

Program EVALPLOT  
(Version 2021-1)

by

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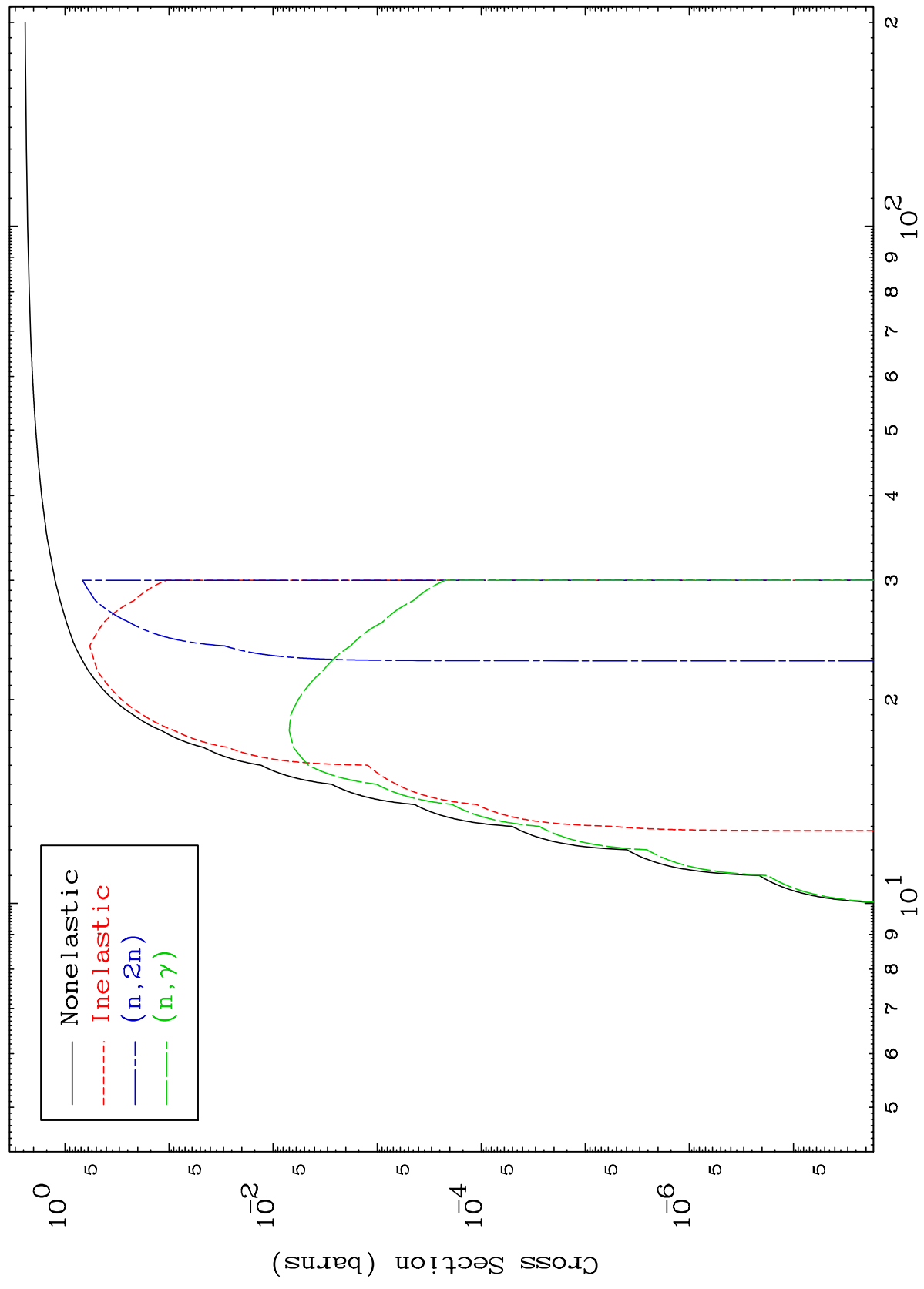
Press Mouse Button to Start

