

Program EVALPLOT
(Version 2021-1)

by

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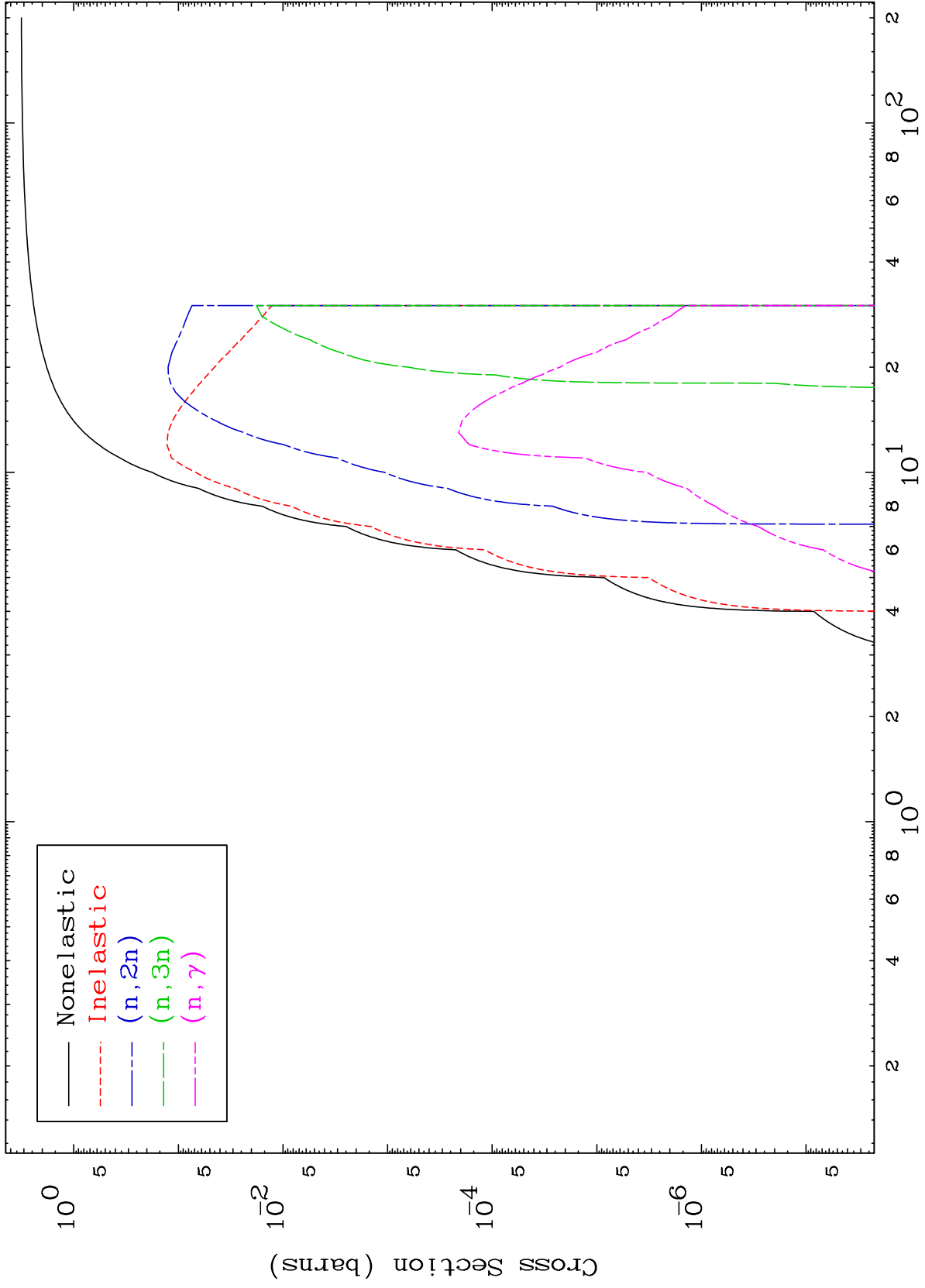
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8523

