

Program EVALPLOT  
(Version 2021-1)

by

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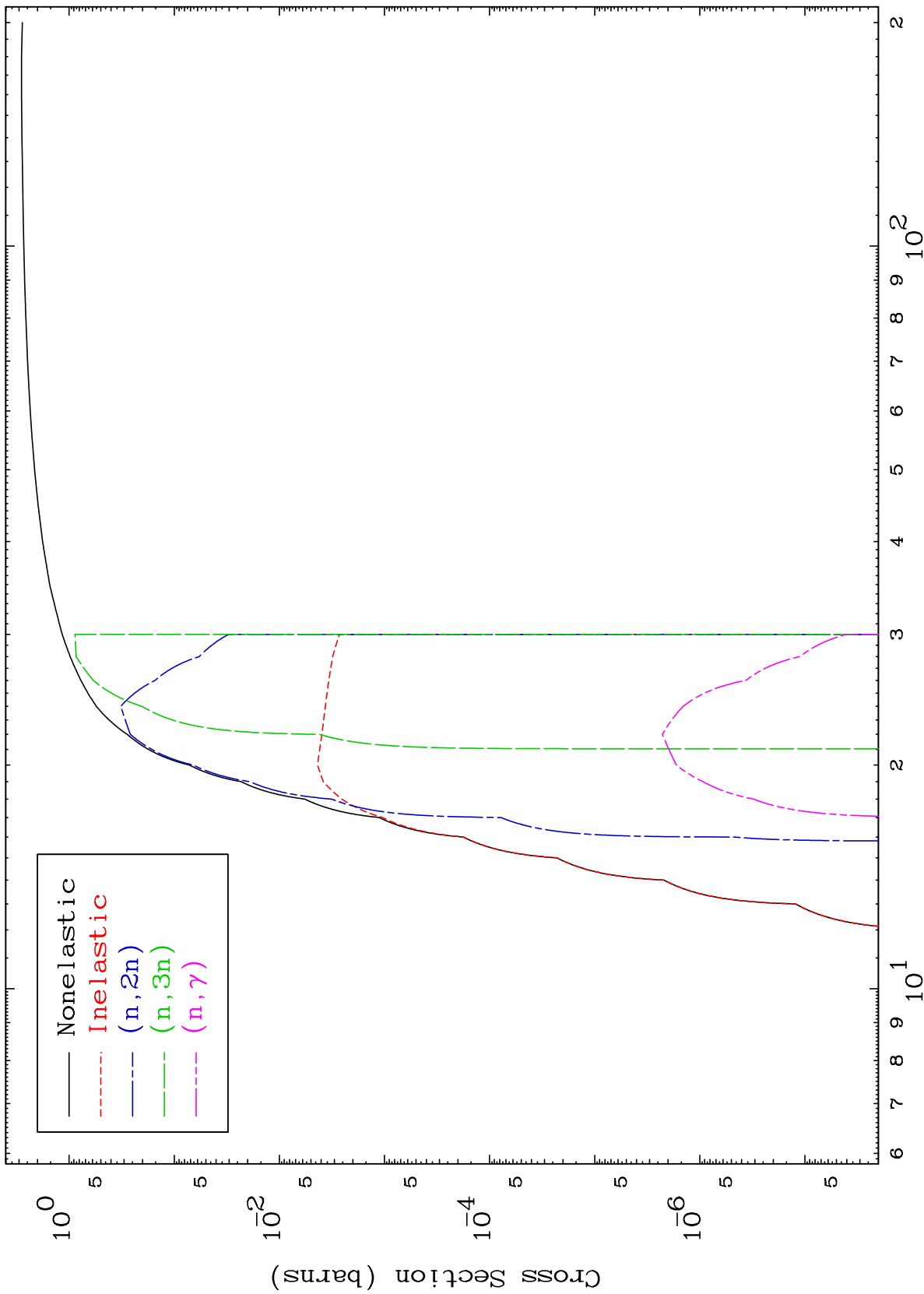
Press Mouse Button to Start

MAT 8143

$\alpha$  Major

81-T1-209

0 Kelvin Cross Sections



1

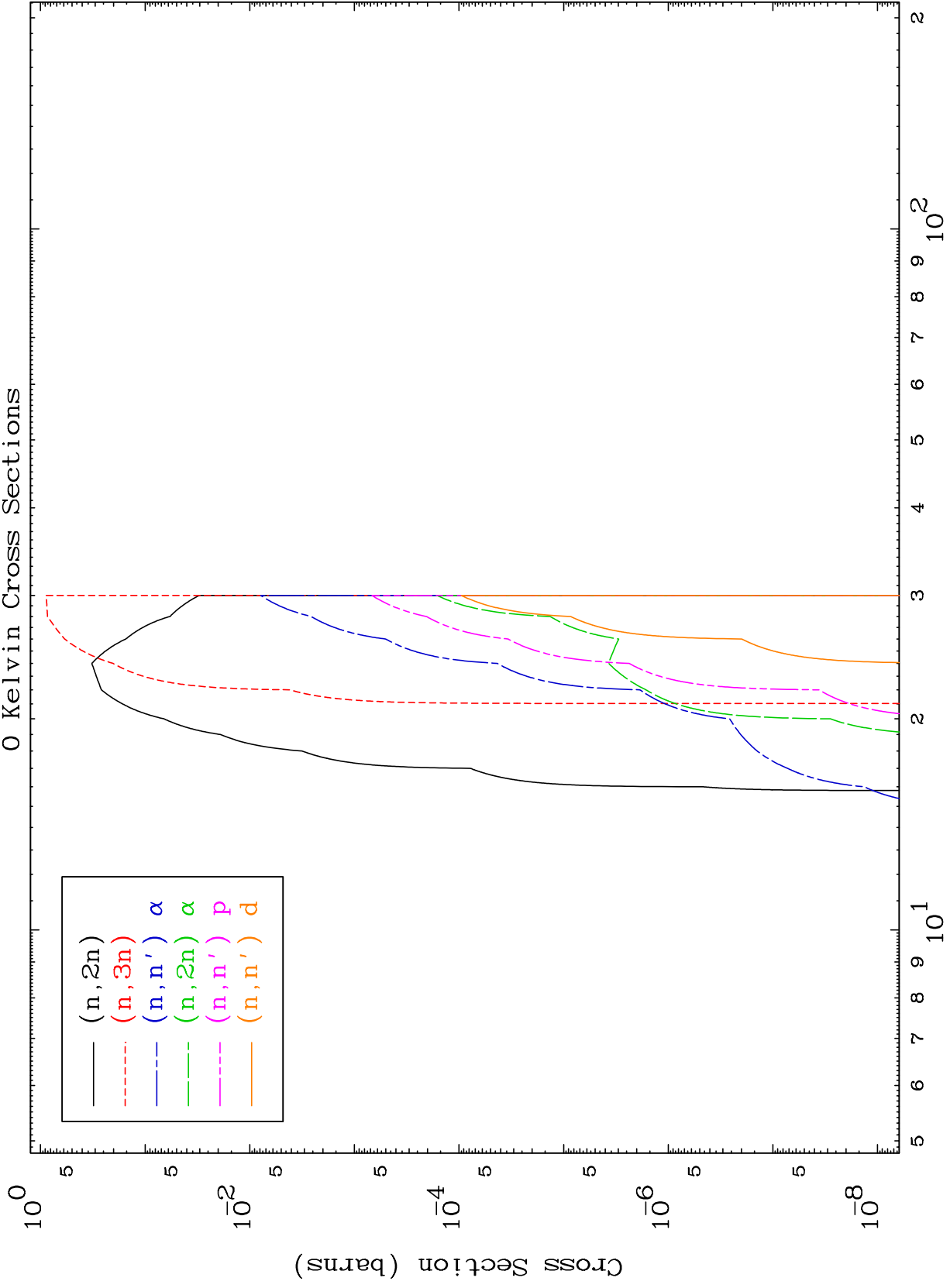
Incident Energy (MeV)

