

Program EVALPLOT  
(Version 2021-1)

by

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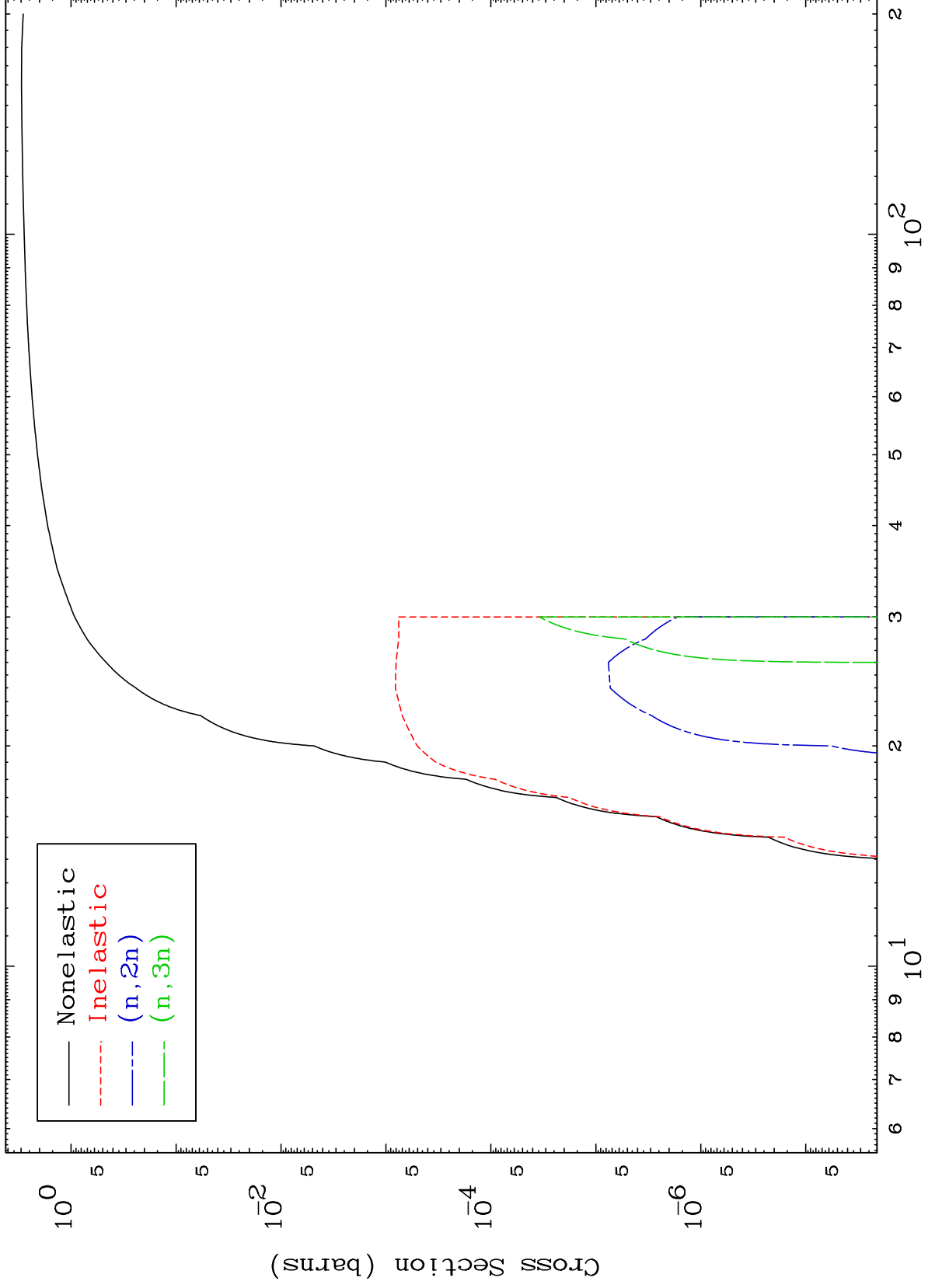
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9222

