

Program EVALPLOT
(Version 2021-1)

by

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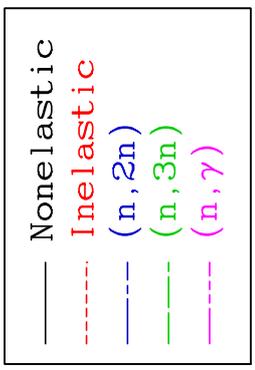
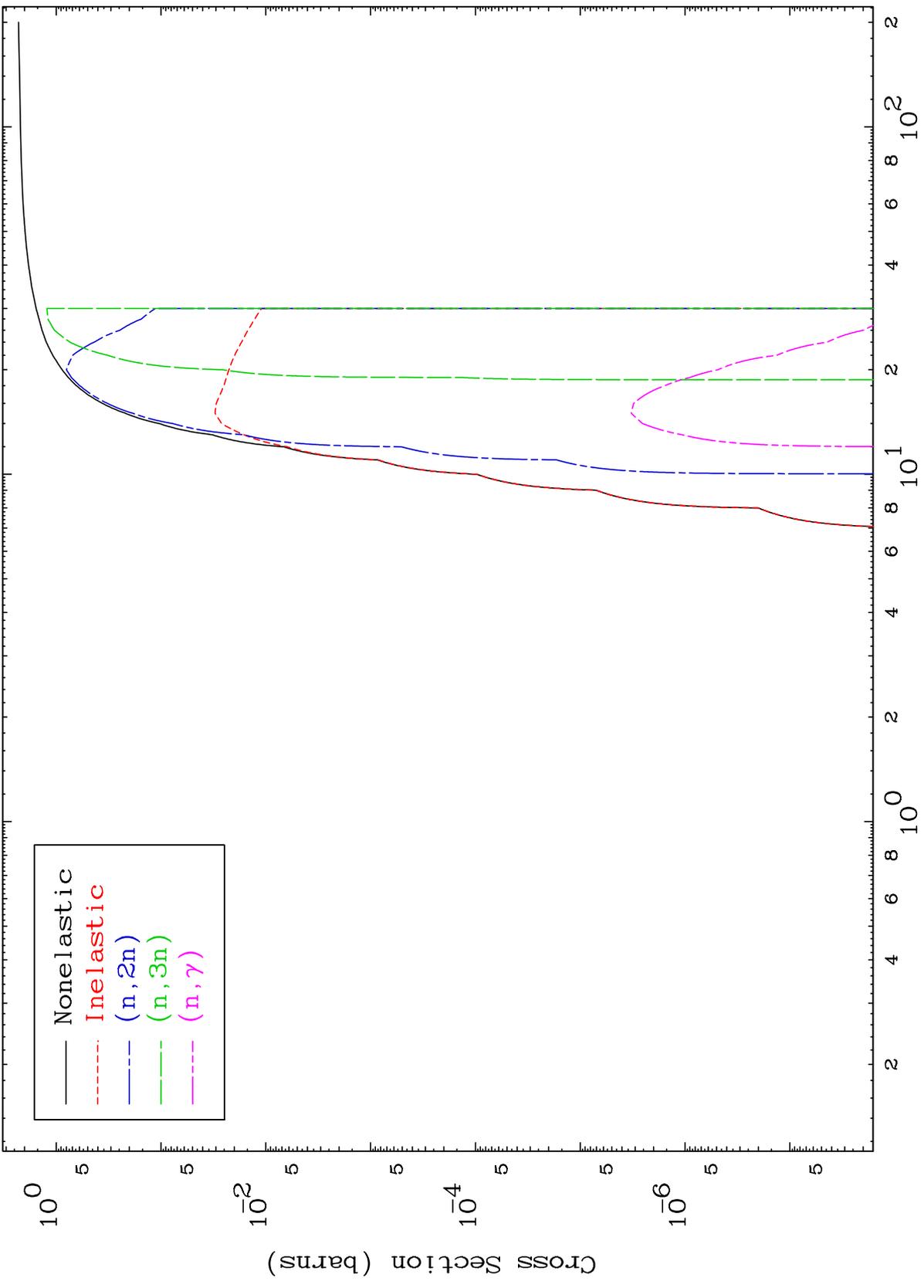
Press Mouse Button to Start

MAT 5073

α Major

50-Sn-128

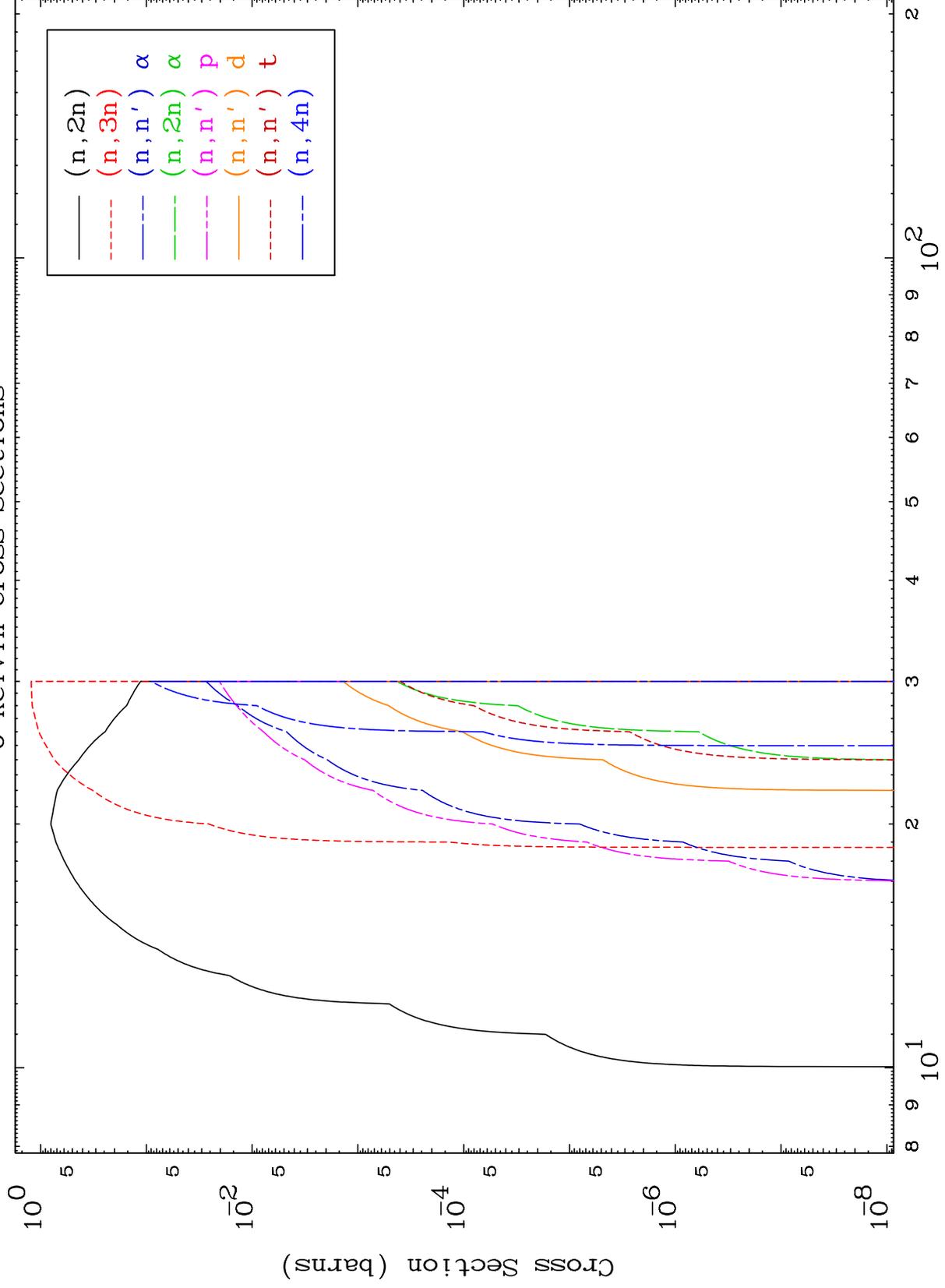
0 Kelvin Cross Sections



MAT 5073

α Neutron Absorption
0 Kelvin Cross Sections

50-Sn-128



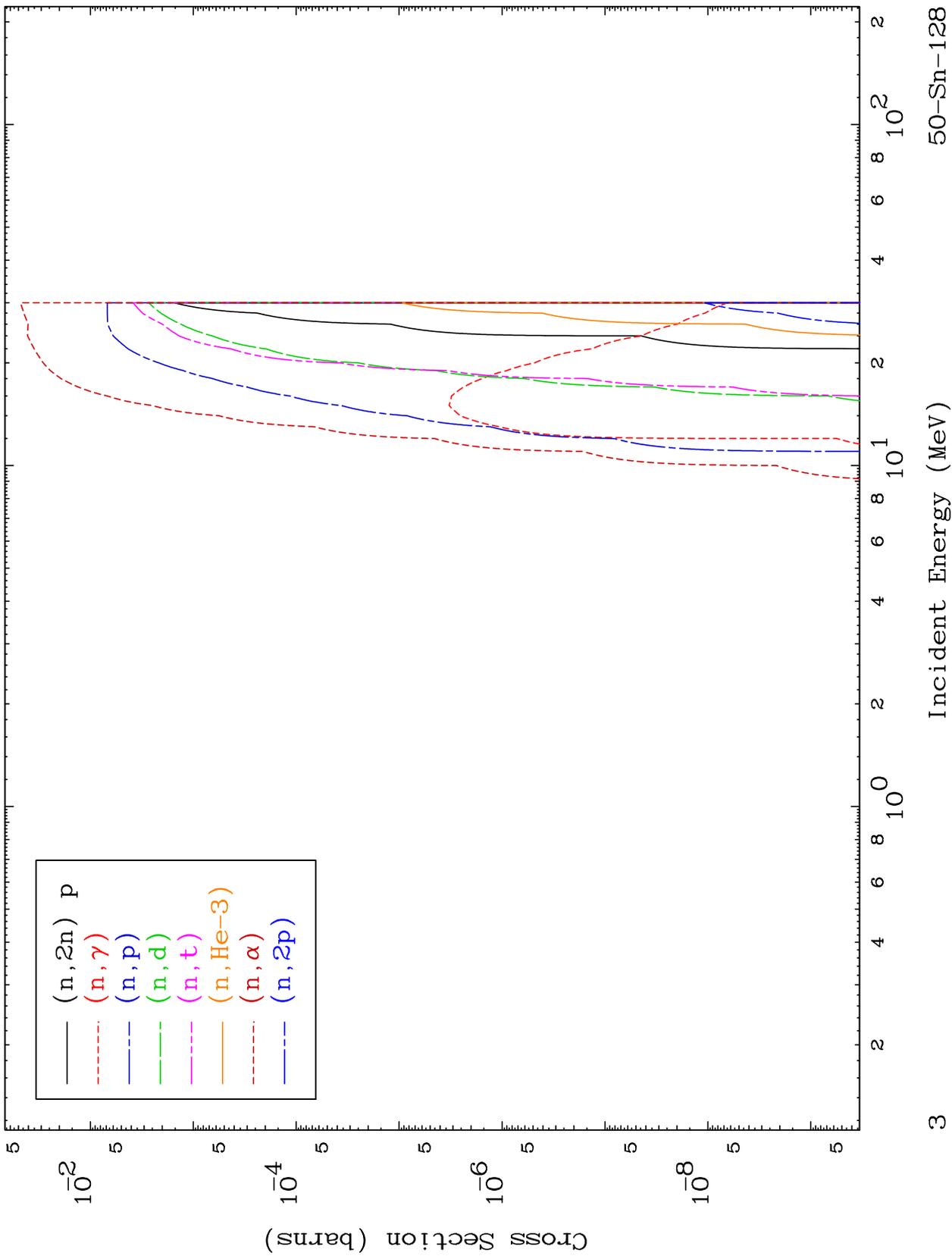
Incident Energy (MeV)

