

Program EVALPLOT
(Version 2021-1)

by

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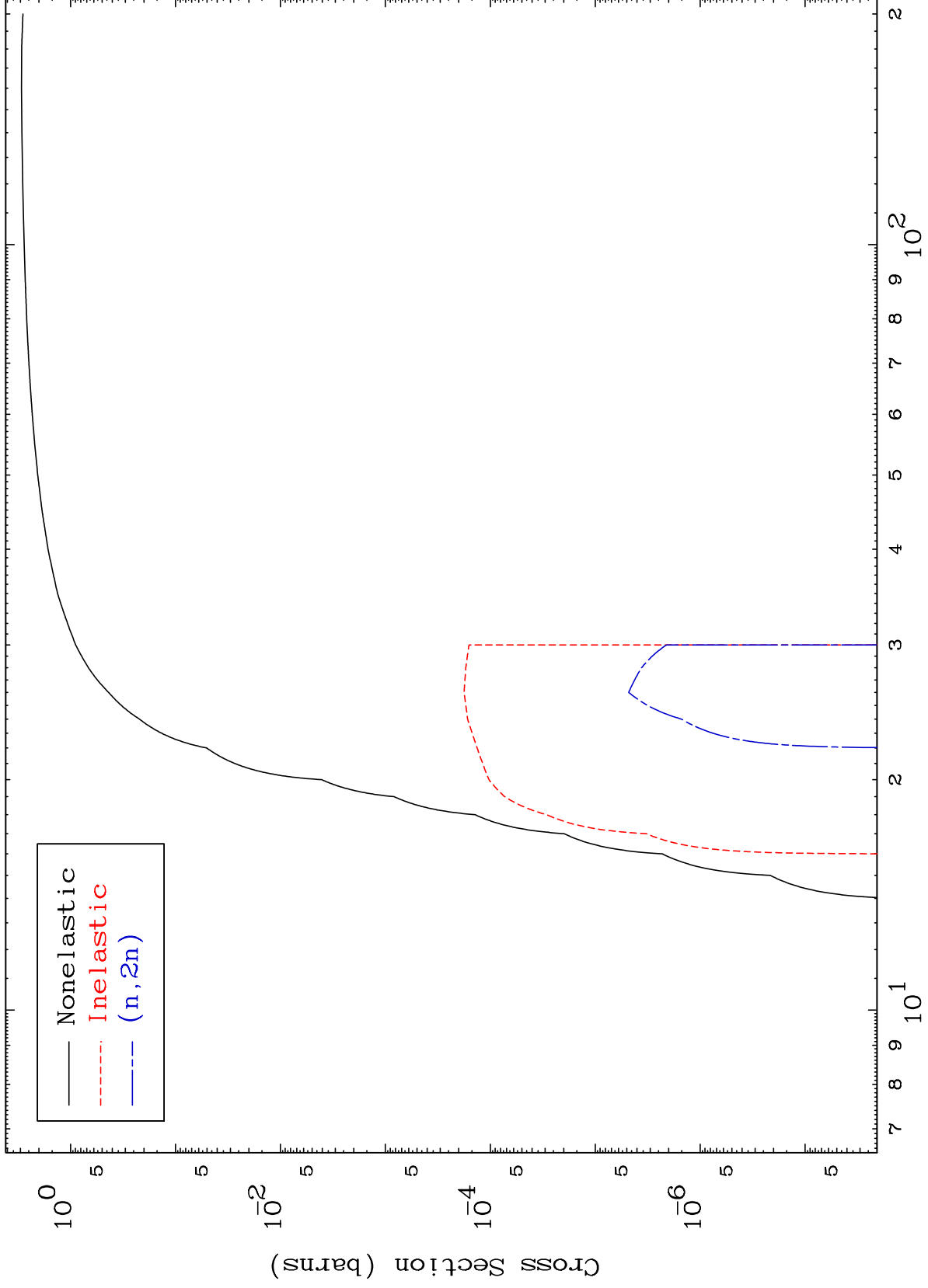
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9207

