

Program EVALPLOT  
(Version 2021-1)

by

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(Present Contact Information)

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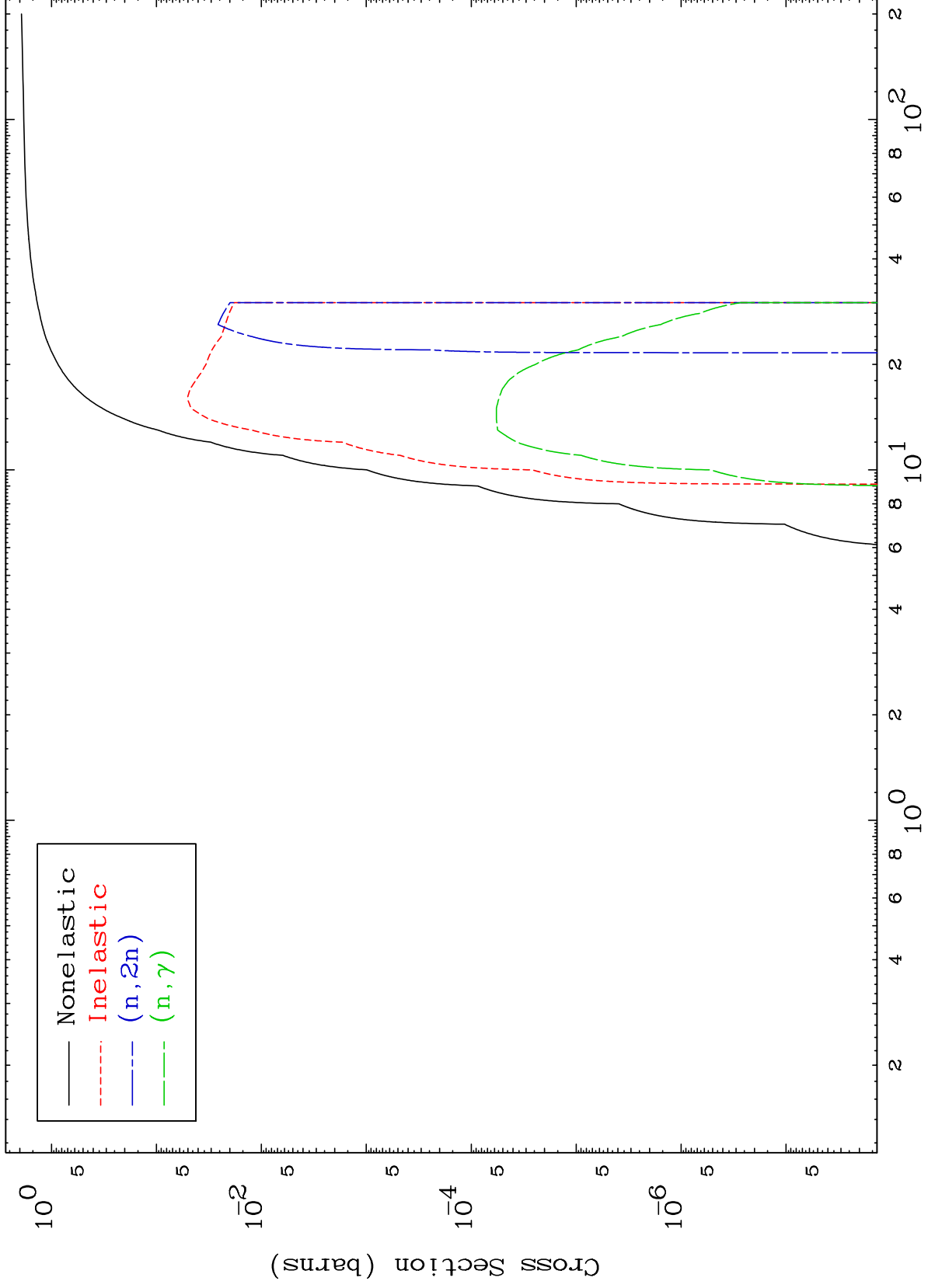
Press Mouse Button to Start