81-T1-209

MAT 8143

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

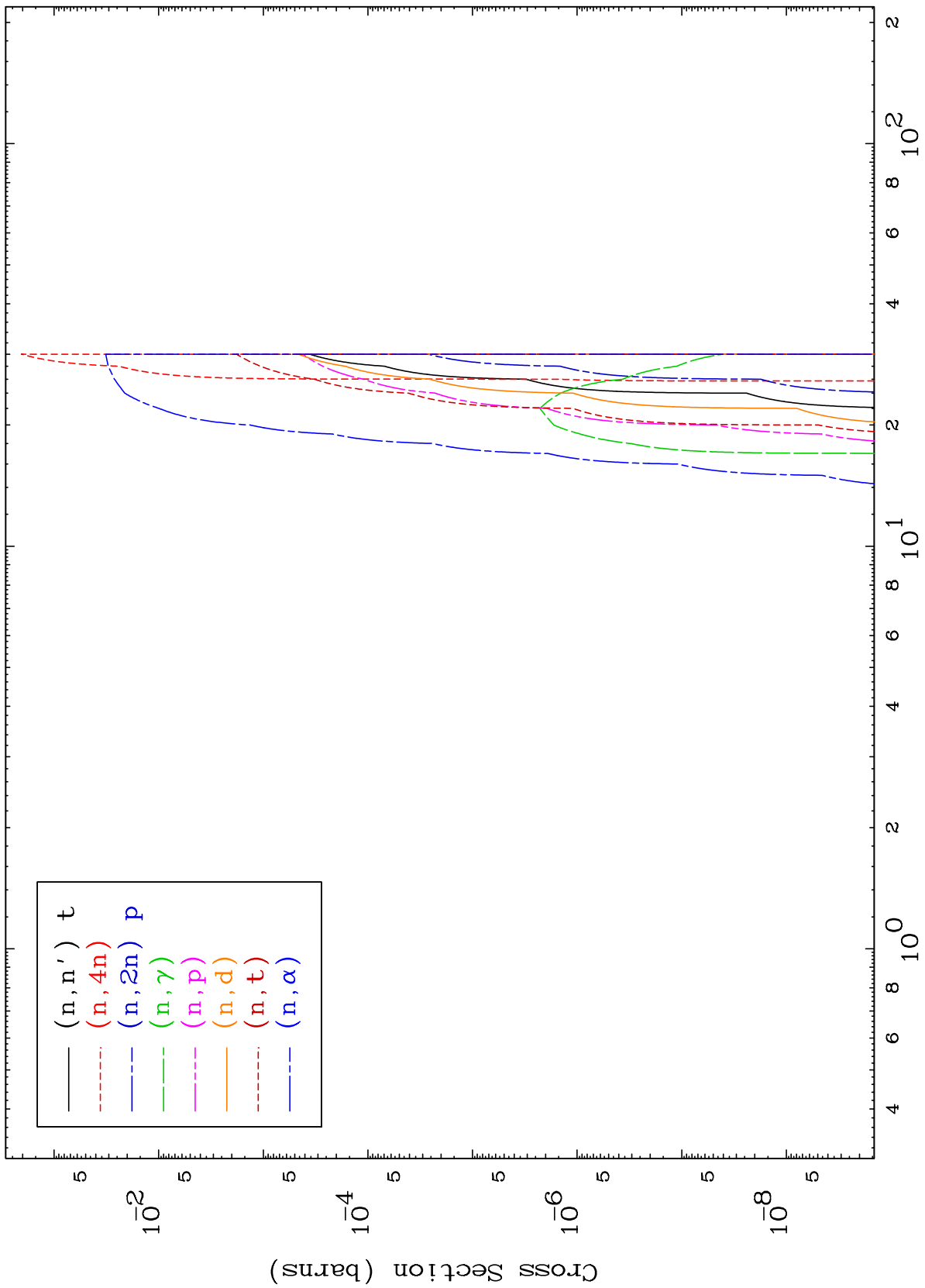
81-Tl-209

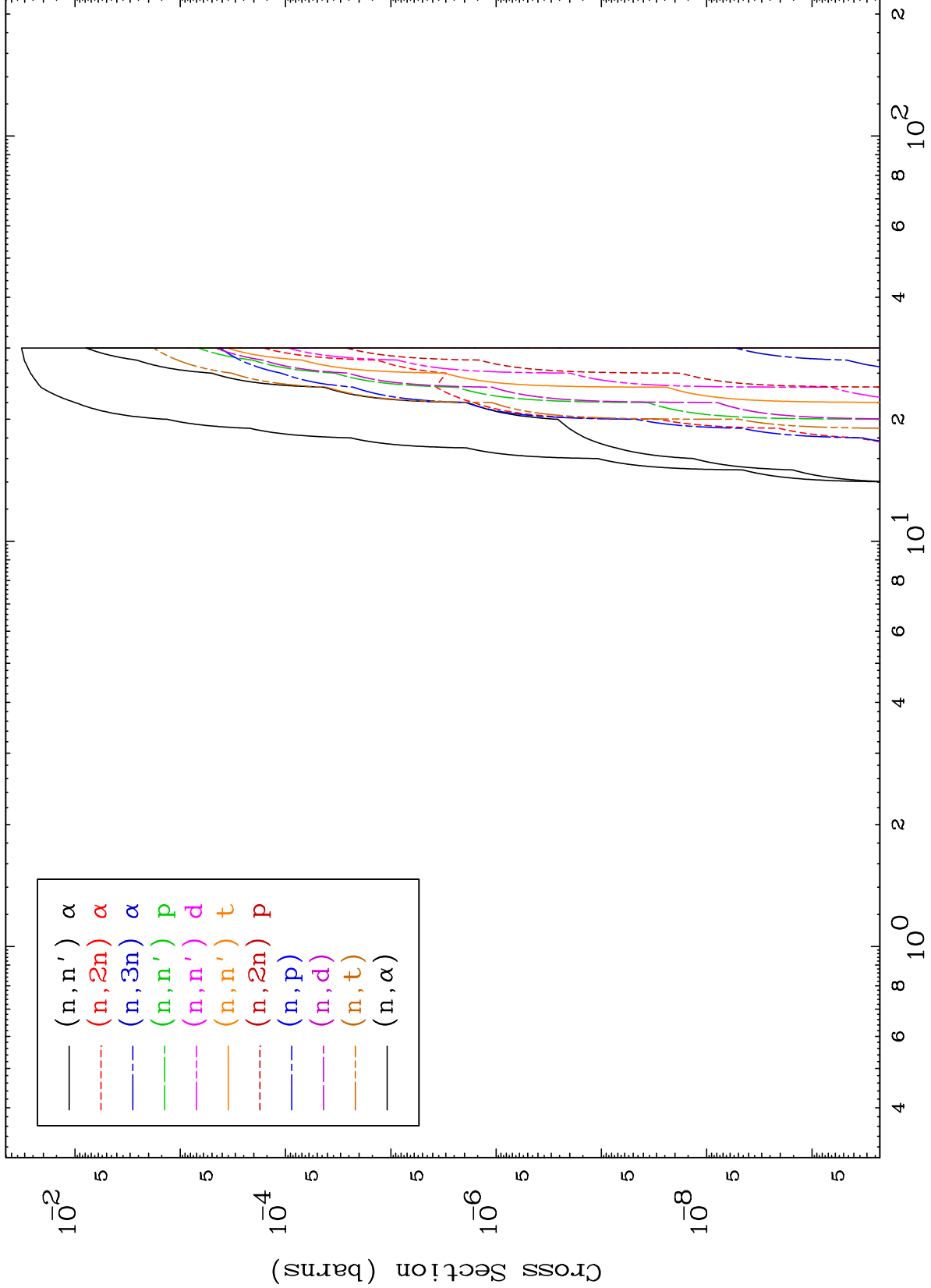


2

Incident Energy (MeV)