0 Kelvin Cross Sections

92-U -228



1

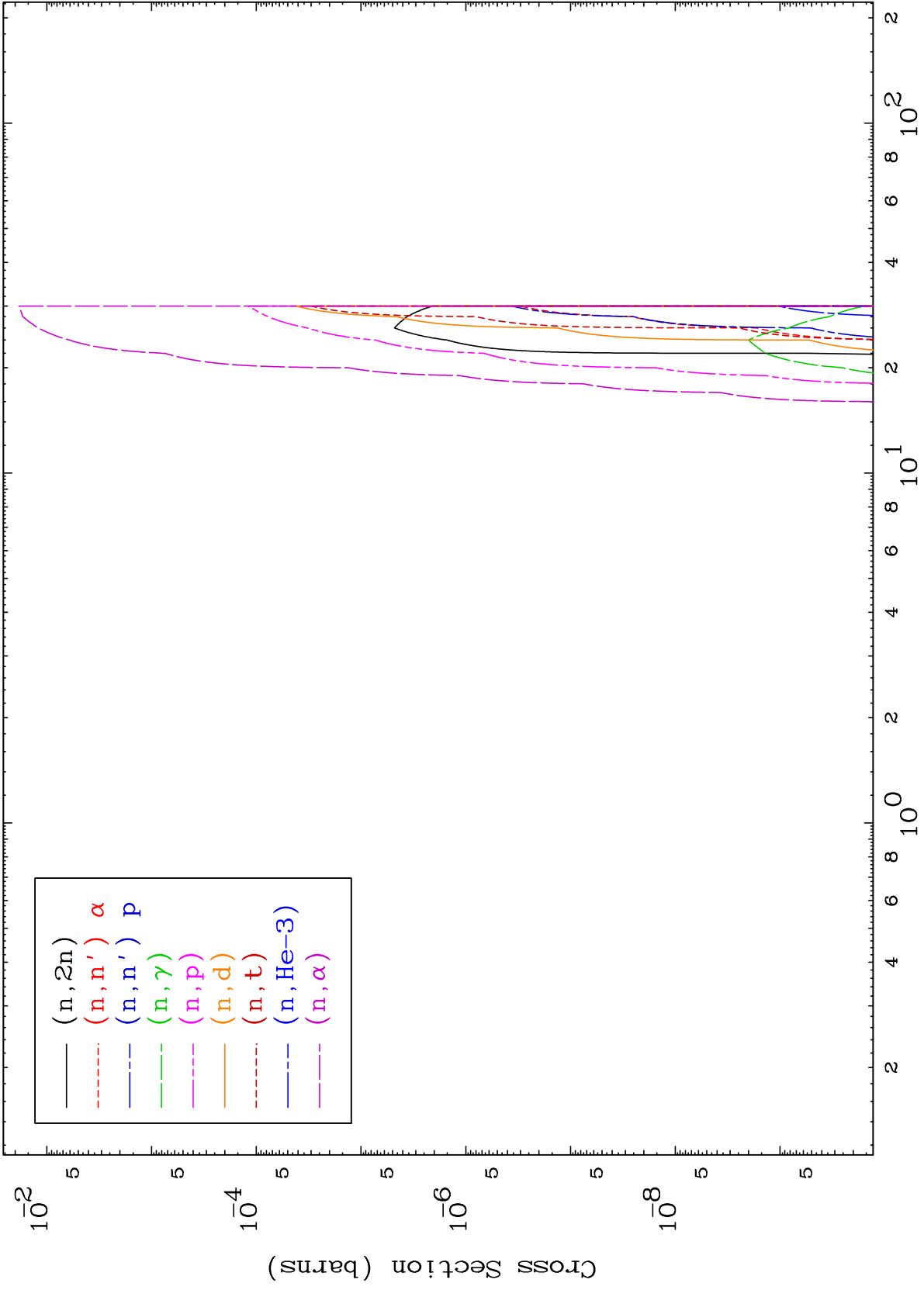
Incident Energy (MeV)

