

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail: [redcullen1@comcast.net](mailto:redcullen1@comcast.net)  
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

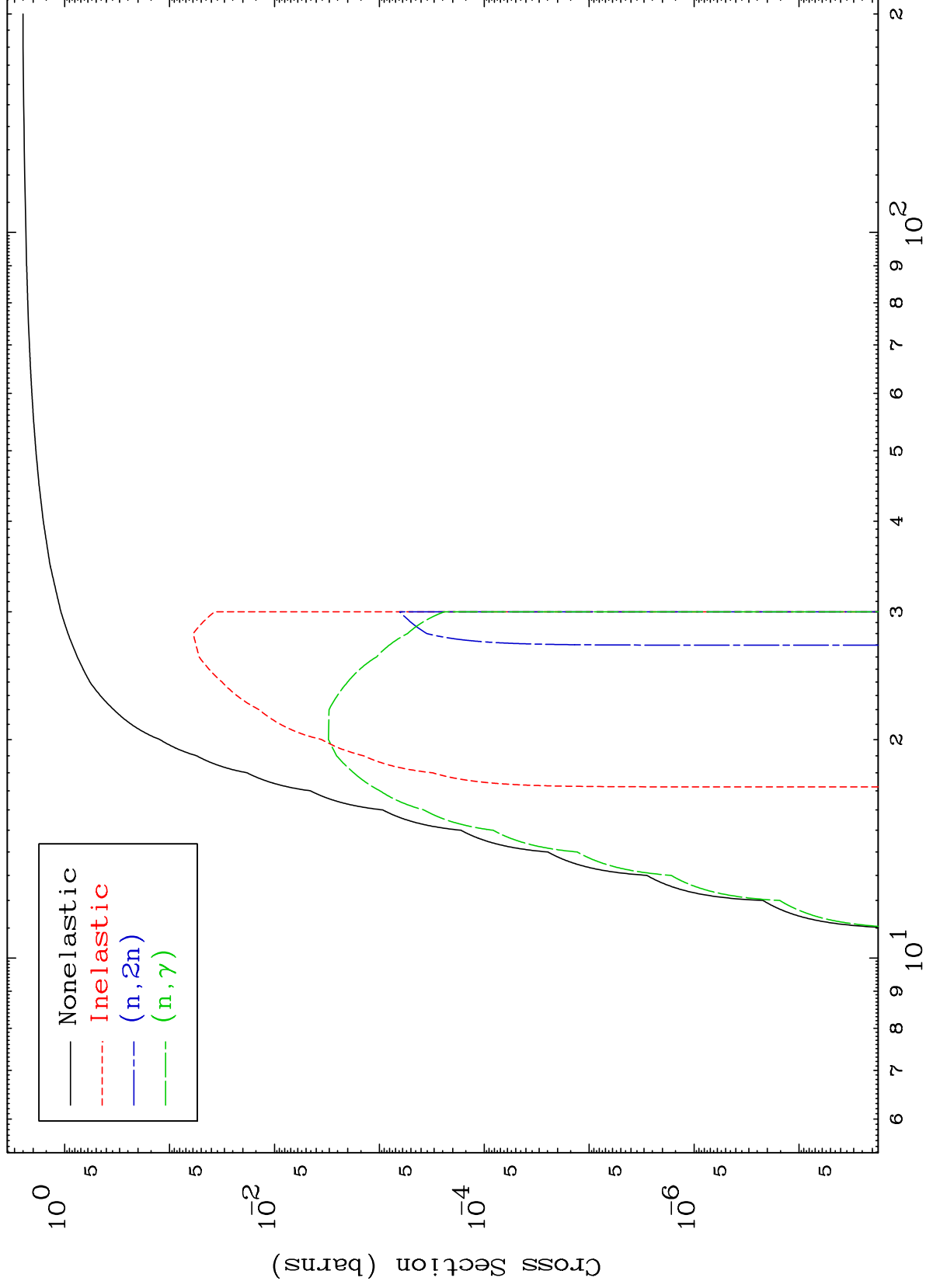
Press Mouse Button to Start

MAT 7269

0 Kelvin

$\alpha$  Major Cross Sections

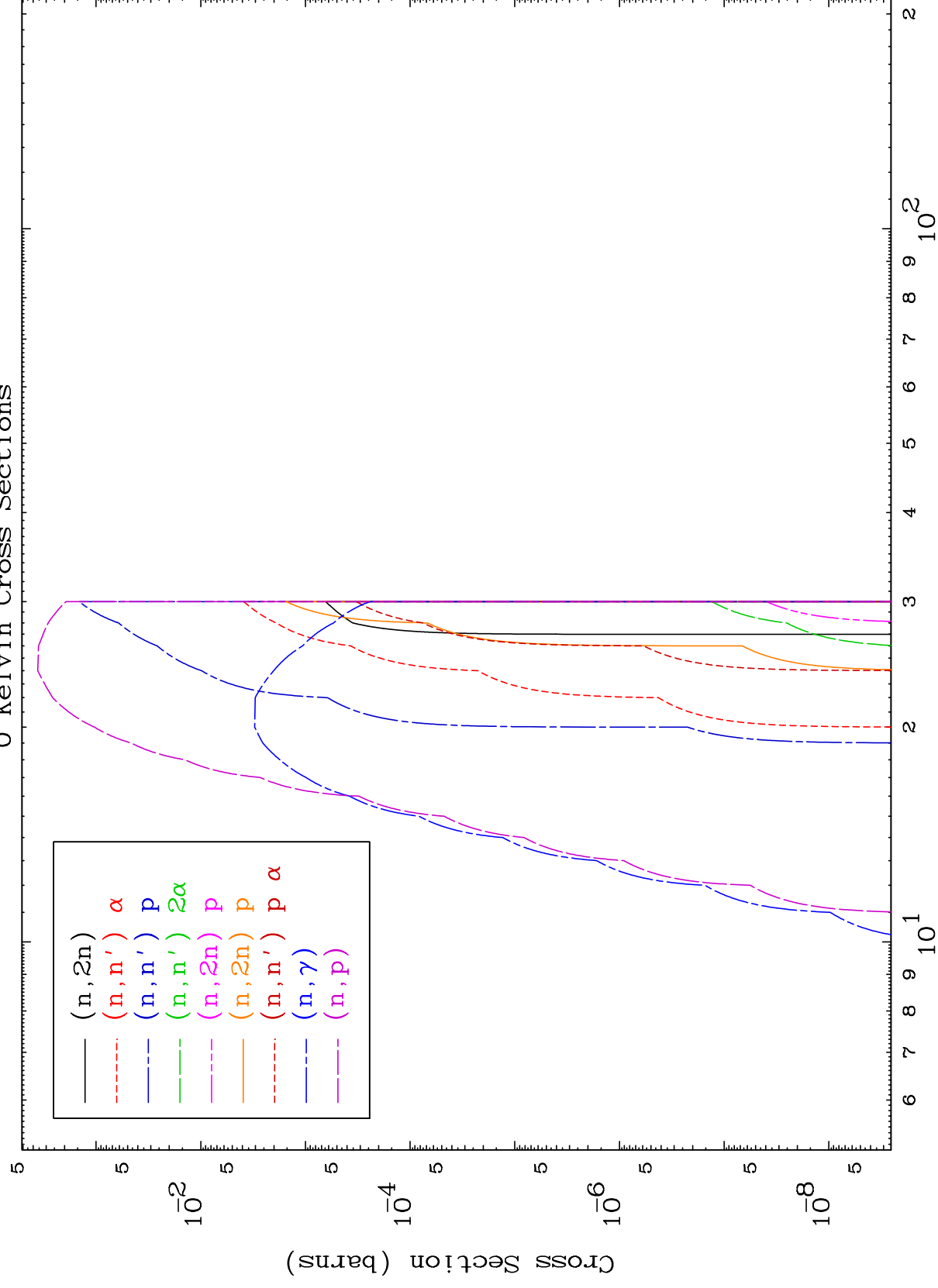
$^{73}\text{Ta-161m}$



1

Incident Energy (MeV)

