

Program EVALPLOT  
(Version 2021-1)

by

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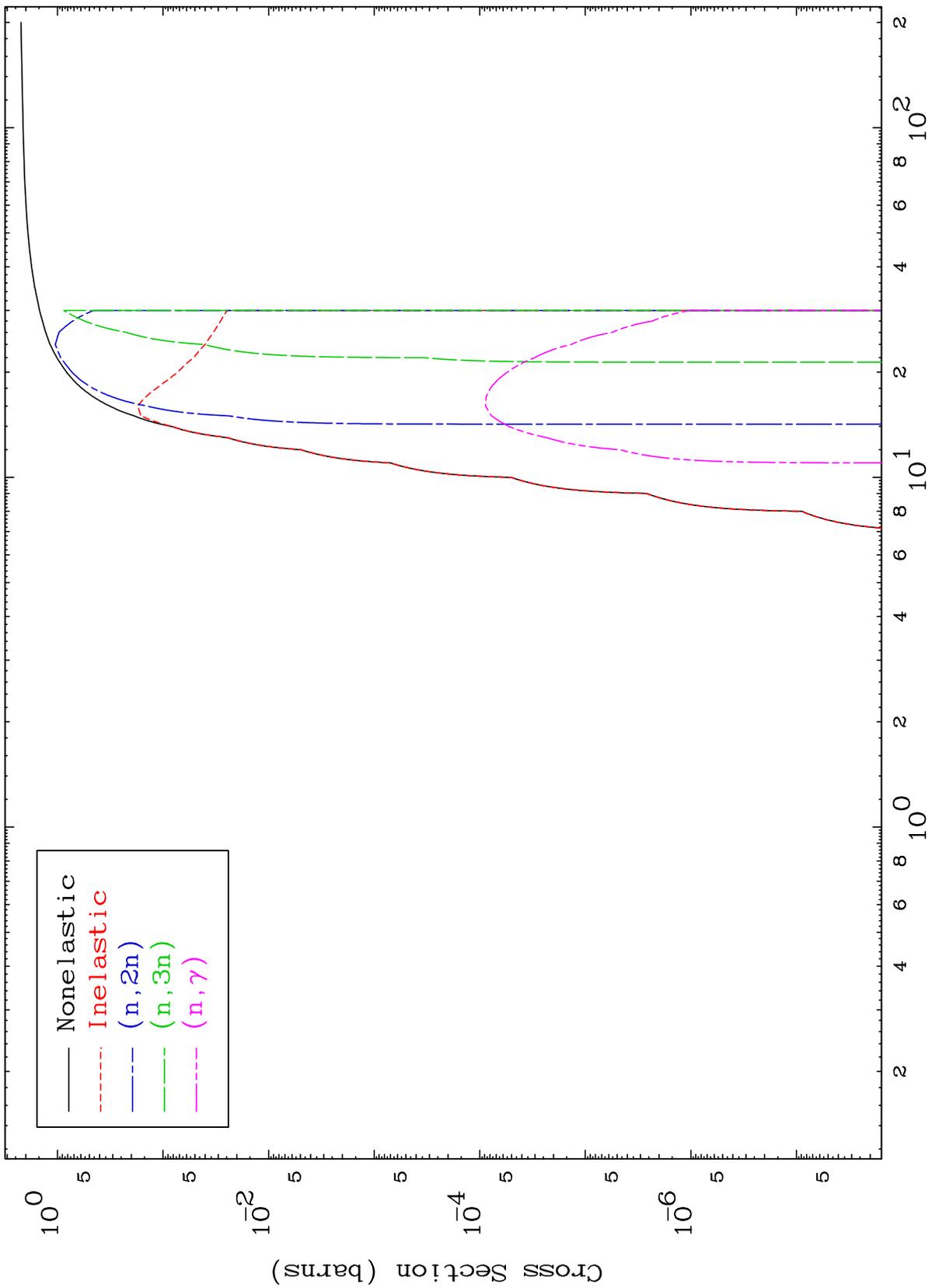
Press Mouse Button to Start

