

Program EVALPLOT  
(Version 2021-1)

by

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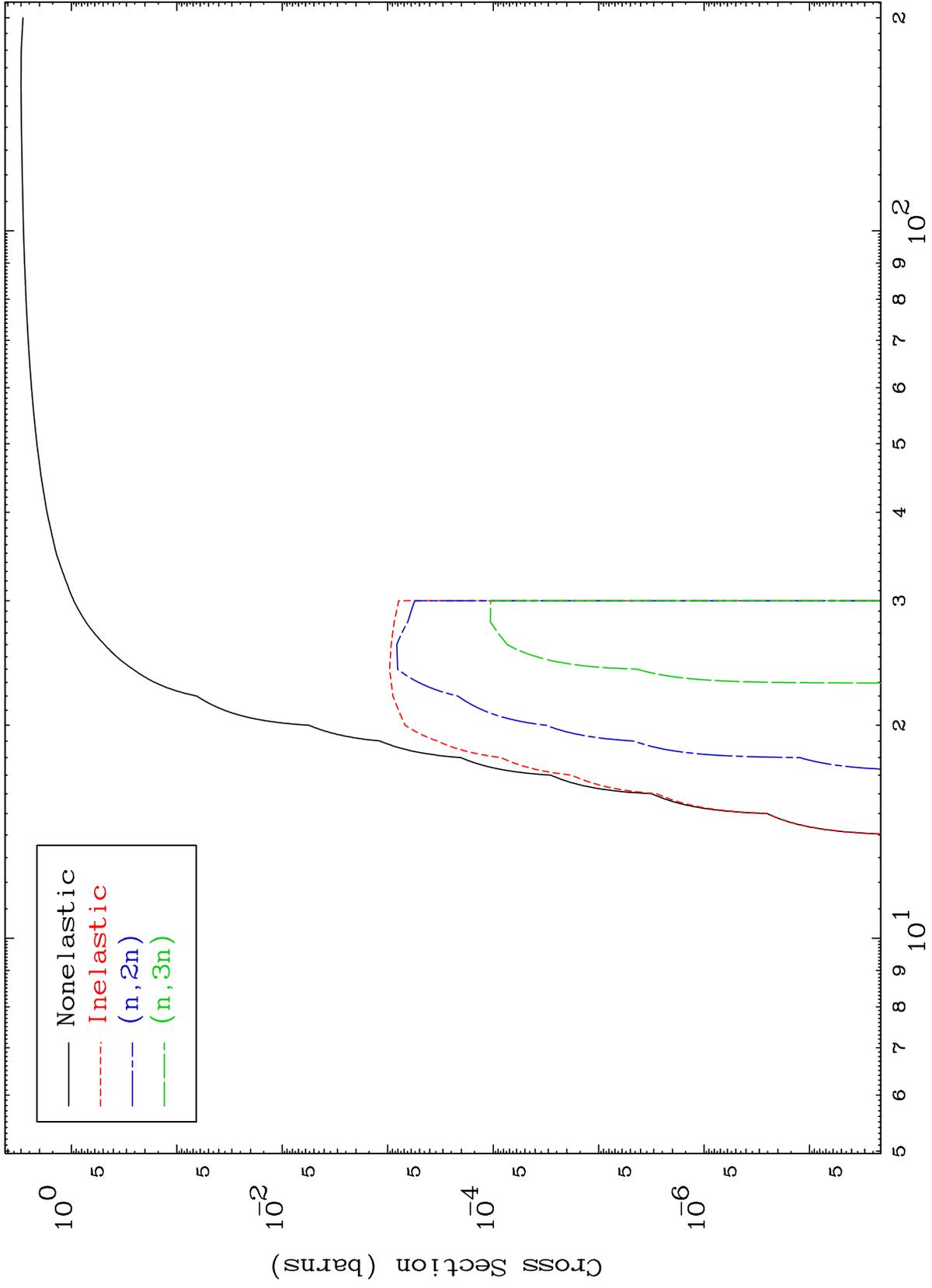
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9234