MAT 4301

$\alpha$  Major

43-Tc-91

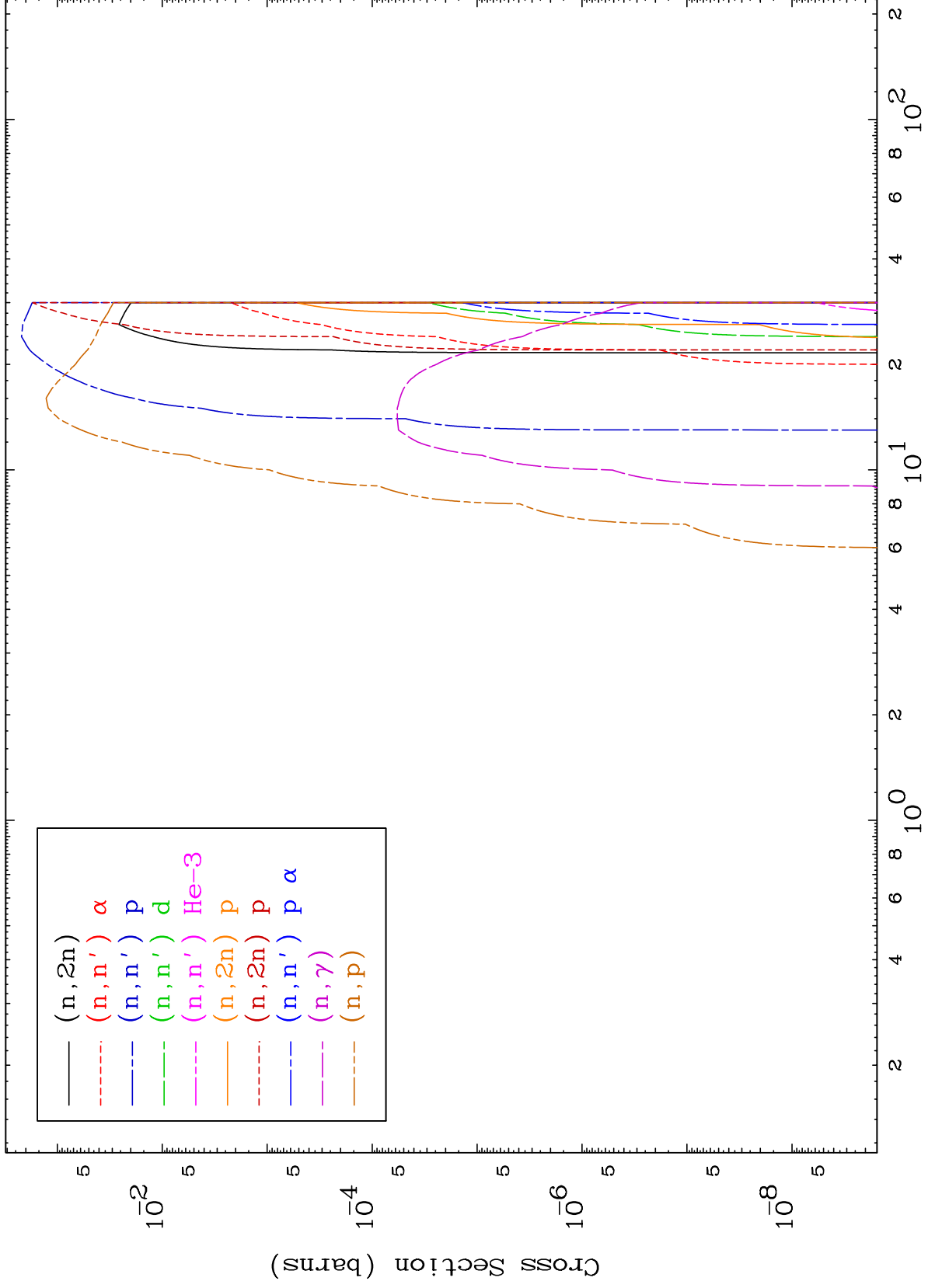
0 Kelvin Cross Sections



MAT 4301

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

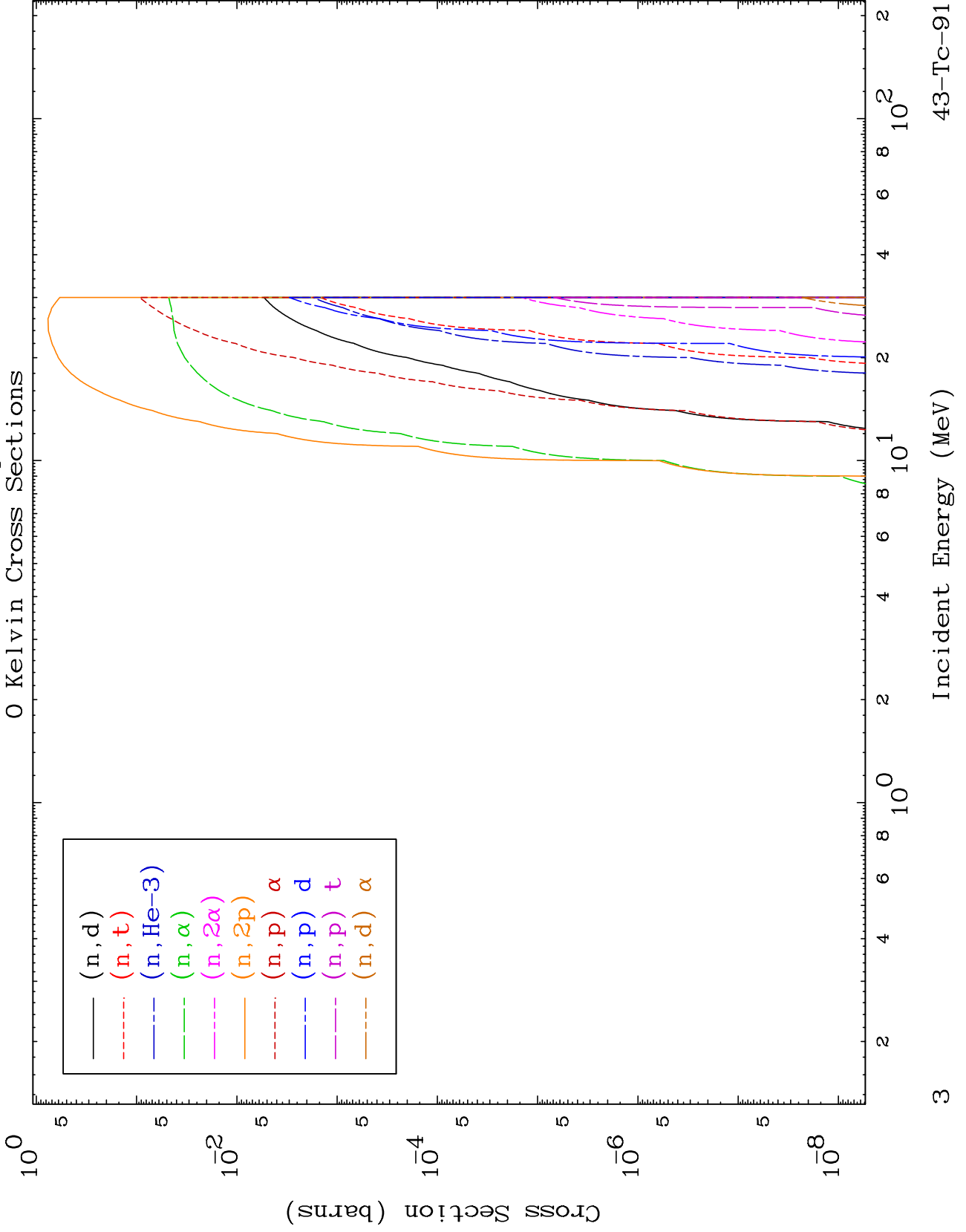
43-Tc-91

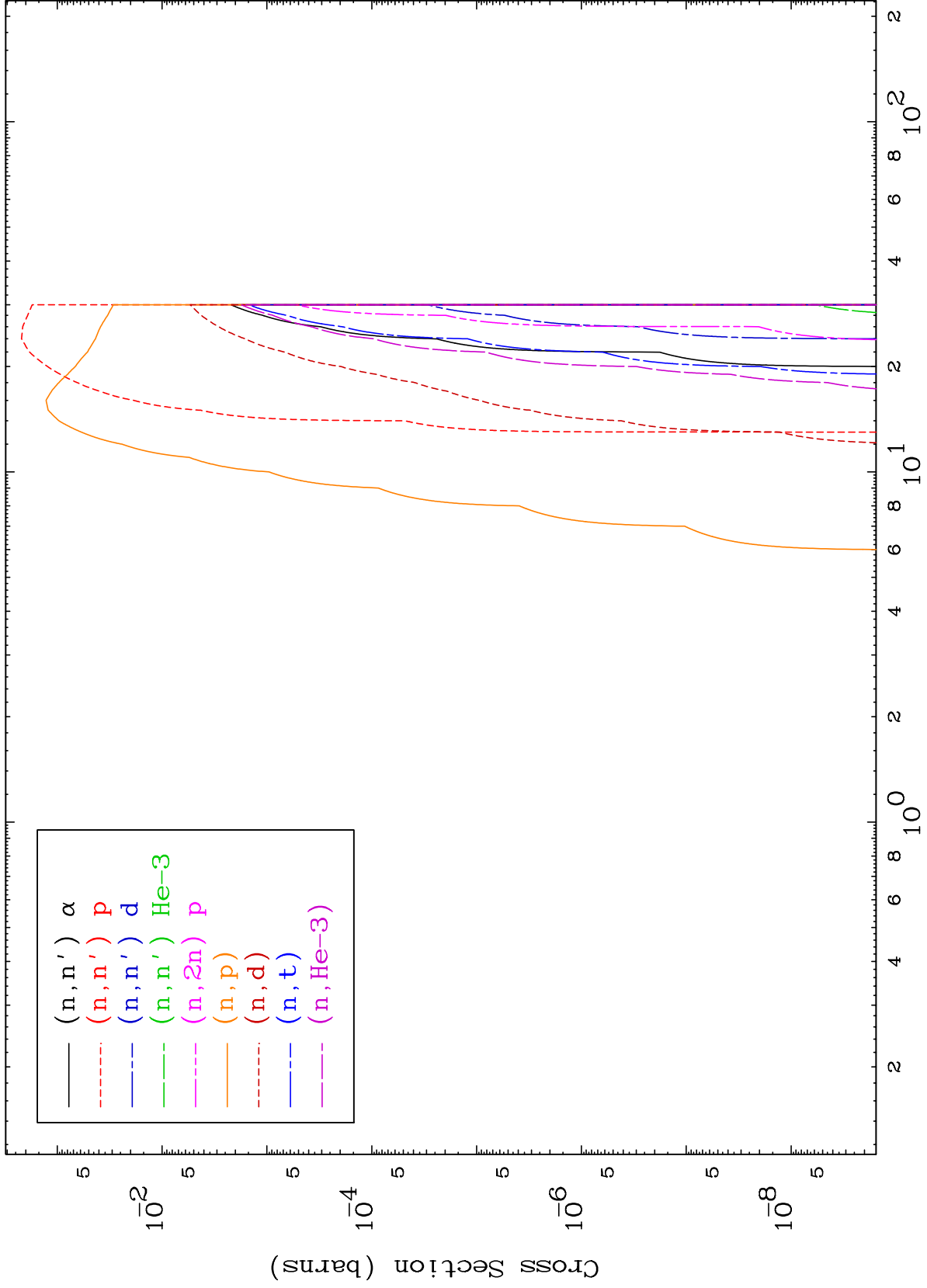


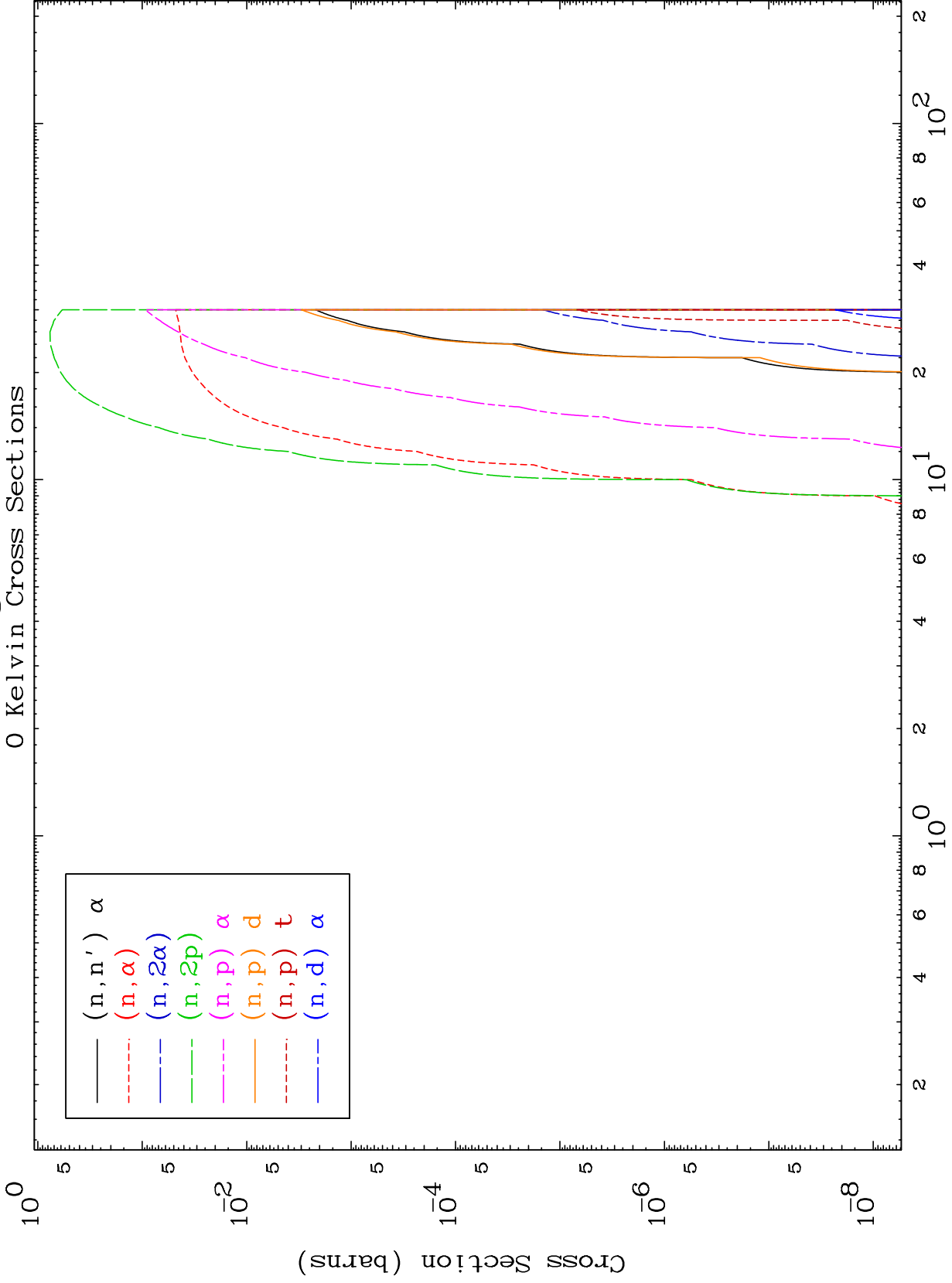
MAT 4301