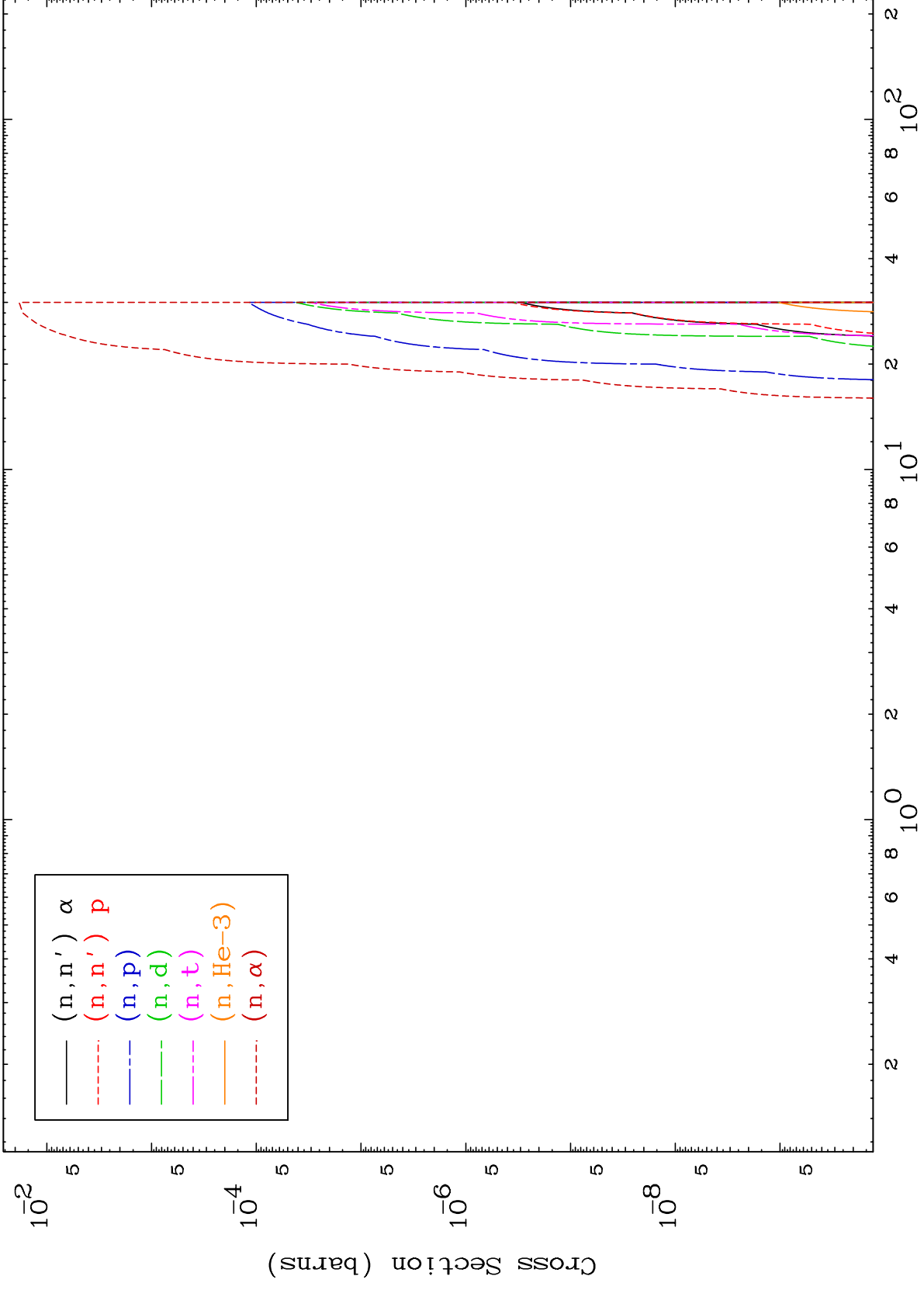
92-U -228

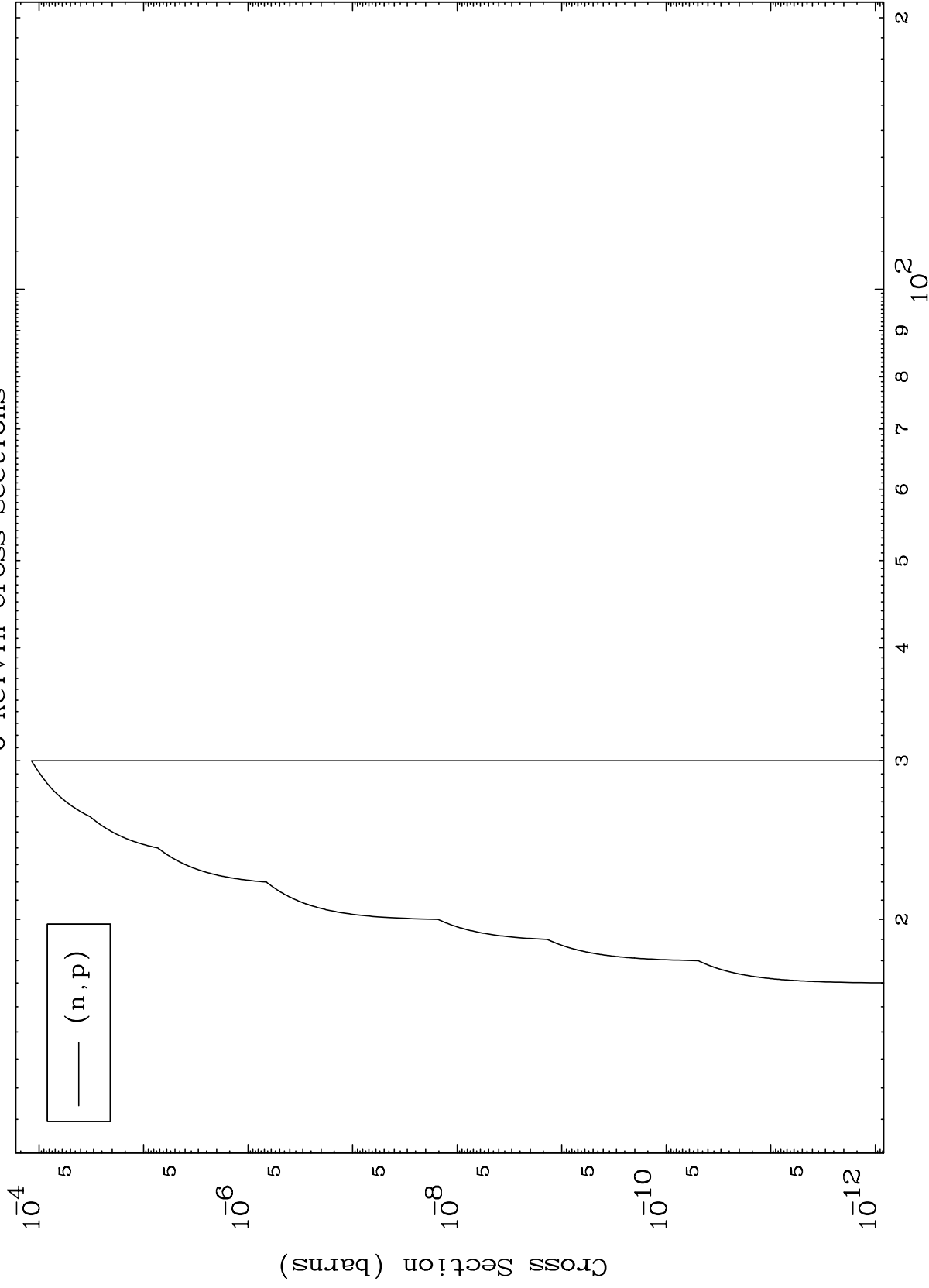
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α Neutron Absorption
0 Kelvin Cross Sections