$^{73}\text{Ta-161m}$

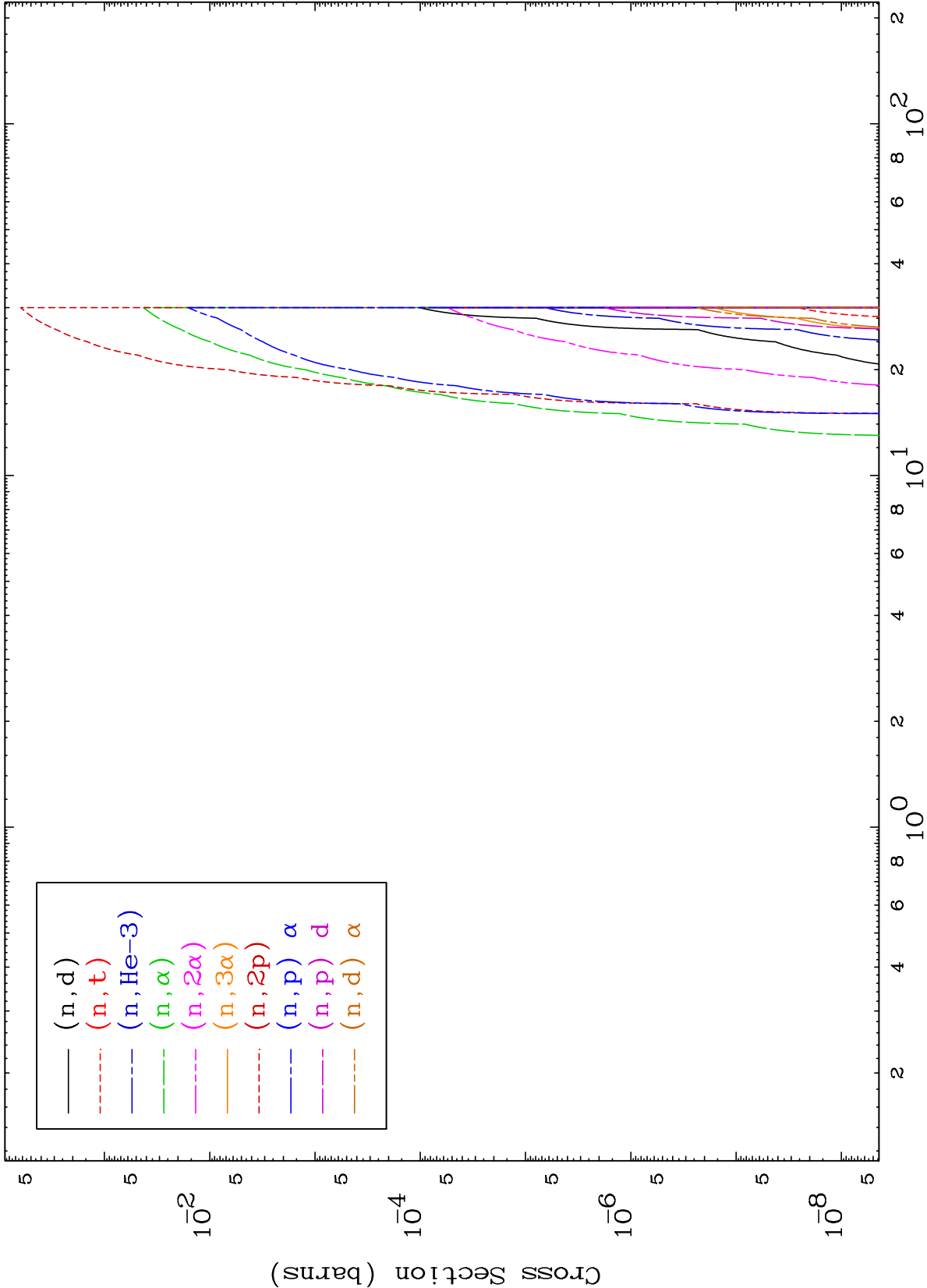


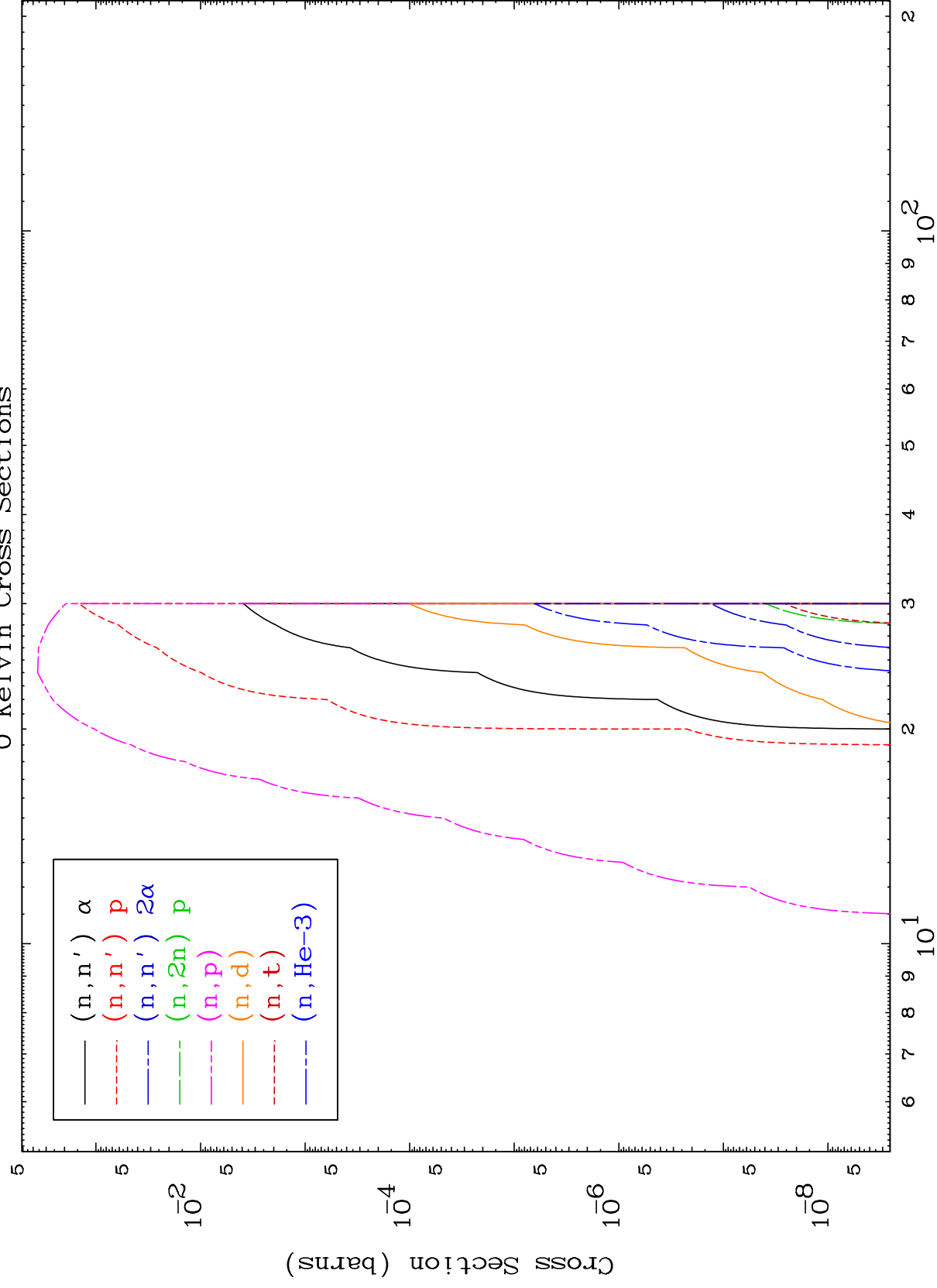
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$\alpha$  Neutron Absorption