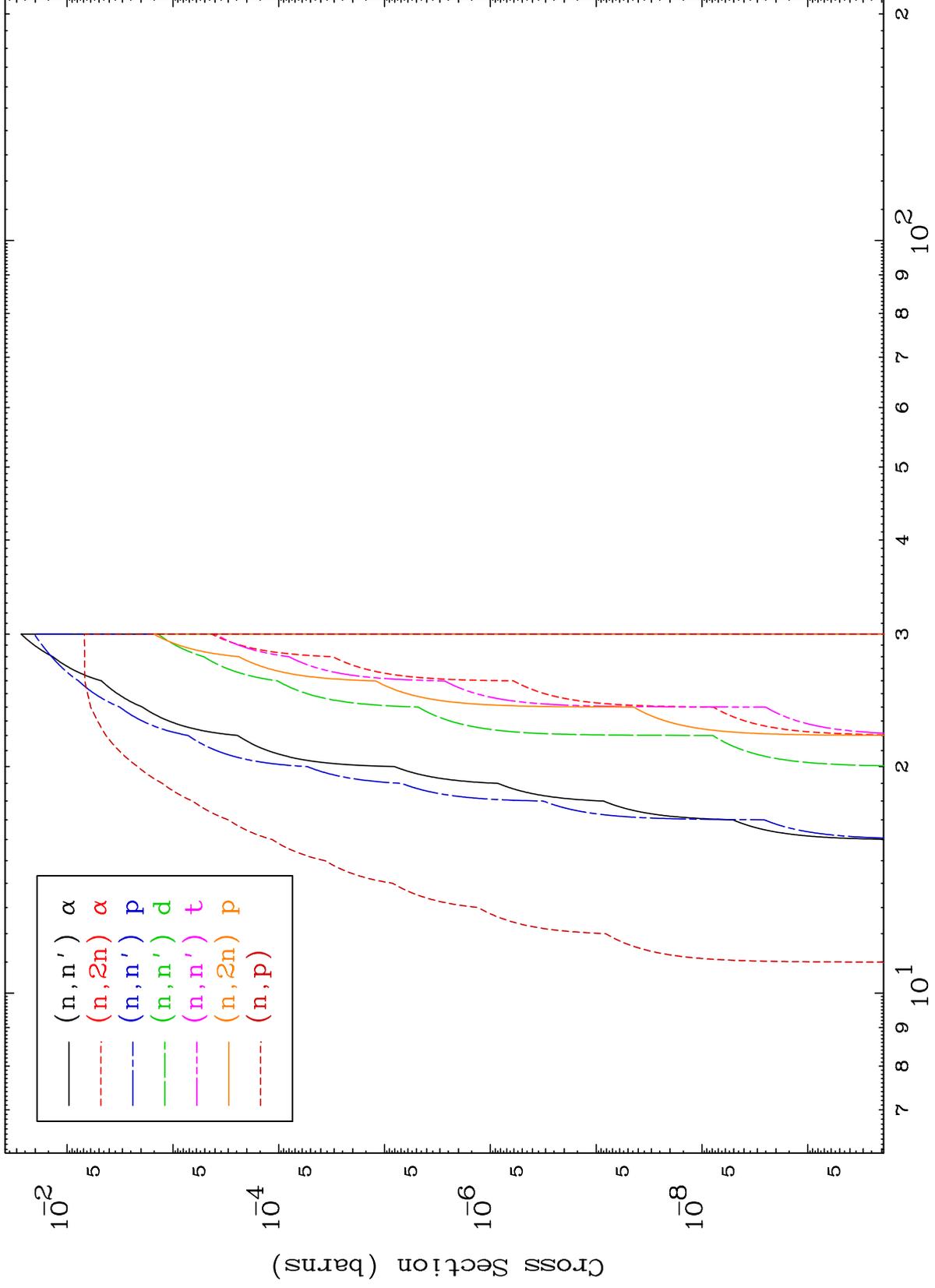
50-Sn-128

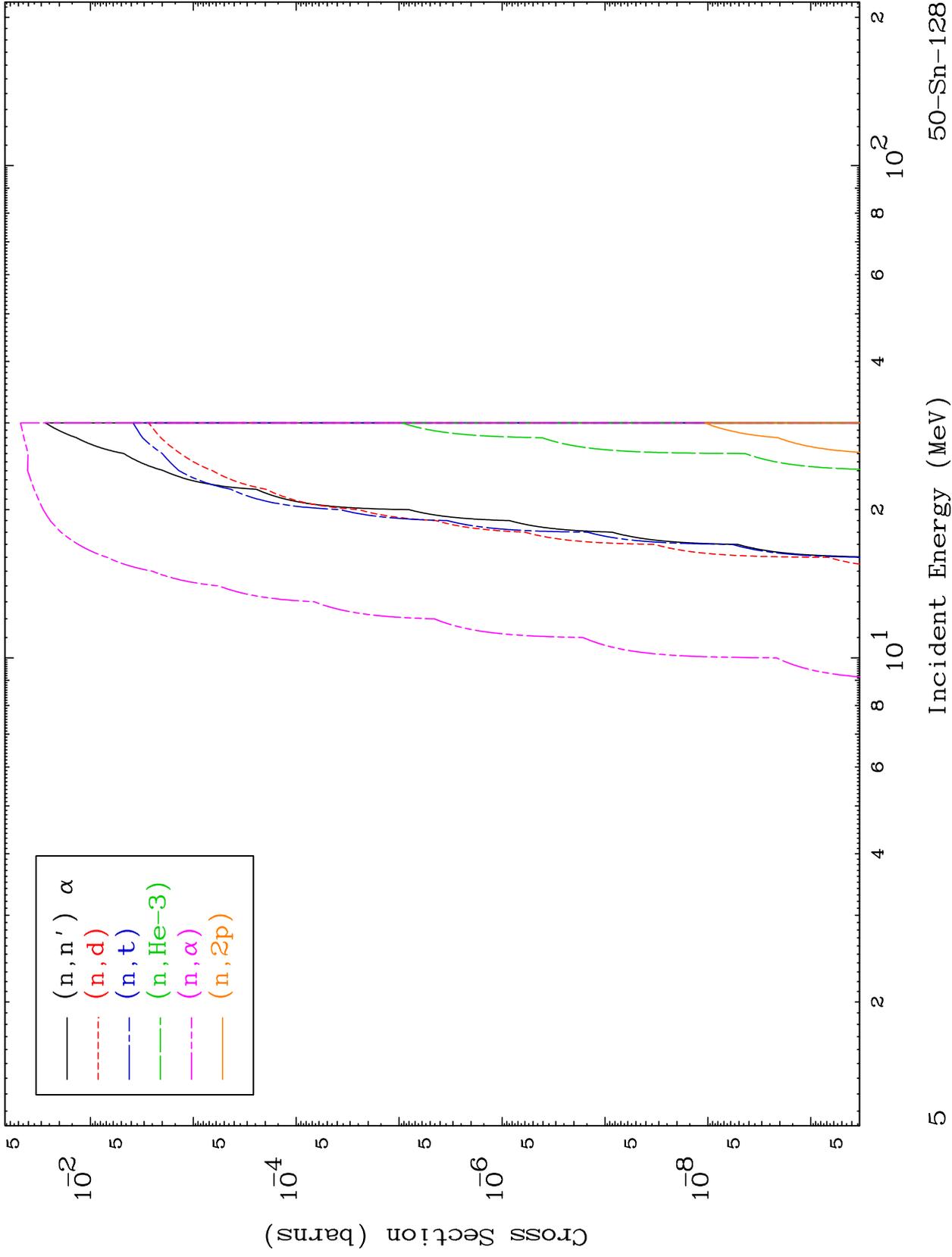
MAT 5073

α Neutron Absorption
0 Kelvin Cross Sections

50-Sn-128



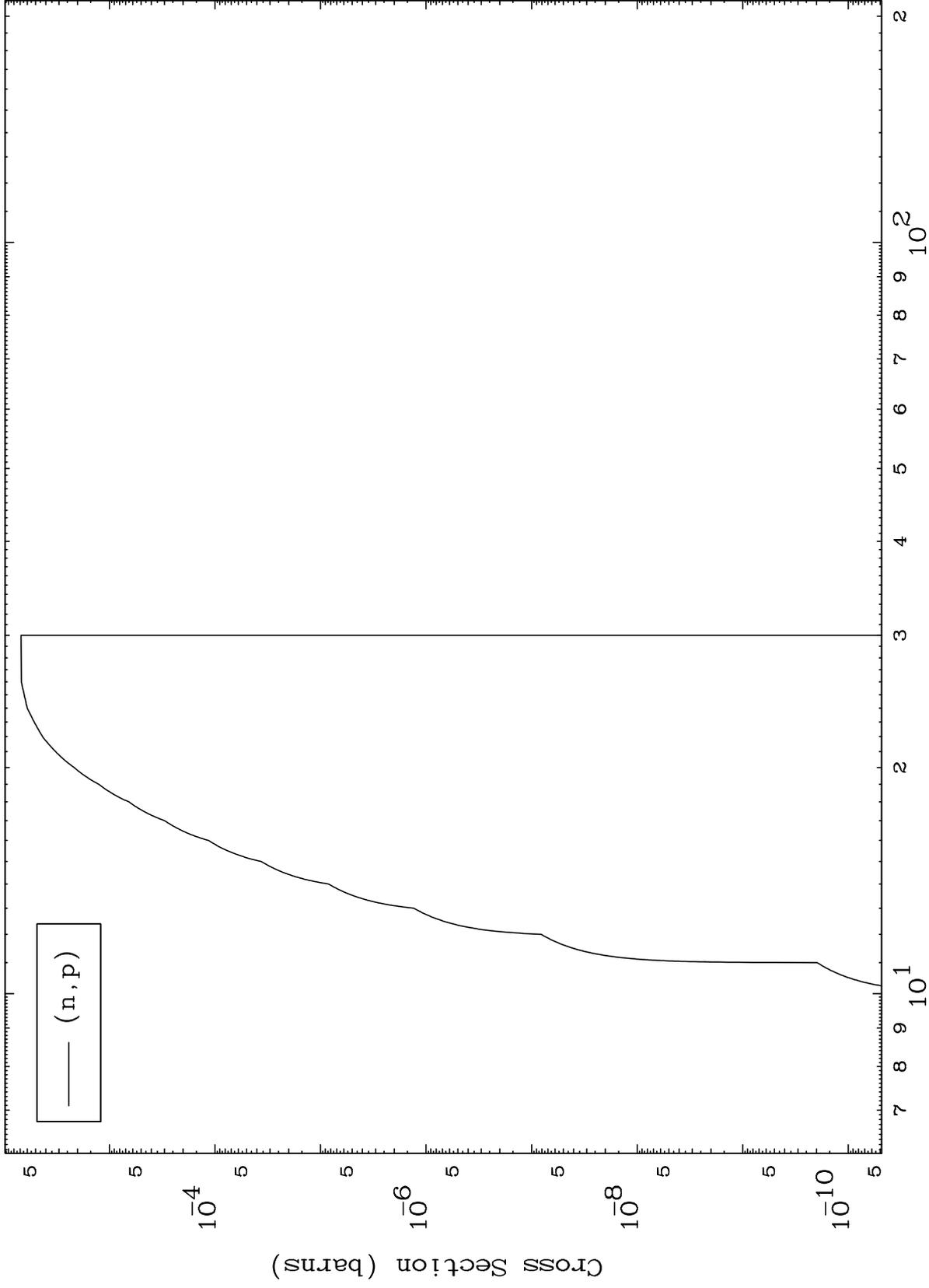




MAT 5073

(α, p) Levels