$\alpha$  Neutron Absorption

43-Tc-91





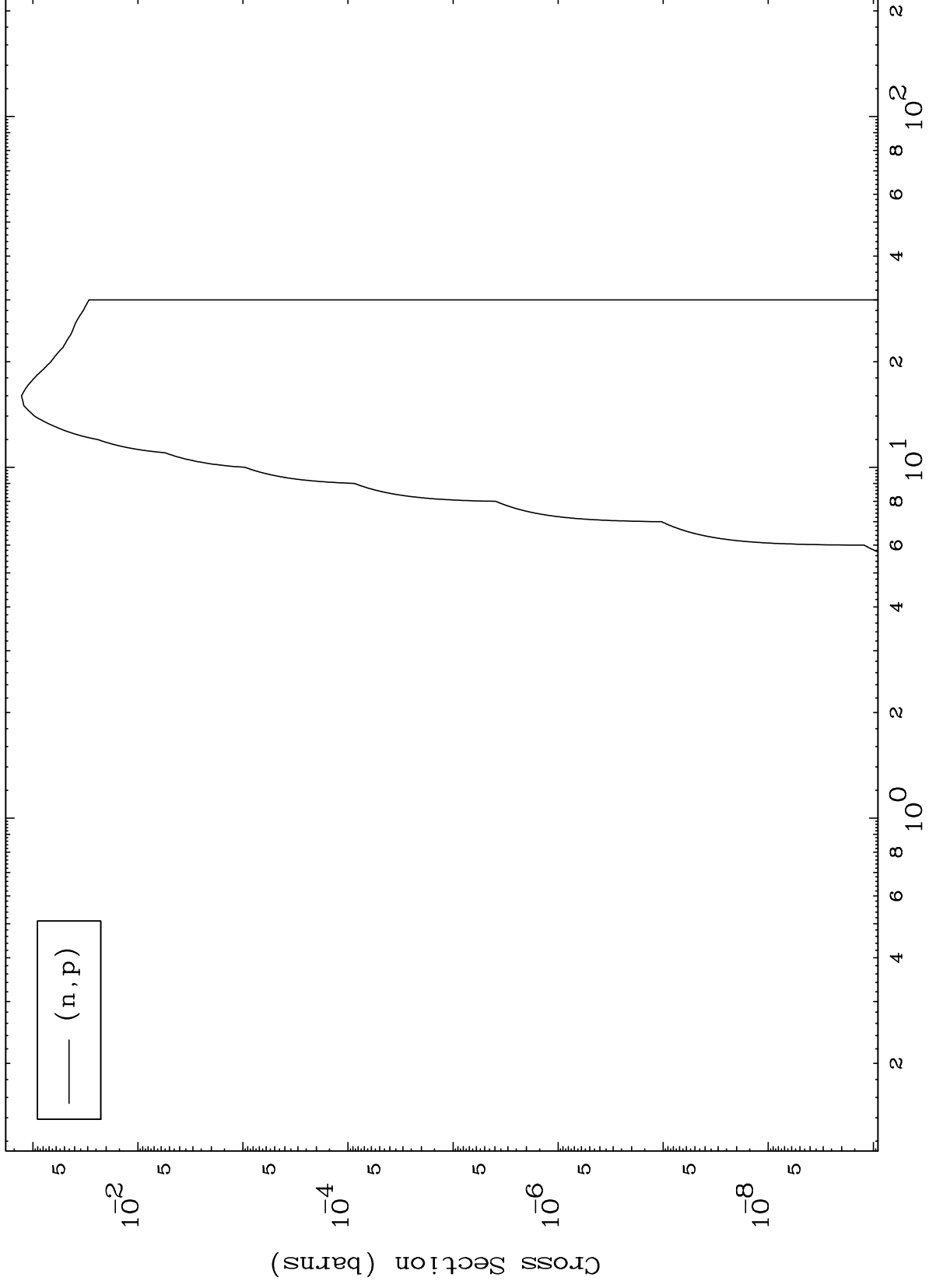


MAT 4301

( $\alpha, p$ ) Levels

43-Tc-91

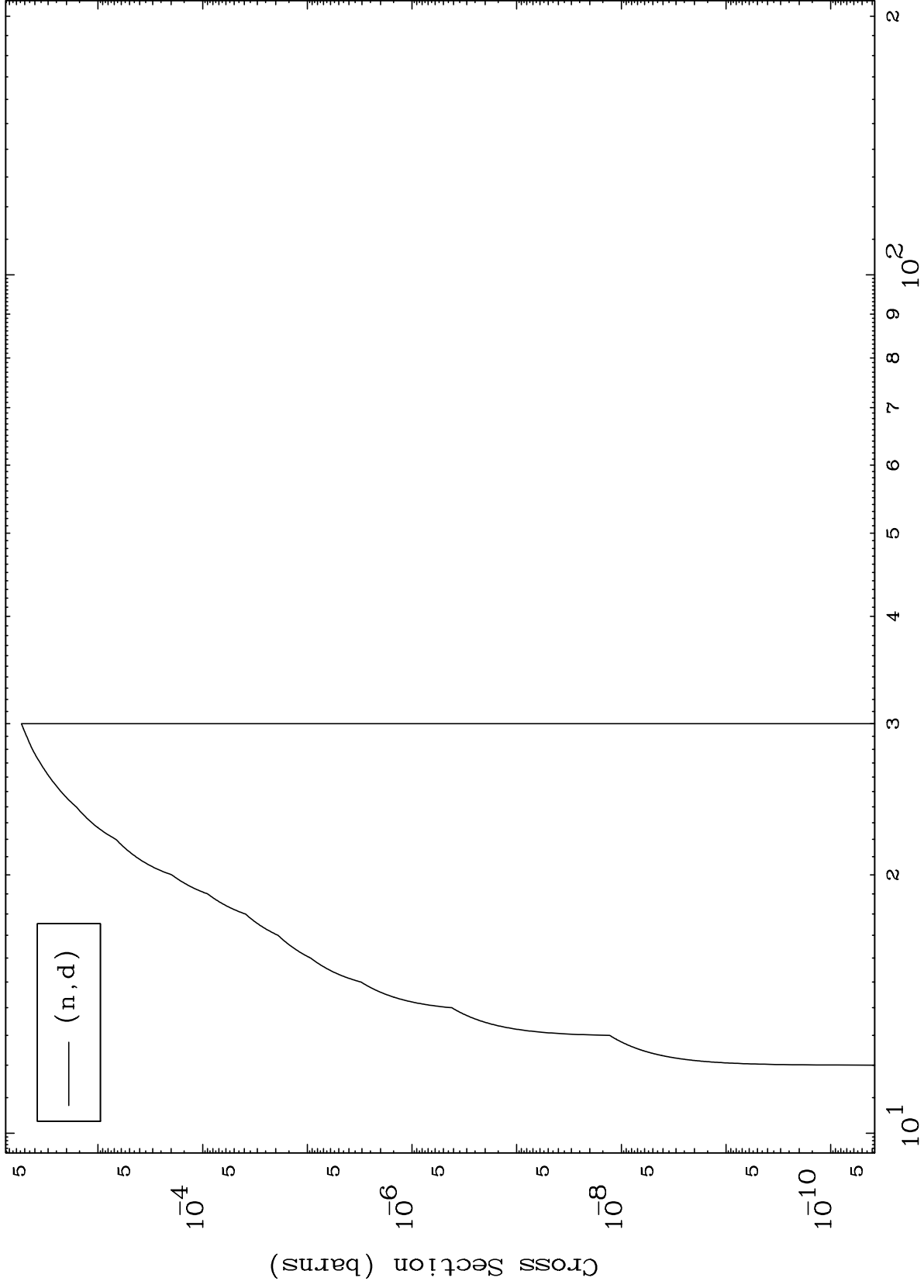
0 Kelvin Cross Sections



MAT 4301

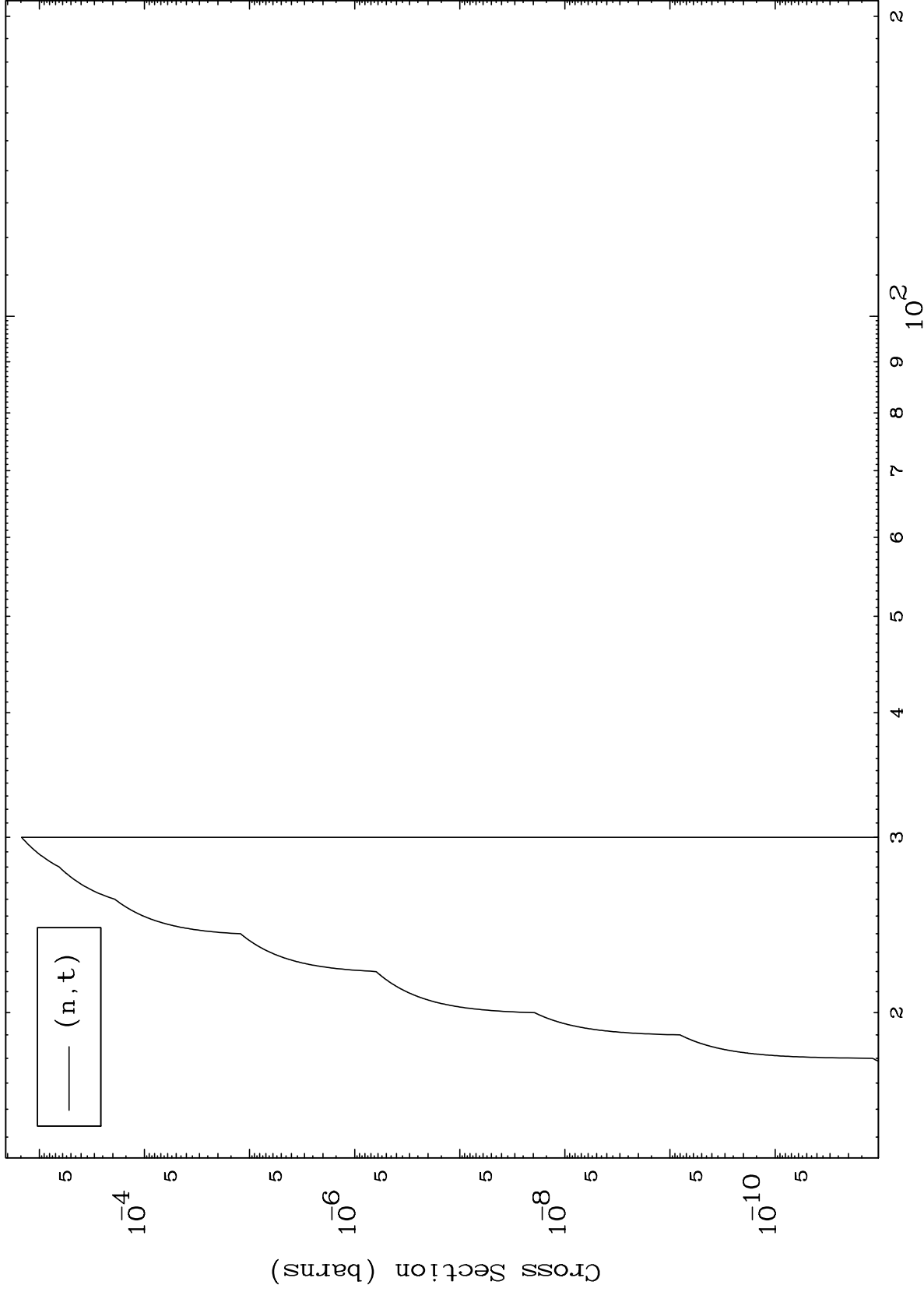
( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

43-Tc-91



Incident Energy (MeV)

