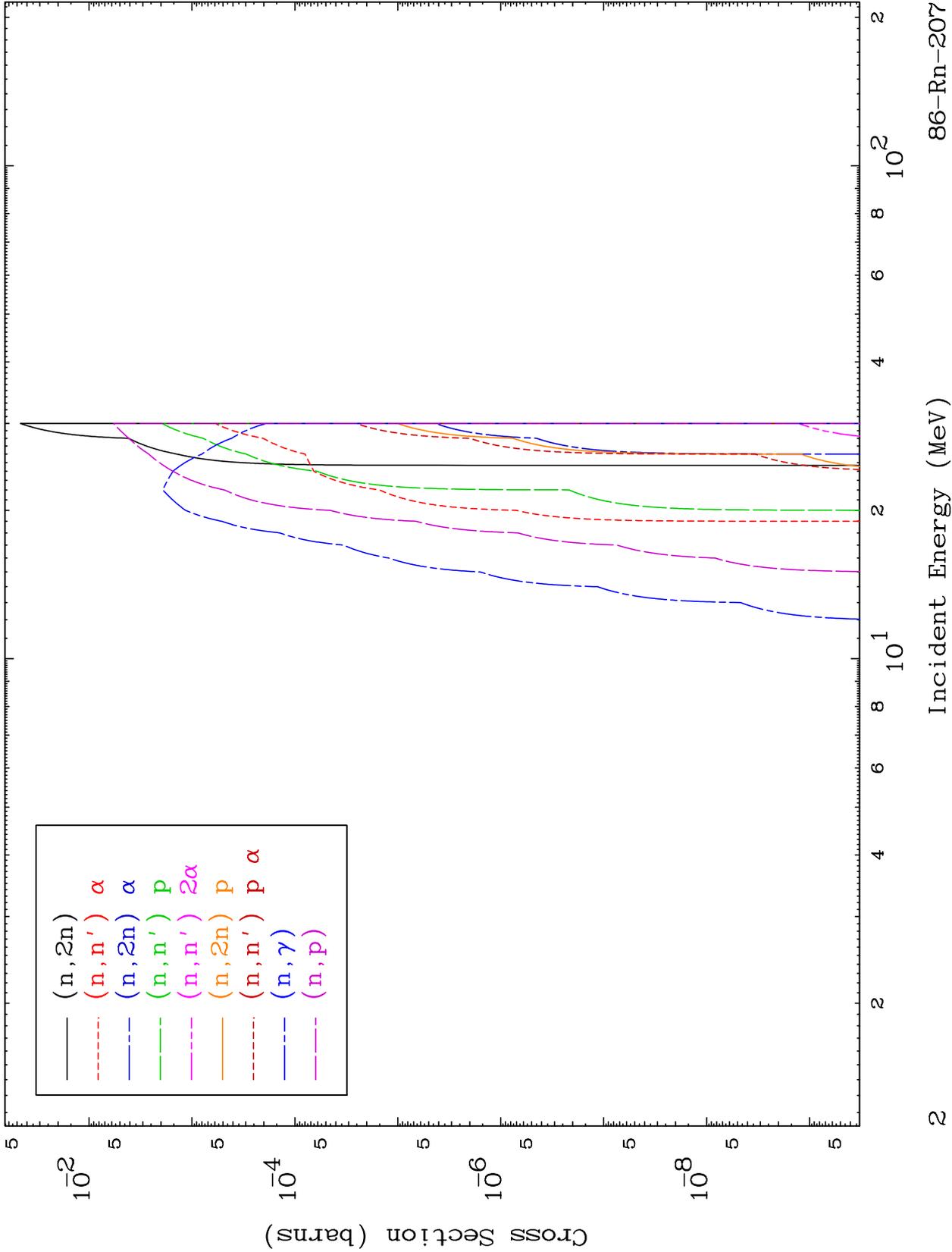
0 Kelvin Cross Sections



1

Incident Energy (MeV)