MAT 6493

$\alpha$  Major

0 Kelvin Cross Sections

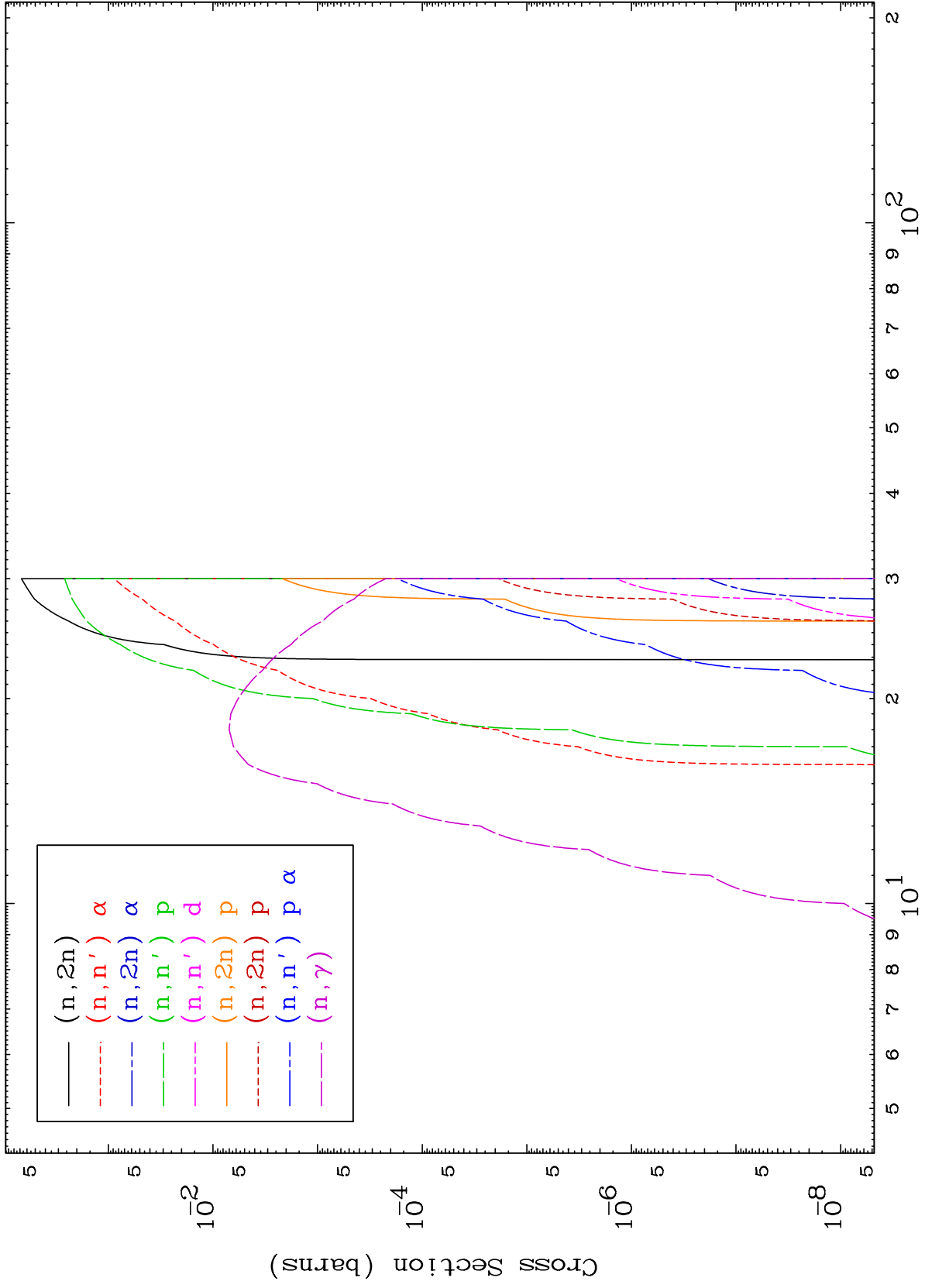
65-Tb-148m



MAT 6493

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

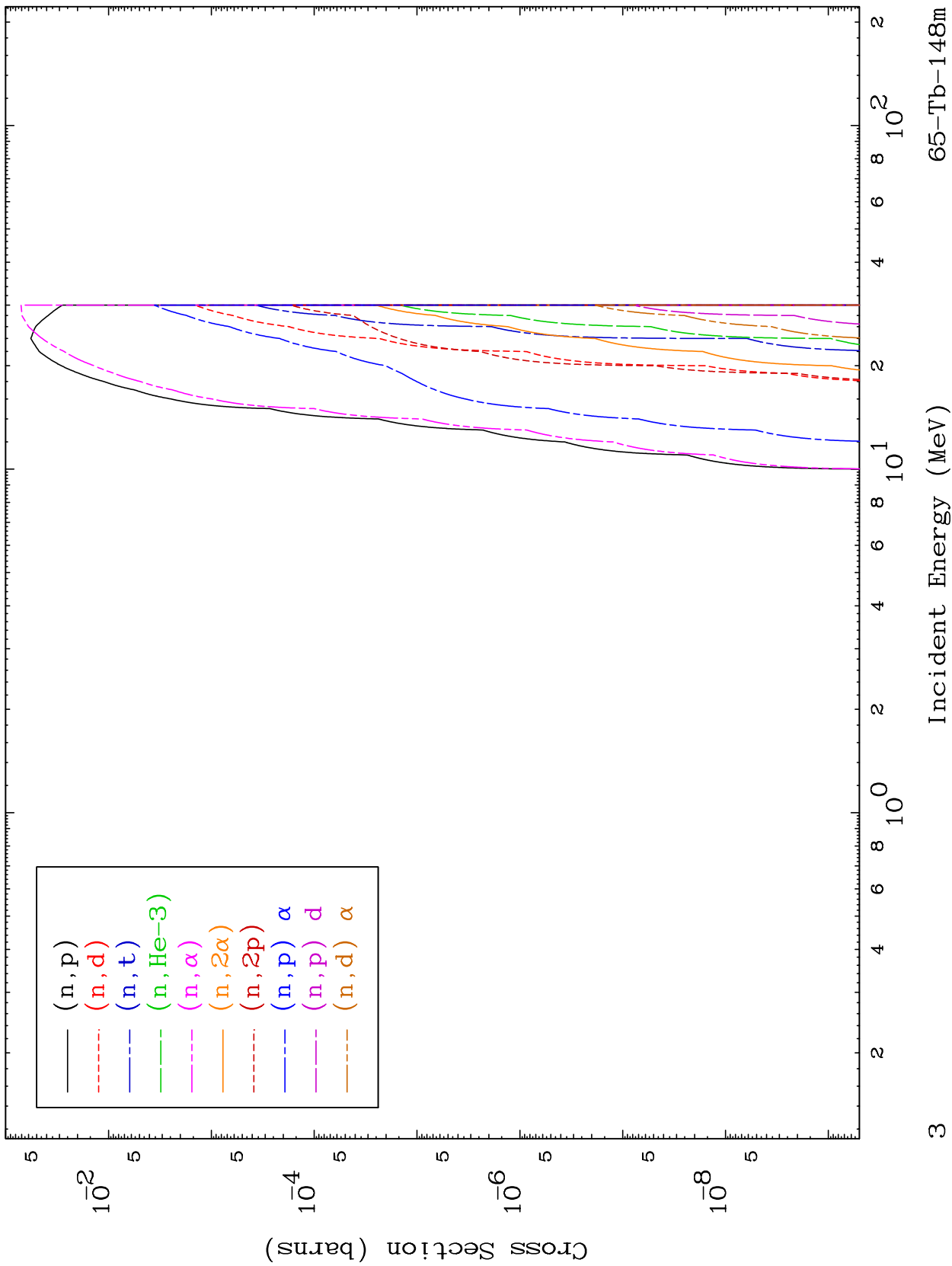
65-Tb-148m

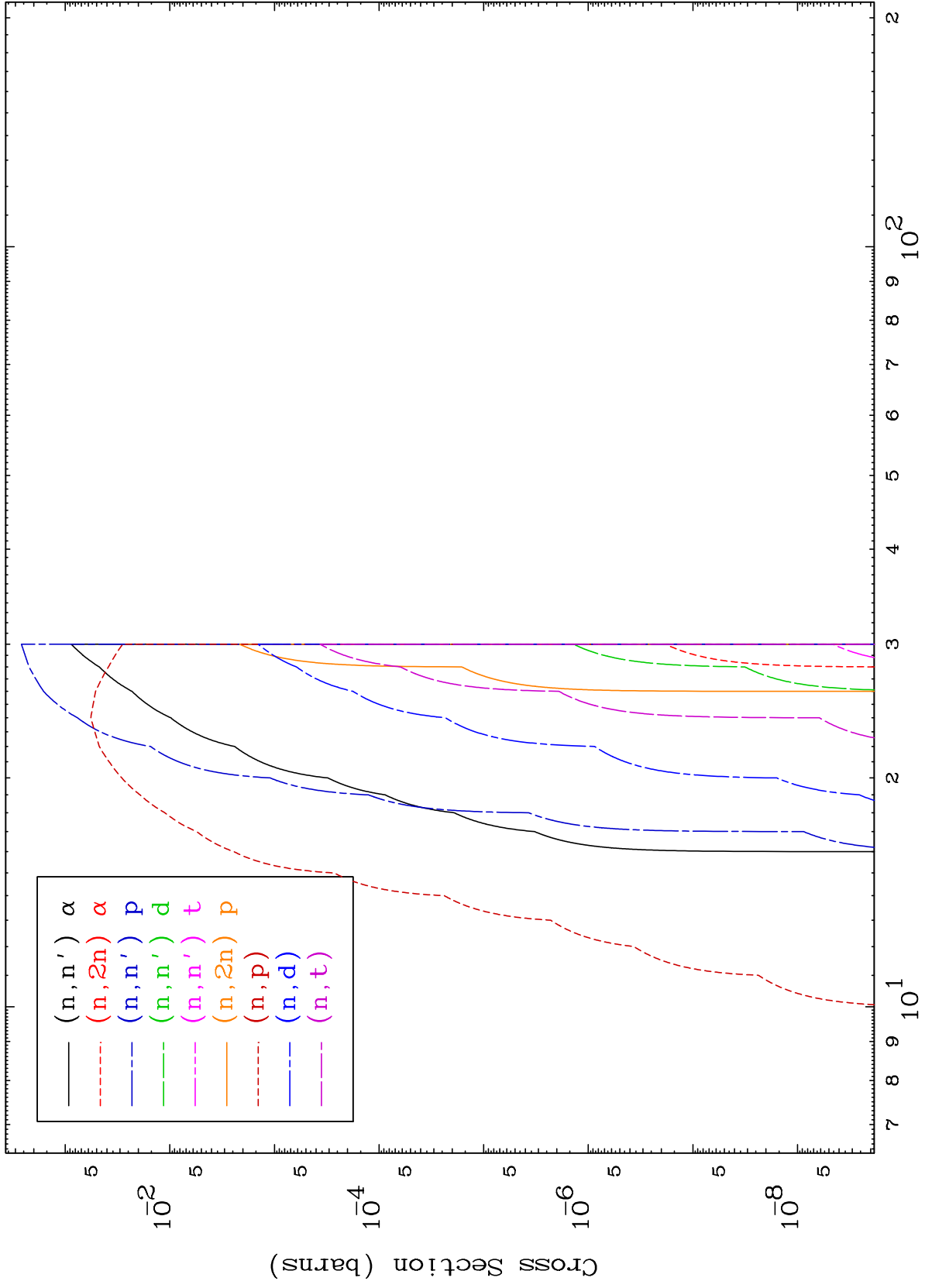


MAT 6493

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

65-Tb-148m

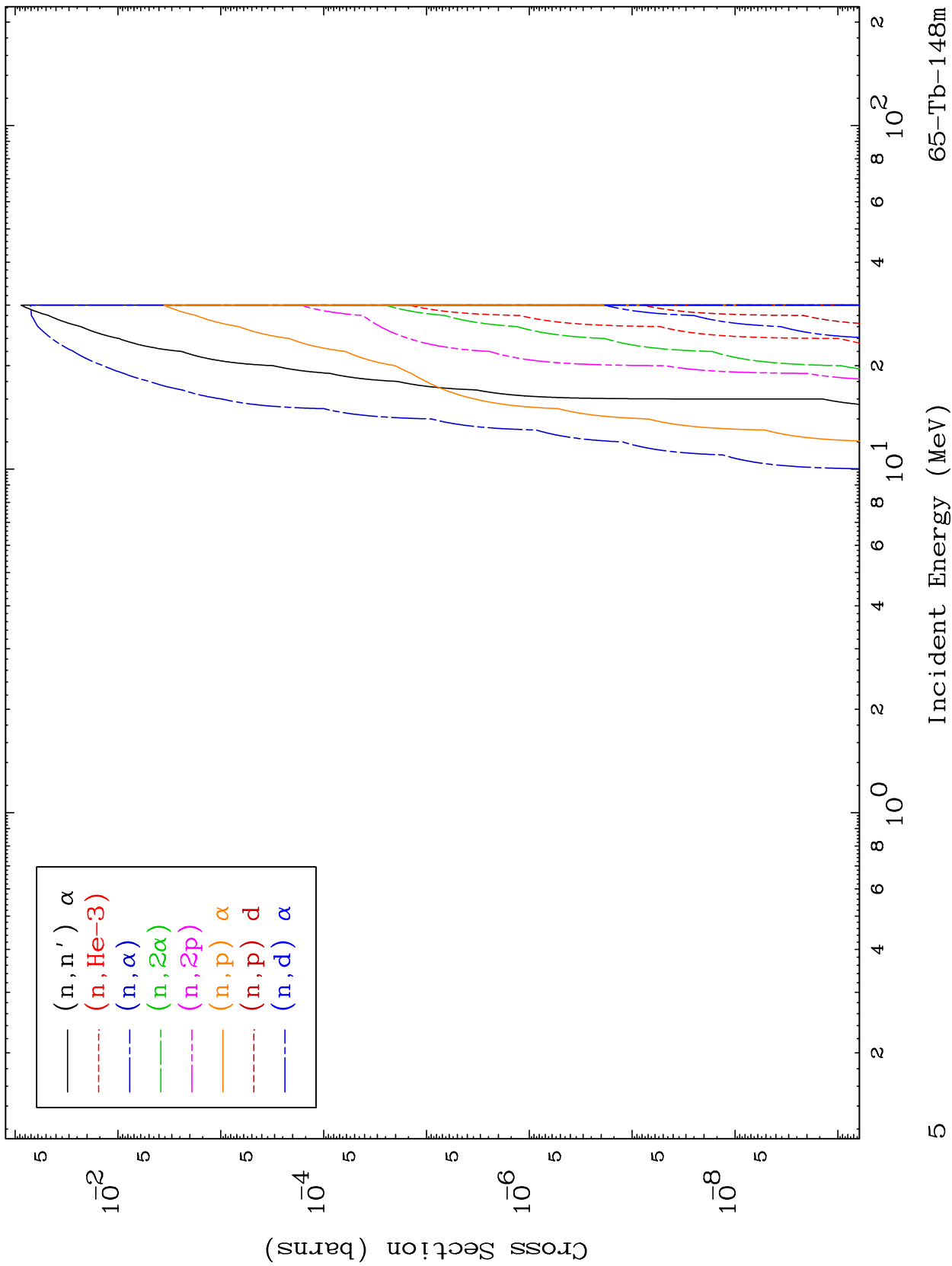




MAT 6493

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

65-Tb-148m

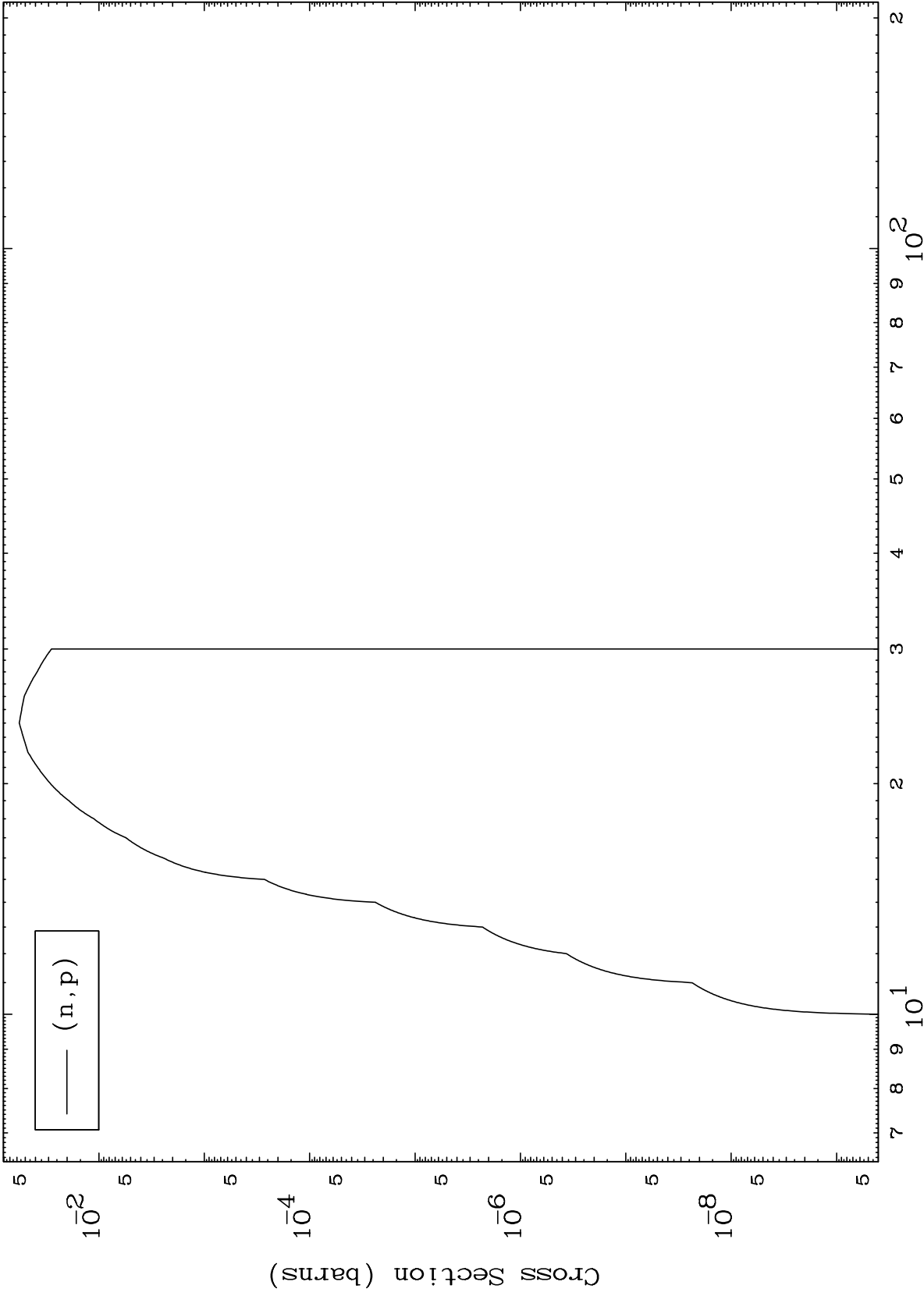


MAT 6493

( $\alpha, p$ ) Levels

65-Tb-148m

0 Kelvin Cross Sections



6

Incident Energy (MeV)