0 Kelvin Cross Sections

50-Sn-128



6

Incident Energy (MeV)

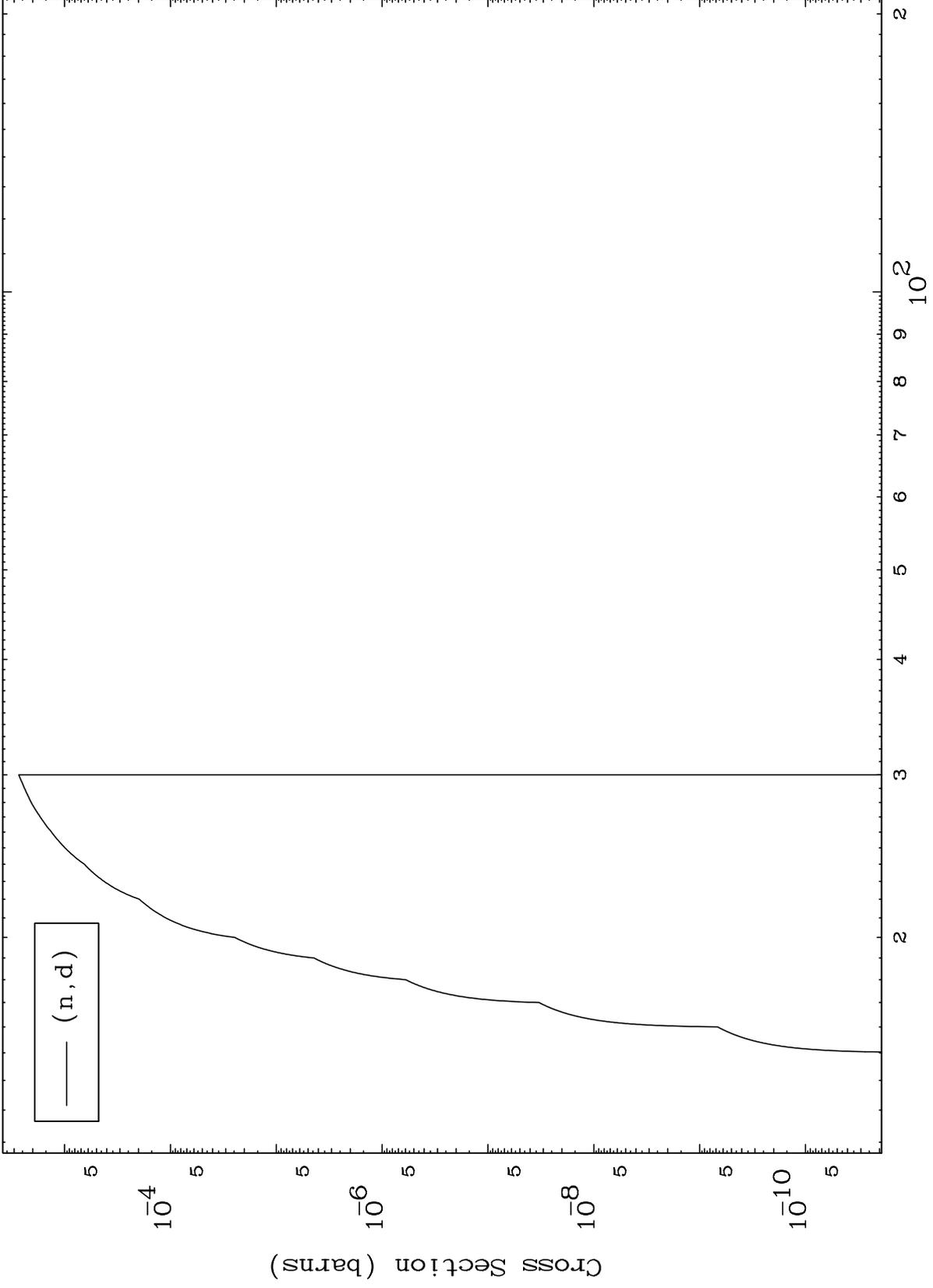
50-Sn-128

MAT 5073

(α, d) Levels

50-Sn-128

0 Kelvin Cross Sections



7

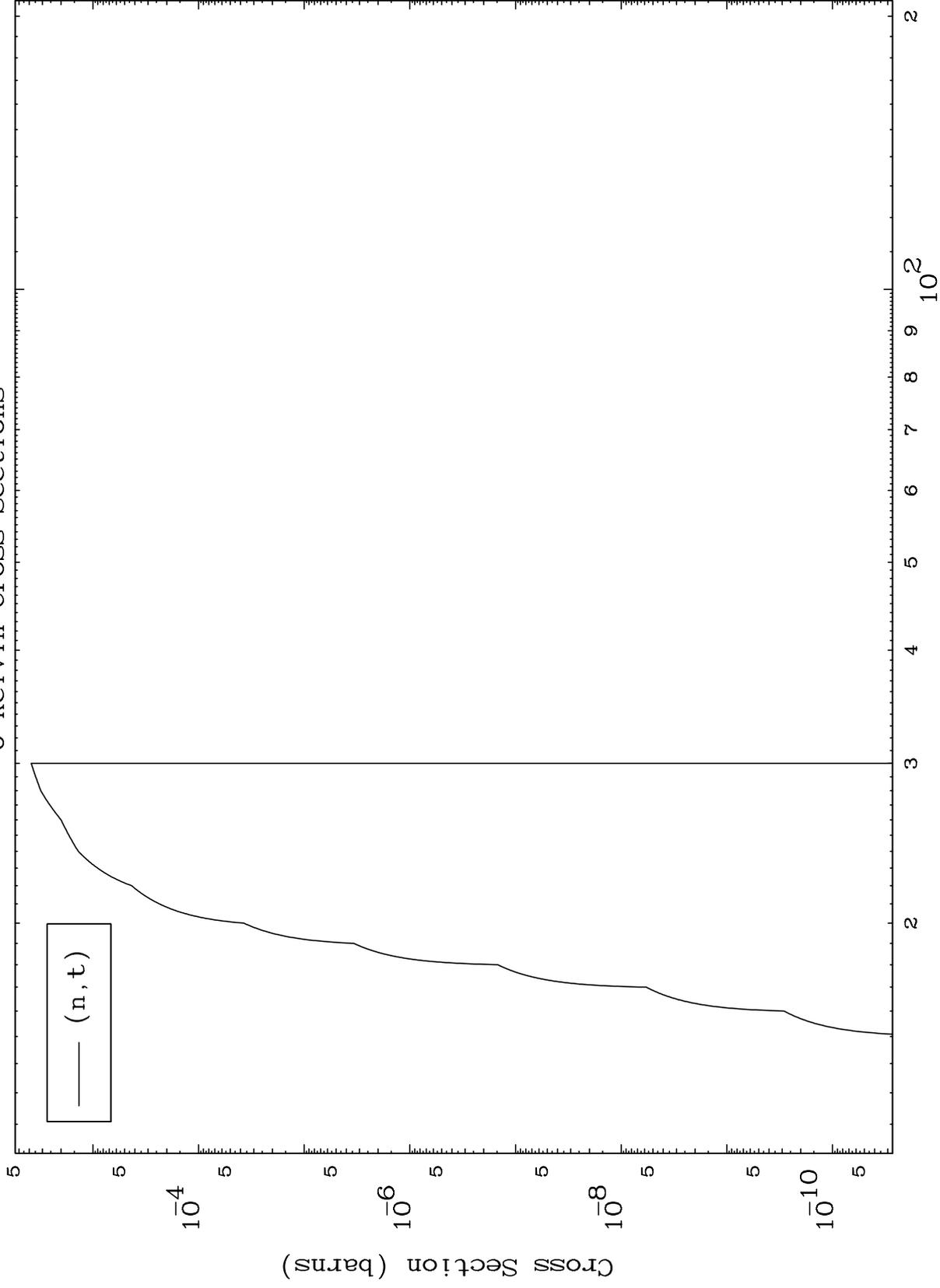
Incident Energy (MeV)

