

Program EVALPLOT  
(Version 2021-1)

by

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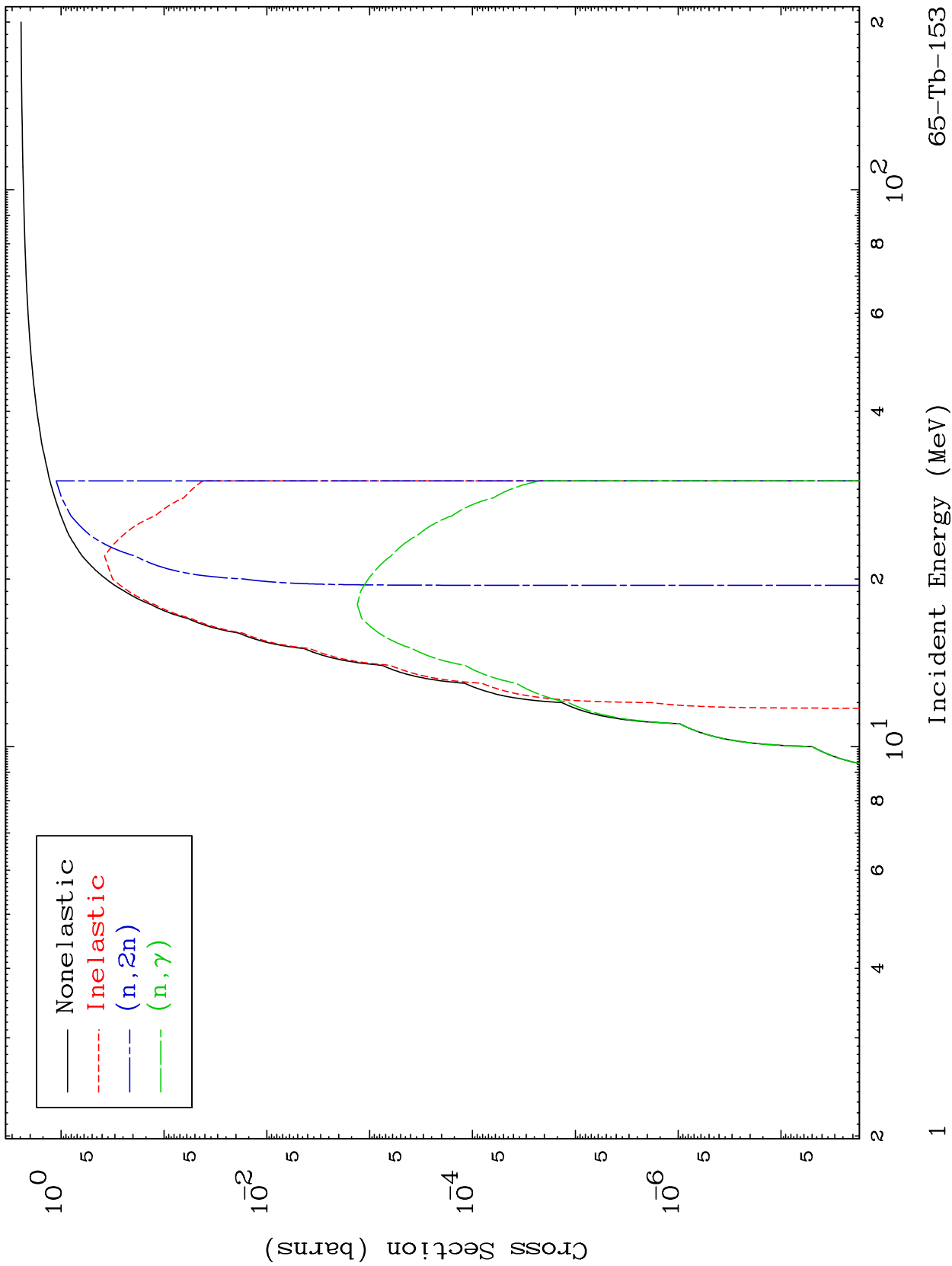
Press Mouse Button to Start