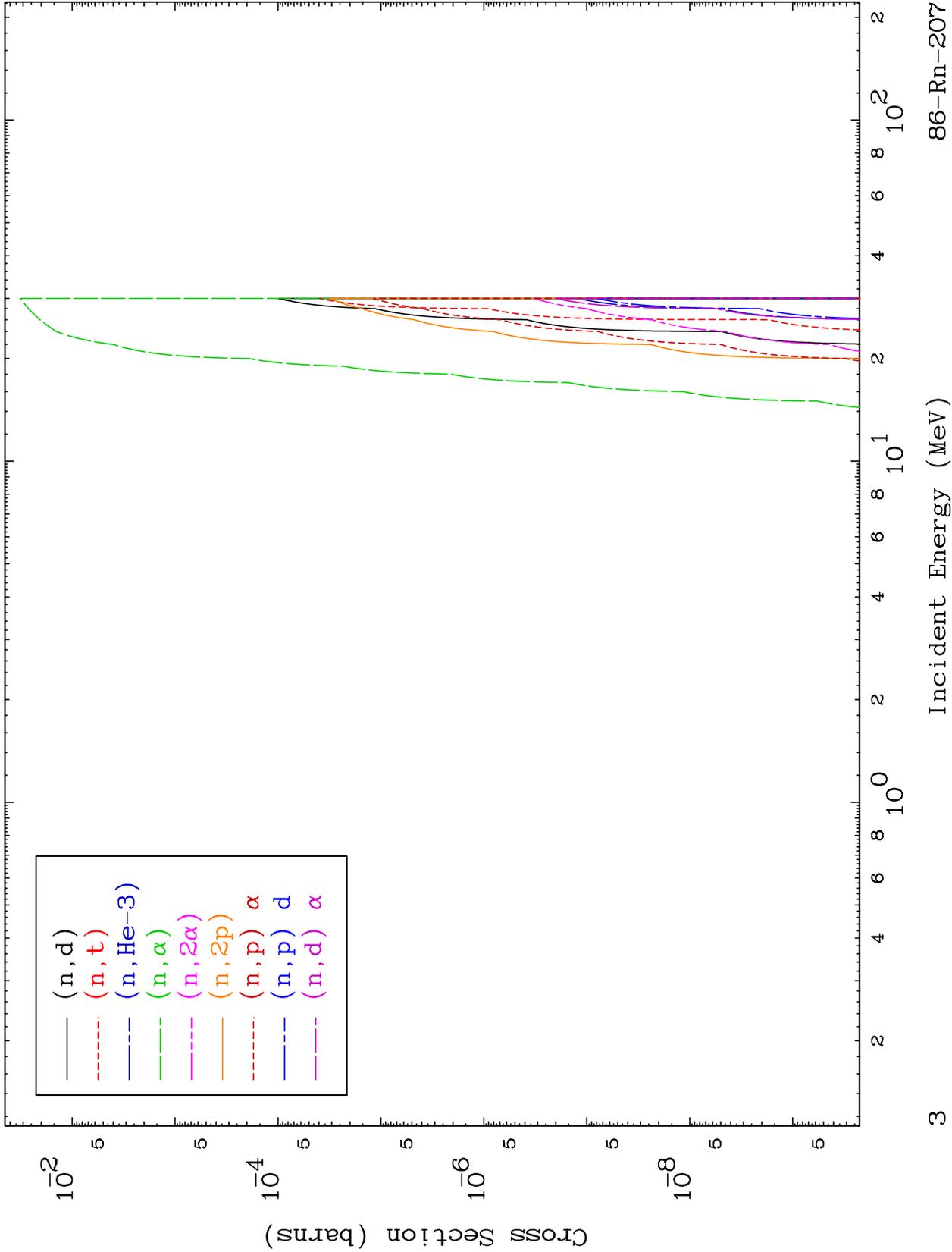
86-Rn-207

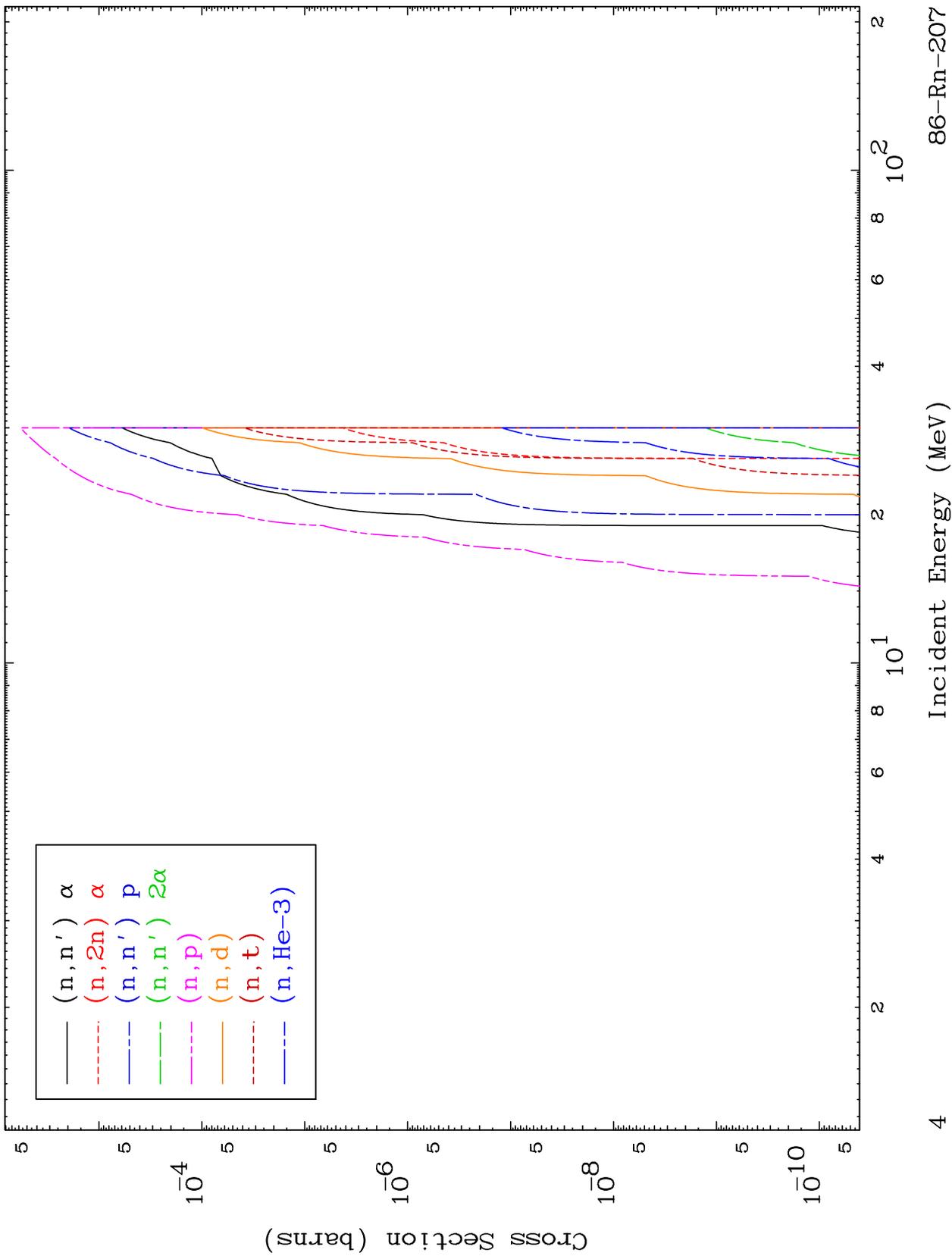


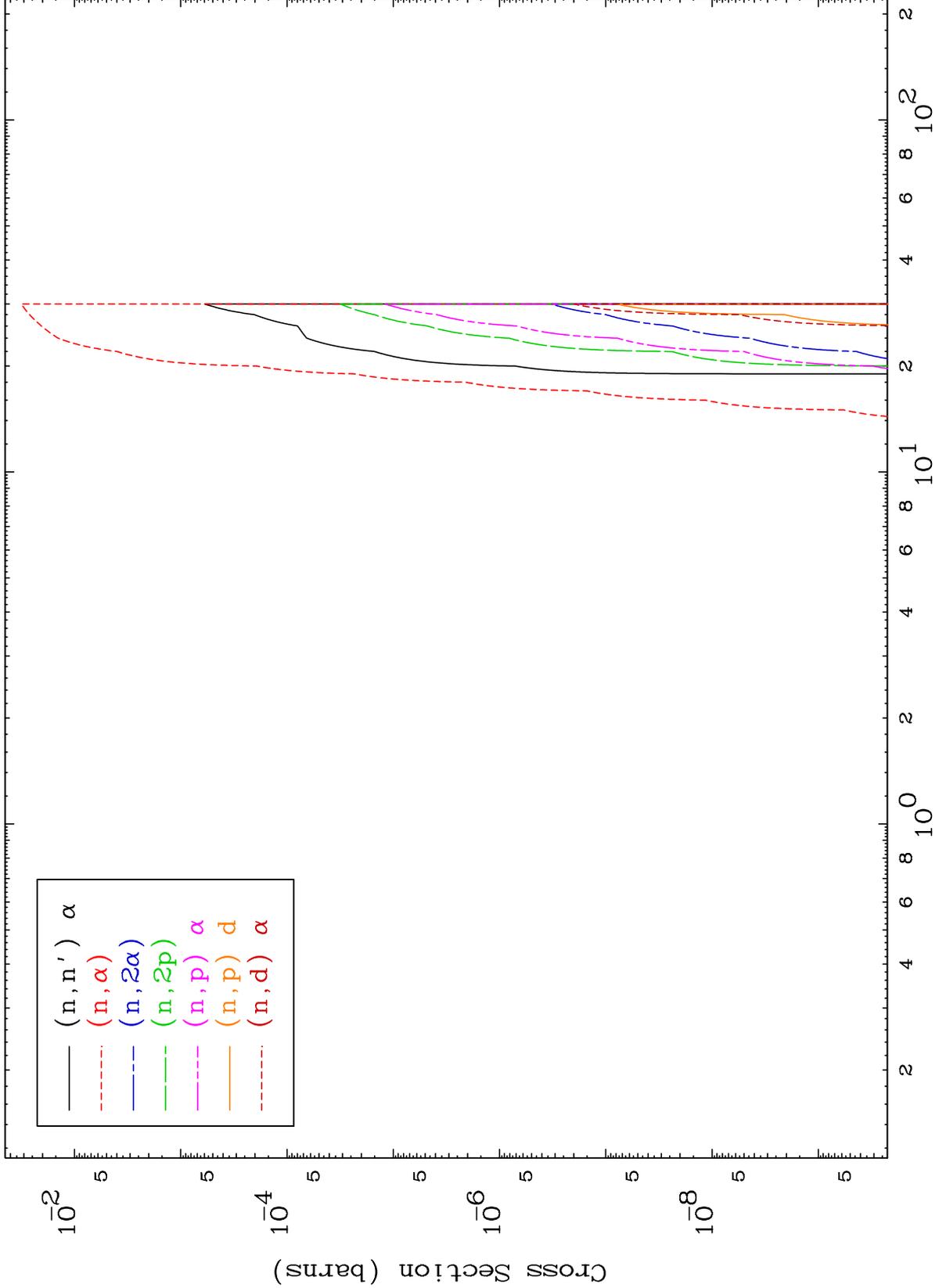