50-Sn-128

MAT 5073

(α, t) Levels
0 Kelvin Cross Sections

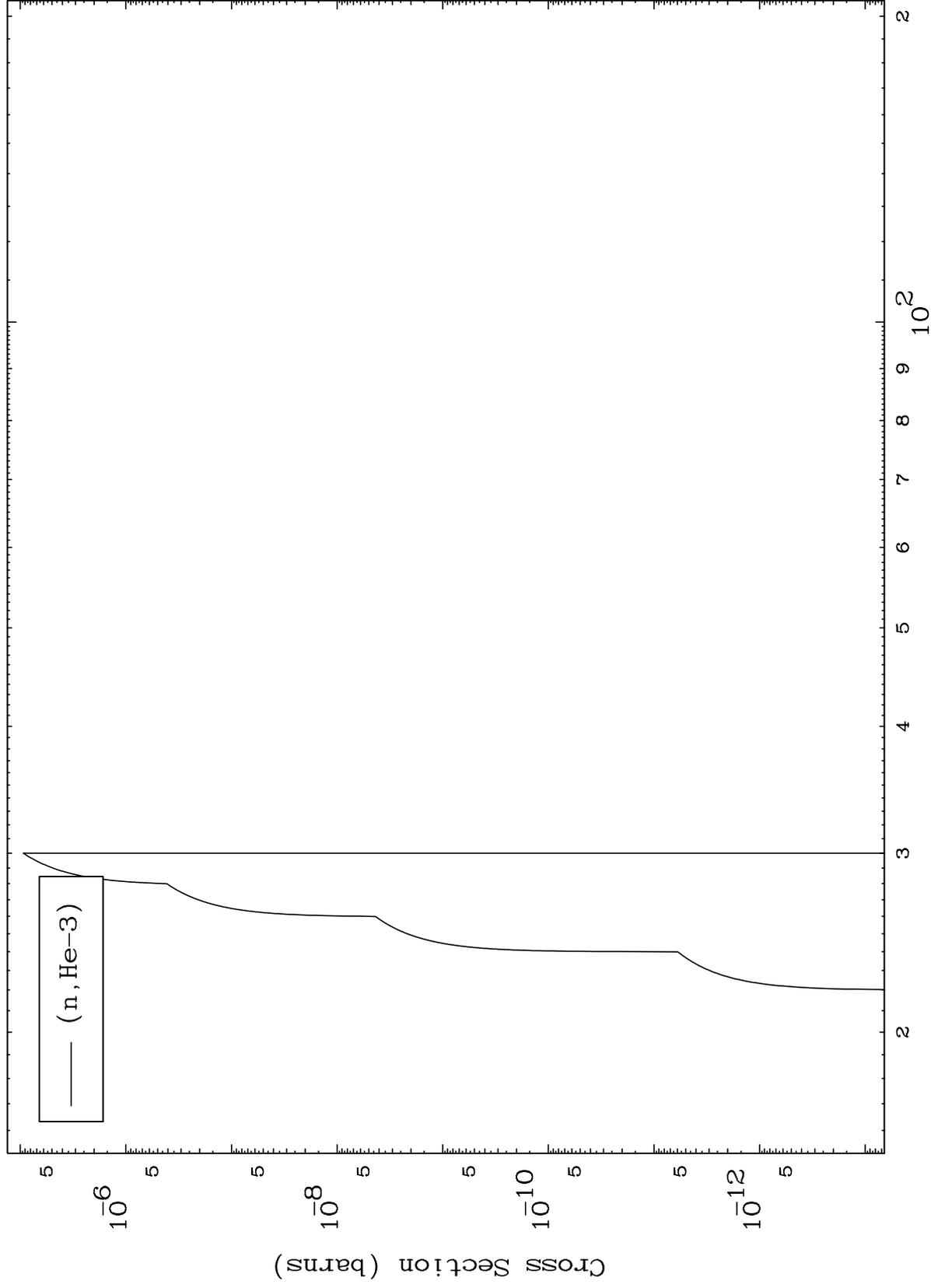
50-Sn-128



MAT 5073

($\alpha, \text{He}3$) Levels
0 Kelvin Cross Sections

50-Sn-128



9

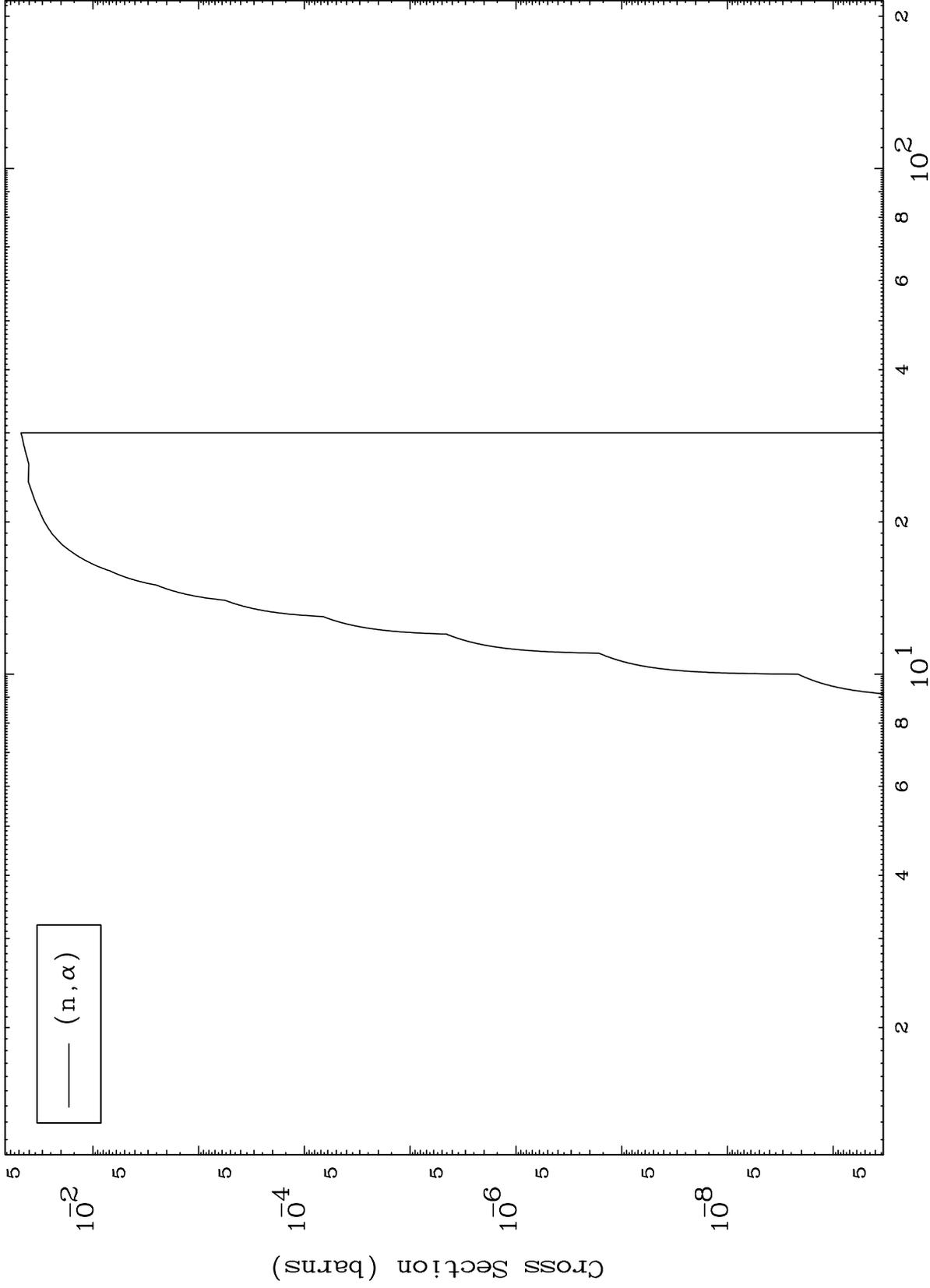
Incident Energy (MeV)

50-Sn-128

MAT 5073

(α, α) Levels
0 Kelvin Cross Sections

50-Sn-128



10

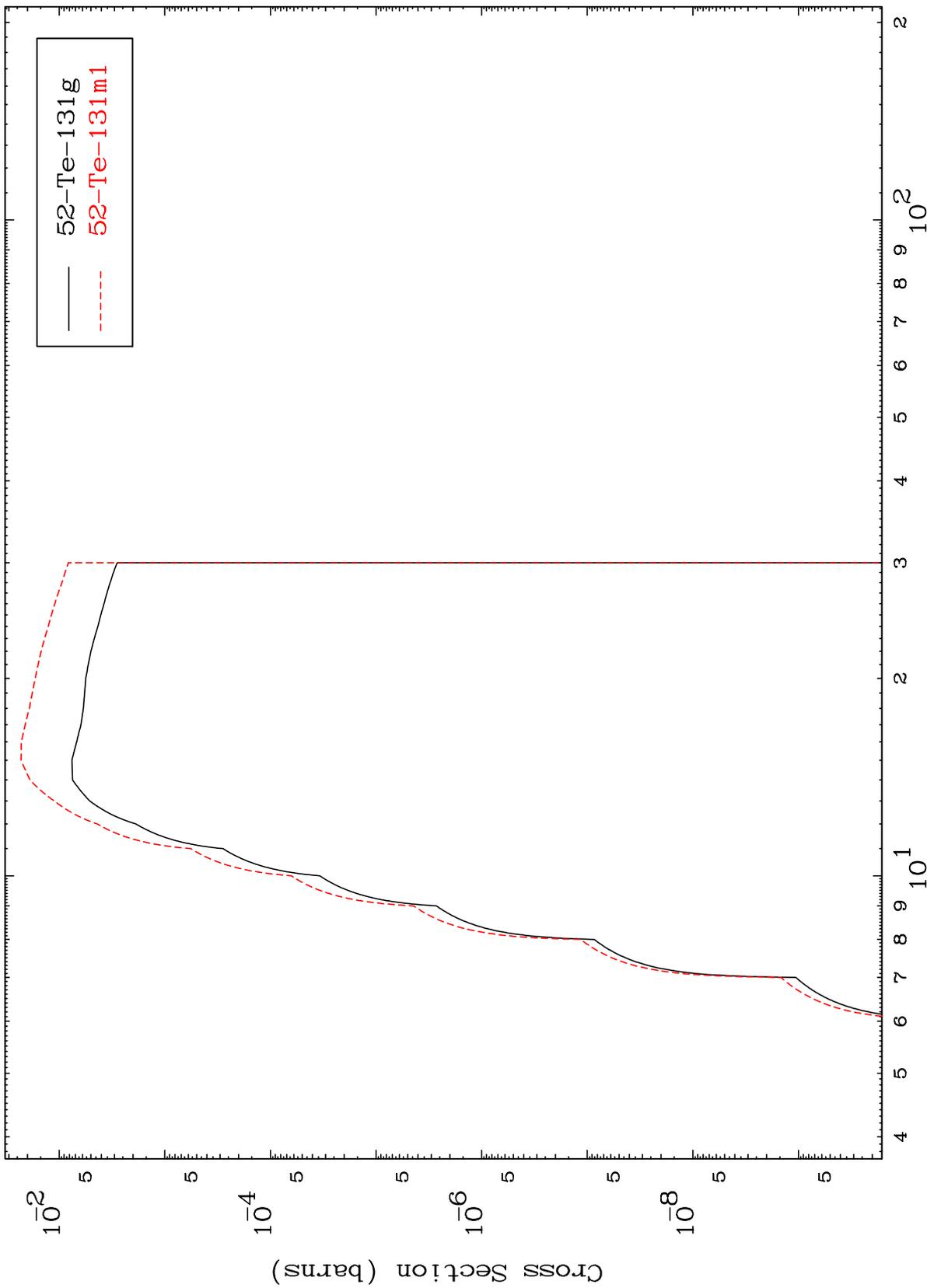
Incident Energy (MeV)