Deuteron Major
0 Kelvin Cross Sections

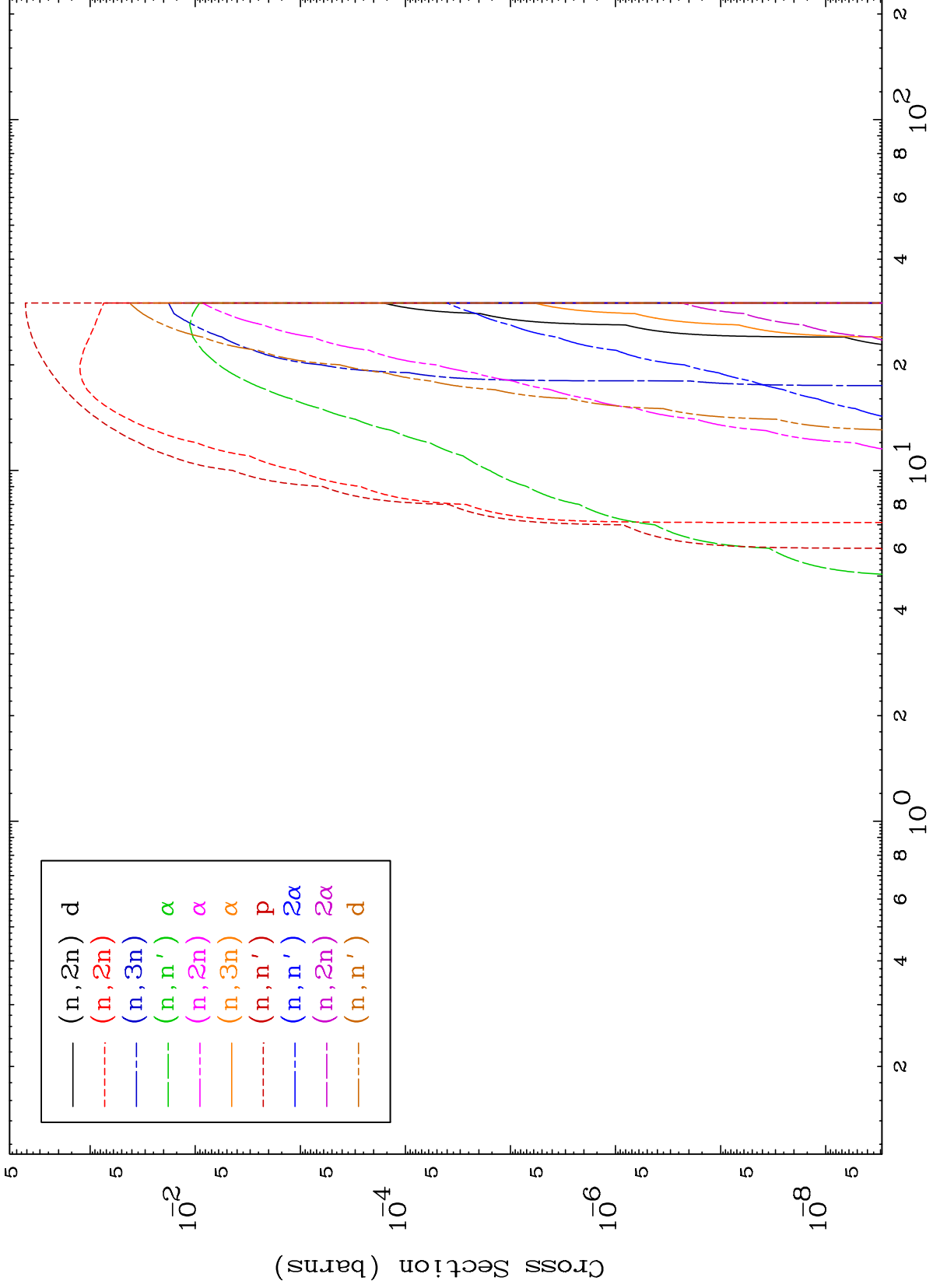
85-At-202m

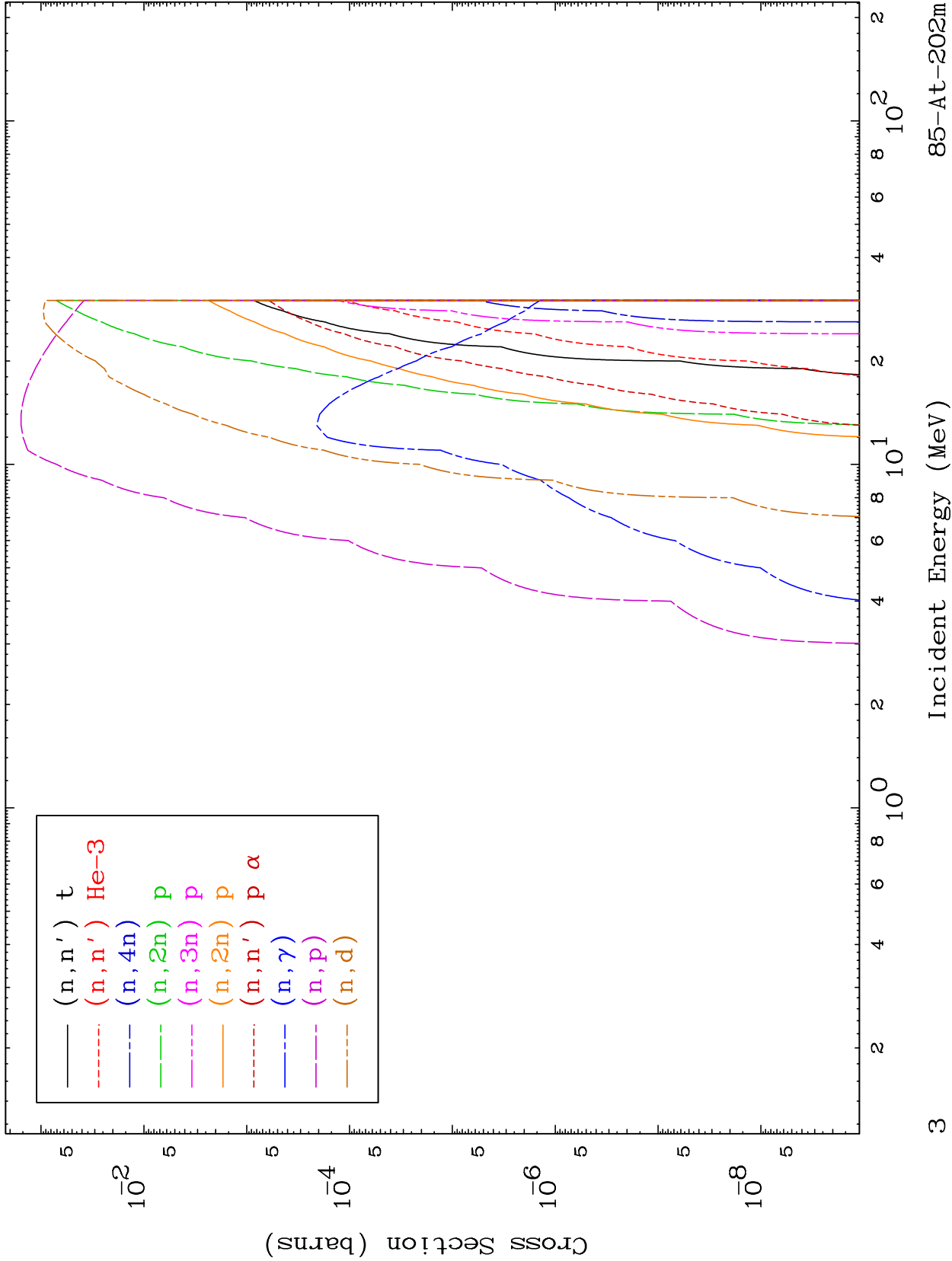


MAT 8523

Deuteron Neutron Absorption
0 Kelvin Cross Sections

85-At-202m

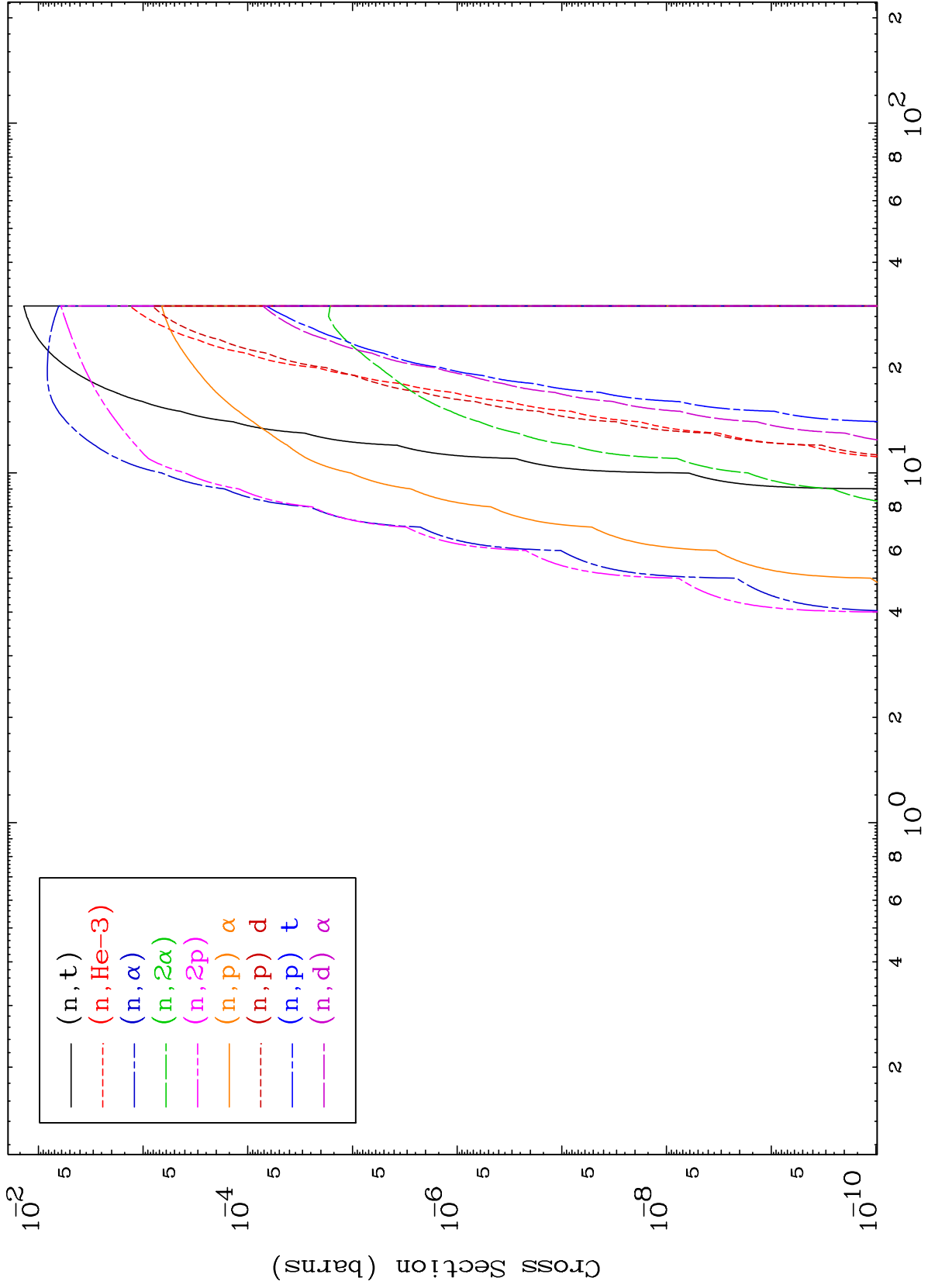




MAT 8523

Deuteron Neutron Absorption
0 Kelvin Cross Sections

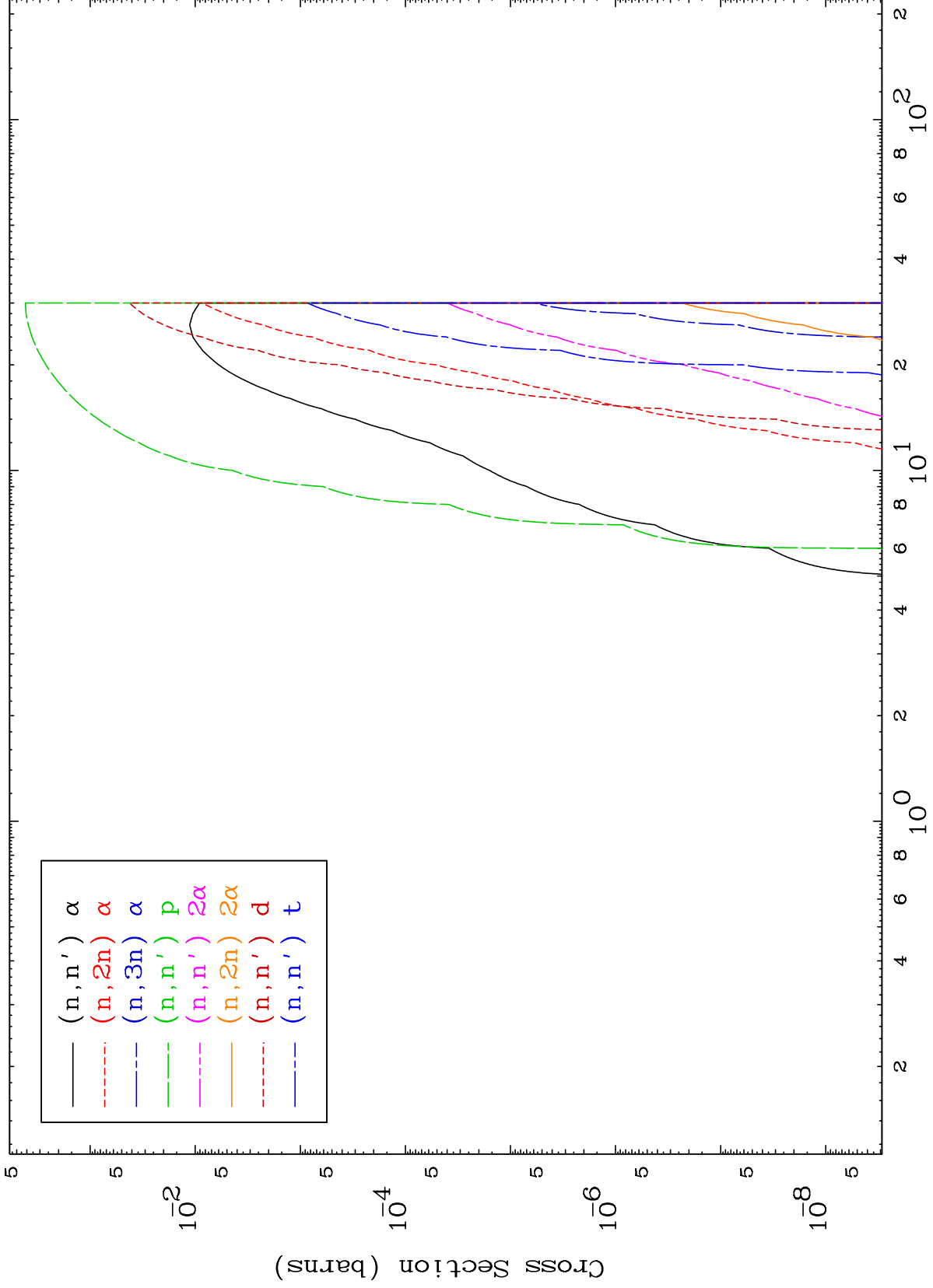
85-At-202m



MAT 8523

Deuteron Charged Particle
0 Kelvin Cross Sections

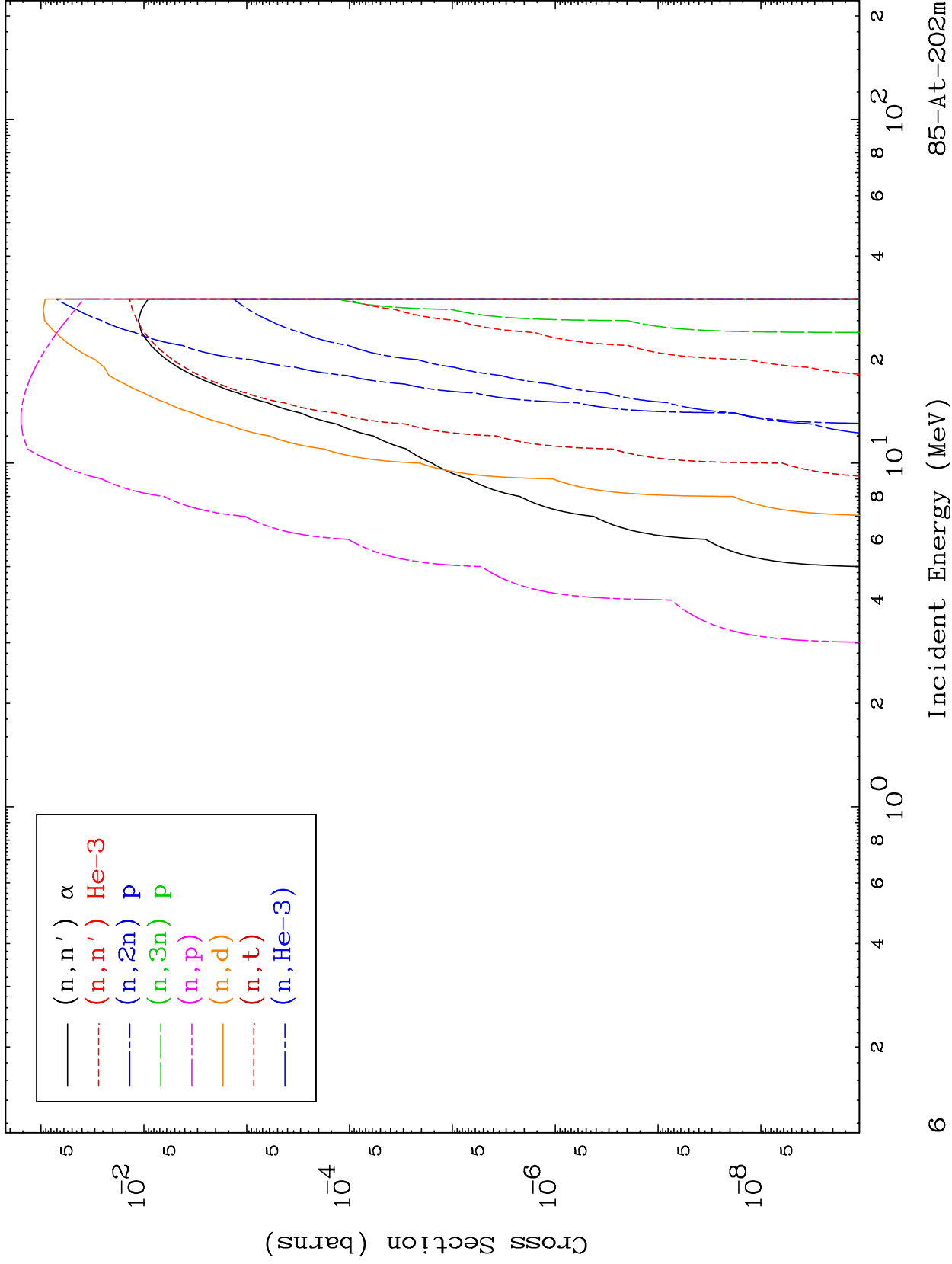
85-At-202m



5

Incident Energy (MeV)

