

Program EVALPLOT  
(Version 2021-1)

by

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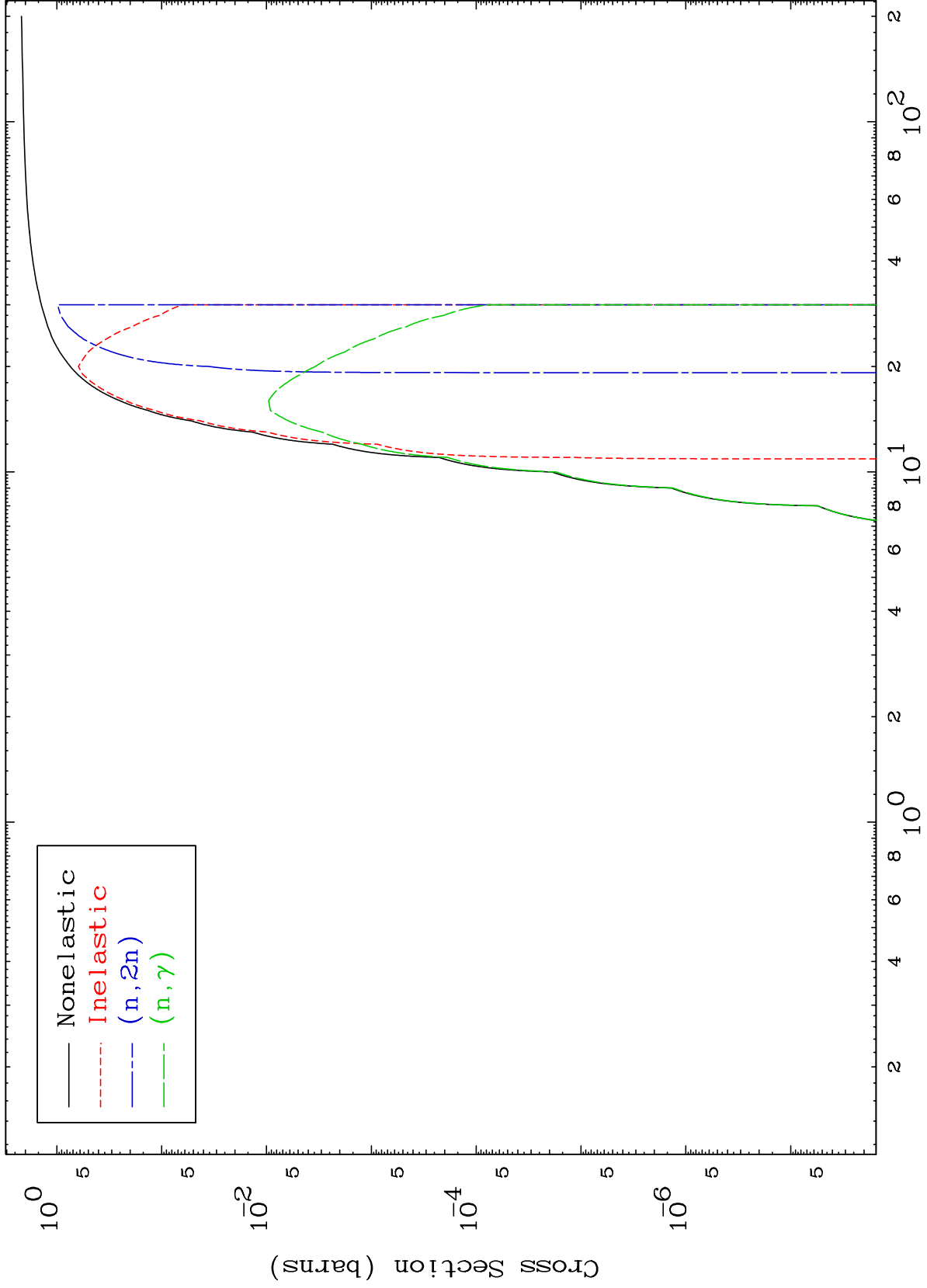
Press Mouse Button to Start