MAT 8613

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

86-Rn-207



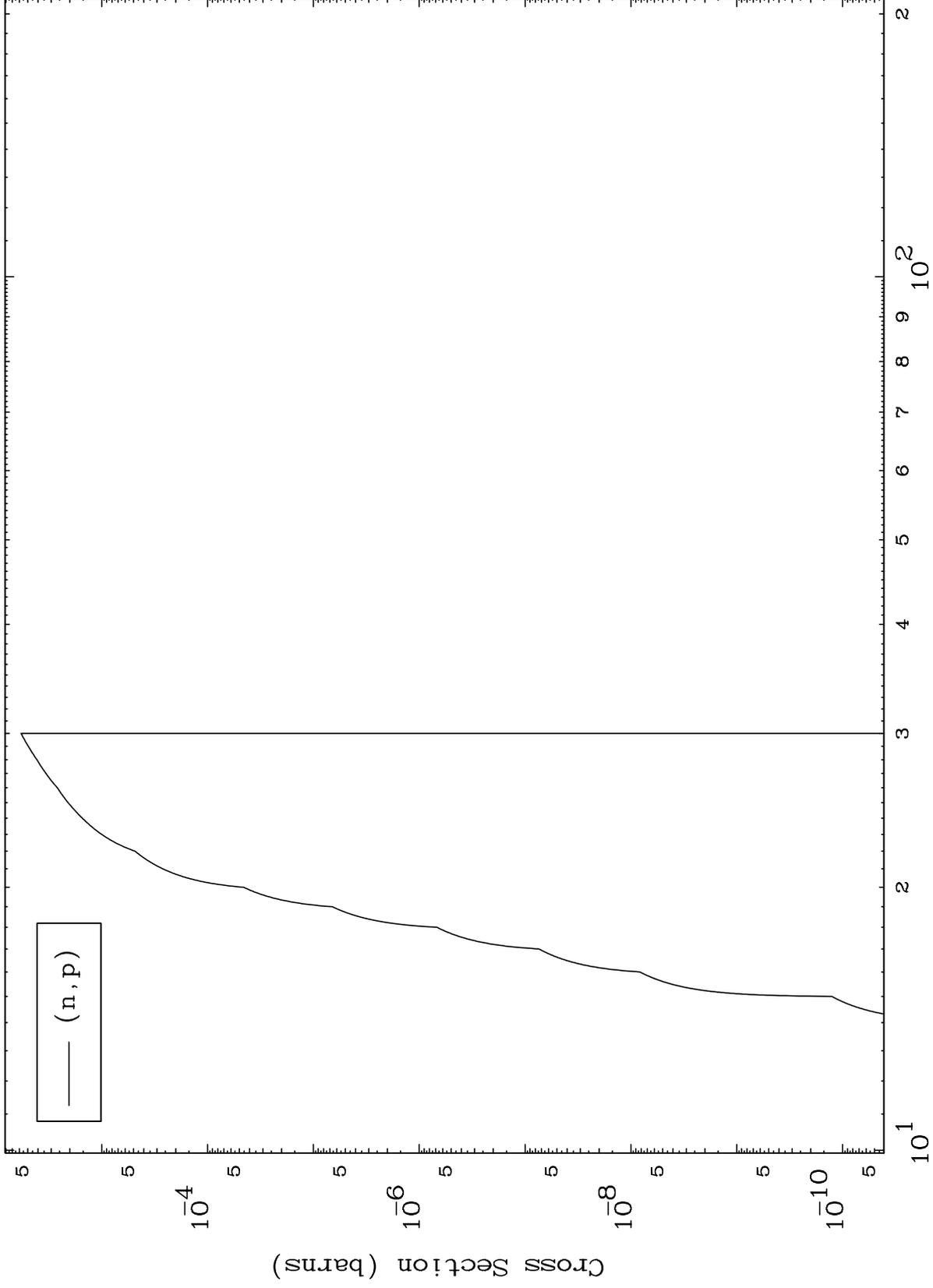




MAT 8613