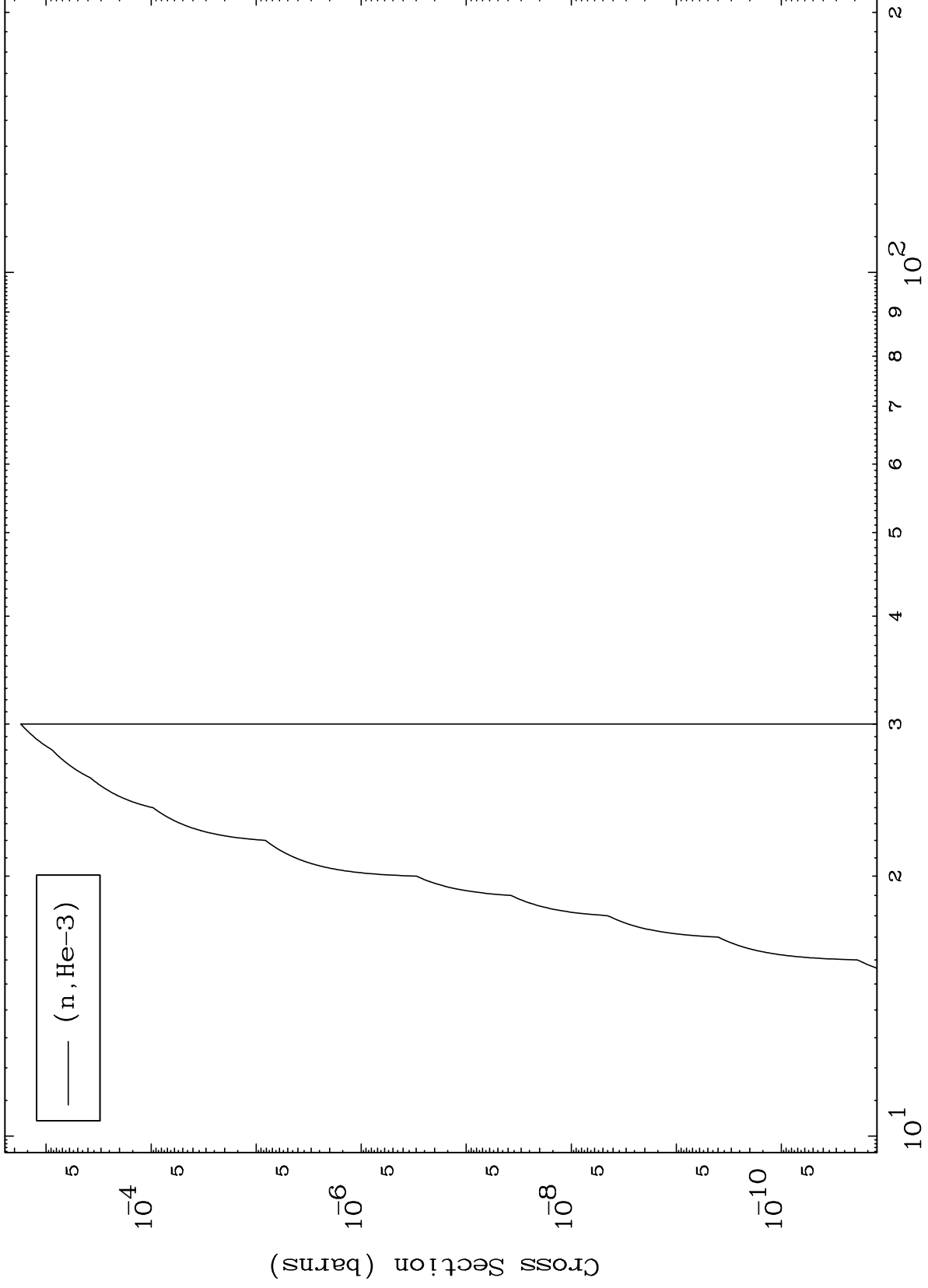
43-Tc-91



MAT 4301

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

43-Tc-91



Incident Energy (MeV)

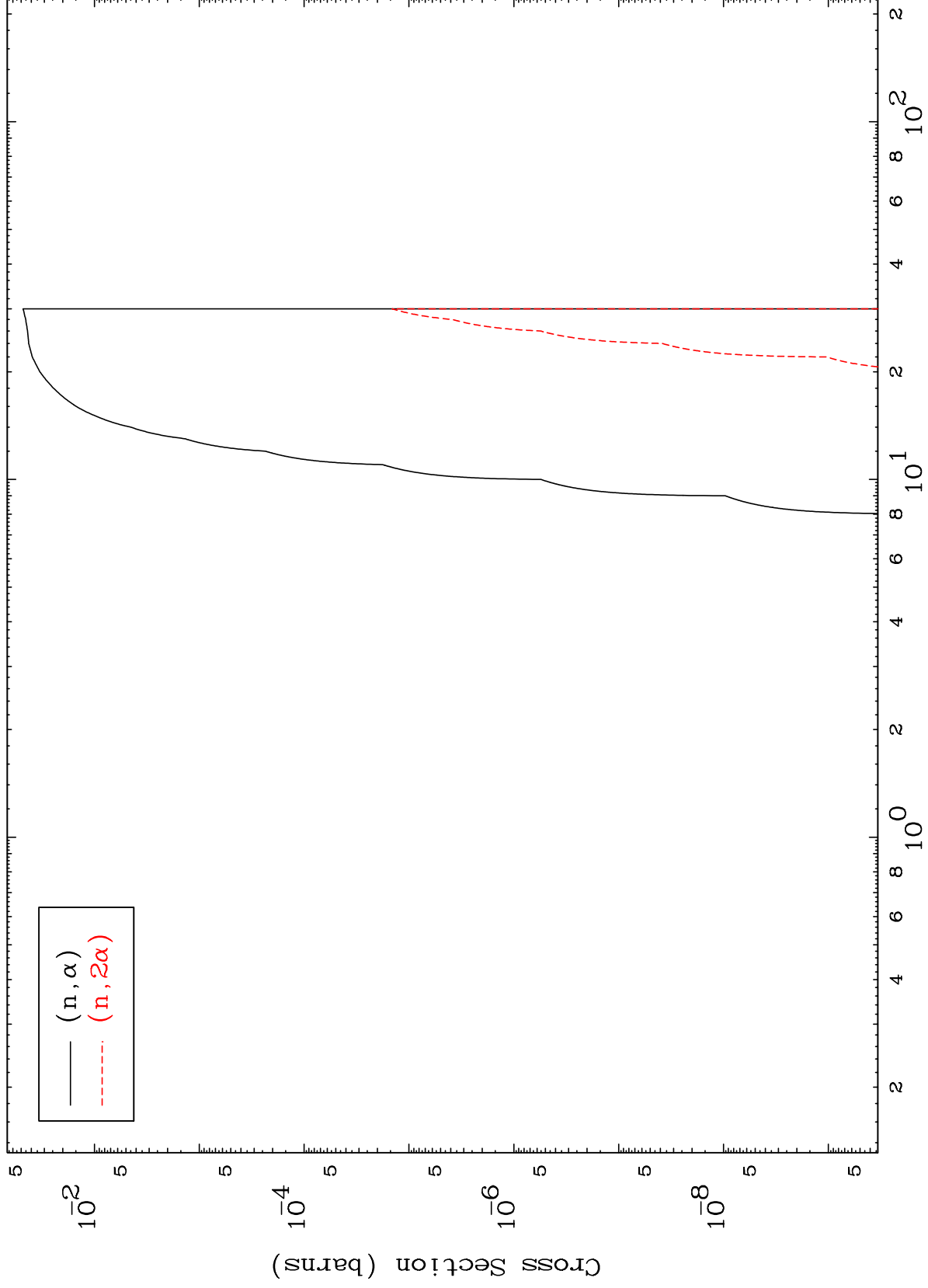
43-Tc-91

MAT 4301

( $\alpha, \alpha$ ) Levels

43-Tc-91

0 Kelvin Cross Sections



10

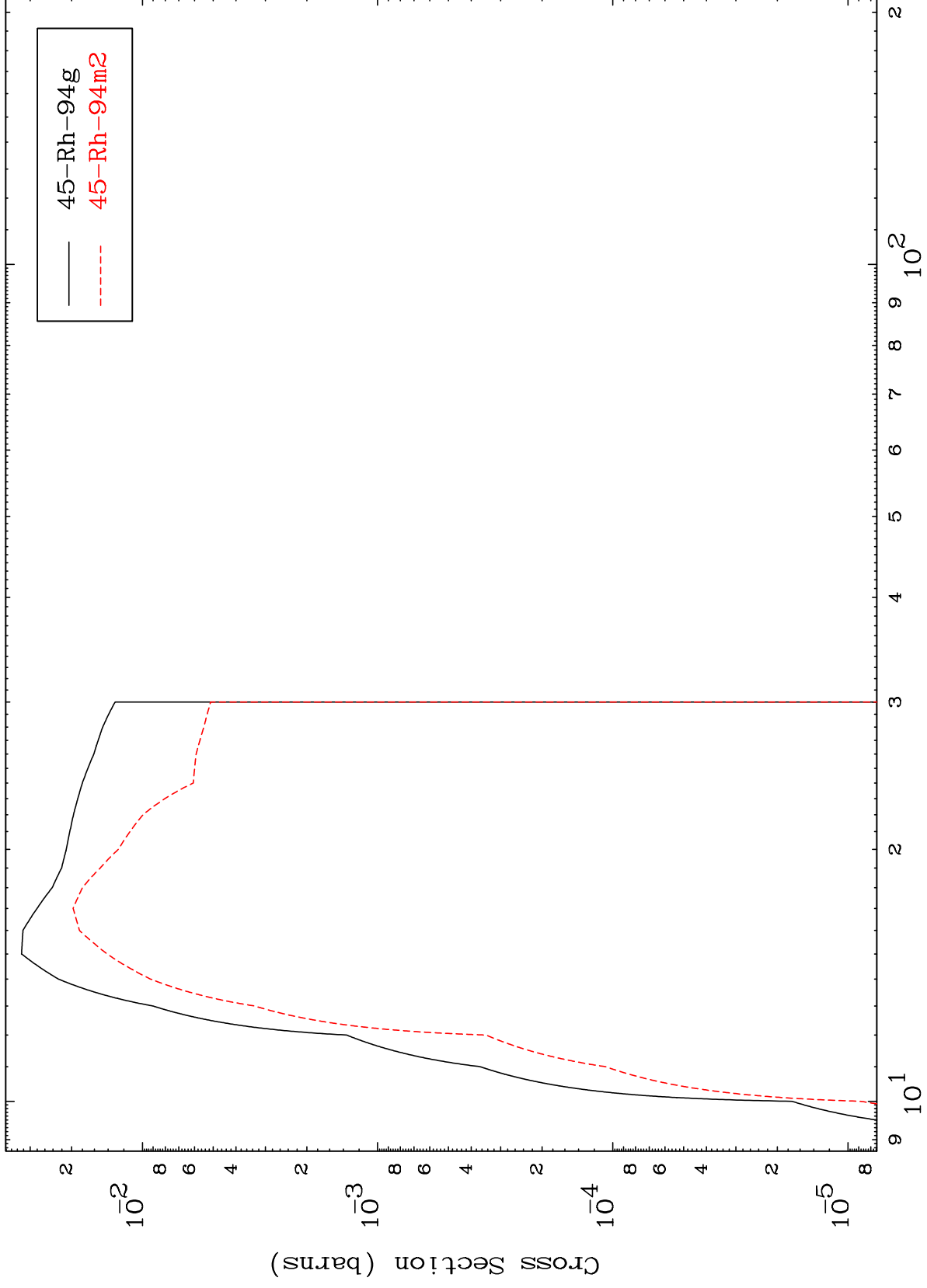
Incident Energy (MeV)

