Program EVALPLOT
(Version 2021-1)
by
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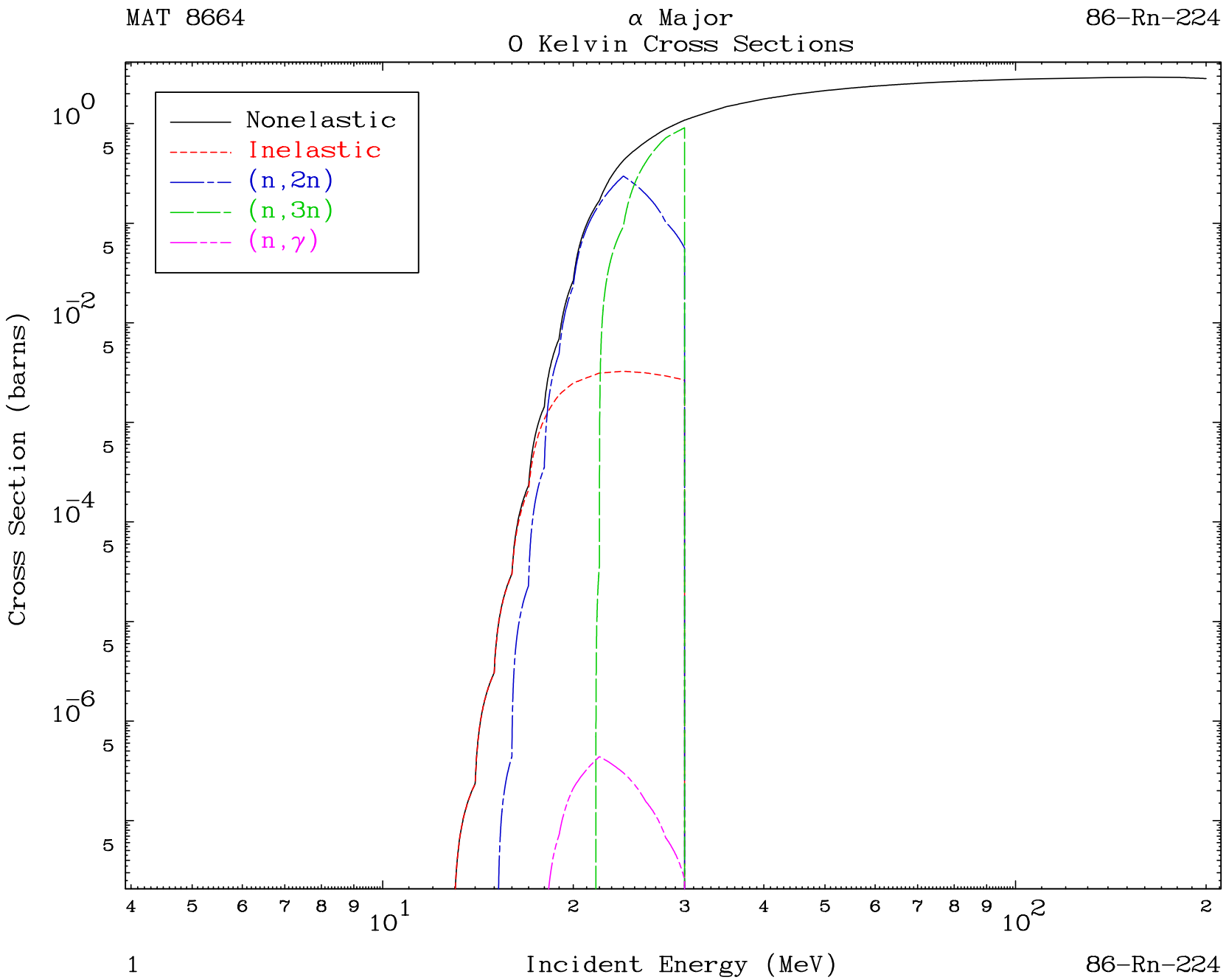
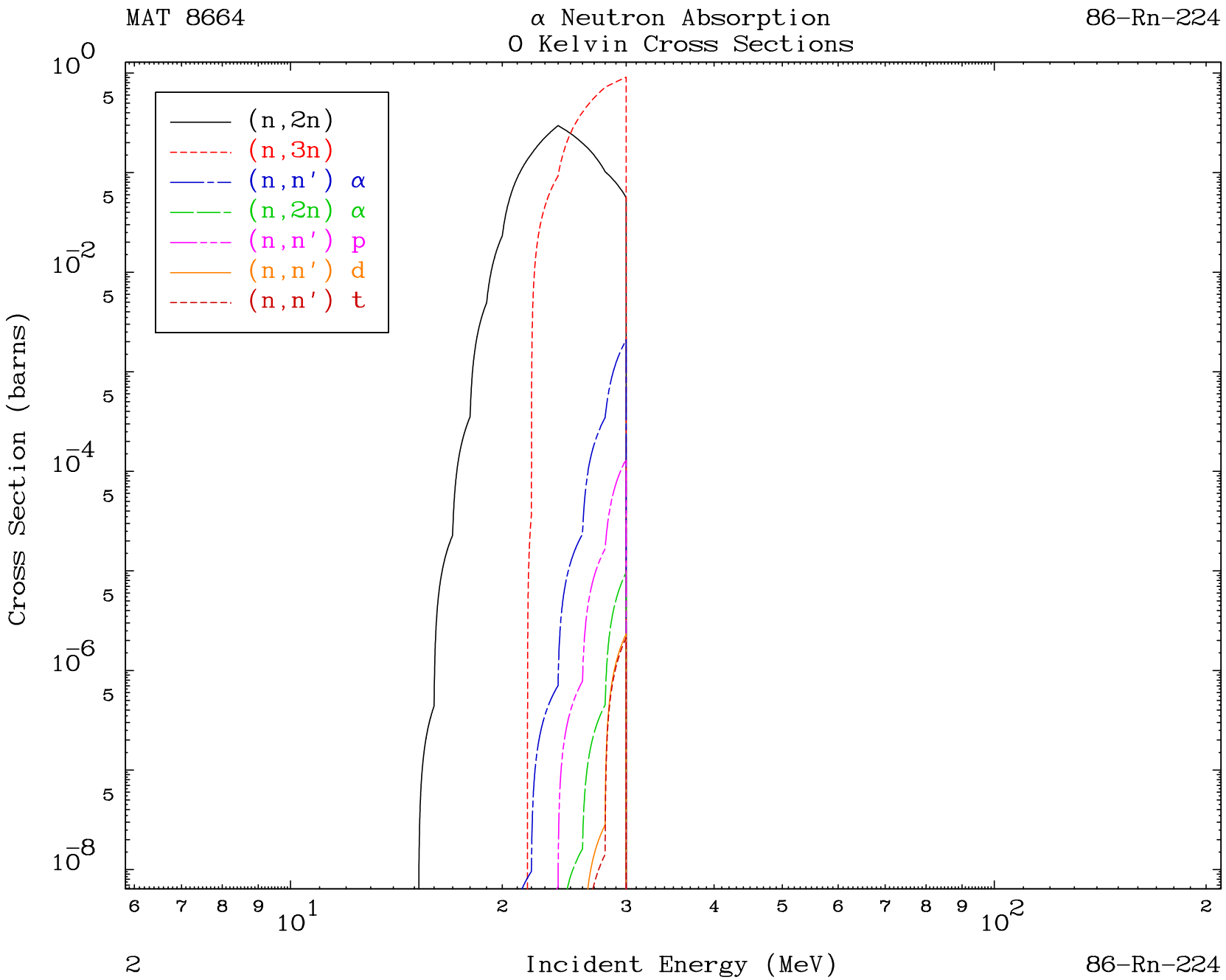