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

86-Rn-207



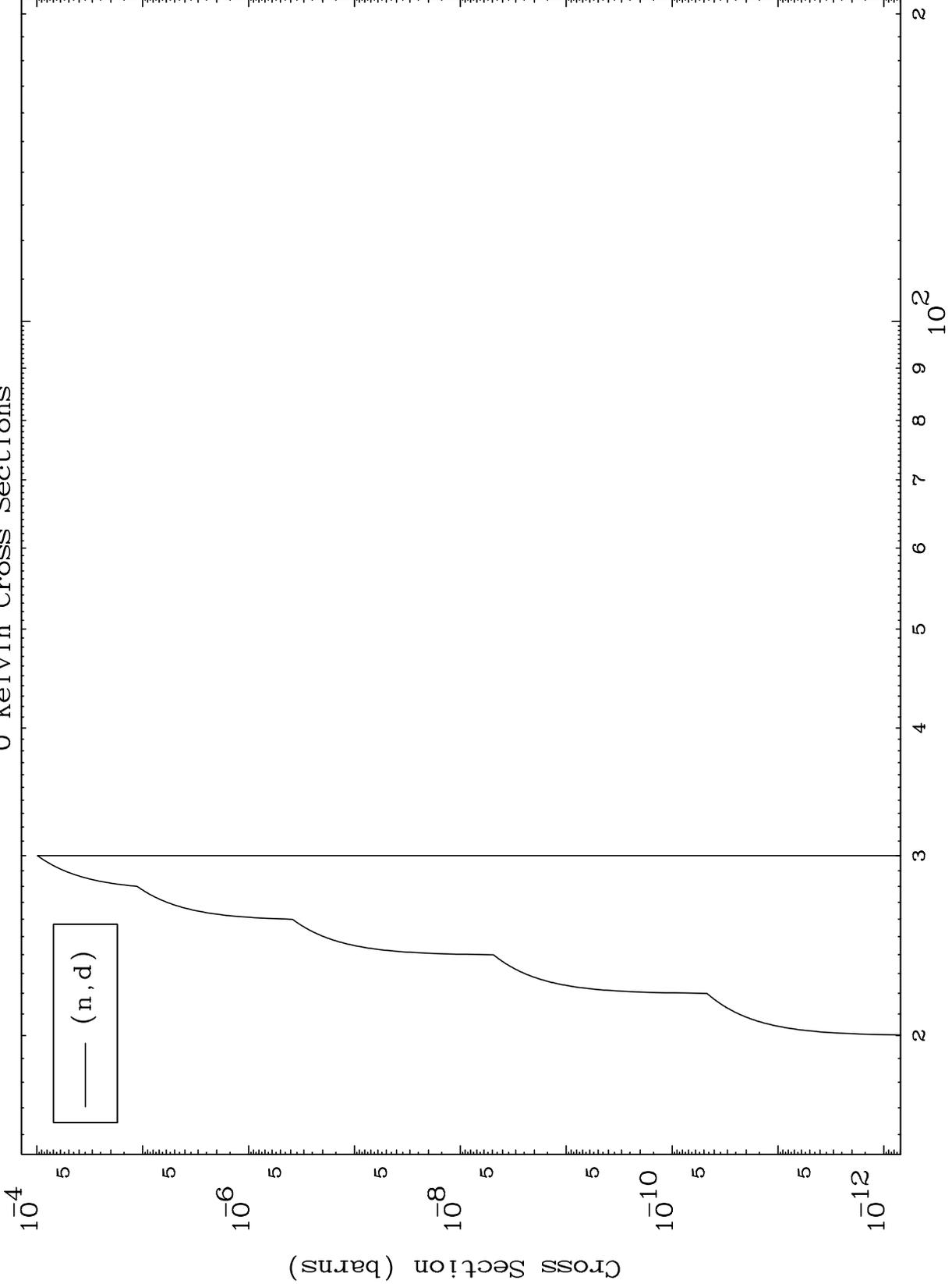
Incident Energy (MeV)

