

Program EVALPLOT  
(Version 2021-1)

by

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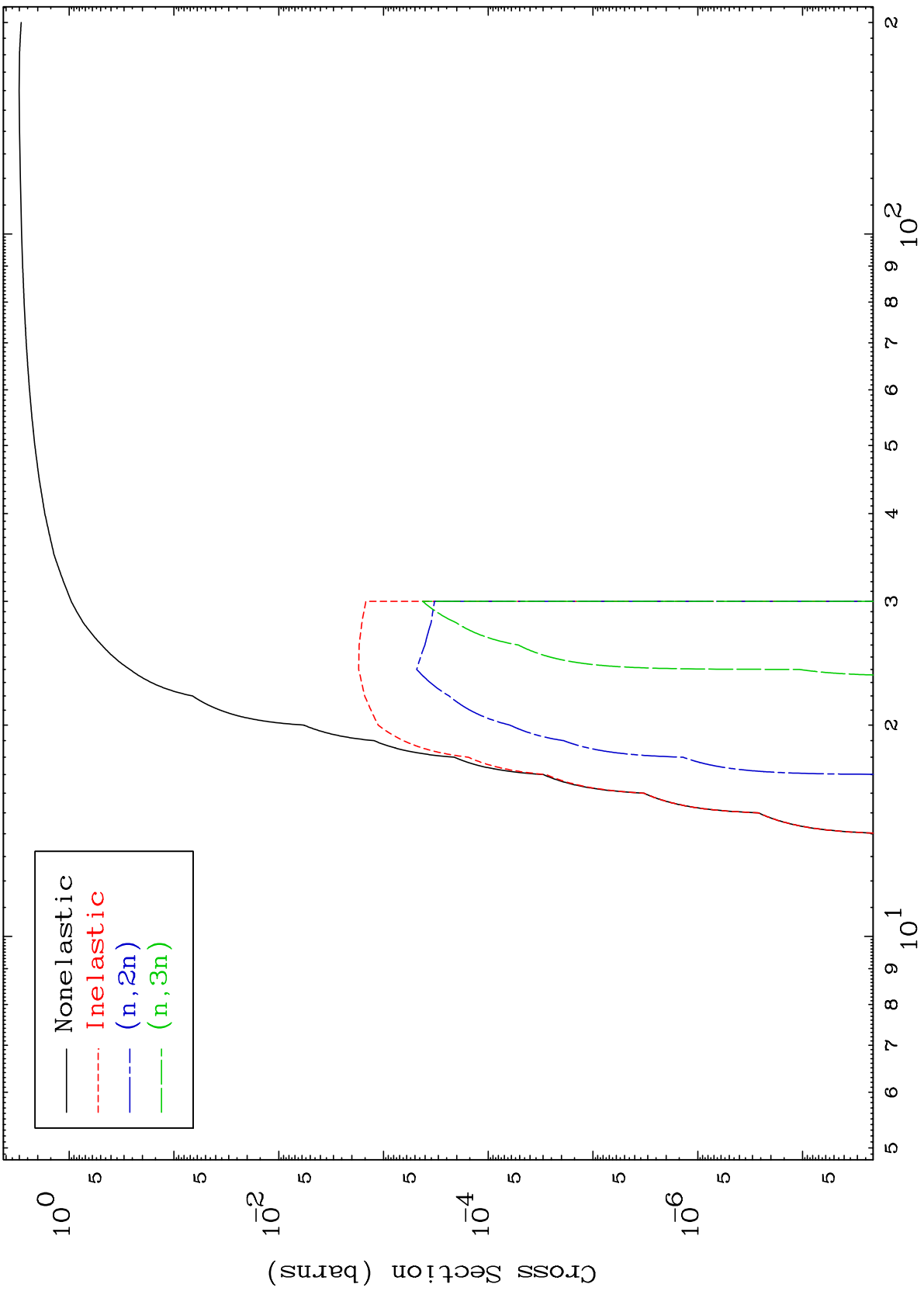
Press Mouse Button to Start