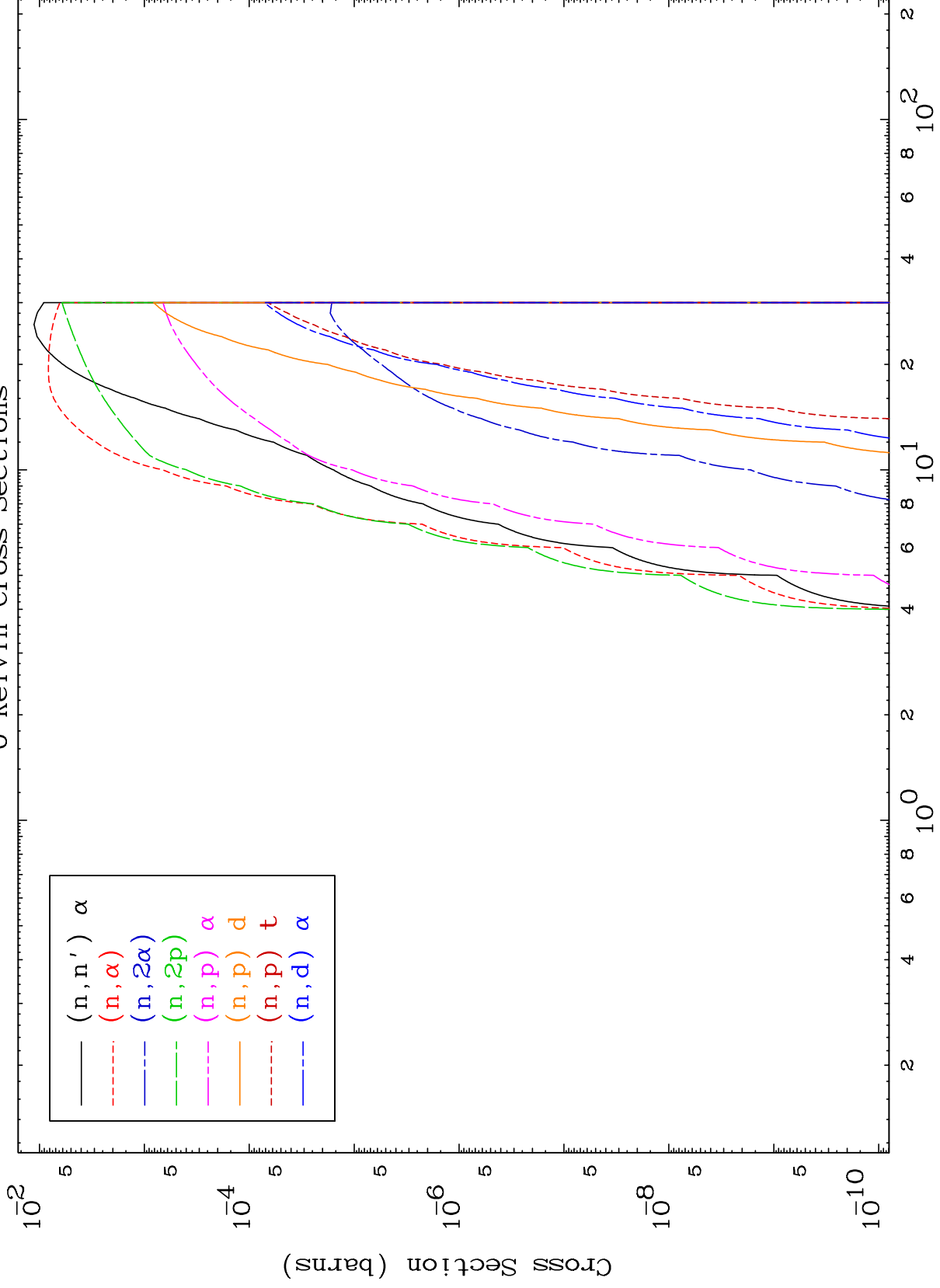
85-At-202m



MAT 8523

Deuteron Charged Particle
0 Kelvin Cross Sections

85-At-202m

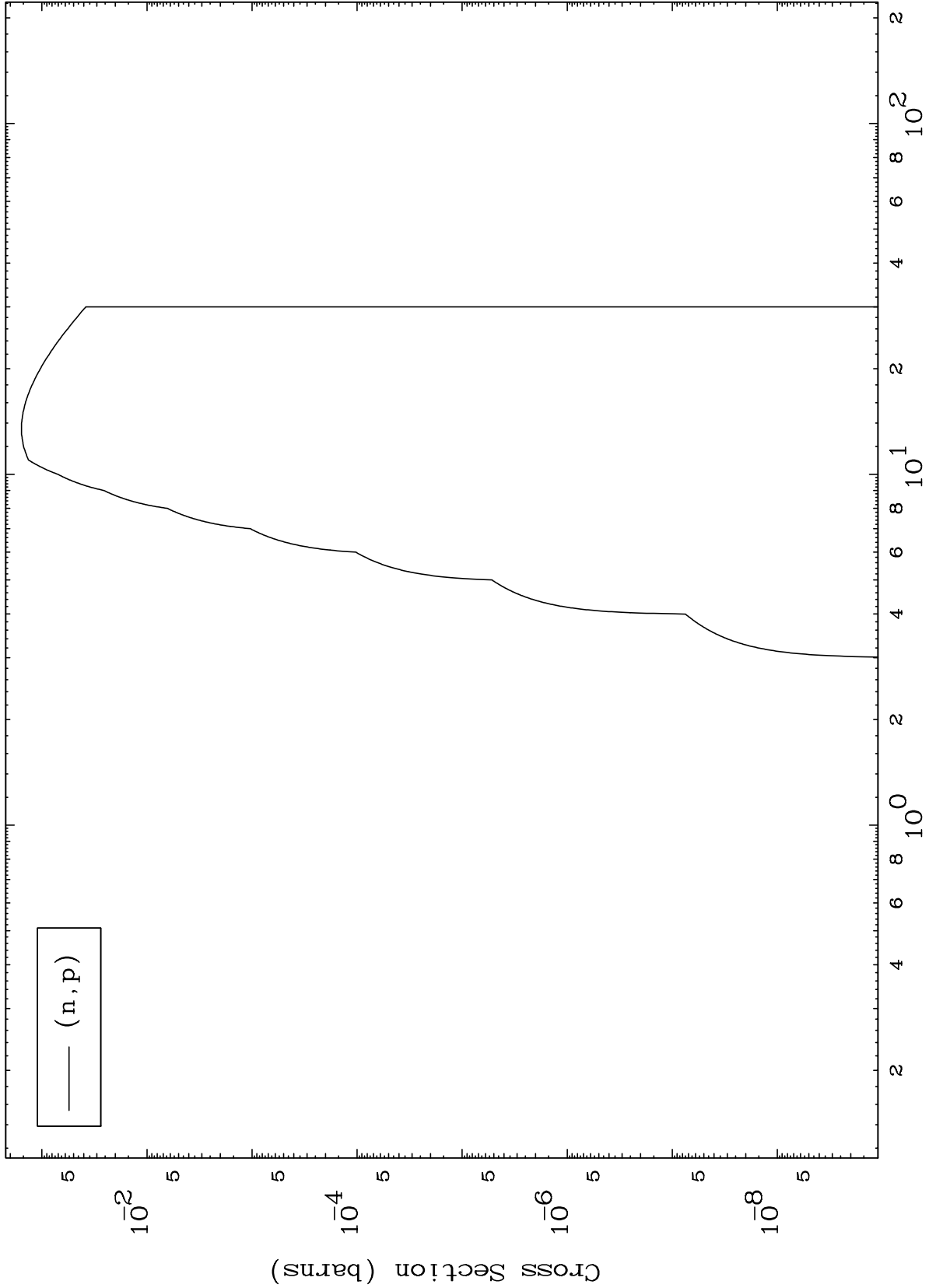


MAT 8523

(d,p) Levels

85-At-202m

0 Kelvin Cross Sections

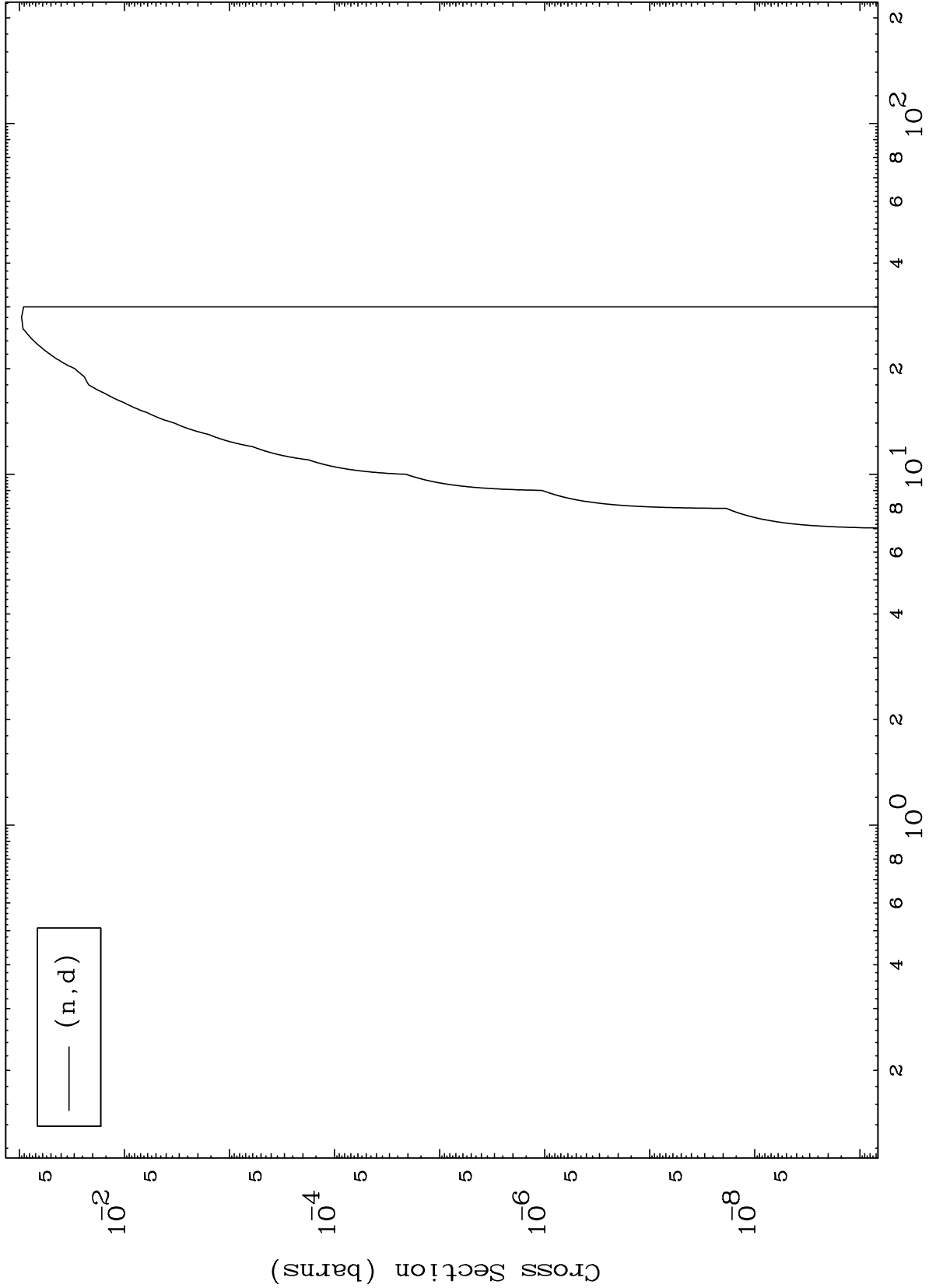


MAT 8523

(d,d) Levels

85-At-202m

0 Kelvin Cross Sections

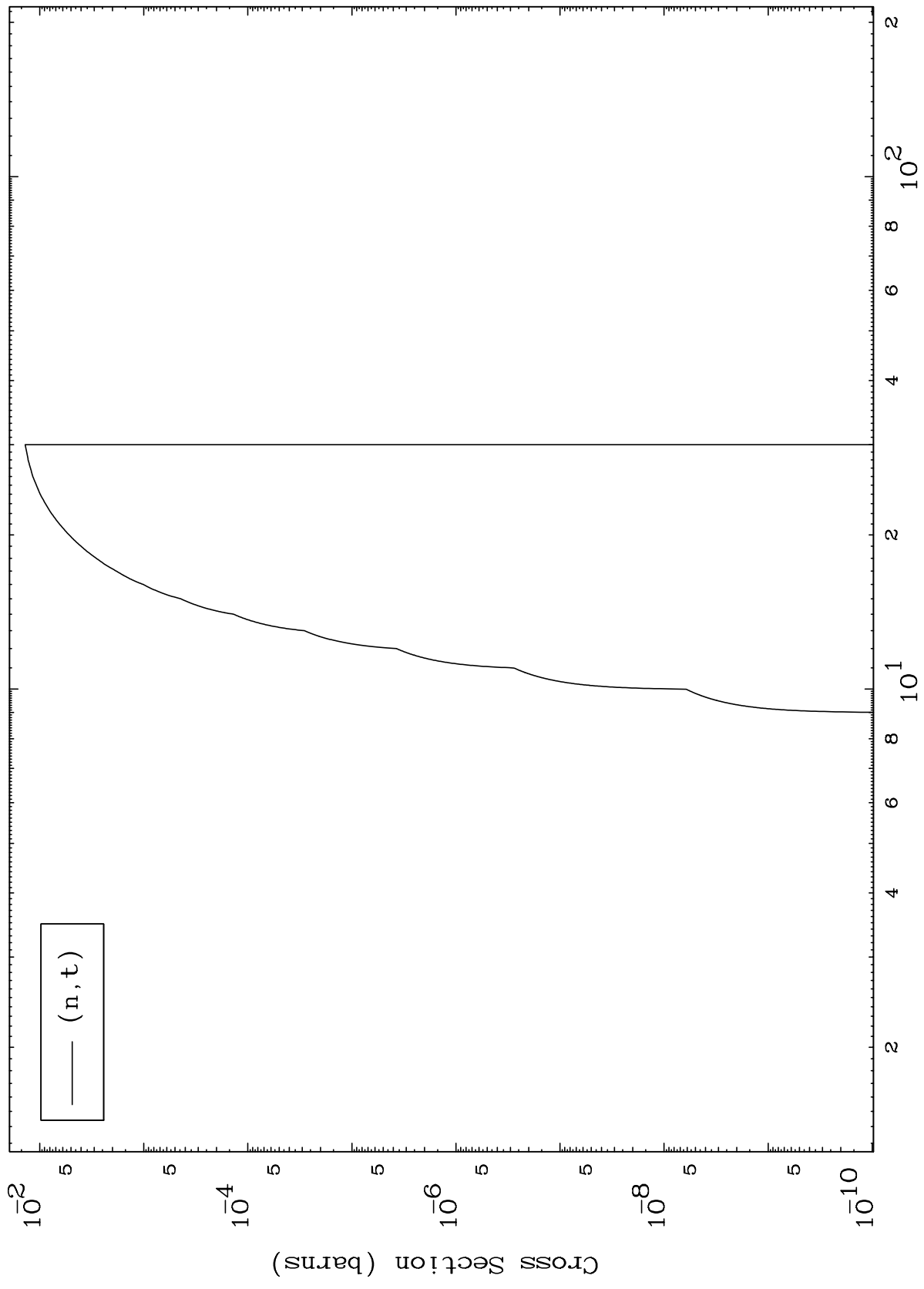


MAT 8523

(d,t) Levels

85-At-202m

0 Kelvin Cross Sections



10

Incident Energy (MeV)