MAT 5052

$\alpha$  Major

50-Sn-121

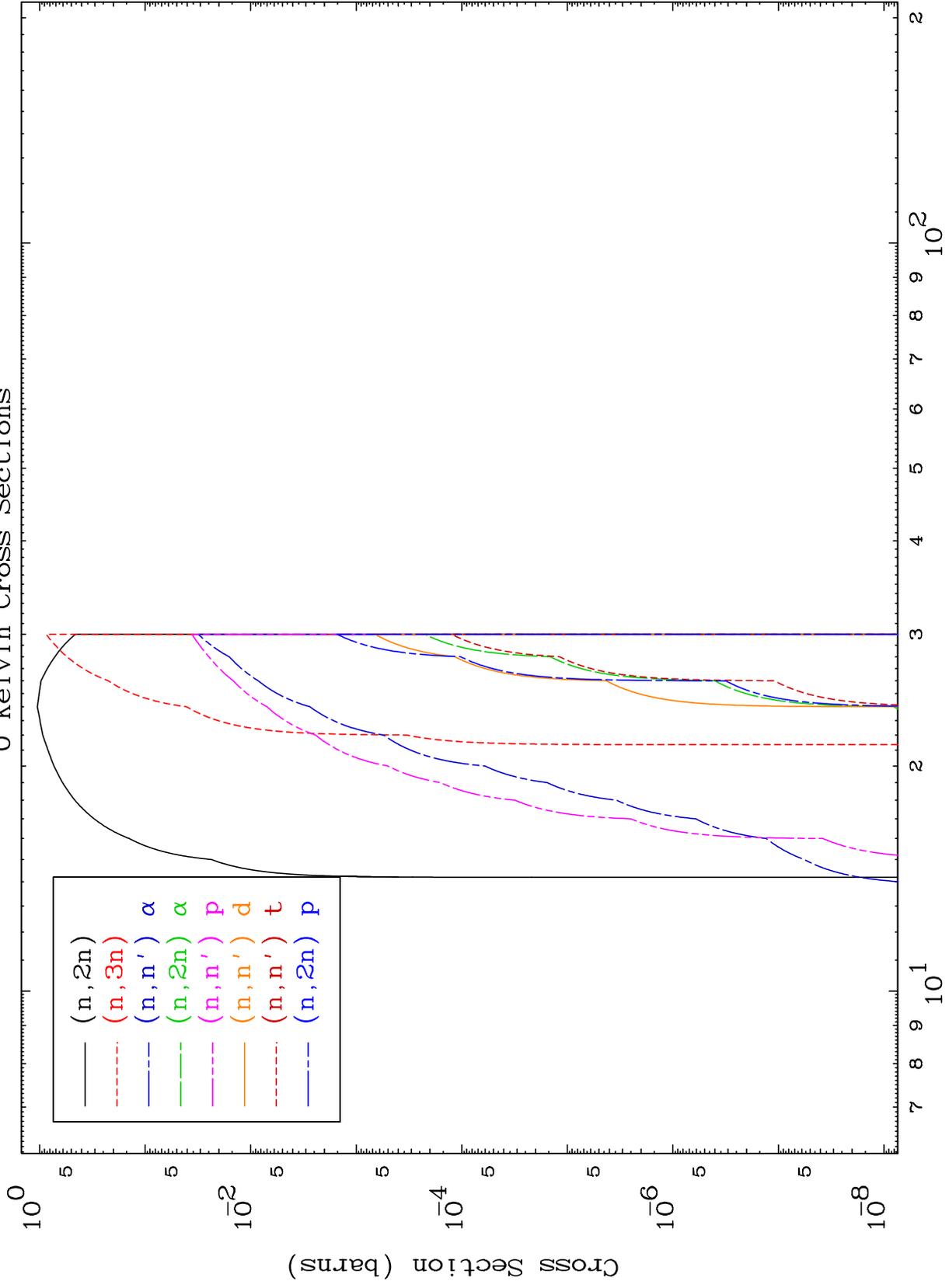
0 Kelvin Cross Sections



MAT 5052

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

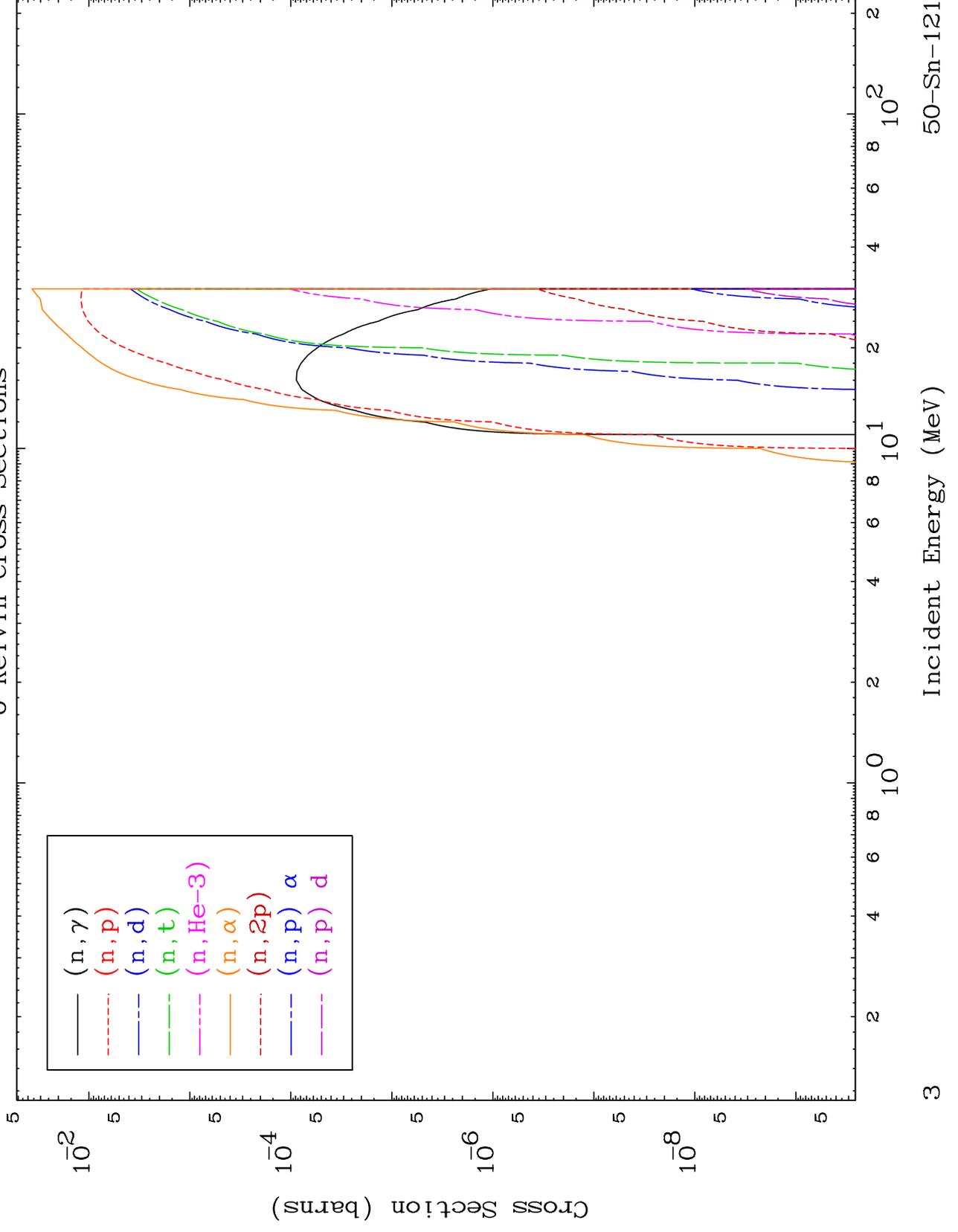
50-Sn-121

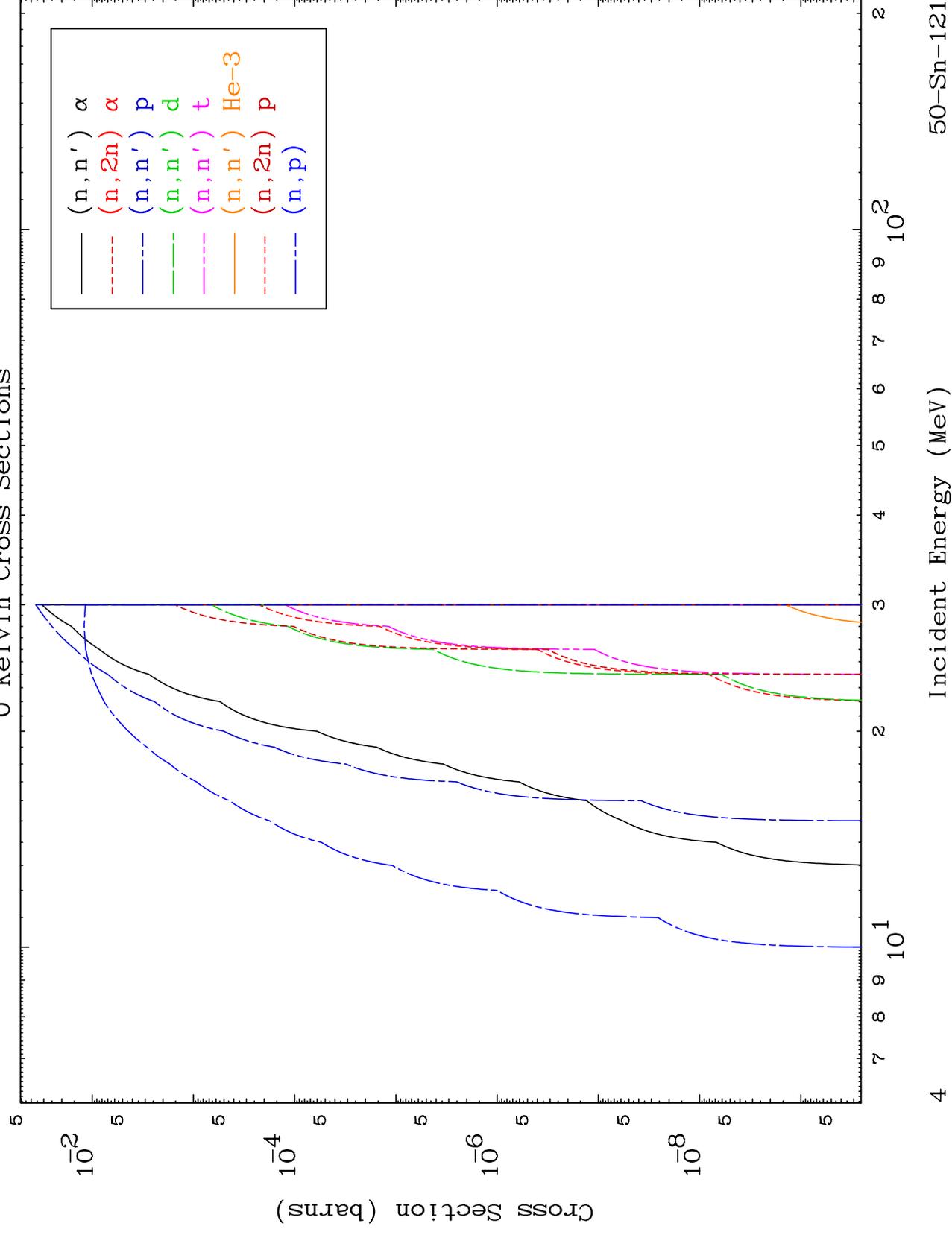


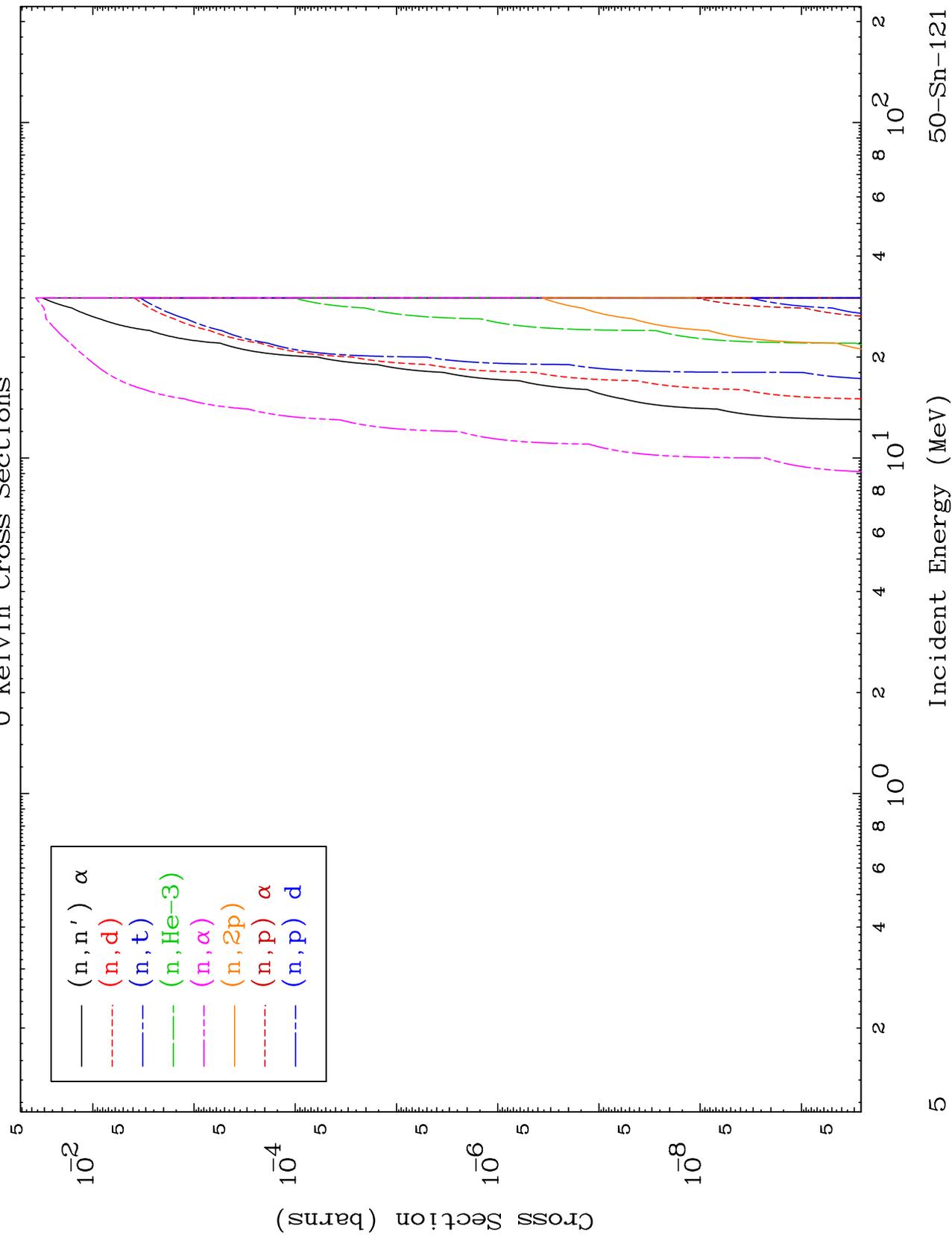
MAT 5052

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