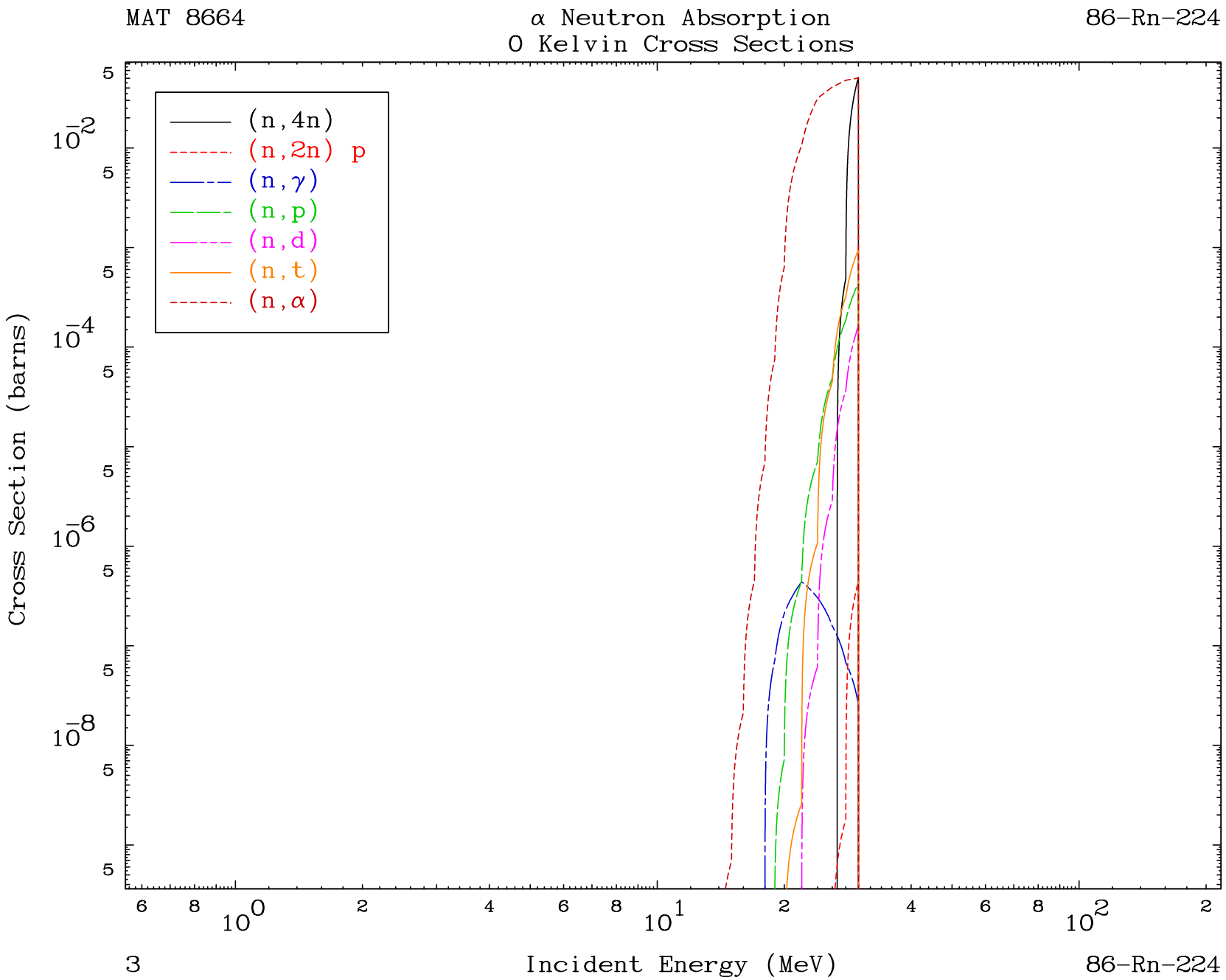
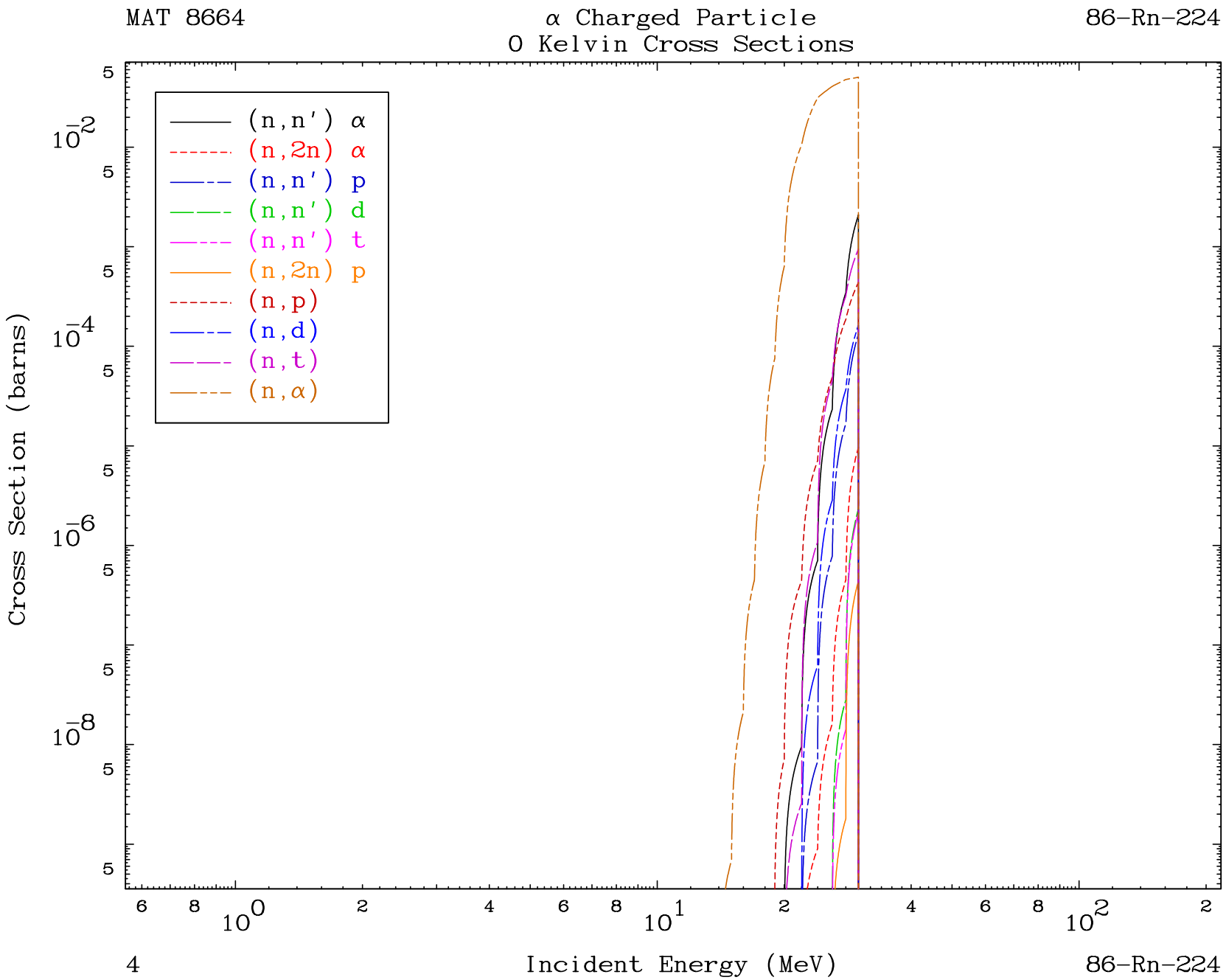