0 Kelvin Cross Sections

92-U -233



1

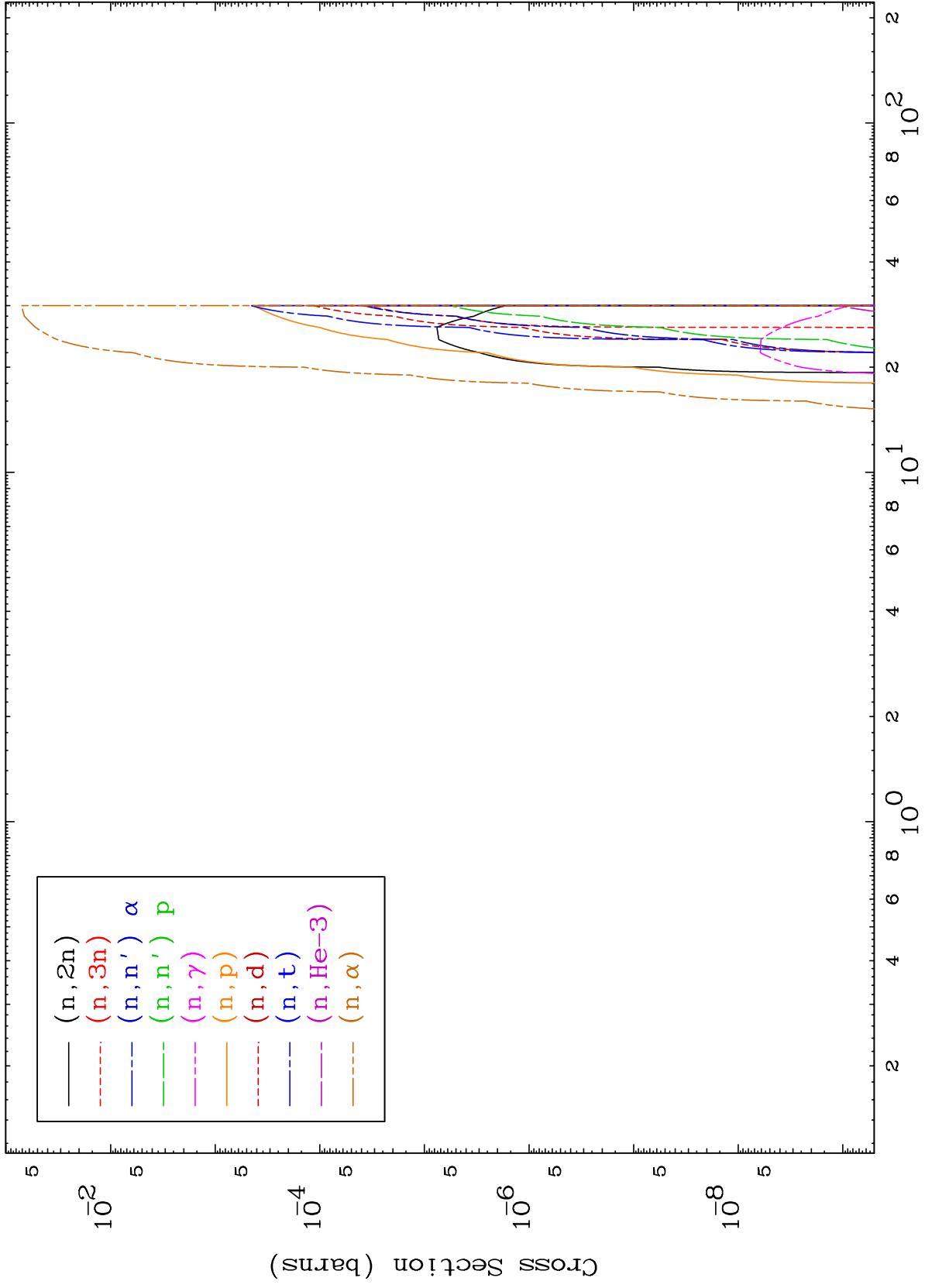
Incident Energy (MeV)