MAT 5113

0 Kelvin

$\alpha$  Major Cross Sections

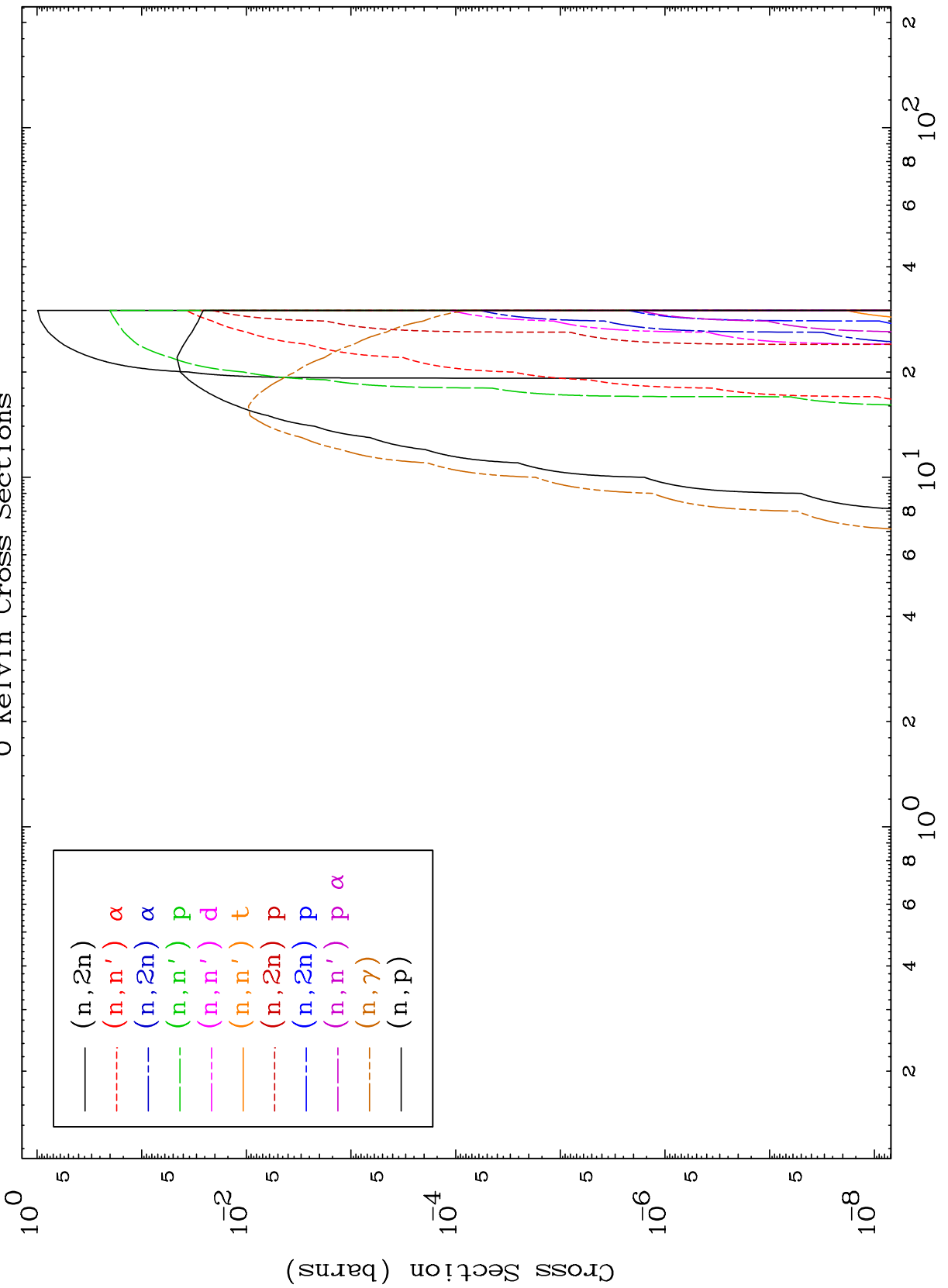
51-Sb-117



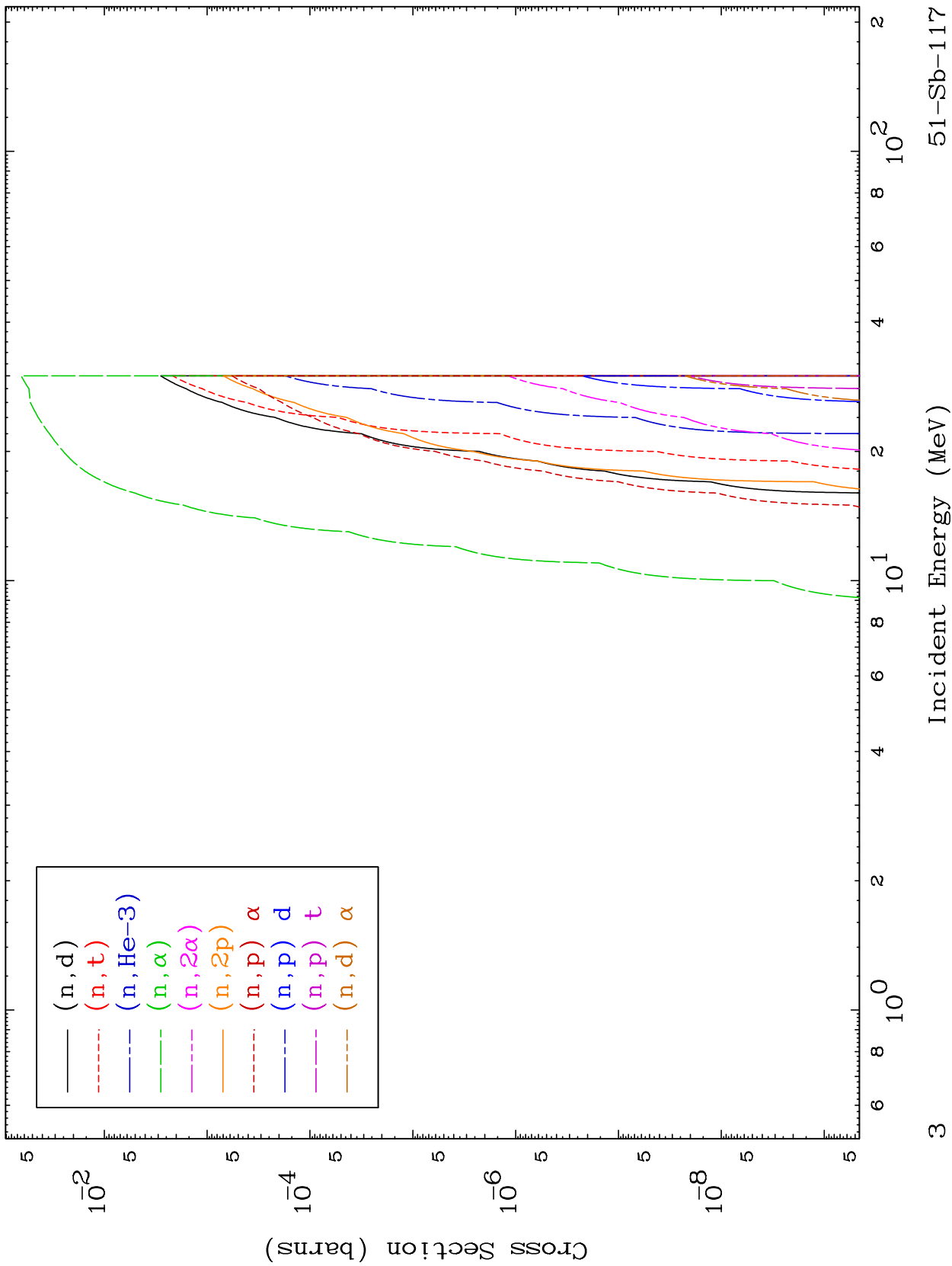
MAT 5113

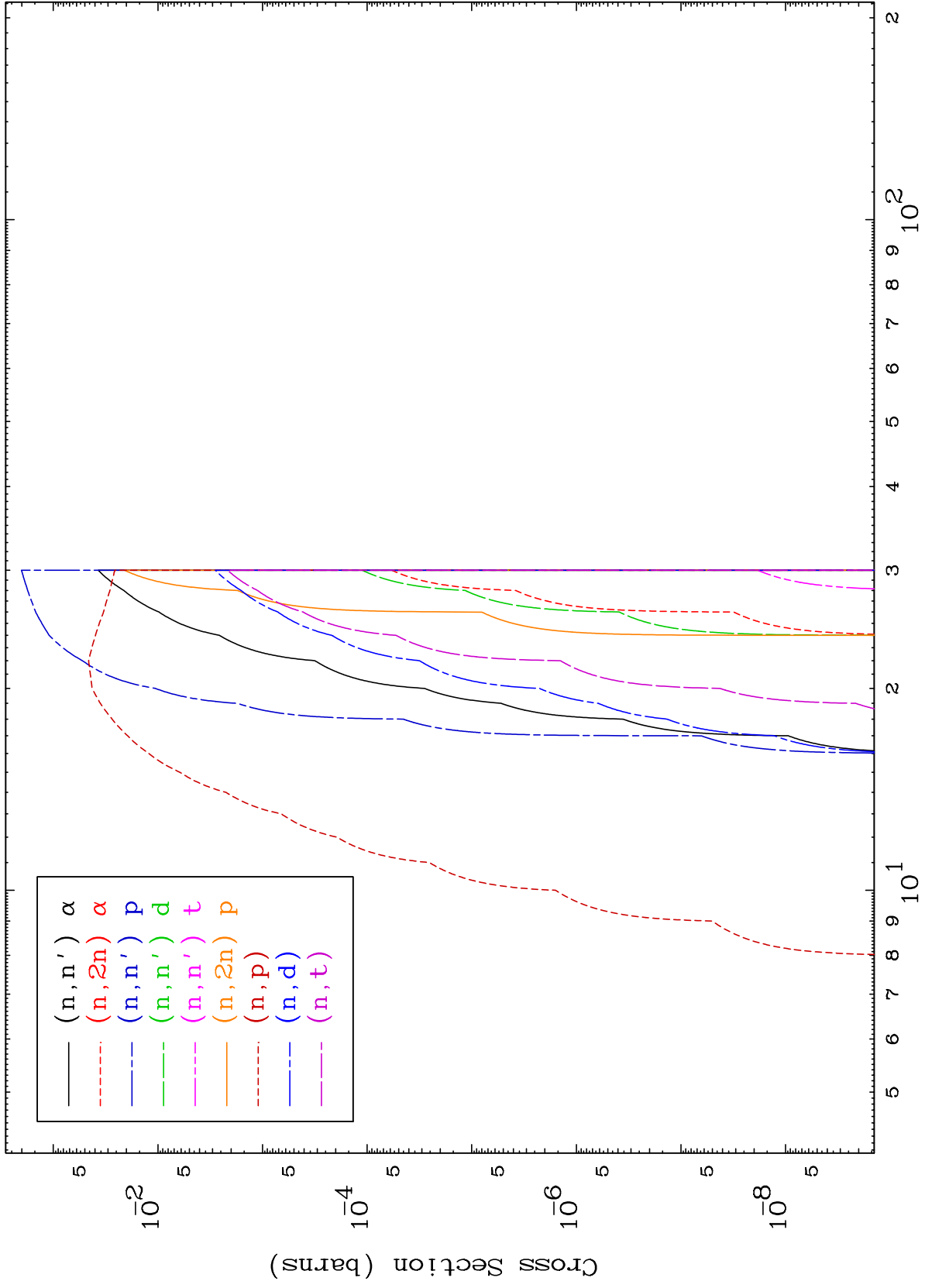
$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