43-Tc-91

MAT 4301

43-Tc-91

Inelastic  
Radionuclide Production Cross Section

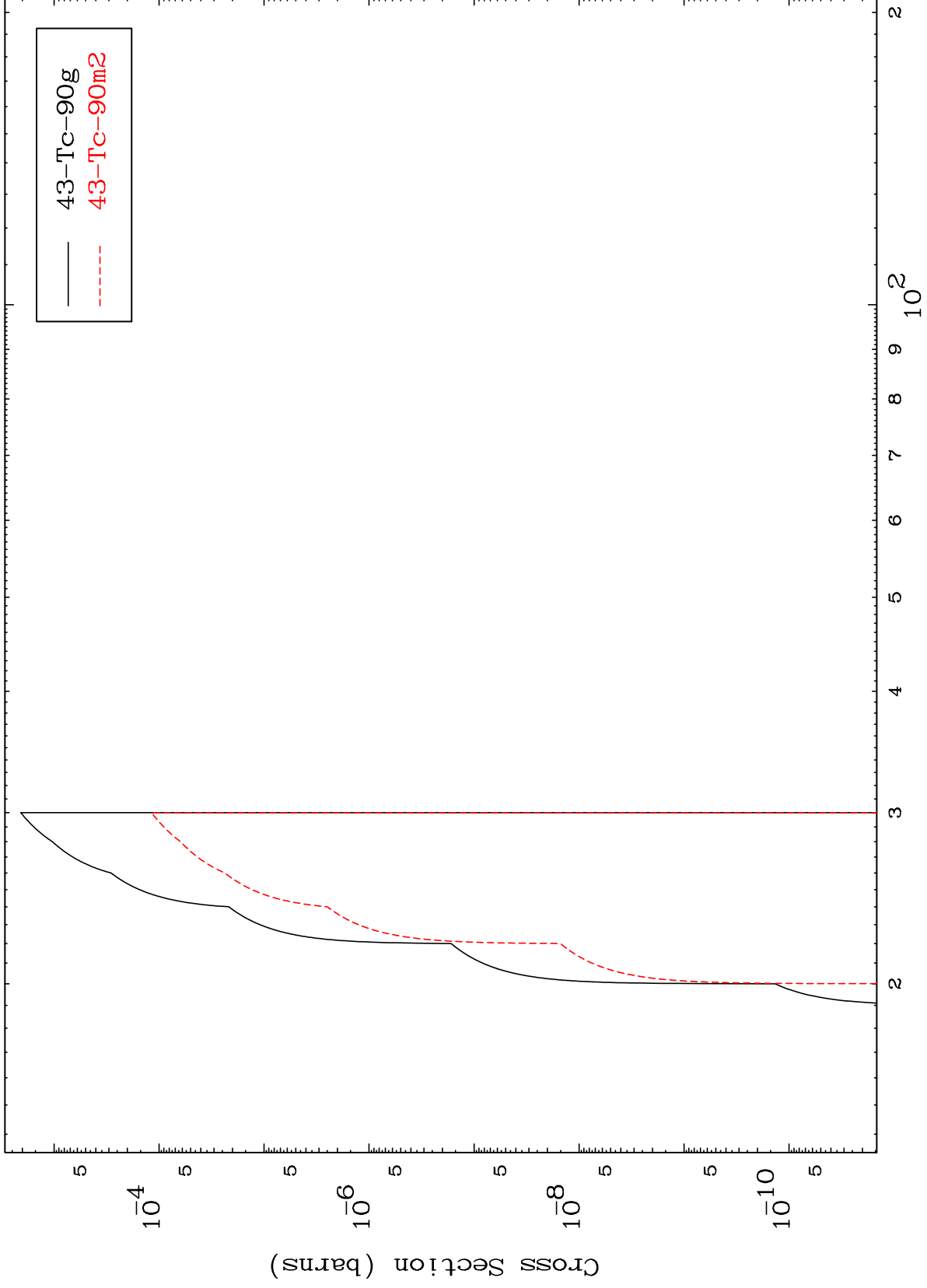


11

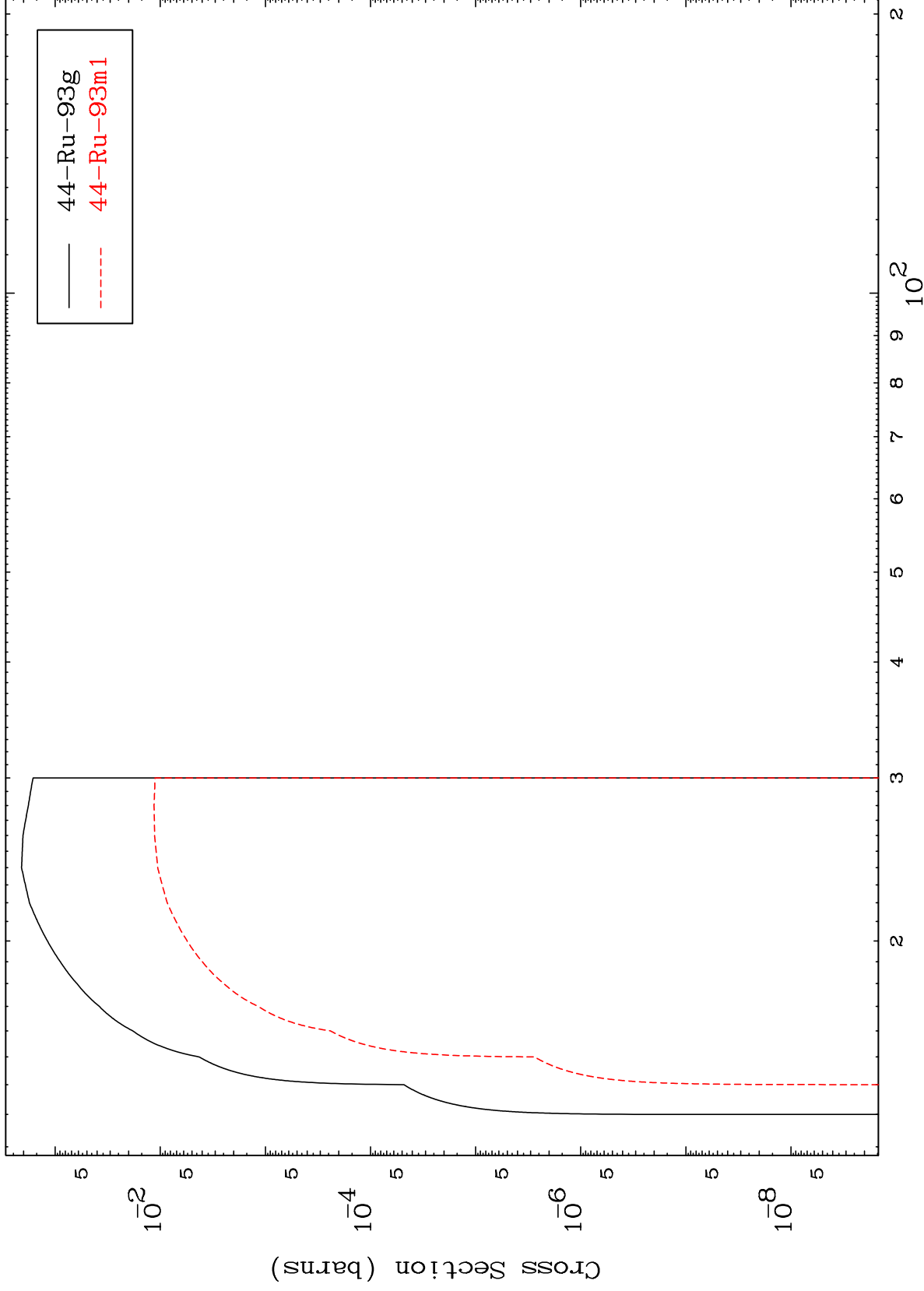
Incident Energy (MeV)