92-U -233

MAT 9222

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

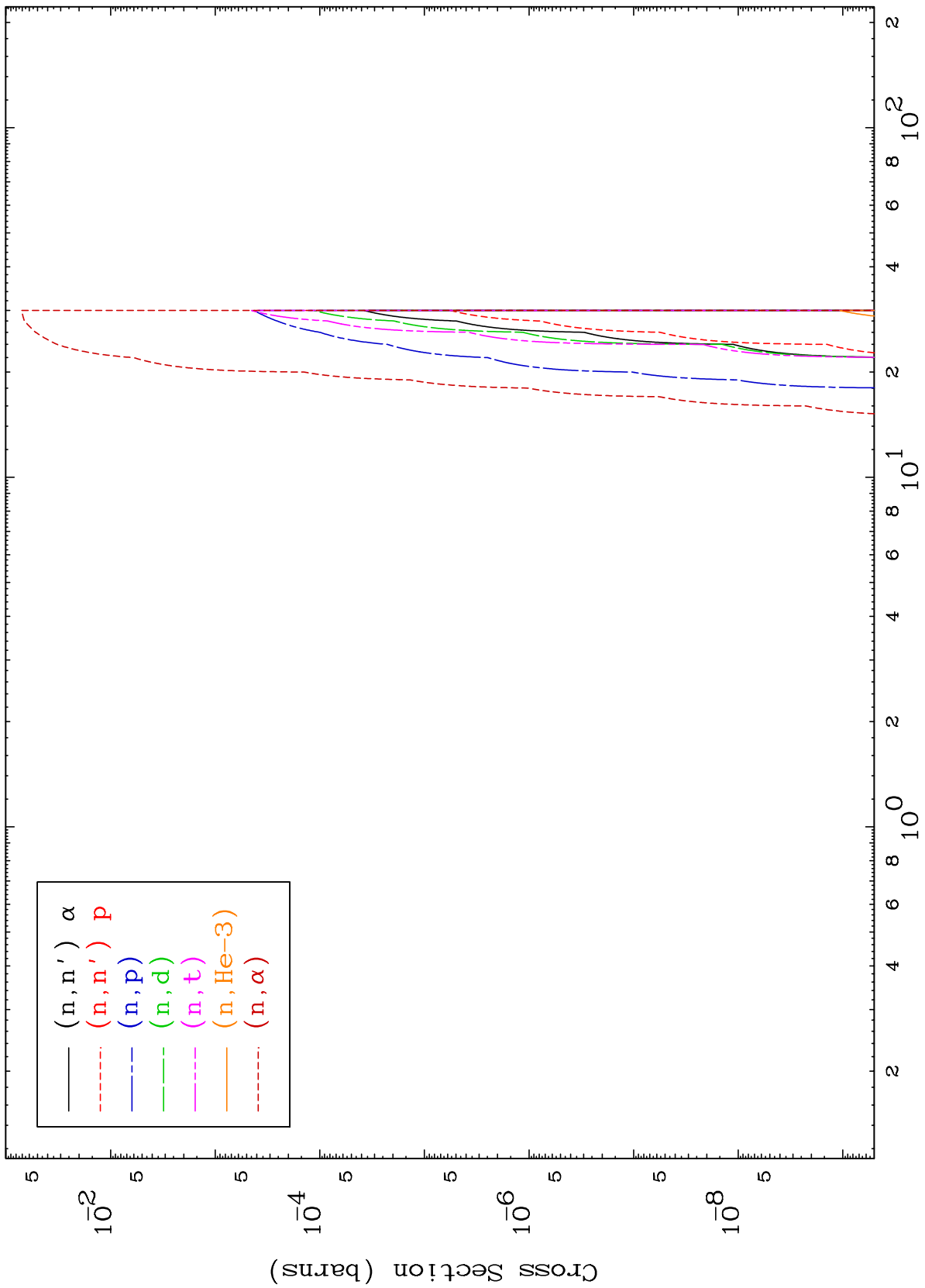
92-U -233



MAT 9222

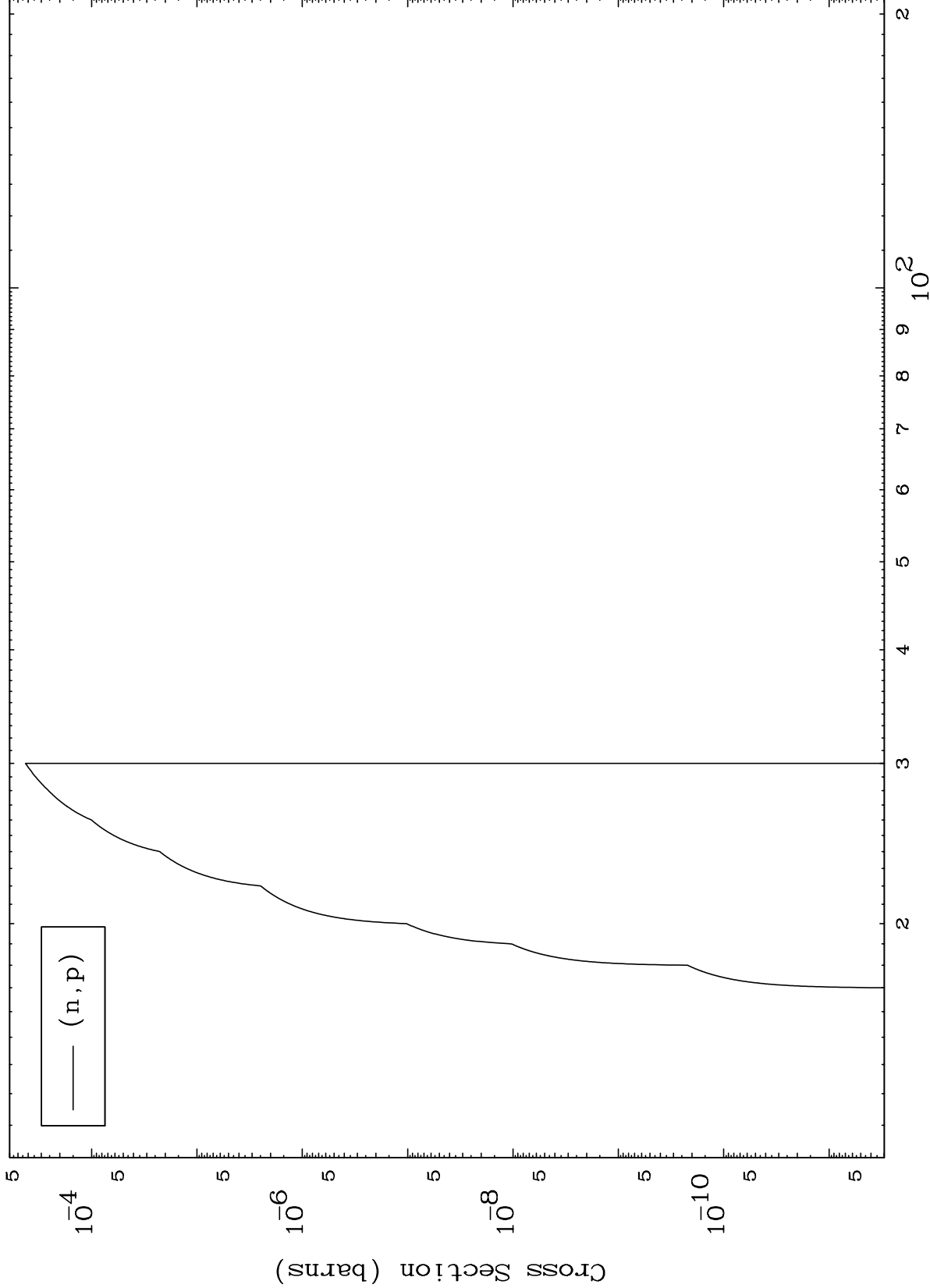
$\alpha$  Charged Particle  
0 Kelvin Cross Sections

92-U -233



Incident Energy (MeV)