50-Sn-121



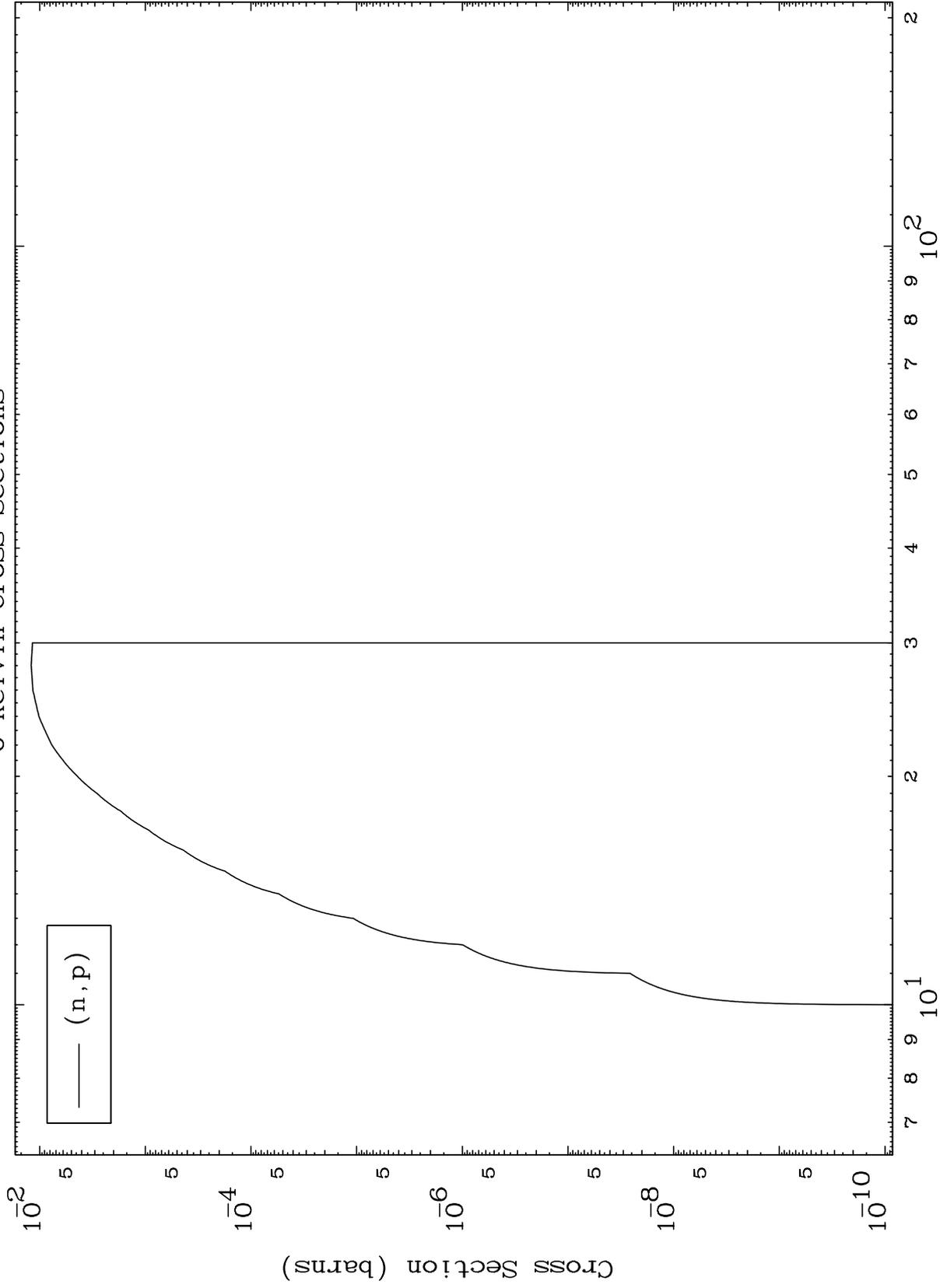




MAT 5052

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

50-Sn-121



6

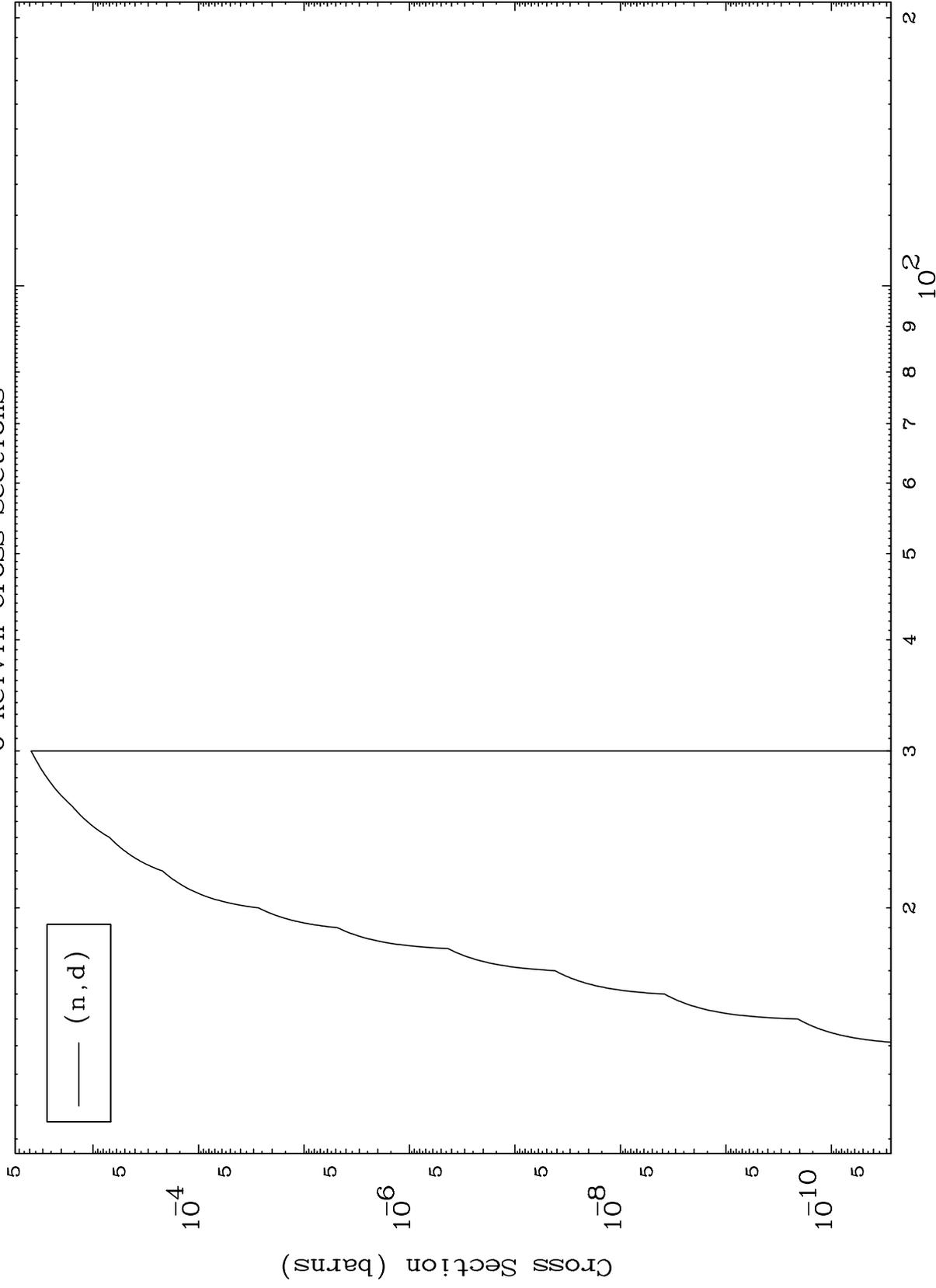
Incident Energy (MeV)

50-Sn-121

MAT 5052

50-Sn-121

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

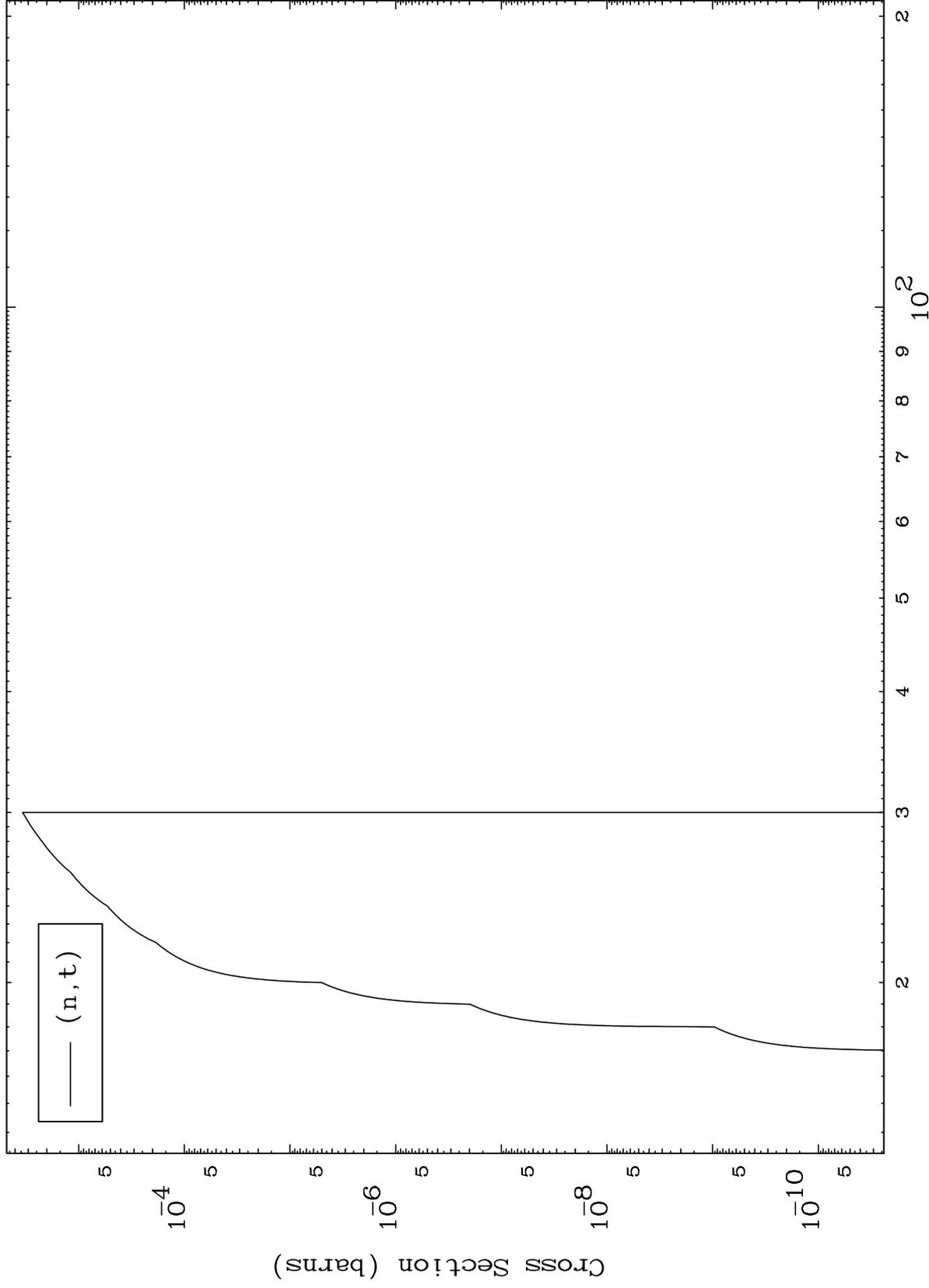


50-Sn-121

Incident Energy (MeV)