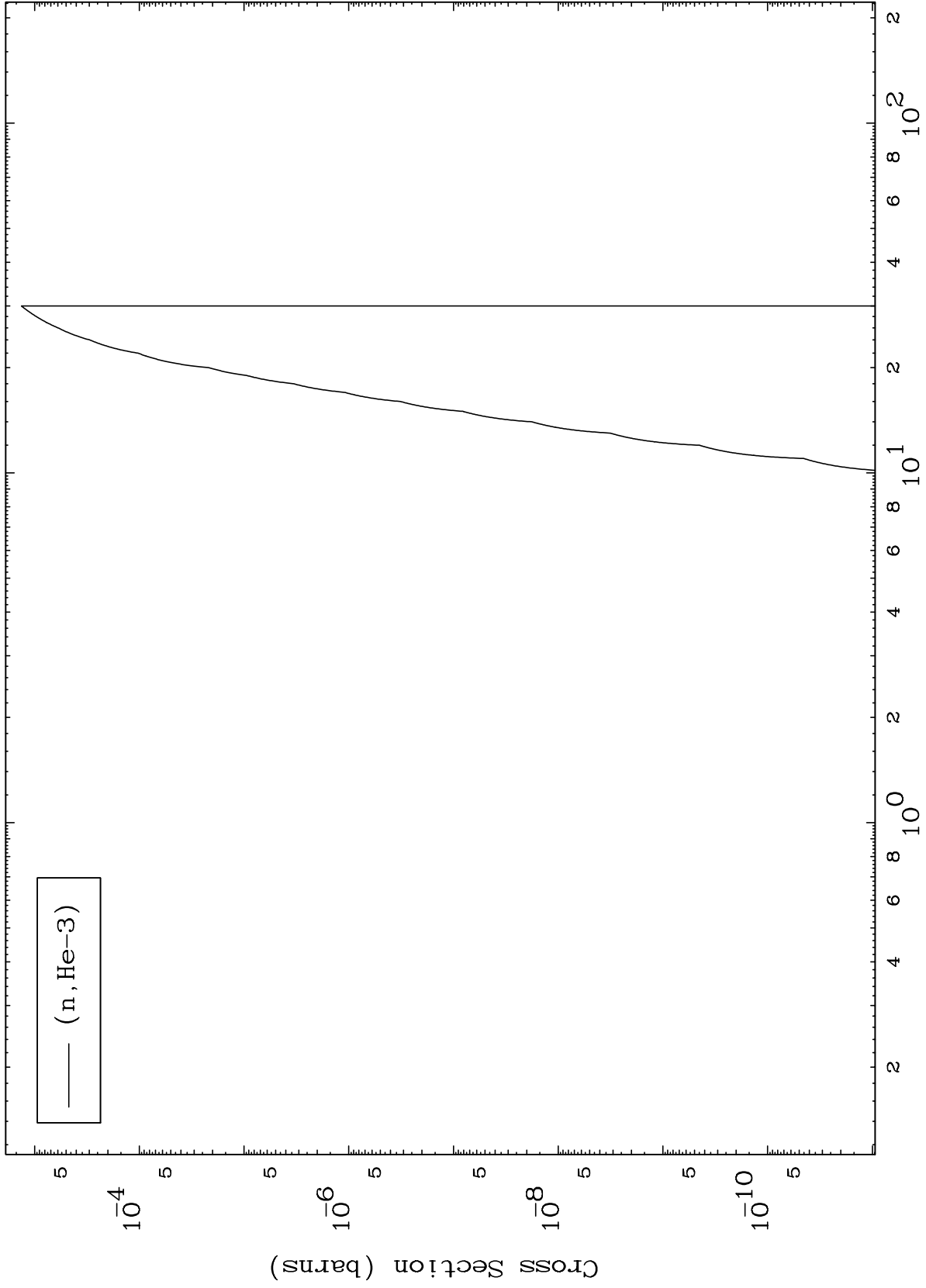
85-At-202m

MAT 8523

(d,He3) Levels

85-At-202m

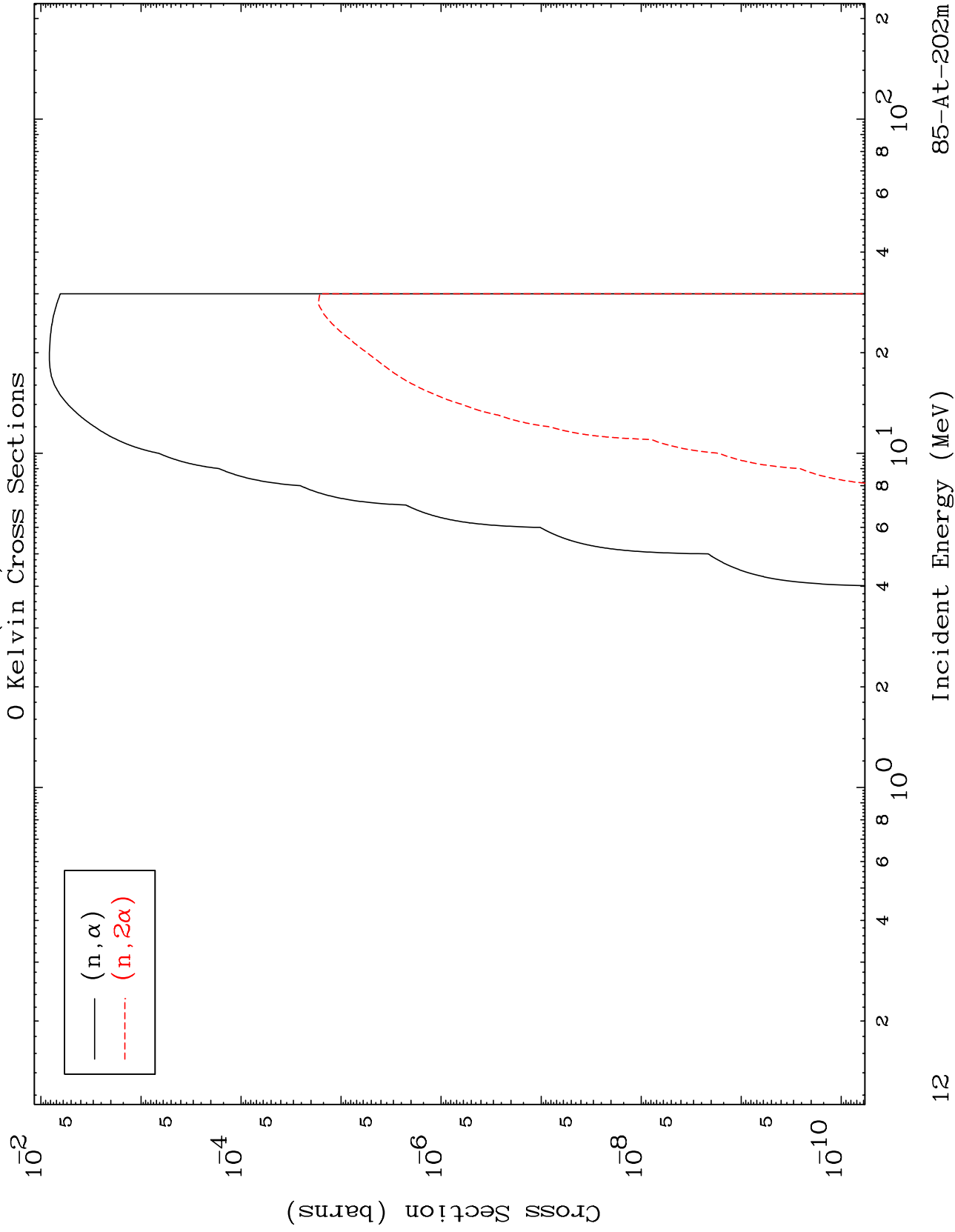
0 Kelvin Cross Sections



MAT 8523

(d, α) Levels

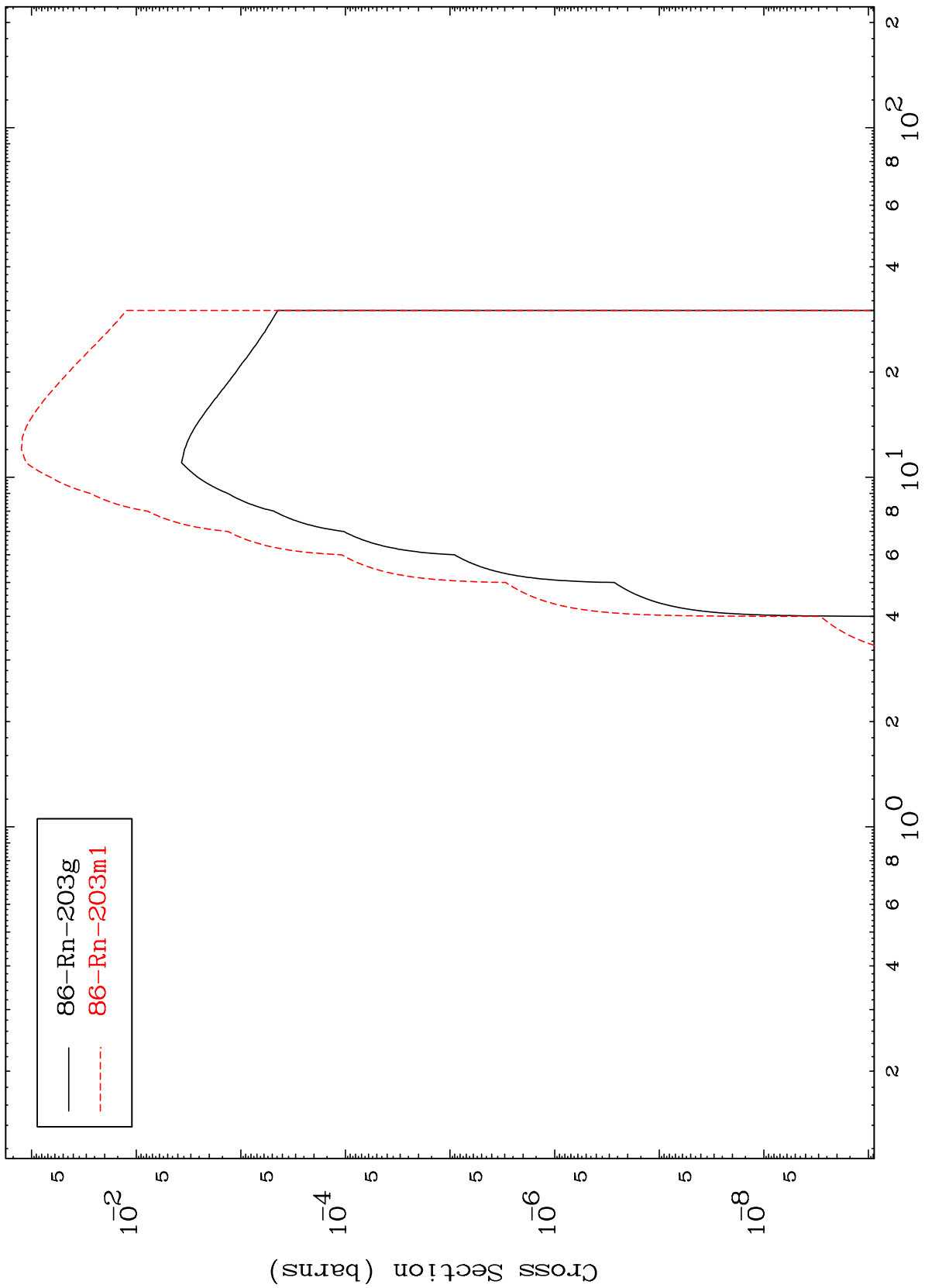
85-At-202m



MAT 8523

85-At-202m

Radionuclide Production Cross Section



86-Rn-203g
86-Rn-203m1

85-At-202m

Incident Energy (MeV)

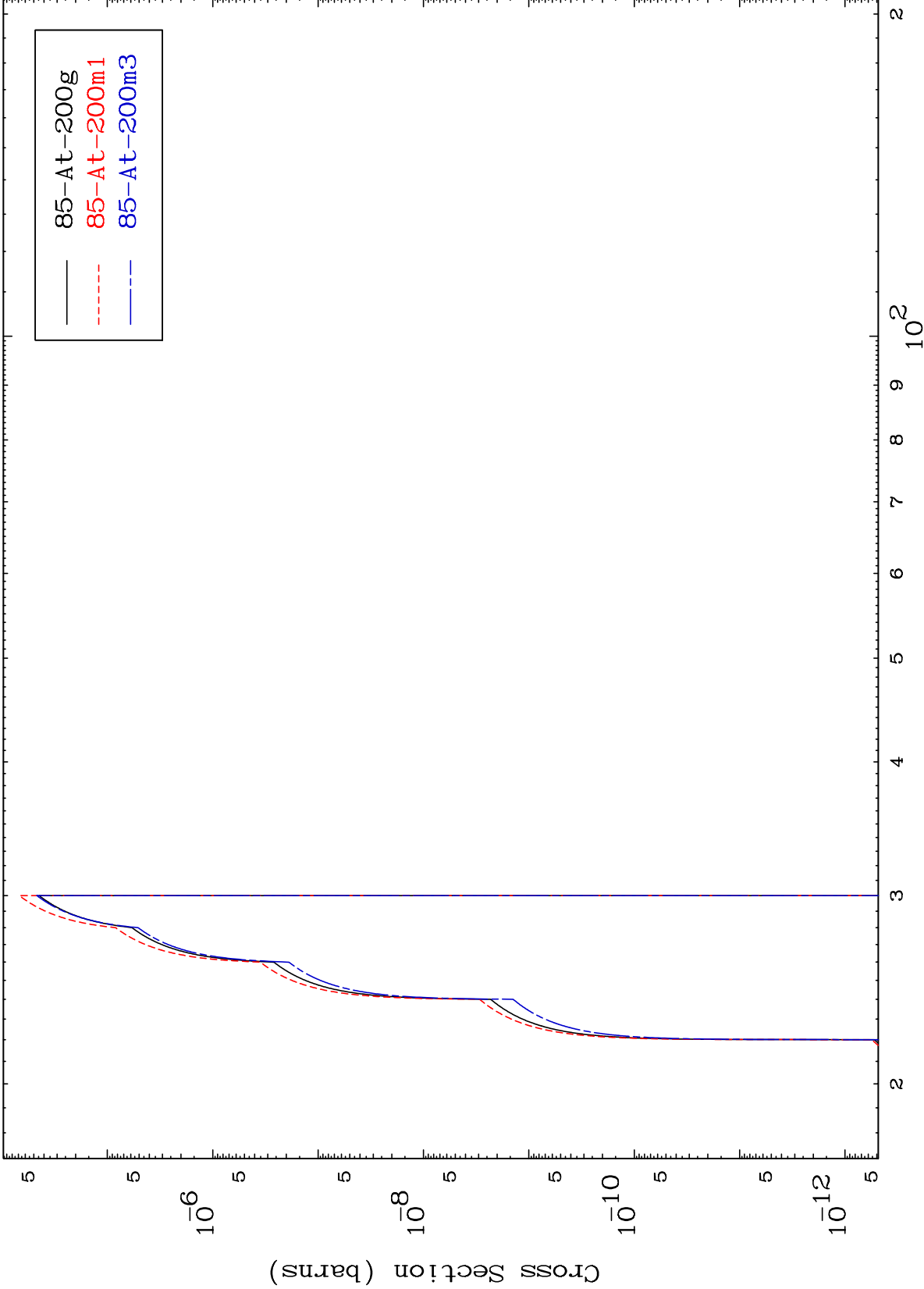
13

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(n,2n) d

85-At-202m

Radionuclide Production Cross Section



14

Incident Energy (MeV)