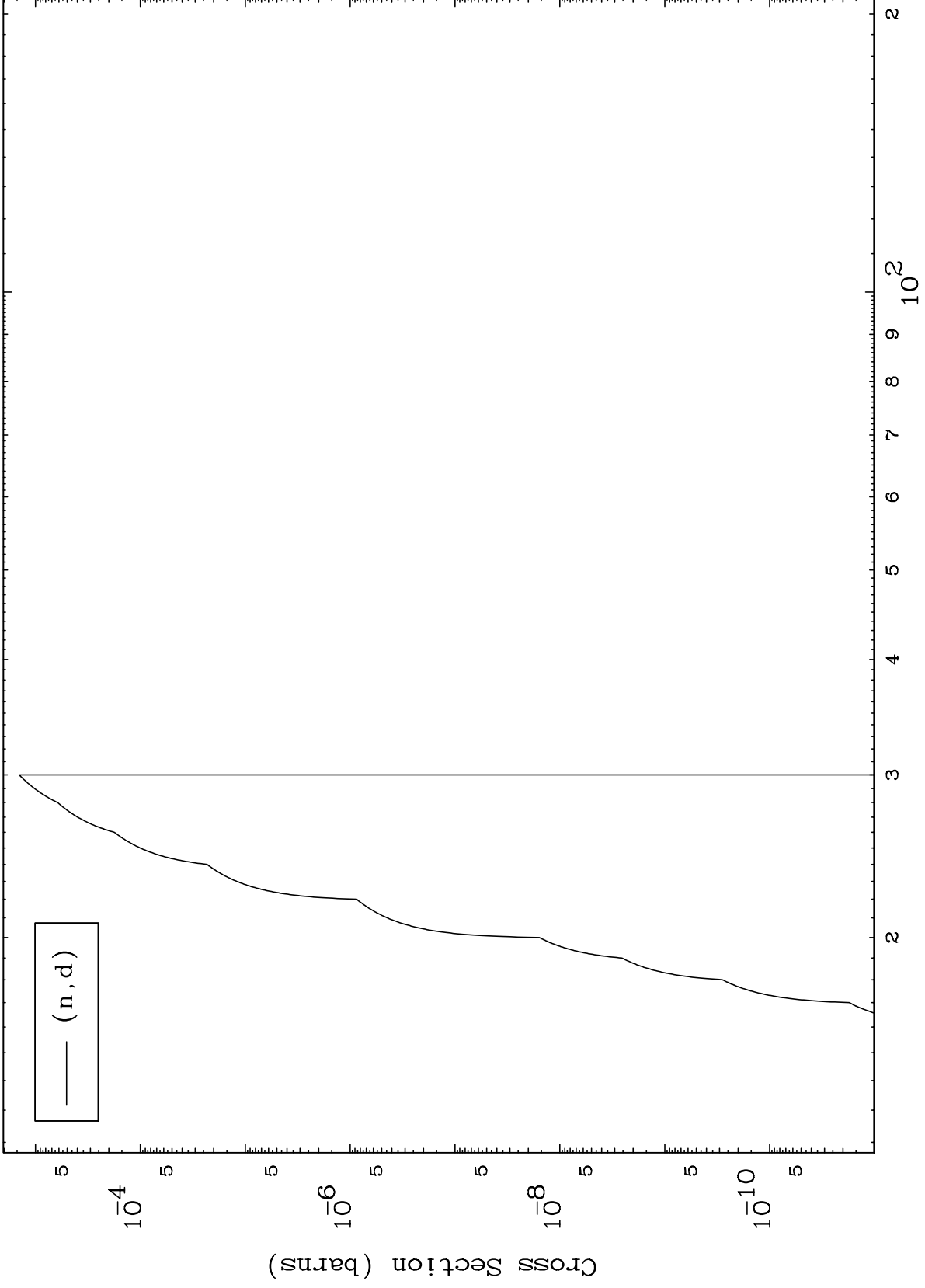
65-Tb-148m

MAT 6493

( $\alpha, d$ ) Levels

65-Tb-148m

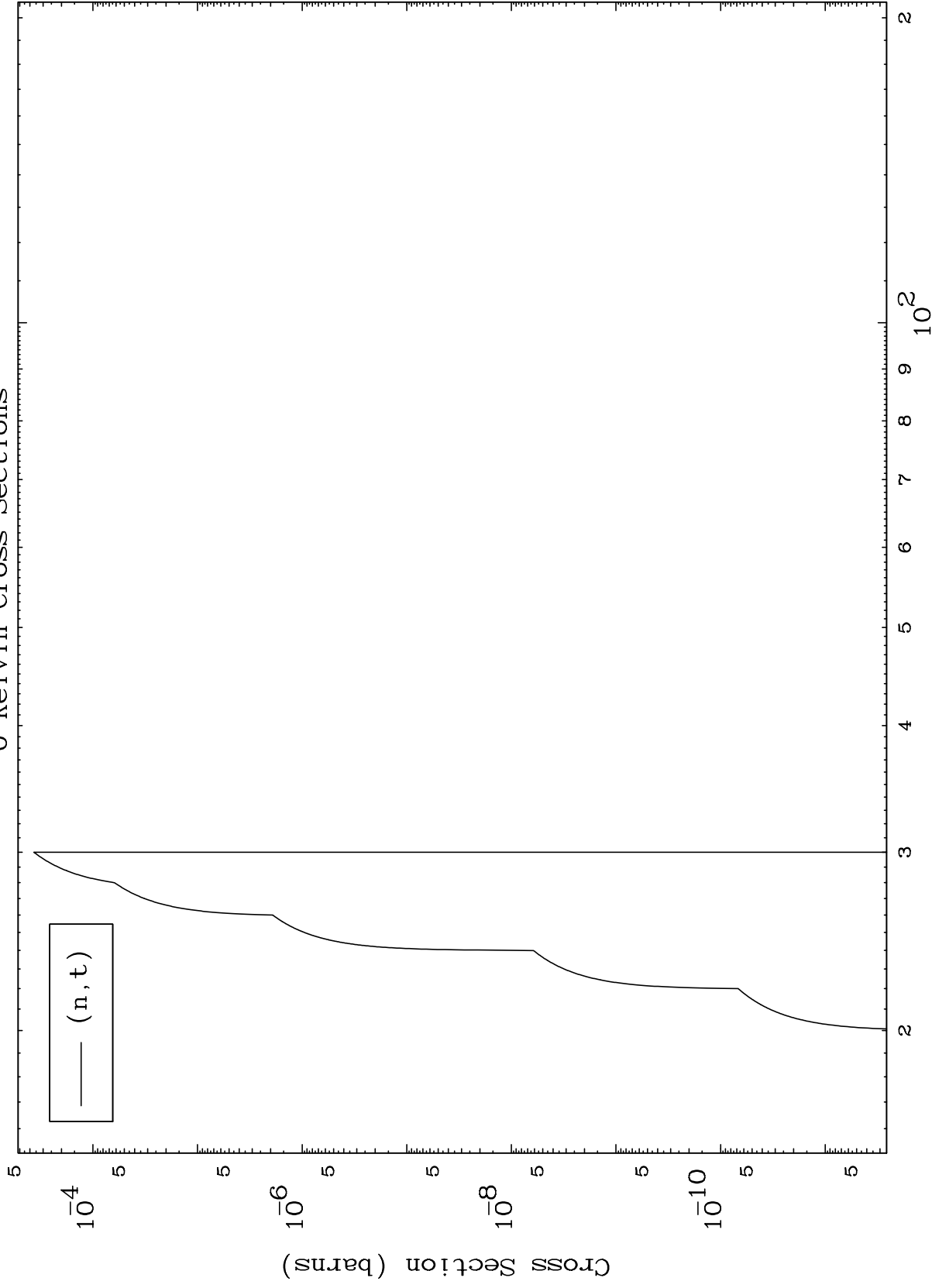
0 Kelvin Cross Sections



MAT 6493

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

65-Tb-148m



8

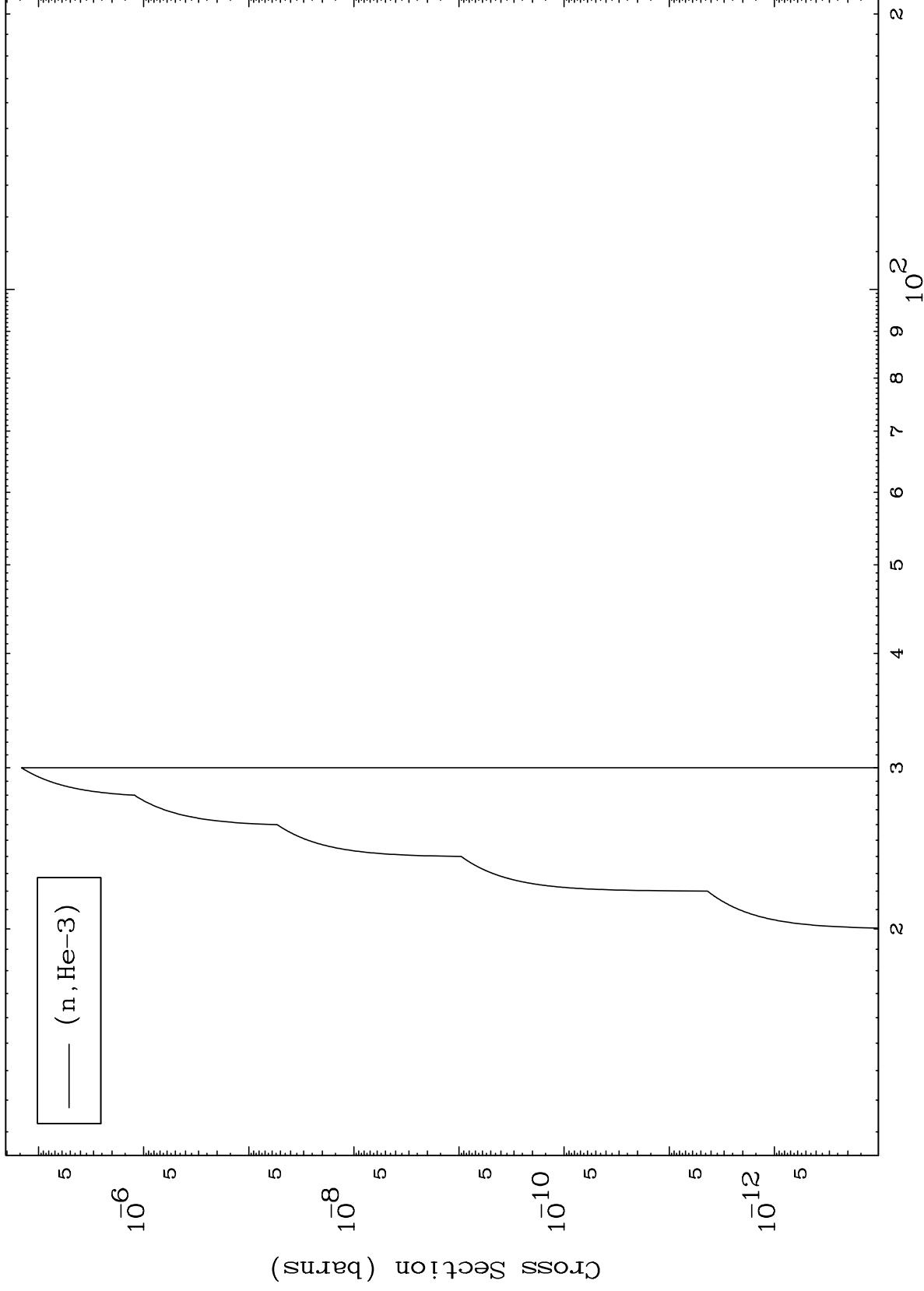
Incident Energy (MeV)