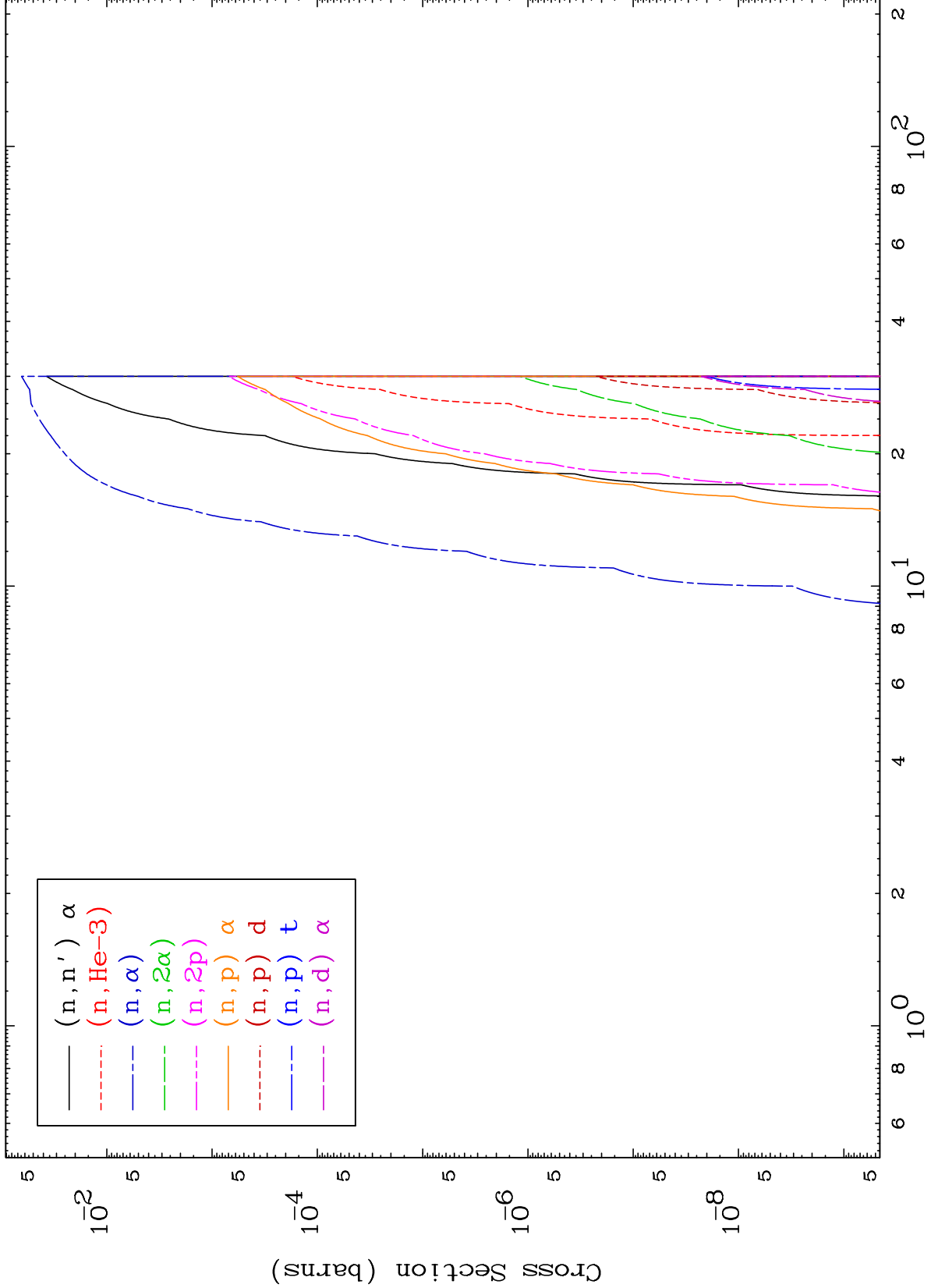
51-Sb-117



$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections



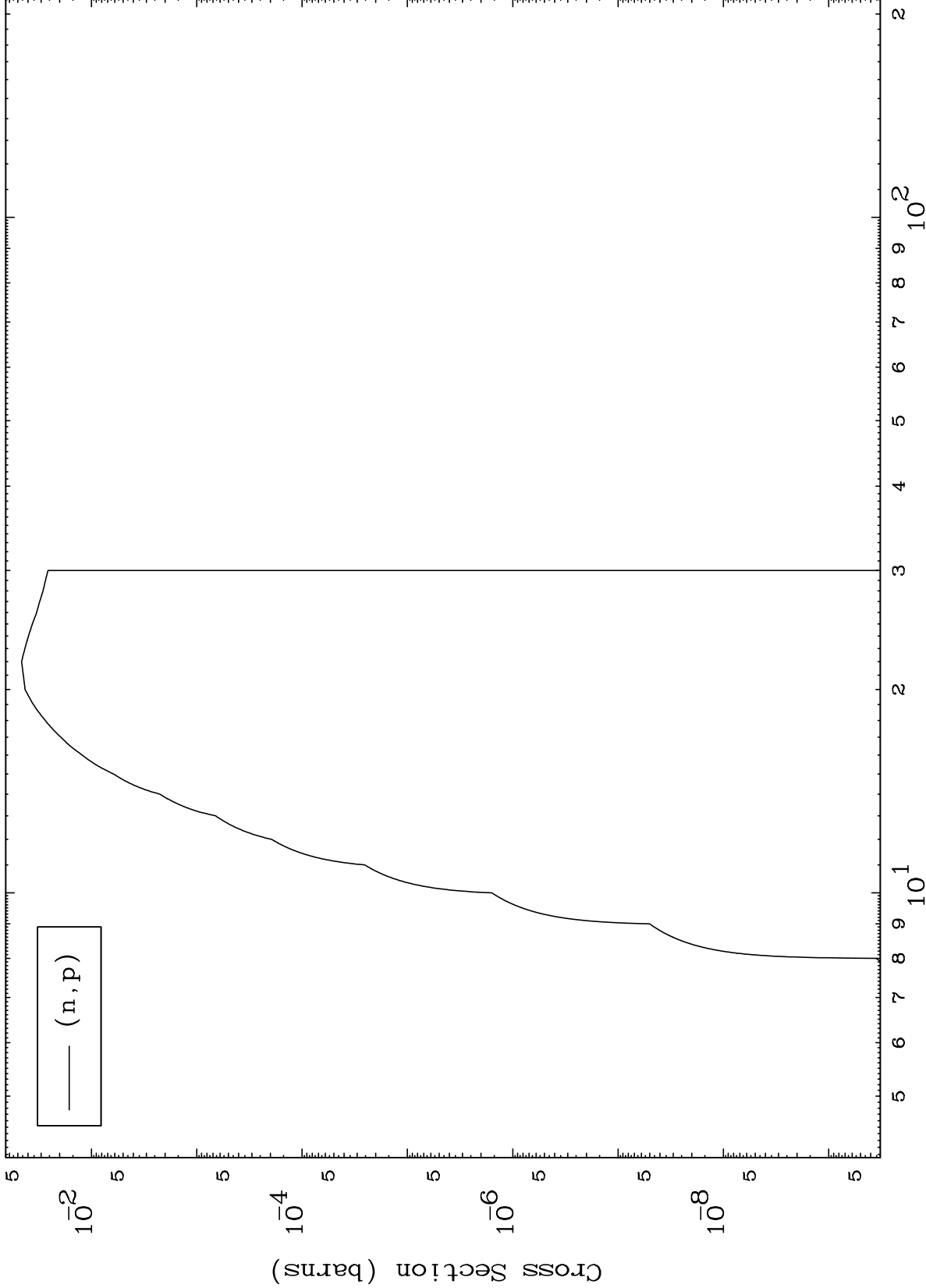




MAT 5113