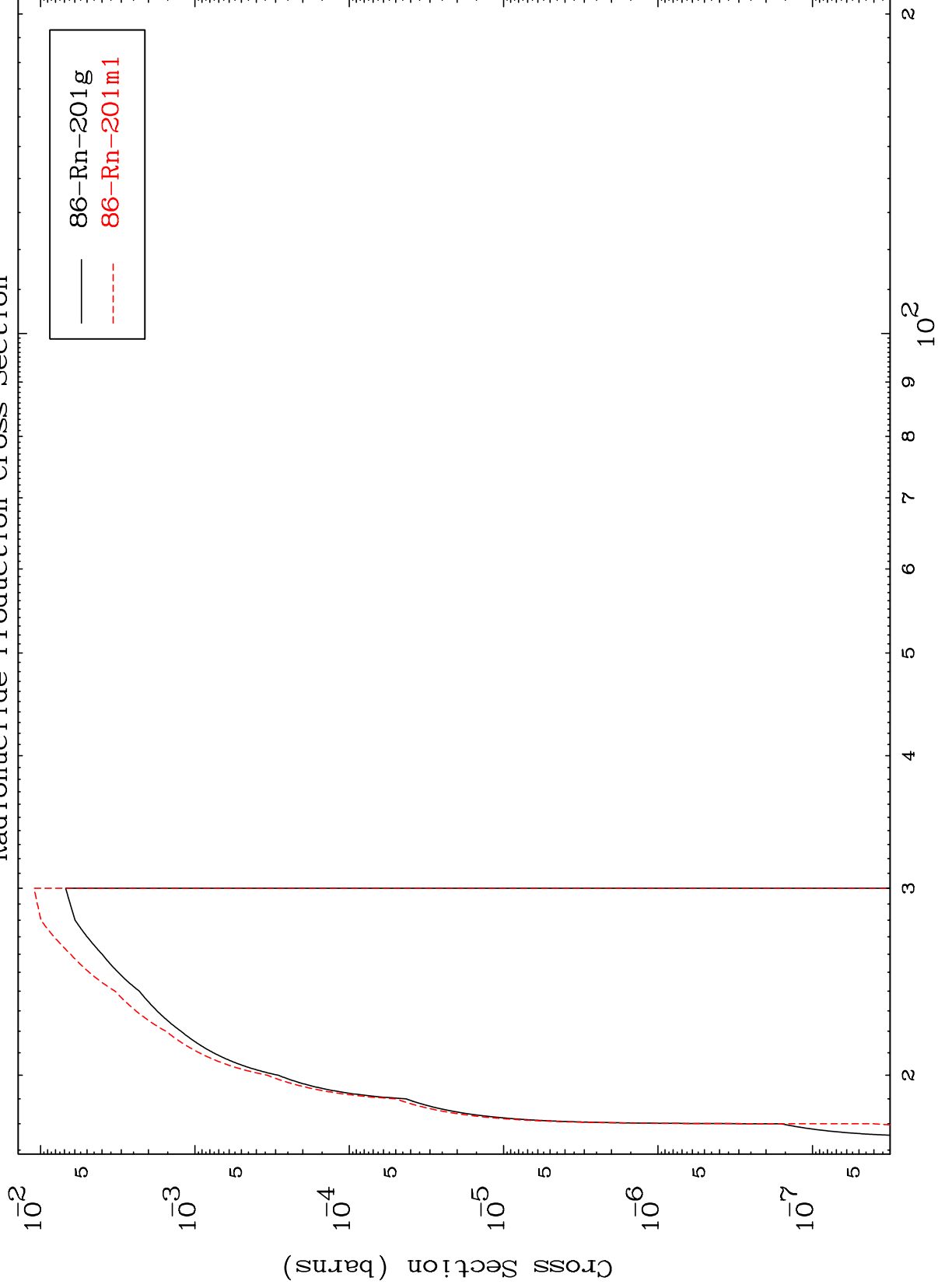
85-At-202m

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(n,3n)

85-At-202m

Radionuclide Production Cross Section



15

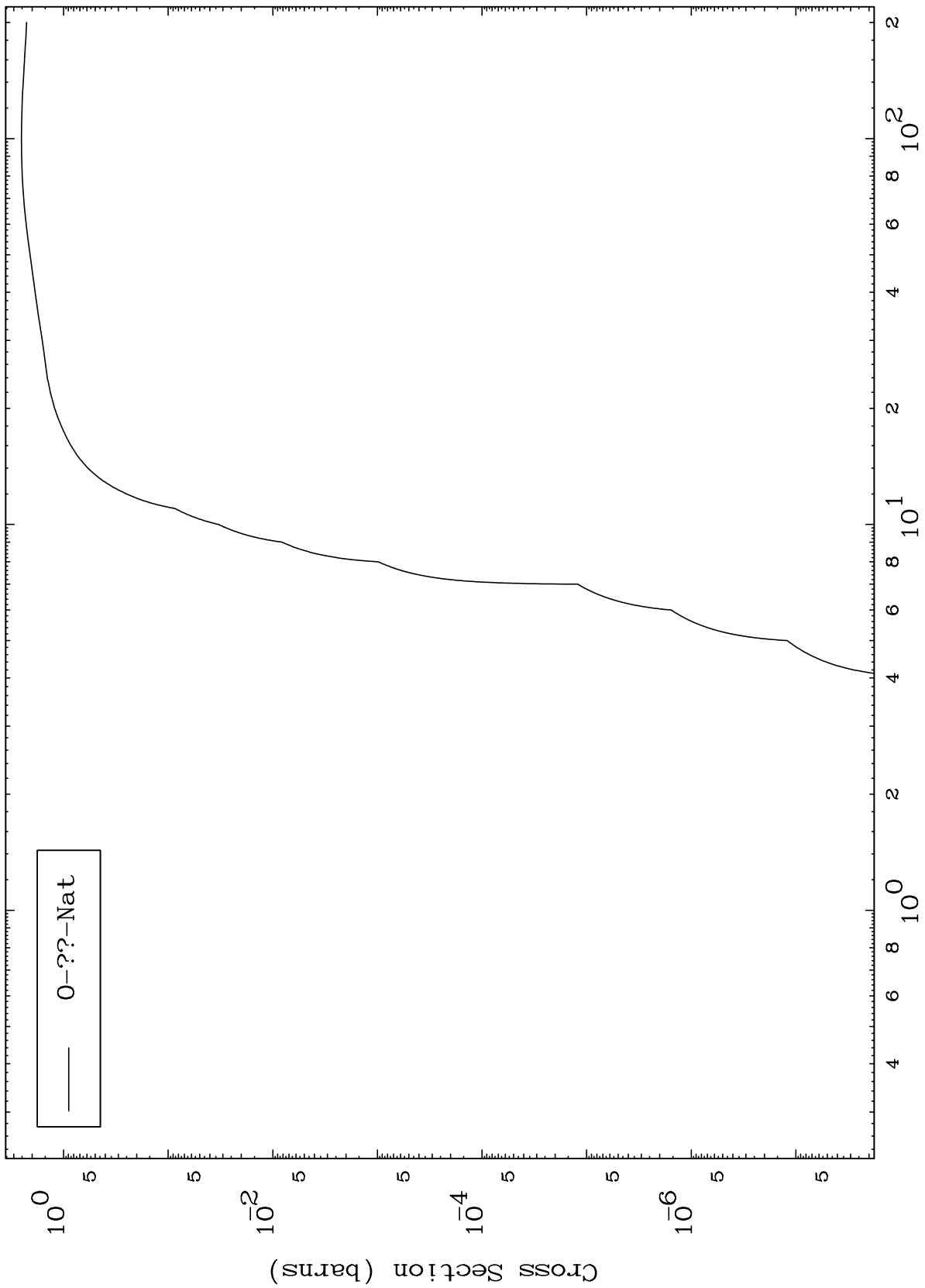
Incident Energy (MeV)

85-At-202m

MAT 8523

85-At-202m

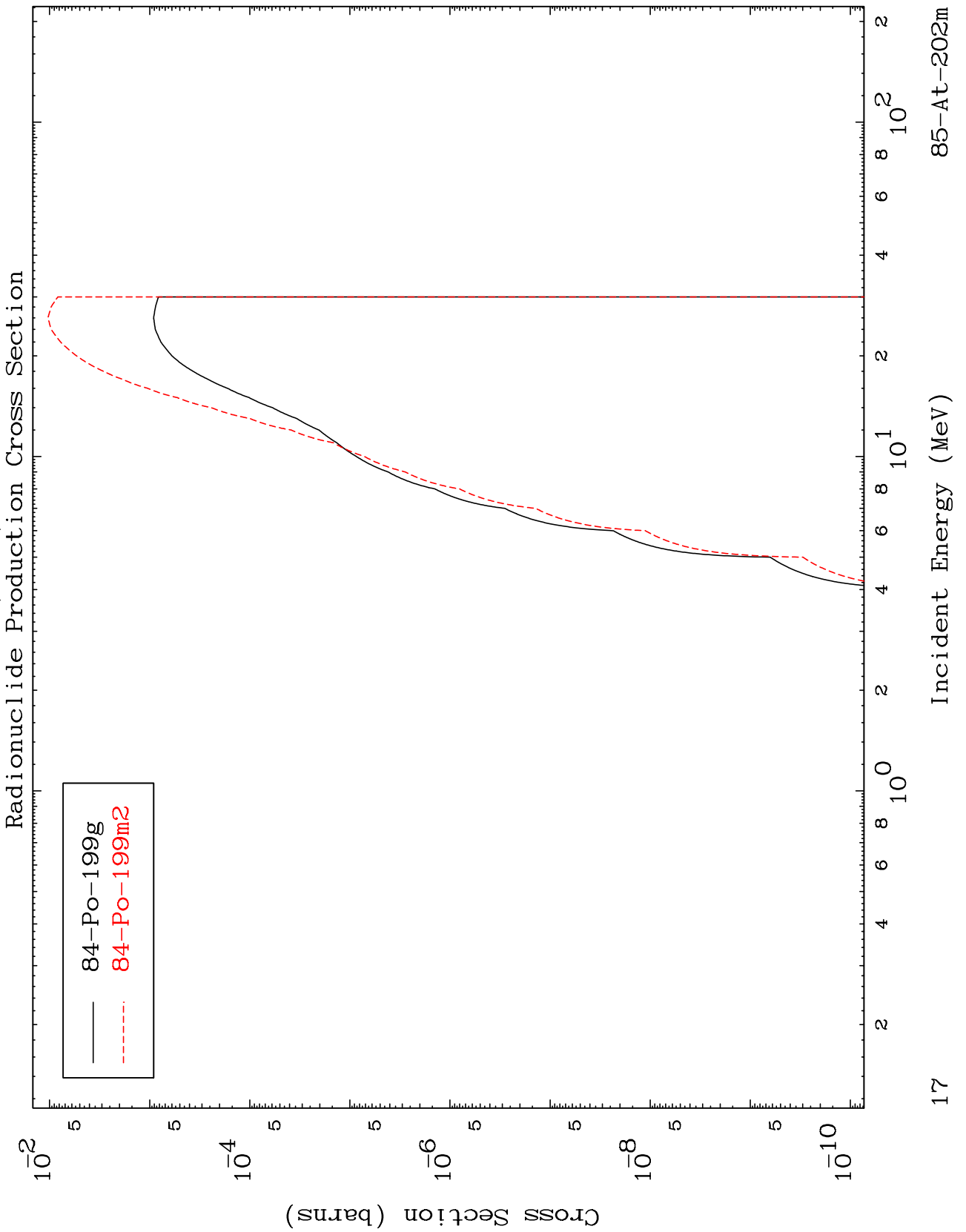
Fission
Radionuclide Production Cross Section