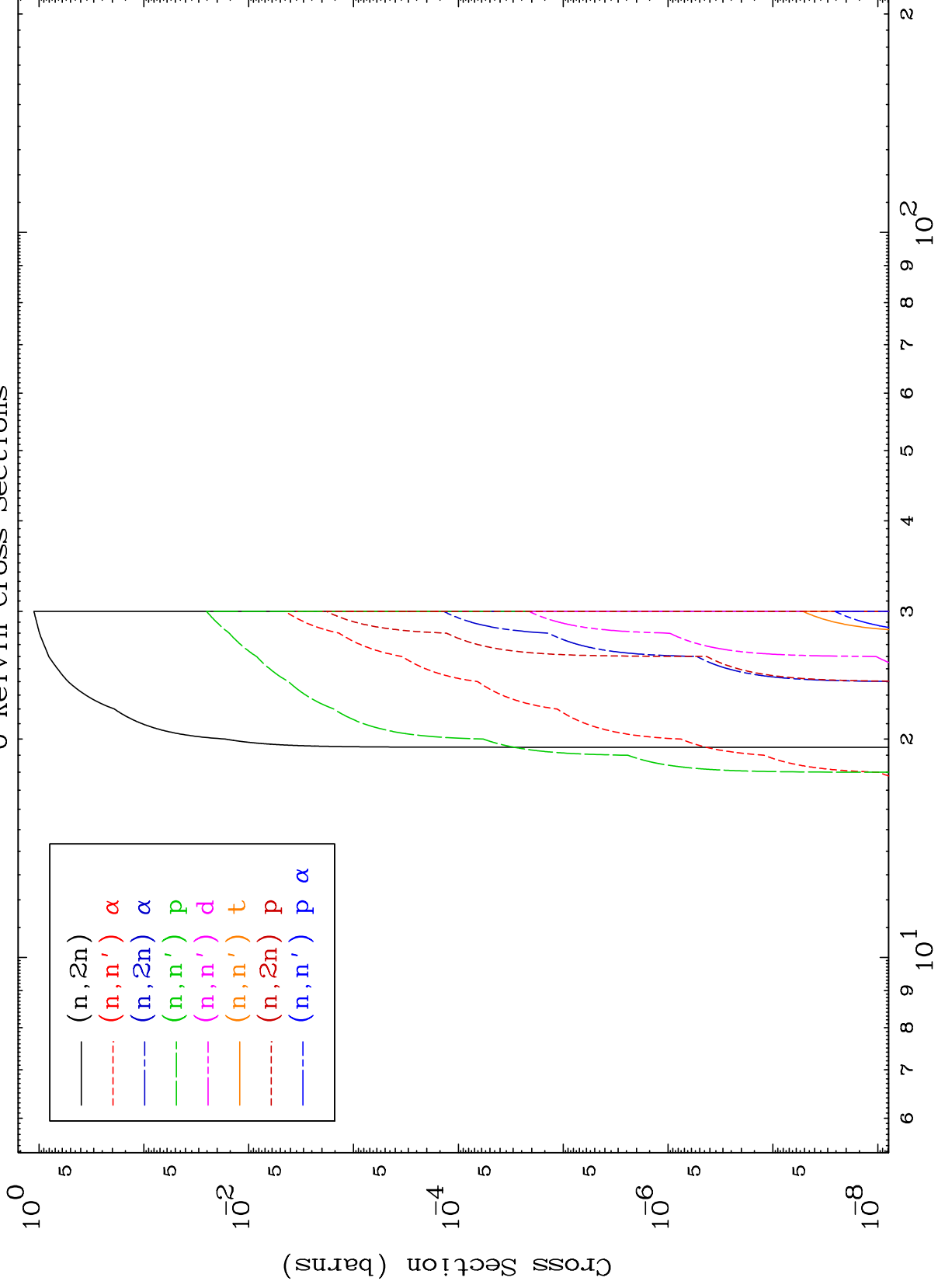
MAT 6507

0 Kelvin

Major Cross Sections

65-Tb-153

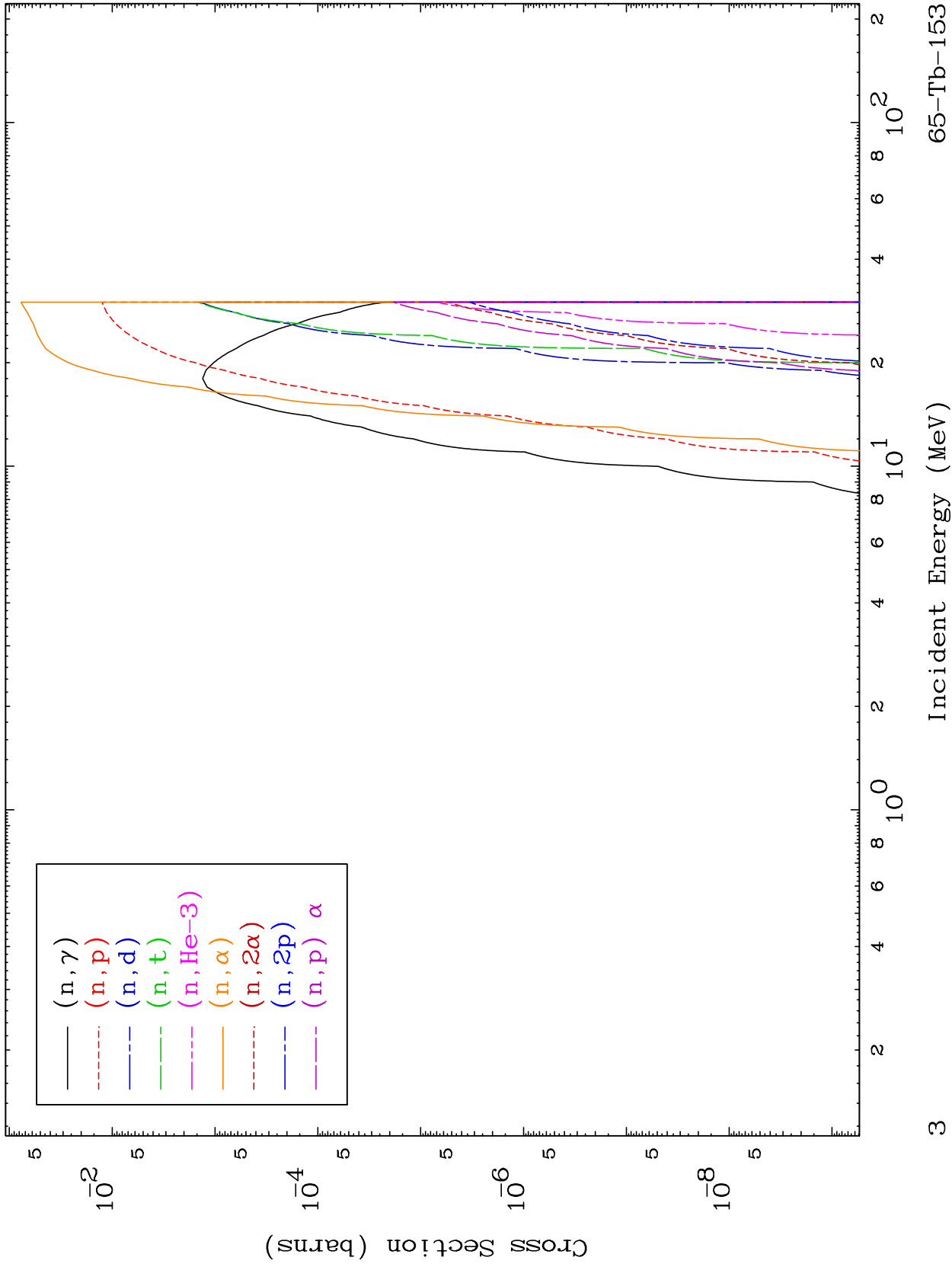


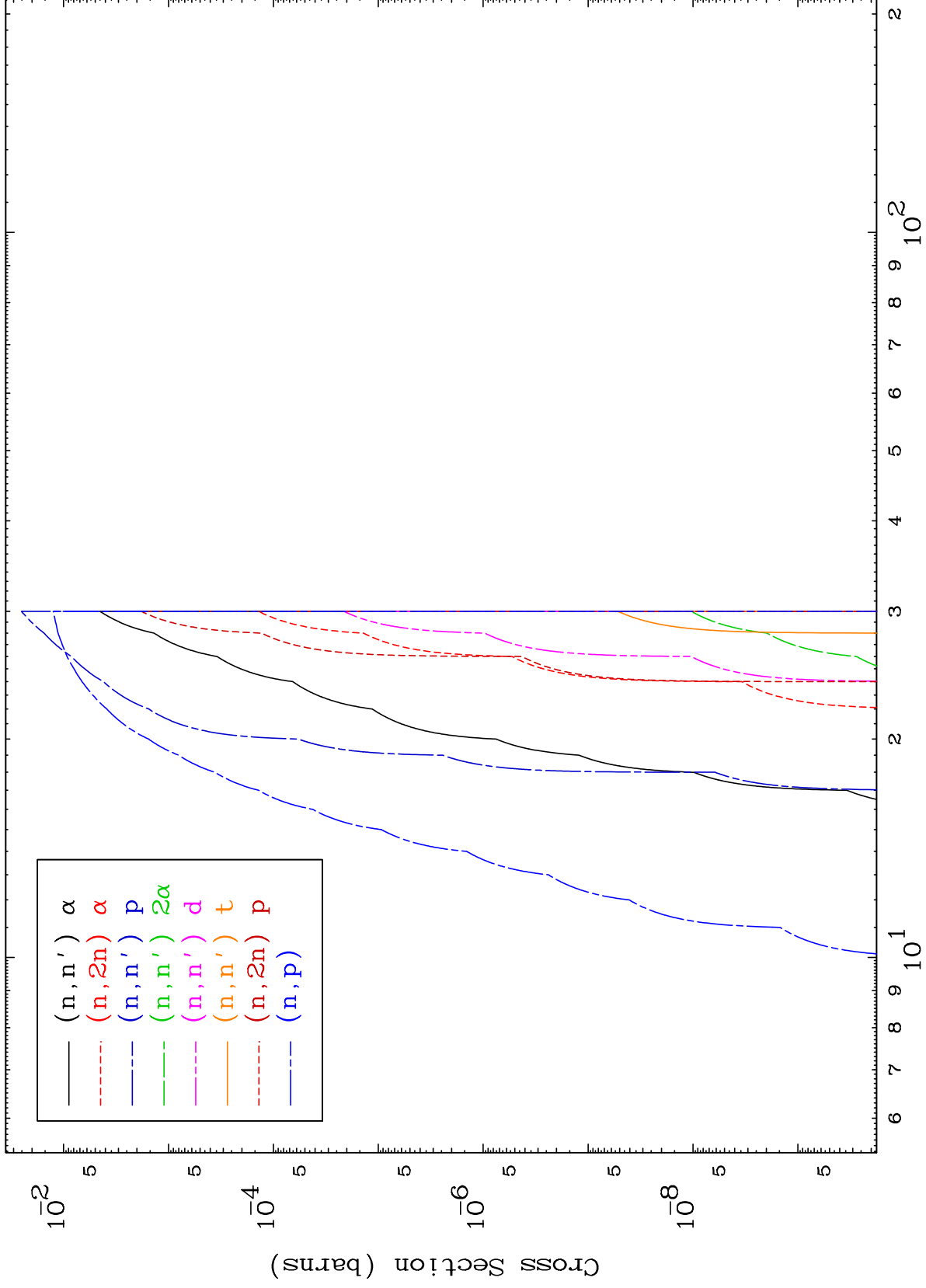