65-Tb-148m

MAT 6493

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

65-Tb-148m

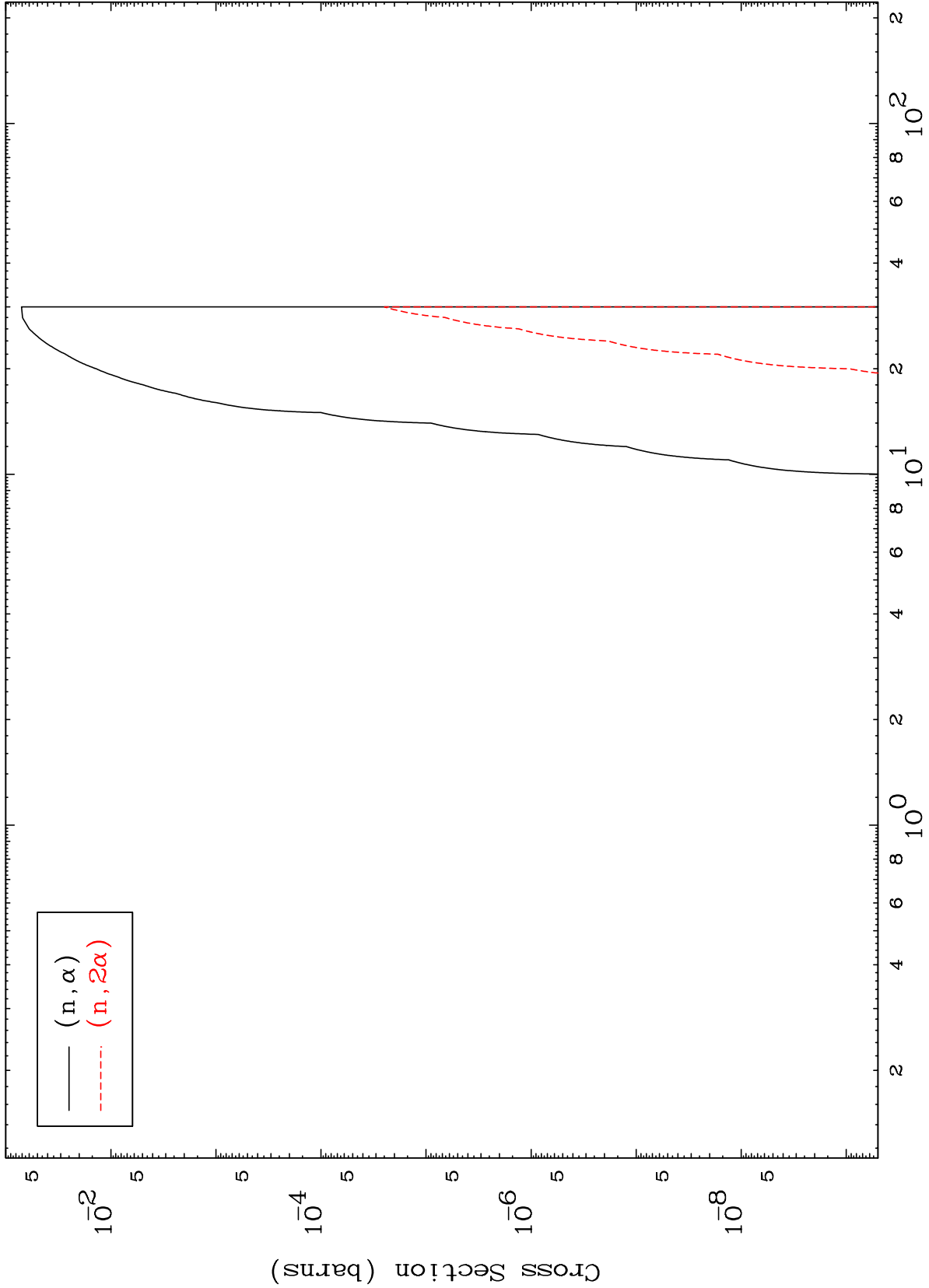


MAT 6493

( $\alpha, \alpha$ ) Levels

65-Tb-148m

0 Kelvin Cross Sections



— ( $n, \alpha$ )  
- - - ( $n, 2\alpha$ )

10

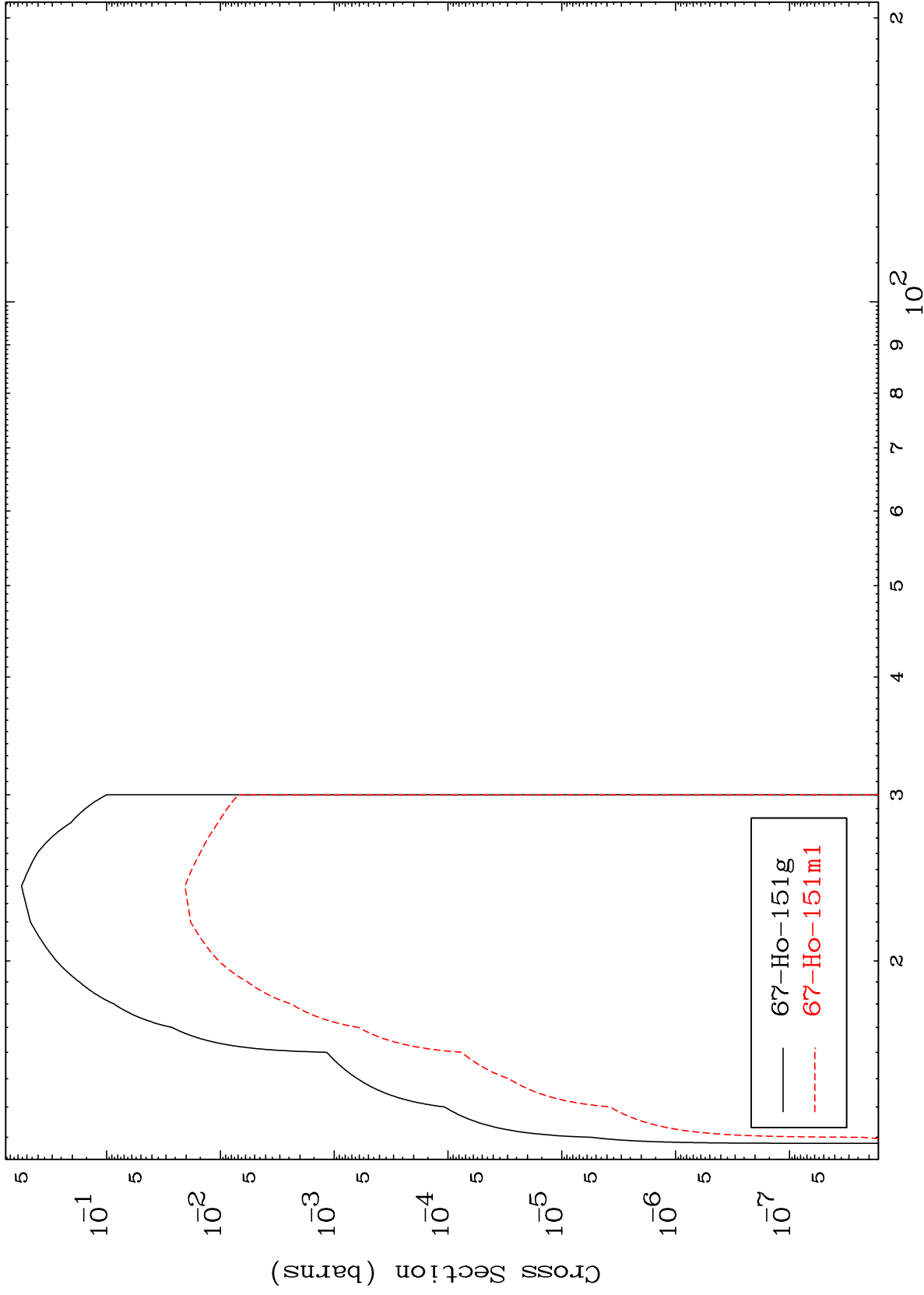
Incident Energy (MeV)