MAT 9237

$\alpha$  Major

92-U -238

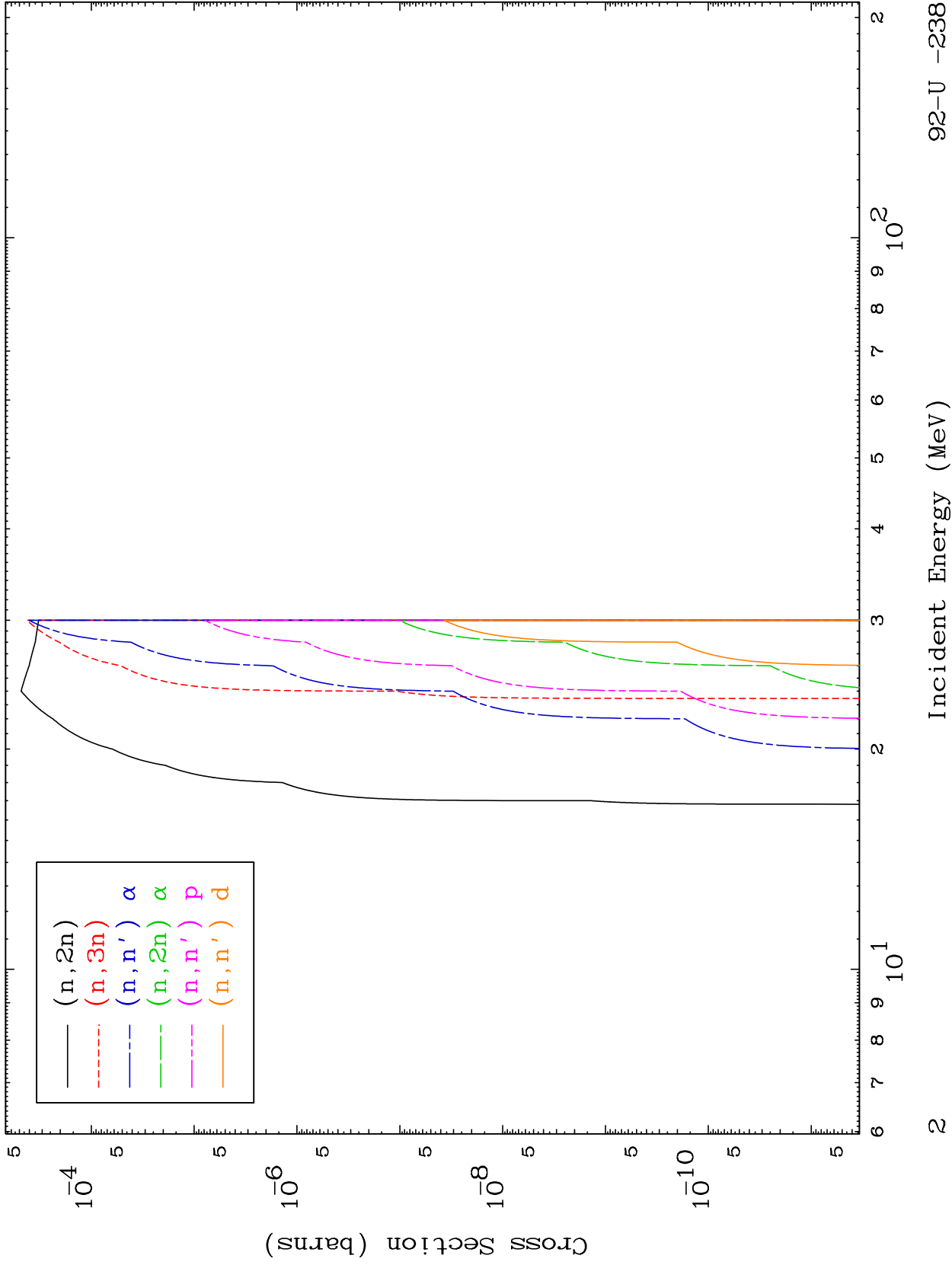