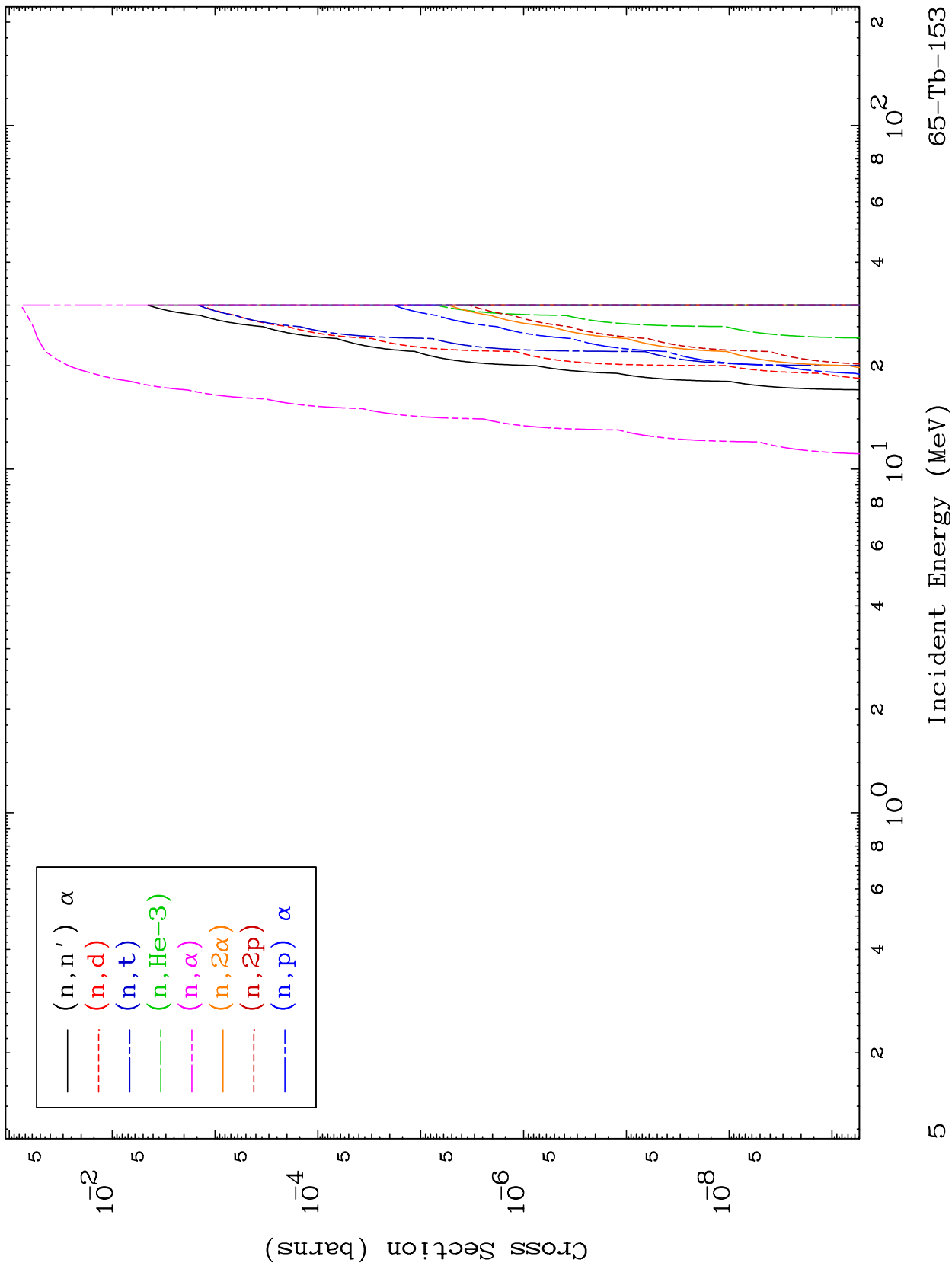
MAT 6507

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

65-Tb-153



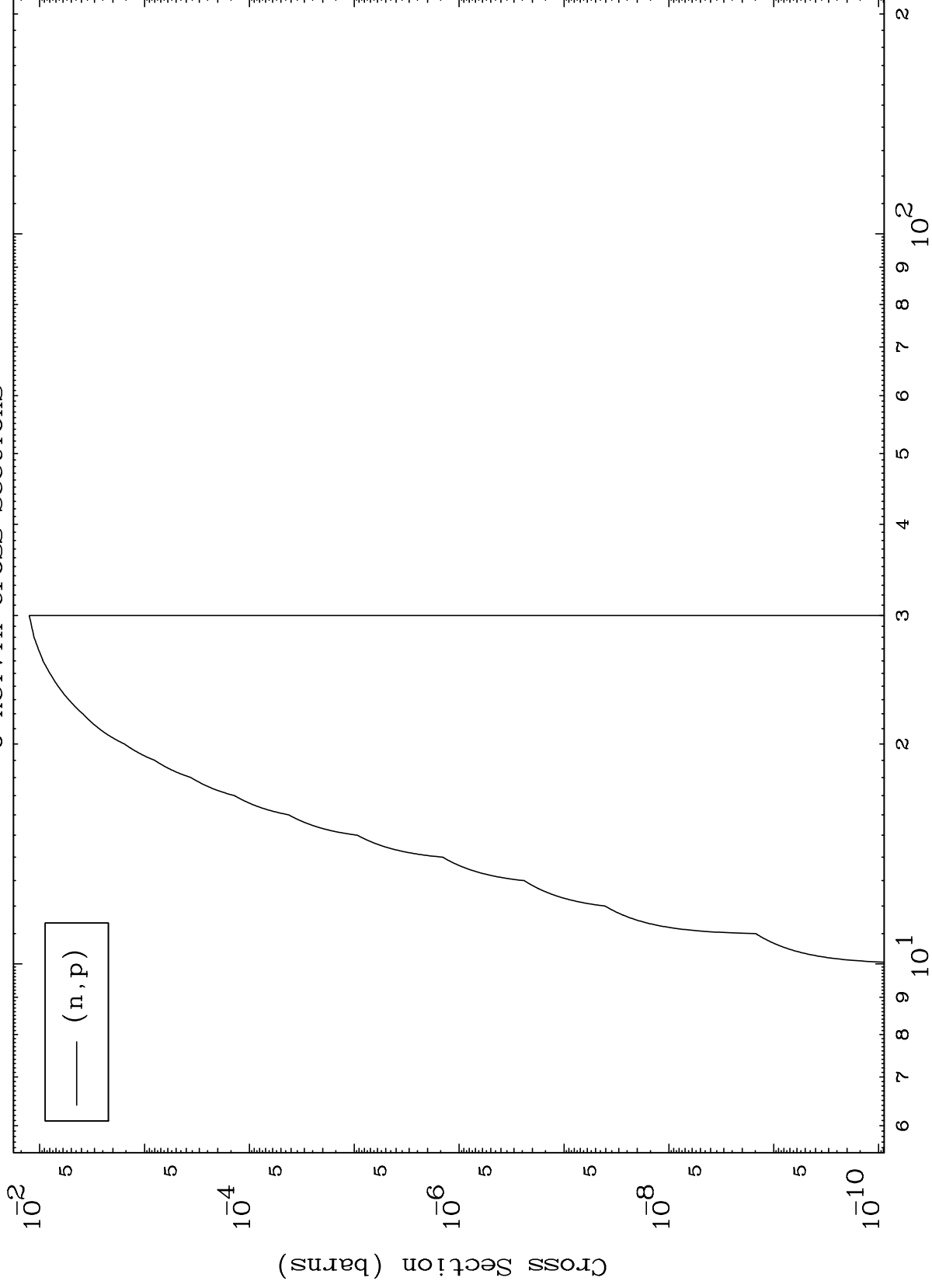




MAT 6507