7

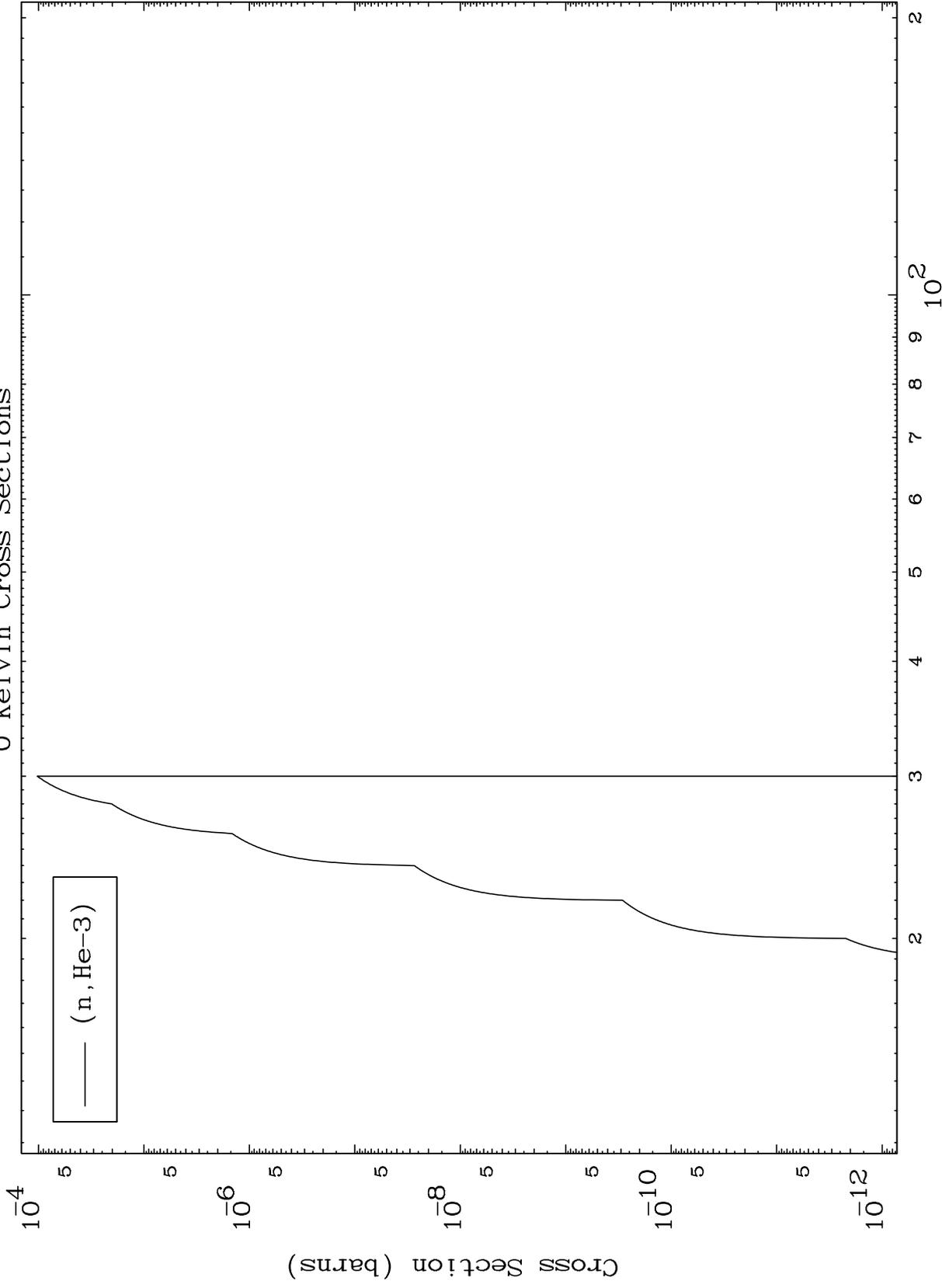
( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections



MAT 5052

50-Sn-121

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

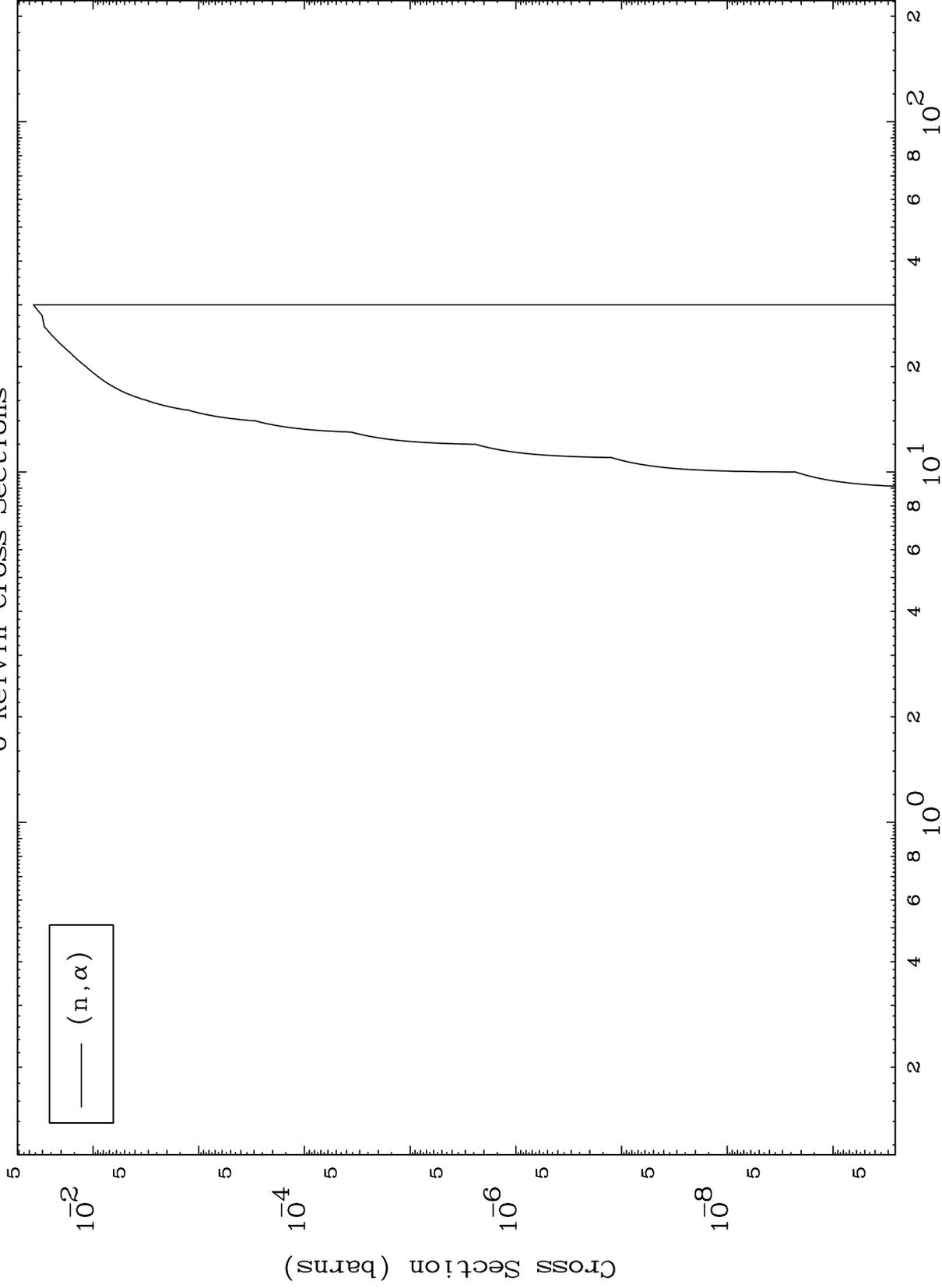


MAT 5052

( $\alpha, \alpha$ ) Levels

50-Sn-121

0 Kelvin Cross Sections



10

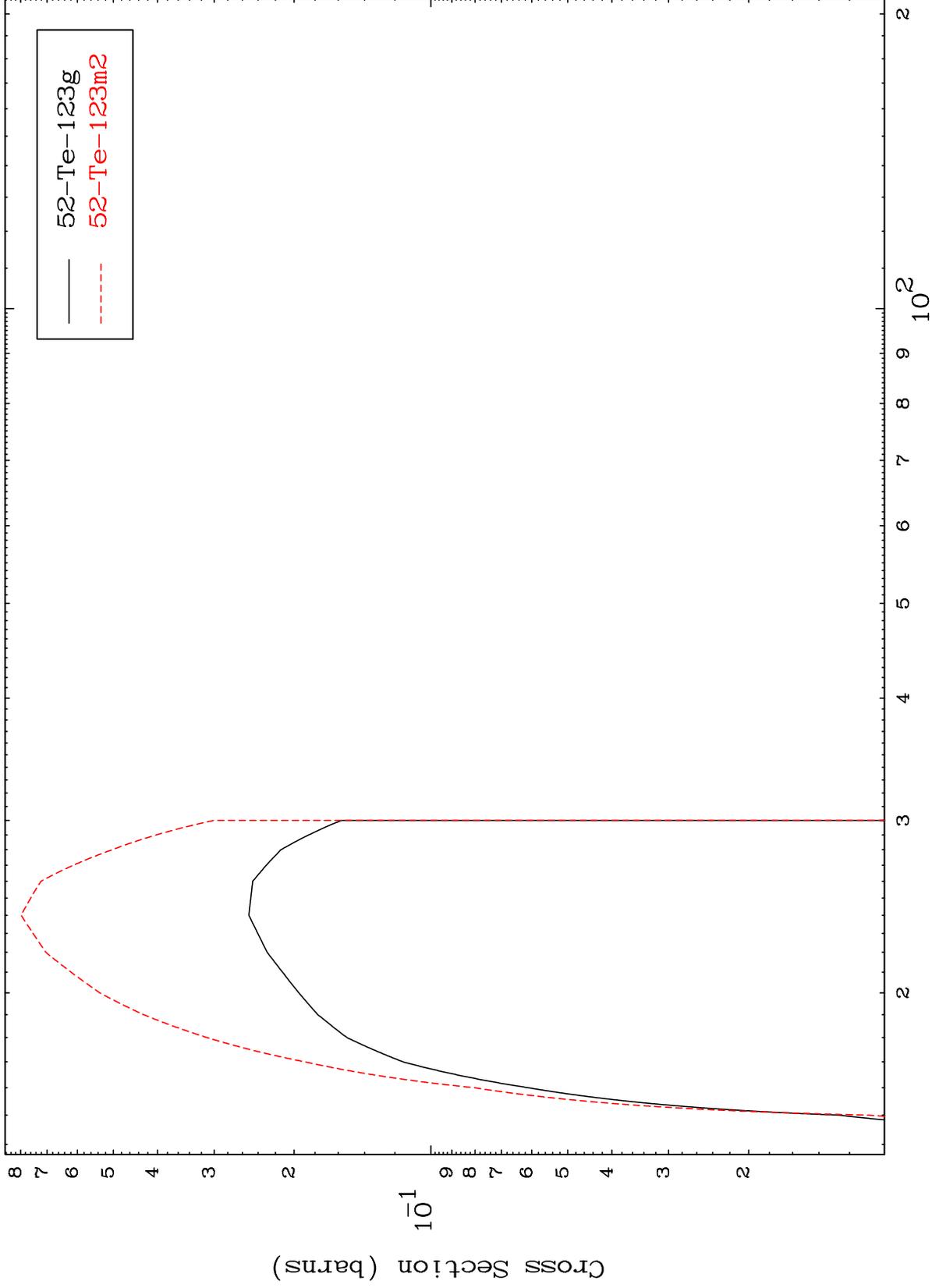
Incident Energy (MeV)