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

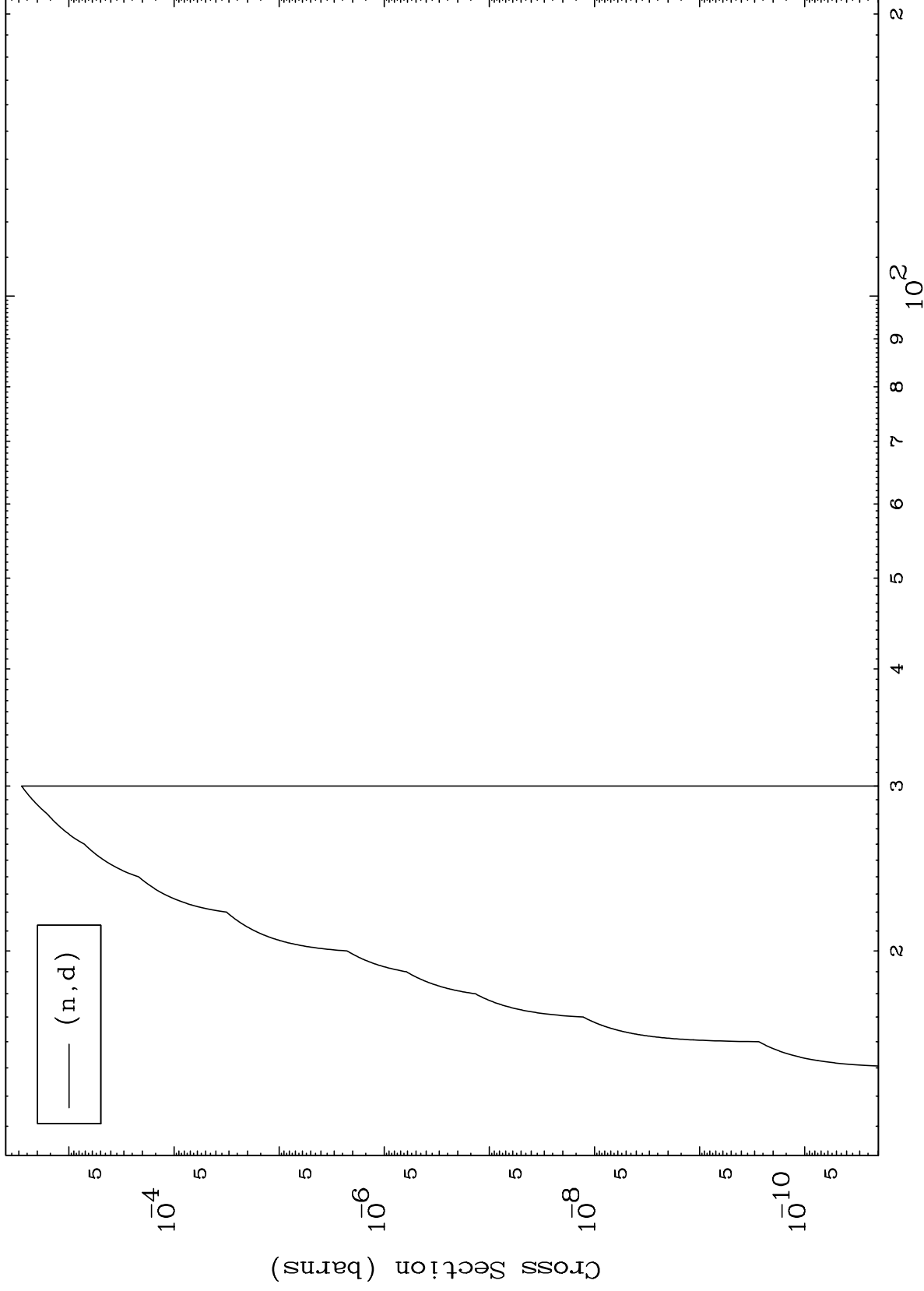
51-Sb-117

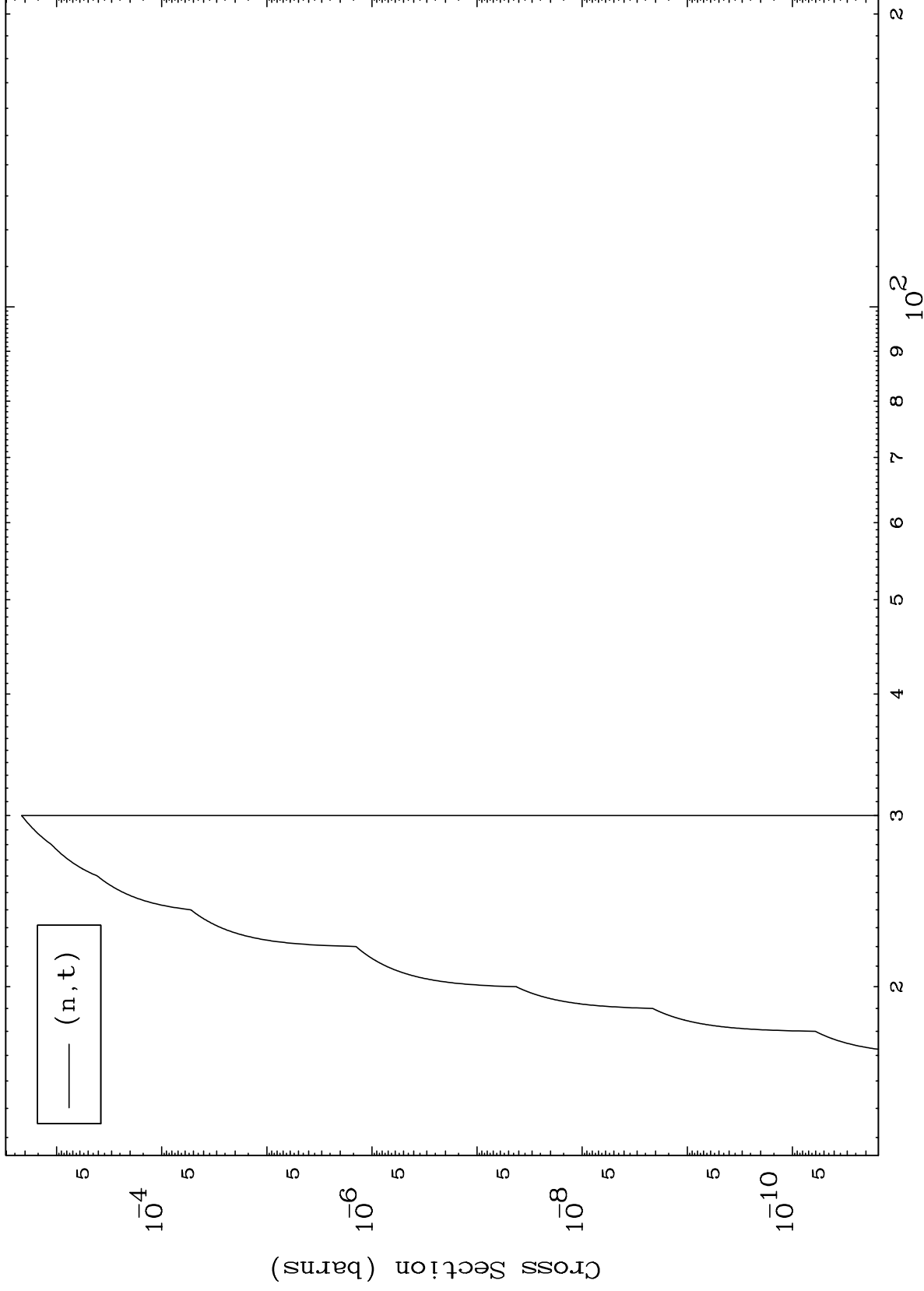


6

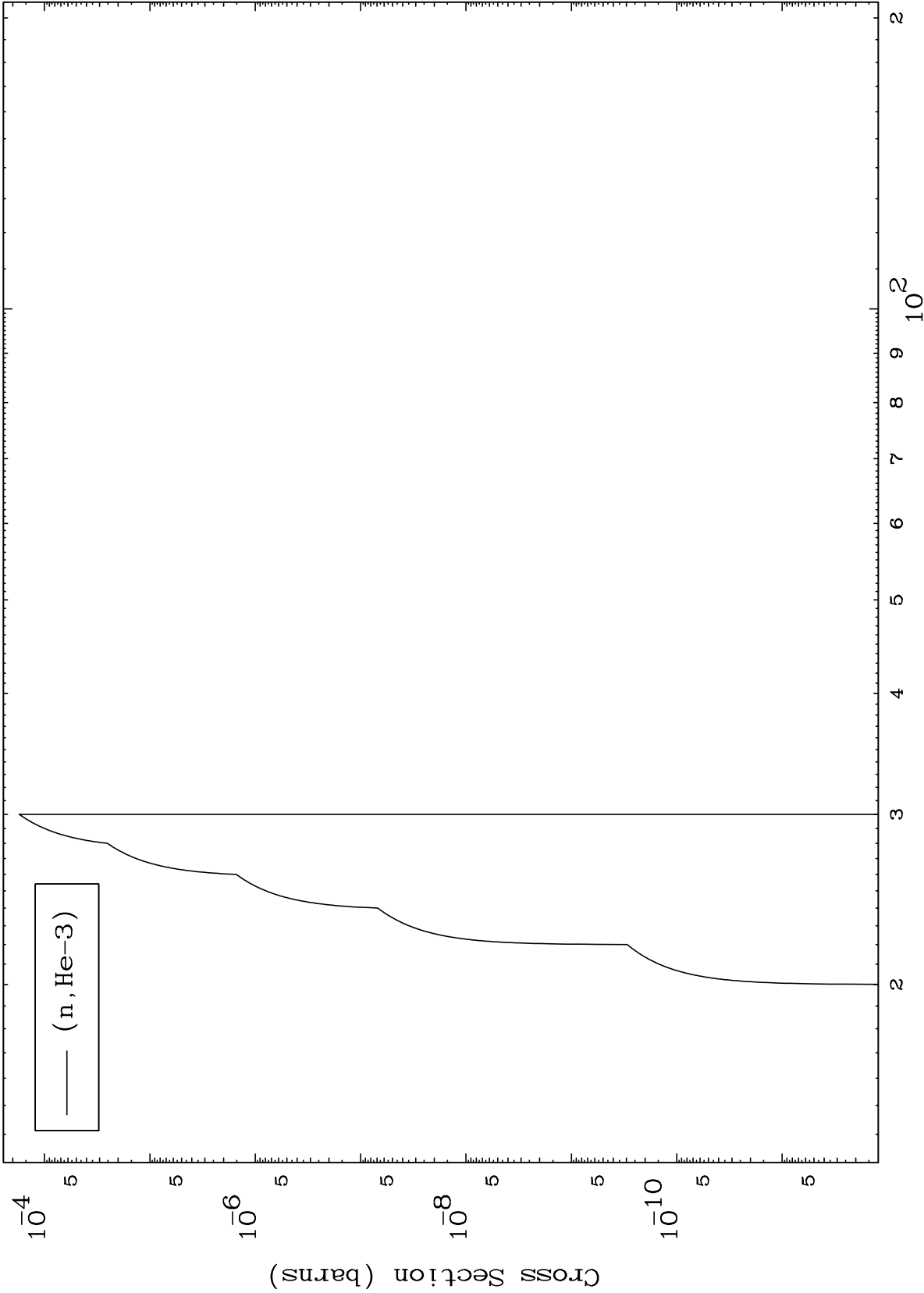
Incident Energy (MeV)