92-U -228



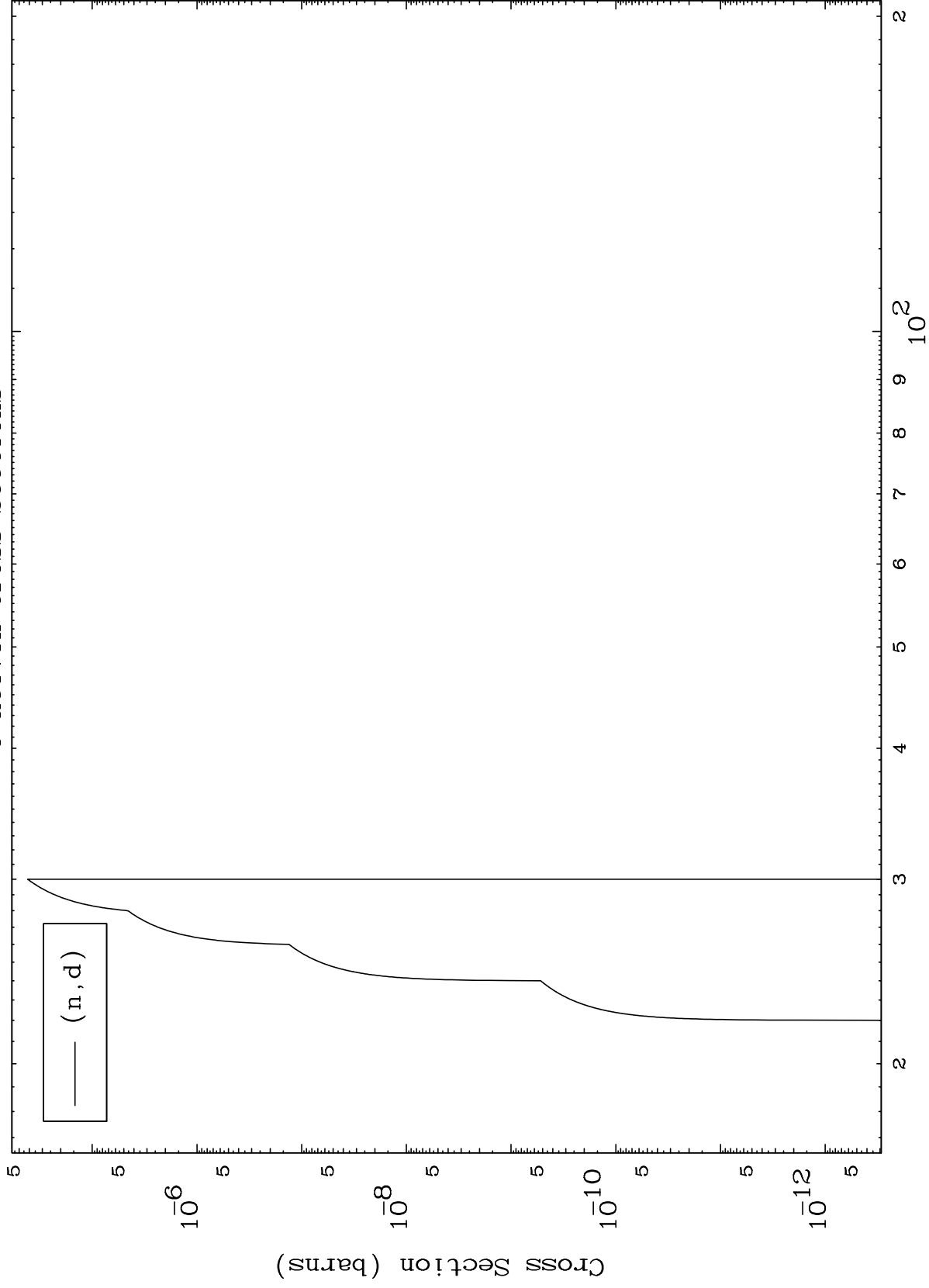




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(α, d) Levels
0 Kelvin Cross Sections