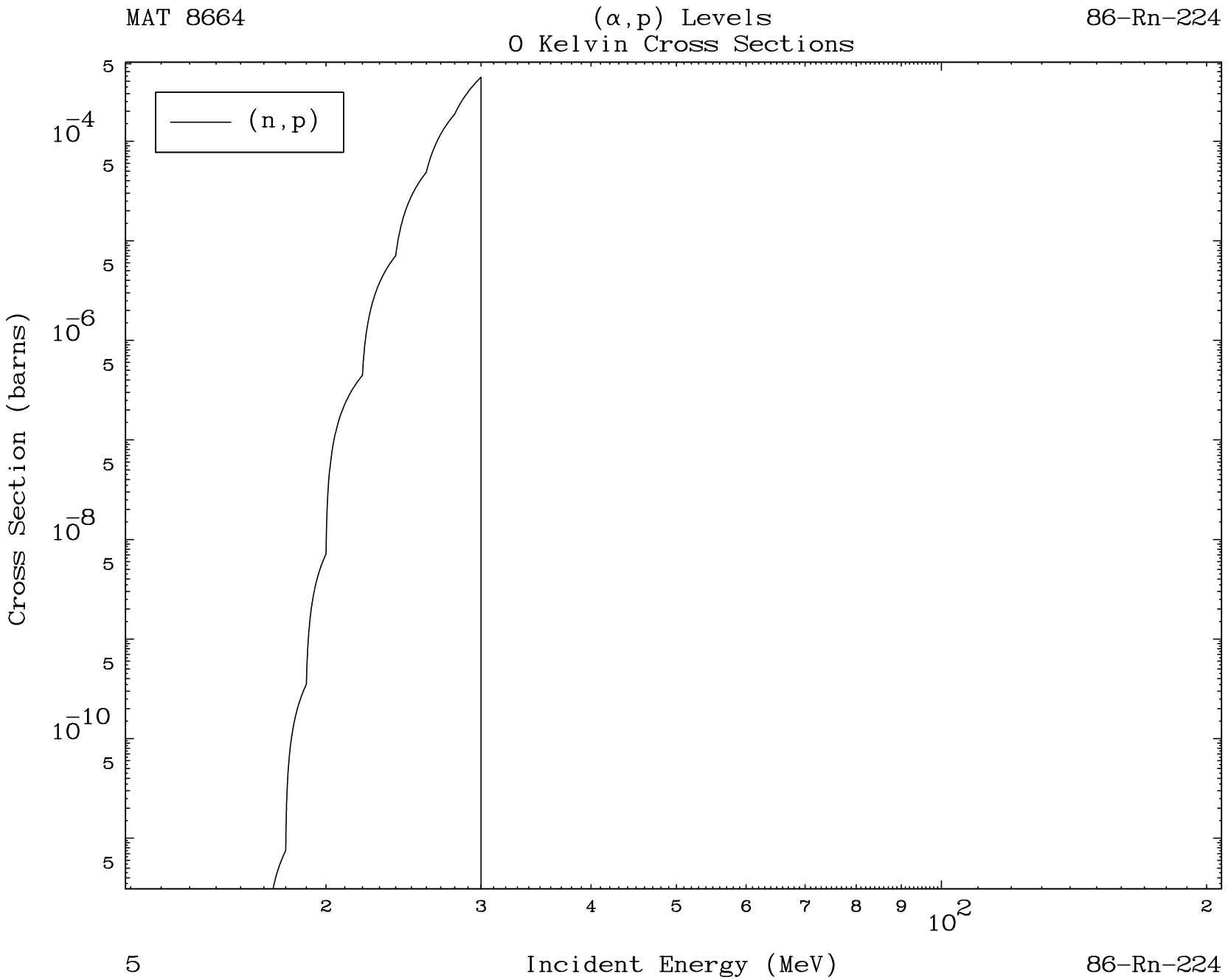
Press Mouse Button to Start