50-Sn-128

MAT 5073

50-Sn-128

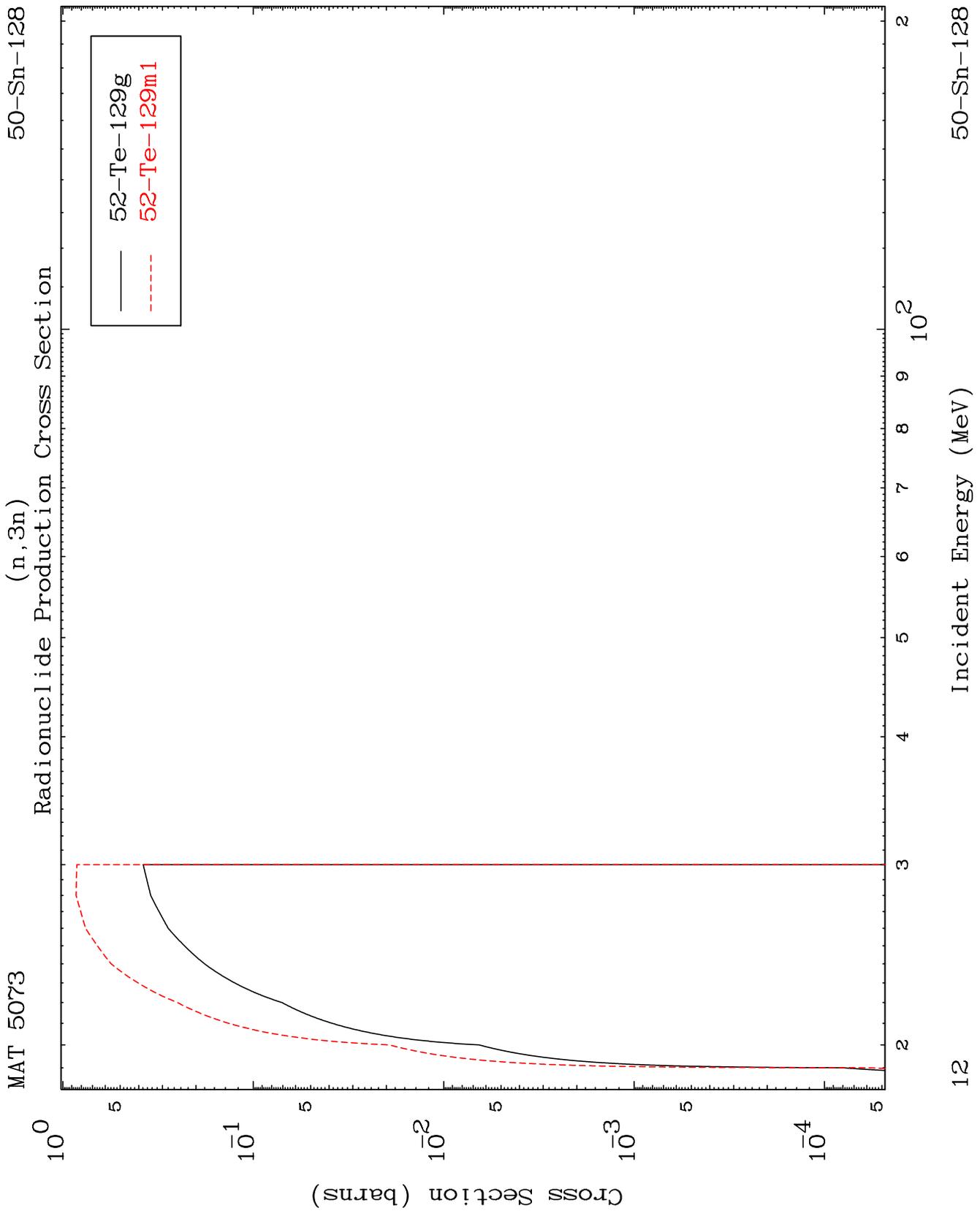
Inelastic
Radionuclide Production Cross Section



11

Incident Energy (MeV)

50-Sn-128

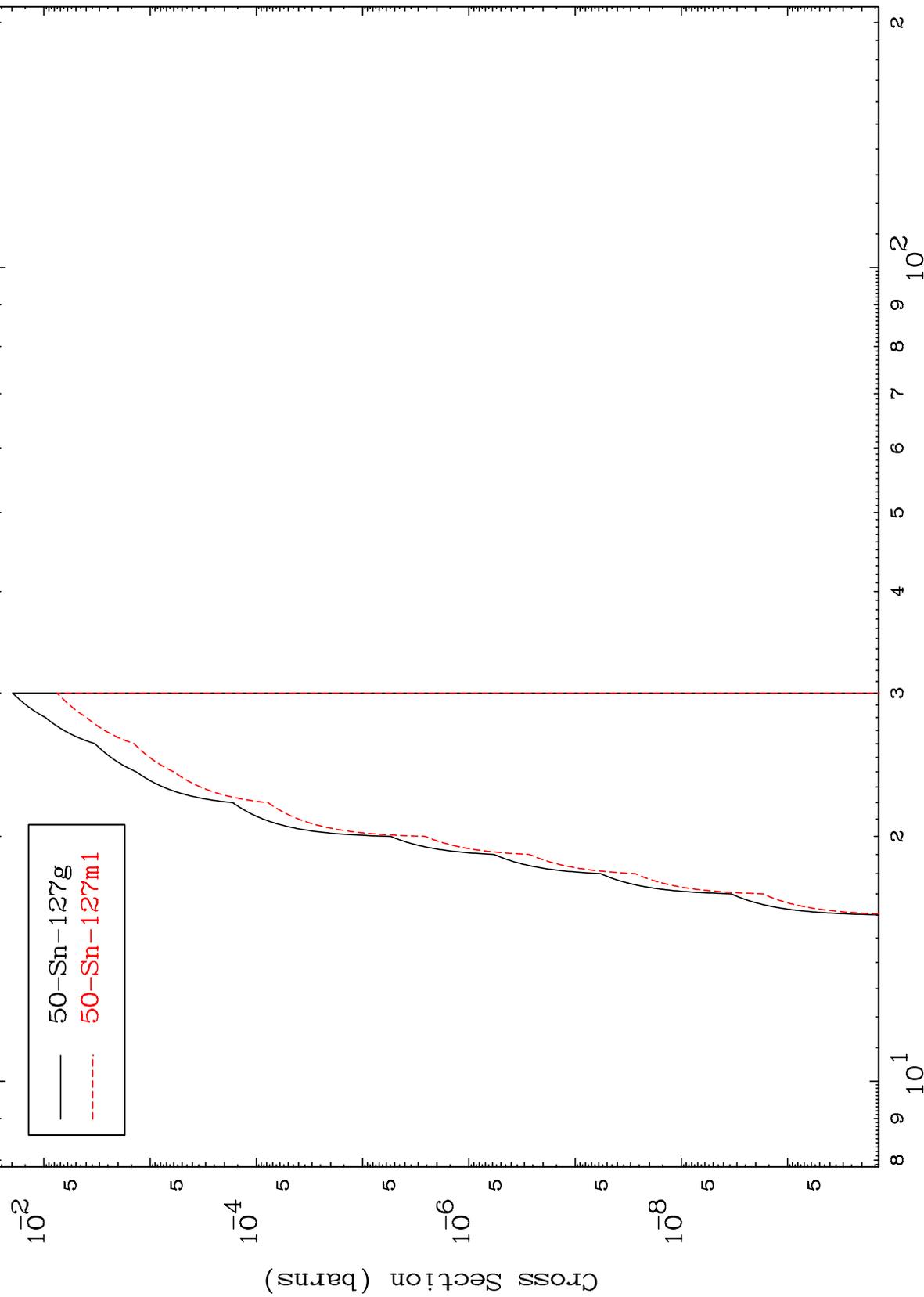


MAT 5073

(n,n') α

50-Sn-128

Radionuclide Production Cross Section



13

Incident Energy (MeV)

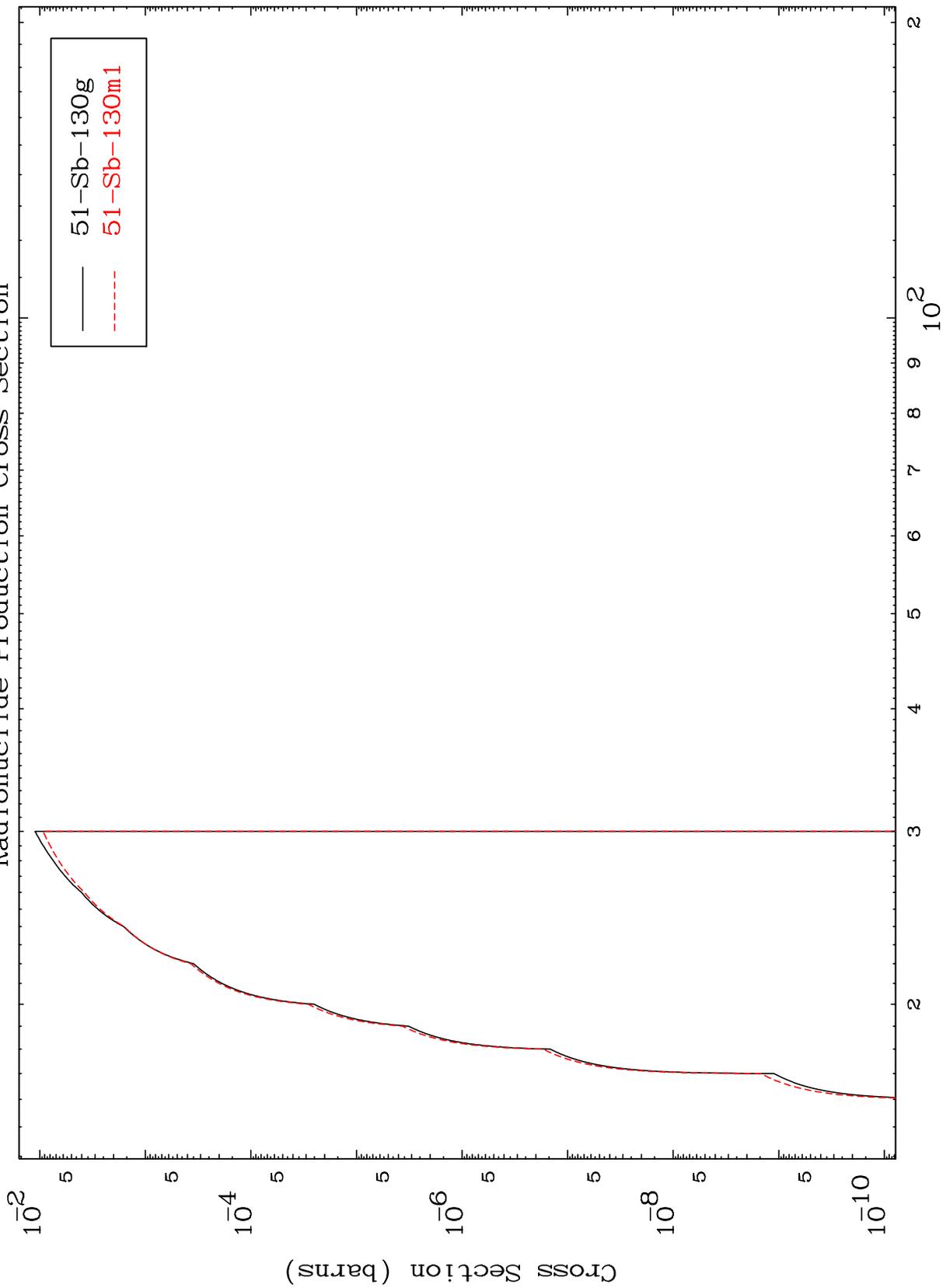
50-Sn-128

MAT 5073

(n,n') p

50-Sn-128

Radionuclide Production Cross Section



14

Incident Energy (MeV)