86-Rn-207

MAT 8613

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

86-Rn-207



7

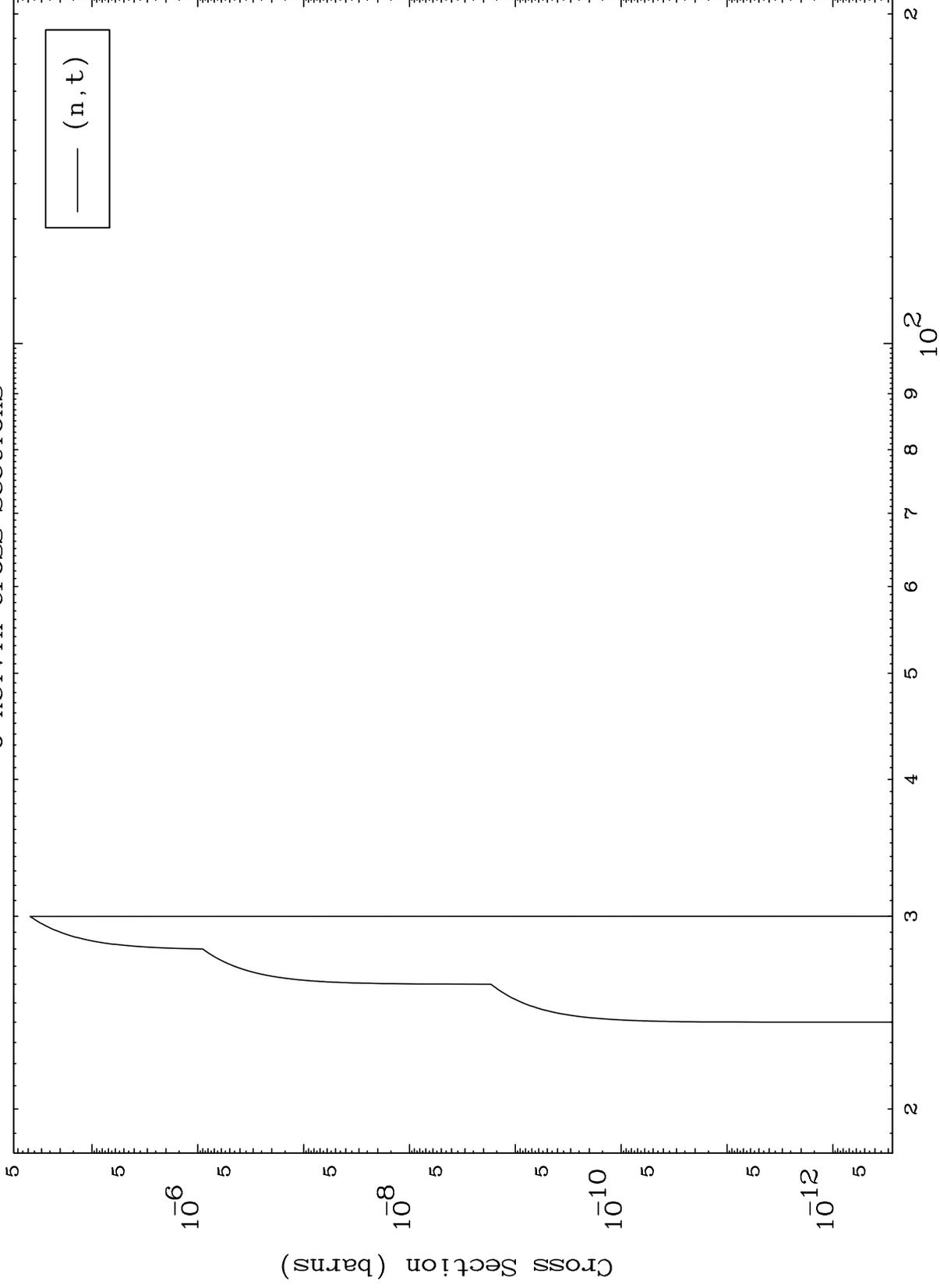
Incident Energy (MeV)