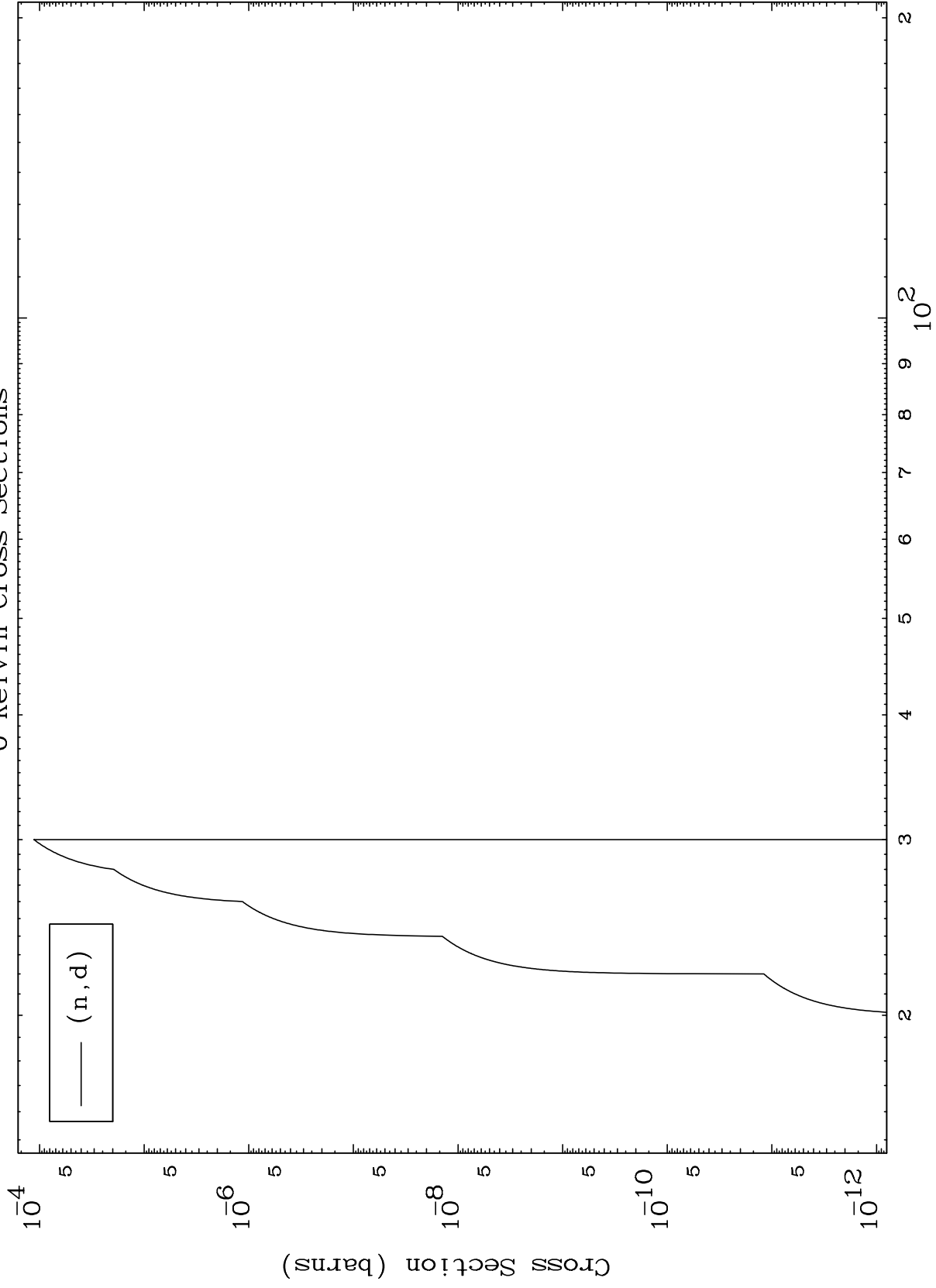
92-U -233



MAT 92222

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