50-Sn-121

MAT 5052

50-Sn-121

(n,2n)  
Radionuclide Production Cross Section



11

Incident Energy (MeV)

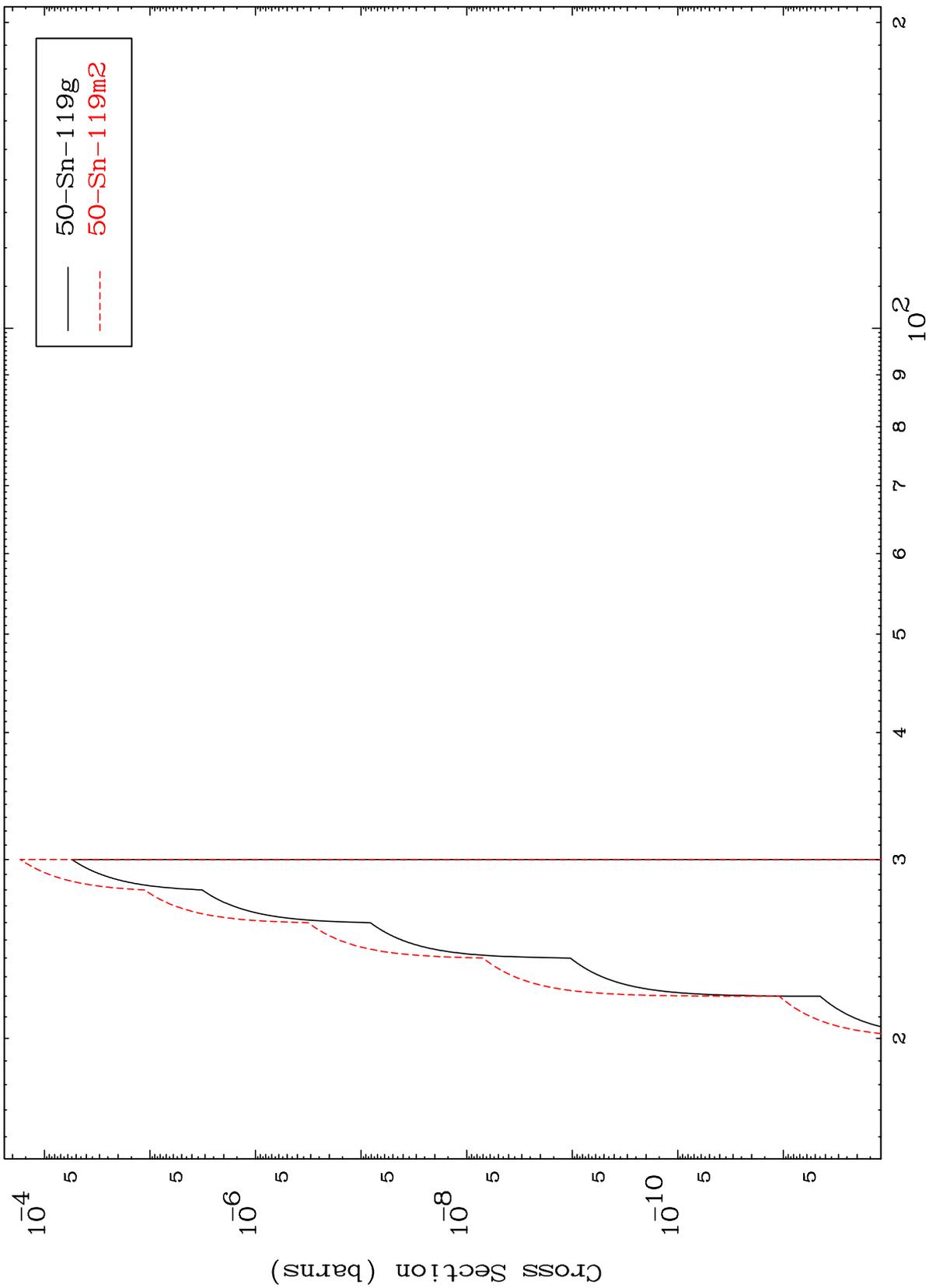
50-Sn-121

MAT 5052

(n,2n)  $\alpha$

50-Sn-121

Radionuclide Production Cross Section



12

Incident Energy (MeV)

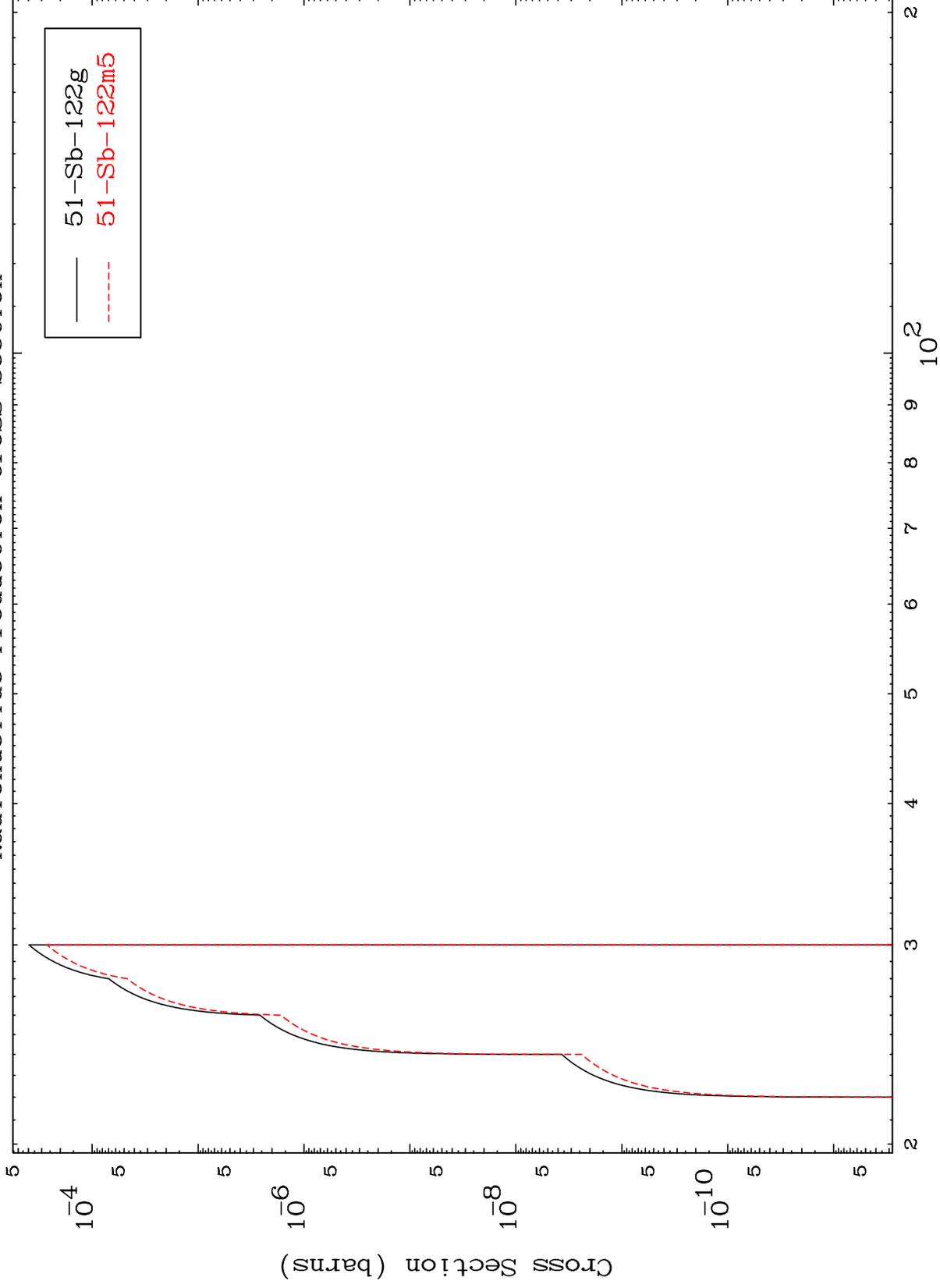
50-Sn-121

MAT 5052

(n,n') d

50-Sn-121

Radionuclide Production Cross Section



13

Incident Energy (MeV)