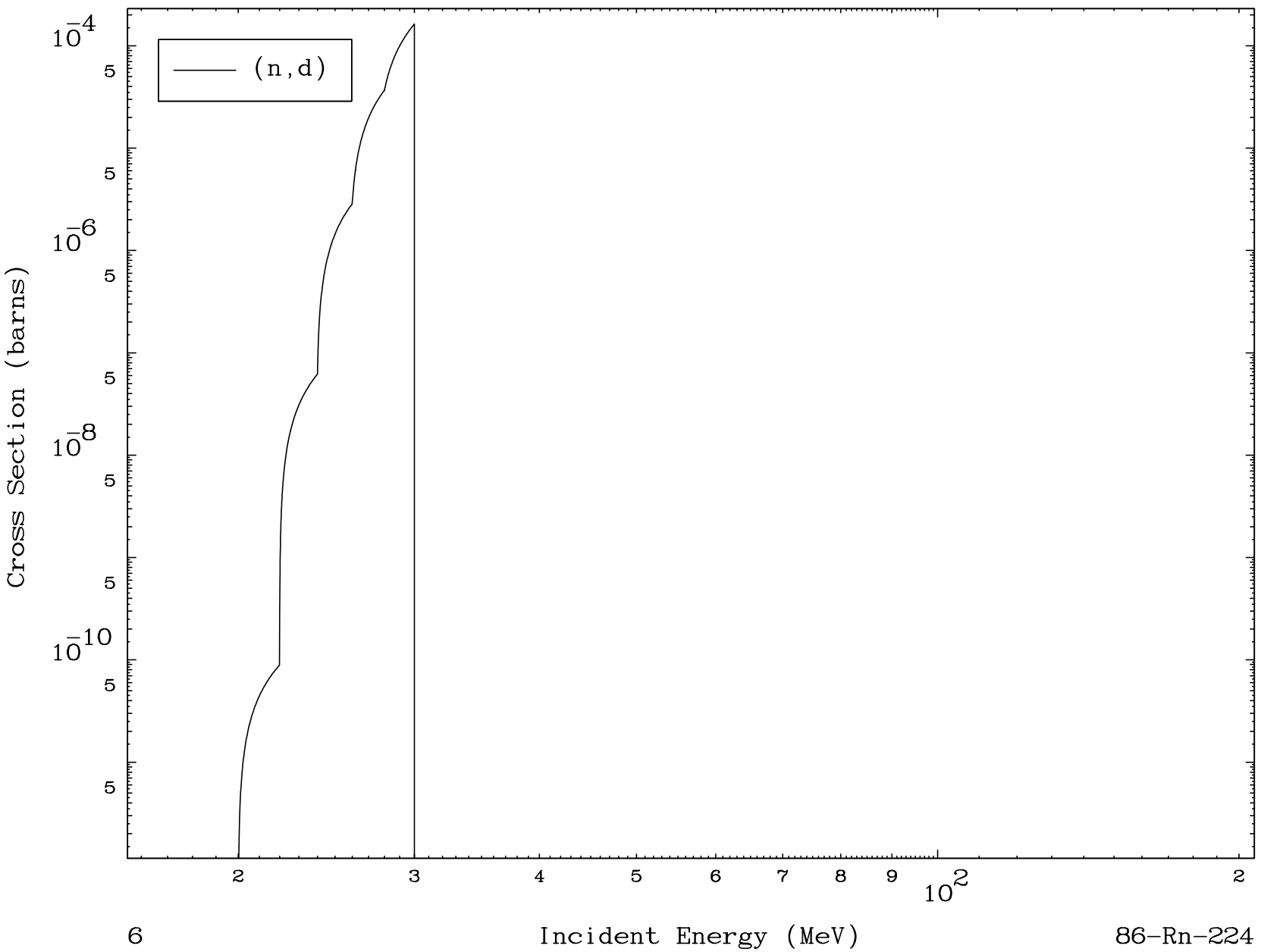

MAT 8664
α Major
86-Rn-224
0 Kelvin Cross Sections
Nonelastic
Inelastic
(n,2n)
(n,3n)
(n,γ)
Cross Section (barns)
Incident Energy (MeV)
86-Rn-224


MAT 8664
α Neutron Absorption
0 Kelvin Cross Sections
86-Rn-224
(n,2n)
(n,3n)
(n,n') α
(n,2n) α
(n,n') p
(n,n') d
(n,n') t
Cross Section (barns)
Incident Energy (MeV)
86-Rn-224