81-Tl-209

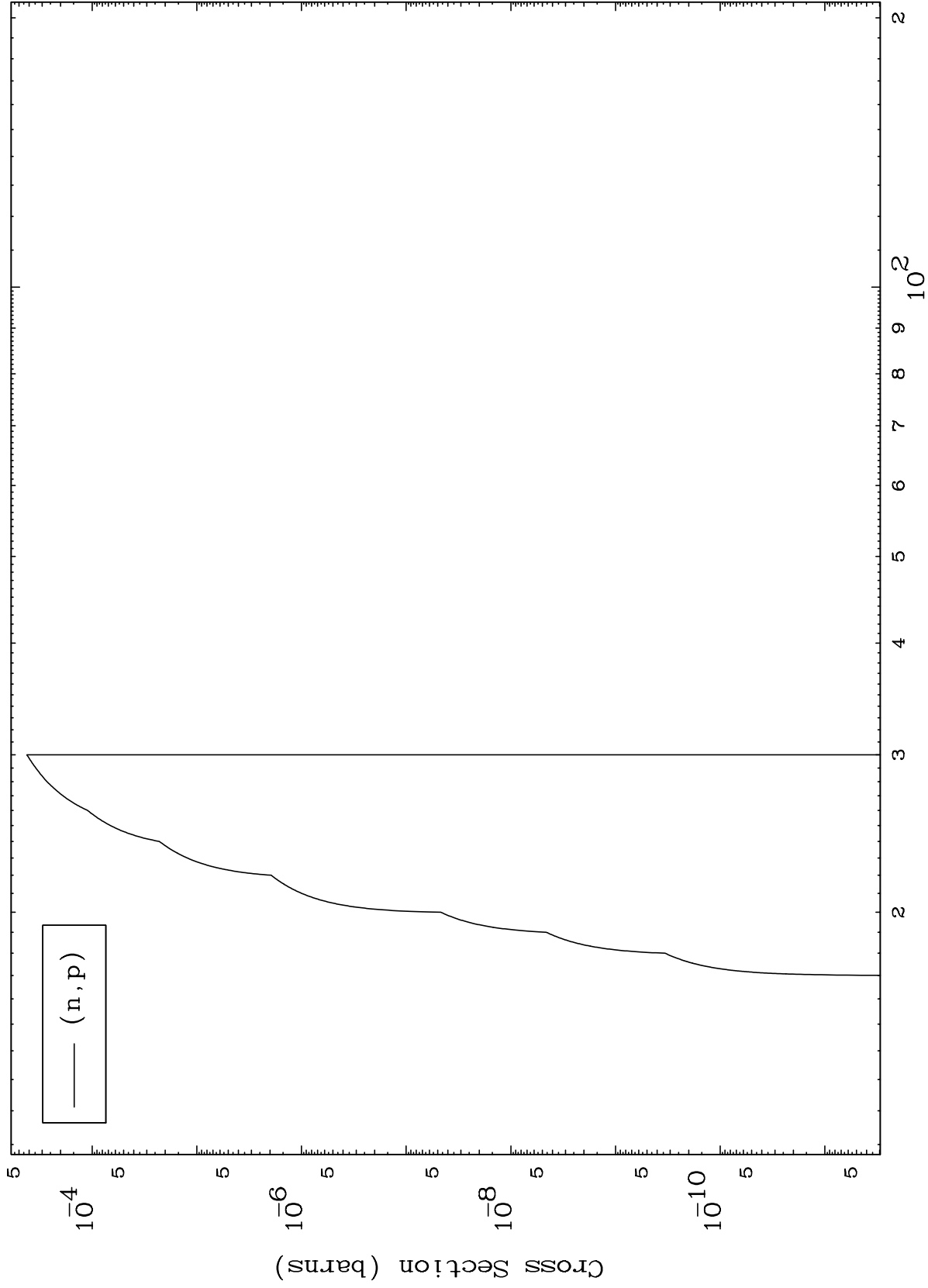




MAT 8143

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

81-Tl-209



5

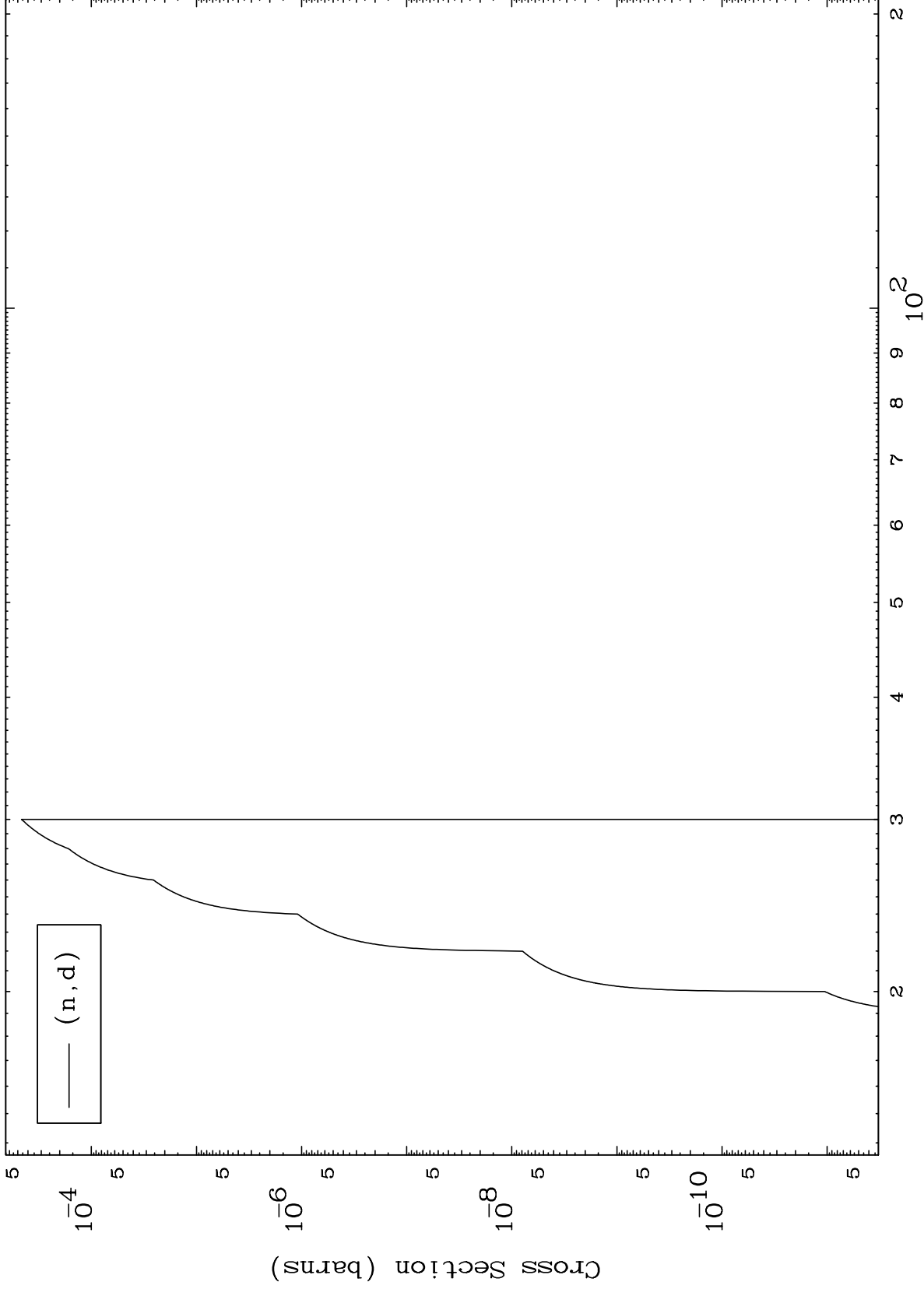
Incident Energy (MeV)