0 Kelvin Cross Sections

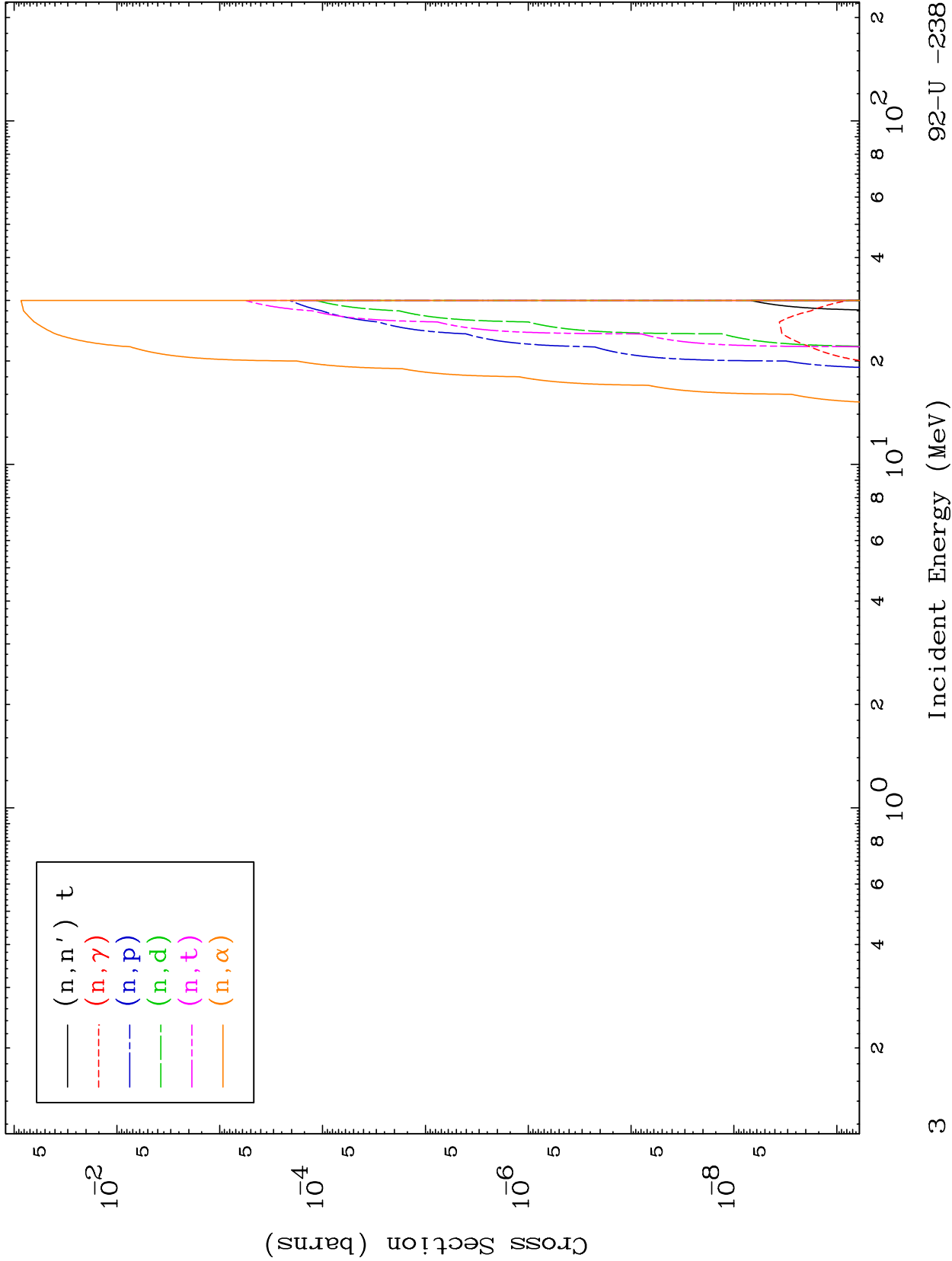


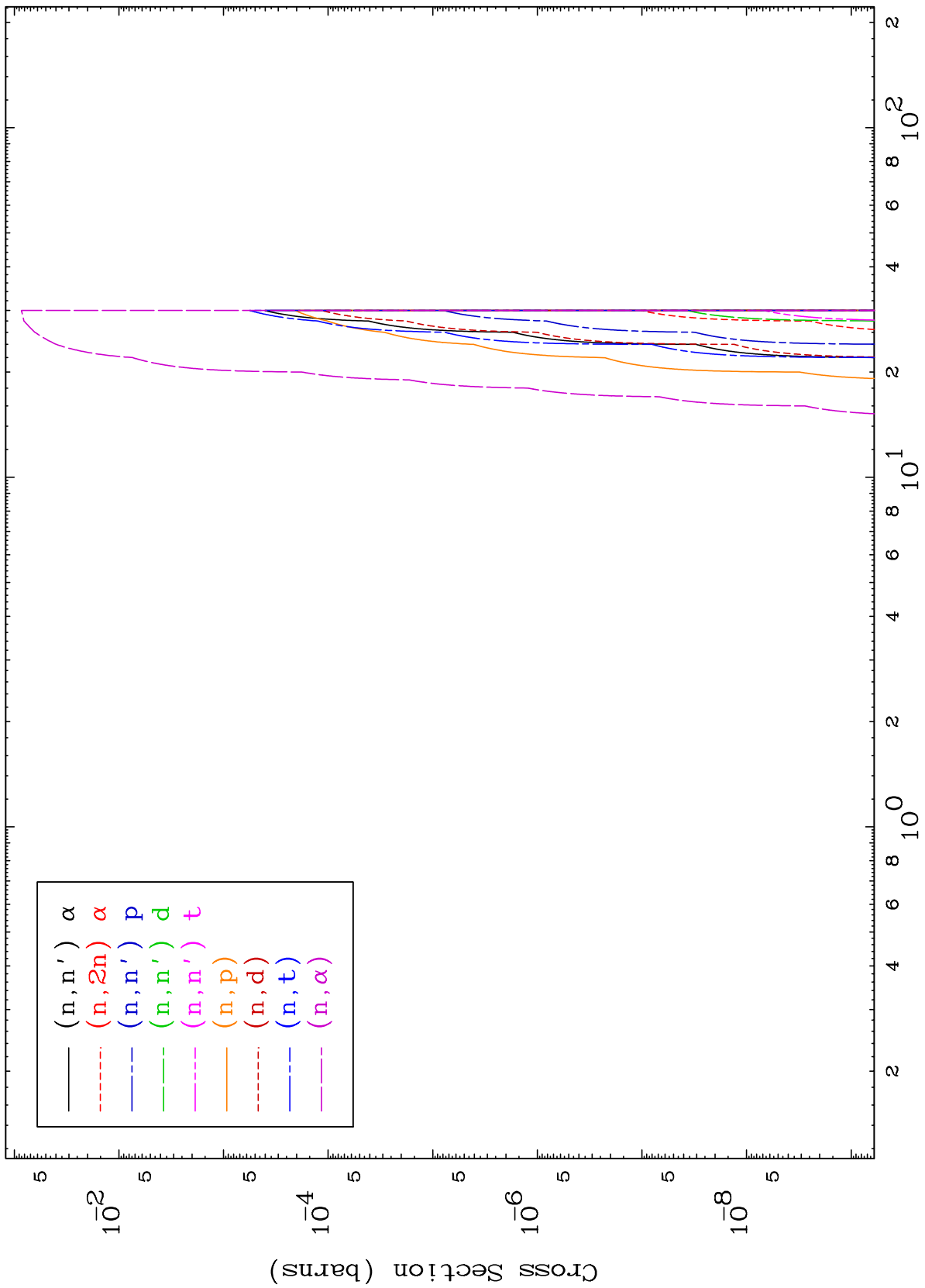