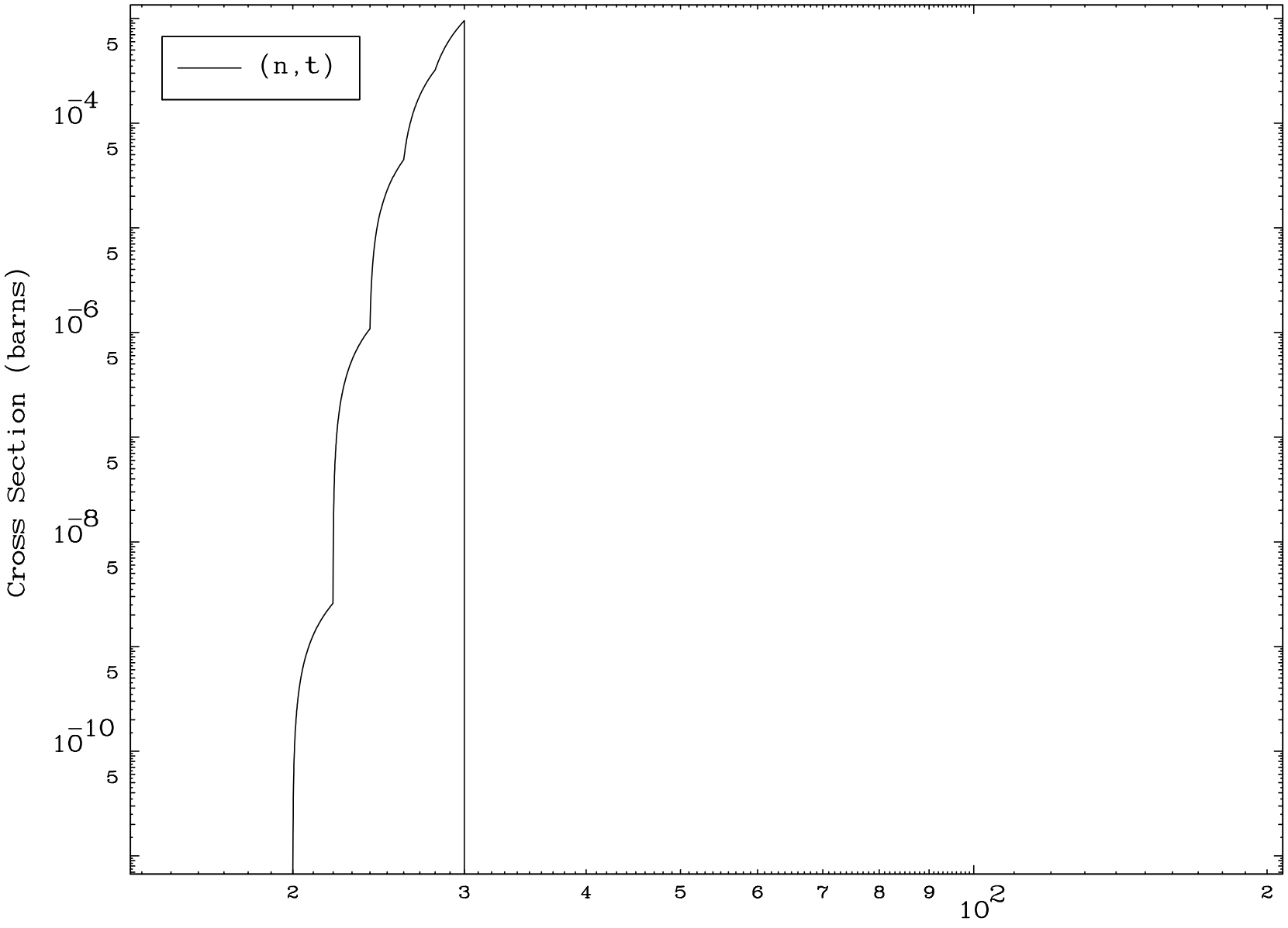

MAT 8664
α Neutron Absorption
0 Kelvin Cross Sections
86-Rn-224
(n,4n)
(n,2n) p
(n,γ)
(n,p)
(n,d)
(n,t)
(n,α)
Cross Section (barns)
Incident Energy (MeV)
3
86-Rn-224