81-Tl-209

MAT 8143

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

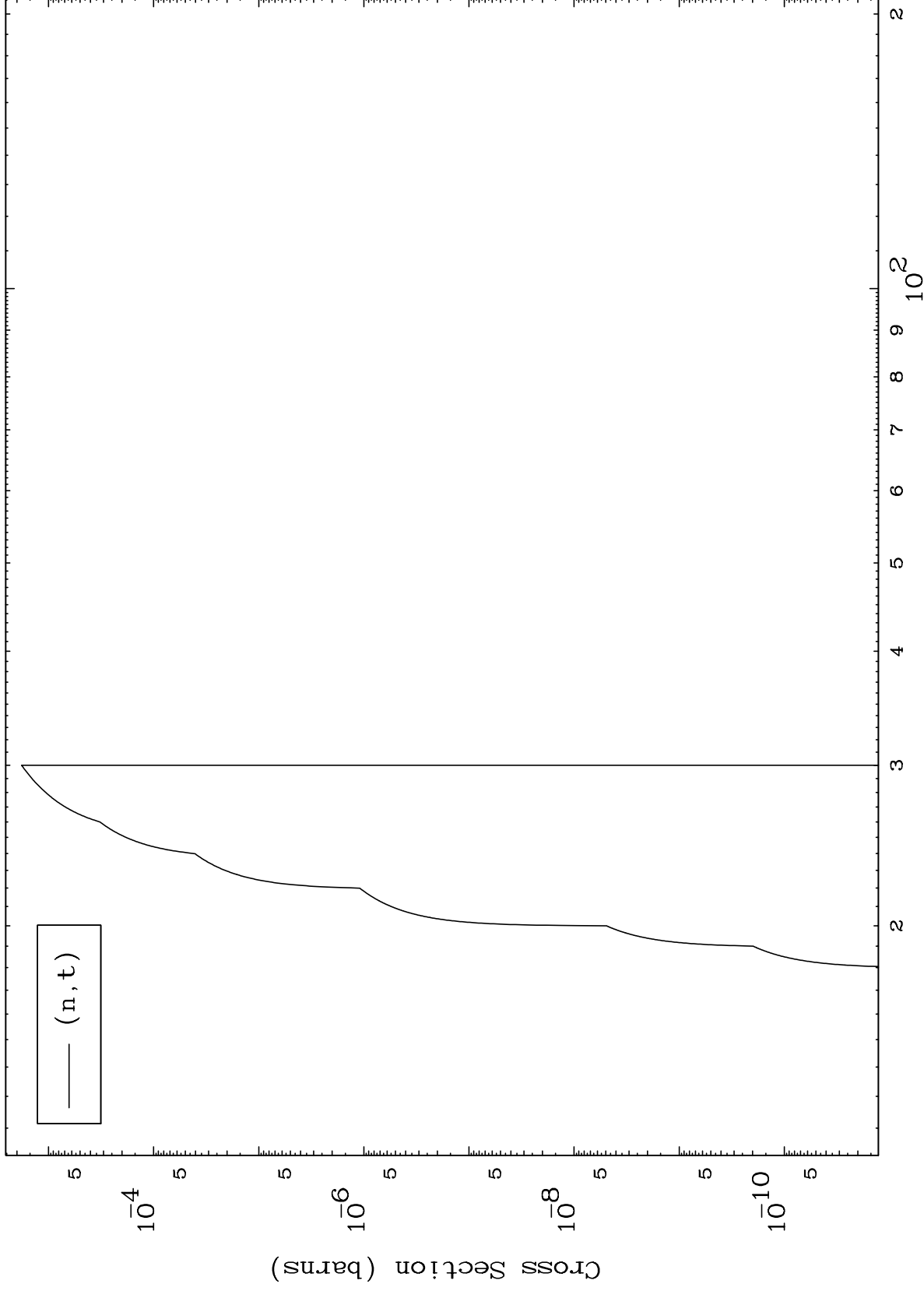
81-Tl-209



6

Incident Energy (MeV)