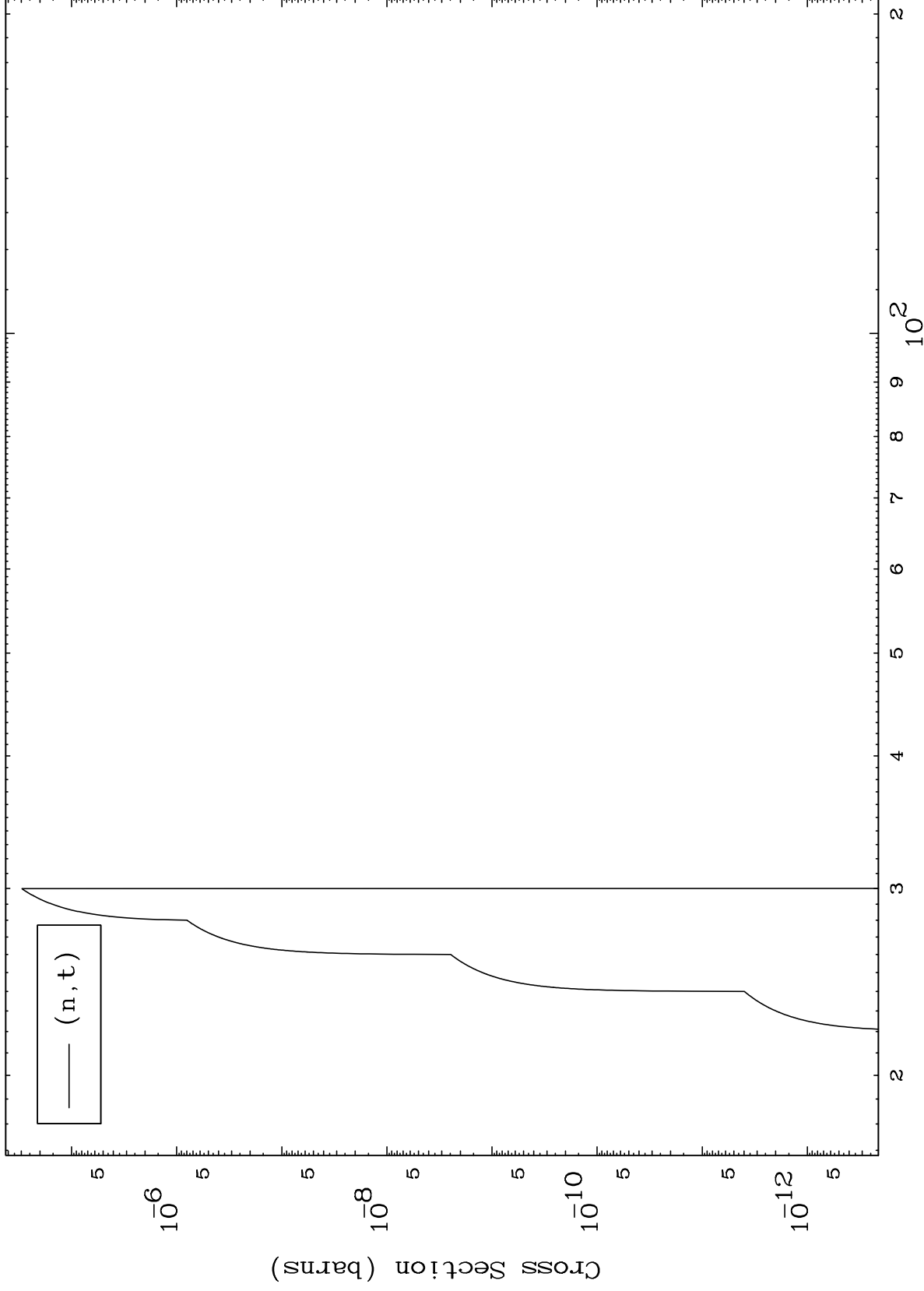
92-U -228