0 Kelvin Cross Sections

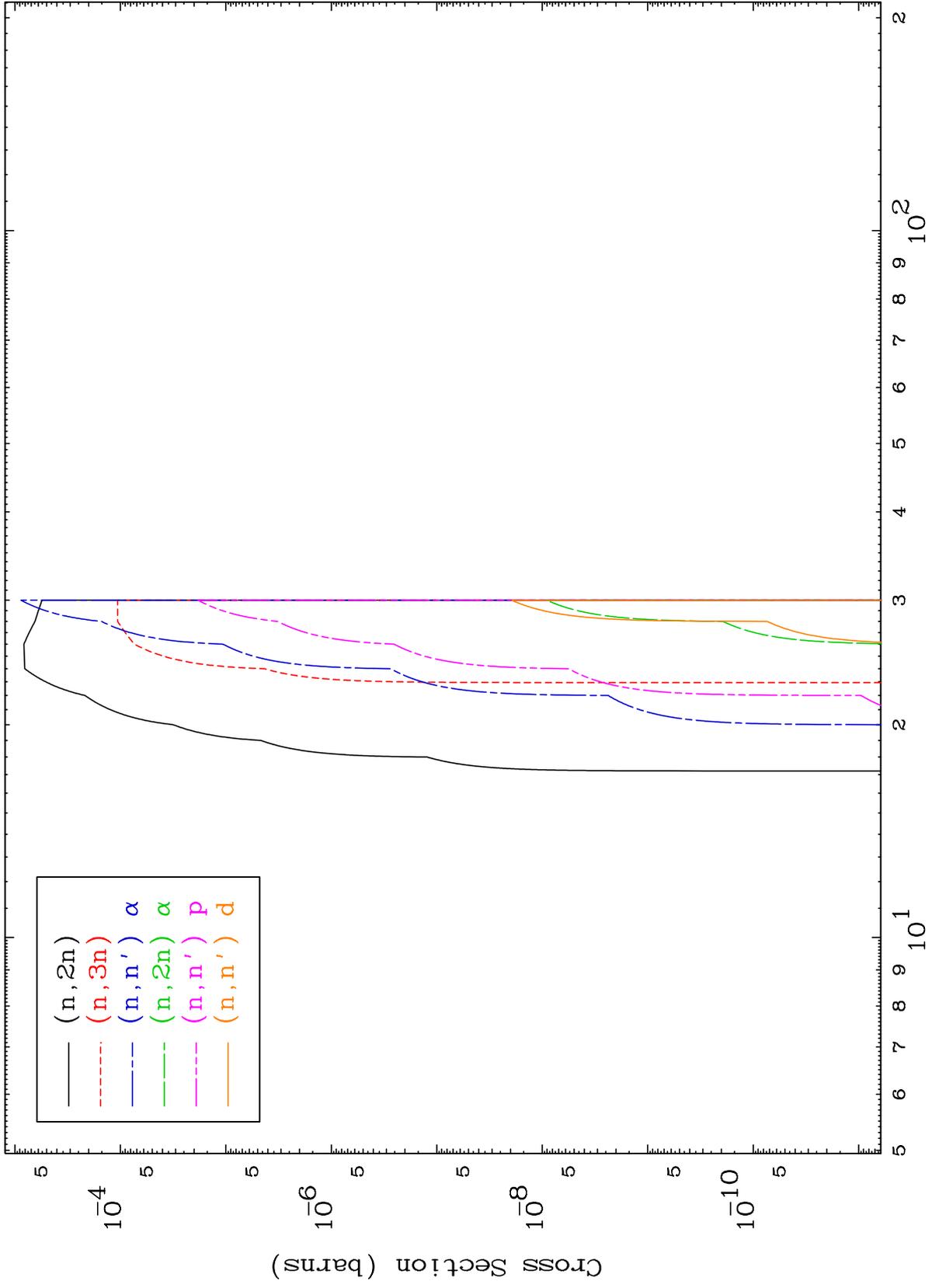
92-U -237