1

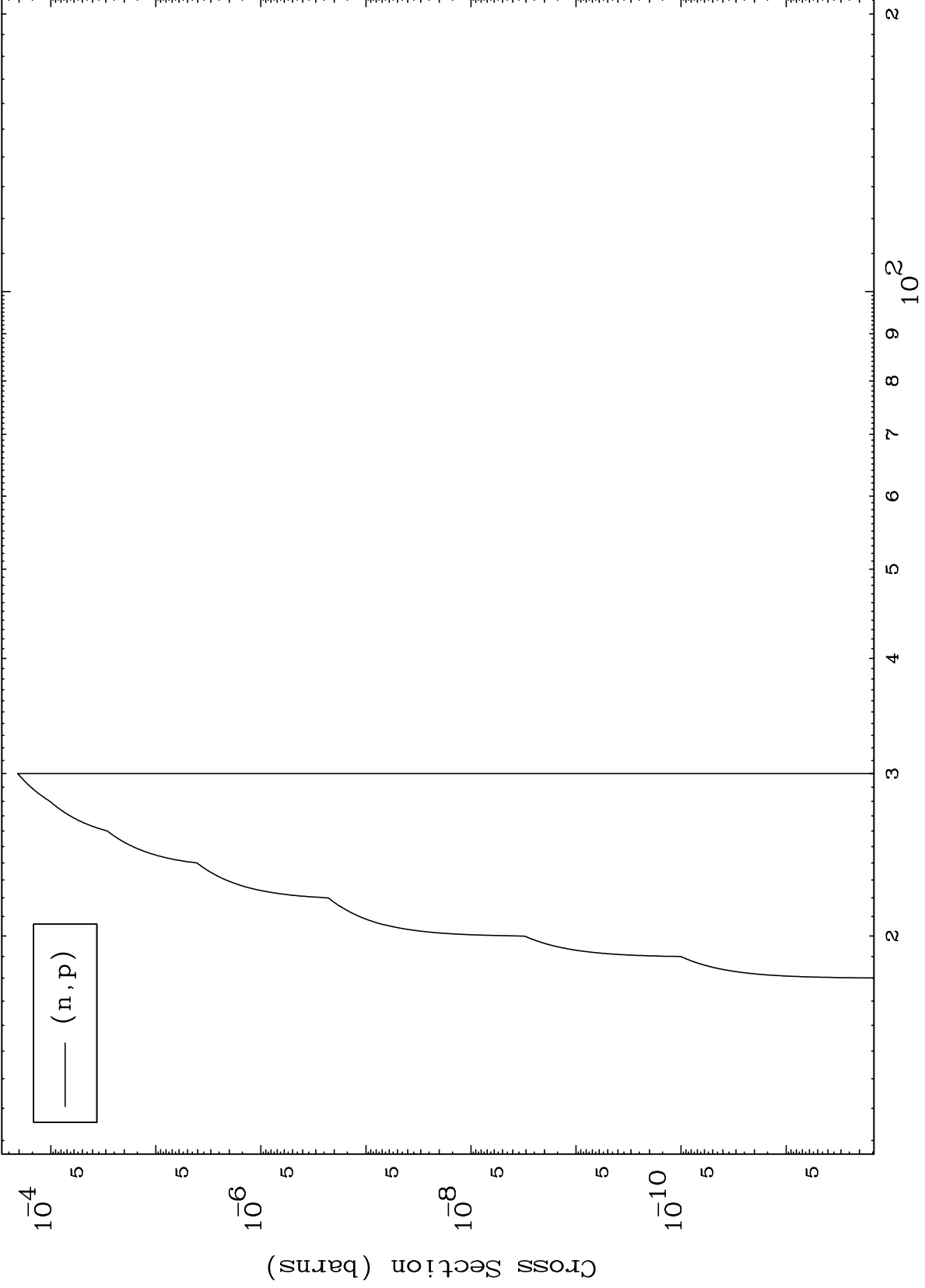
Incident Energy (MeV)