MAT 8523

$(n, n') \alpha$

85-At-202m

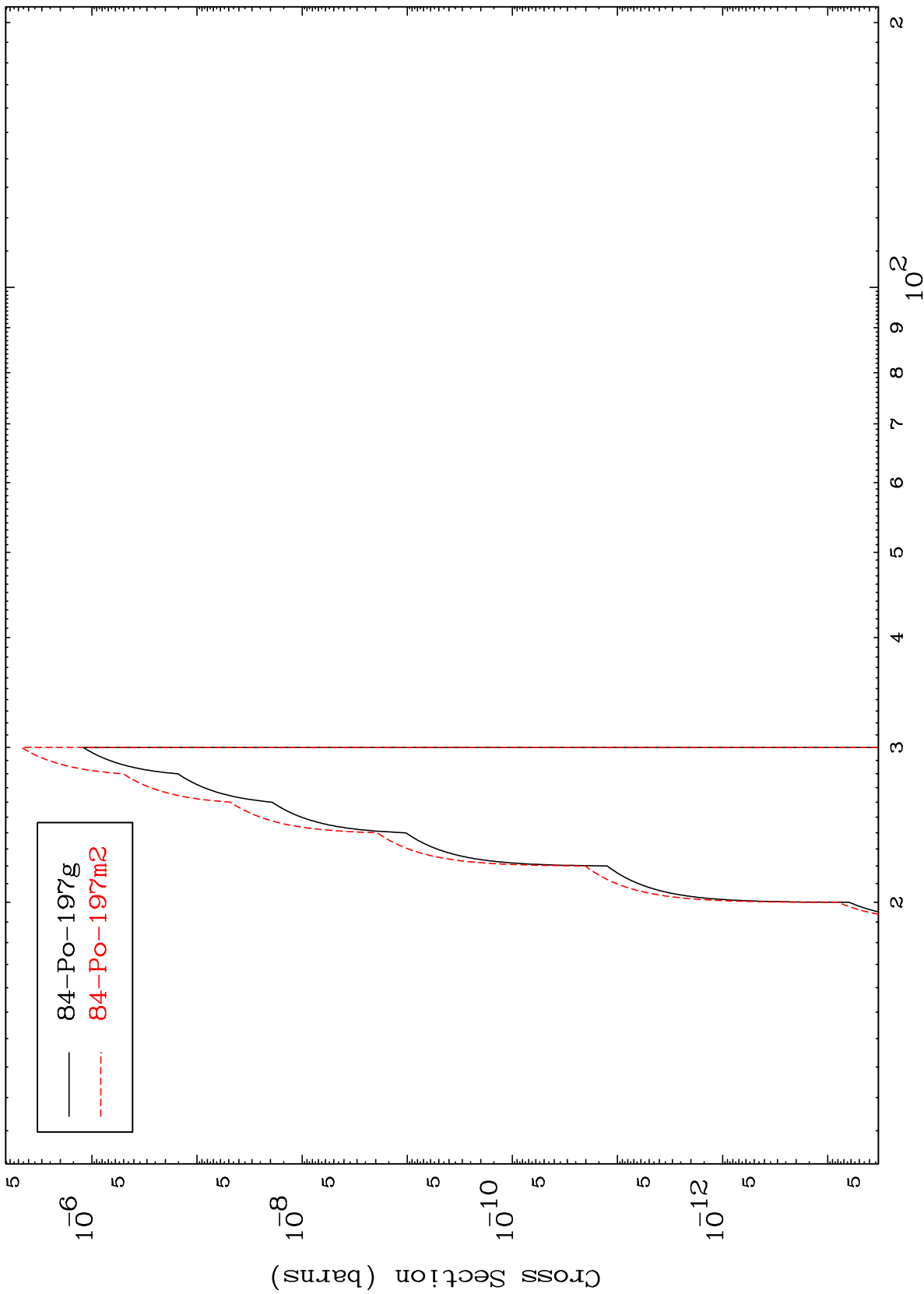


MAT 8523

(n,3n) α

85-At-202m

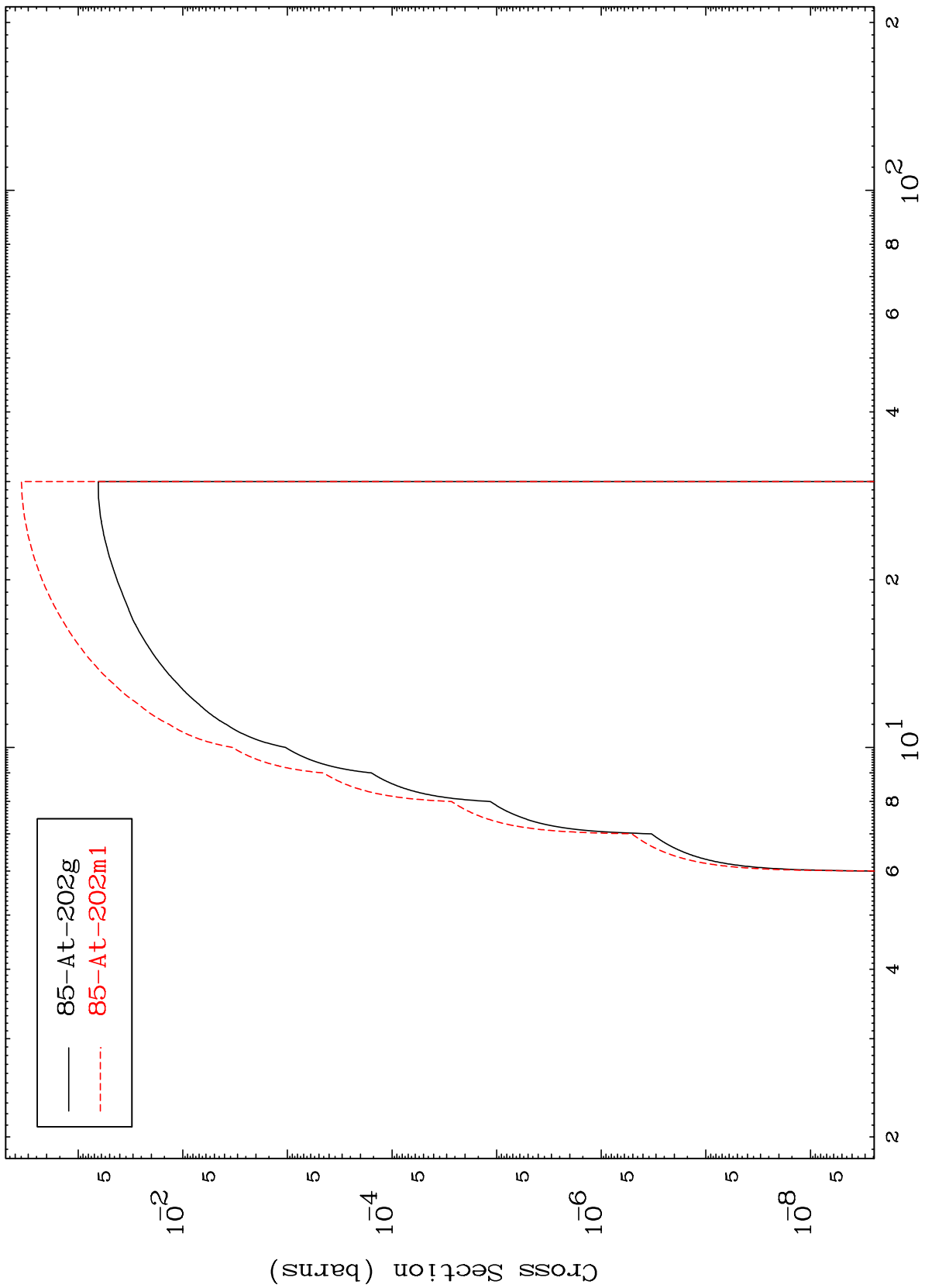
Radionuclide Production Cross Section



MAT 8523

85-At-202m

(n,n') p
Radionuclide Production Cross Section



19

Incident Energy (MeV)

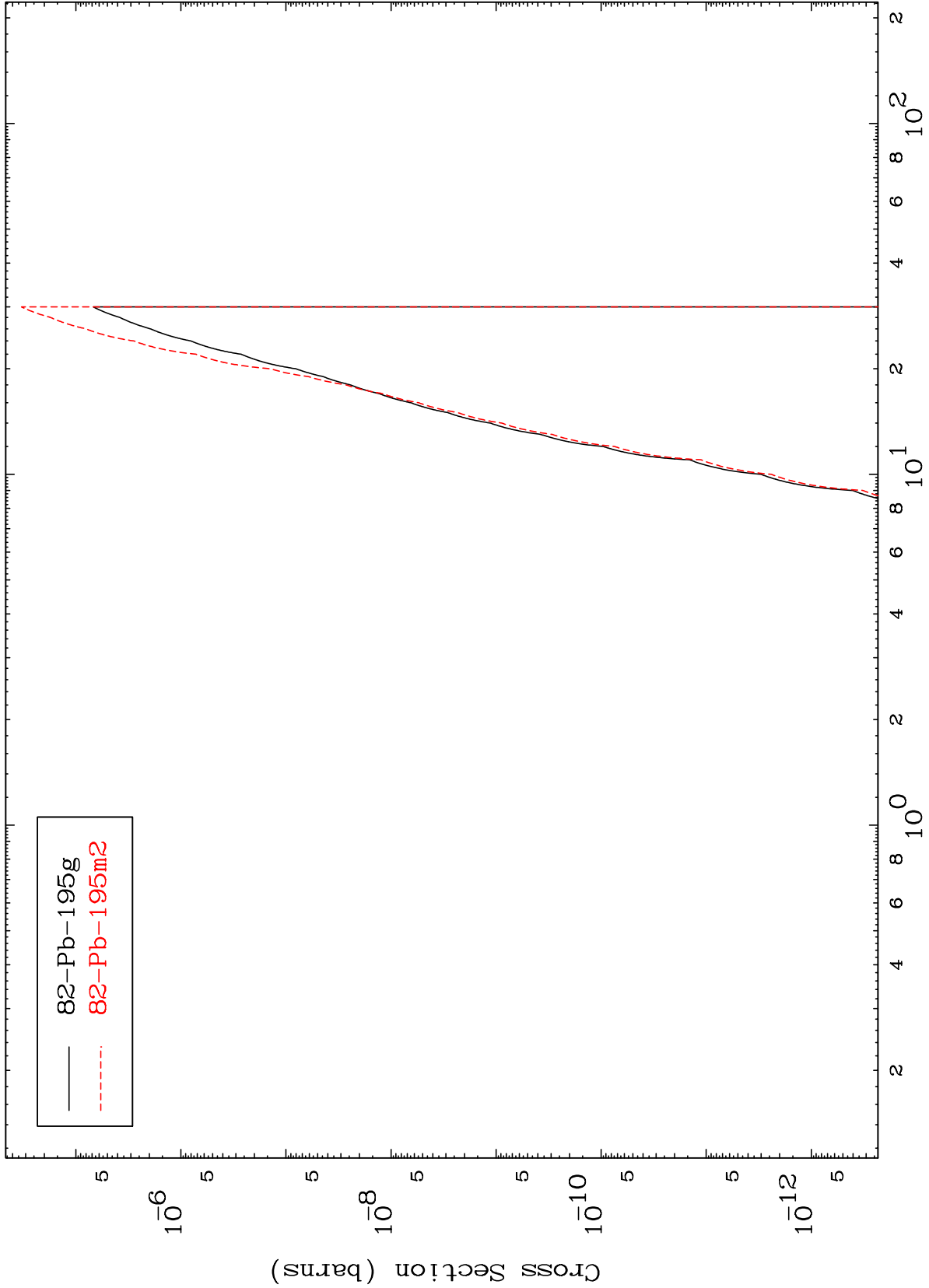
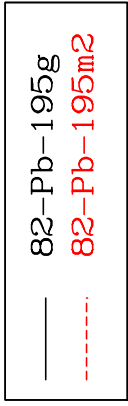
85-At-202m