51-Sb-117





( $\alpha, \text{He3}$ ) Levels  
0 Kelvin Cross Sections

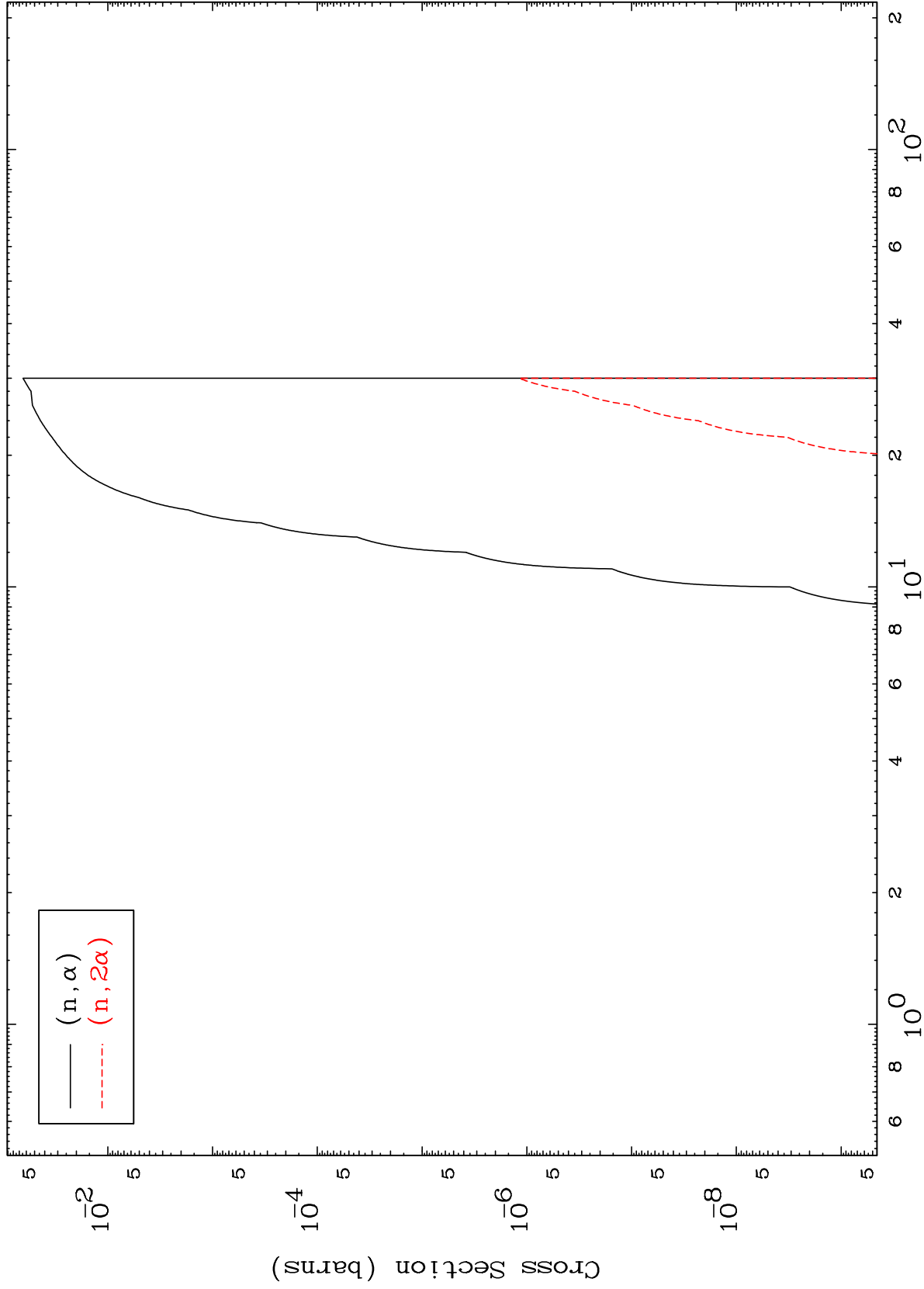


MAT 5113

( $\alpha, \alpha$ ) Levels

51-Sb-117

0 Kelvin Cross Sections



10

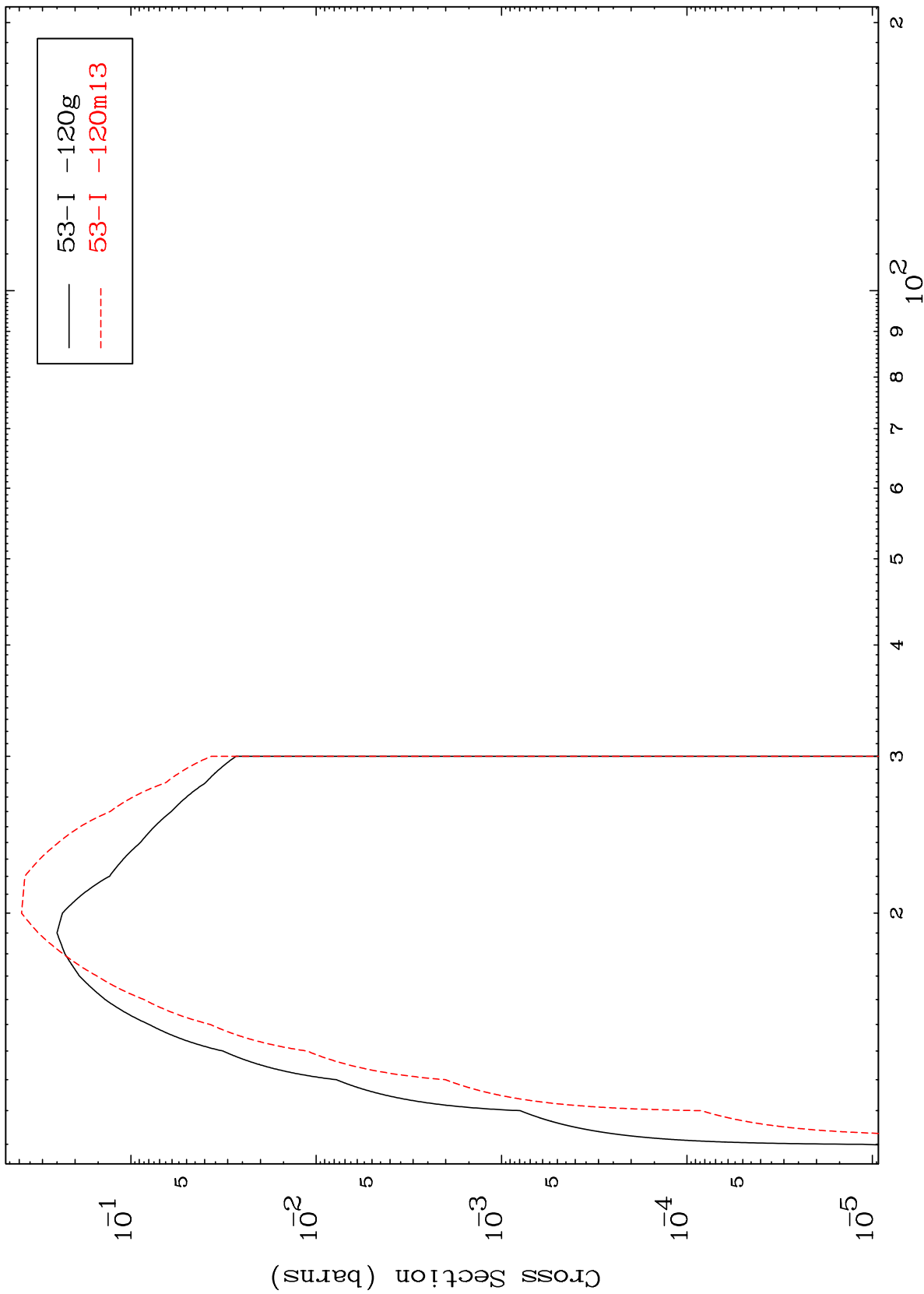
Incident Energy (MeV)

51-Sb-117

MAT 5113

51-Sb-117

Inelastic  
Radionuclide Production Cross Section



11

Incident Energy (MeV)