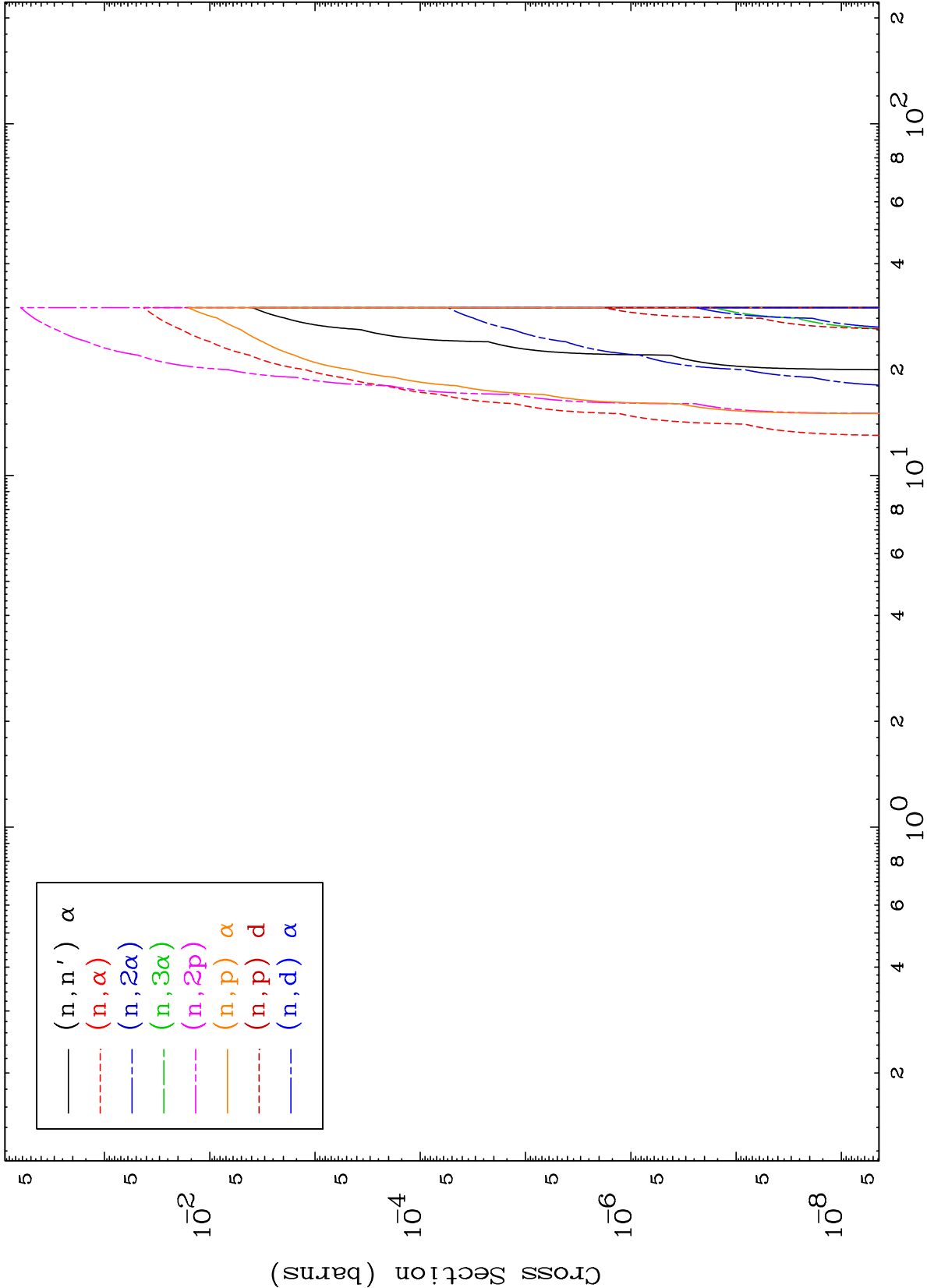
$^{73}\text{Ta-161m}$

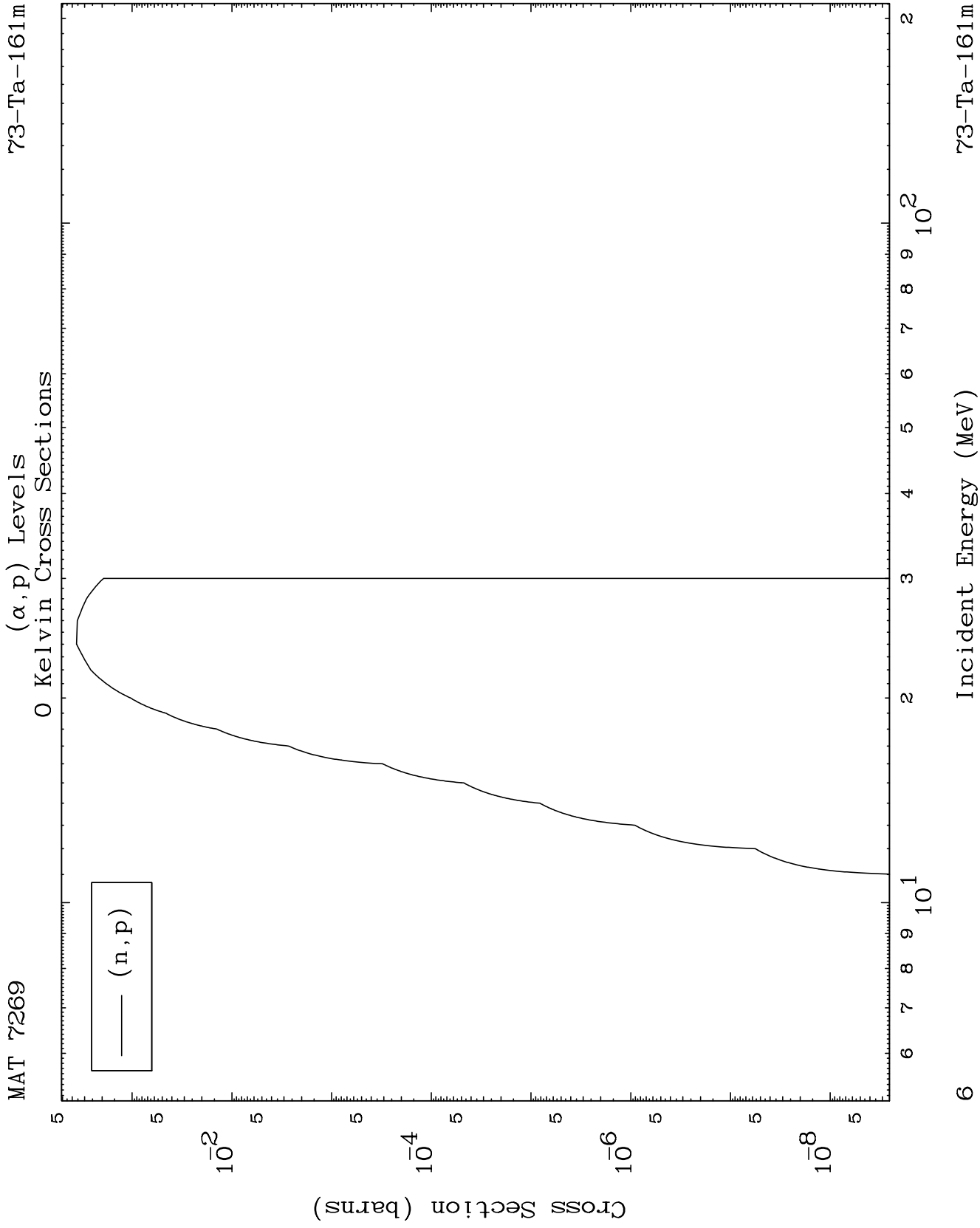
0 Kelvin Cross Sections