81-Tl-209

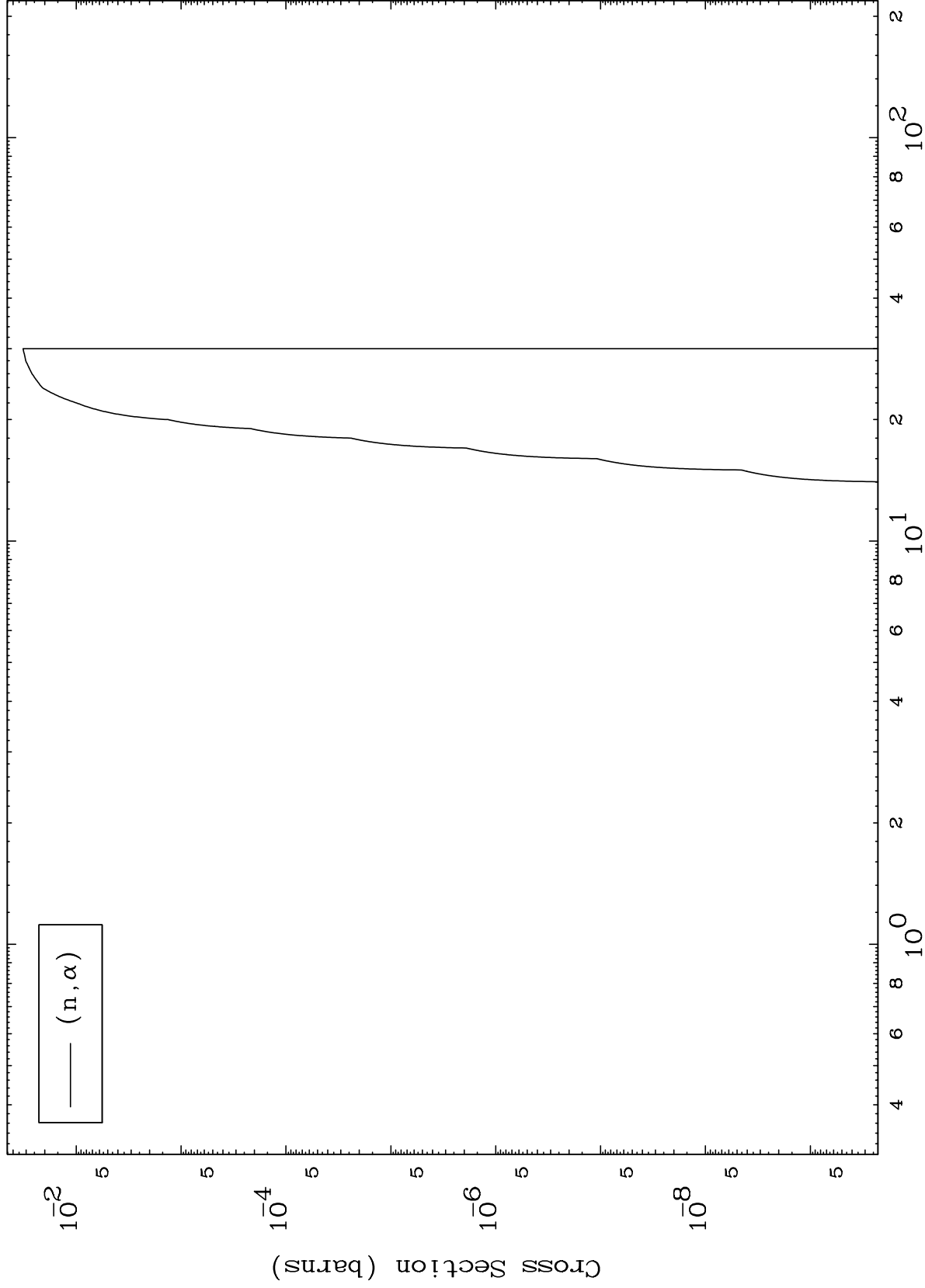


MAT 8143

( $\alpha, \alpha$ ) Levels

81-Tl-209

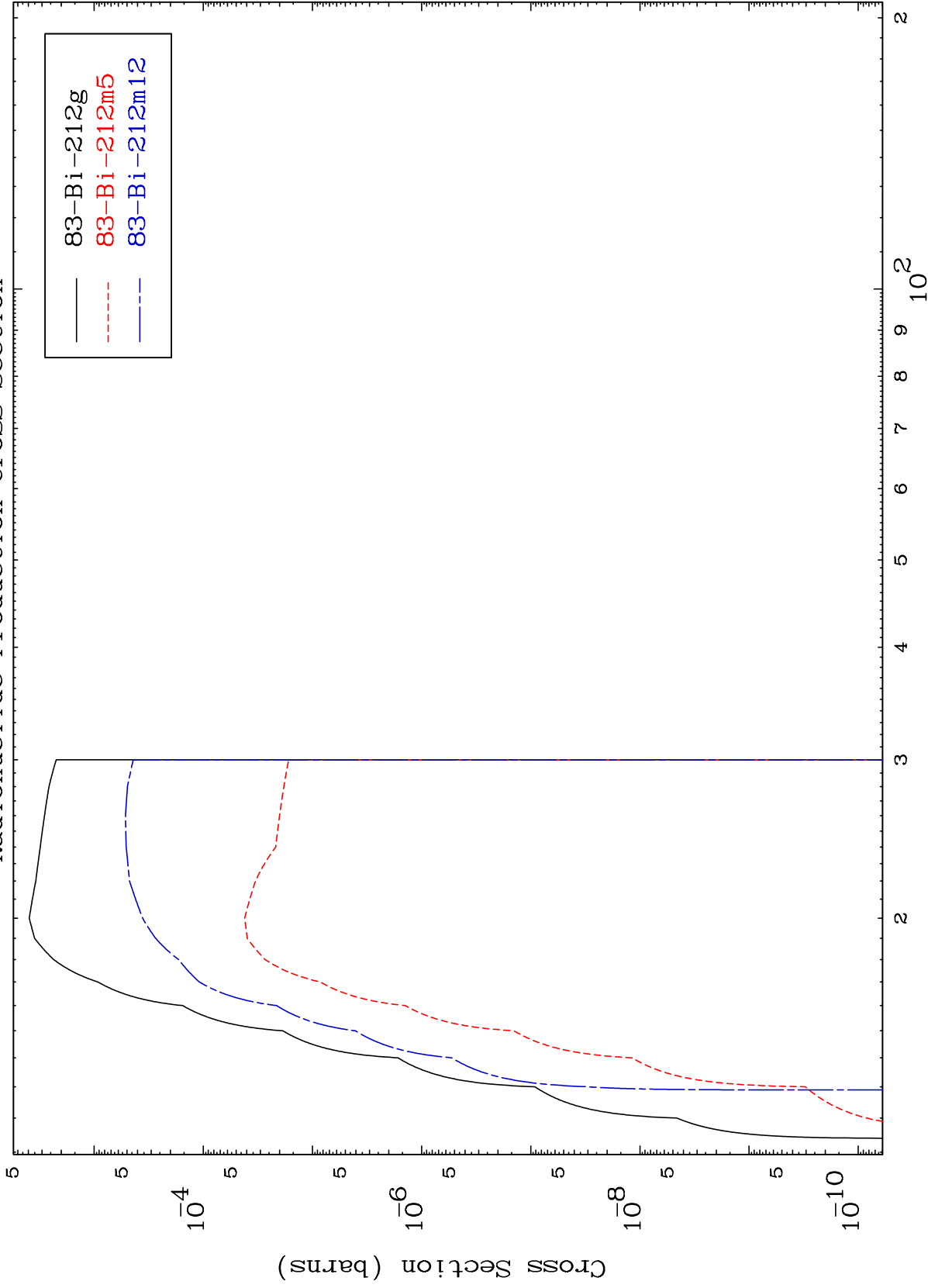