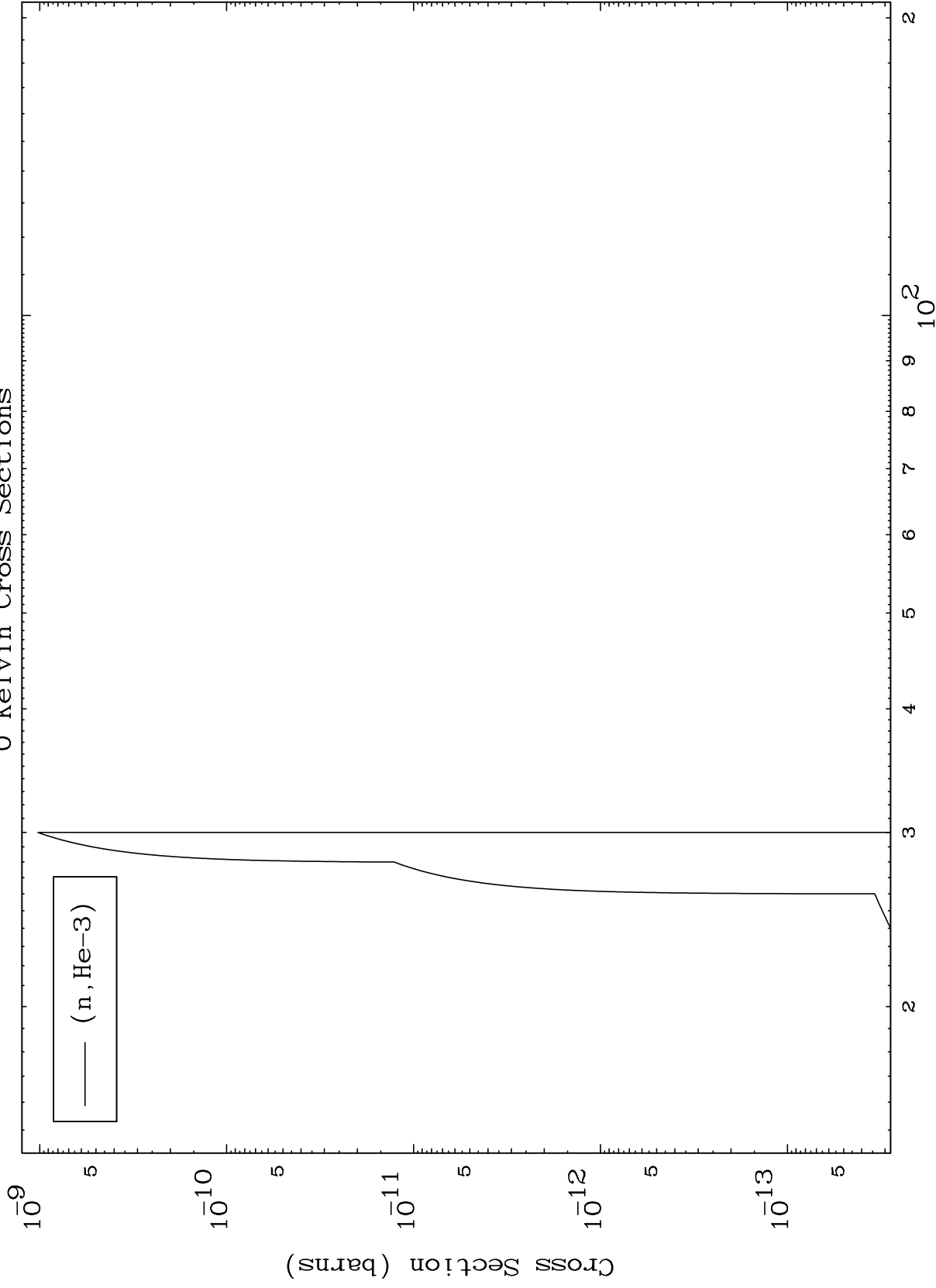


5

Incident Energy (MeV)