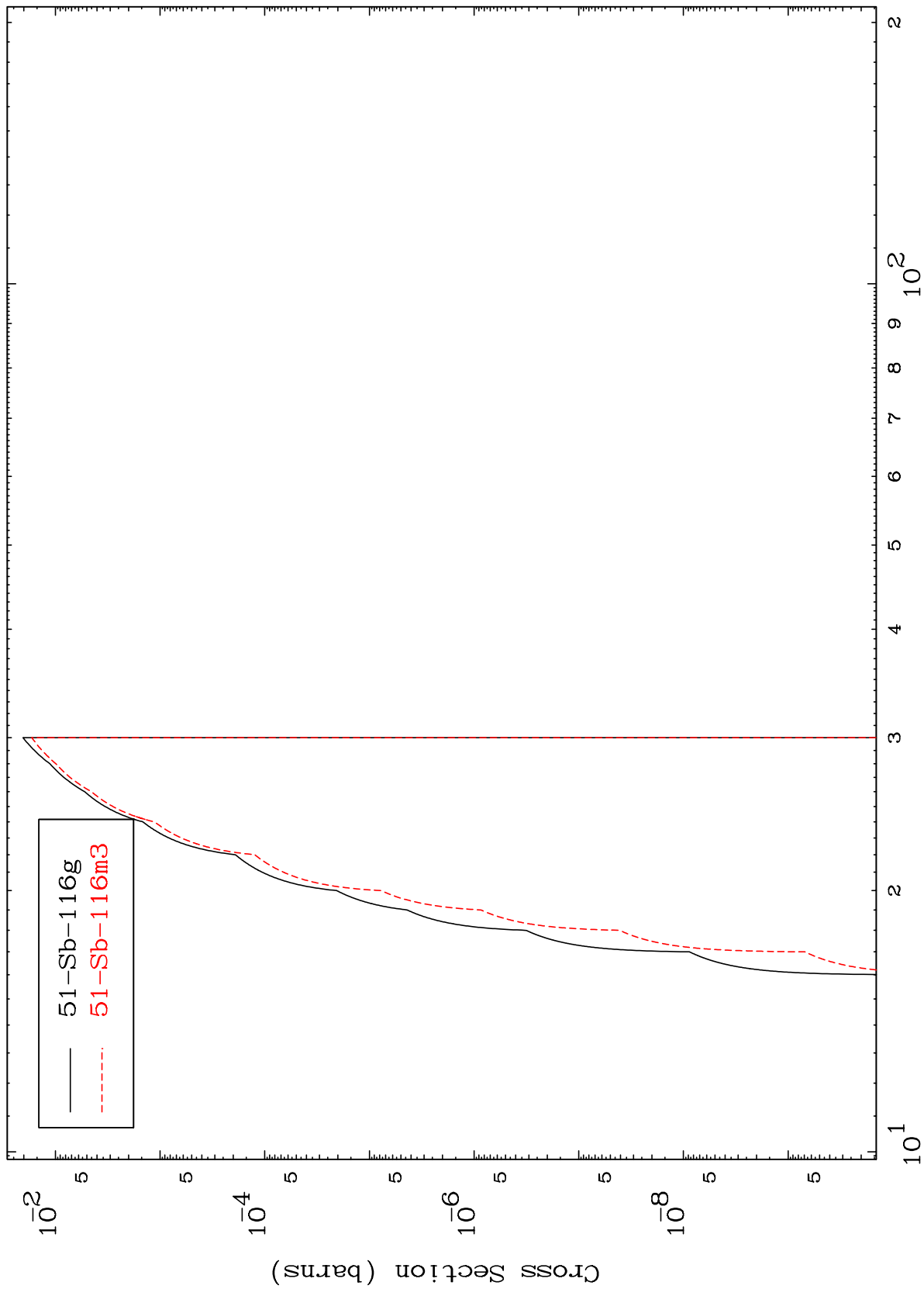
51-Sb-117

MAT 5113

(n,n')  $\alpha$

51-Sb-117

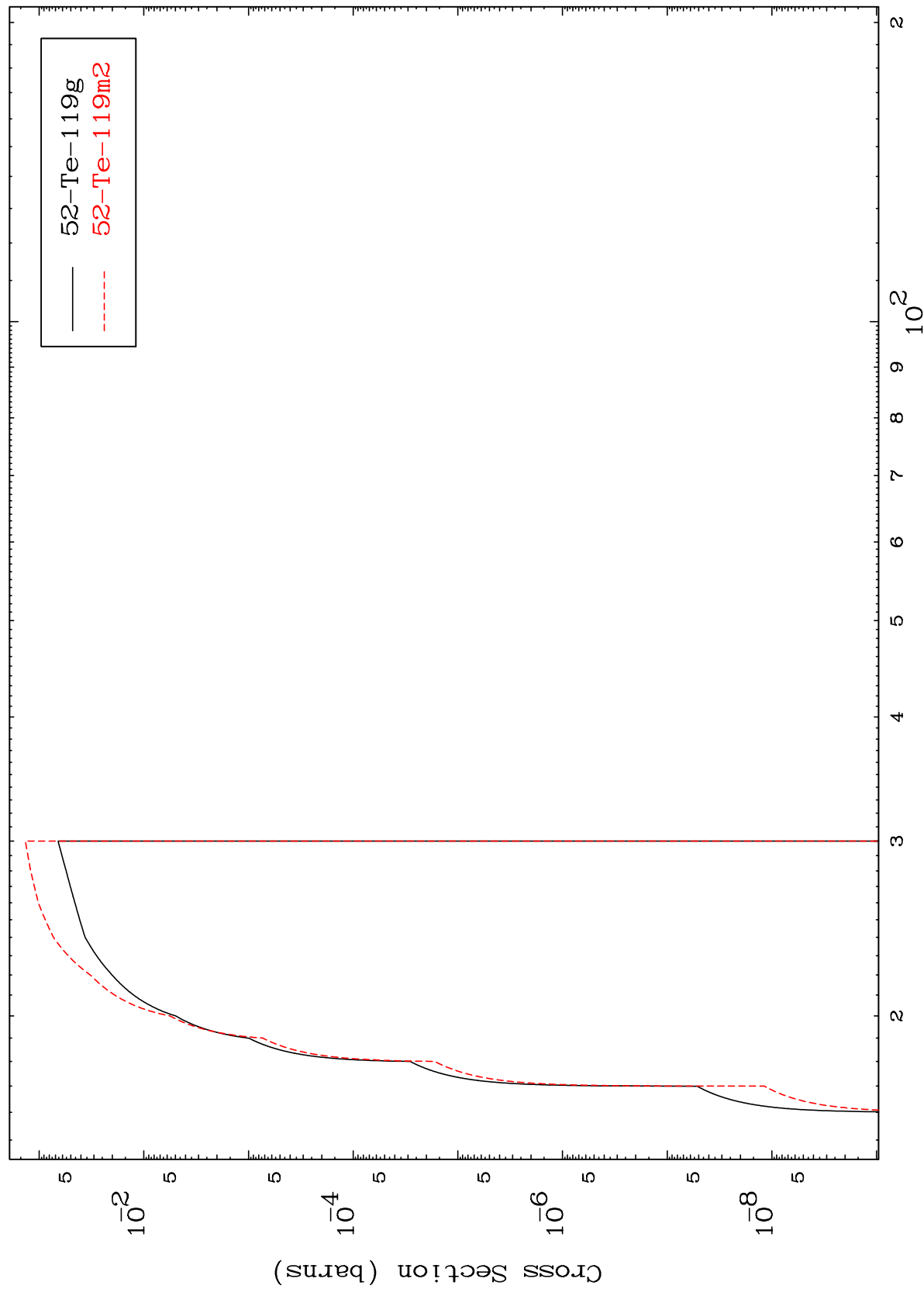
Radionuclide Production Cross Section



Incident Energy (MeV)

51-Sb-117

Radionuclide Production Cross Section

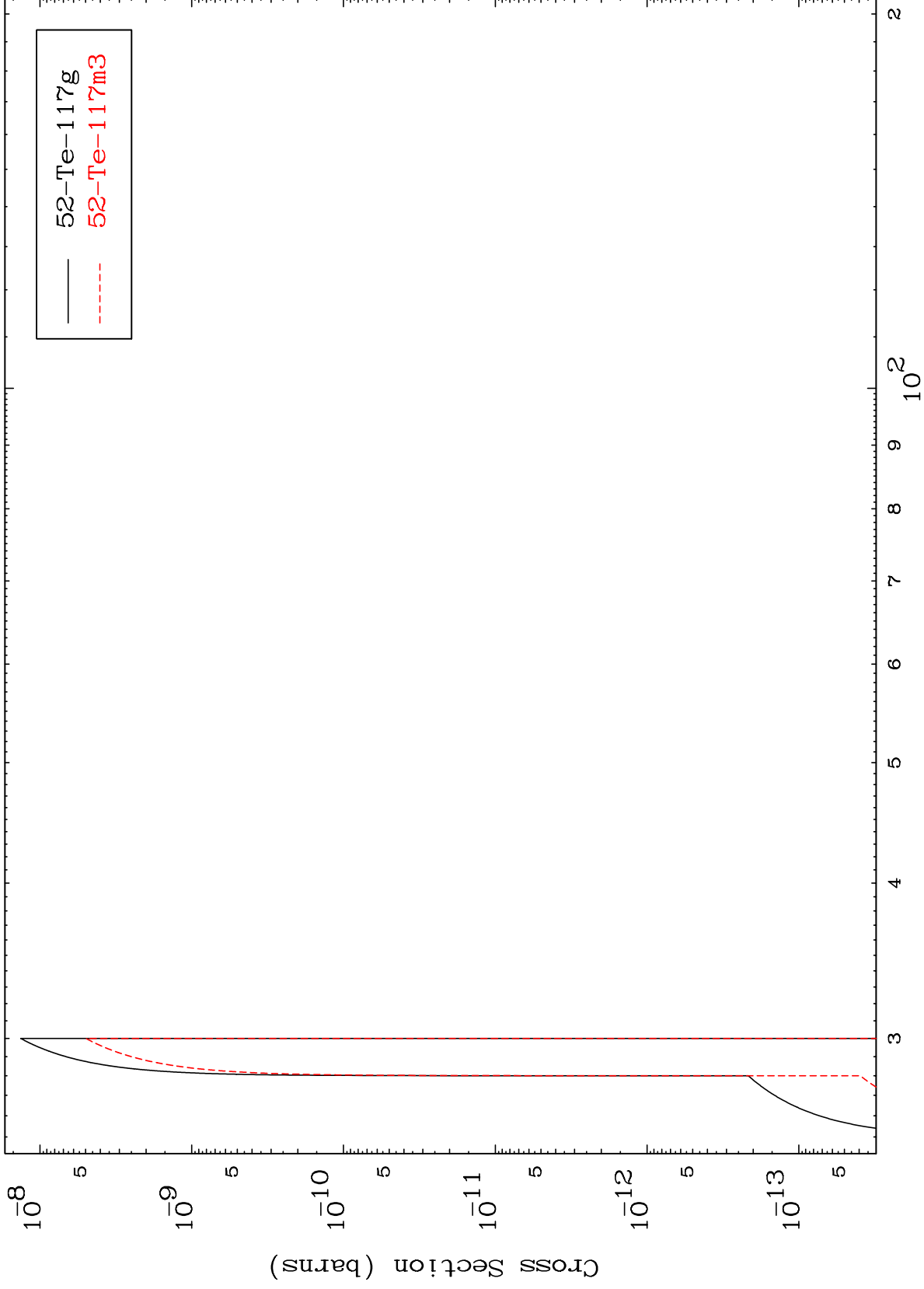


MAT 5113

(n,n') t

51-Sb-117

Radionuclide Production Cross Section



14

Incident Energy (MeV)