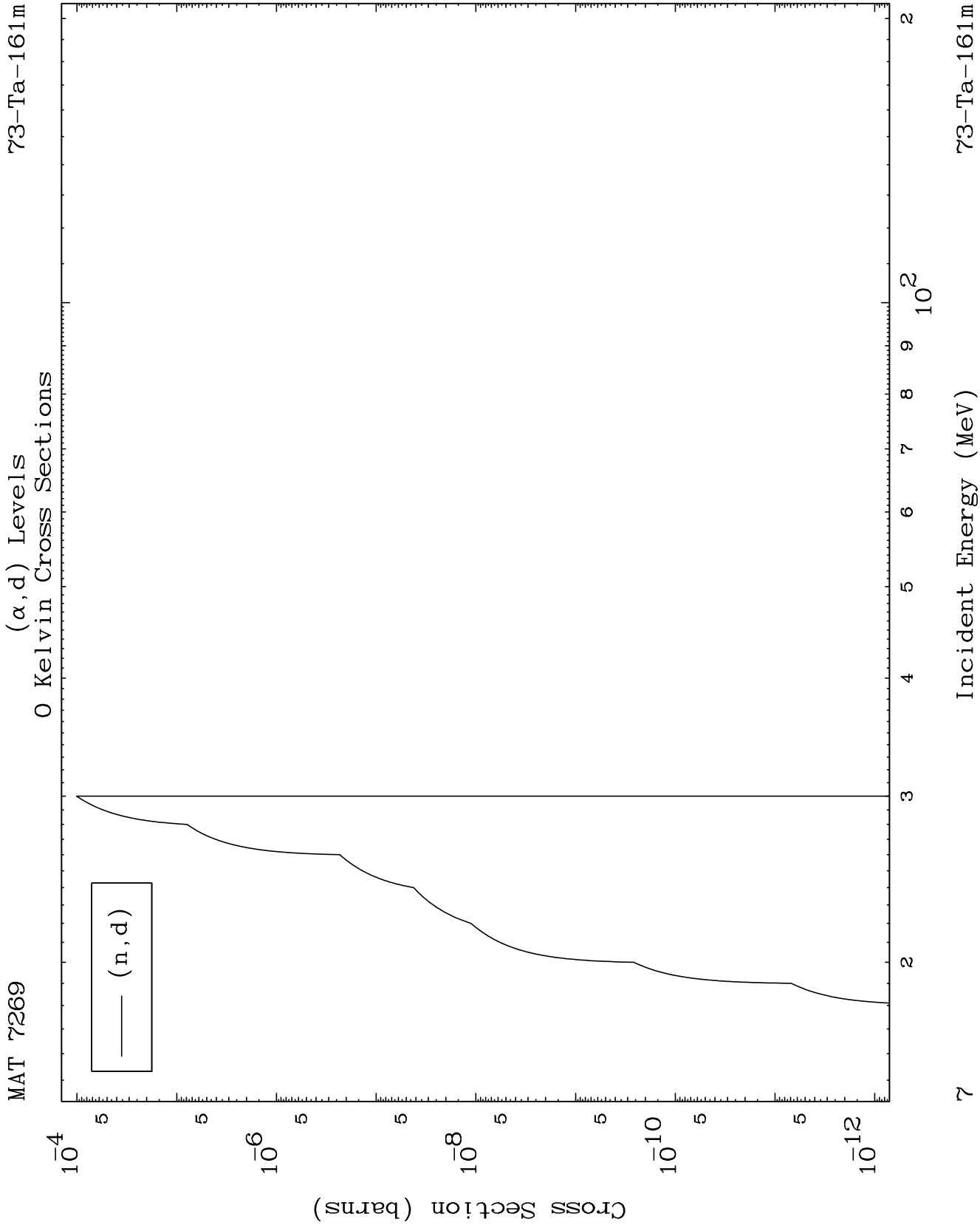




0 Kelvin Cross Sections



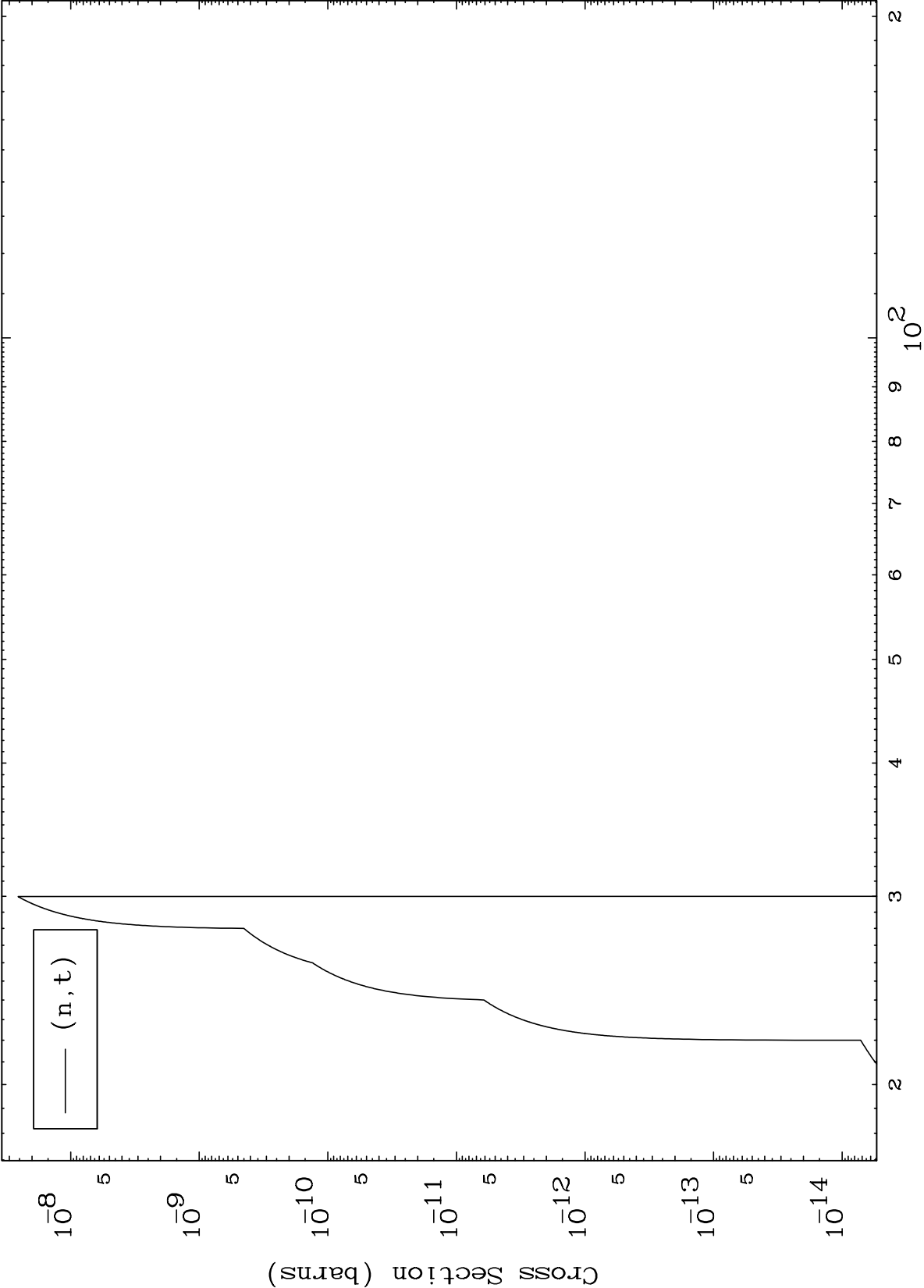