65-Tb-148m

MAT 6493

Inelastic  
Radionuclide Production Cross Section

65-Tb-148m

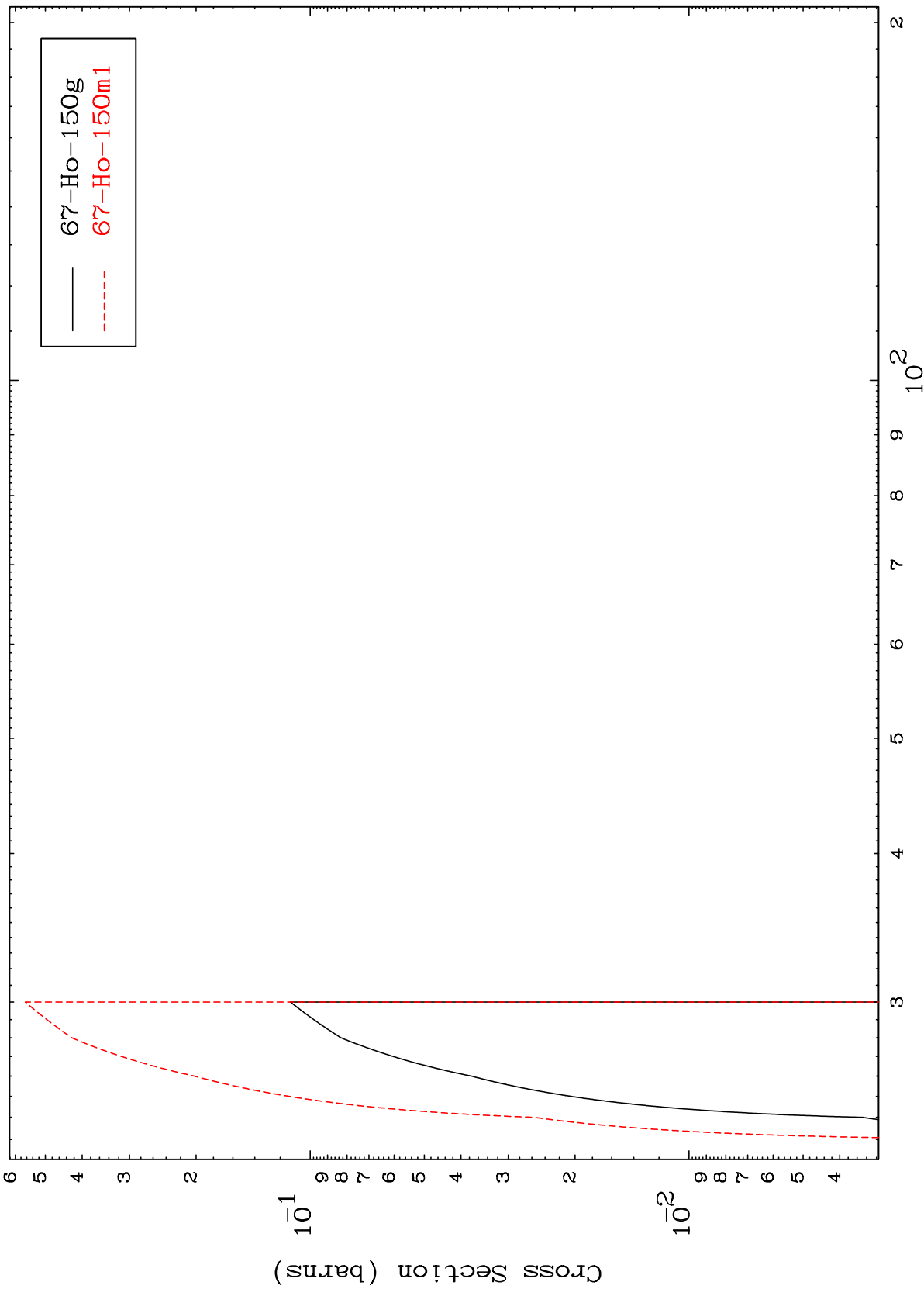


65-Tb-148m

MAT 6493

65-Tb-148m

(n,2n)  
Radionuclide Production Cross Section



65-Tb-148m

Incident Energy (MeV)

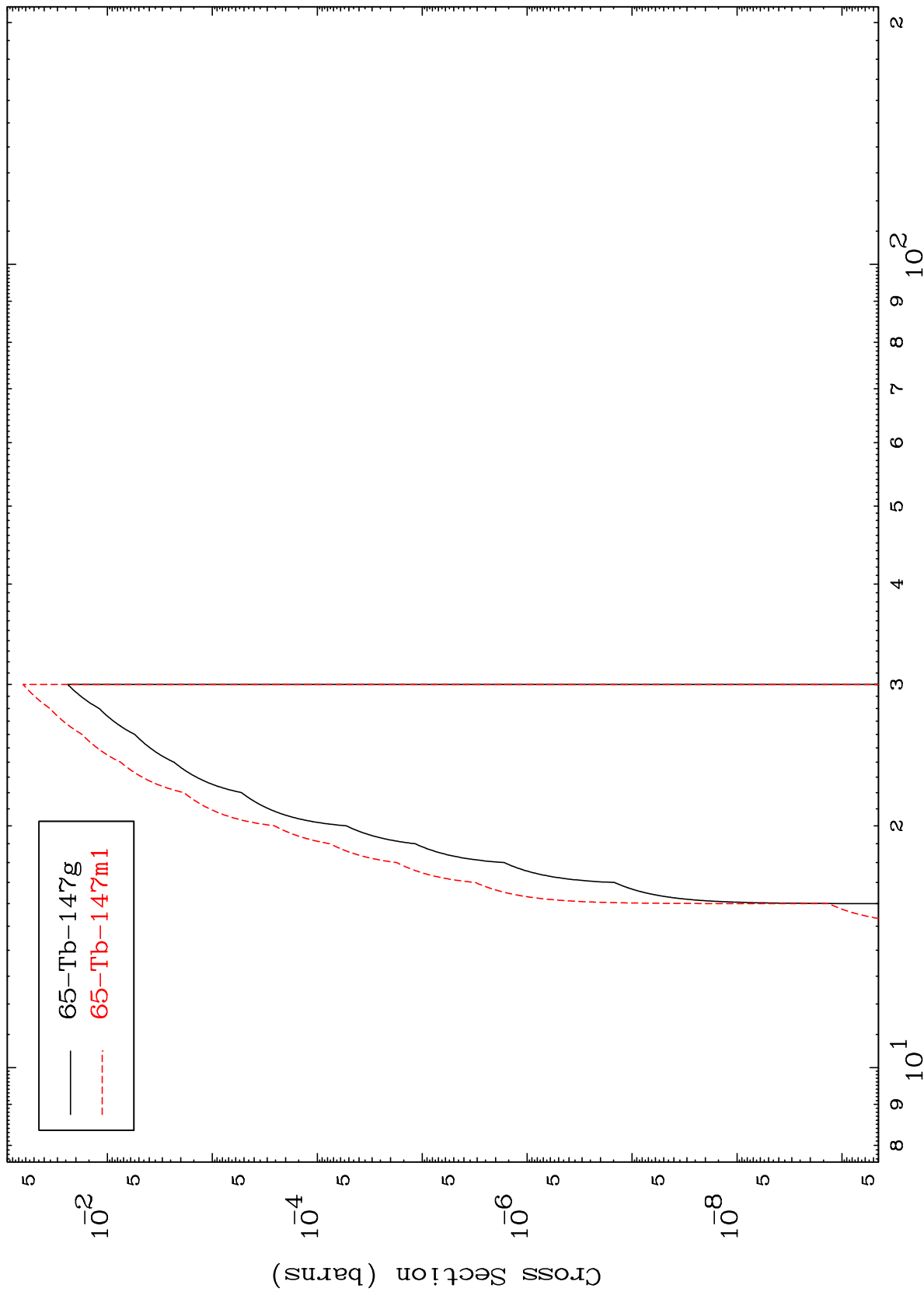
12

MAT 6493

$(n, n') \alpha$

65-Tb-148m

Radionuclide Production Cross Section



13

Incident Energy (MeV)

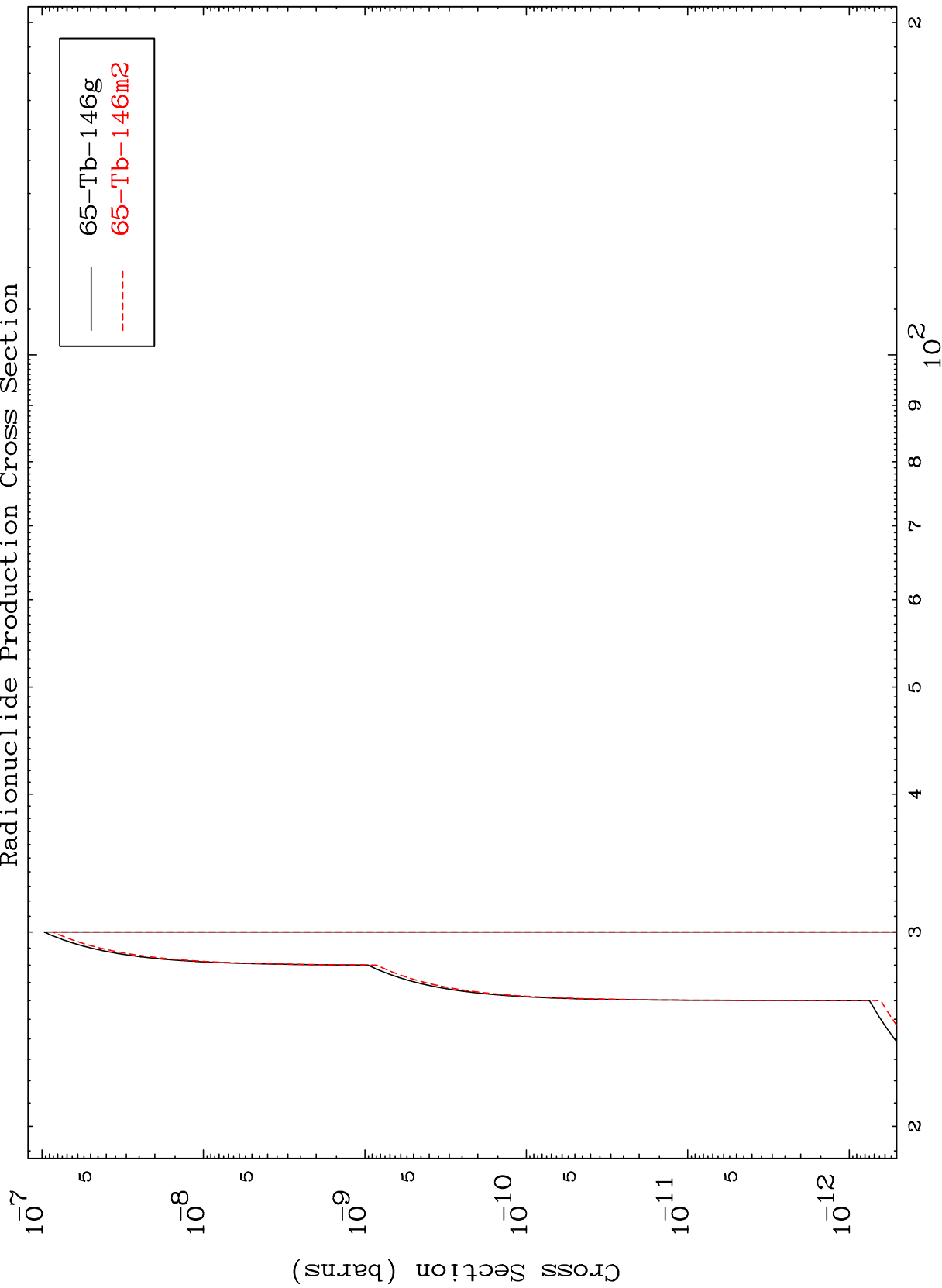
65-Tb-148m

MAT 6493

$(n,2n) \alpha$

65-Tb-148m

Radionuclide Production Cross Section



14

Incident Energy (MeV)