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

65-Tb-153



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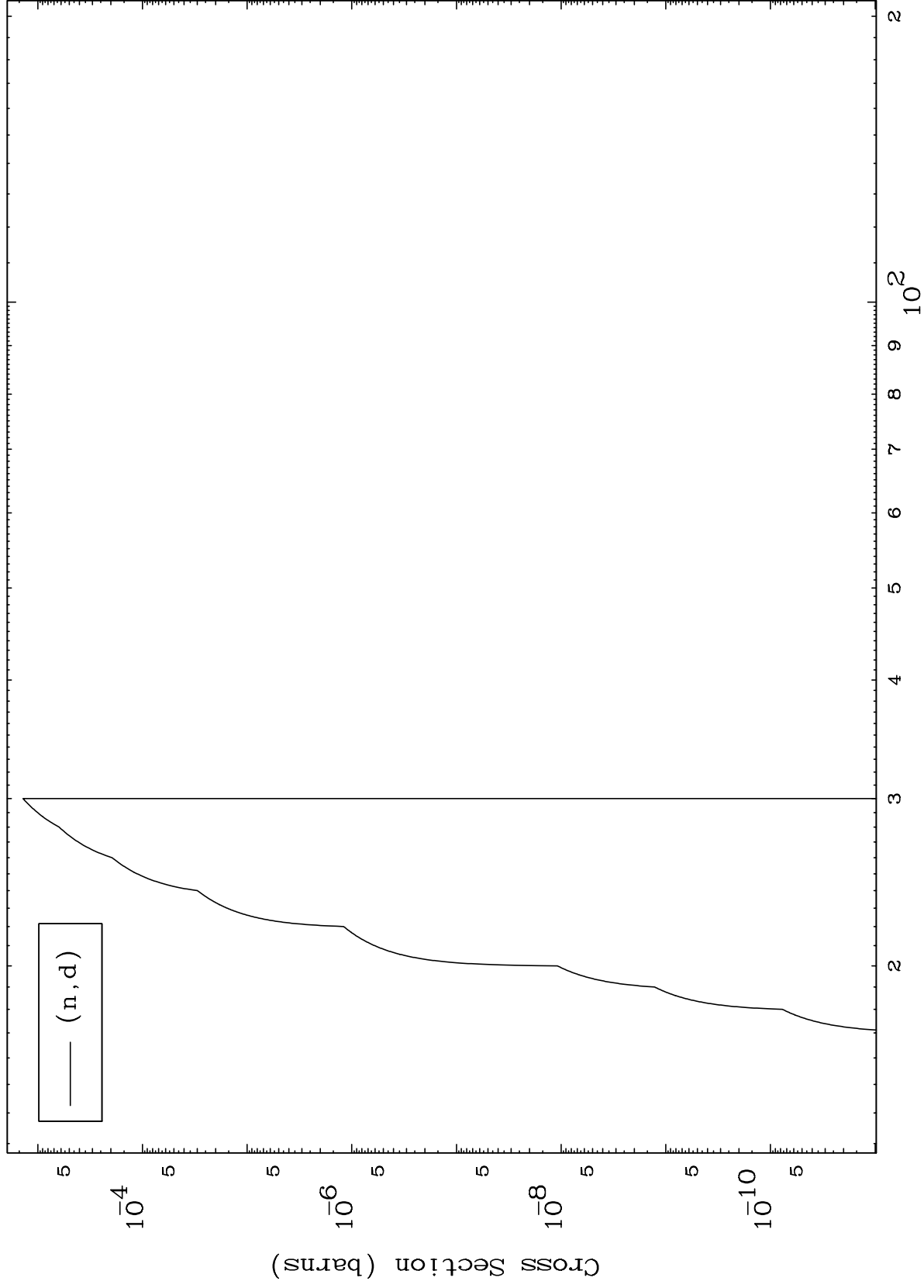
Incident Energy (MeV)