MAT 7269

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

$^{73}\text{Ta-161m}$

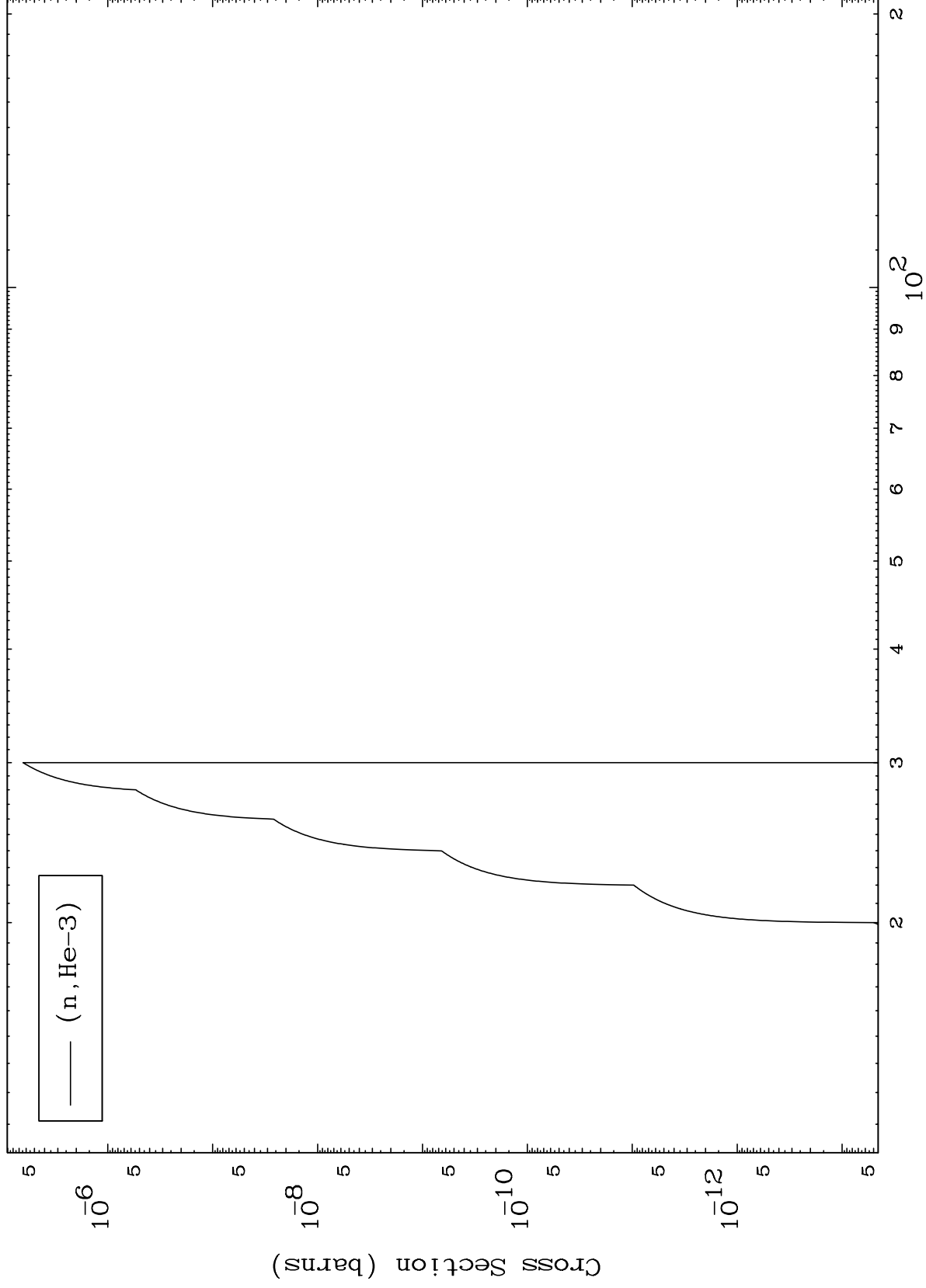