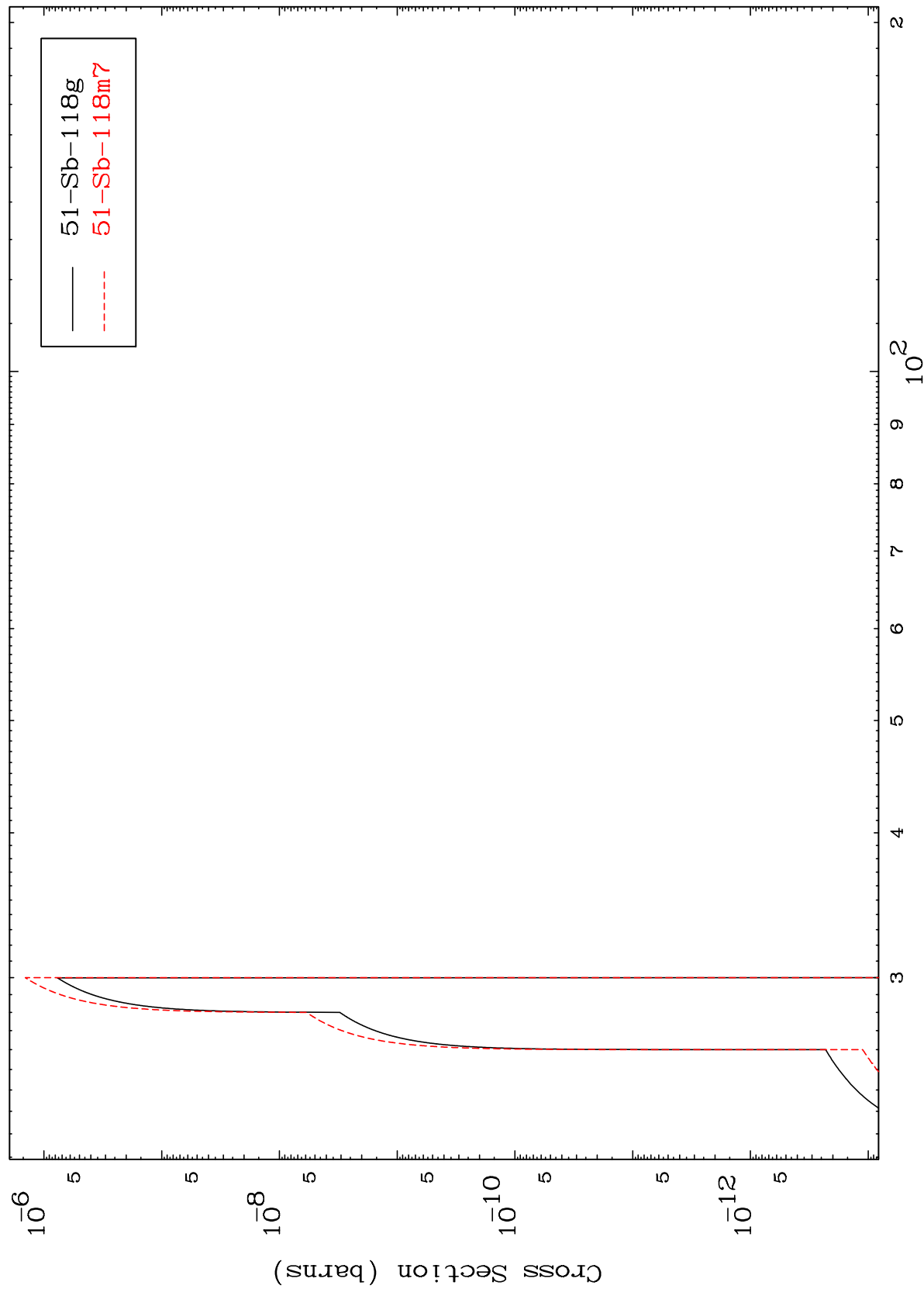
51-Sb-117

MAT 5113

(n,2n) p

51-Sb-117

Radionuclide Production Cross Section



51-Sb-118g  
51-Sb-118m7

15

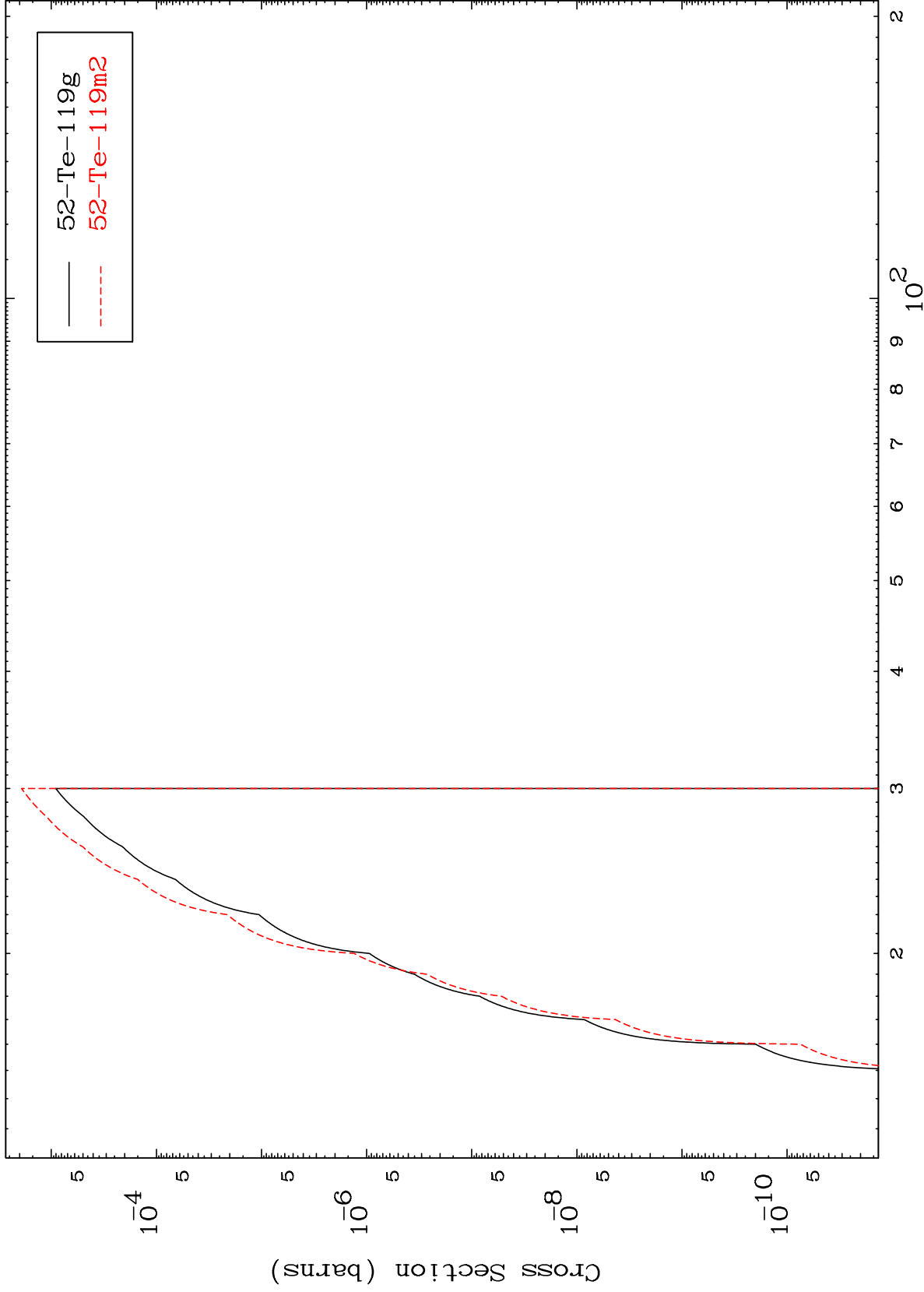
Incident Energy (MeV)

51-Sb-117

MAT 5113

51-Sb-117

(n,d)  
Radionuclide Production Cross Section



16

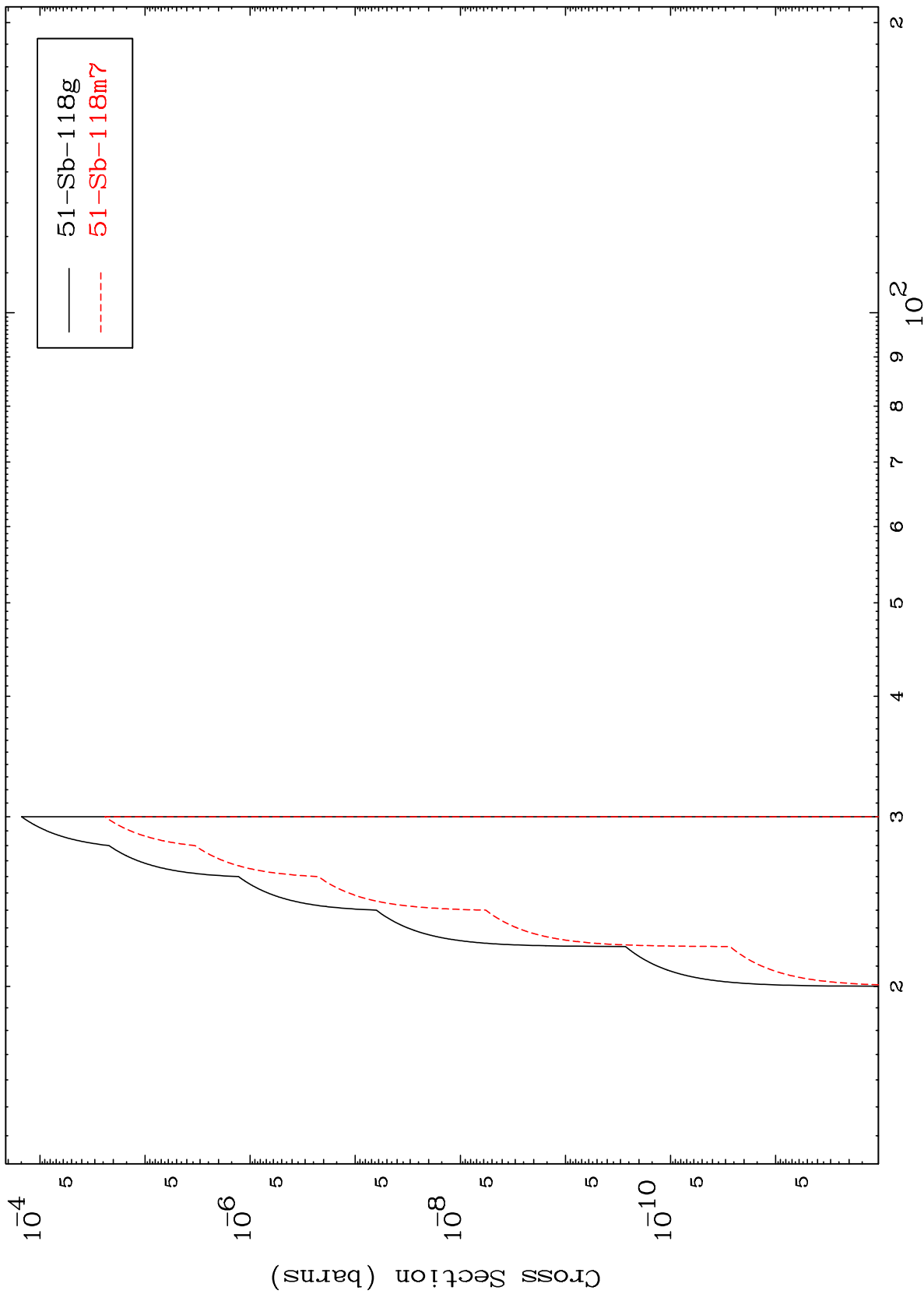
Incident Energy (MeV)