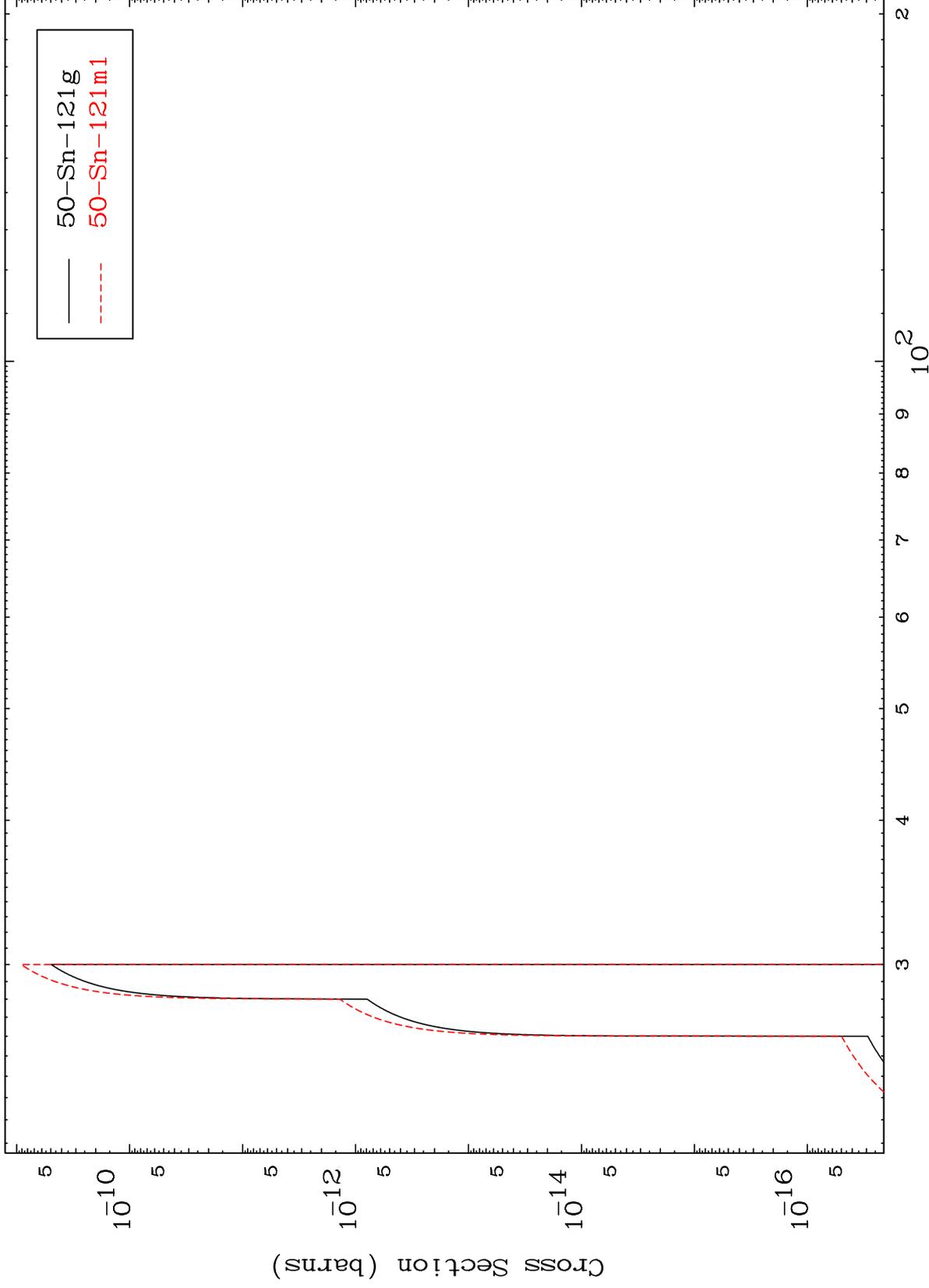
50-Sn-121

MAT 5052

(n,n') He-3

50-Sn-121

Radionuclide Production Cross Section



14

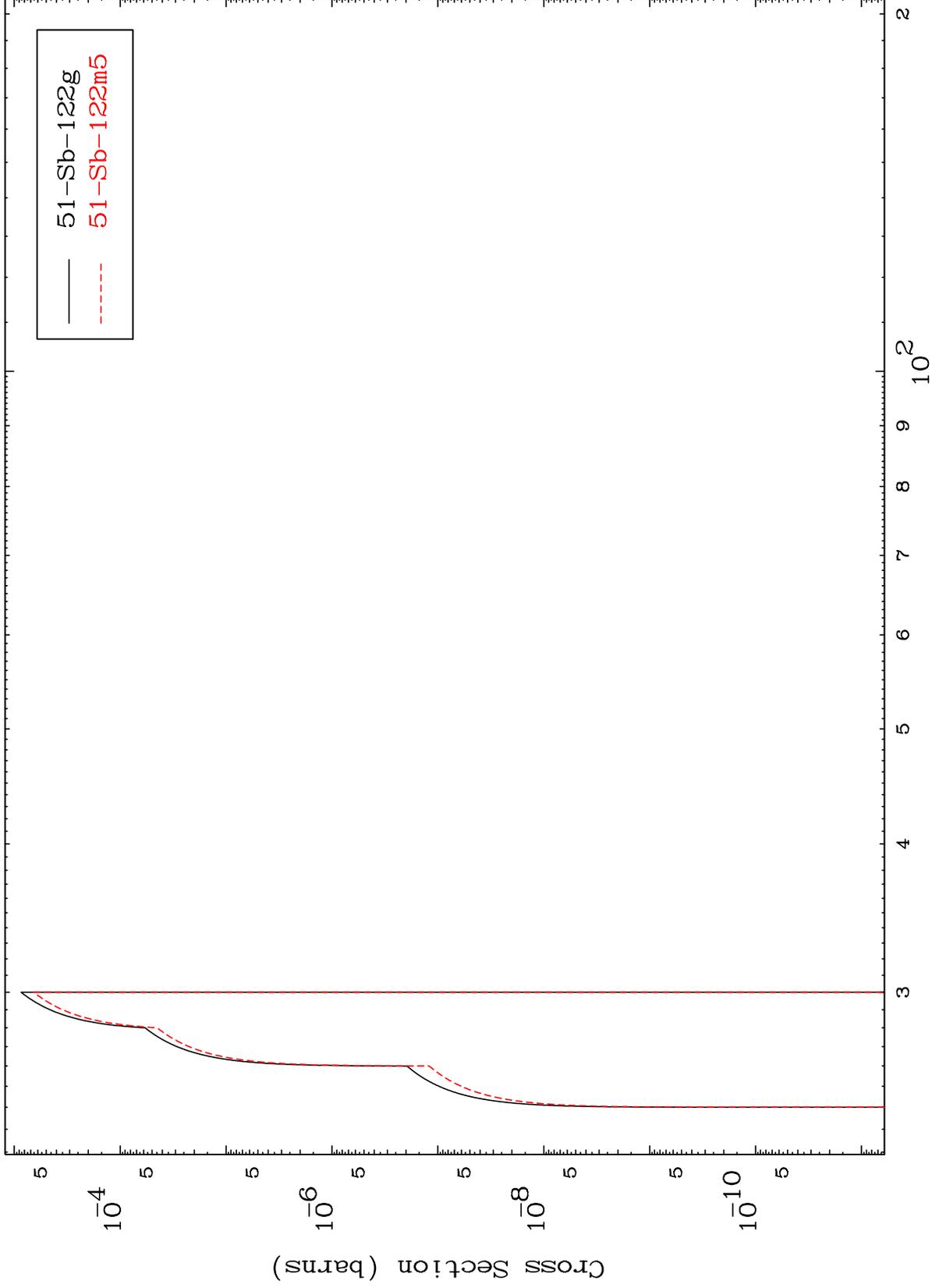
Incident Energy (MeV)

50-Sn-121

MAT 5052

50-Sn-121

(n,2n) p  
Radionuclide Production Cross Section



50-Sn-121

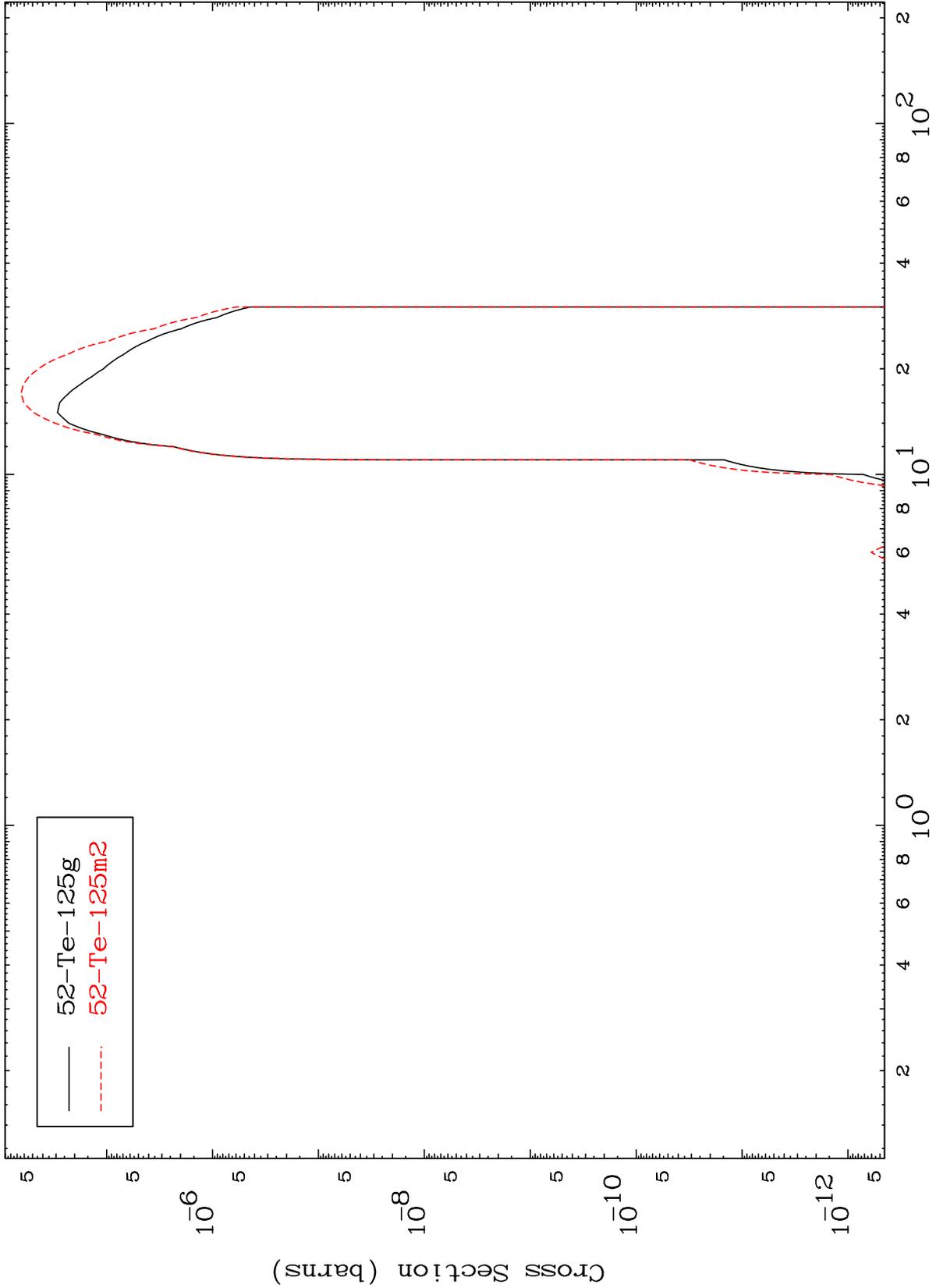
Incident Energy (MeV)