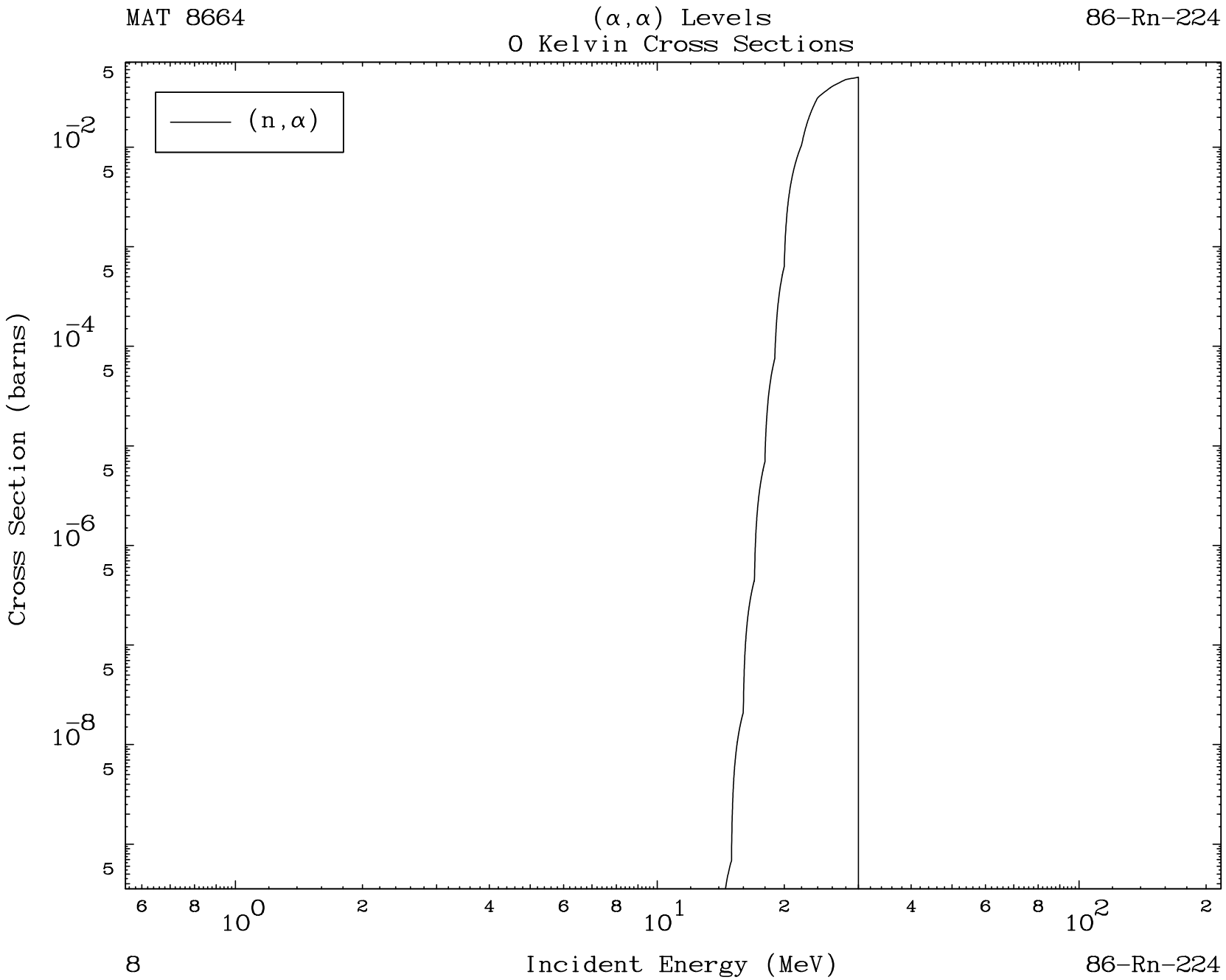

MAT 8664
α Charged Particle
0 Kelvin Cross Sections
86-Rn-224
(n,n') α
(n,2n) α
(n,n') p
(n,n') d
(n,n') t
(n,2n) p
(n,p)
(n,d)
(n,t)
(n,α)
Cross Section (barns)
Incident Energy (MeV)
86-Rn-224
4

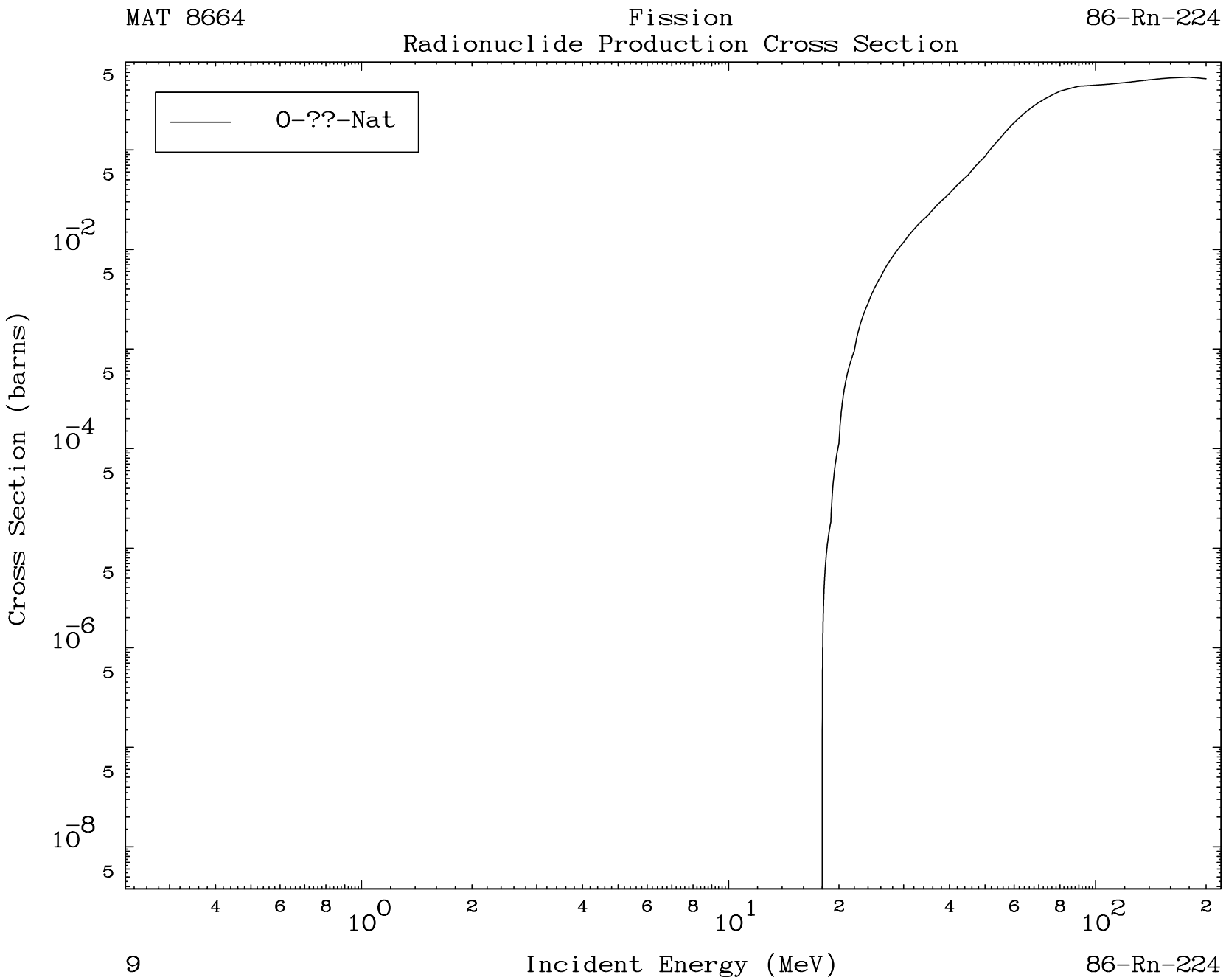

MAT 8664
(α,p) Levels
86-Rn-224
0 Kelvin Cross Sections
(n,p)
Cross Section (barns)
10^-4
10^-6
10^-8
10^-10
Incident Energy (MeV)
10^2
86-Rn-224