43-Tc-91

Radionuclide Production Cross Section



Radionuclide Production Cross Section

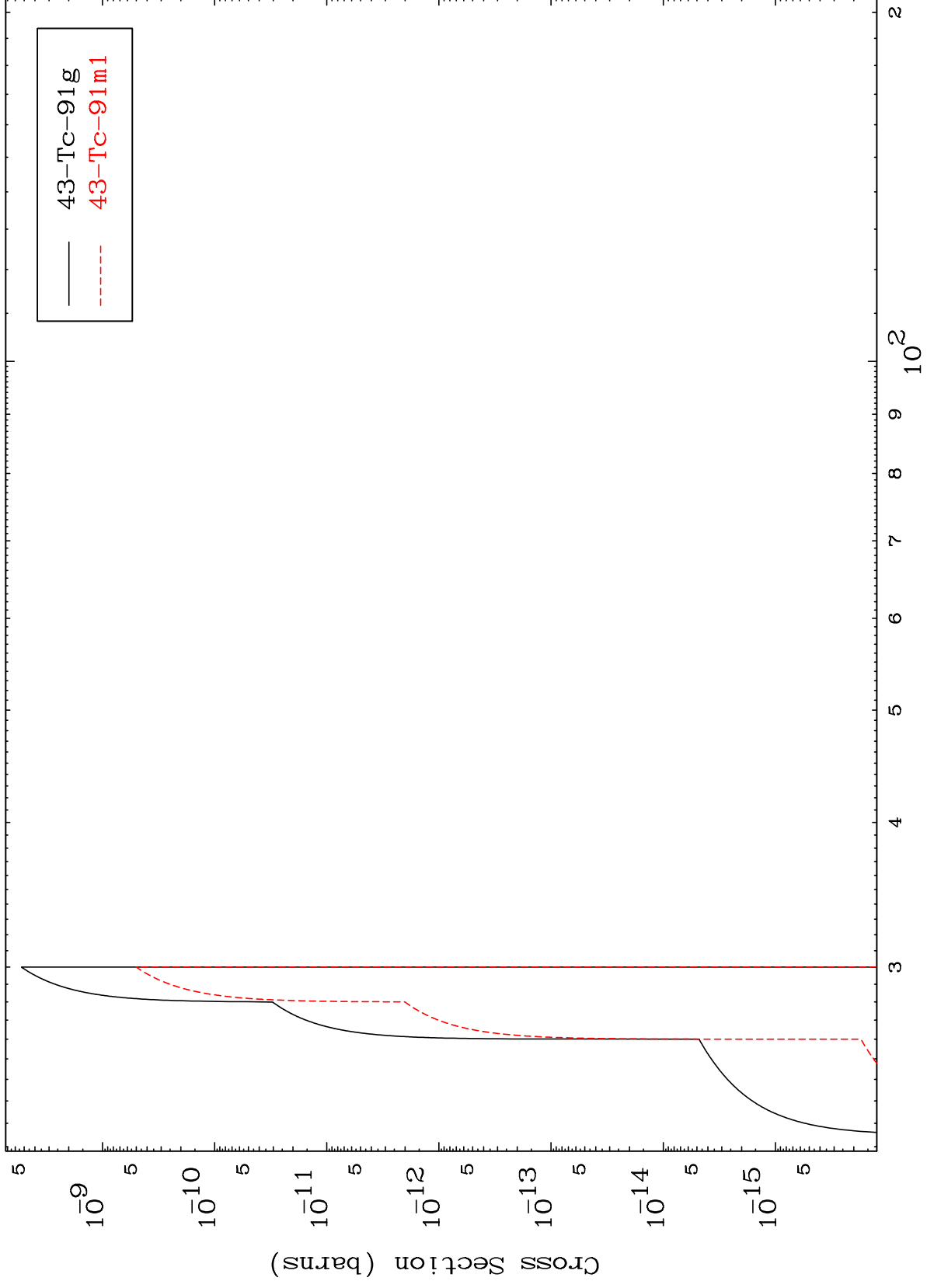


MAT 4301

(n,n') He-3

43-Tc-91

Radionuclide Production Cross Section

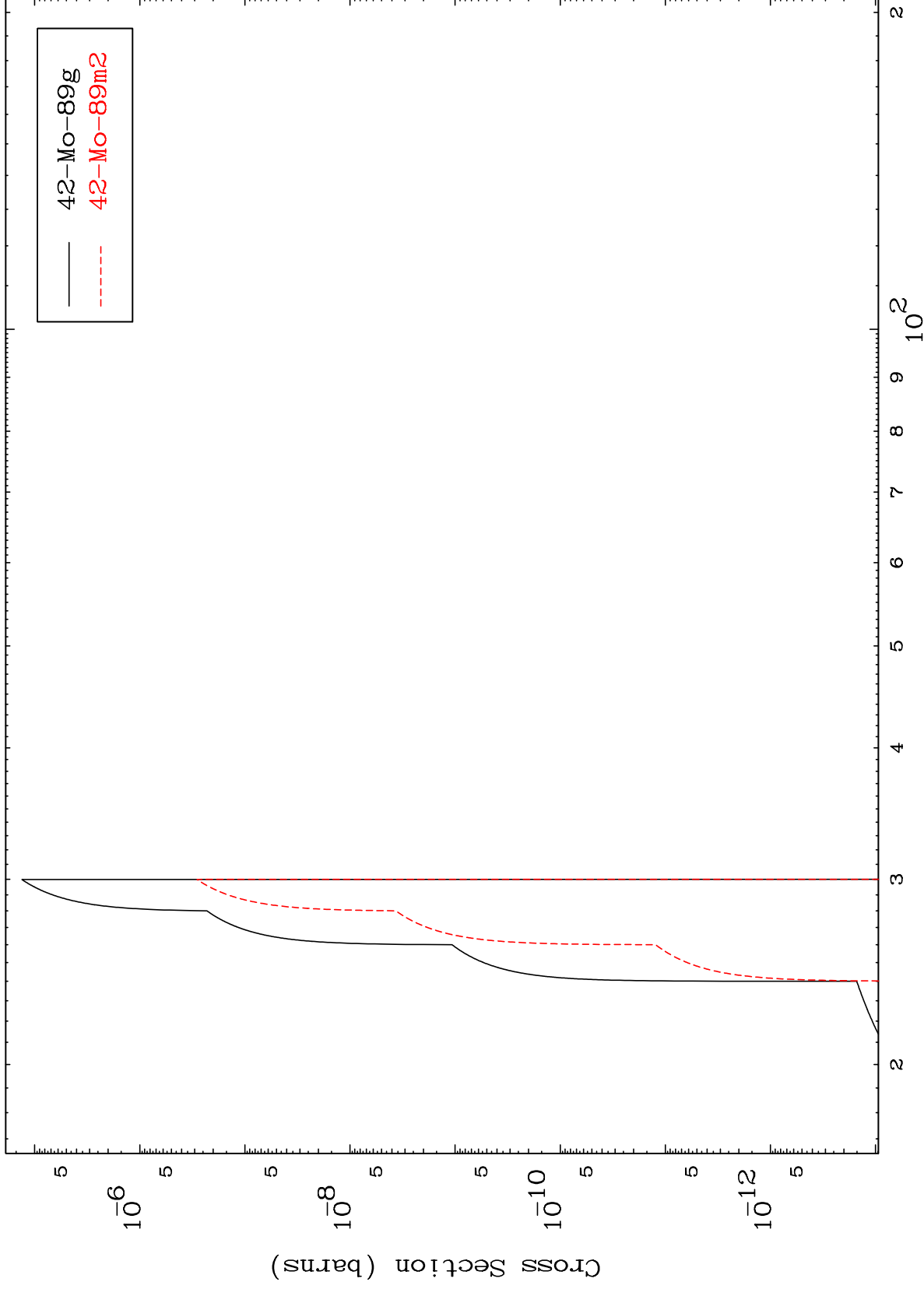


14

Incident Energy (MeV)

43-Tc-91

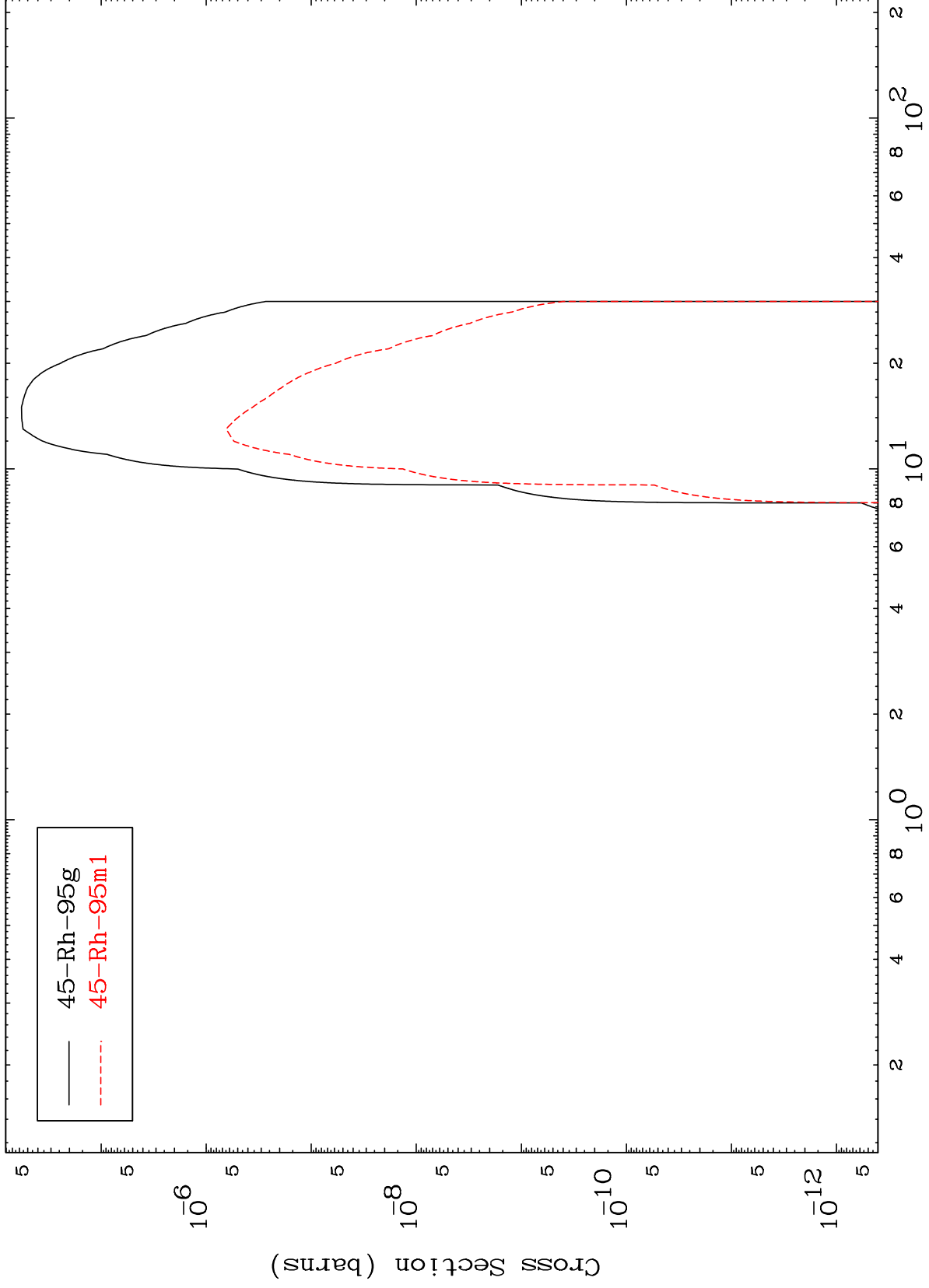
Radionuclide Production Cross Section



MAT 4301

43-Tc-91

(n,  $\gamma$ )  
Radionuclide Production Cross Section



16

Incident Energy (MeV)