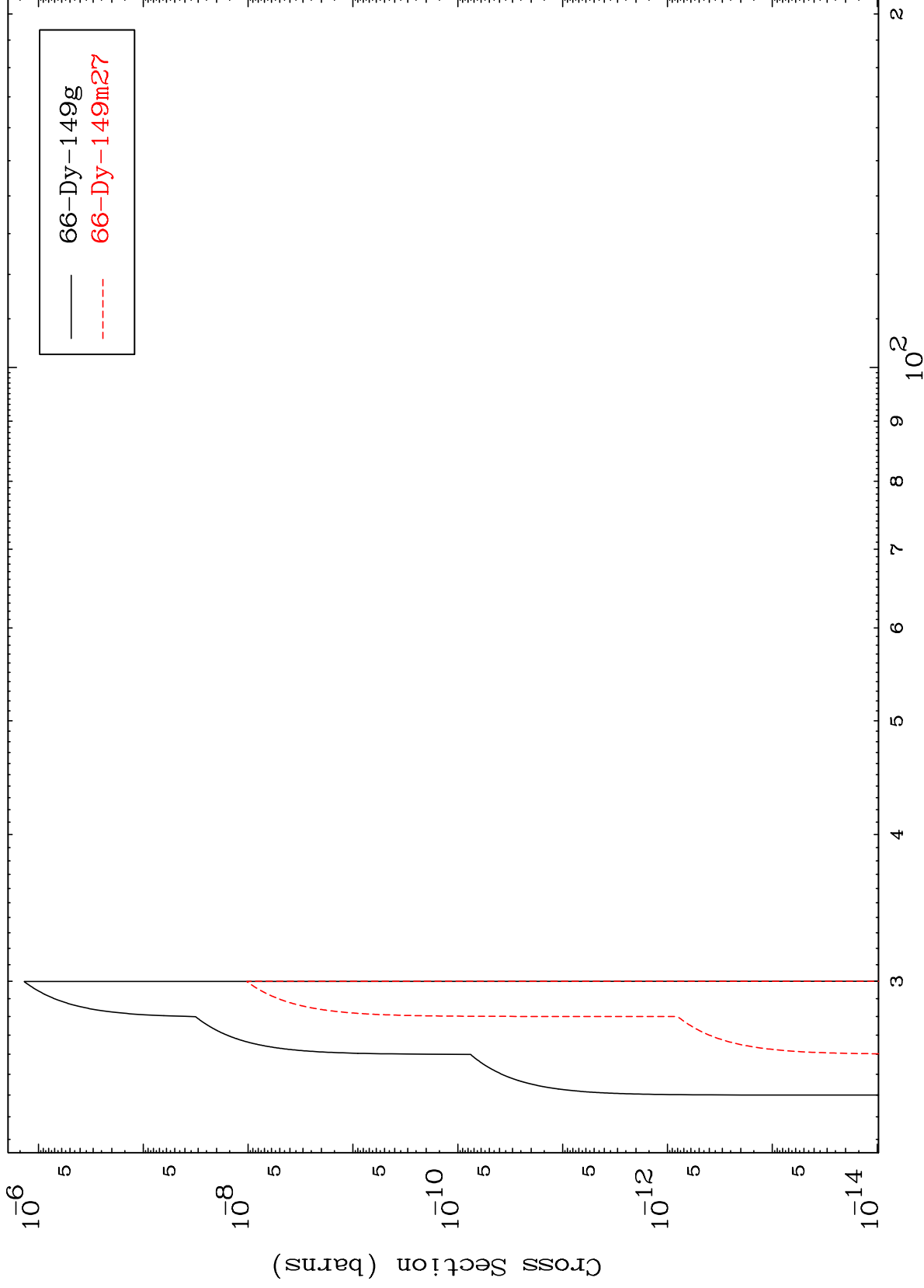
65-Tb-148m

MAT 6493

(n,n') d

65-Tb-148m

Radionuclide Production Cross Section



15

Incident Energy (MeV)

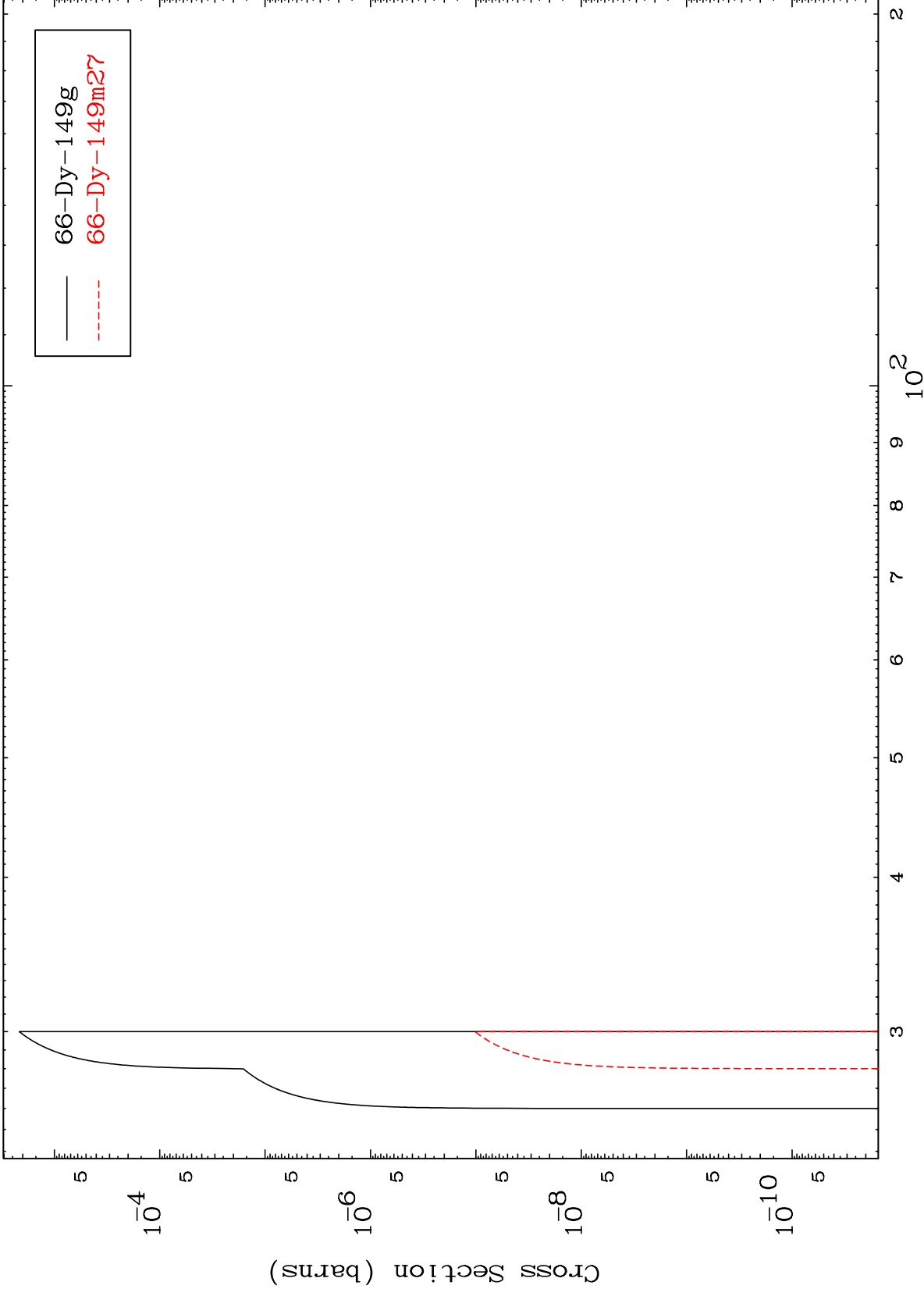
65-Tb-148m

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(n,2n) p

65-Tb-148m

Radionuclide Production Cross Section



16

Incident Energy (MeV)