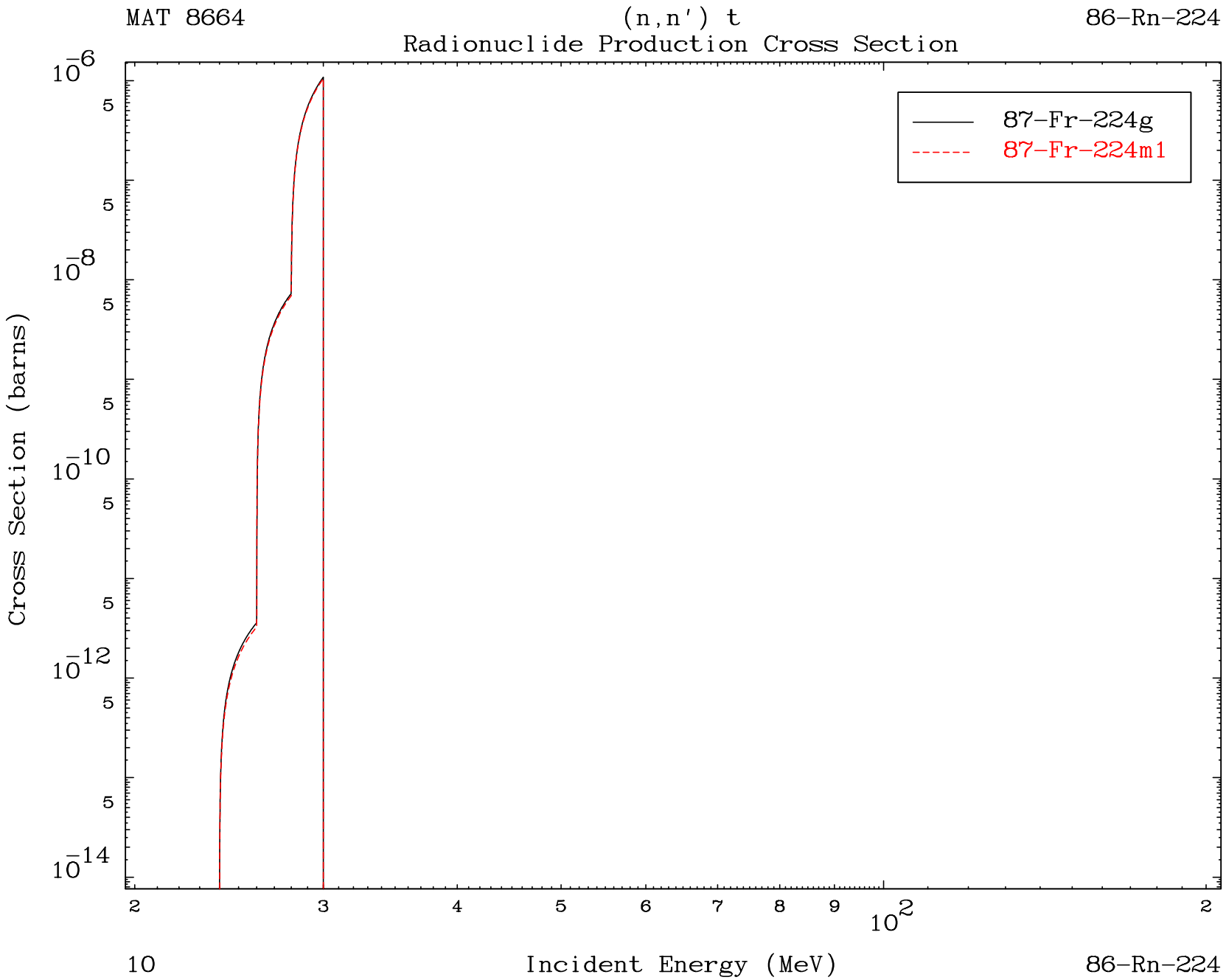

0 Kelvin Cross Sections
(n,d)
Cross Section (barns)
10^-4
5
10^-6
5
10^-8
5
10^-10
5
5
2
3
4
5
6
7
8
9
10^2
2
Incident Energy (MeV)

0 Kelvin Cross Sections




MAT 8664
(α,α) Levels
86-Rn-224
0 Kelvin Cross Sections
(n,α)
Cross Section (barns)
Incident Energy (MeV)
86-Rn-224
8


MAT 8664
Fission
86-Rn-224
Radionuclide Production Cross Section
0-??-Nat
Cross Section (barns)
Incident Energy (MeV)
9
86-Rn-224


MAT 8664
(n,n') t
86-Rn-224
Radionuclide Production Cross Section
87-Fr-224g
87-Fr-224m1
Cross Section (barns)
Incident Energy (MeV)
86-Rn-224