86-Rn-207

MAT 8613

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

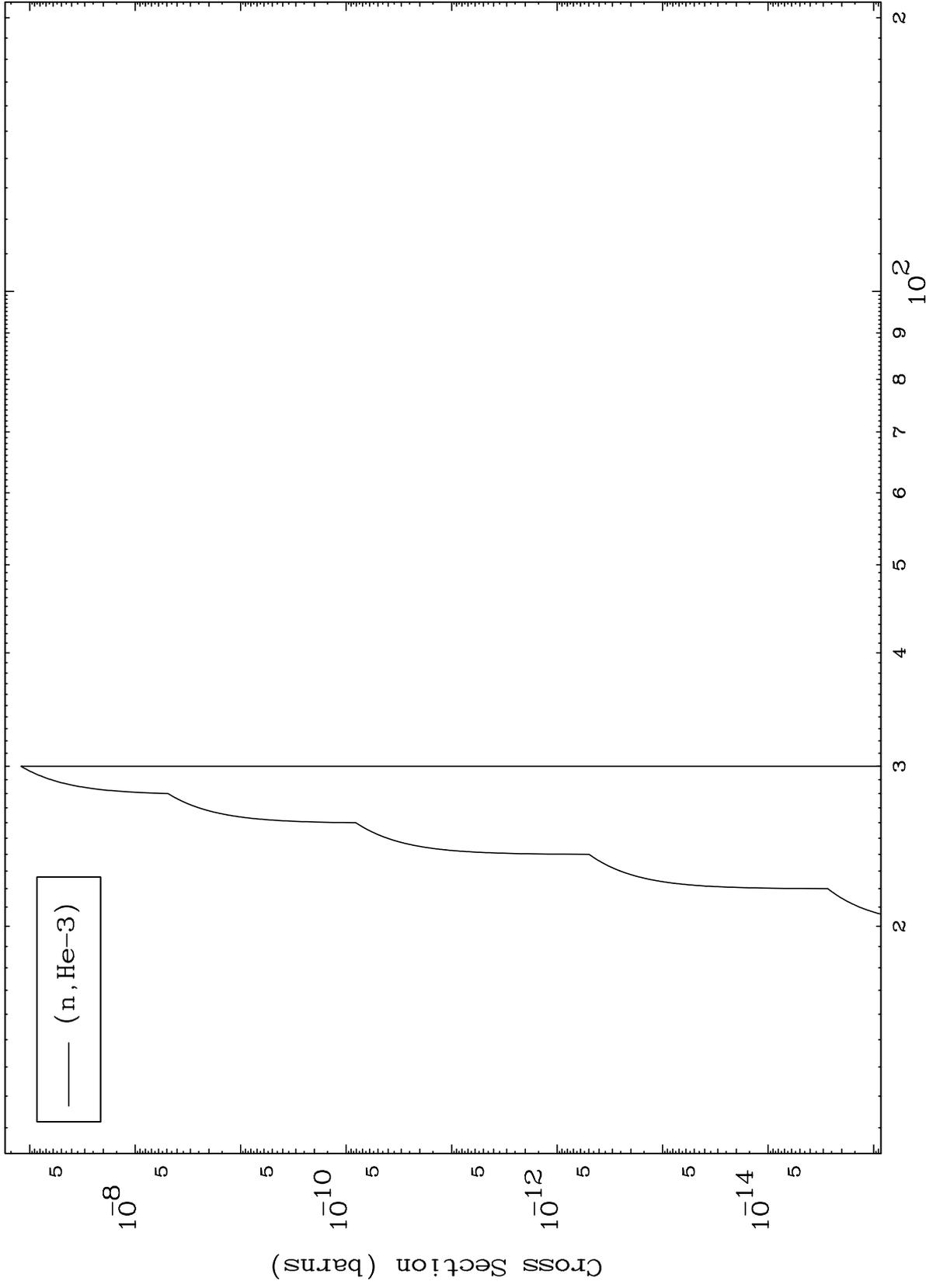
86-Rn-207



8

Incident Energy (MeV)