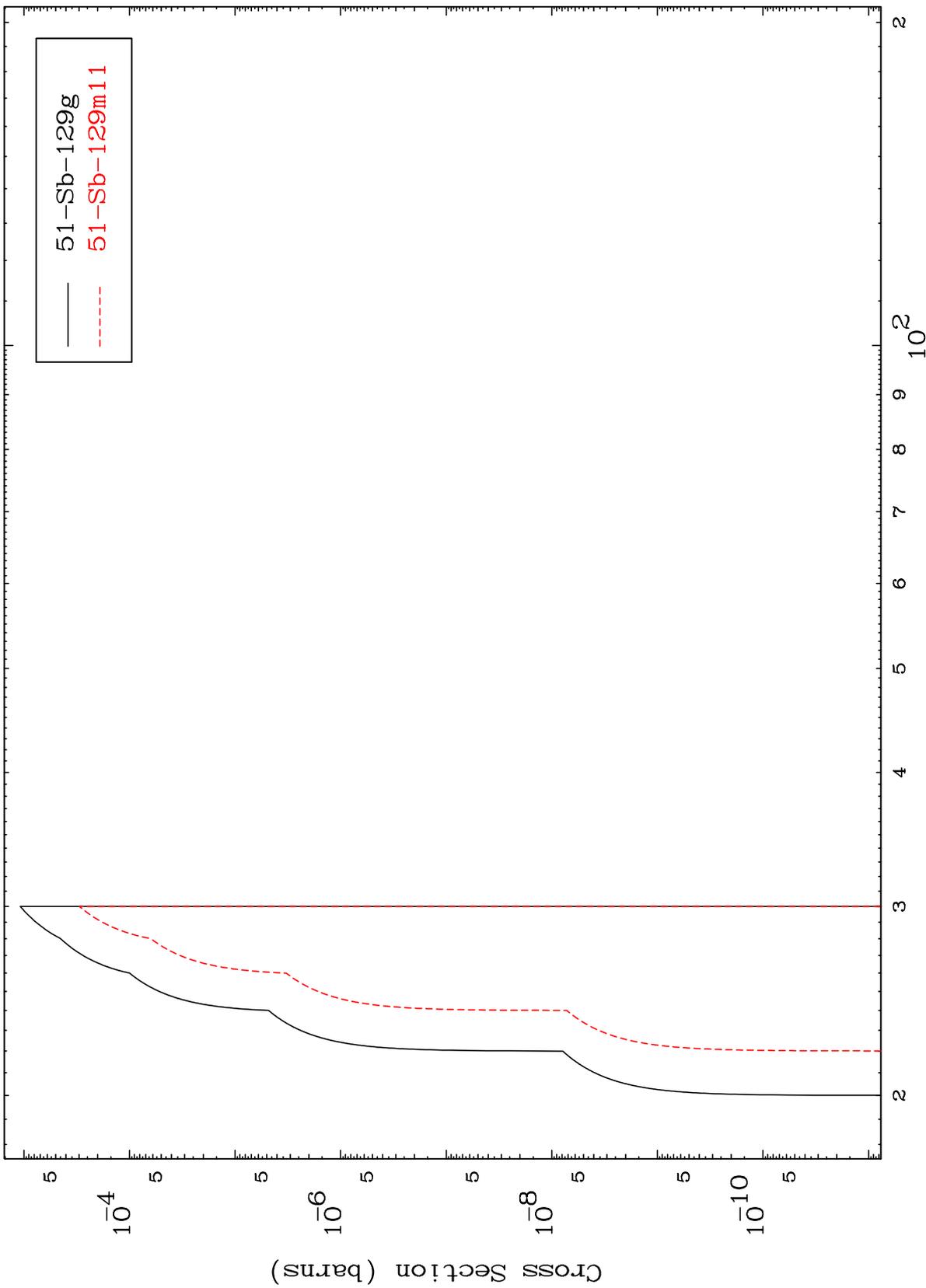
50-Sn-128

MAT 5073

(n,n') d

50-Sn-128

Radionuclide Production Cross Section



15

Incident Energy (MeV)

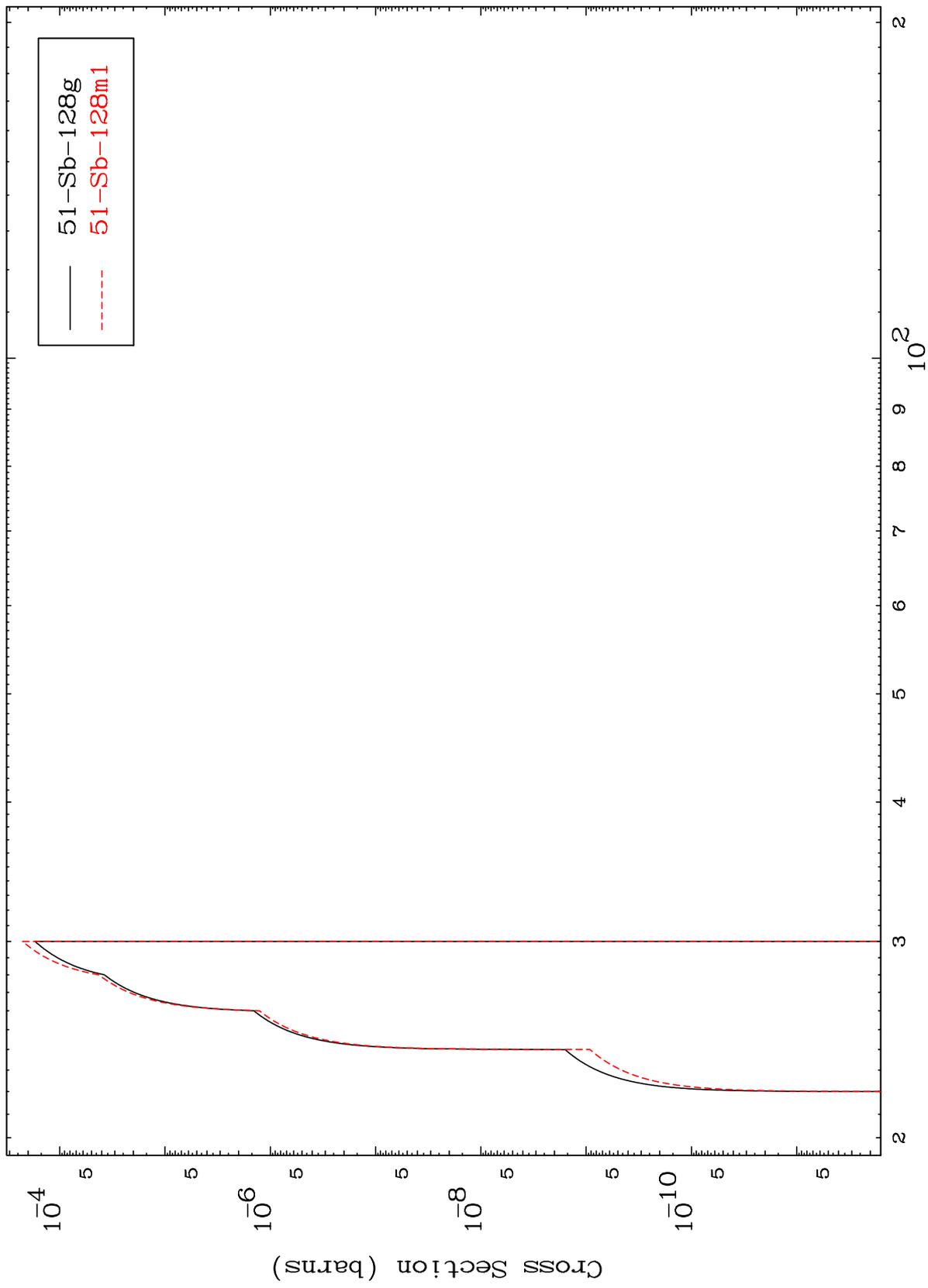
50-Sn-128

MAT 5073

(n,n') t

50-Sn-128

Radionuclide Production Cross Section



16

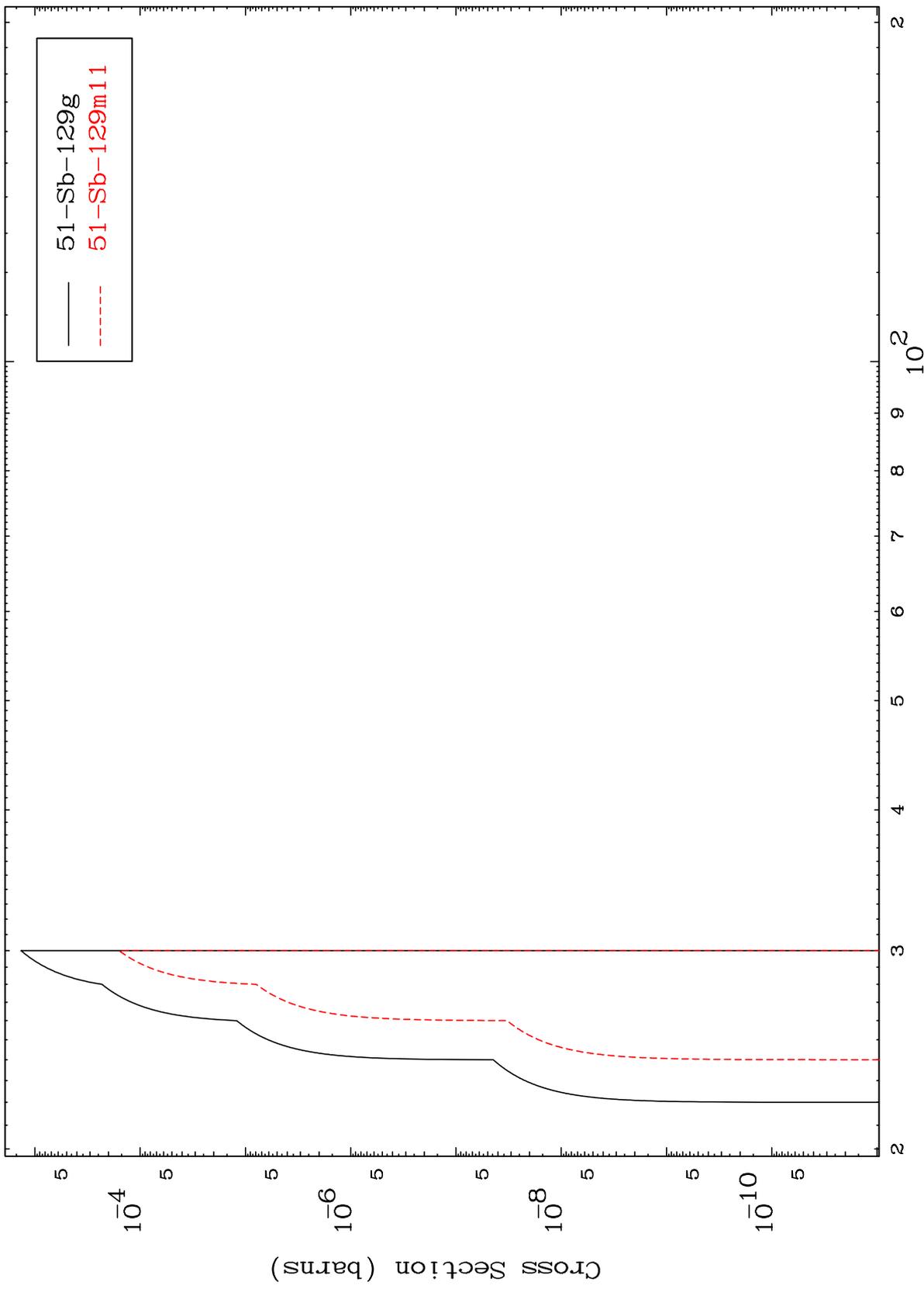
Incident Energy (MeV)