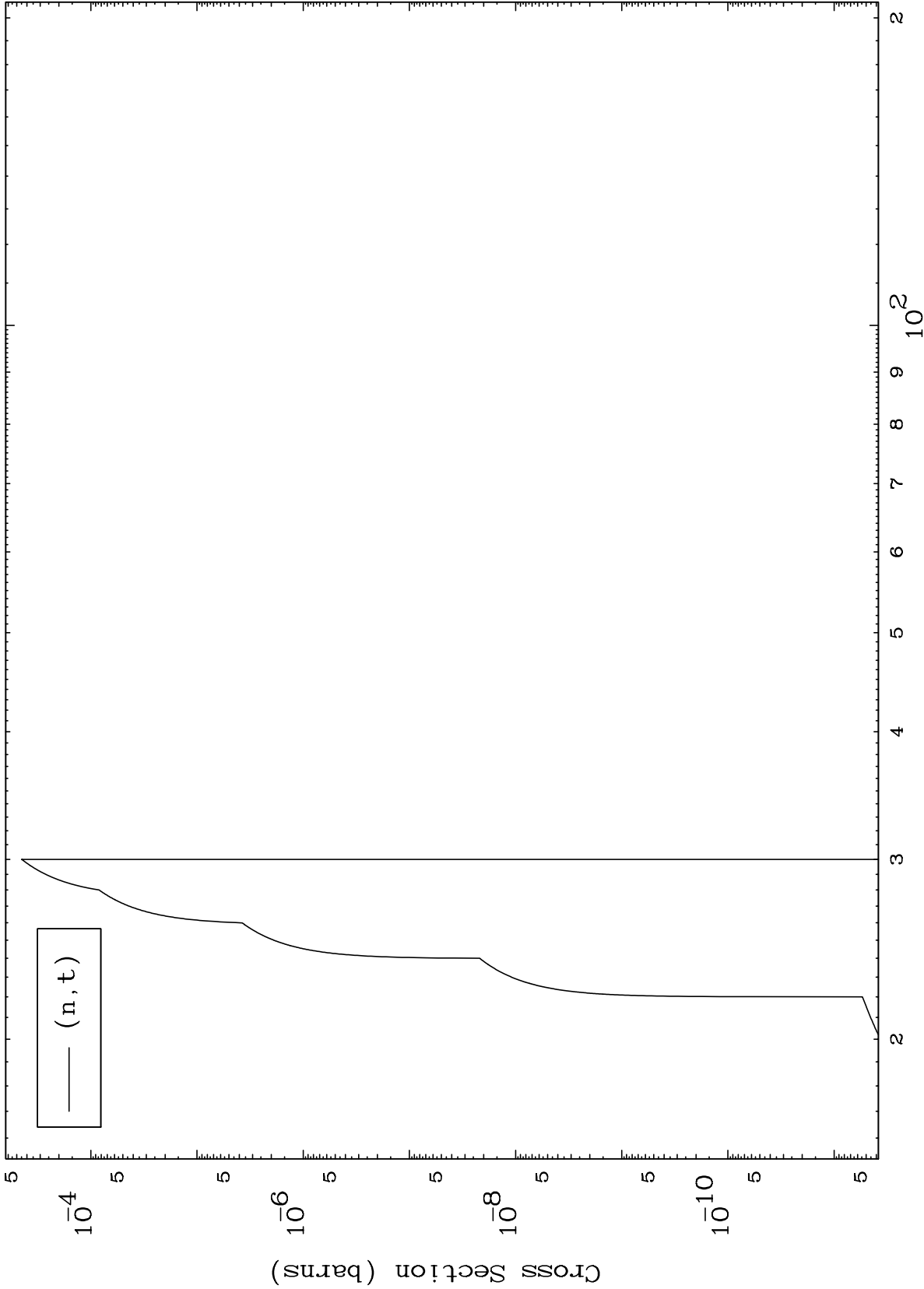
92-U -233

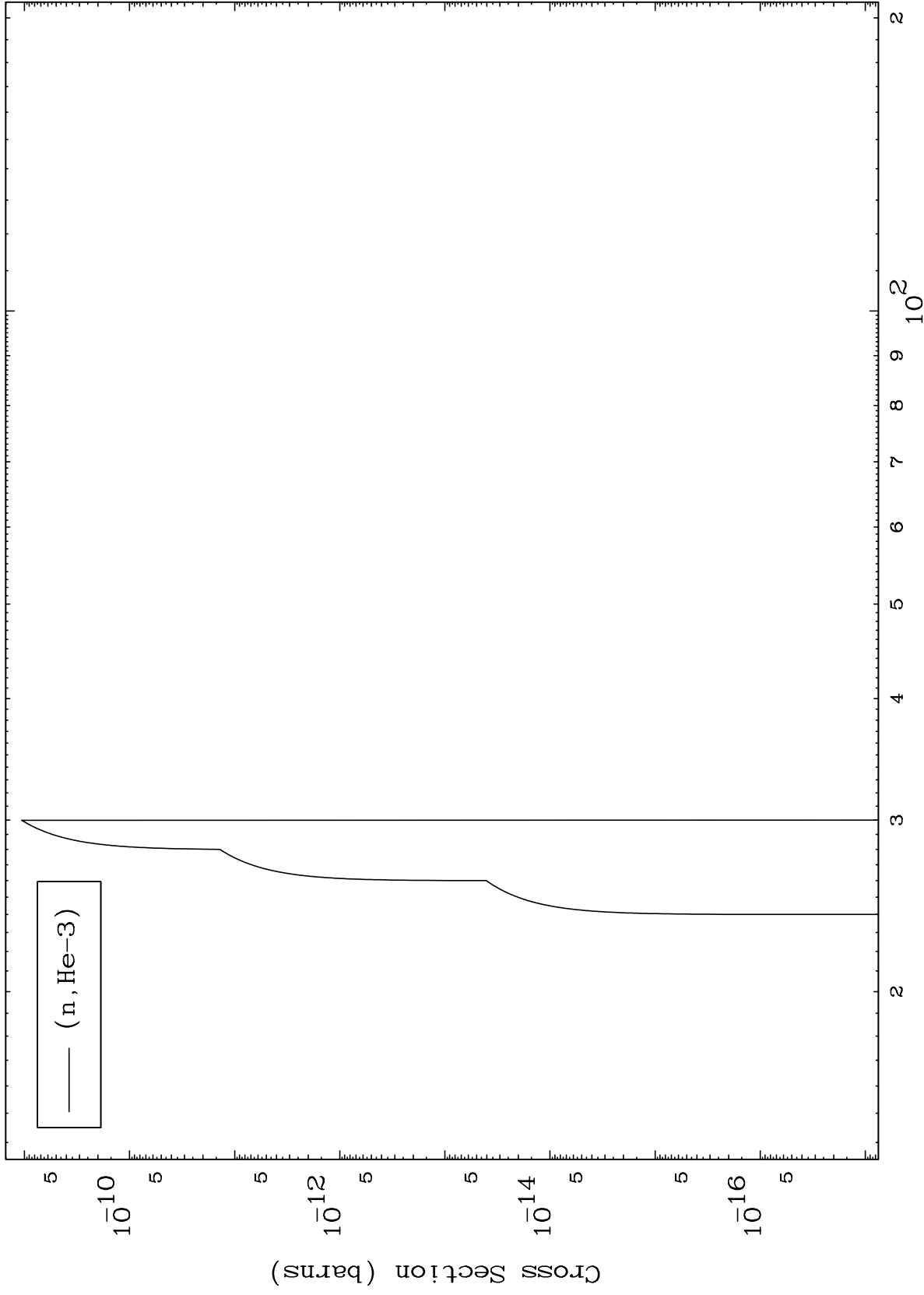


5

Incident Energy (MeV)