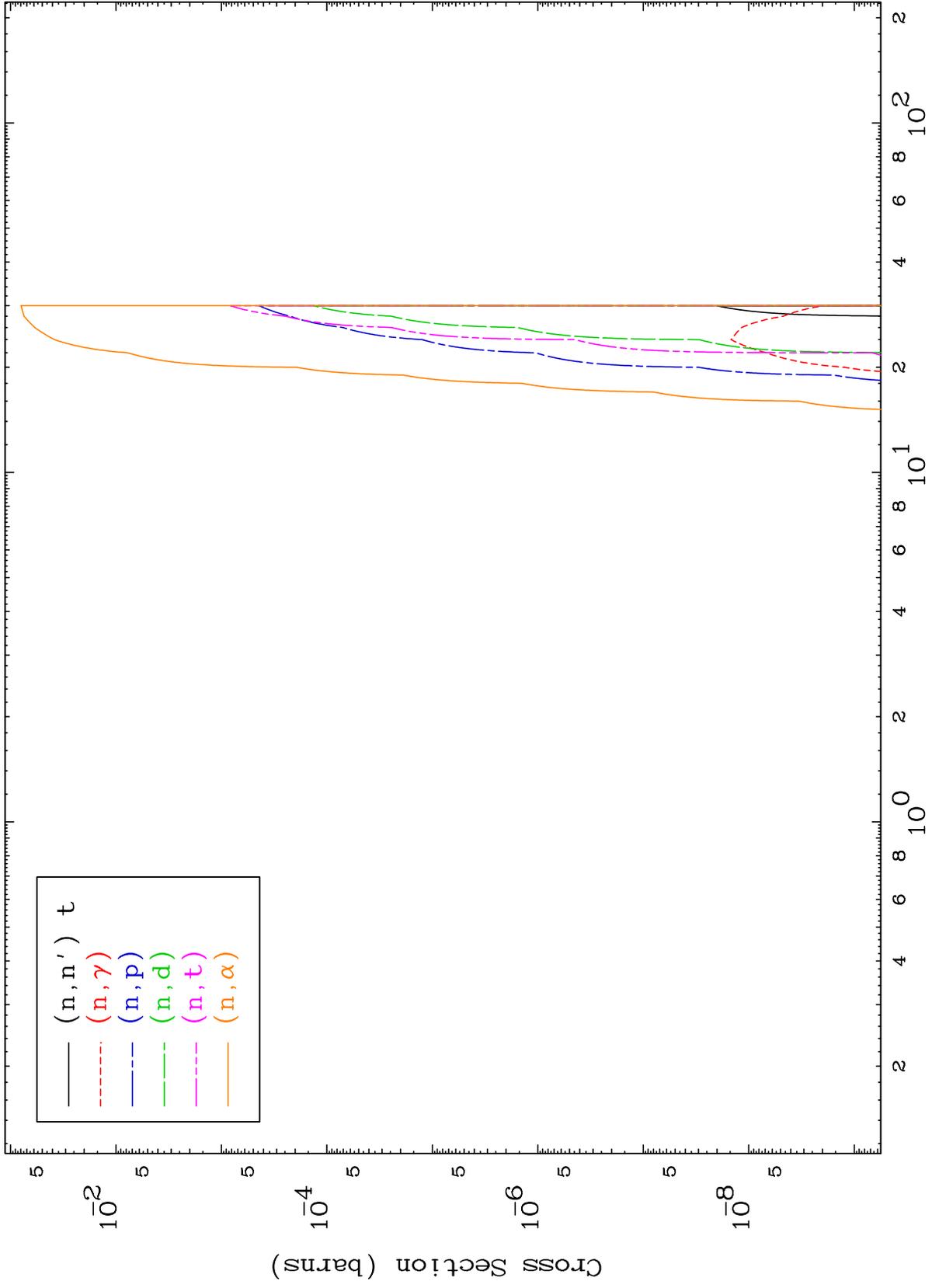


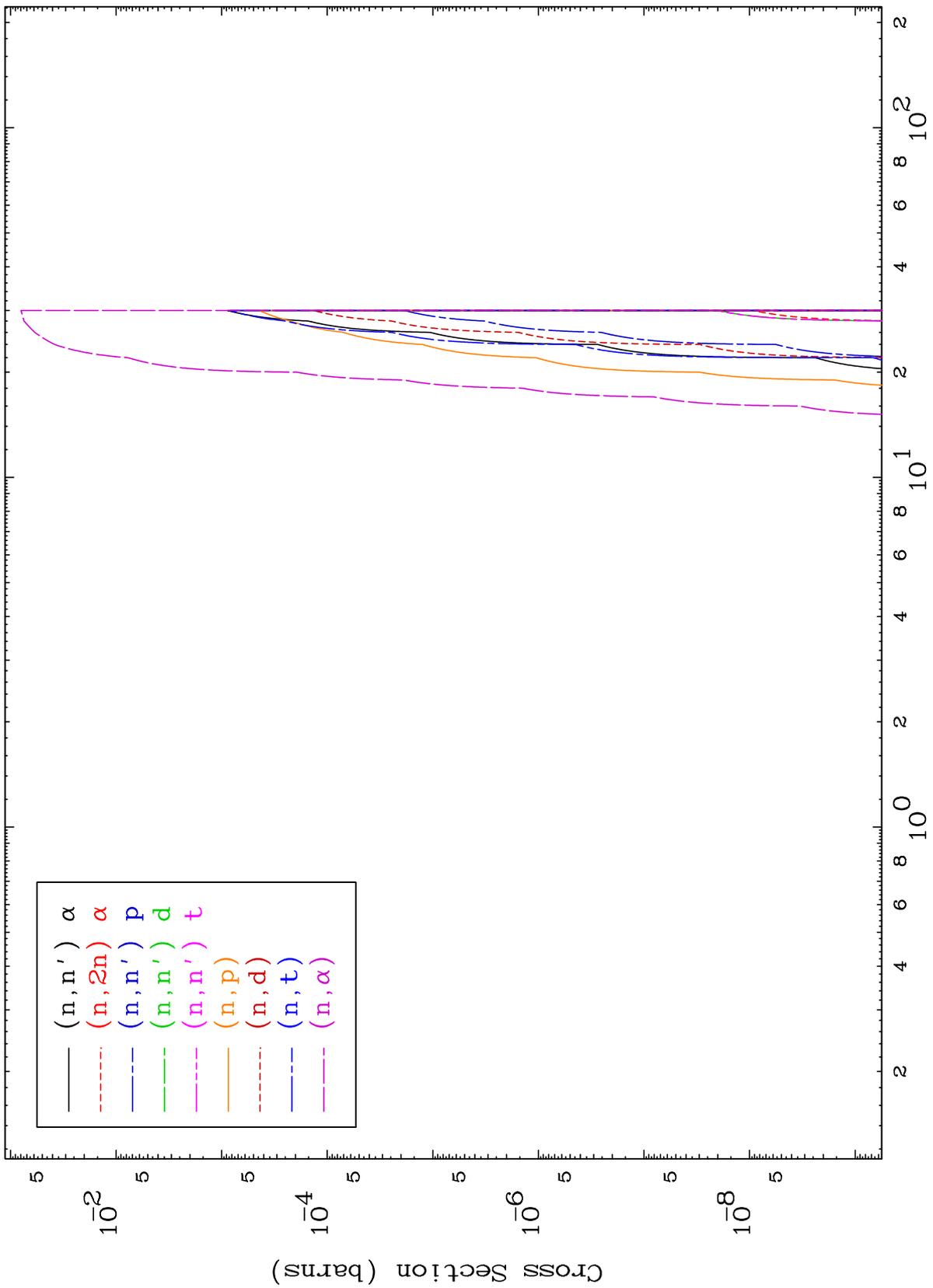