65-Tb-153

MAT 6507

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

65-Tb-153



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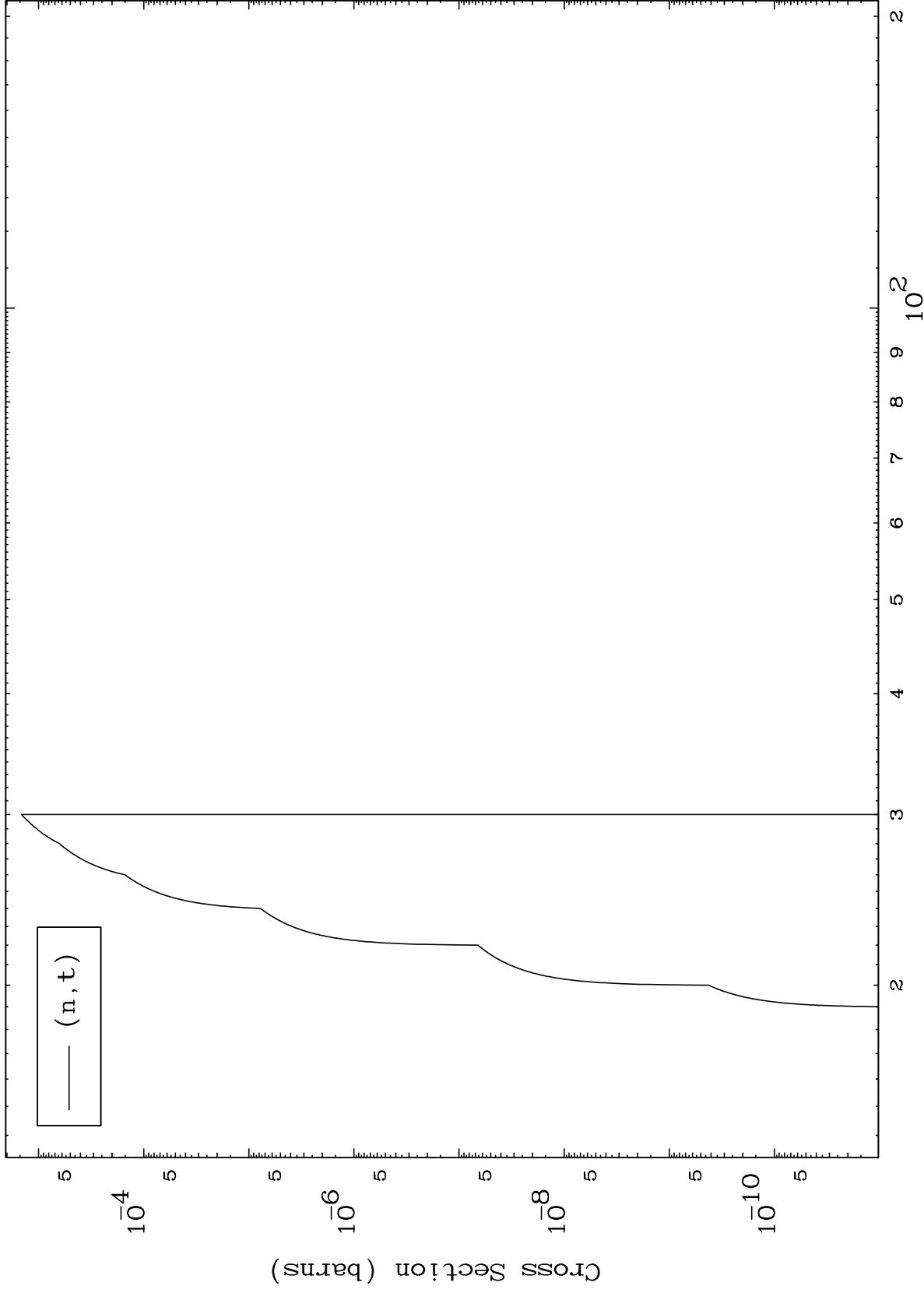
Incident Energy (MeV)