92-U -233



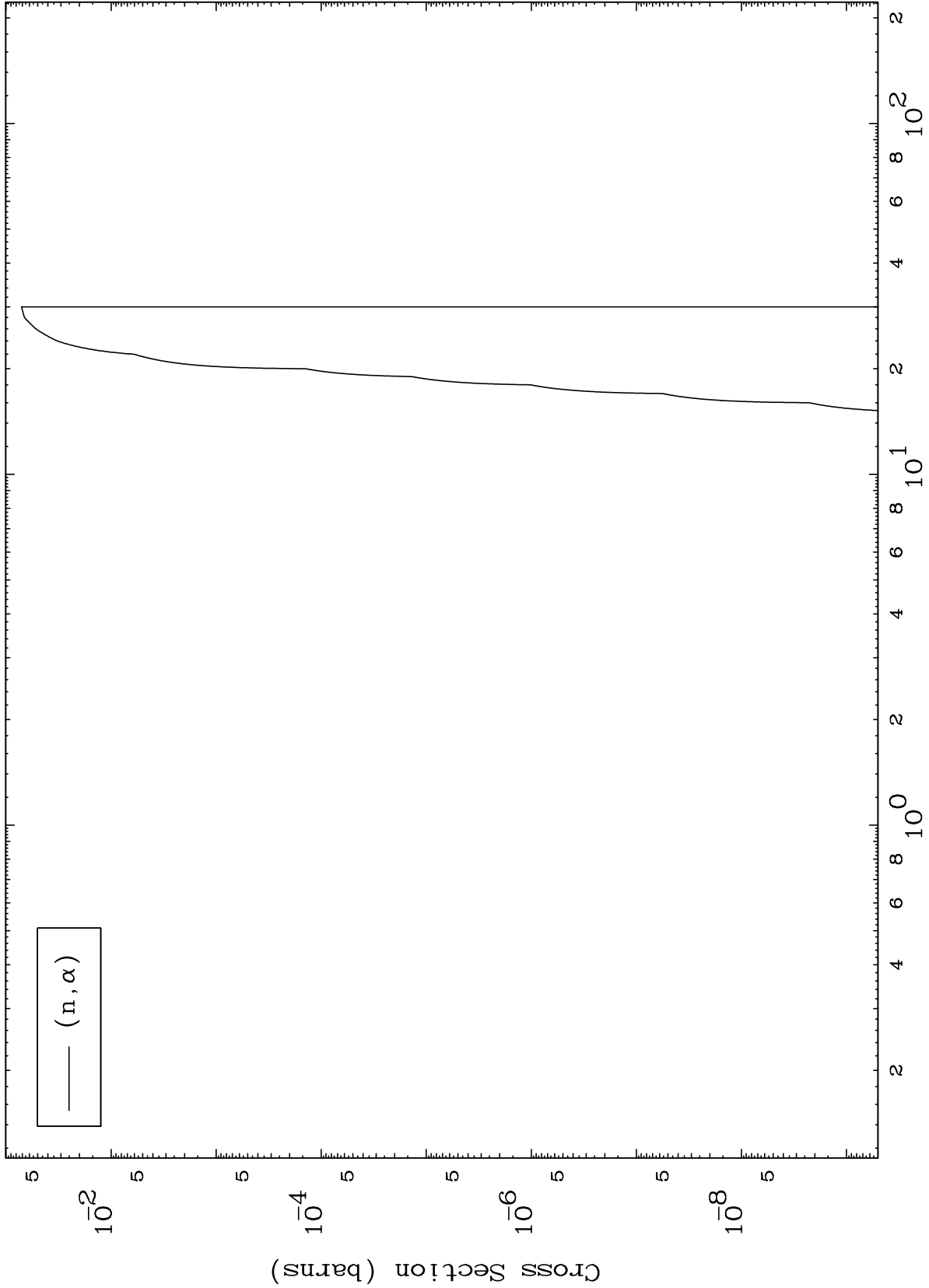


MAT 9222

( $\alpha, \alpha$ ) Levels

92-U -233

0 Kelvin Cross Sections

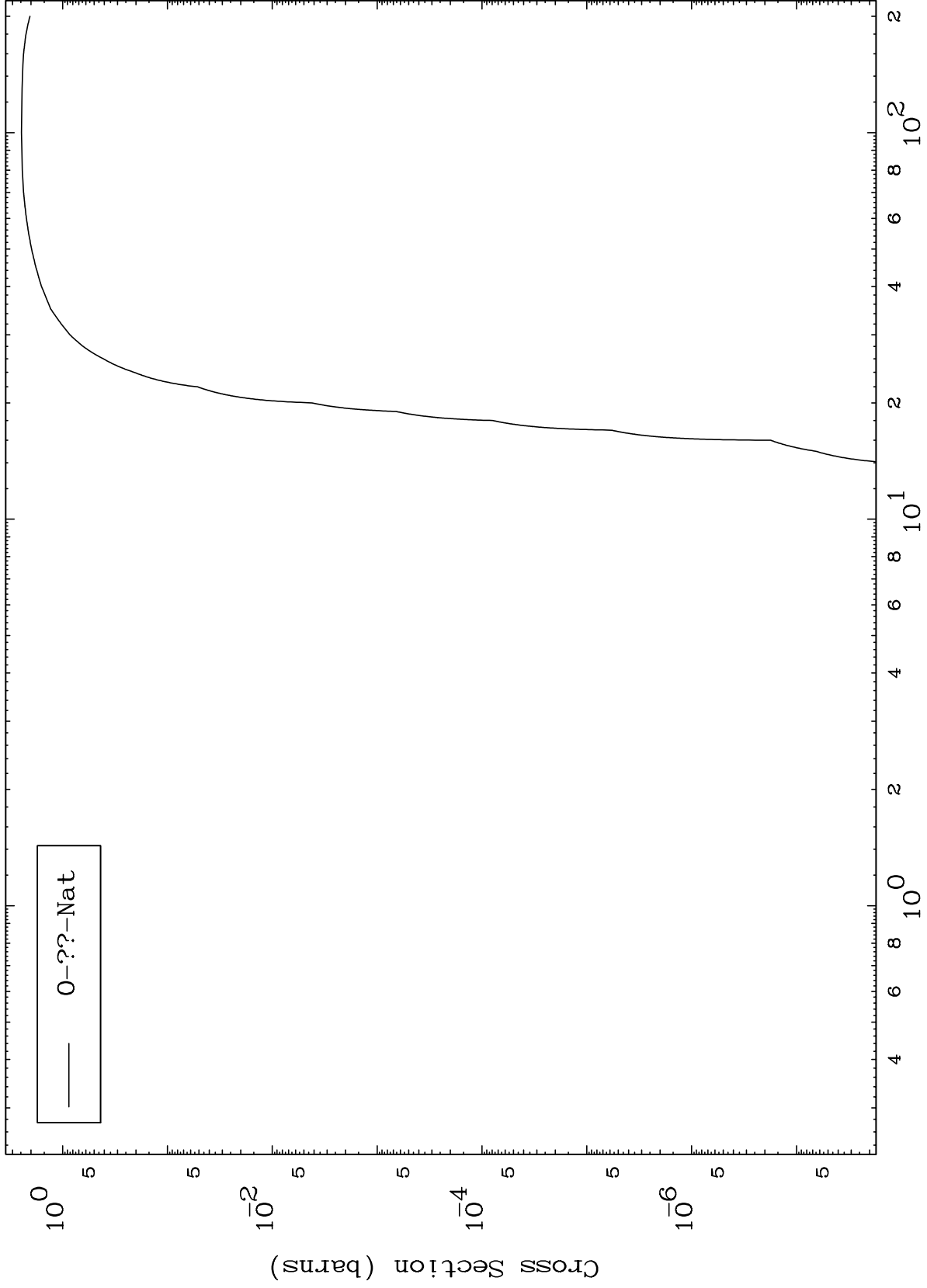


MAT 9222