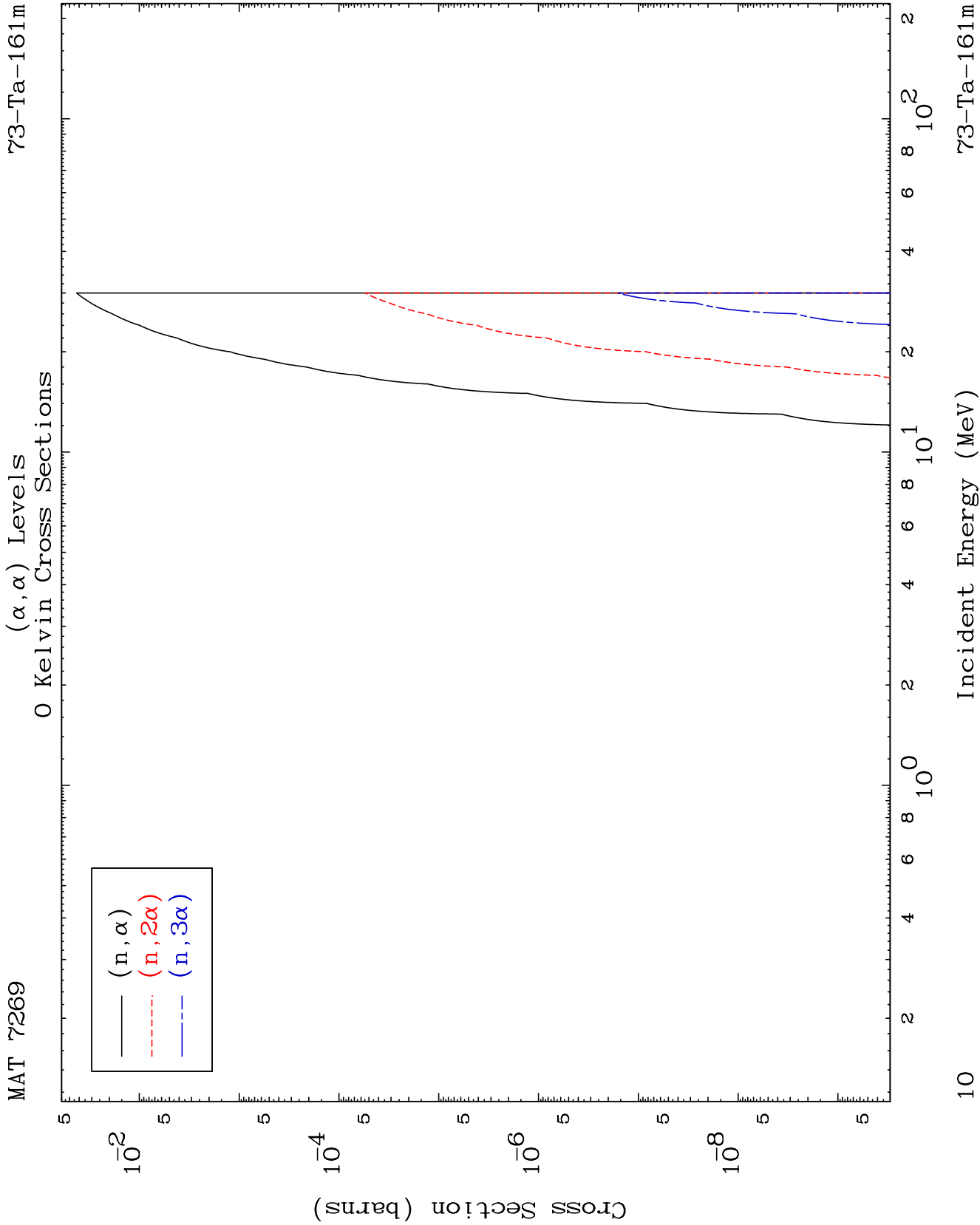
8

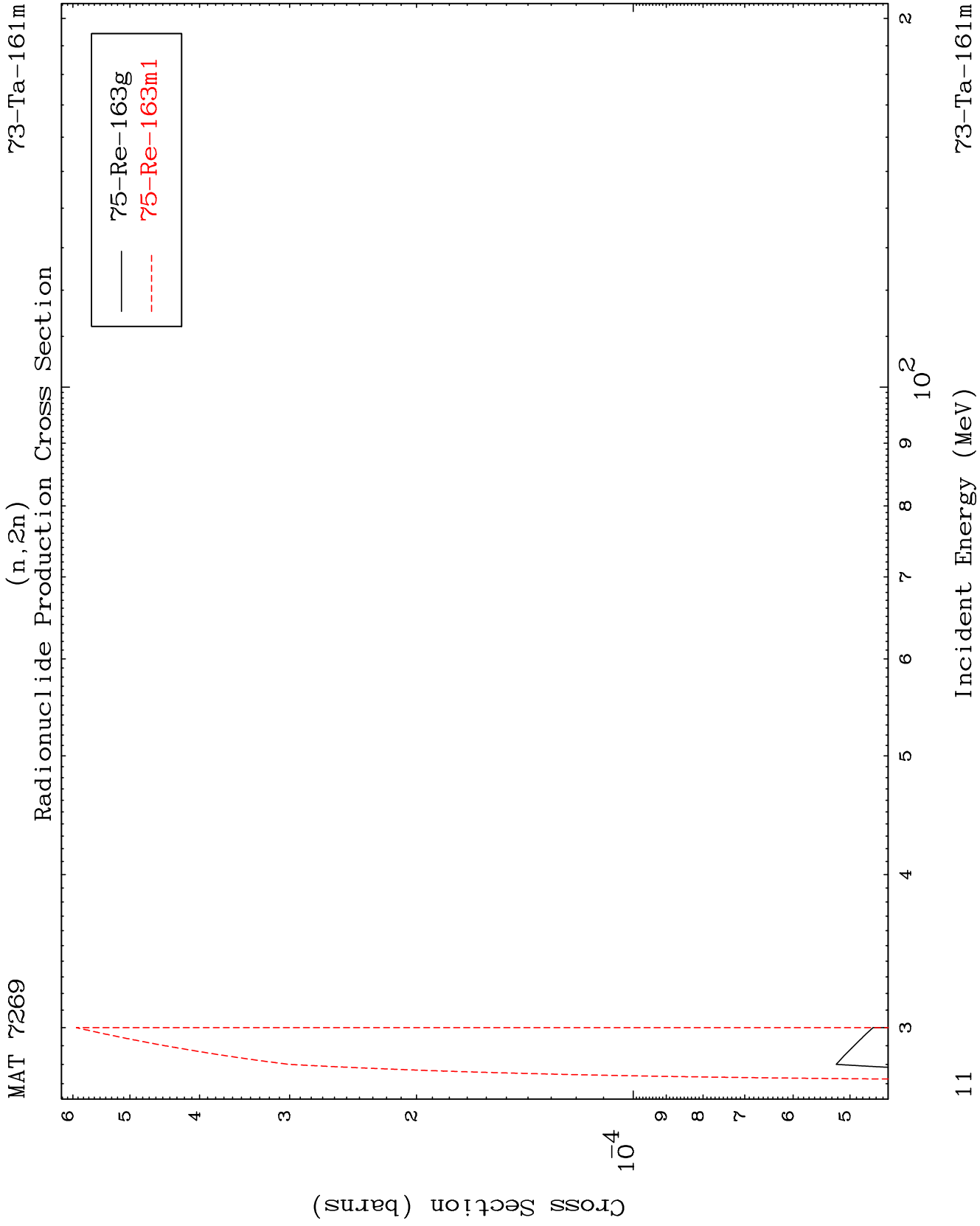
Incident Energy (MeV)