65-Tb-153

MAT 6507

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

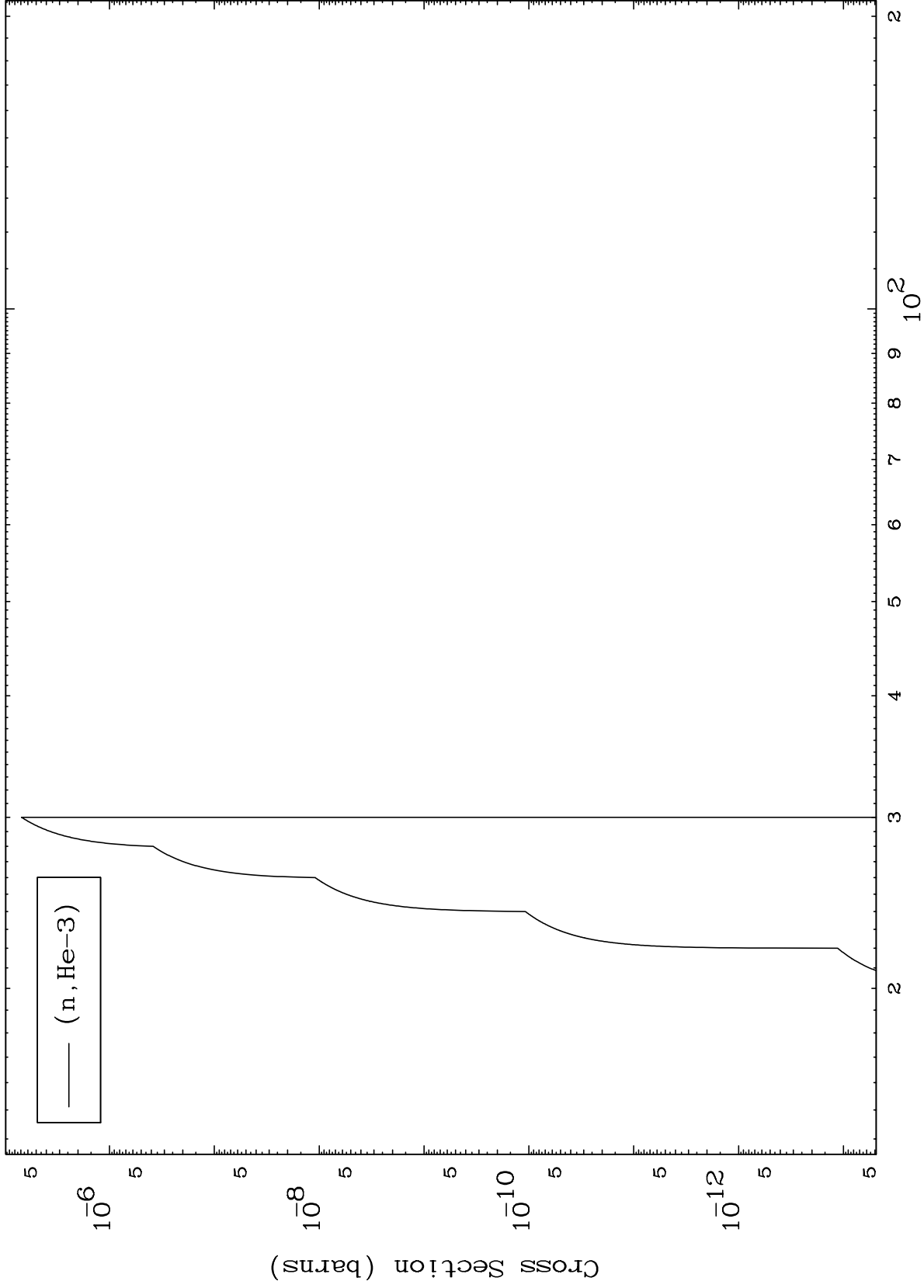
65-Tb-153



MAT 6507

( $\alpha$ , He3) Levels  
0 Kelvin Cross Sections

65-Tb-153



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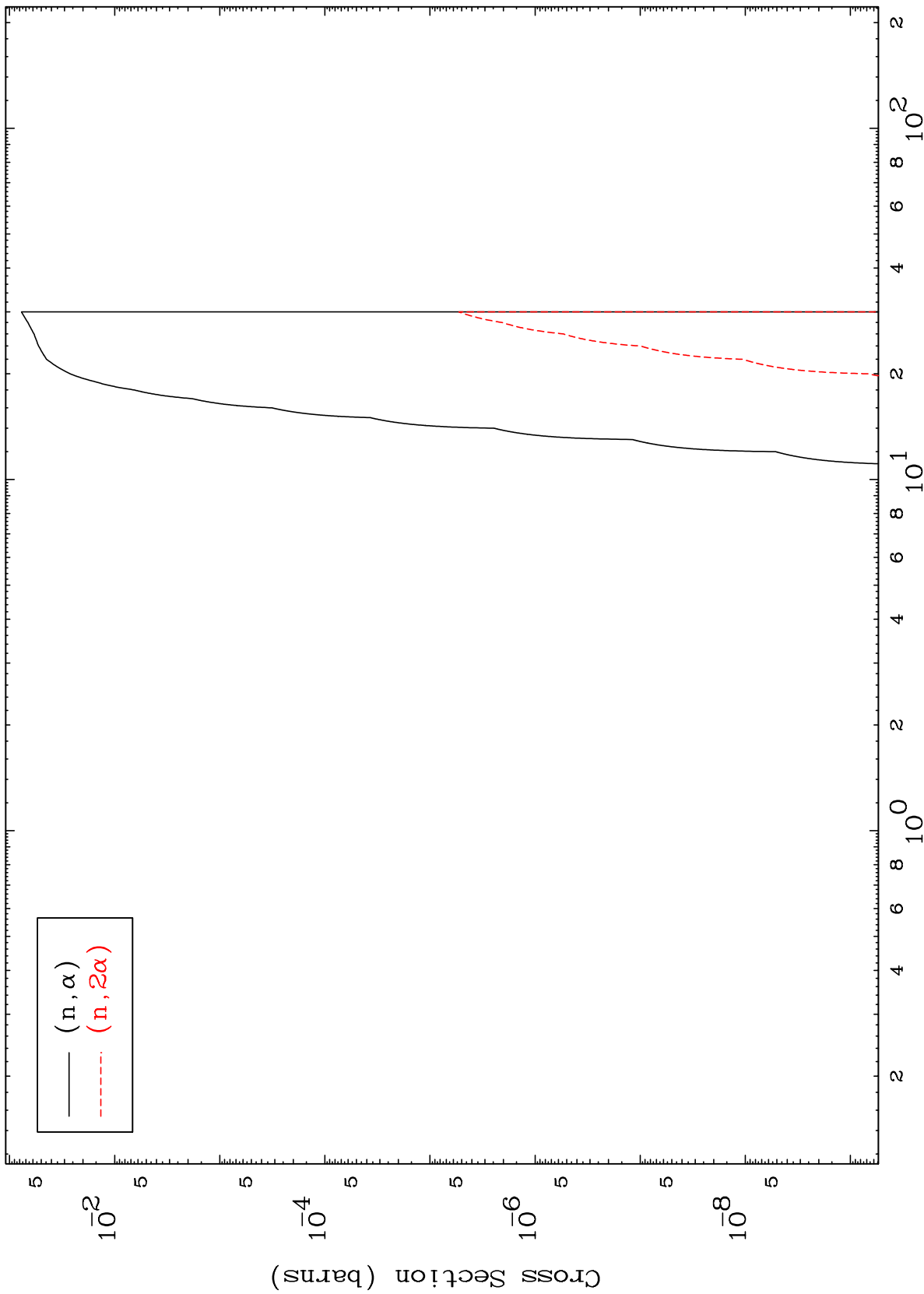
Incident Energy (MeV)

65-Tb-153

MAT 6507

65-Tb-153

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections



65-Tb-153

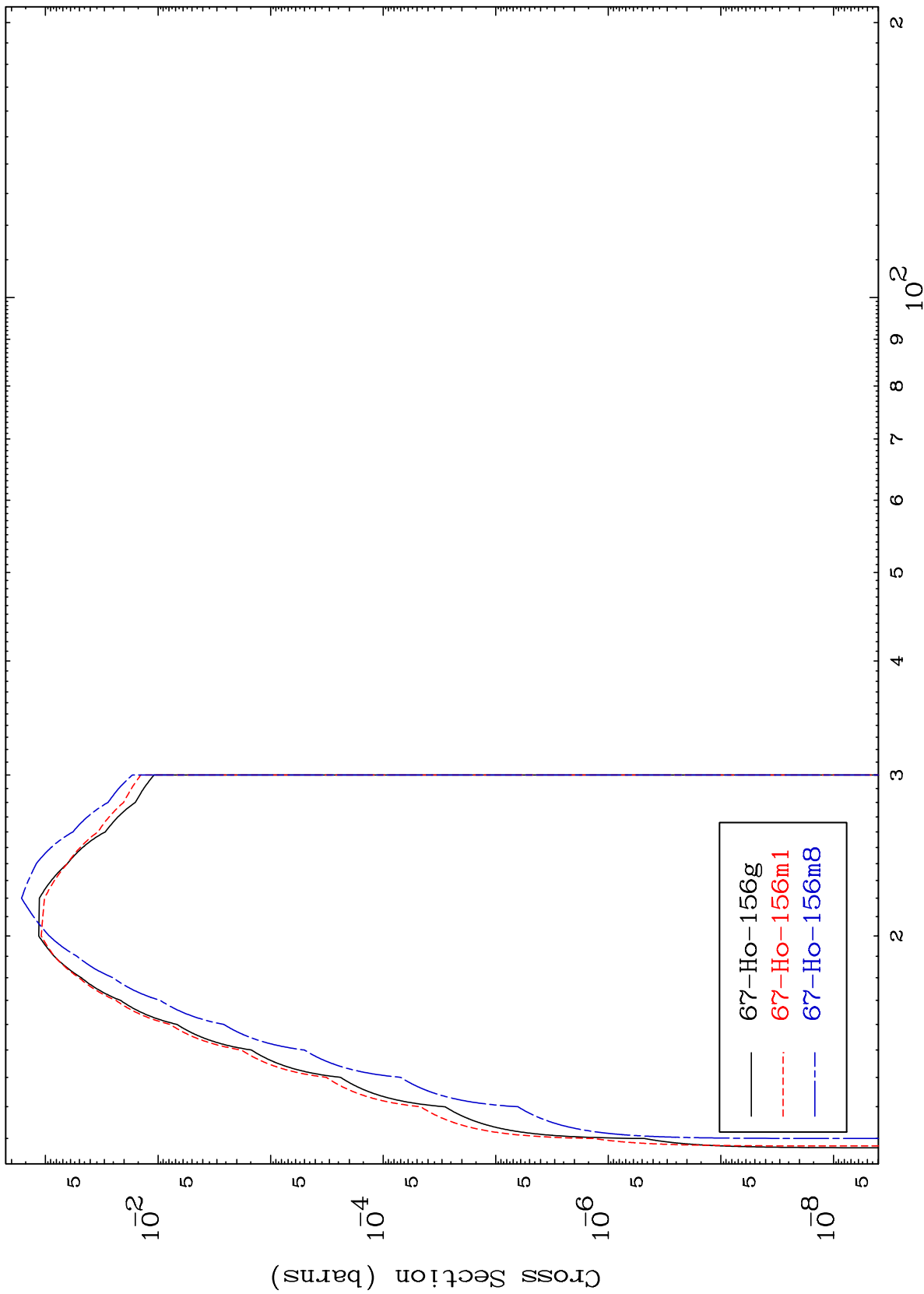
Incident Energy (MeV)