1

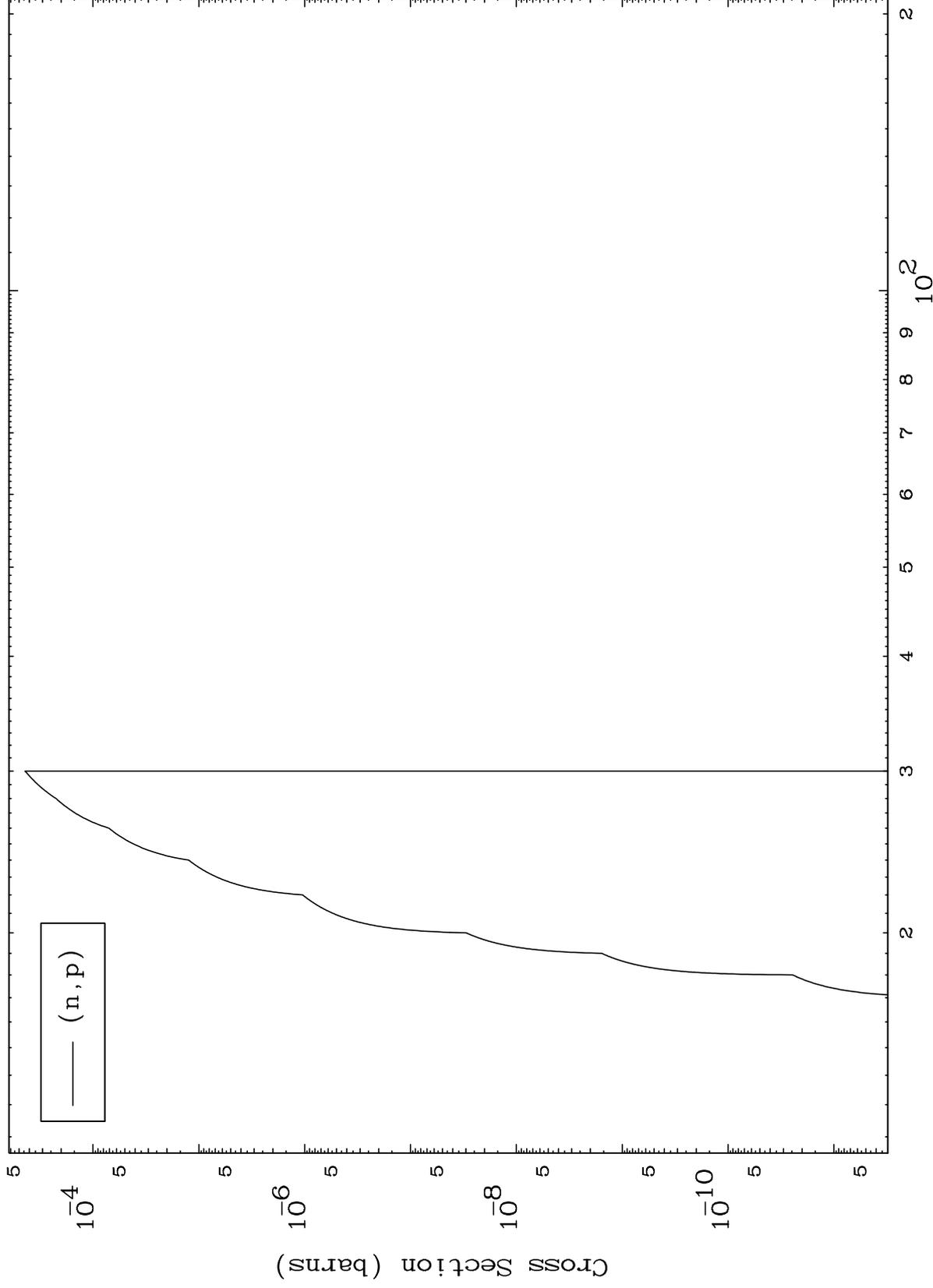
Incident Energy (MeV)