92-U -238





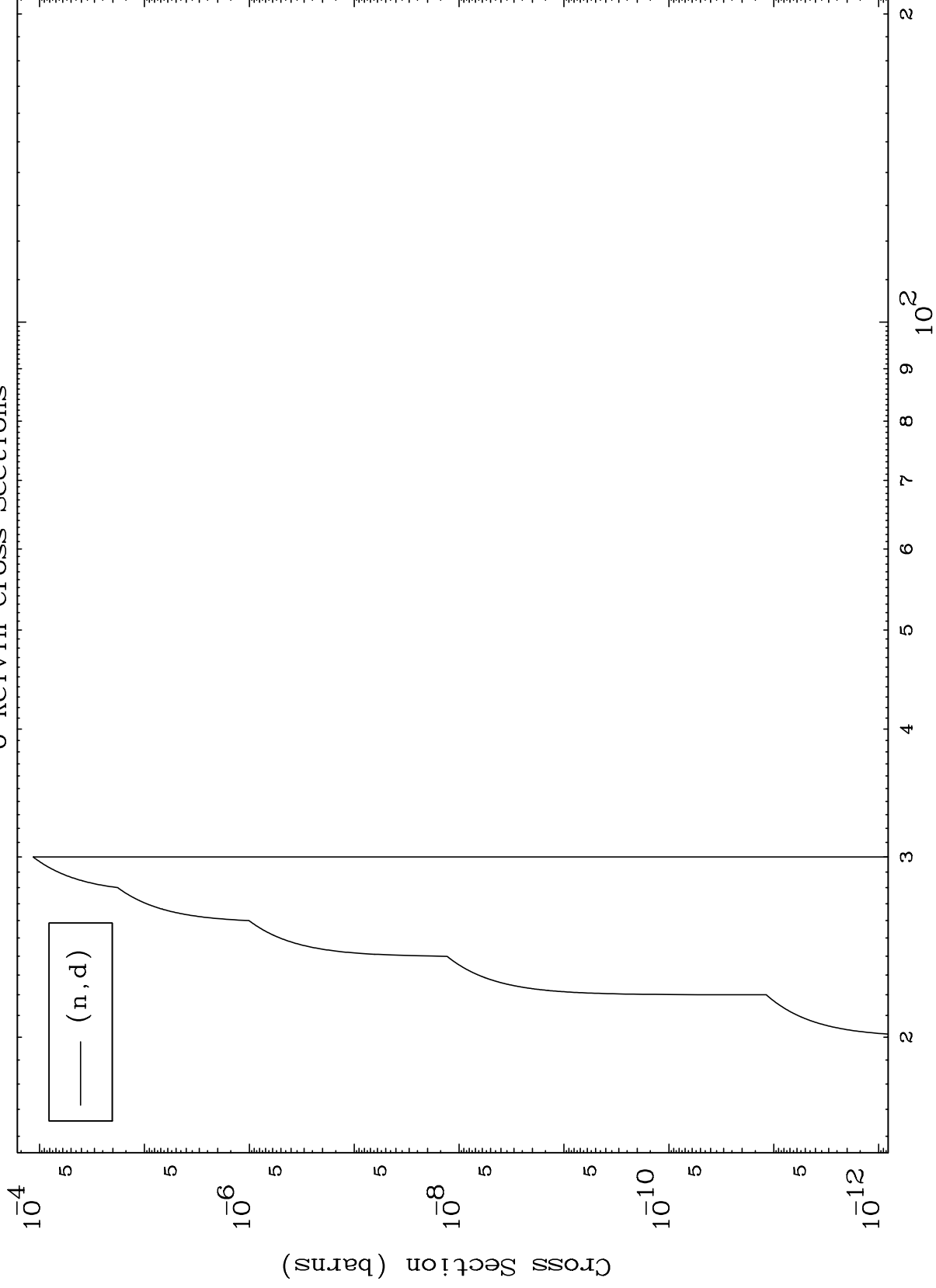




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( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

92-U -238



6

Incident Energy (MeV)