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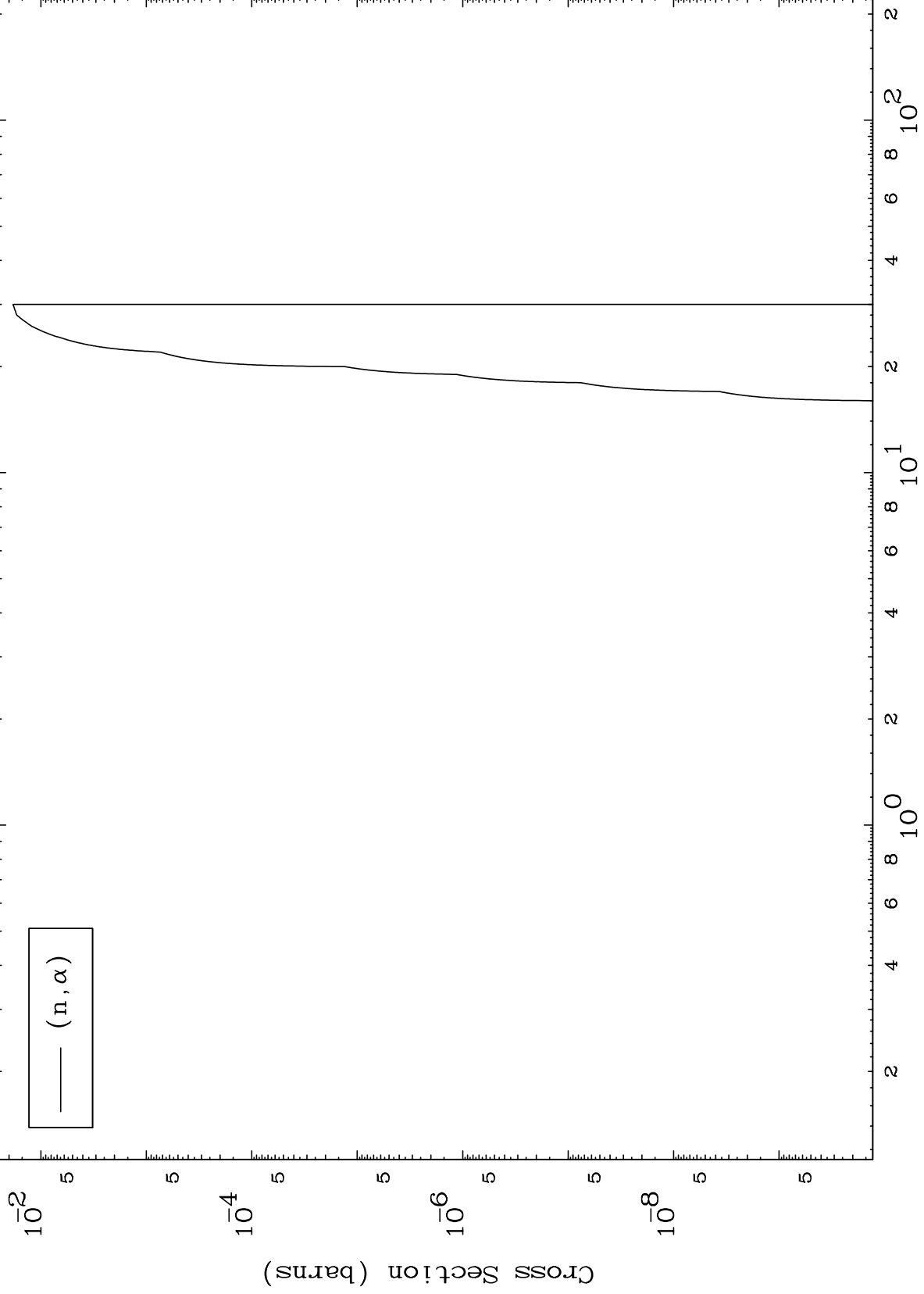


MAT 9207

(α, α) Levels

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0 Kelvin Cross Sections



MAT 9207

Fission

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Radionuclide Production Cross Section