50-Sn-128

MAT 5073

50-Sn-128

(n,2n) p
Radionuclide Production Cross Section



50-Sn-128

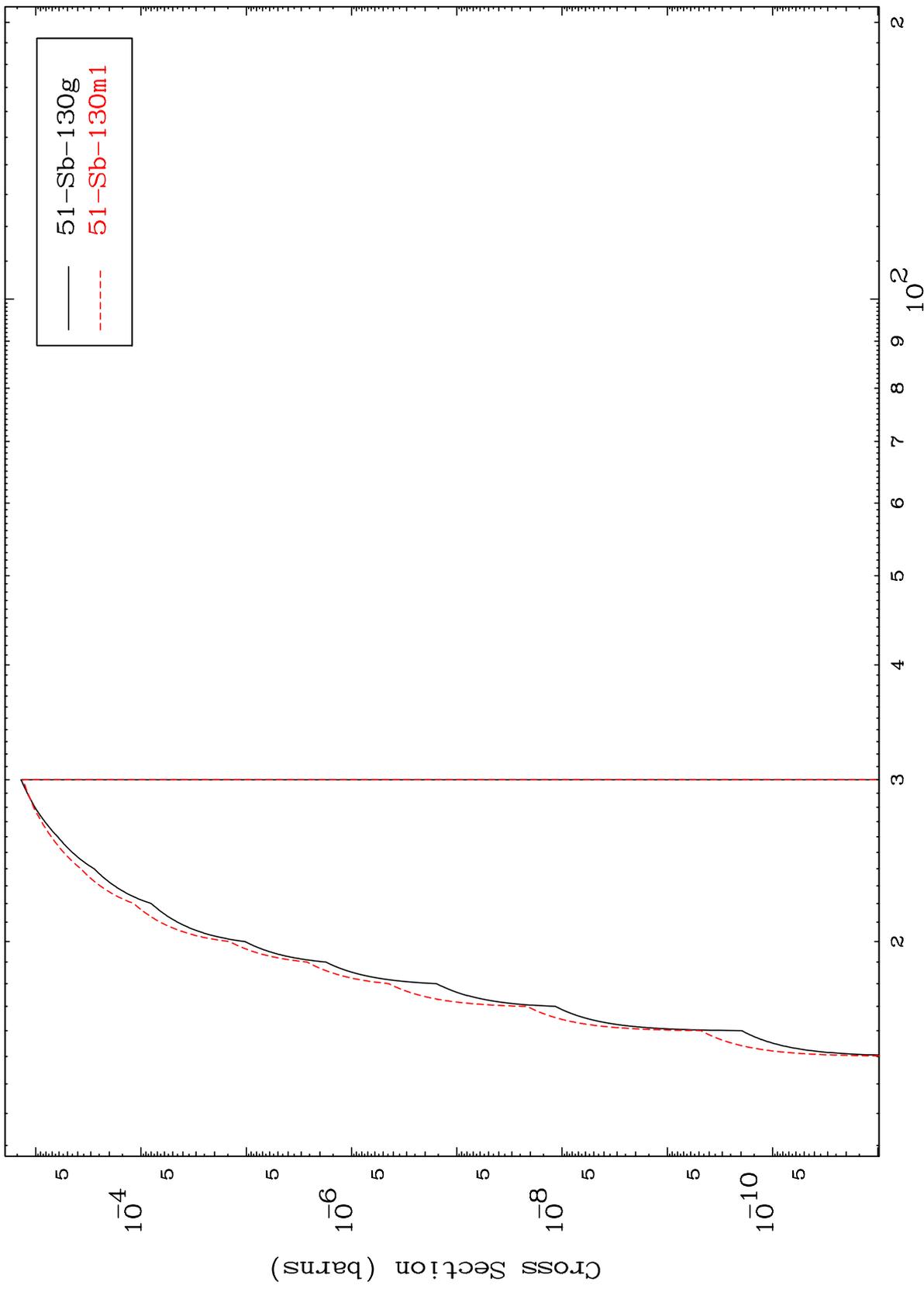
Incident Energy (MeV)

17

MAT 5073

50-Sn-128

(n,d)
Radionuclide Production Cross Section

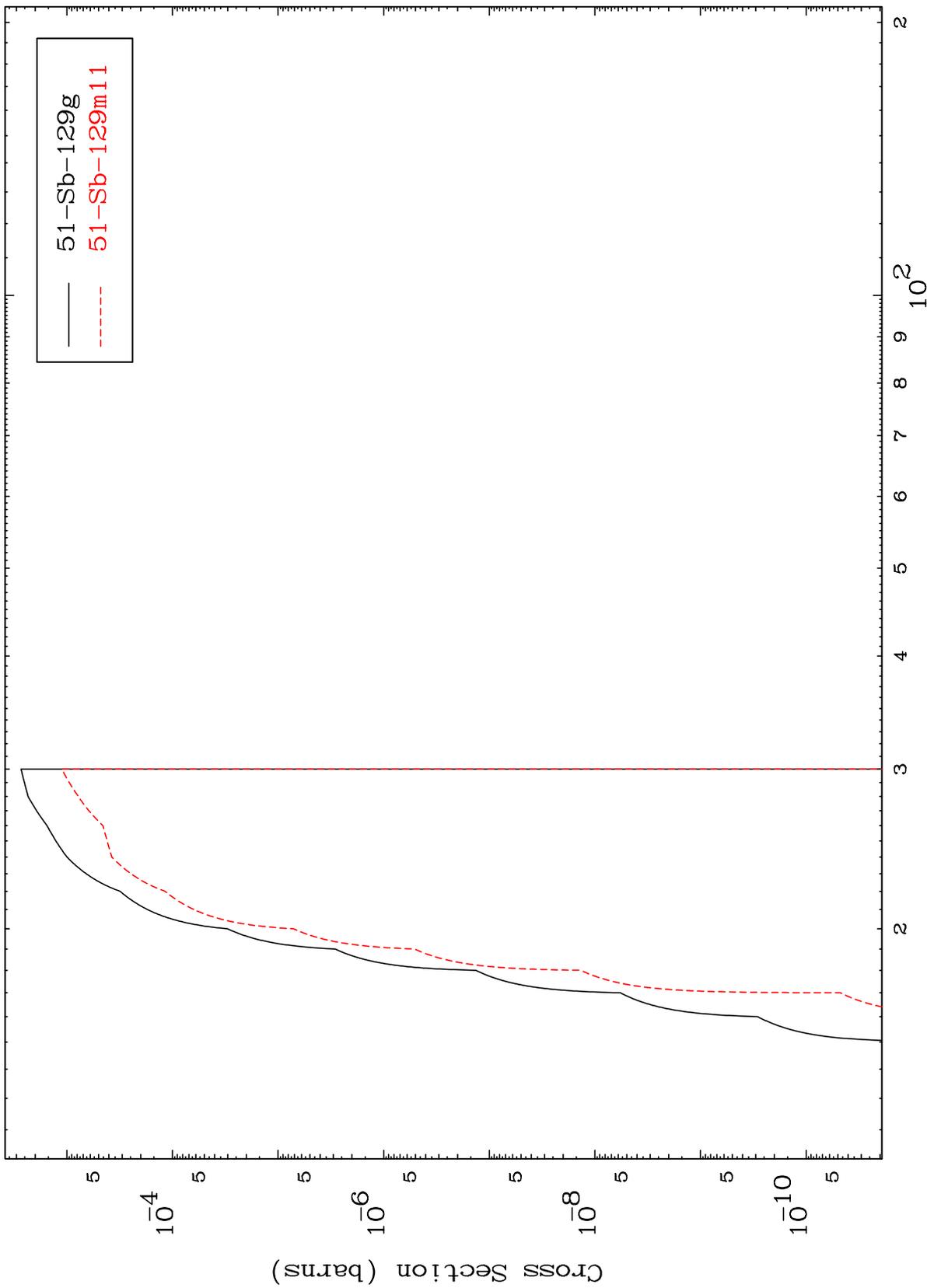


18

Incident Energy (MeV)