10

MAT 6507

65-Tb-153

Inelastic  
Radionuclide Production Cross Section



65-Tb-153

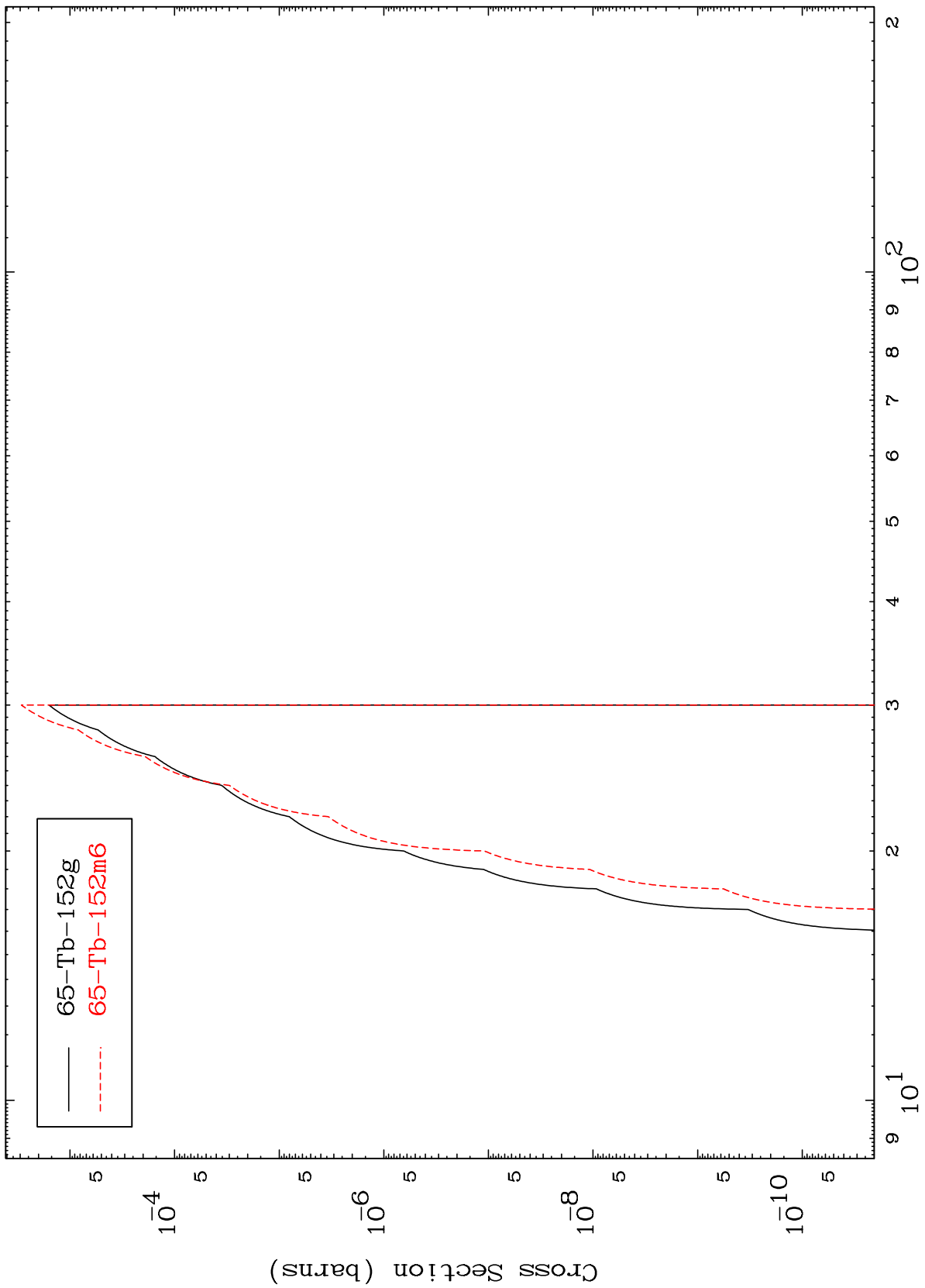
Incident Energy (MeV)