86-Rn-207

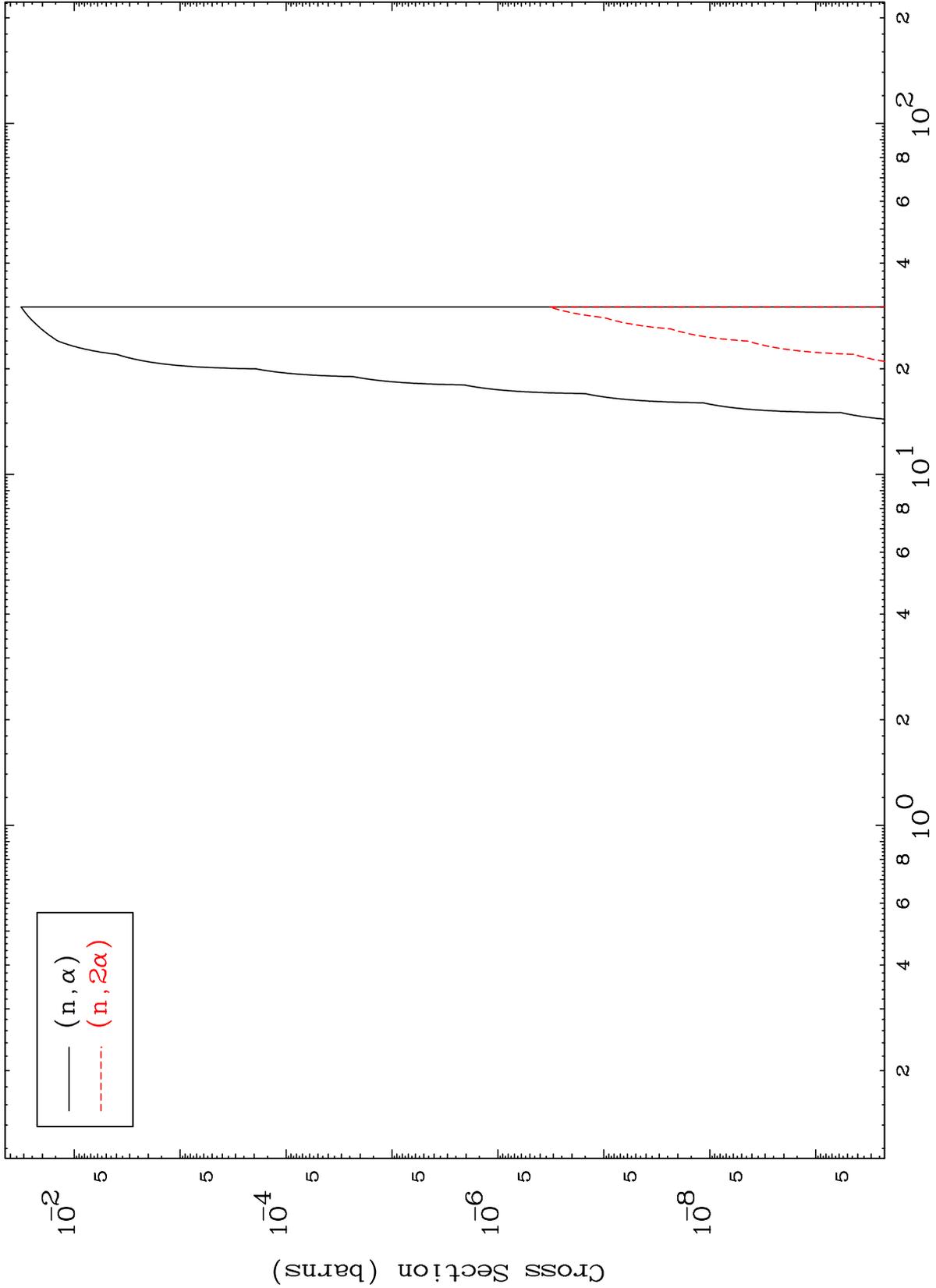


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( $\alpha, \alpha$ ) Levels

86-Rn-207

0 Kelvin Cross Sections



— ( $n, \alpha$ )  
- - - ( $n, 2\alpha$ )

10

Incident Energy (MeV)