92-U -238

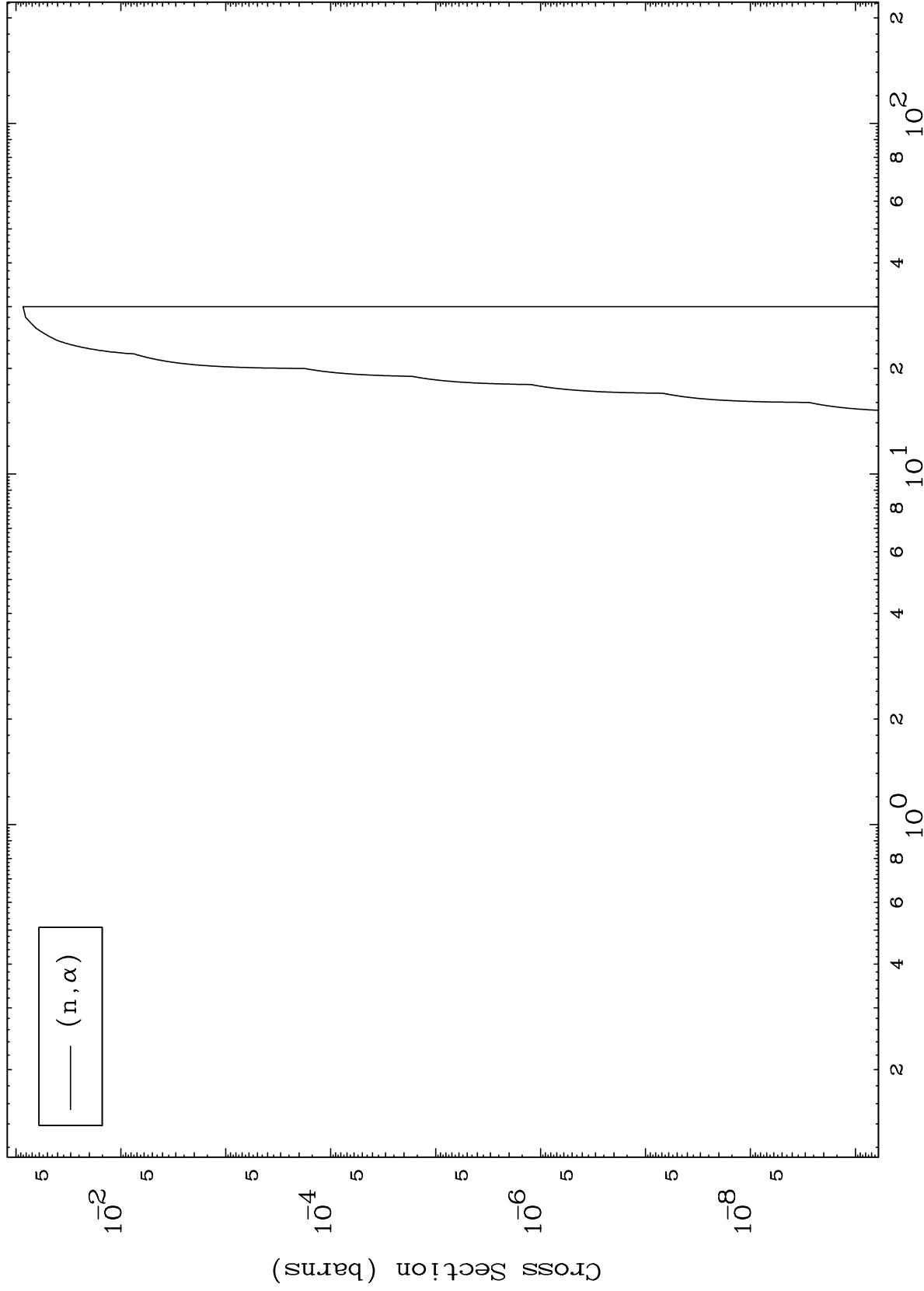


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( $\alpha, \alpha$ ) Levels

92-U -238

0 Kelvin Cross Sections