0 Kelvin Cross Sections



8

Incident Energy (MeV)

81-Tl-209

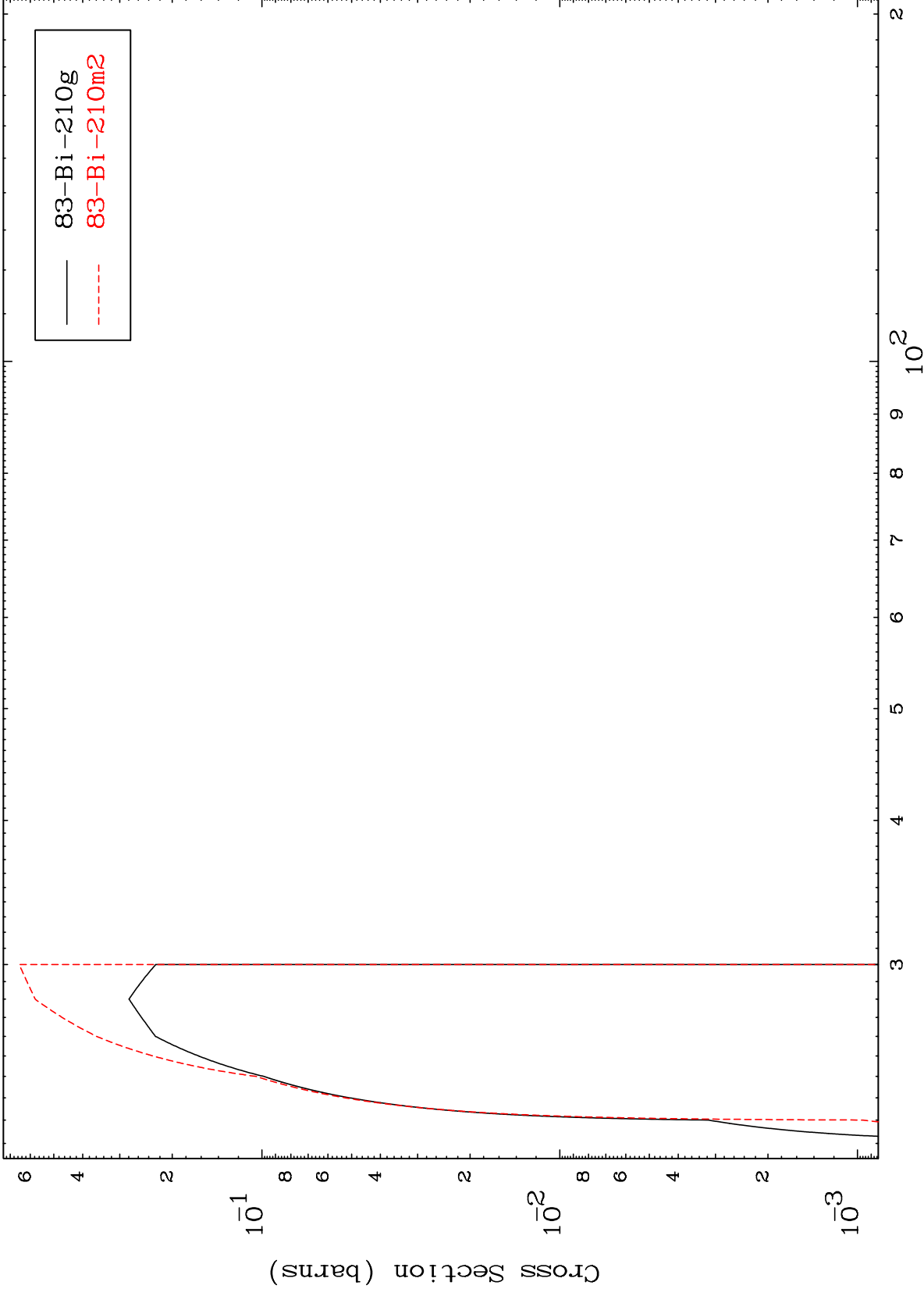


MAT 8143

(n,3n)