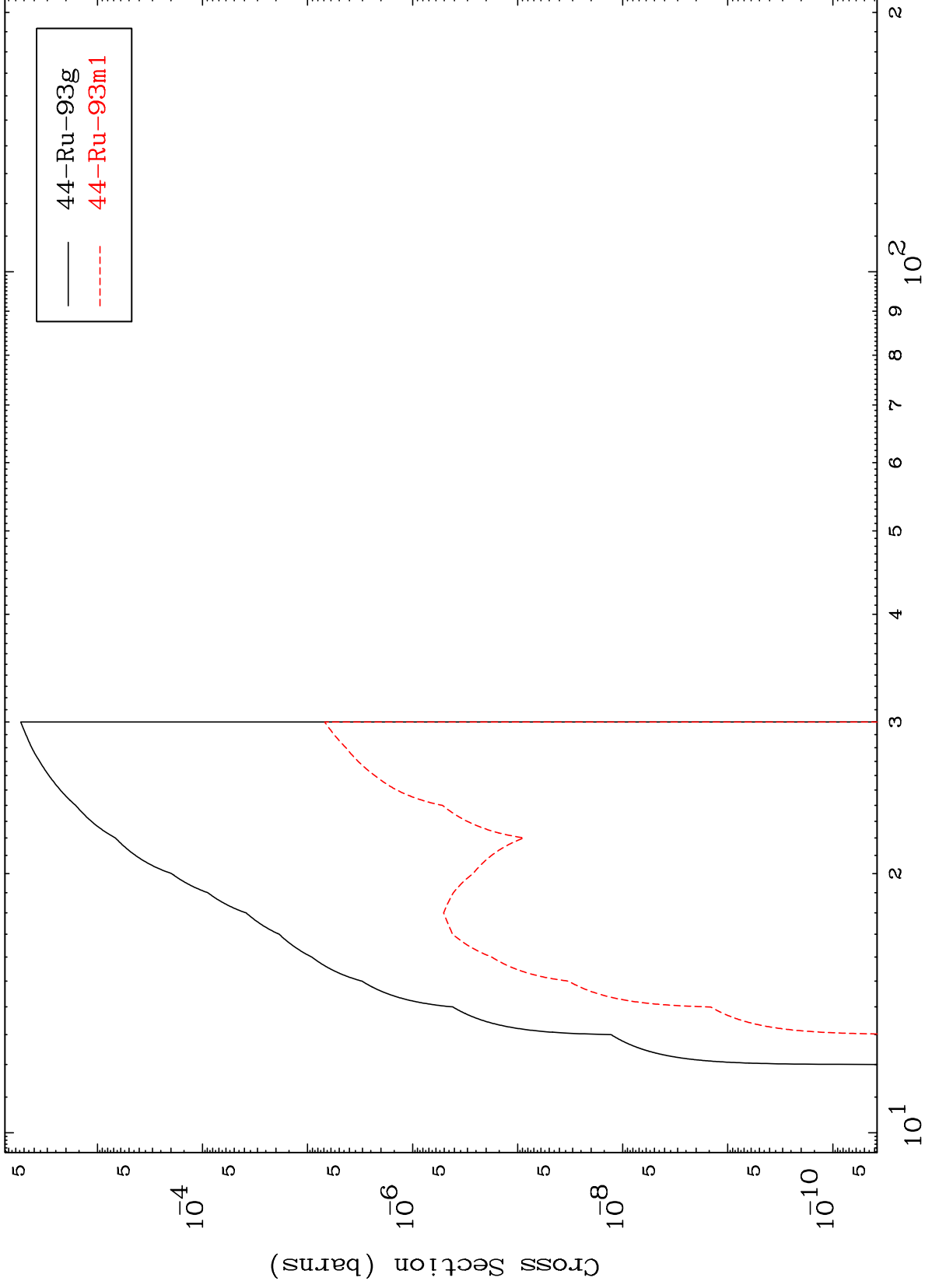
43-Tc-91

MAT 4301

(n,d)

43-Tc-91

Radionuclide Production Cross Section



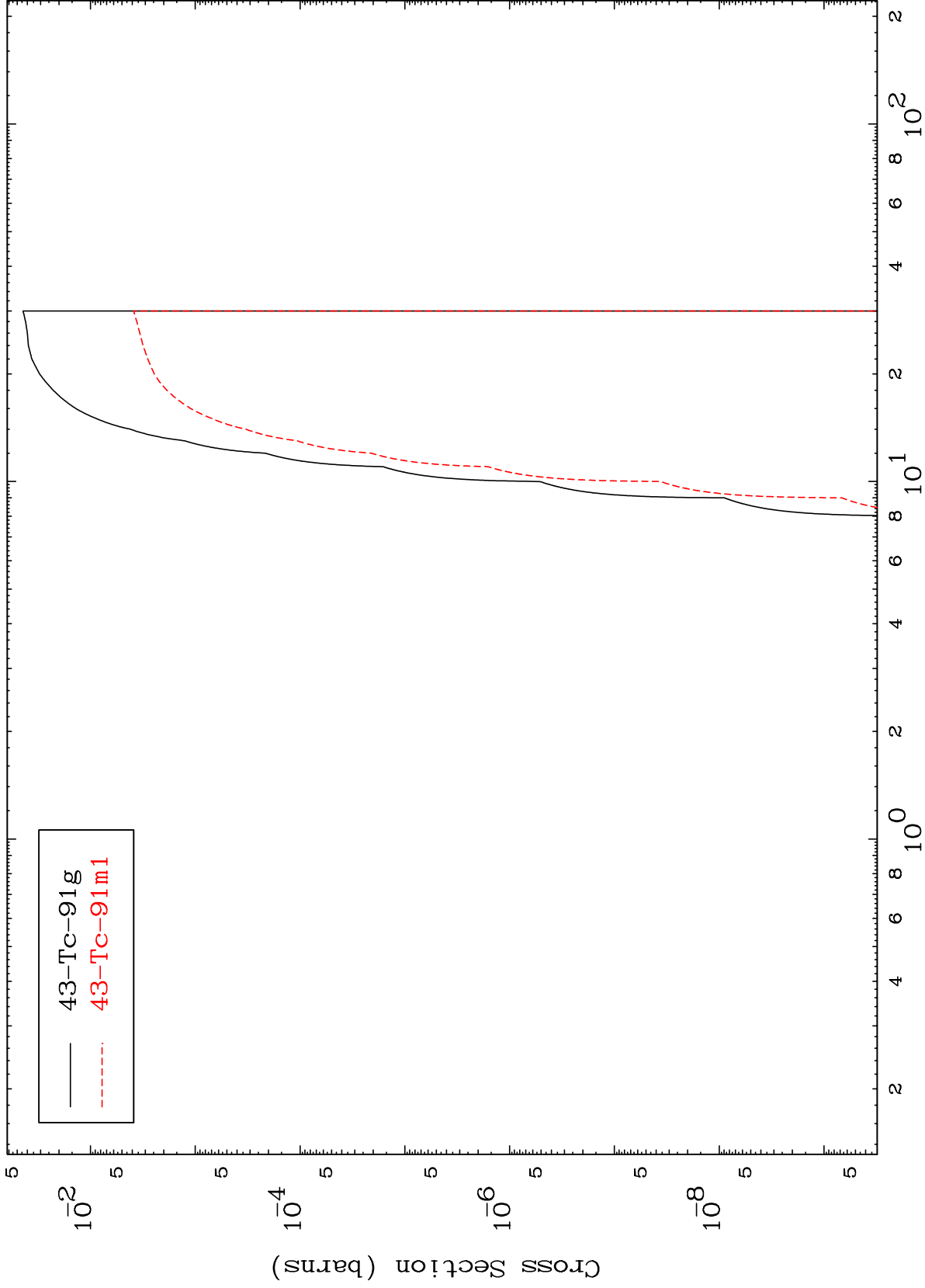
Incident Energy (MeV)

43-Tc-91

MAT 4301

43-Tc-91

Radionuclide Production Cross Section  
(n,  $\alpha$ )



18

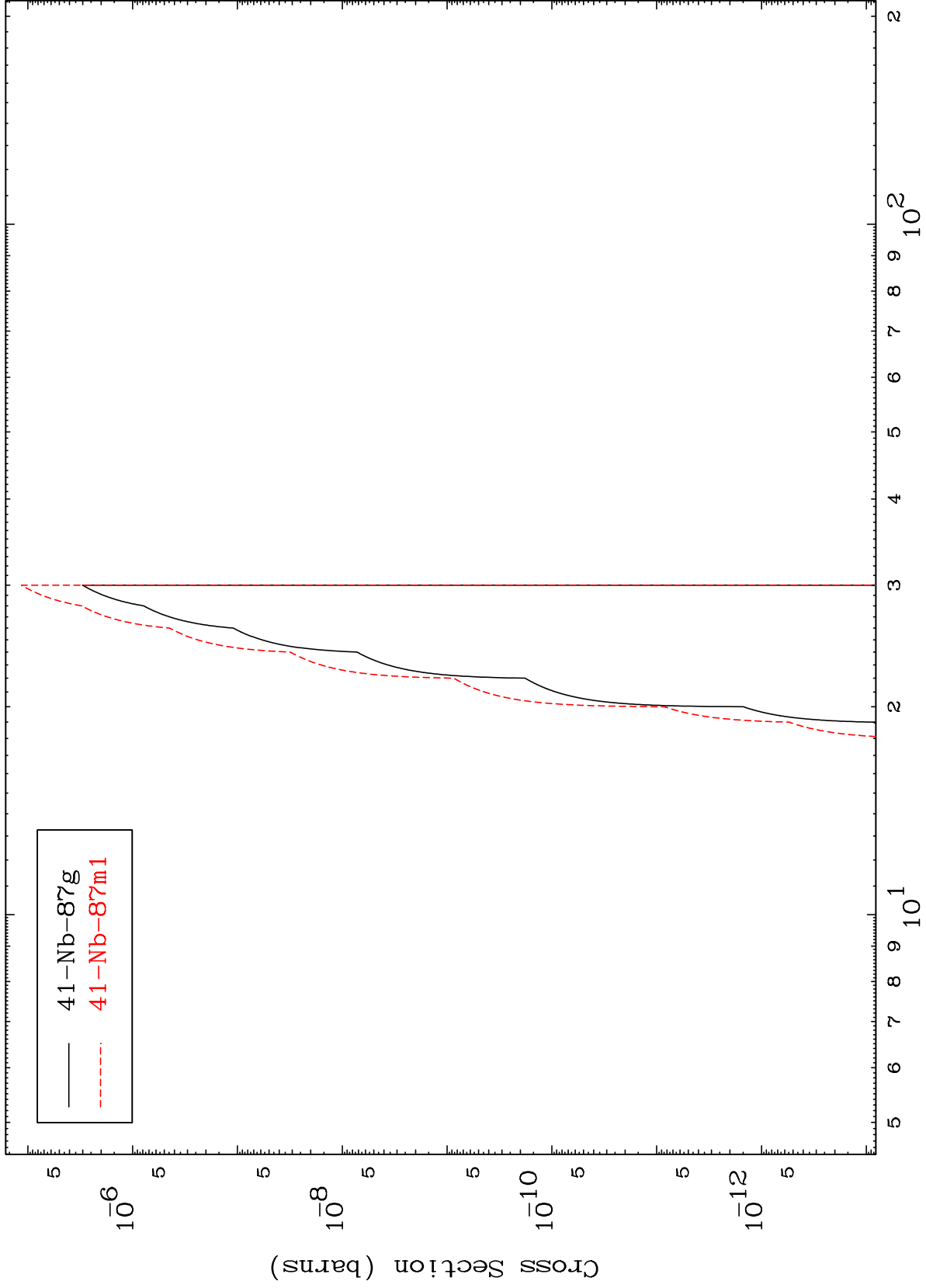
Incident Energy (MeV)