$^{73}\text{Ta-161m}$

( $\alpha$ ,He3) Levels  
0 Kelvin Cross Sections



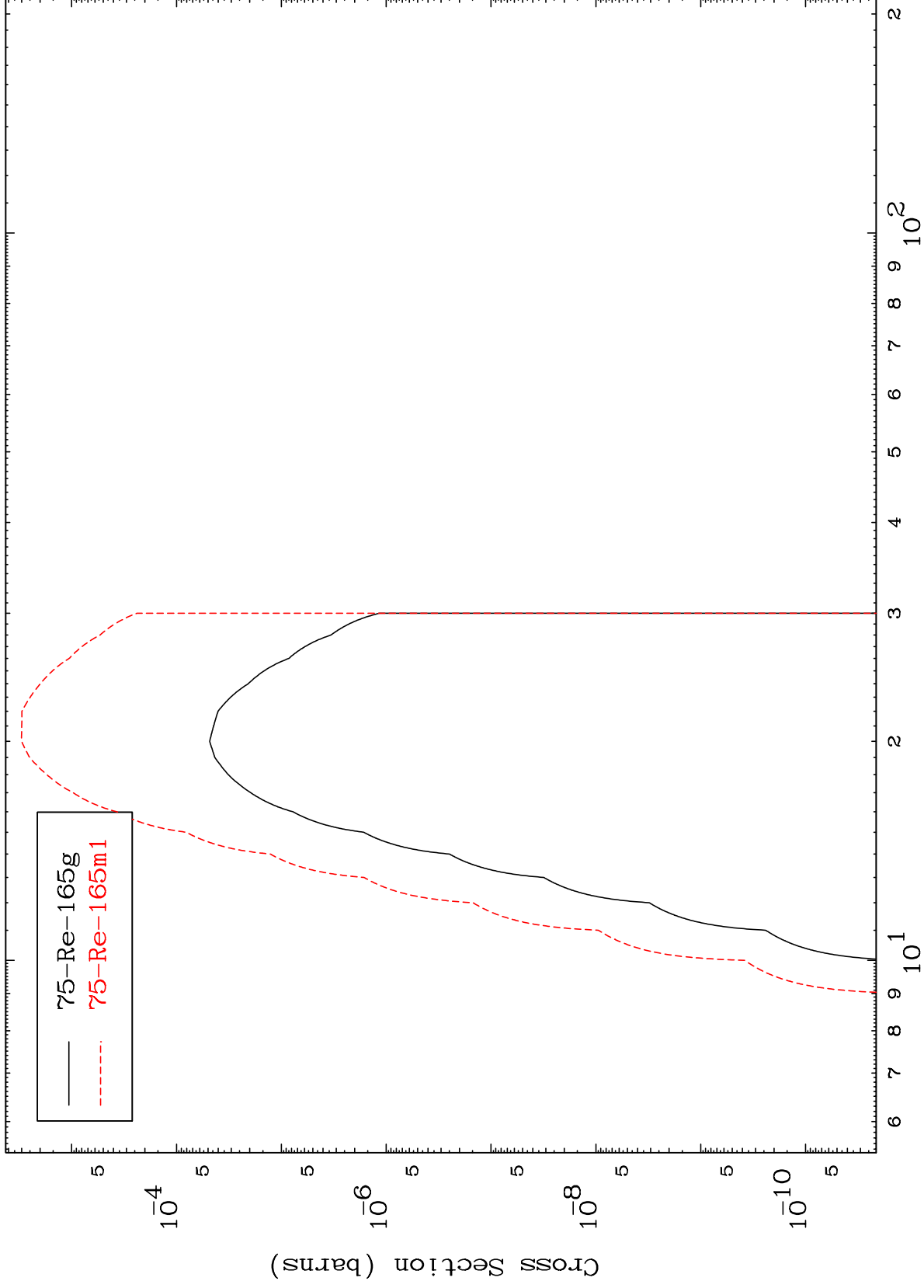




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<sup>73</sup>Ta-161m

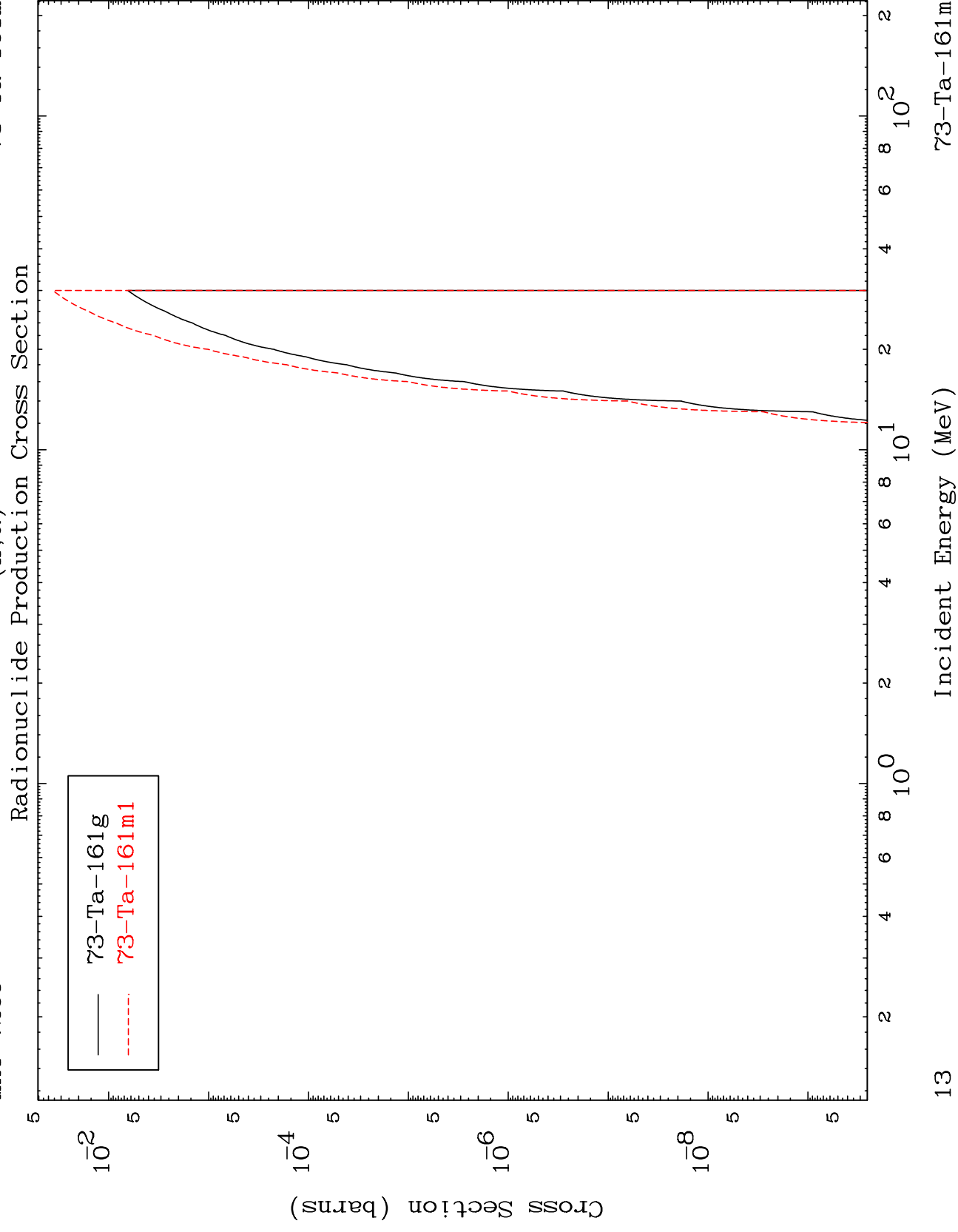
Radionuclide Production Cross Section



<sup>73</sup>Ta-161m

Incident Energy (MeV)