81-Tl-209

Radionuclide Production Cross Section



83-Bi-210g  
83-Bi-210m2

10

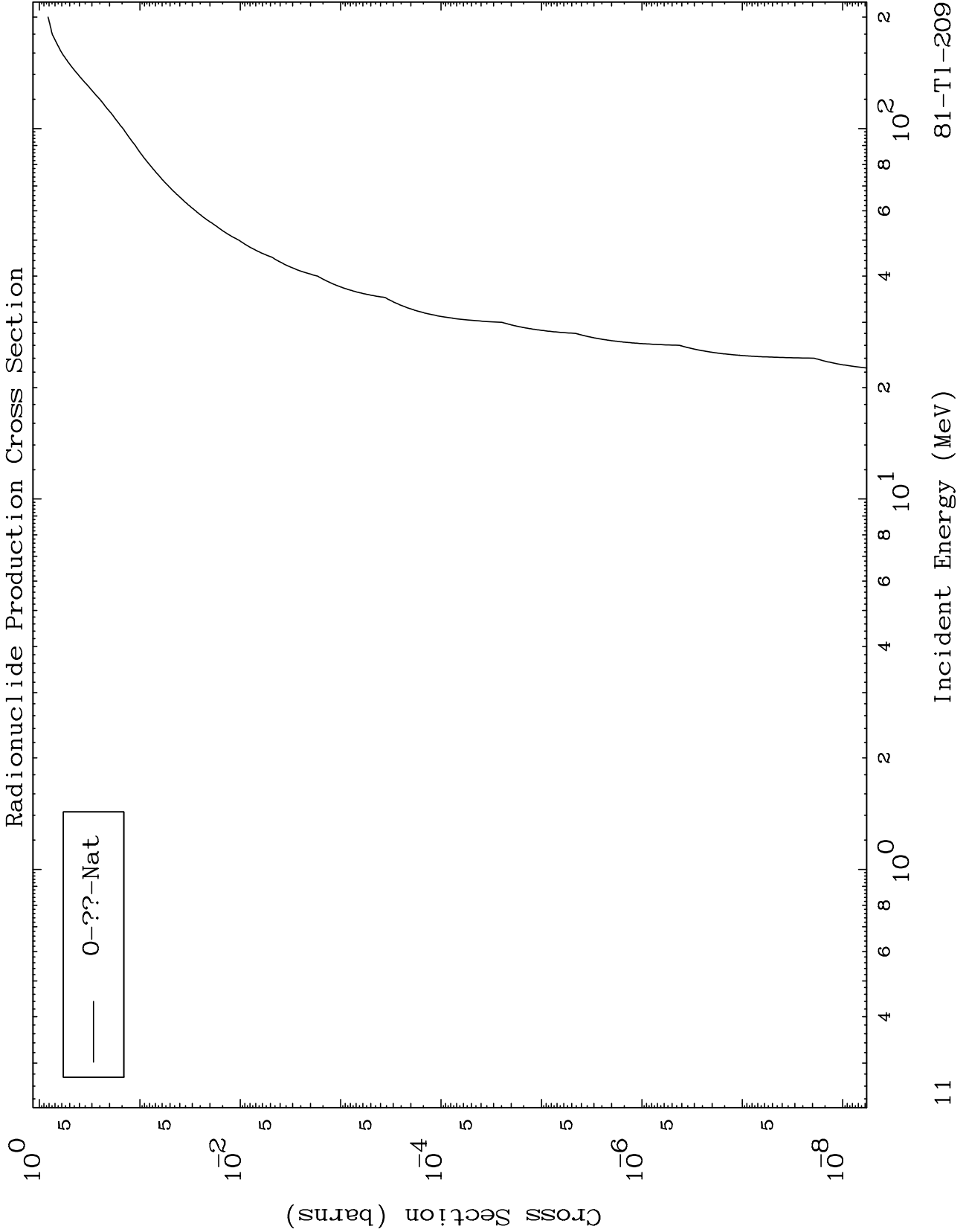
Incident Energy (MeV)