51-Sb-117

MAT 5113

51-Sb-117

(n,He-3)  
Radionuclide Production Cross Section



51-Sb-117

Incident Energy (MeV)

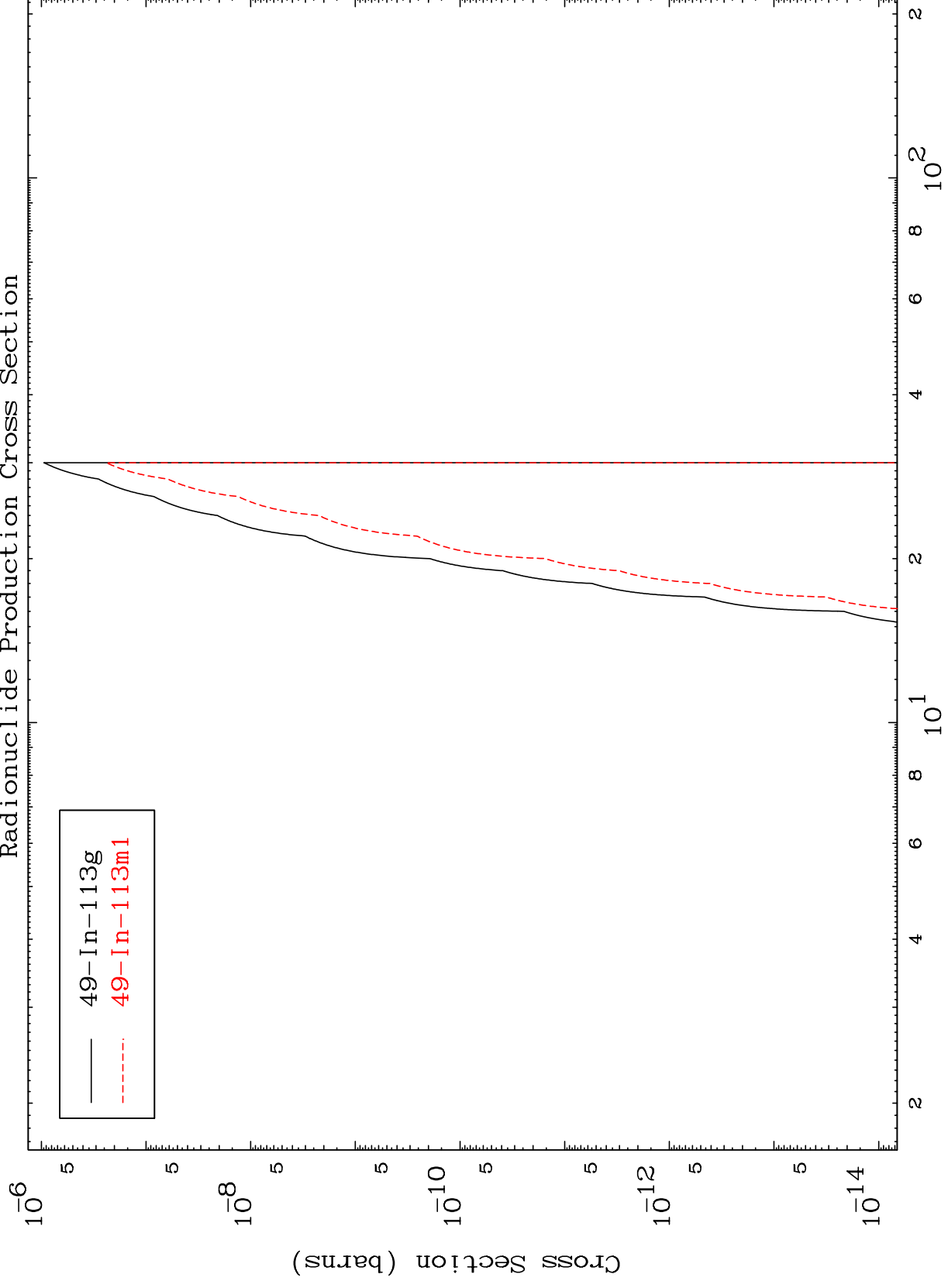
17

MAT 5113

(n,2α)

51-Sb-117

Radionuclide Production Cross Section



— 49-In-113g  
- - - 49-In-113m1

18

Incident Energy (MeV)

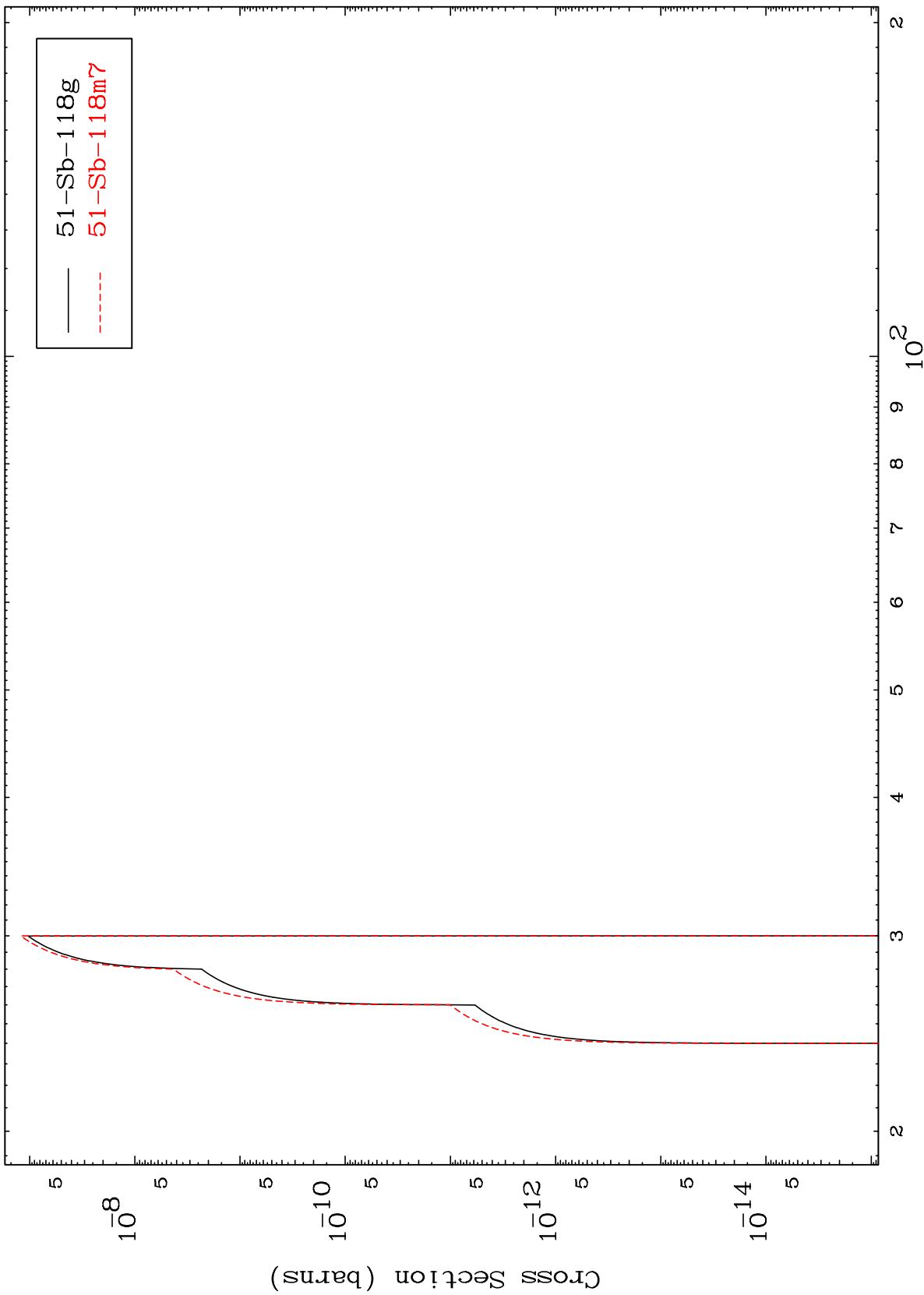
51-Sb-117

MAT 5113

(n,p) d

51-Sb-117

Radionuclide Production Cross Section



19

Incident Energy (MeV)

51-Sb-117