Fission

92-U -233

Radionuclide Production Cross Section

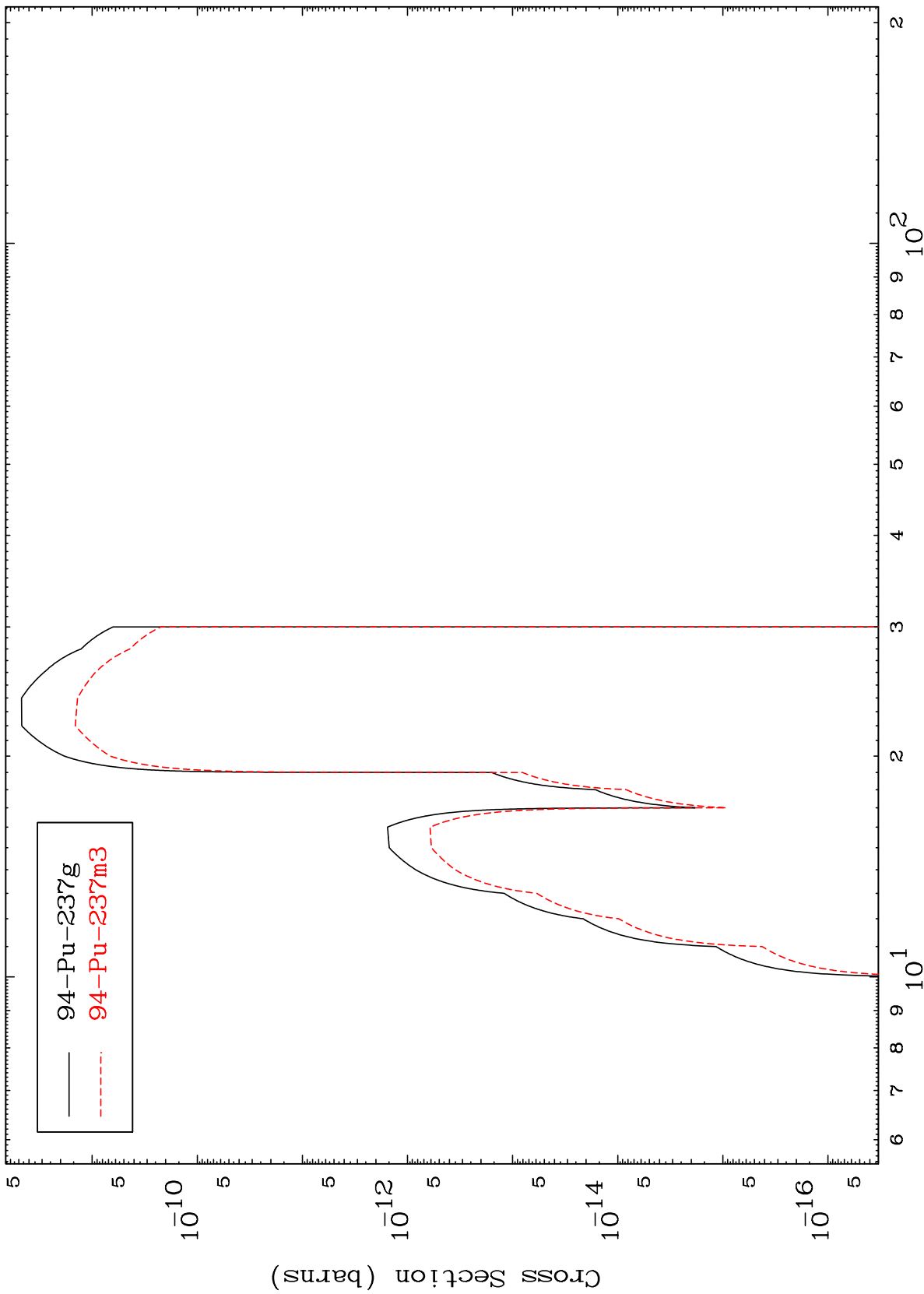


0-??-Nat

MAT 9222

92-U -233

(n,γ)  
Radionuclide Production Cross Section