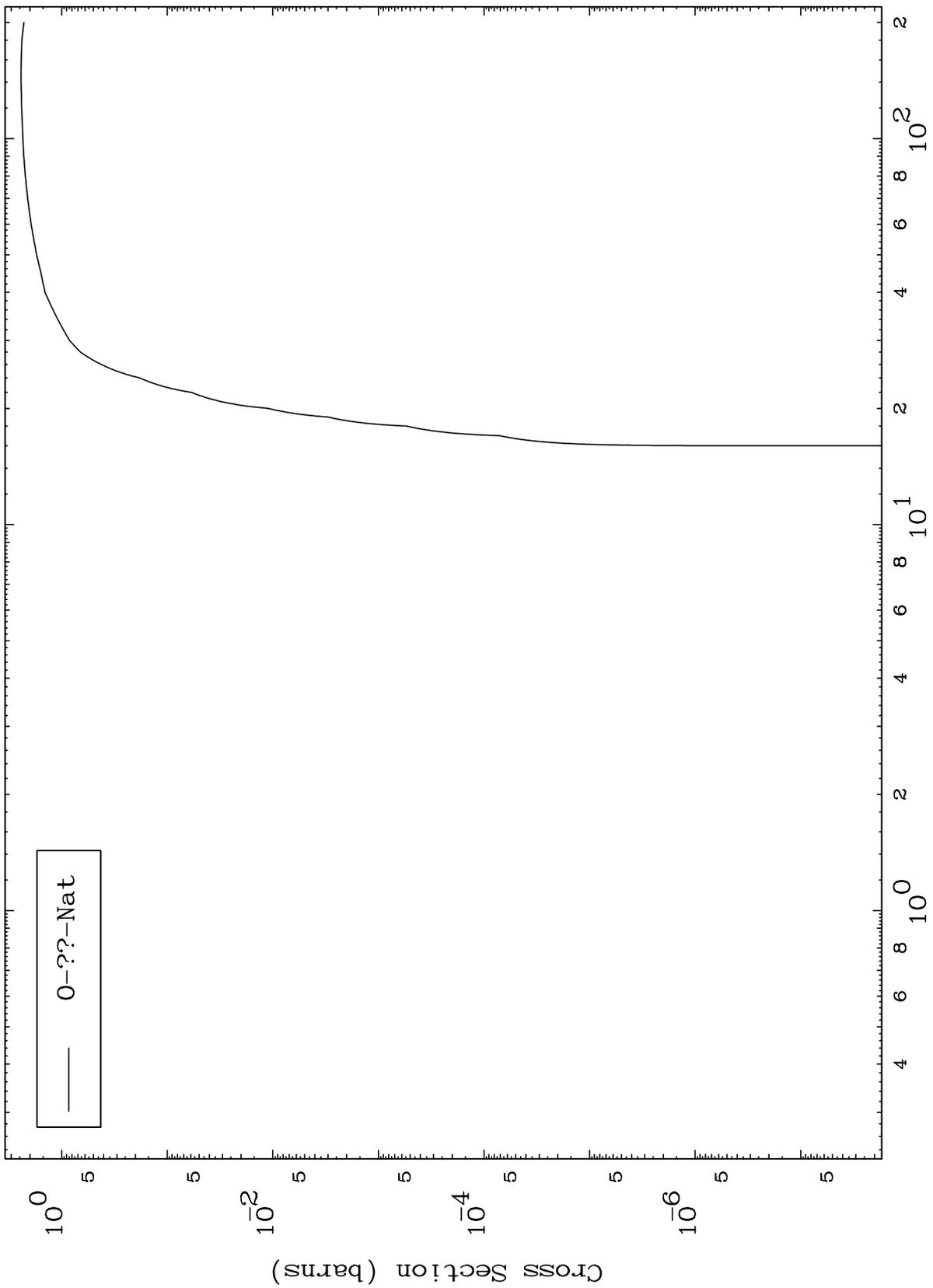
86-Rn-207

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Fission

86-Rn-207

Radionuclide Production Cross Section

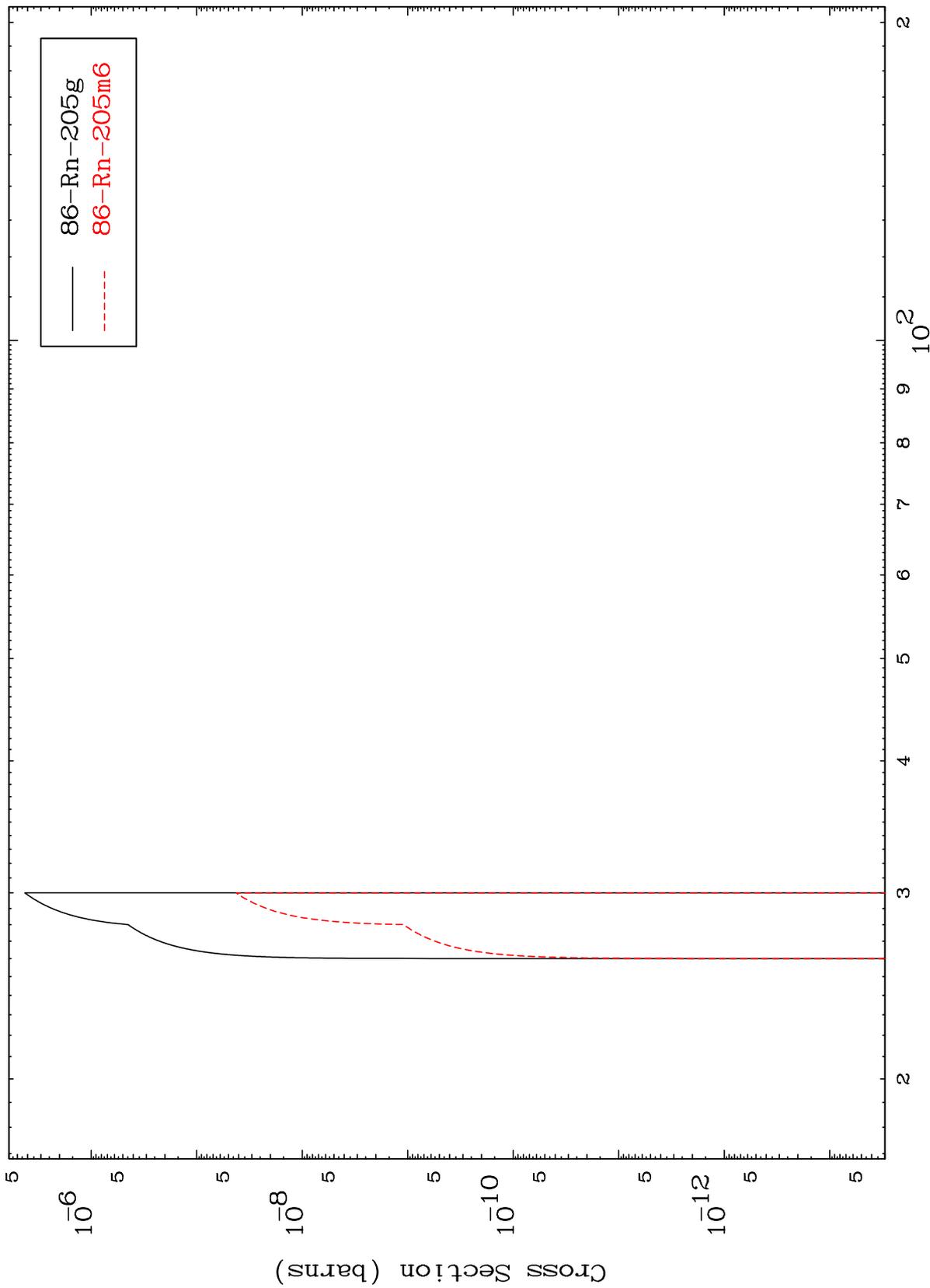


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$(n,2n) \alpha$

86-Rn-207

Radionuclide Production Cross Section



12

Incident Energy (MeV)

86-Rn-207

MAT 8613

(n,2α)

86-Rn-207

Radionuclide Production Cross Section