43-Tc-91

MAT 4301

43-Tc-91

(n,2α)  
Radionuclide Production Cross Section



19

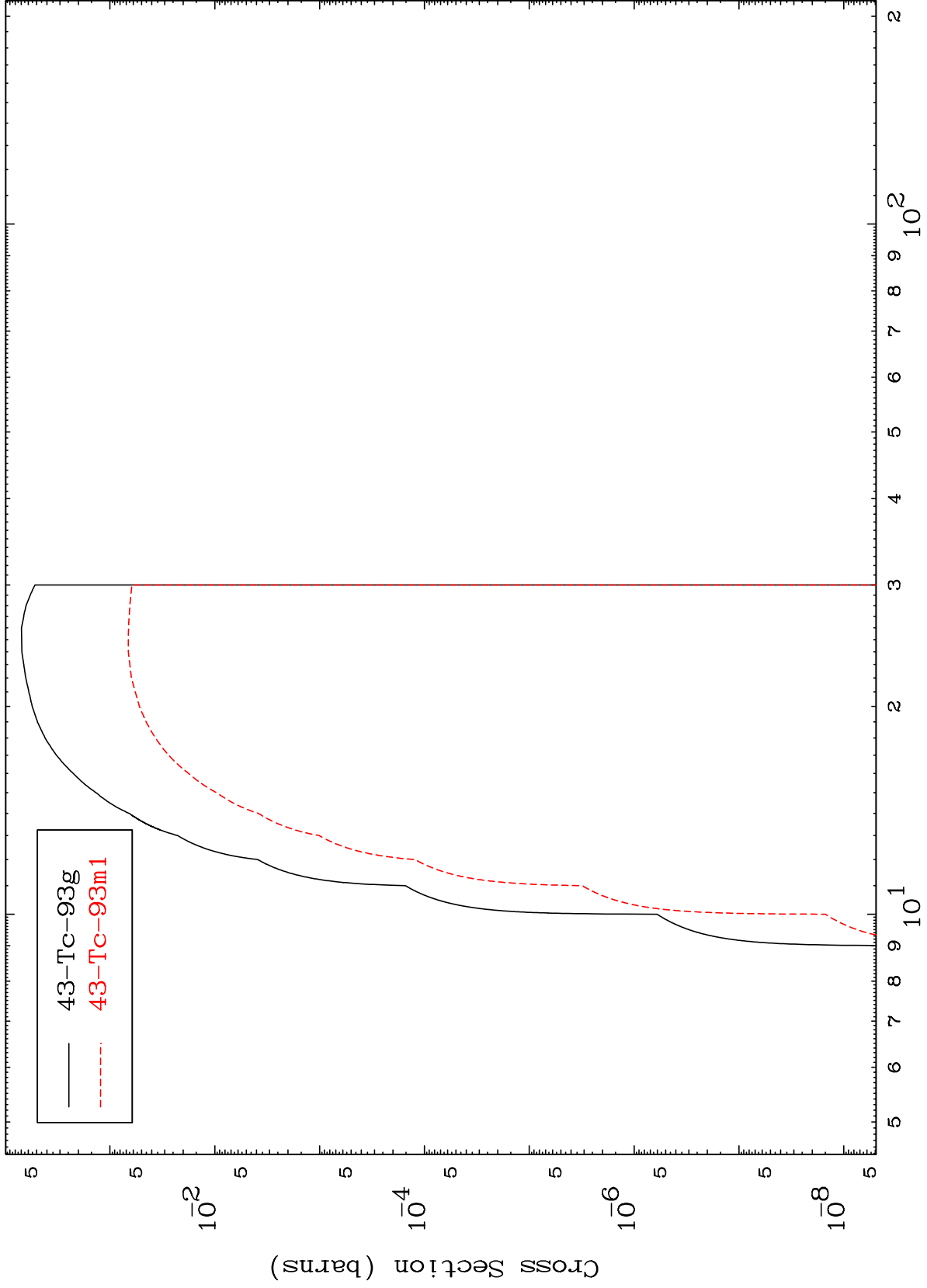
Incident Energy (MeV)

43-Tc-91

MAT 4301

43-Tc-91

(n,2p)  
Radionuclide Production Cross Section

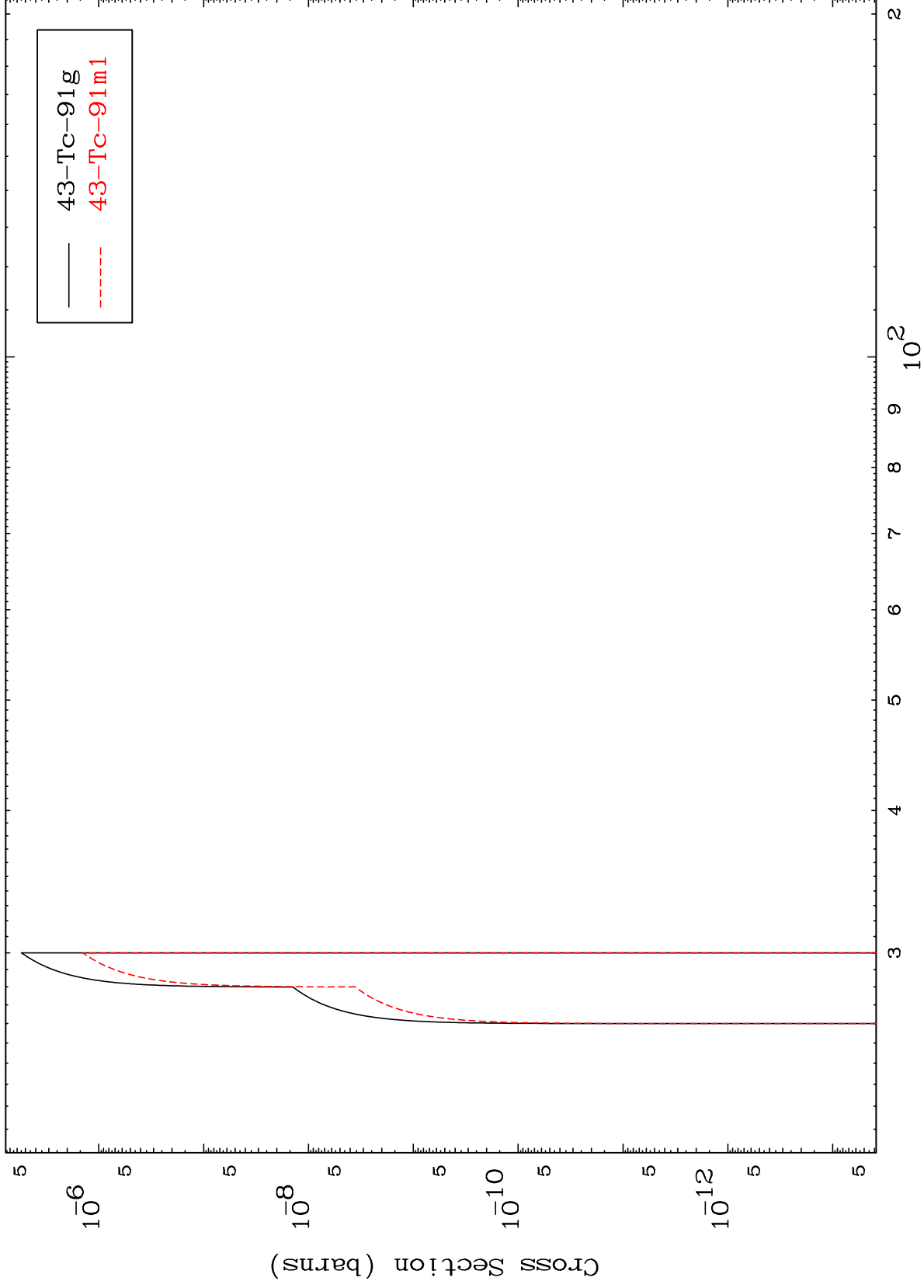


20

Incident Energy (MeV)

43-Tc-91

Radionuclide Production Cross Section

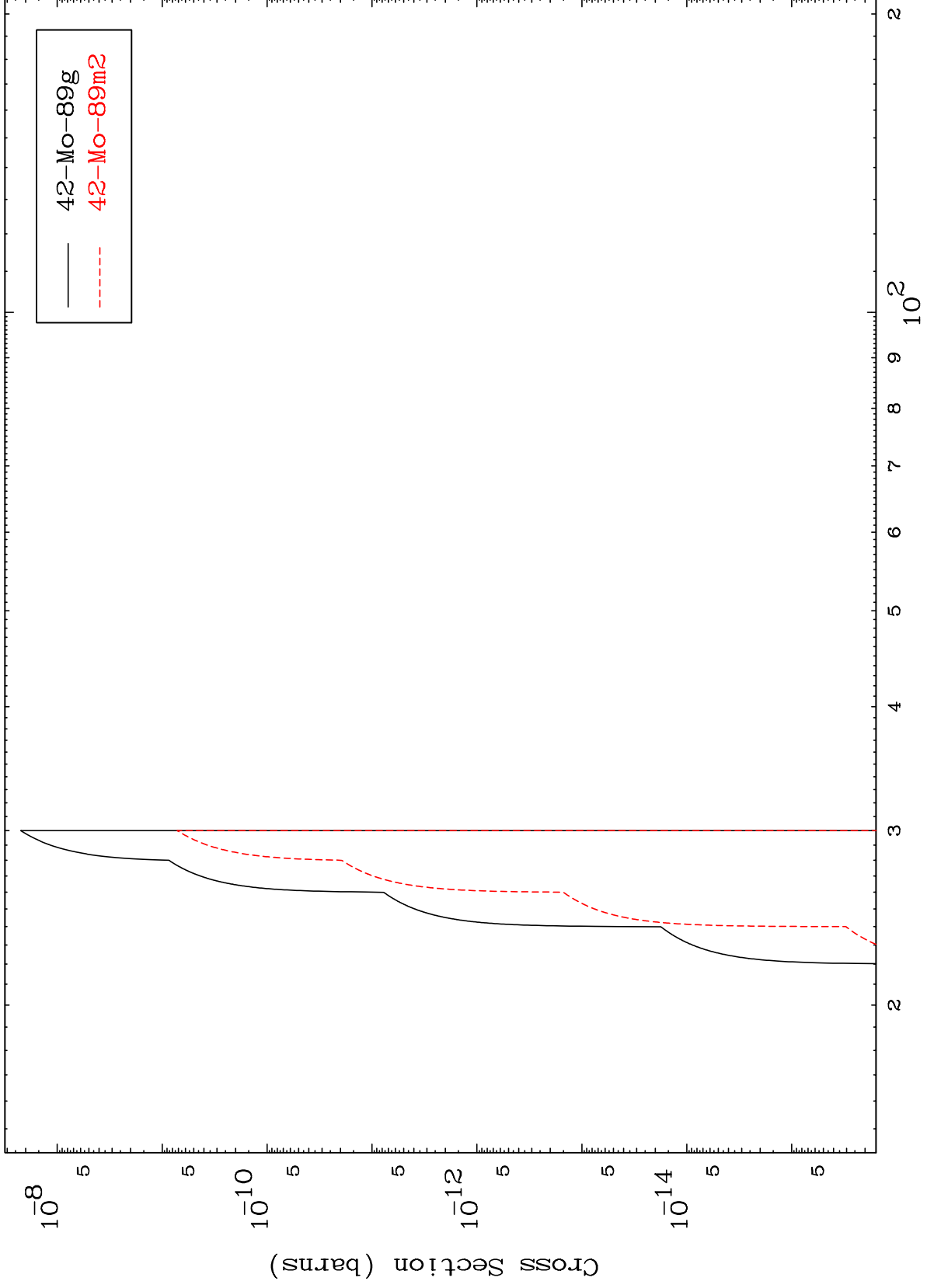


MAT 4301

(n,d)  $\alpha$

43-Tc-91

Radionuclide Production Cross Section



22

Incident Energy (MeV)

43-Tc-91