10

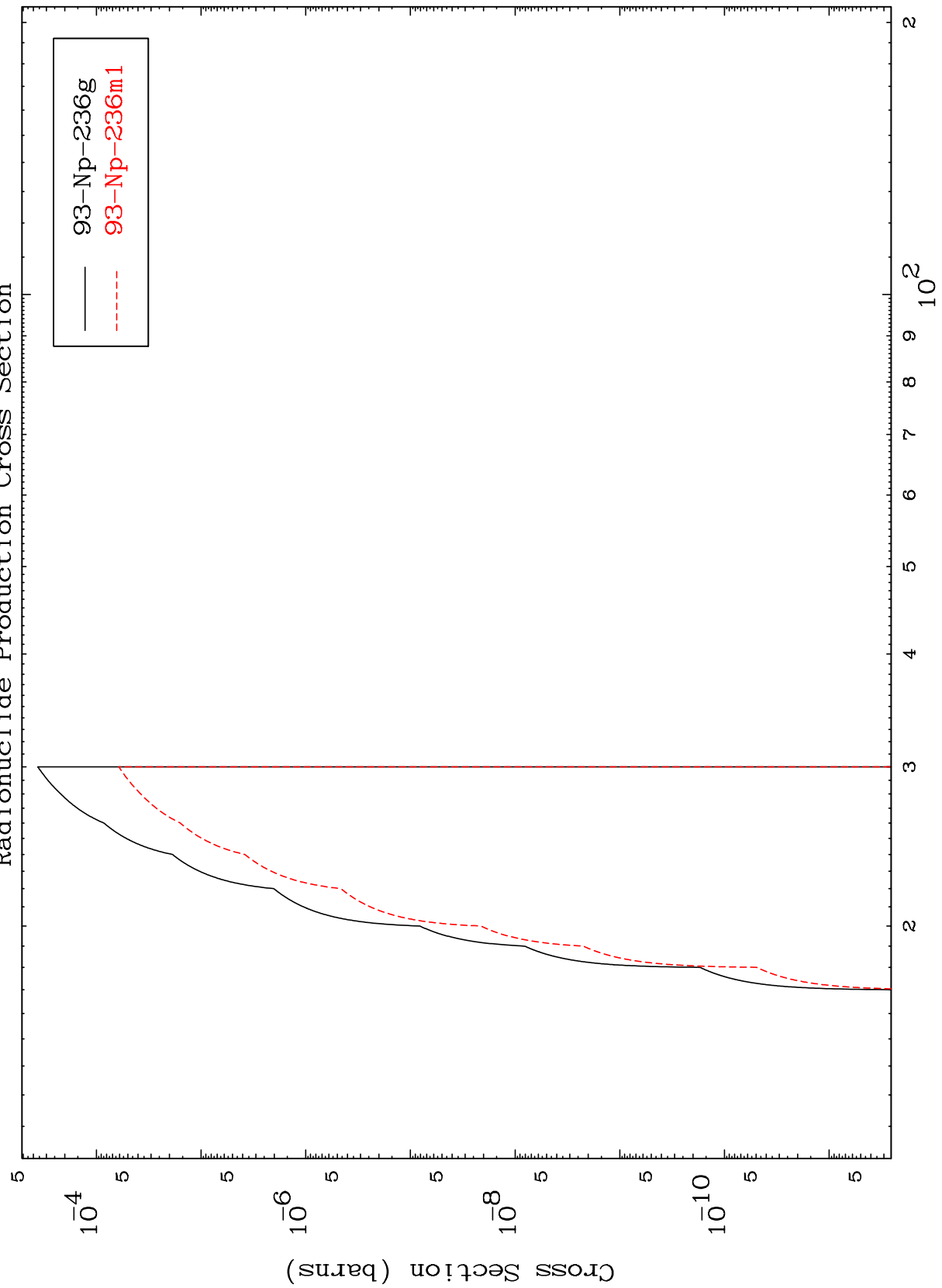
Incident Energy (MeV)

92-U -233

MAT 9222

92-U -233

(n,p)  
Radionuclide Production Cross Section



11

Incident Energy (MeV)

92-U -233