92-U -237







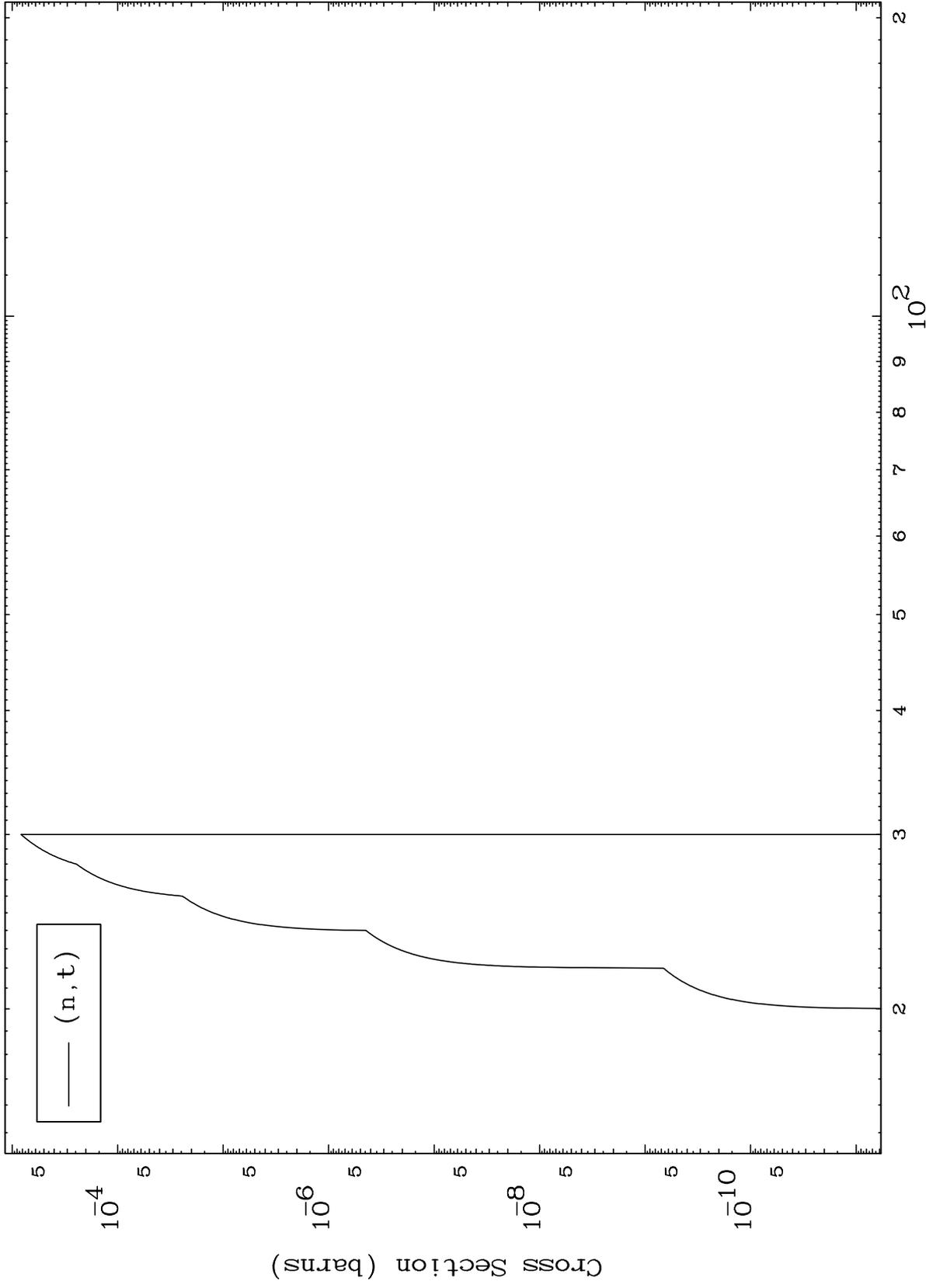




MAT 9234

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

92-U -237



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Incident Energy (MeV)