12

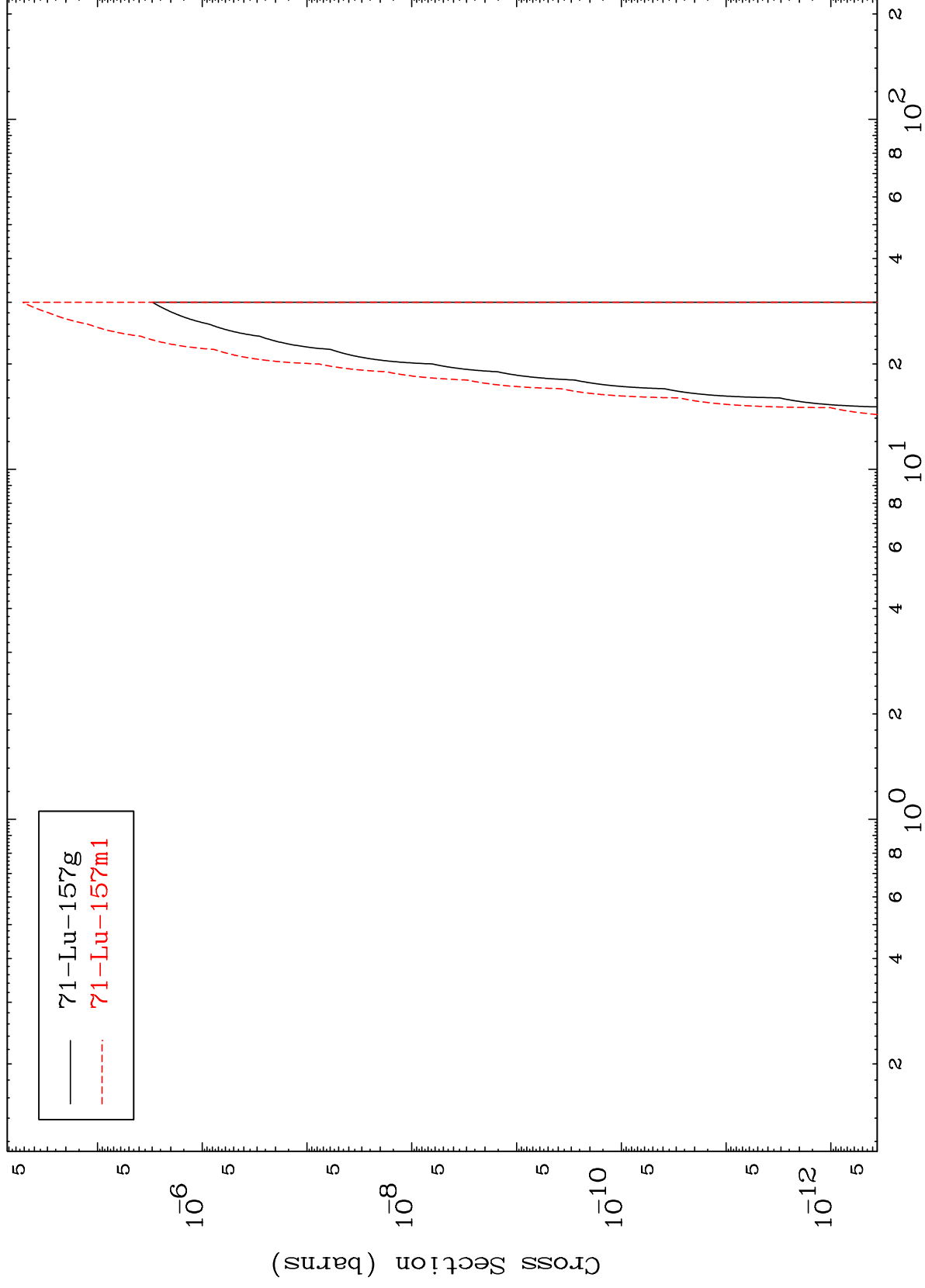


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(n,2α)

<sup>73</sup>Ta-161m

Radionuclide Production Cross Section



14

Incident Energy (MeV)

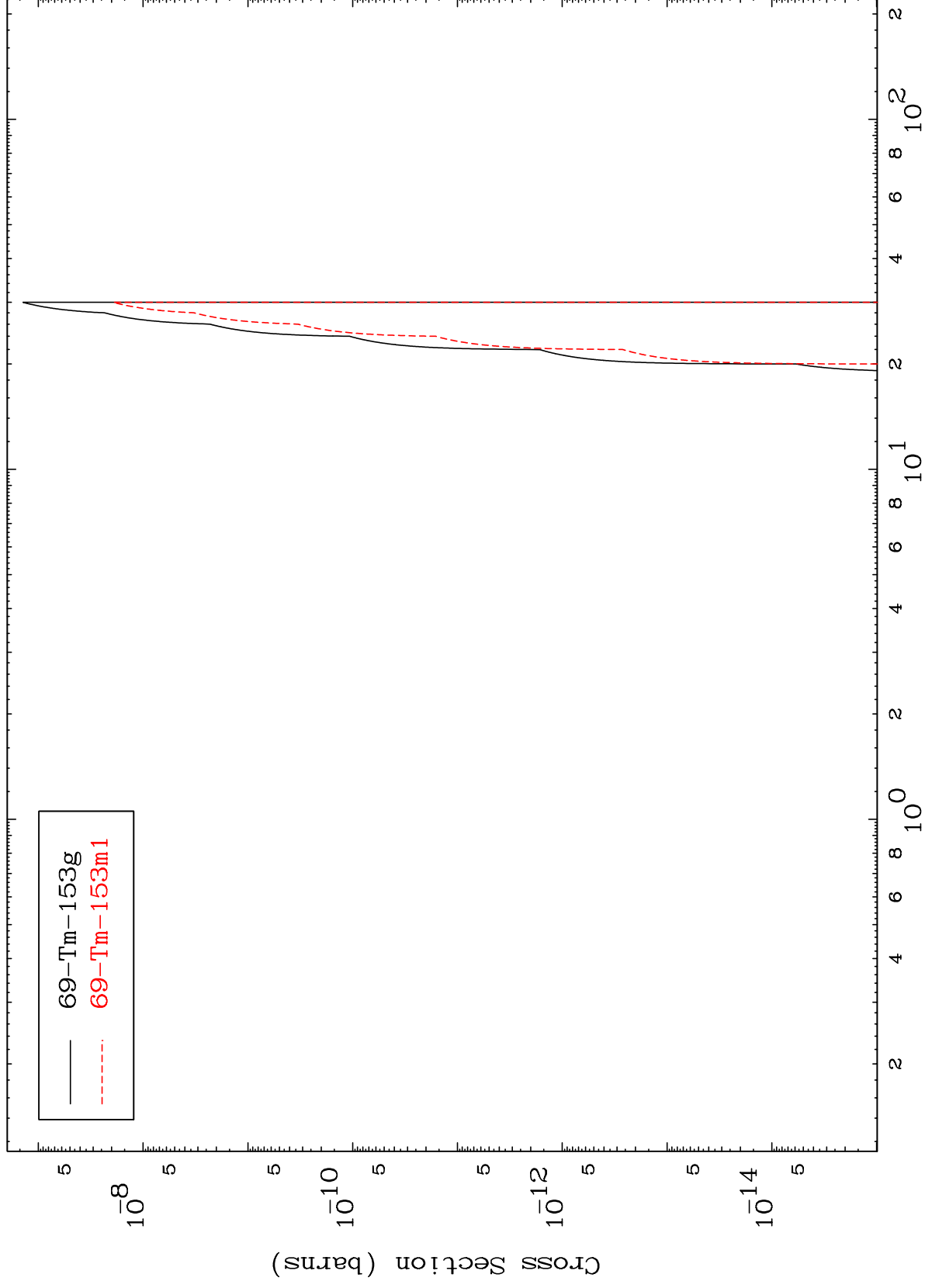
<sup>73</sup>Ta-161m

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(n,3α)

<sup>73</sup>Ta-161m

Radionuclide Production Cross Section



15

Incident Energy (MeV)

<sup>73</sup>Ta-161m