MAT 8523

(n,n') 2α

85-At-202m

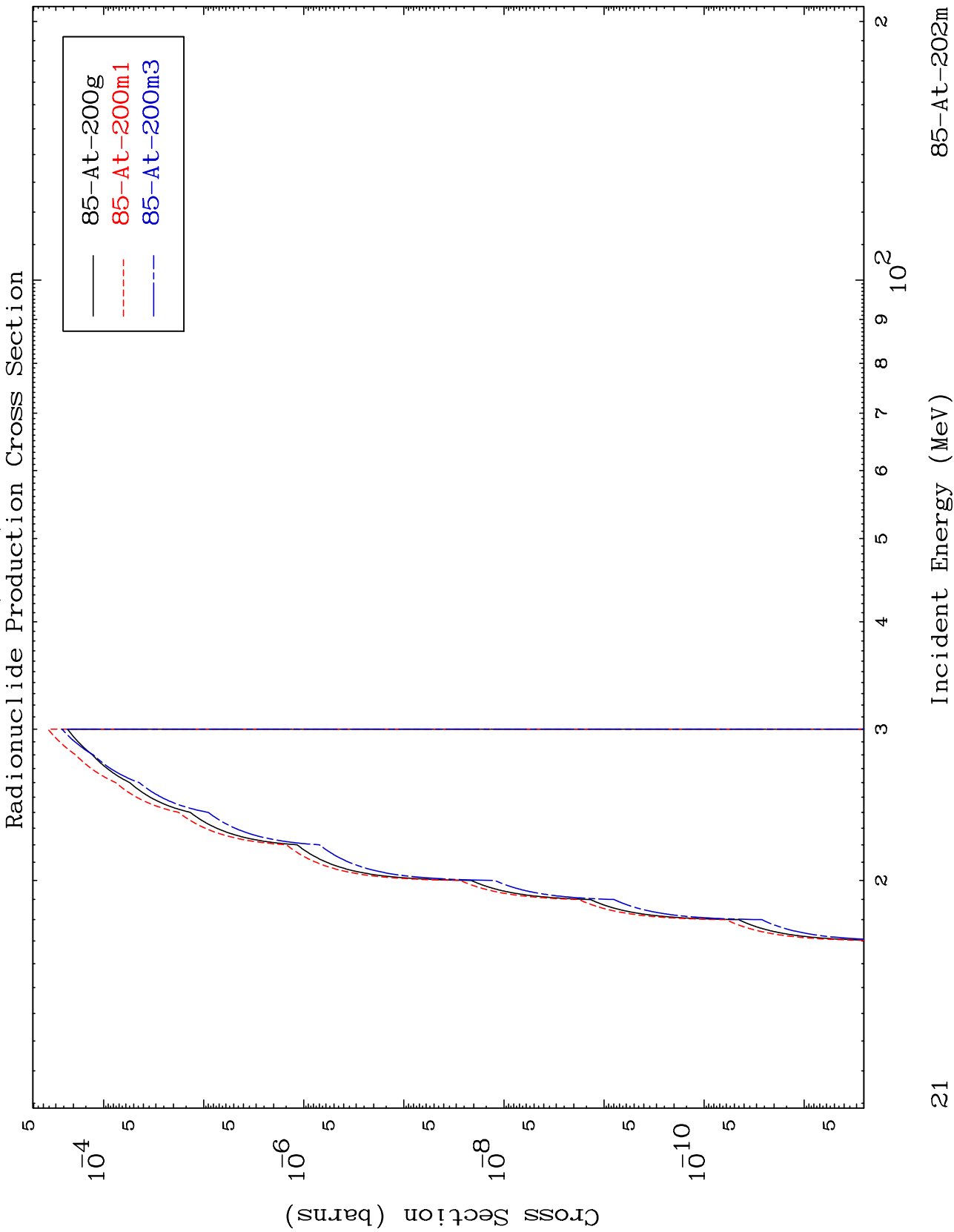
Radionuclide Production Cross Section



MAT 8523

(n,n') t

85-At-202m

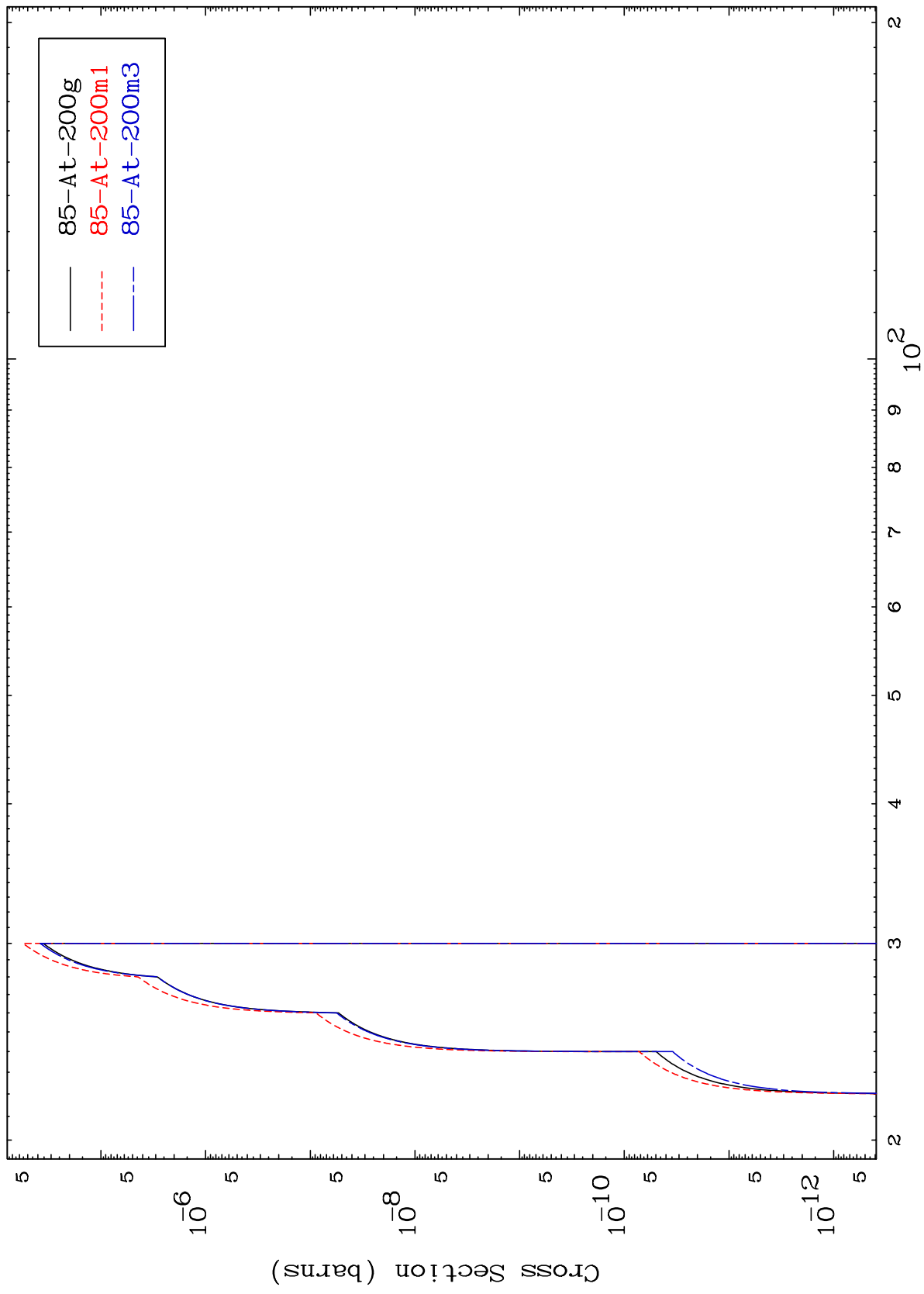


MAT 8523

(n,3n) p

85-At-202m

Radionuclide Production Cross Section



22

Incident Energy (MeV)

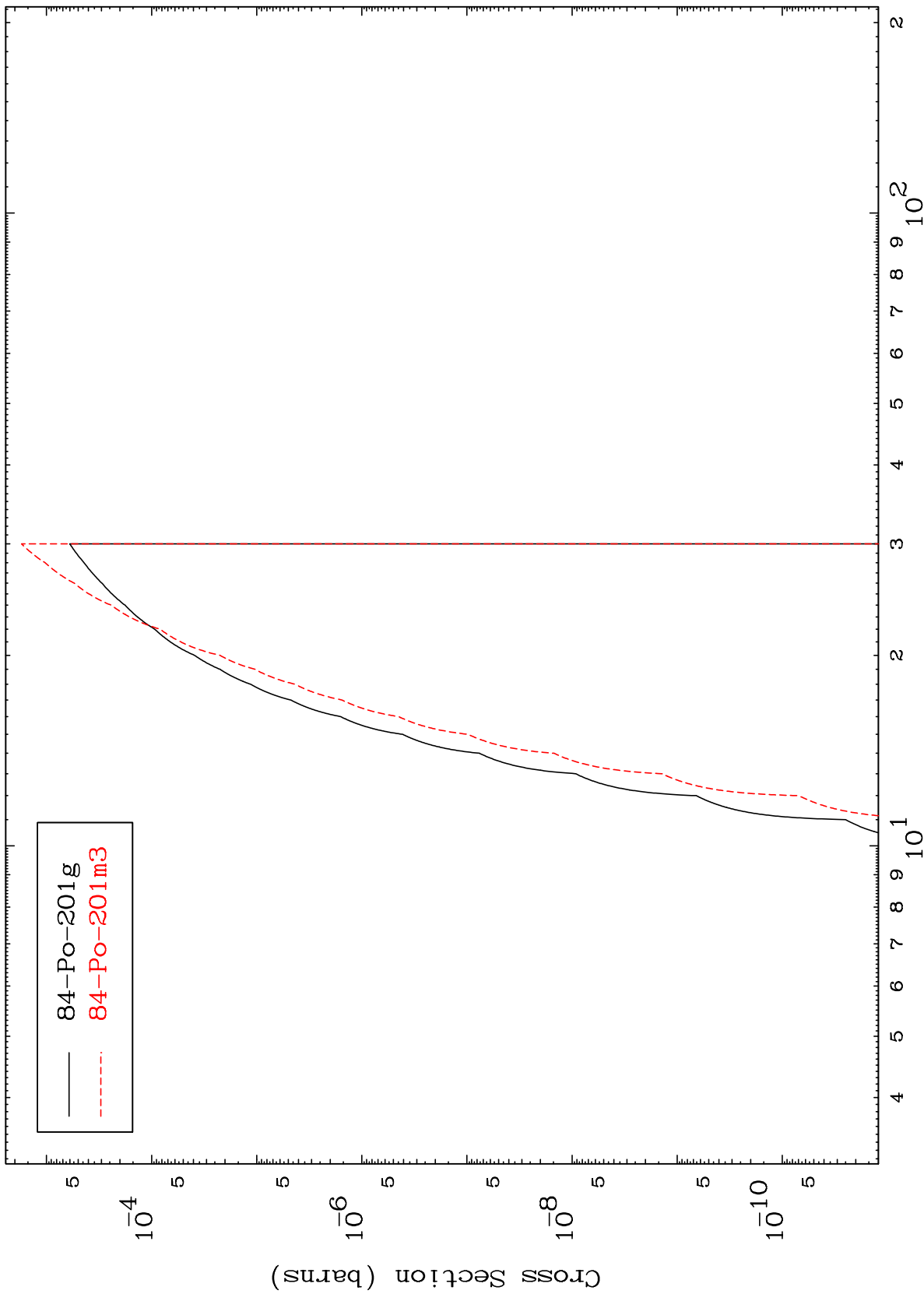
85-At-202m

MAT 8523

(n,2n) p

85-At-202m

Radionuclide Production Cross Section



Incident Energy (MeV)