81-Tl-209

MAT 8143

Fission

81-Tl-209

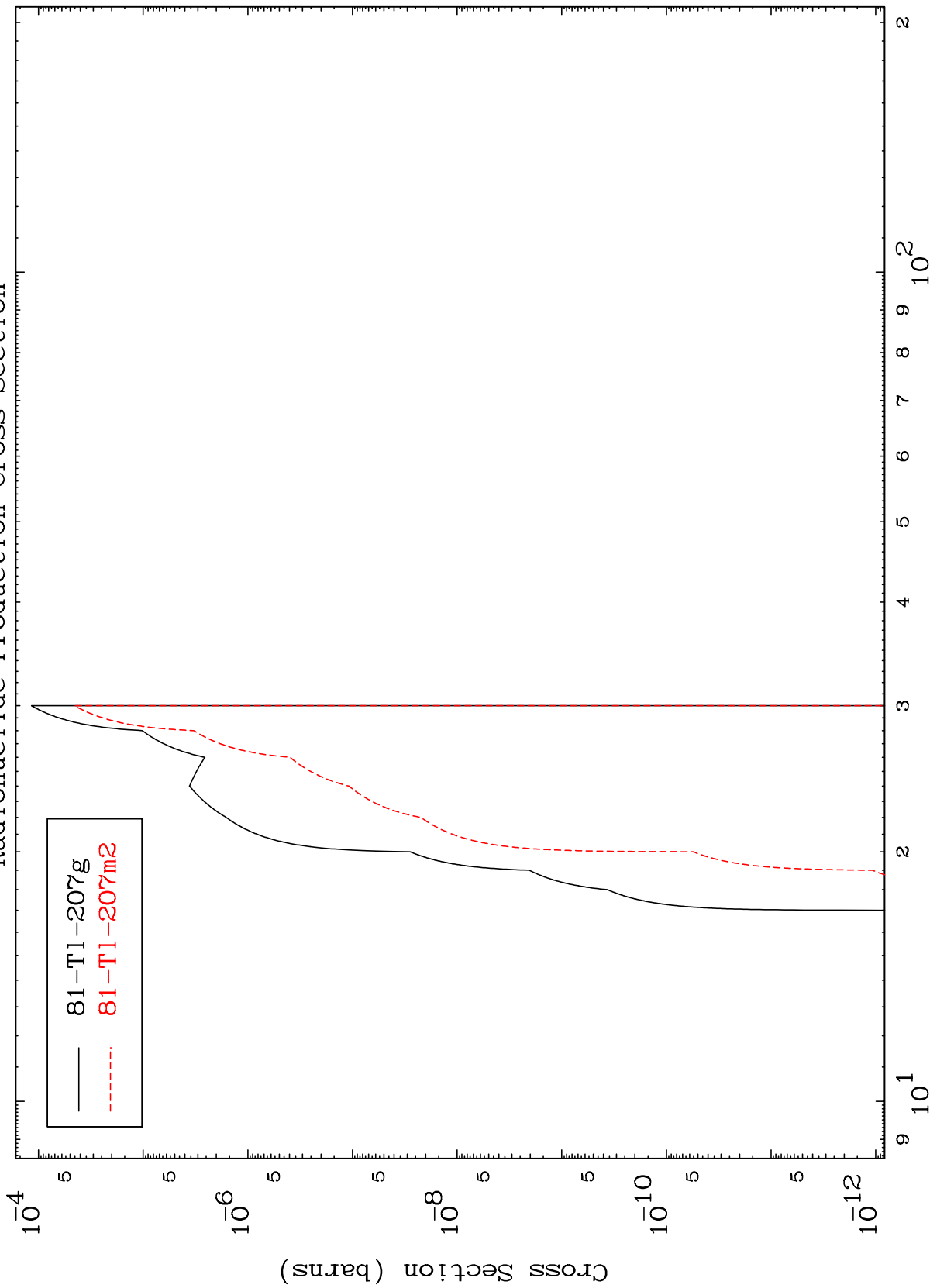


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(n,2n)  $\alpha$

81-Tl-209

Radionuclide Production Cross Section



12

Incident Energy (MeV)

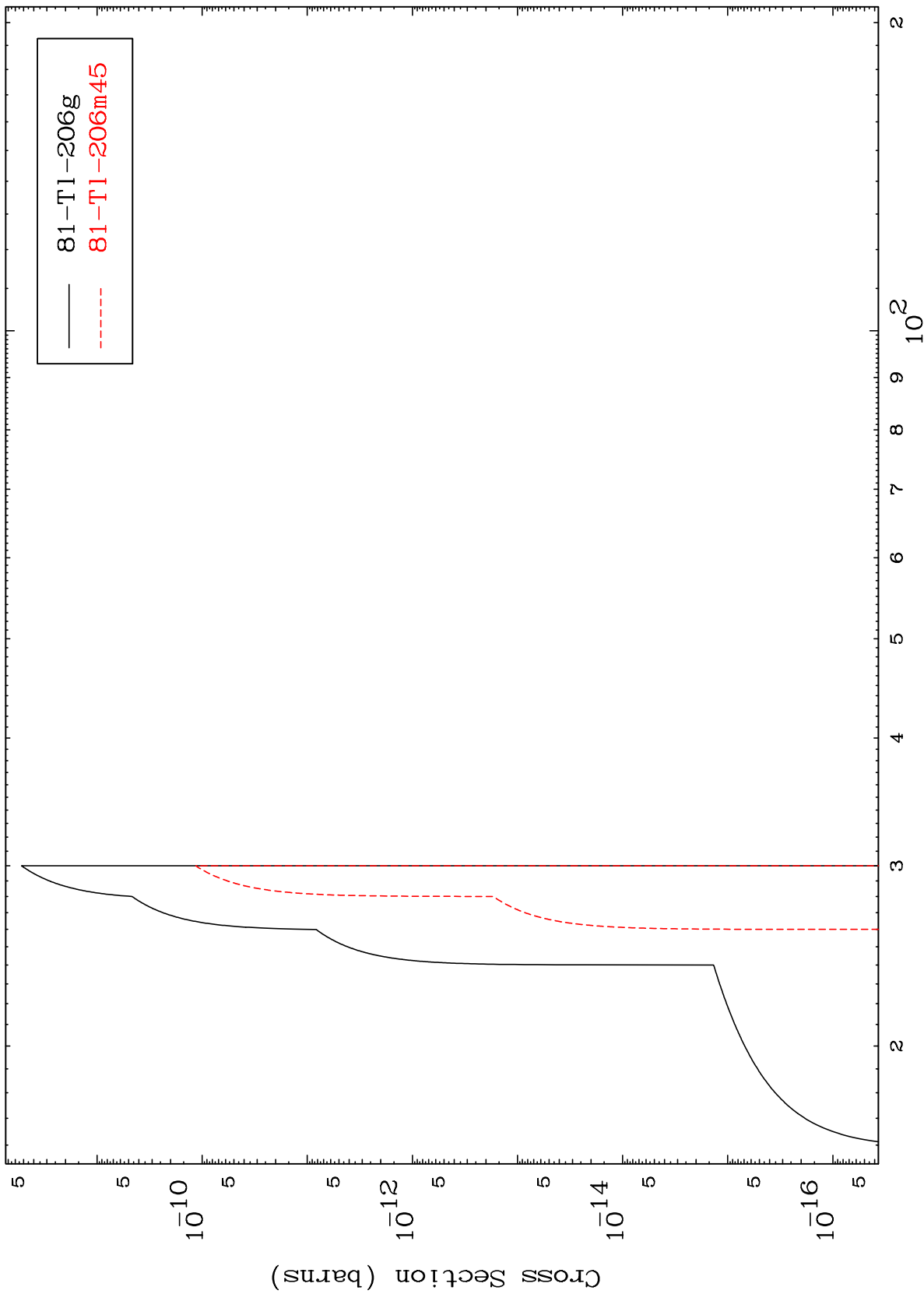
81-Tl-209

MAT 8143

(n,3n)  $\alpha$

81-Tl-209

Radionuclide Production Cross Section



13

Incident Energy (MeV)

81-Tl-209