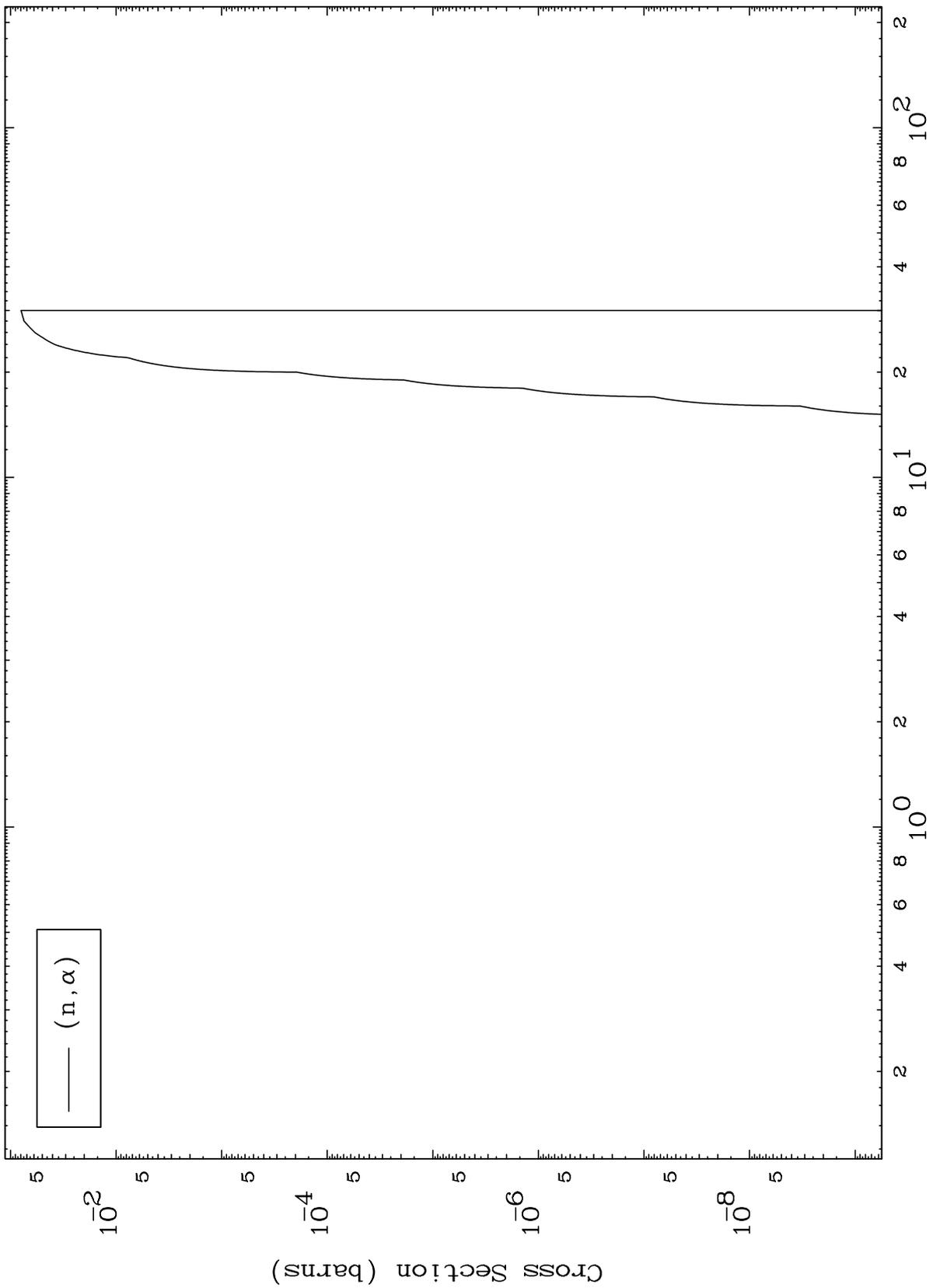
92-U -237

MAT 9234

( $\alpha, \alpha$ ) Levels

92-U -237

0 Kelvin Cross Sections

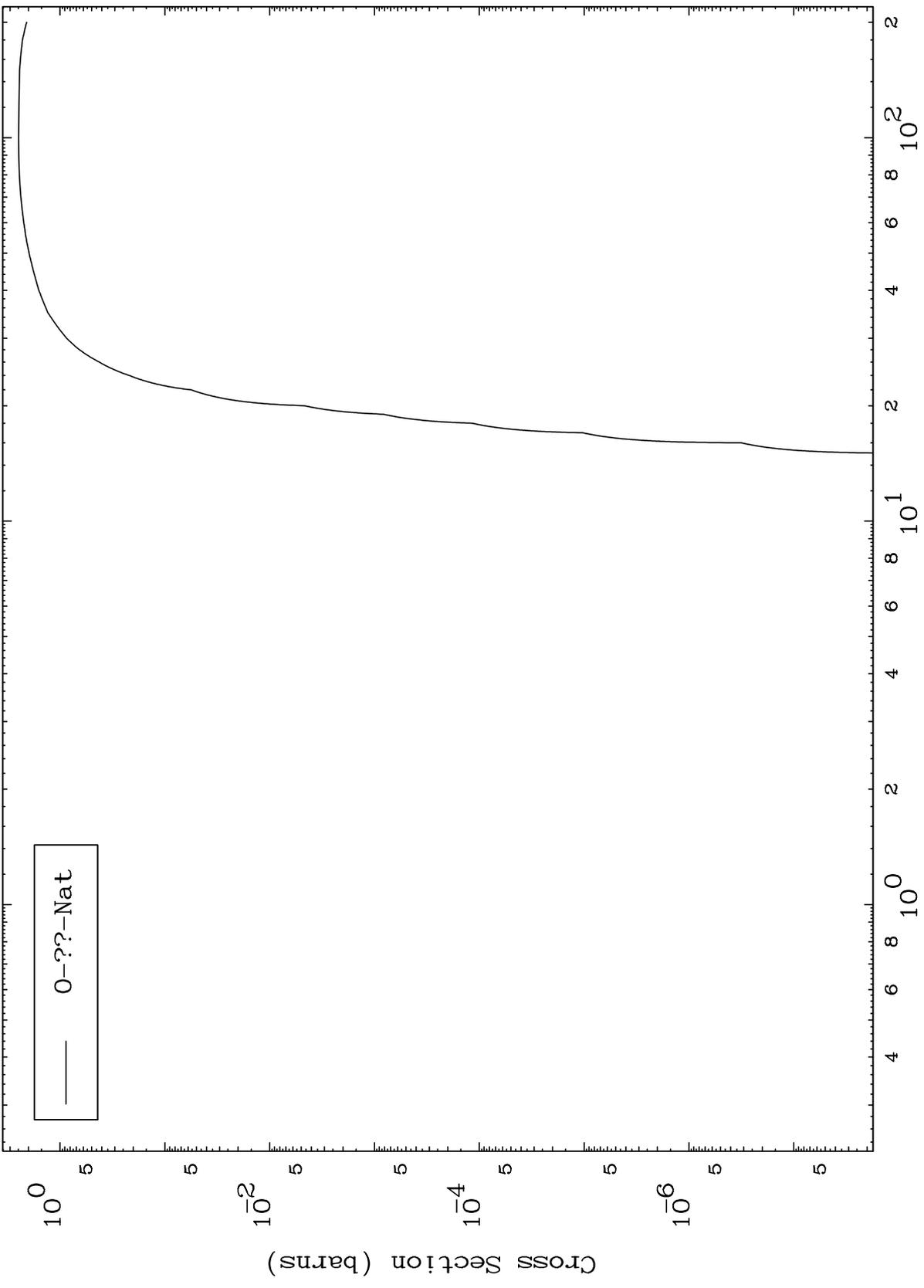