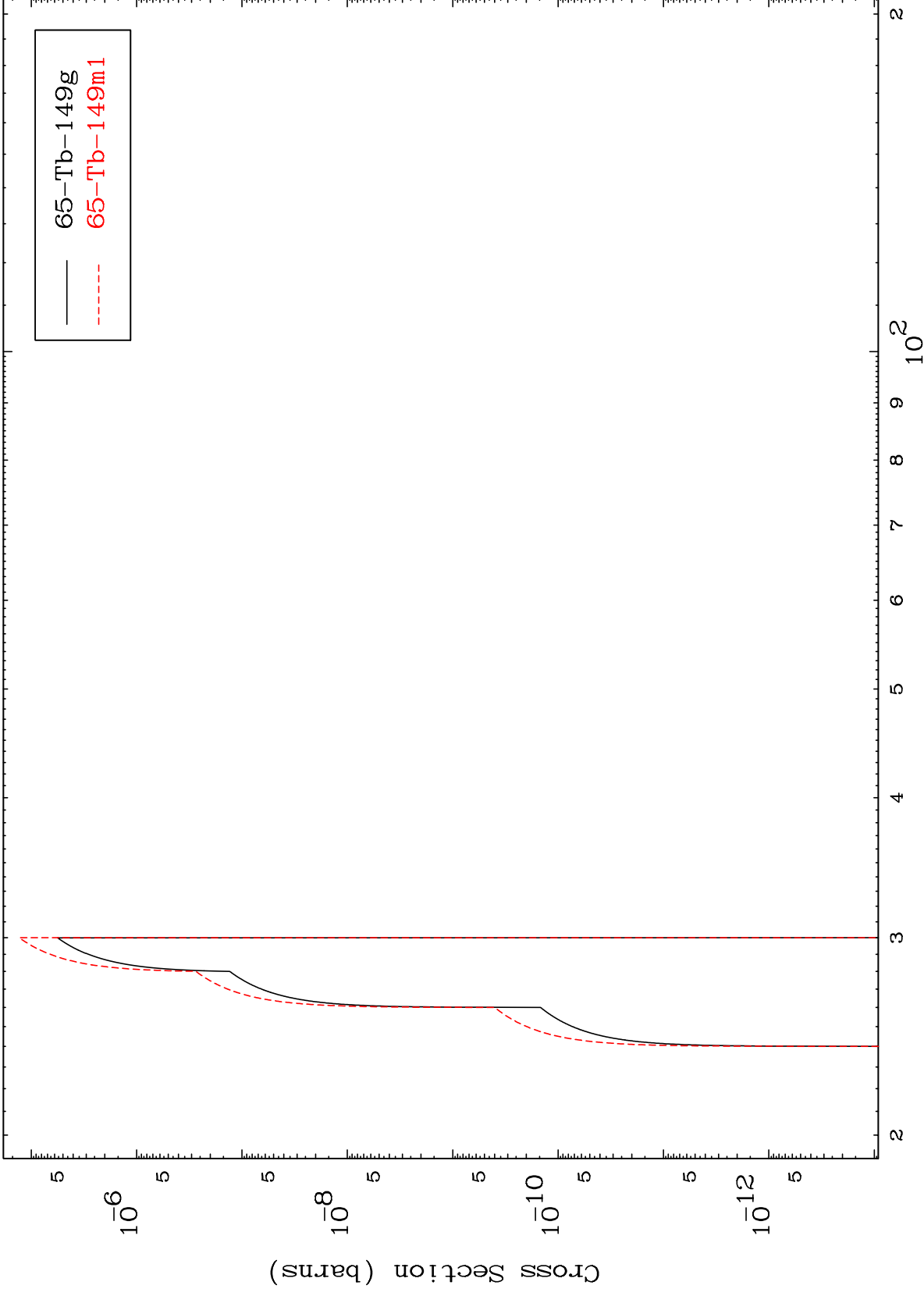
65-Tb-148m

MAT 6493

(n,2n) p

65-Tb-148m

Radionuclide Production Cross Section



17

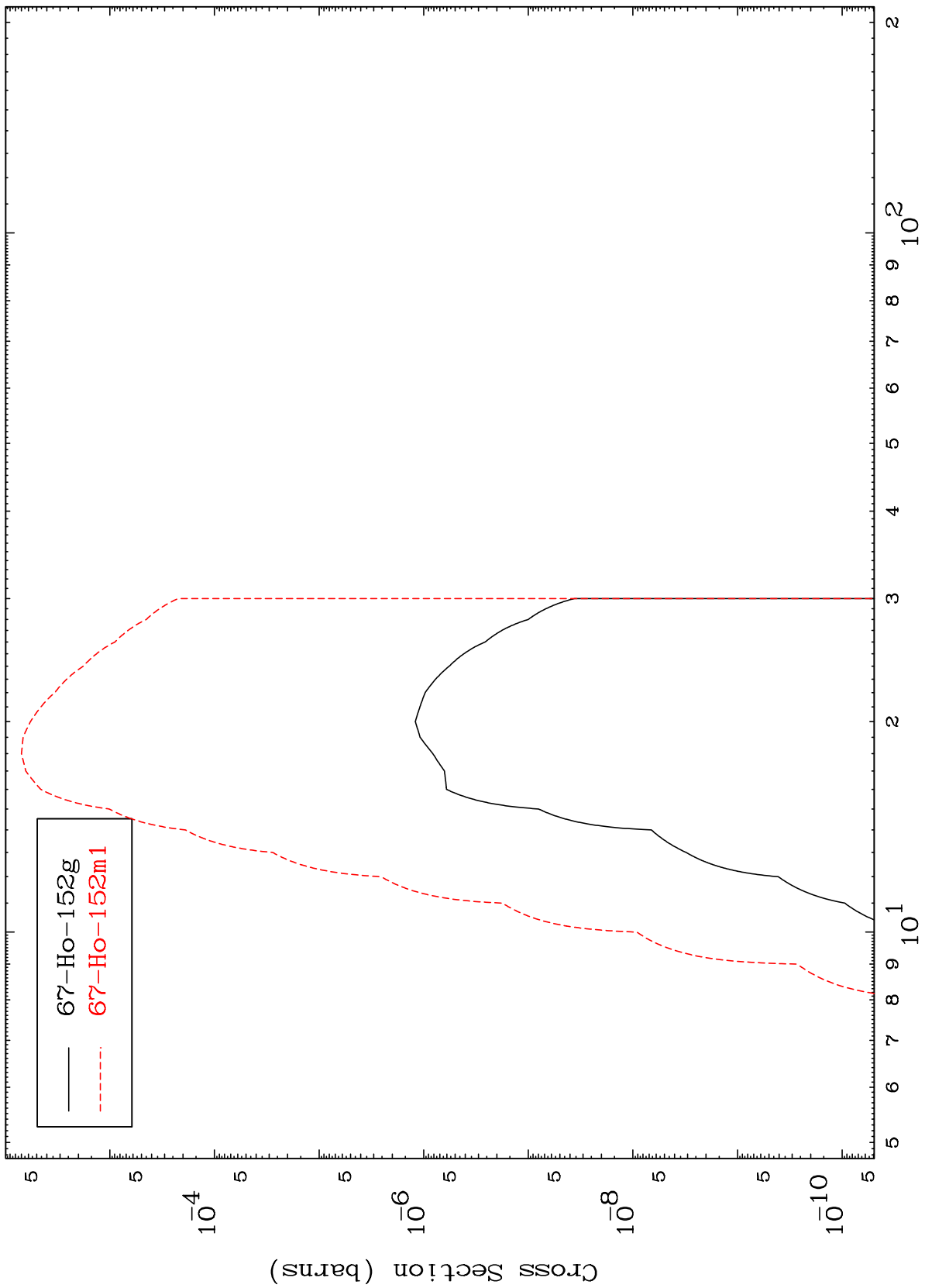
Incident Energy (MeV)

65-Tb-148m

MAT 6493

65-Tb-148m

Radionuclide Production Cross Section  
(n,  $\gamma$ )



18

Incident Energy (MeV)

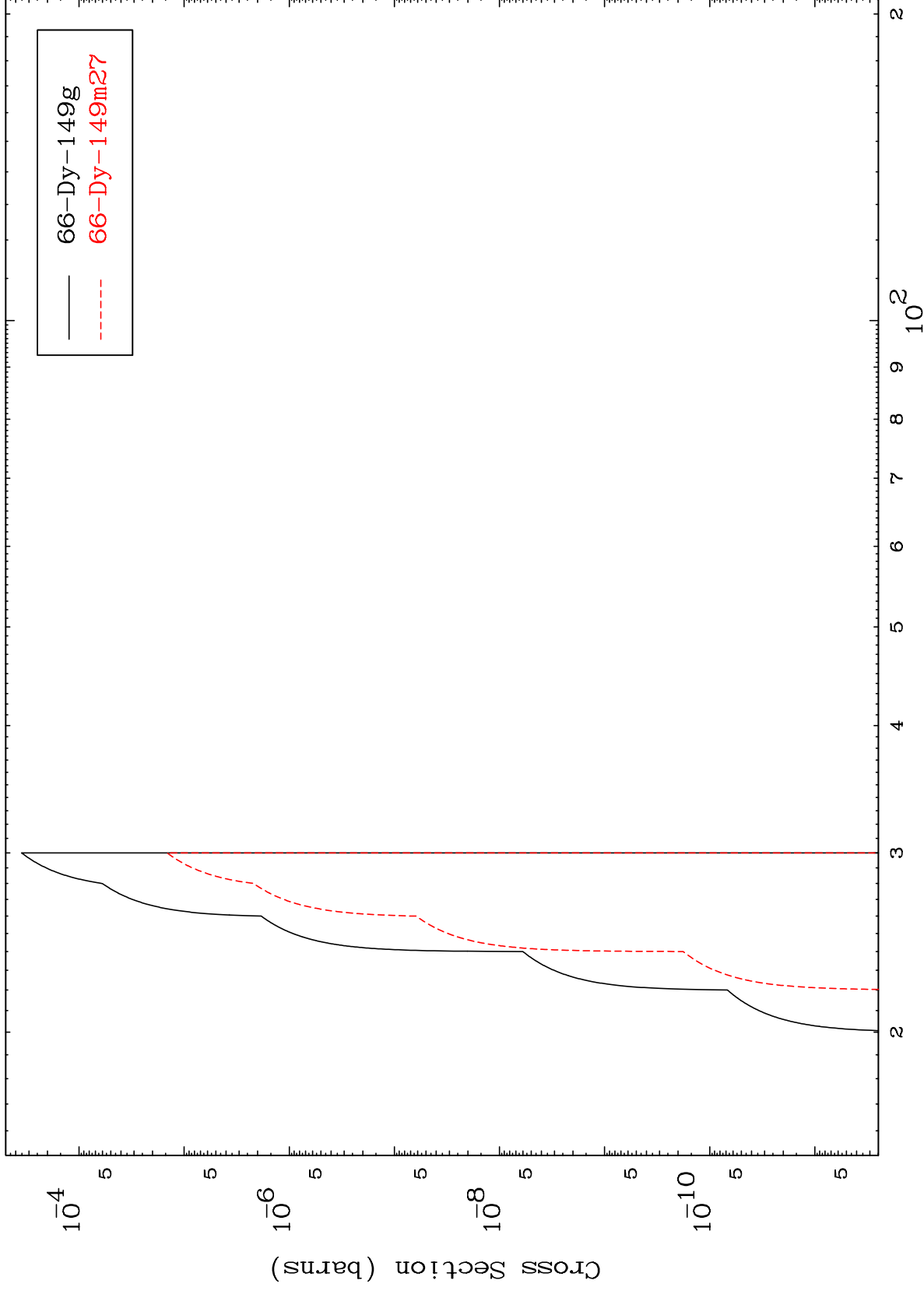
65-Tb-148m

MAT 6493

(n, t)

65-Tb-148m

Radionuclide Production Cross Section



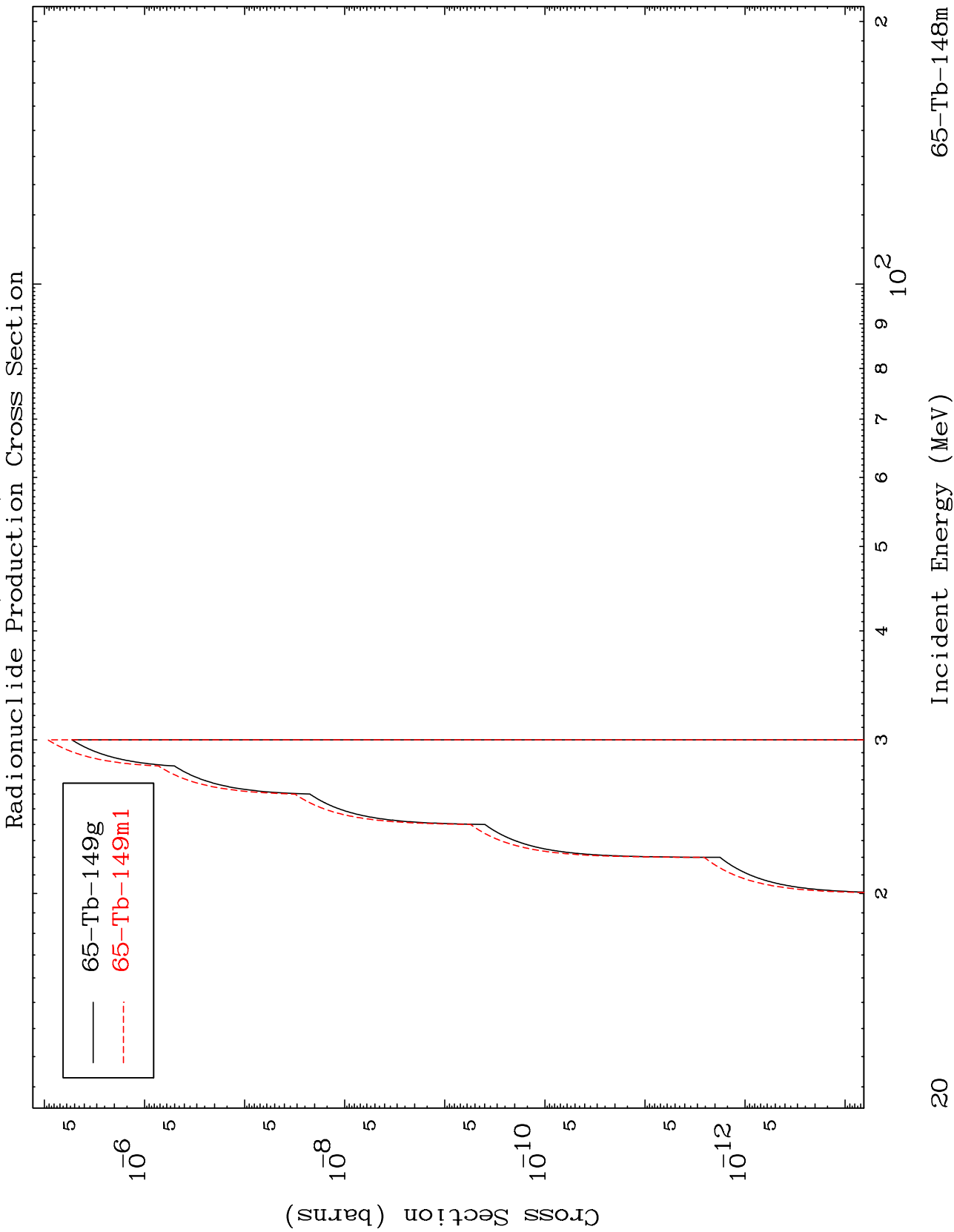
19

Incident Energy (MeV)