MAT 6507

$(n, n') \alpha$

65-Tb-153

Radionuclide Production Cross Section



12

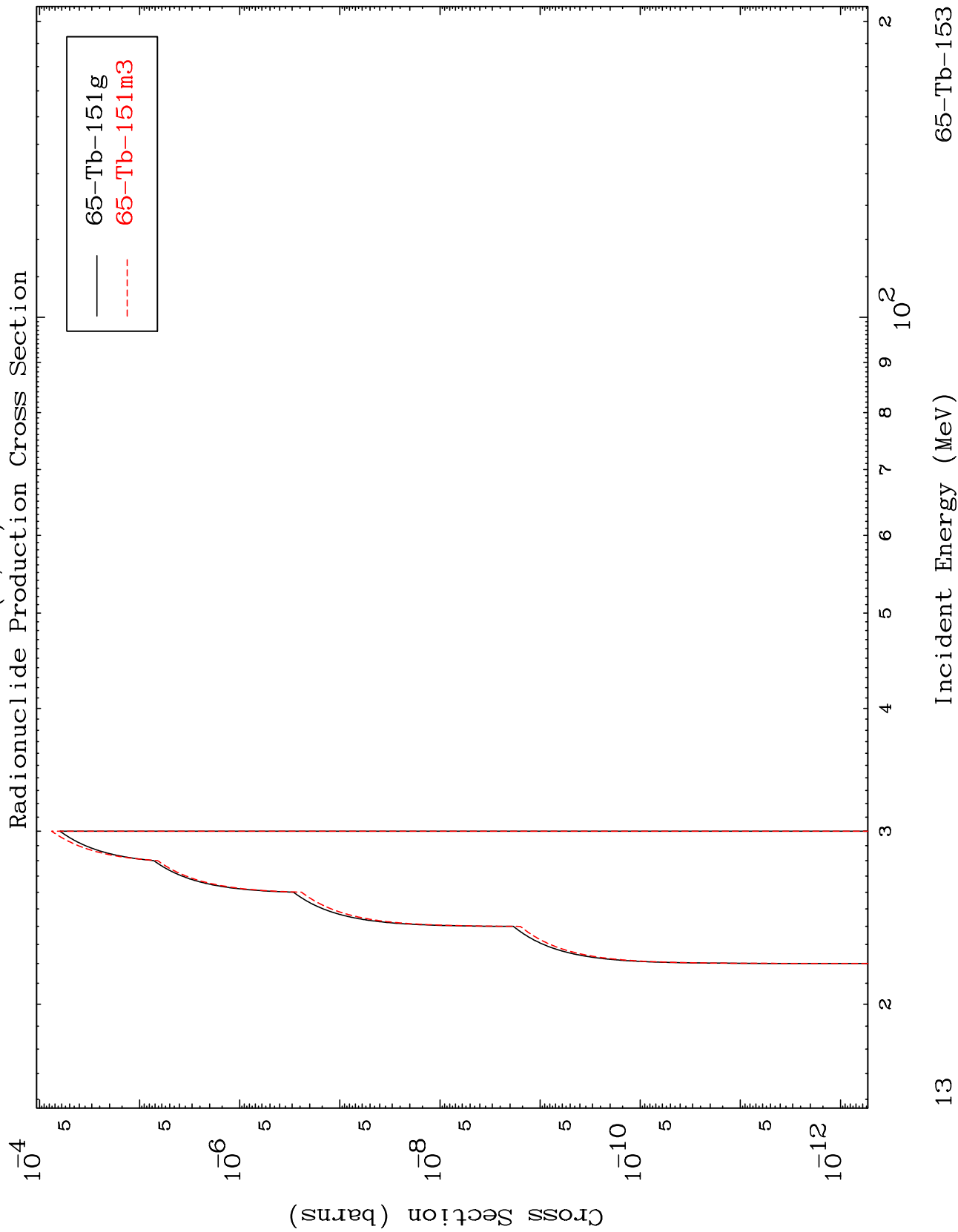
Incident Energy (MeV)

65-Tb-153

MAT 6507

(n,2n)  $\alpha$

65-Tb-153



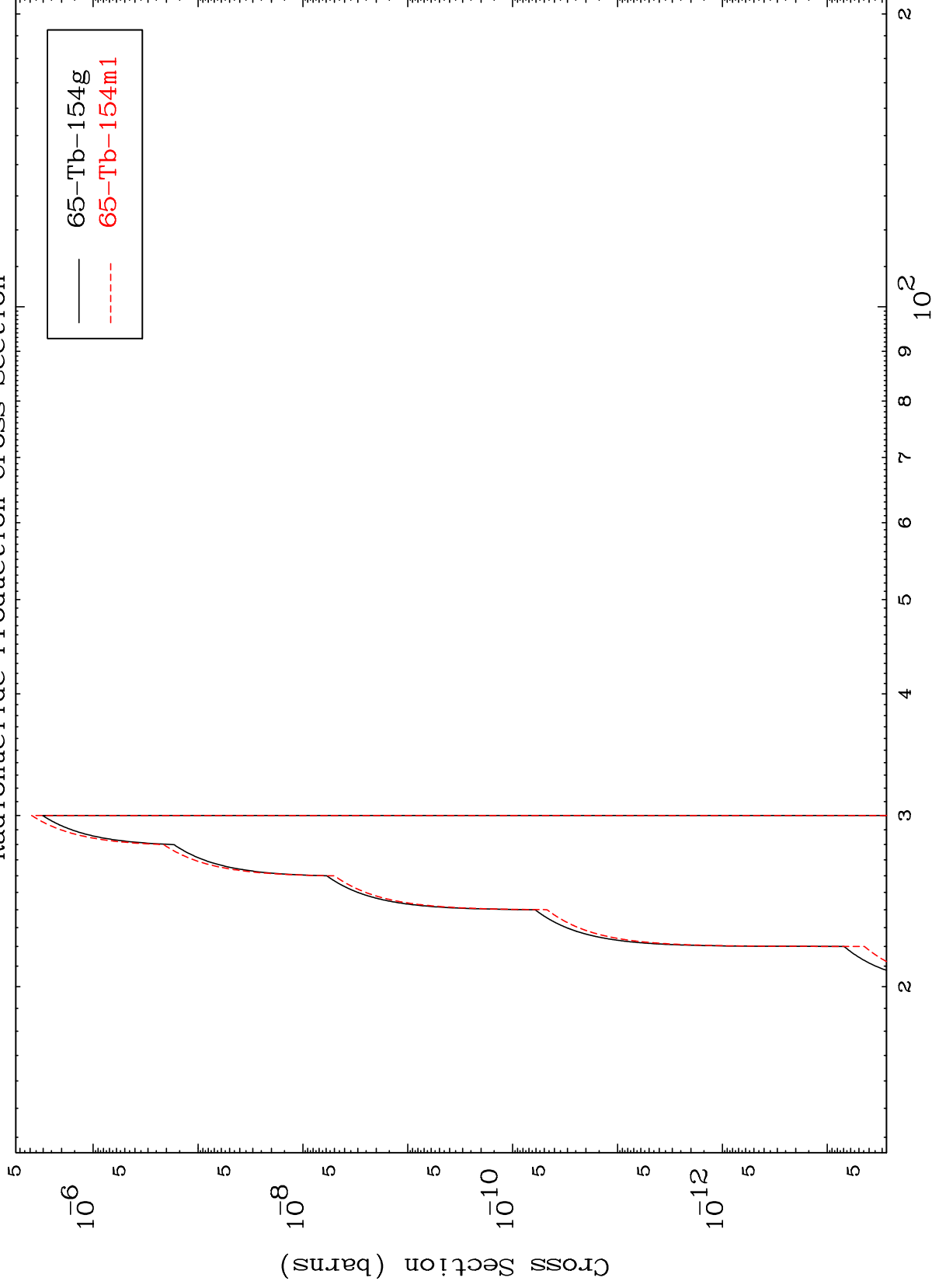
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MAT 6507

(n,He-3)

65-Tb-153

Radionuclide Production Cross Section



14

Incident Energy (MeV)

65-Tb-153