15

MAT 5052

50-Sn-121

(n,  $\gamma$ )  
Radionuclide Production Cross Section



— 52-Te-125g  
- - - 52-Te-125m2

50-Sn-121

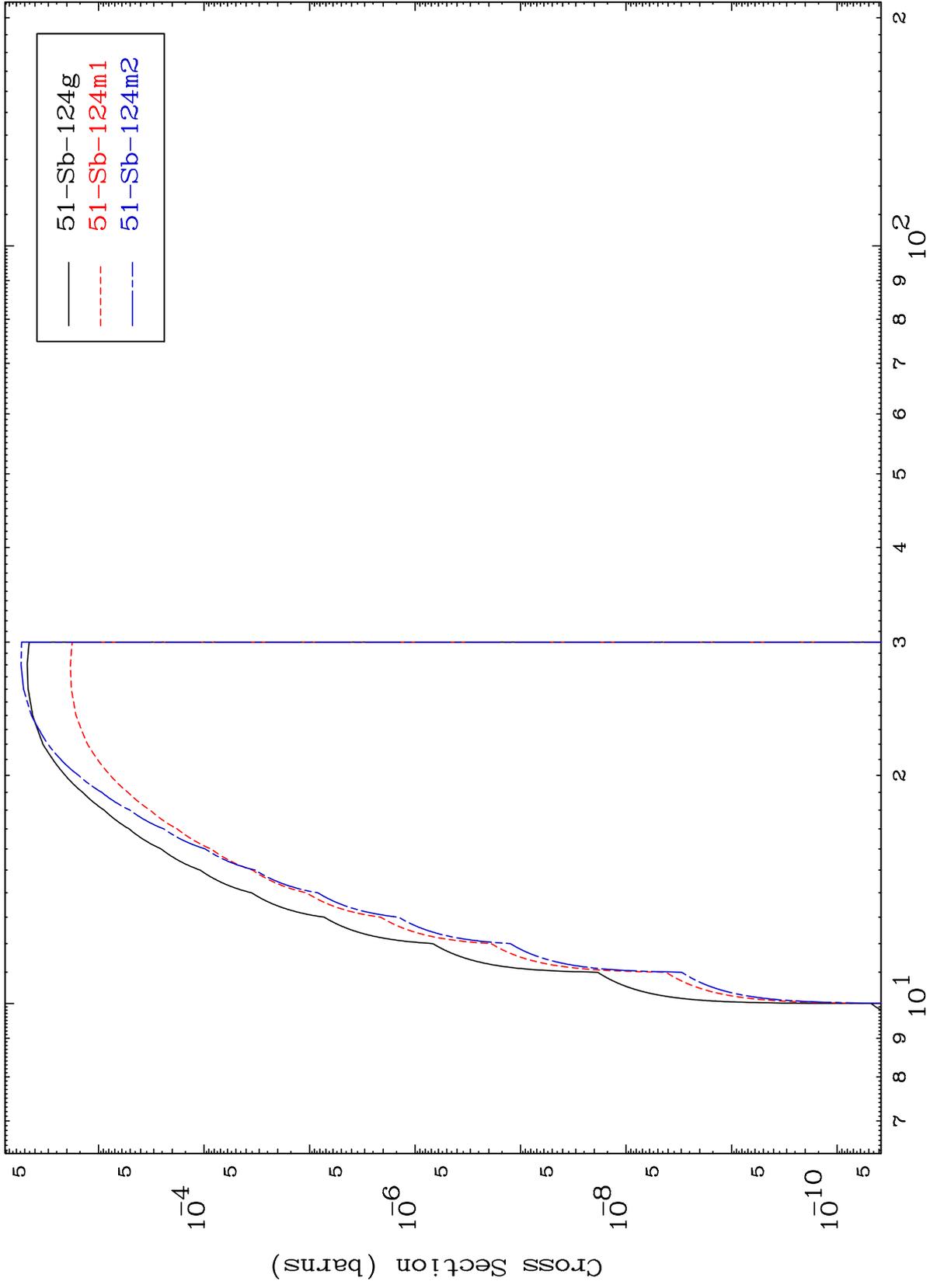
Incident Energy (MeV)

16

MAT 5052

50-Sn-121

(n,p)  
Radionuclide Production Cross Section



17

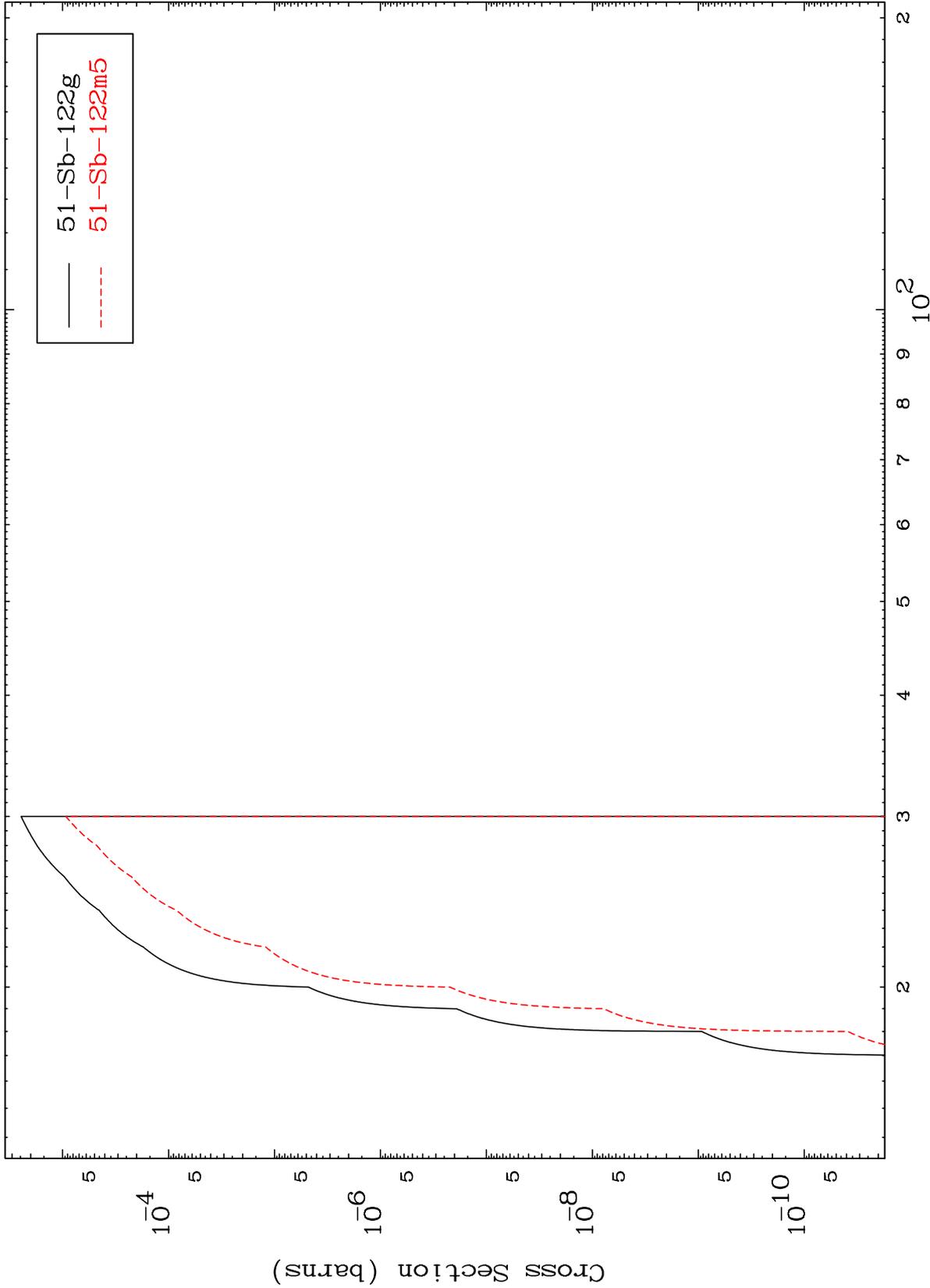
Incident Energy (MeV)