65-Tb-148m

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65-Tb-148m

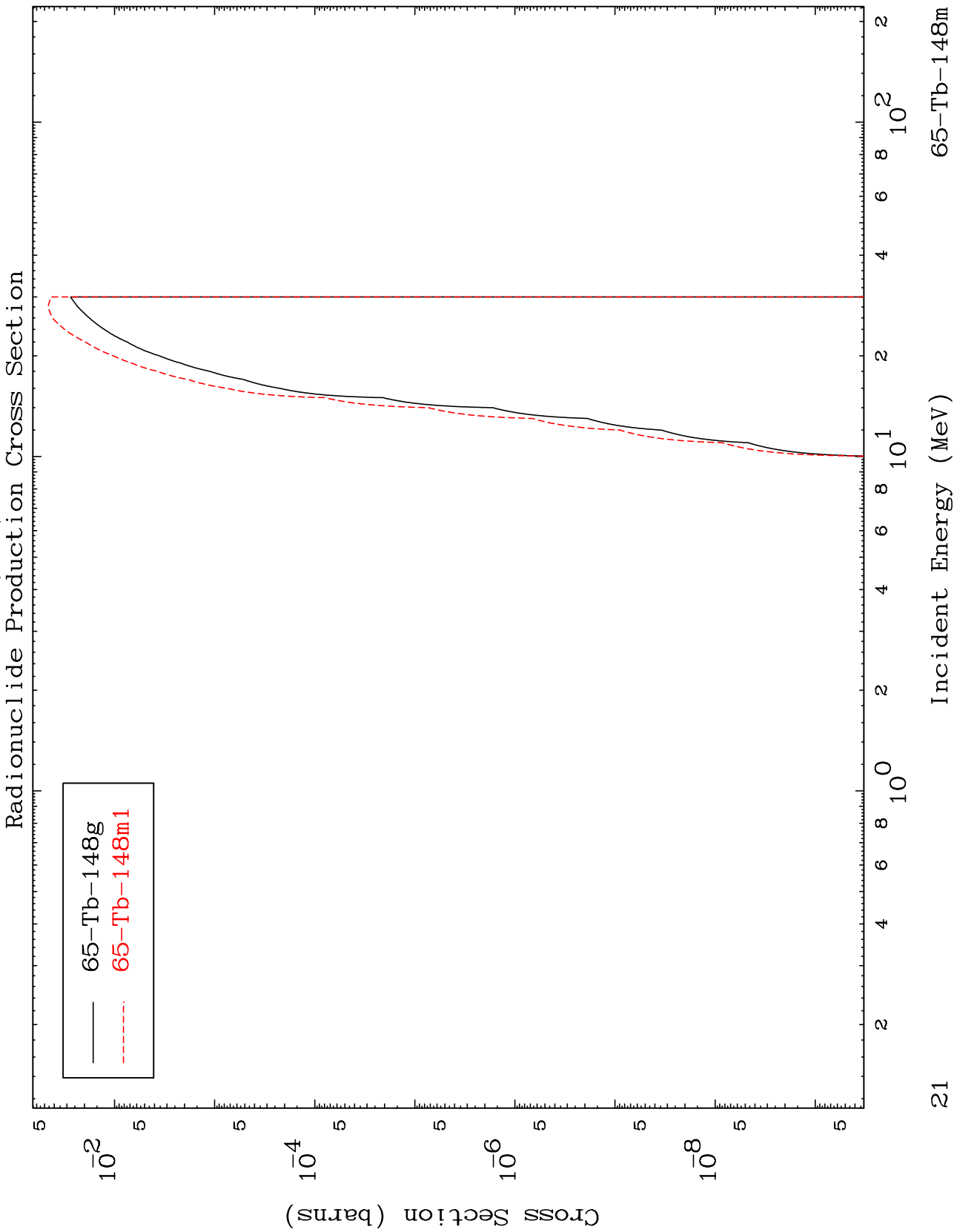


20

65-Tb-148m

MAT 6493

65-Tb-148m

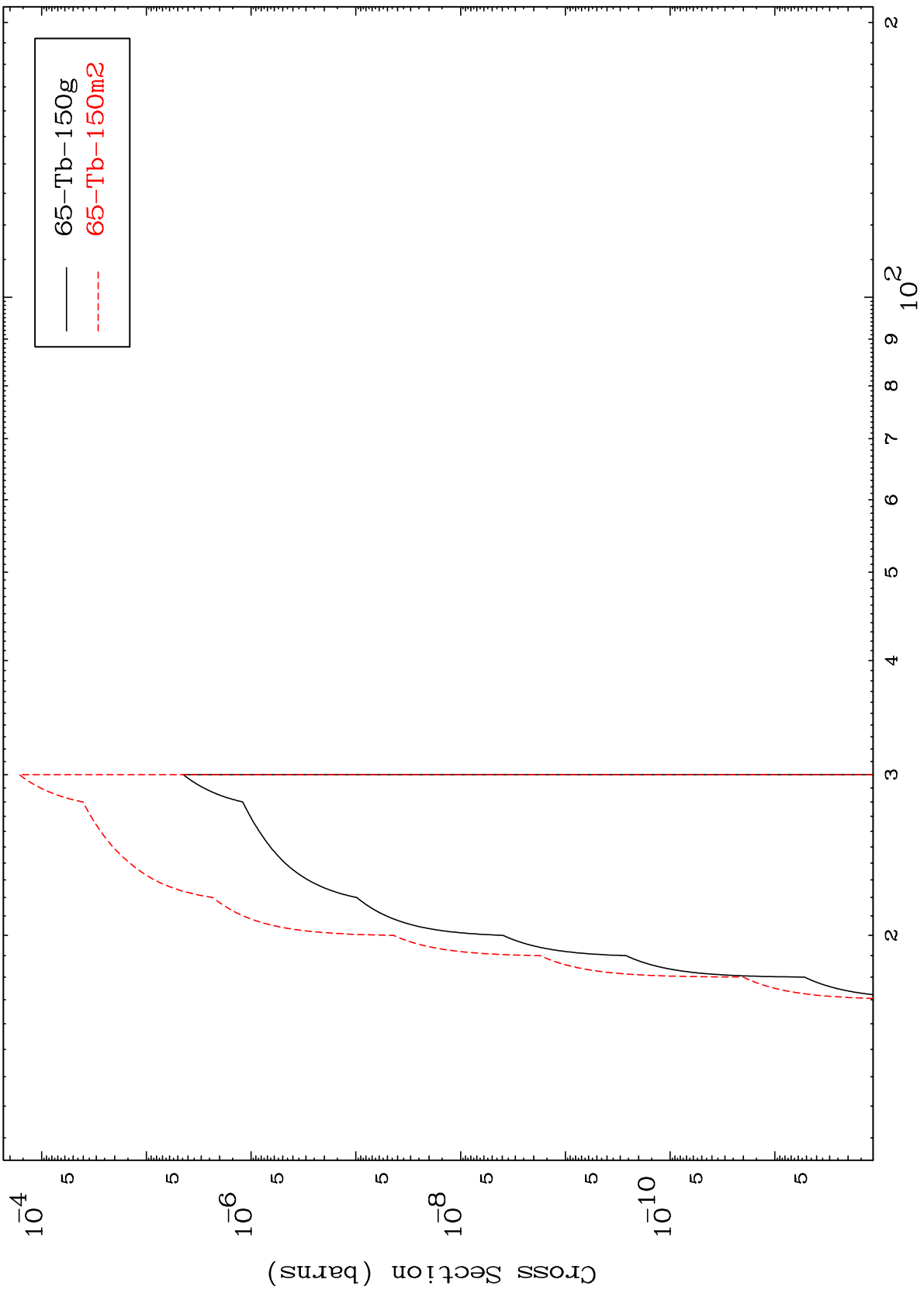


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65-Tb-148m

(n,2p)

Radionuclide Production Cross Section



22

65-Tb-148m

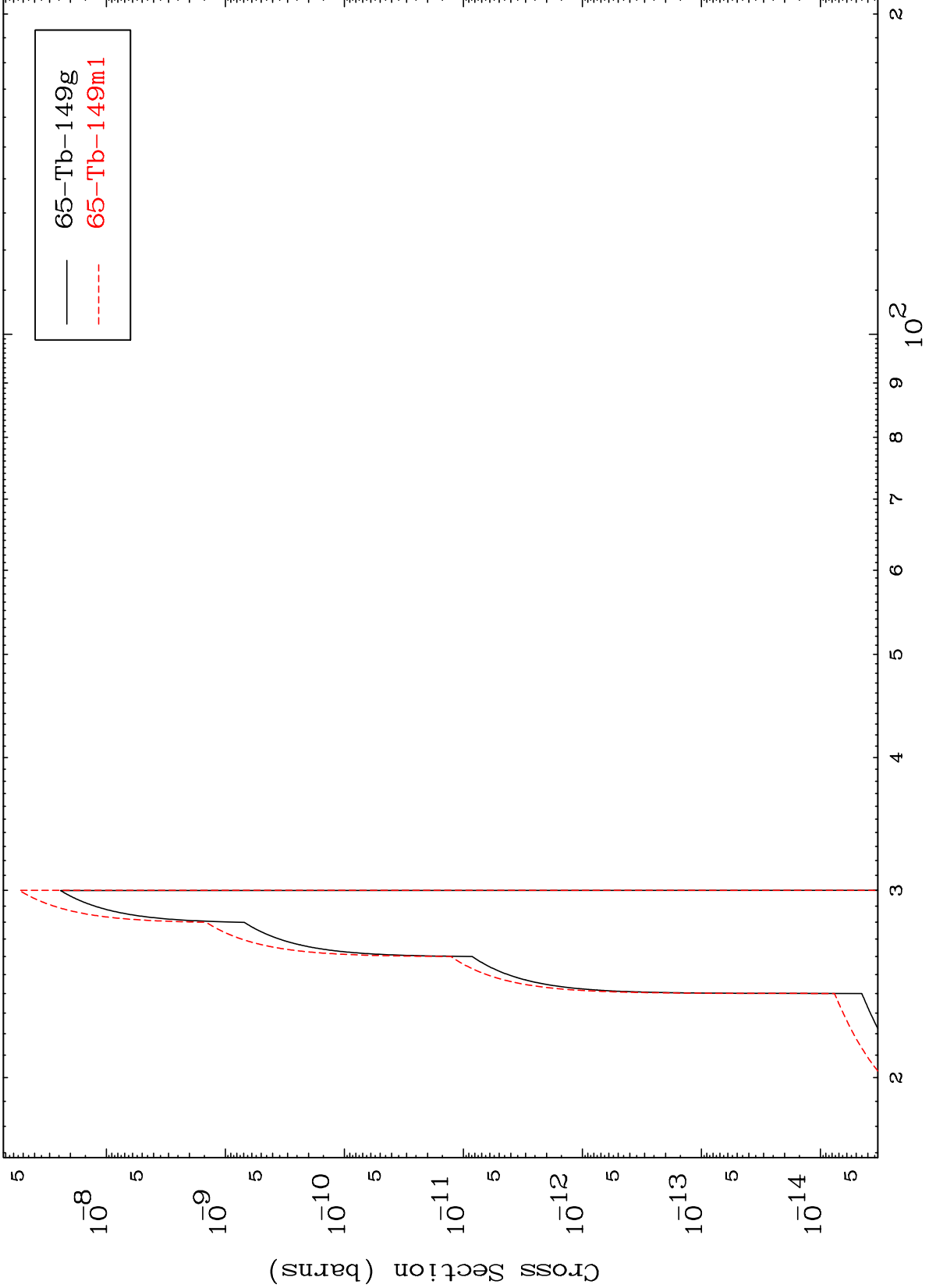
Incident Energy (MeV)

MAT 6493

(n,p) d

65-Tb-148m

Radionuclide Production Cross Section



23

Incident Energy (MeV)

65-Tb-148m