MAT 9234

Fission

92-U -237

Radionuclide Production Cross Section

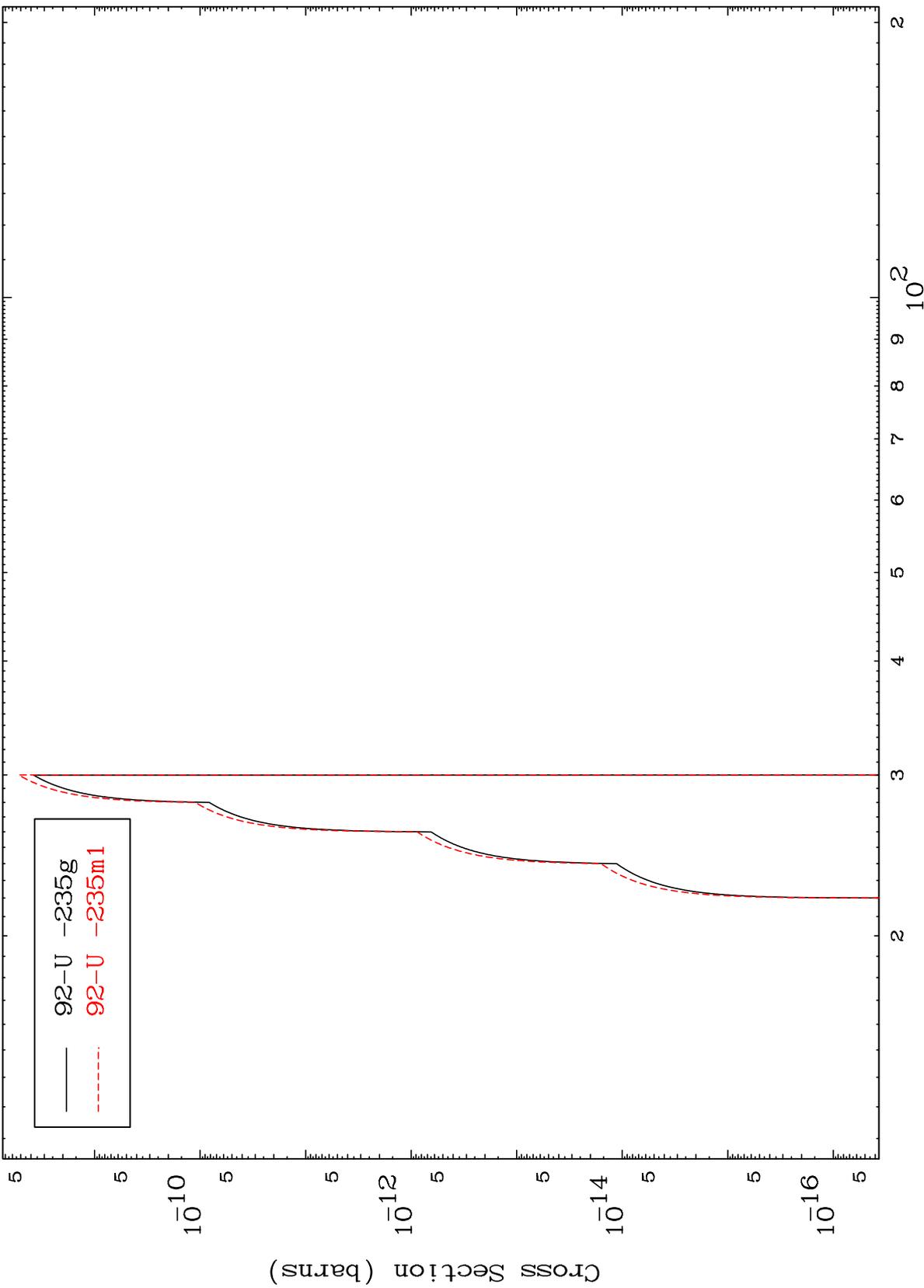


MAT 9234

(n,2n)  $\alpha$

92-U -237

Radionuclide Production Cross Section



10

Incident Energy (MeV)

92-U -237

MAT 92334

92-U -237

(n,p)  
Radionuclide Production Cross Section