50-Sn-121

MAT 5052

50-Sn-121

(n,t)  
Radionuclide Production Cross Section



18

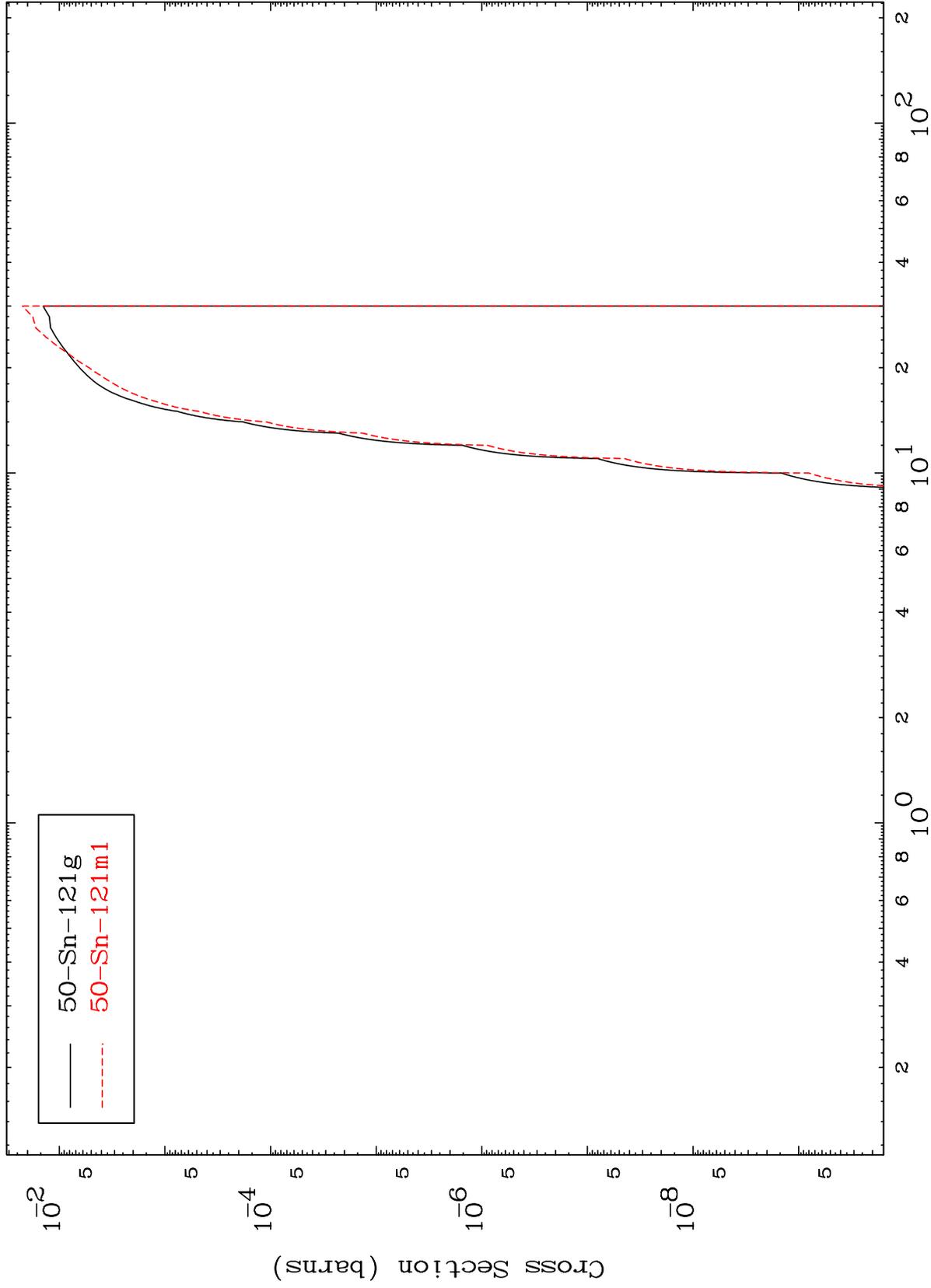
50-Sn-121

Incident Energy (MeV)

MAT 5052

50-Sn-121

Radionuclide Production Cross Section  
(n,  $\alpha$ )



50-Sn-121

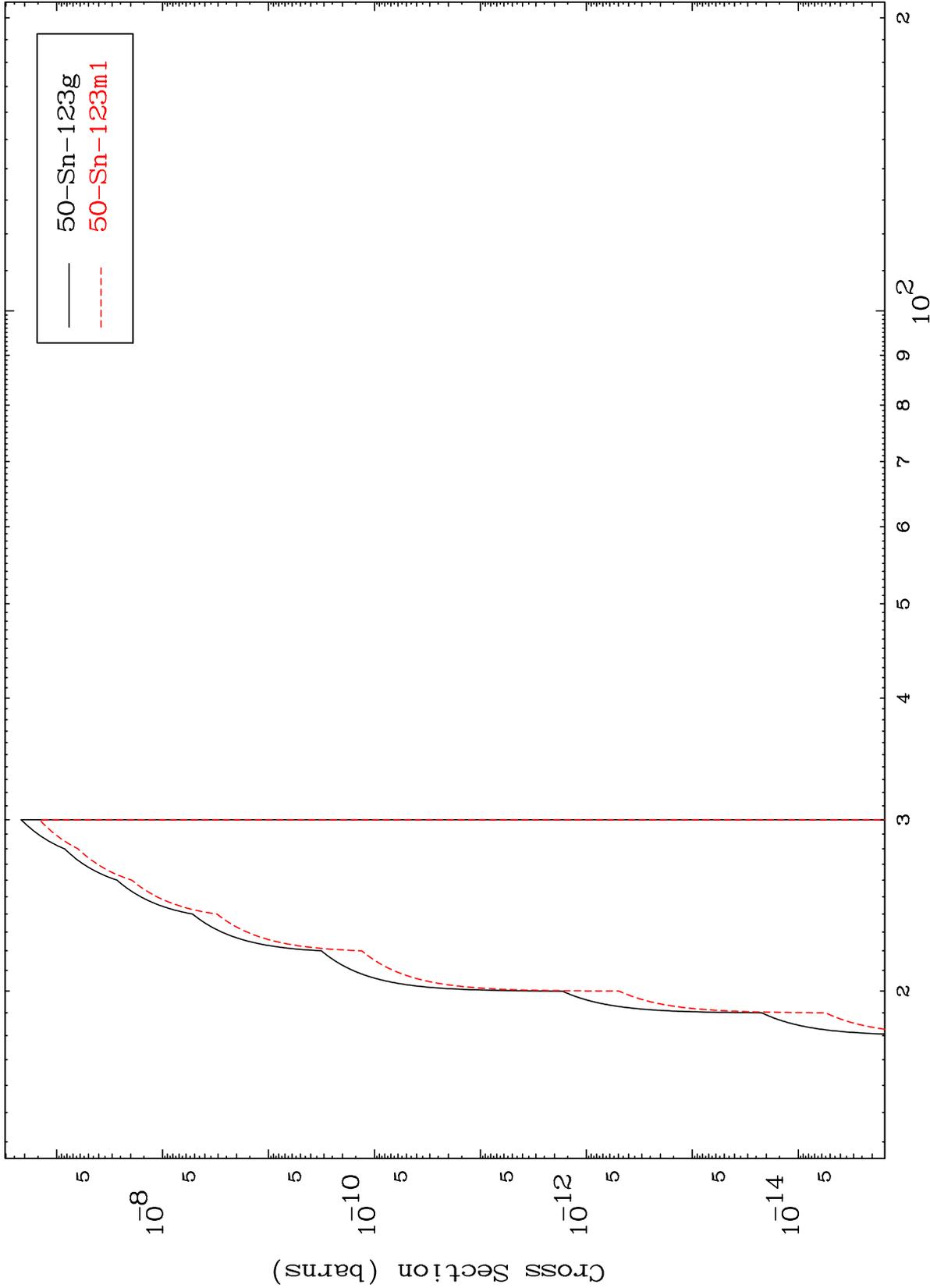
Incident Energy (MeV)

19

MAT 5052

50-Sn-121

(n,2p)  
Radionuclide Production Cross Section



20

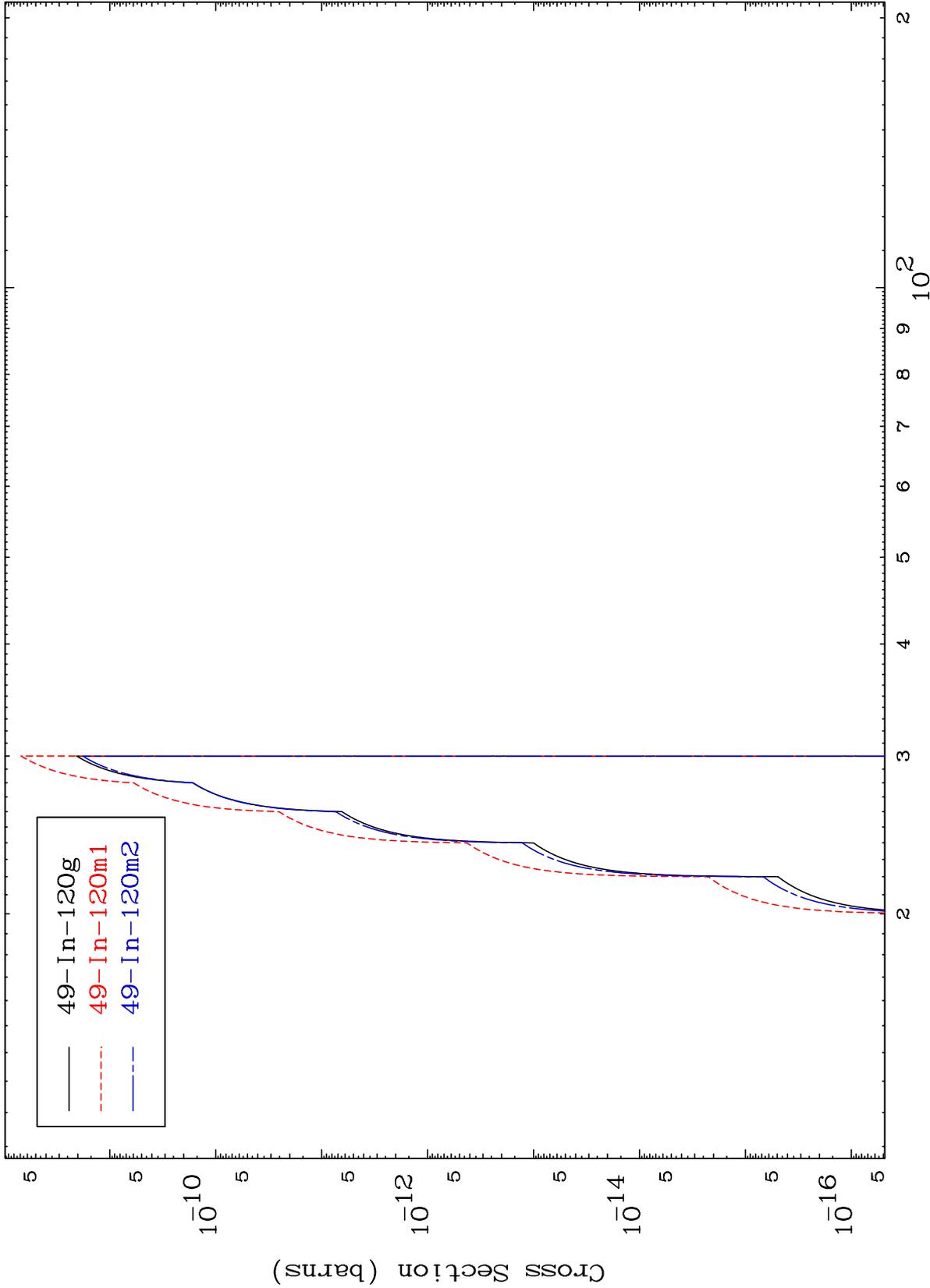
Incident Energy (MeV)

50-Sn-121

MAT 5052

50-Sn-121

(n,p)  $\alpha$   
Radionuclide Production Cross Section



50-Sn-121

Incident Energy (MeV)

21