50-Sn-128

(n,t)
Radionuclide Production Cross Section



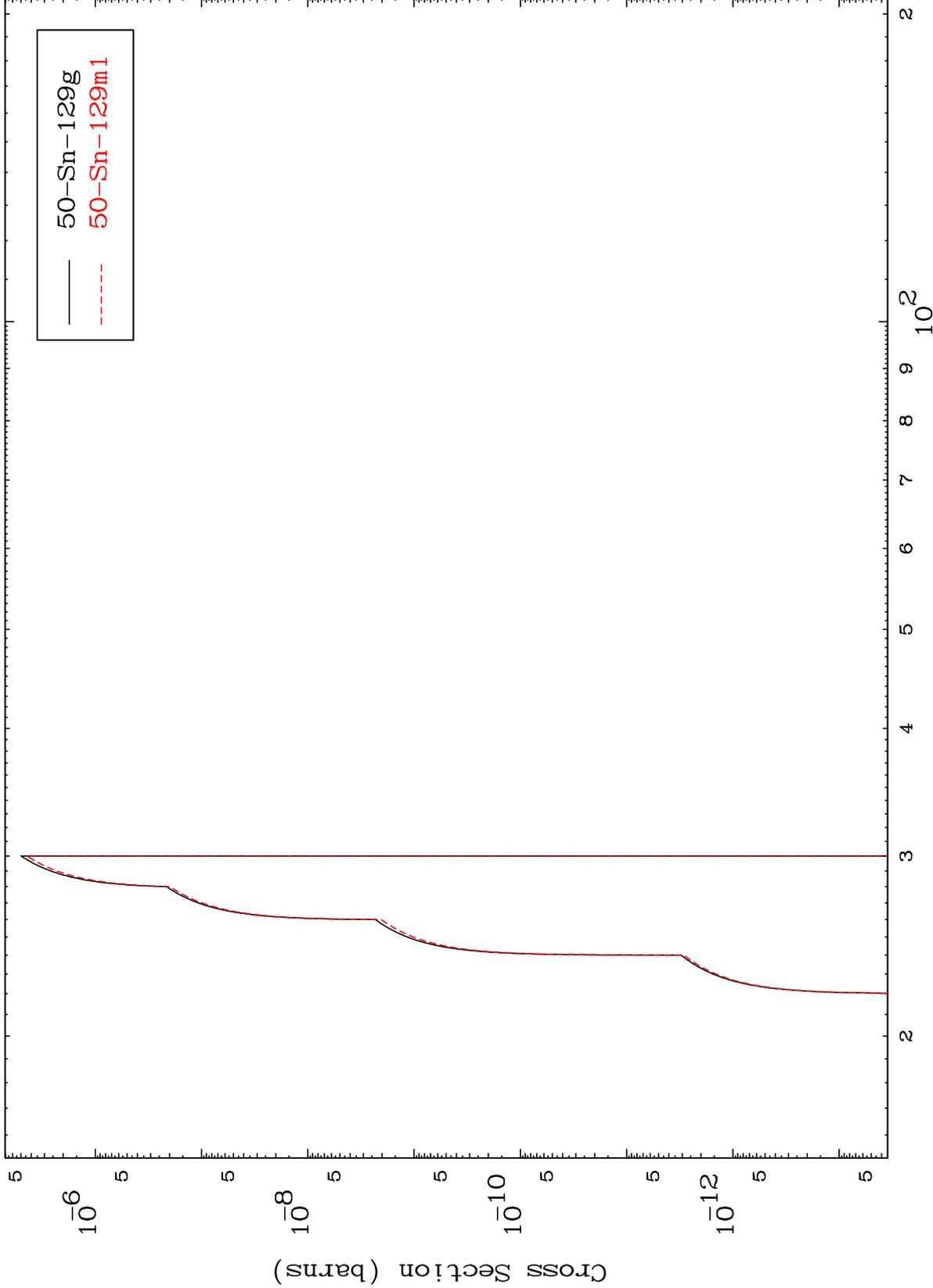
51-Sb-129g
51-Sb-129m11

MAT 5073

(n,He-3)

50-Sn-128

Radionuclide Production Cross Section



20

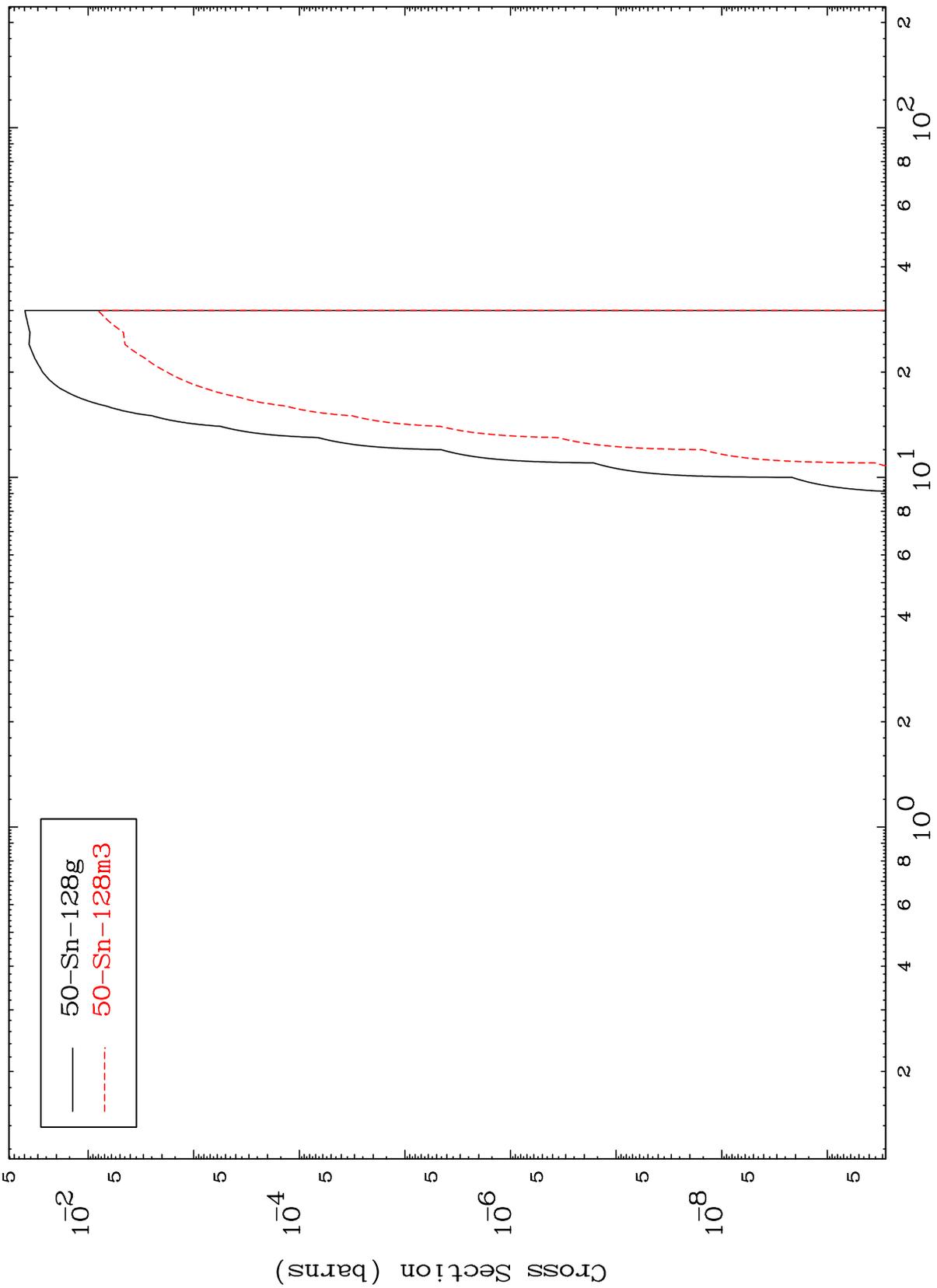
Incident Energy (MeV)

50-Sn-128

MAT 5073

50-Sn-128

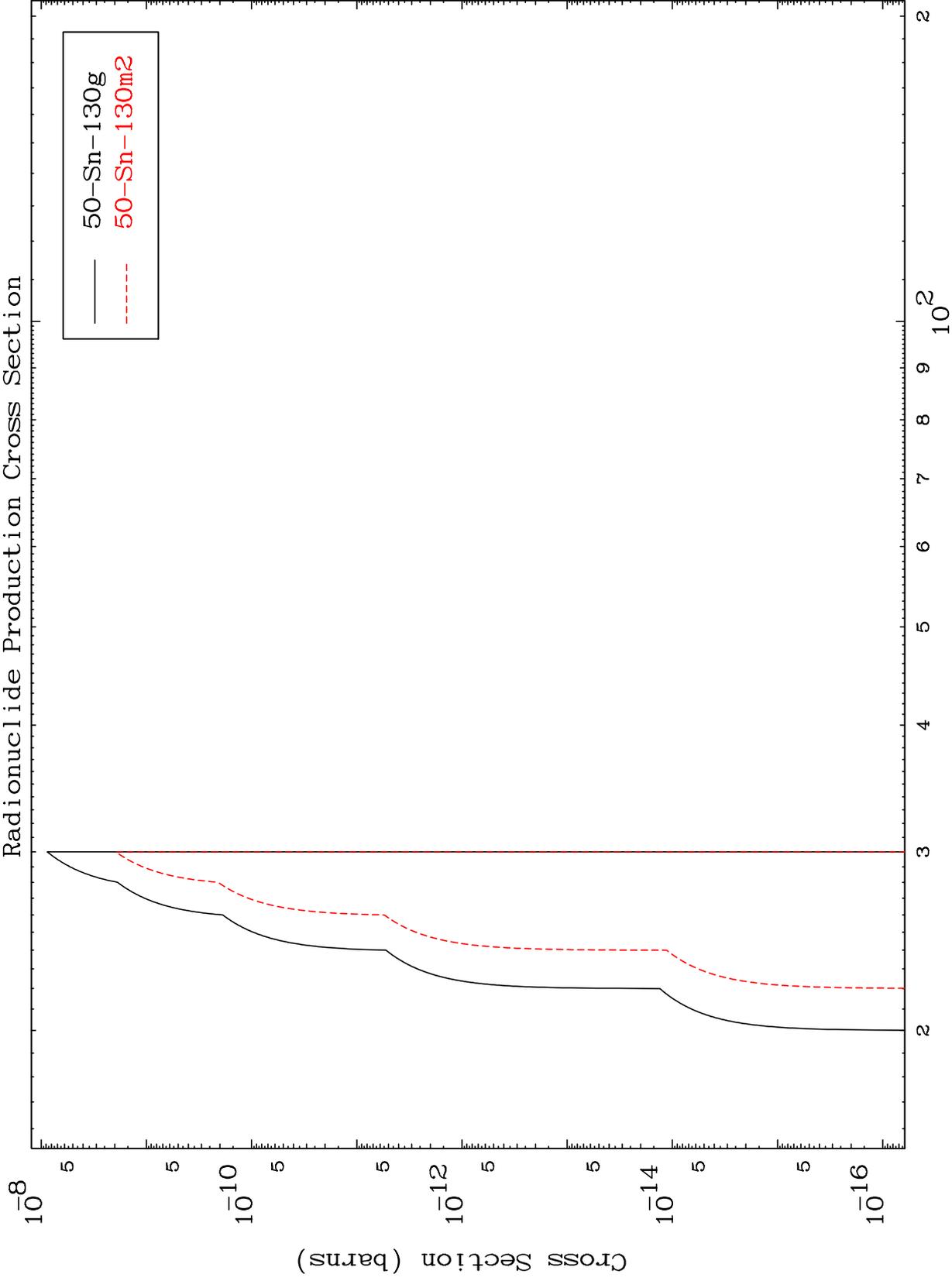
Radionuclide Production Cross Section
(n, α)



MAT 5073

50-Sn-128

(n,2p)
Radionuclide Production Cross Section



22

50-Sn-128

Incident Energy (MeV)