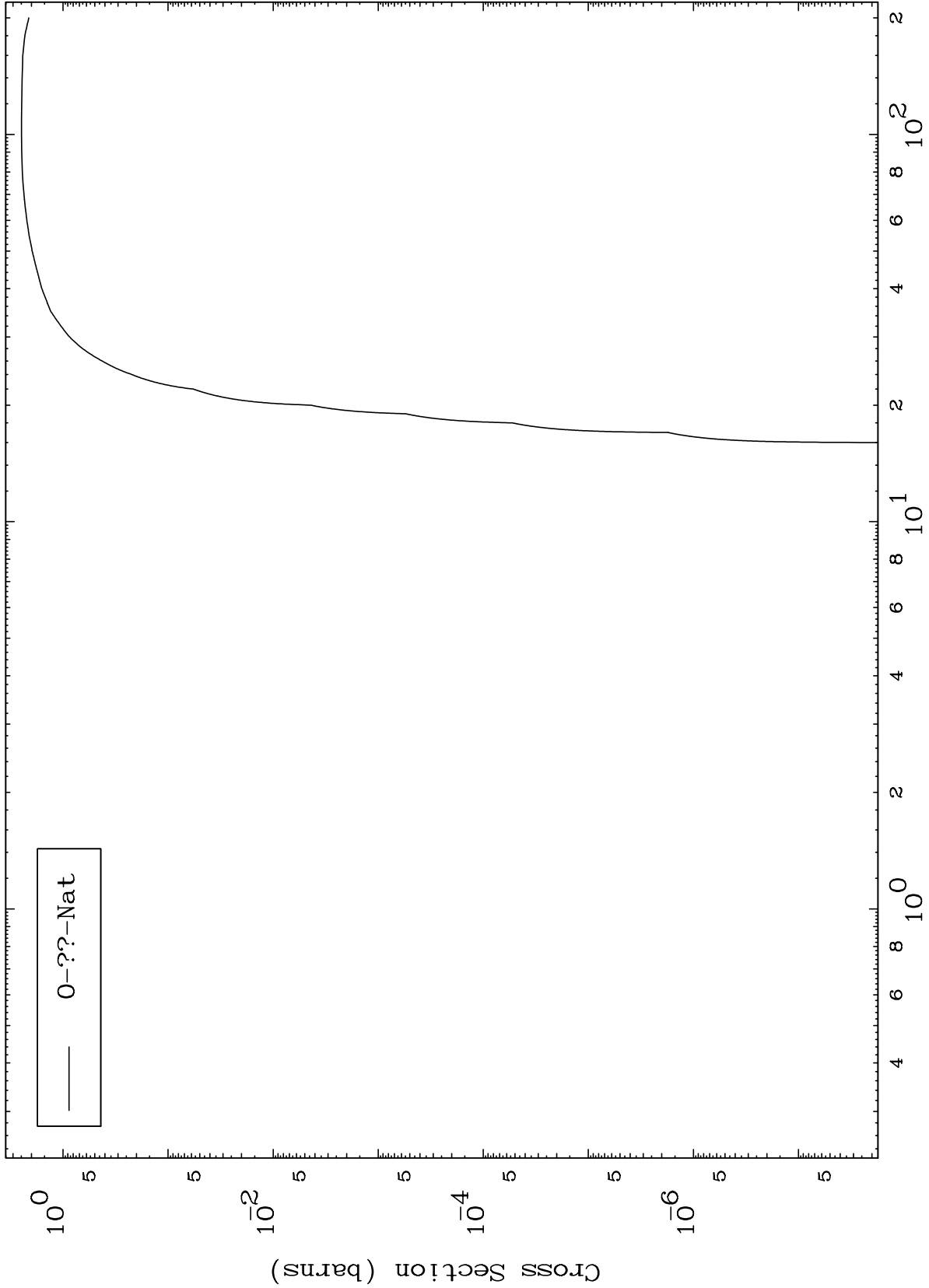


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Fission

92-U -238

Radionuclide Production Cross Section

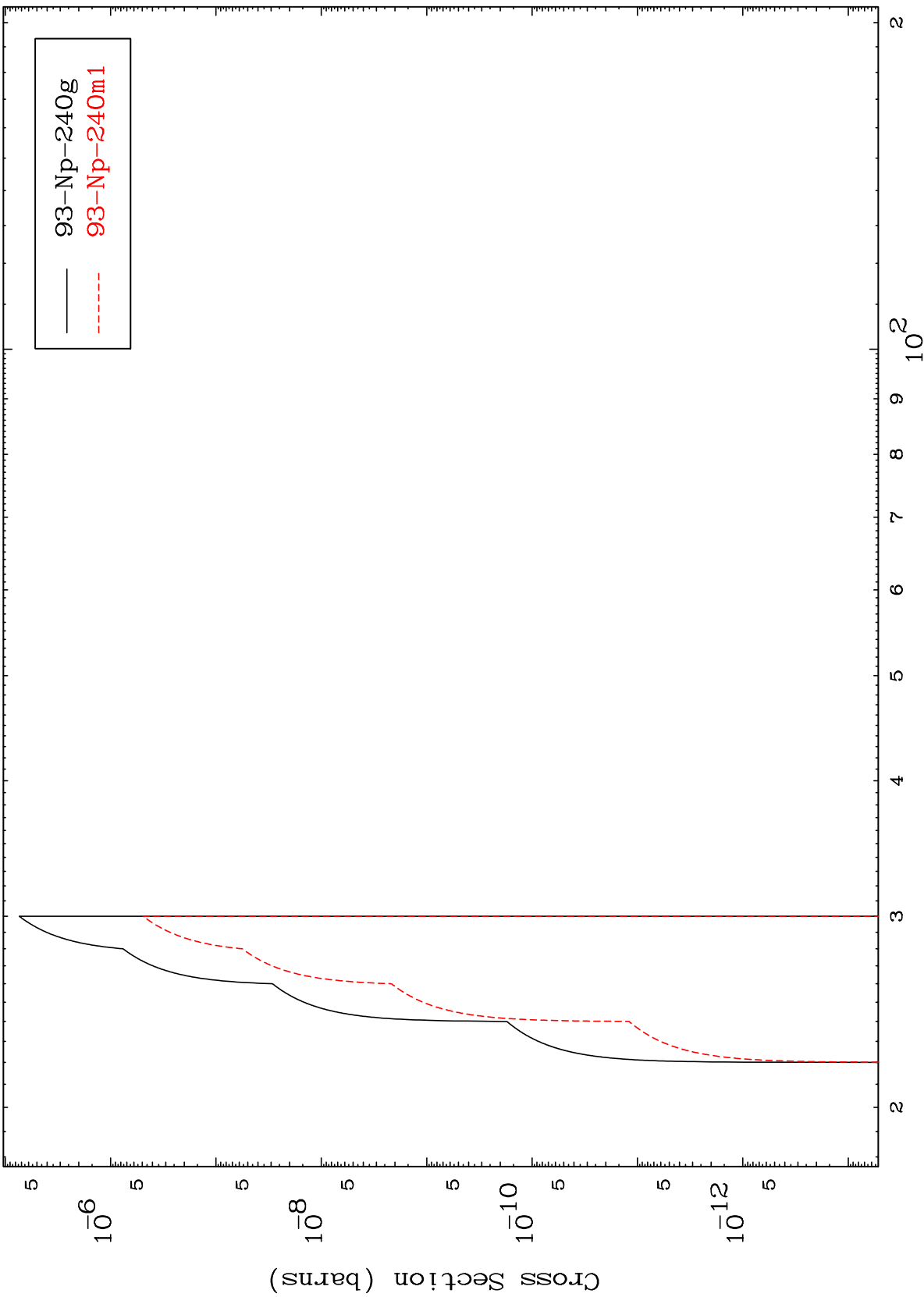


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(n,n') p

92-U -238

Radionuclide Production Cross Section



10

Incident Energy (MeV)

92-U -238

(n,d)  
Radionuclide Production Cross Section