85-At-202m

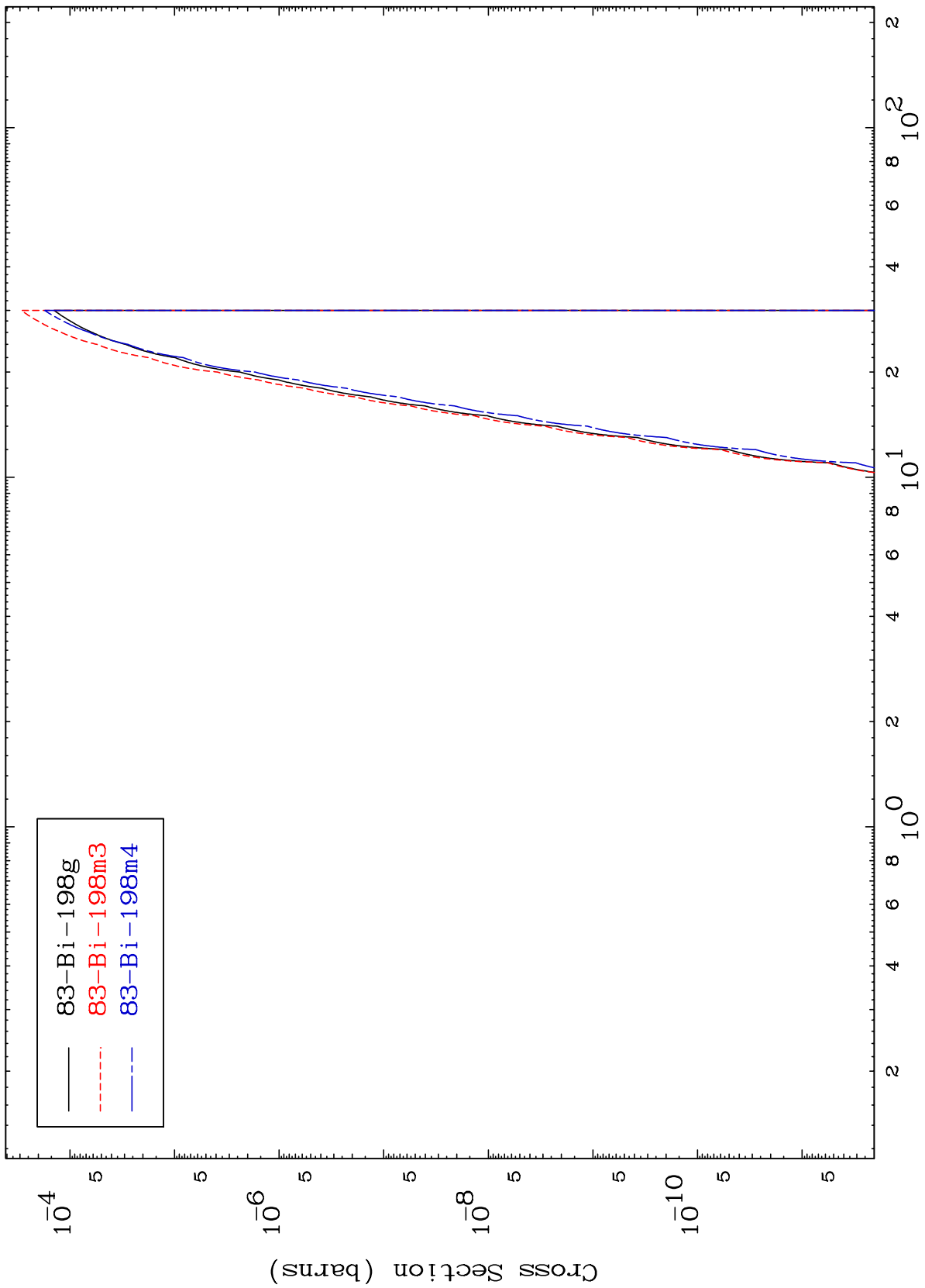
23

MAT 8523

(n,n') p α

85-At-202m

Radionuclide Production Cross Section

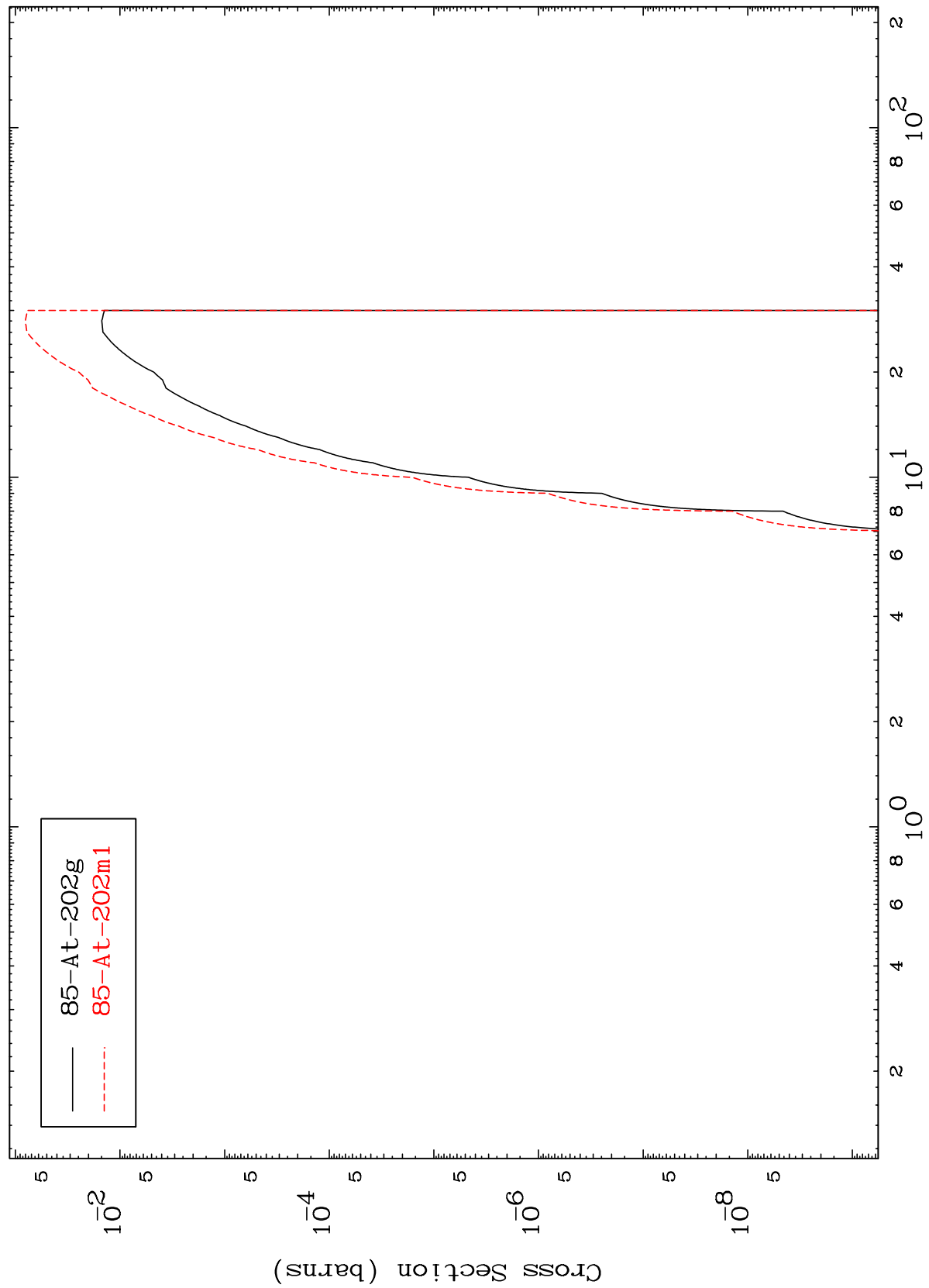


MAT 8523

(n,d)

85-At-202m

Radionuclide Production Cross Section

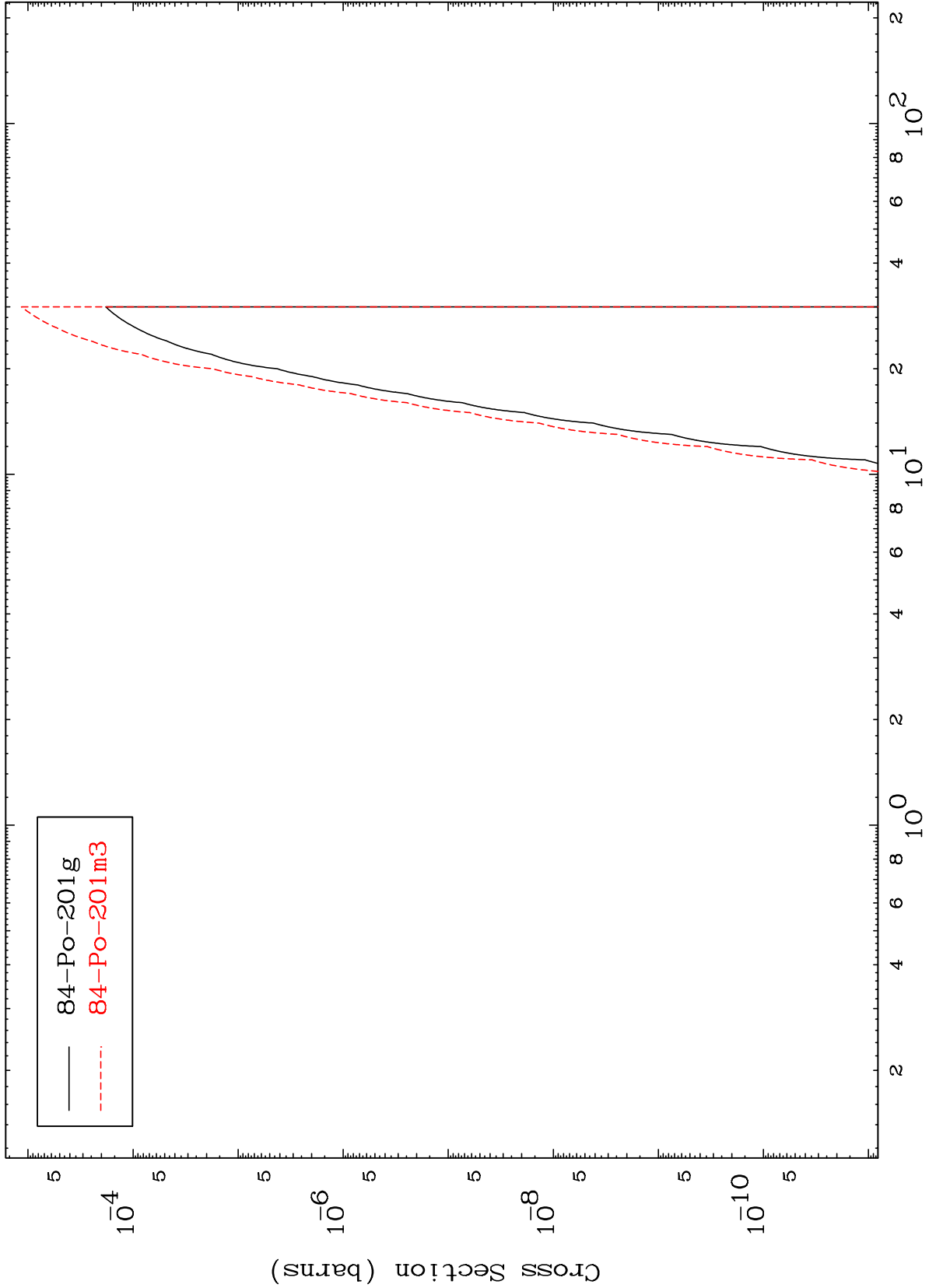


MAT 8523

(n,He-3)

85-At-202m

Radionuclide Production Cross Section



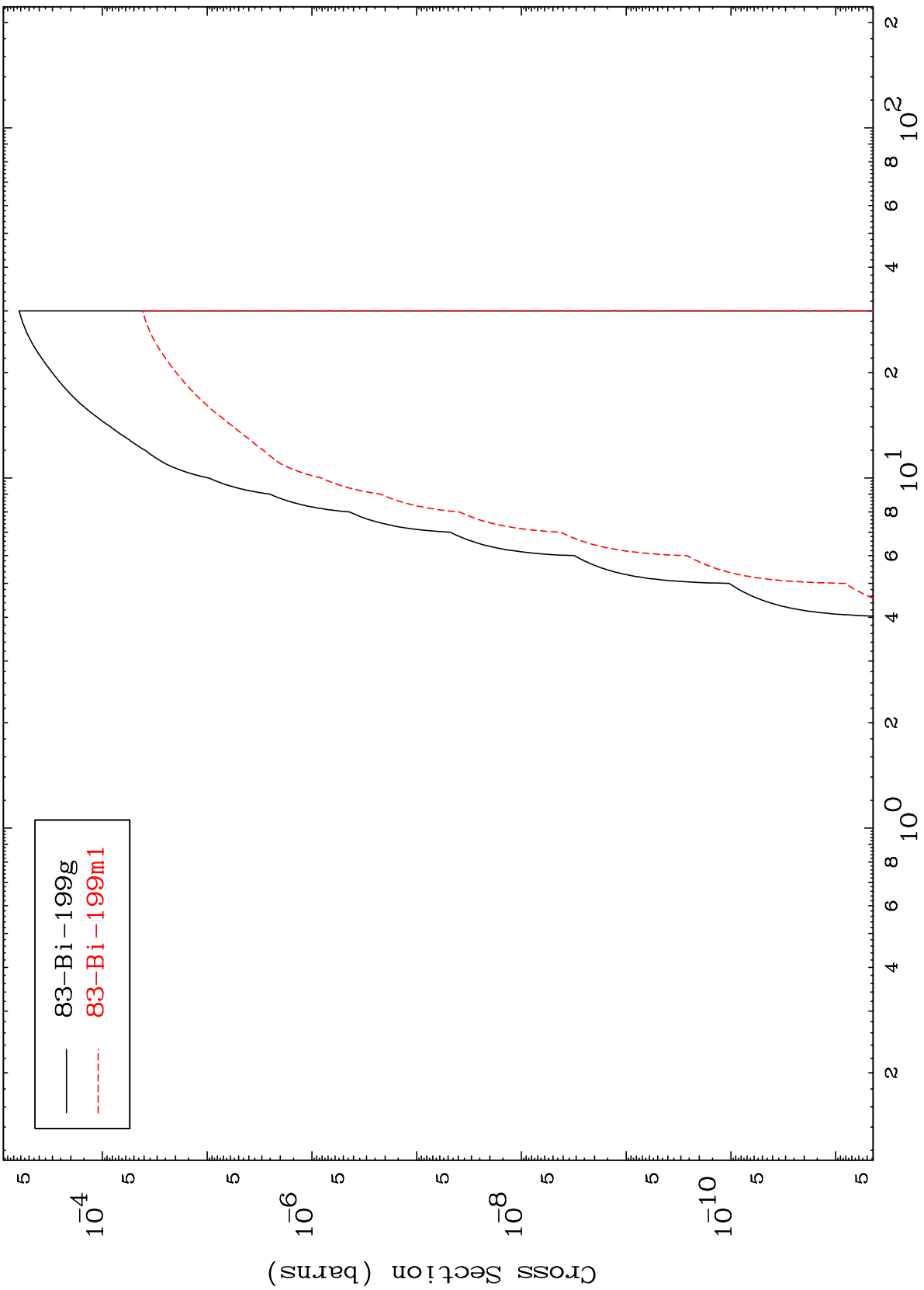
— 84-Po-201 g
- - - 84-Po-201 m3

MAT 8523

(n,p) α

85-At-202m

Radionuclide Production Cross Section



83-Bi-199g
83-Bi-199m1

Incident Energy (MeV)

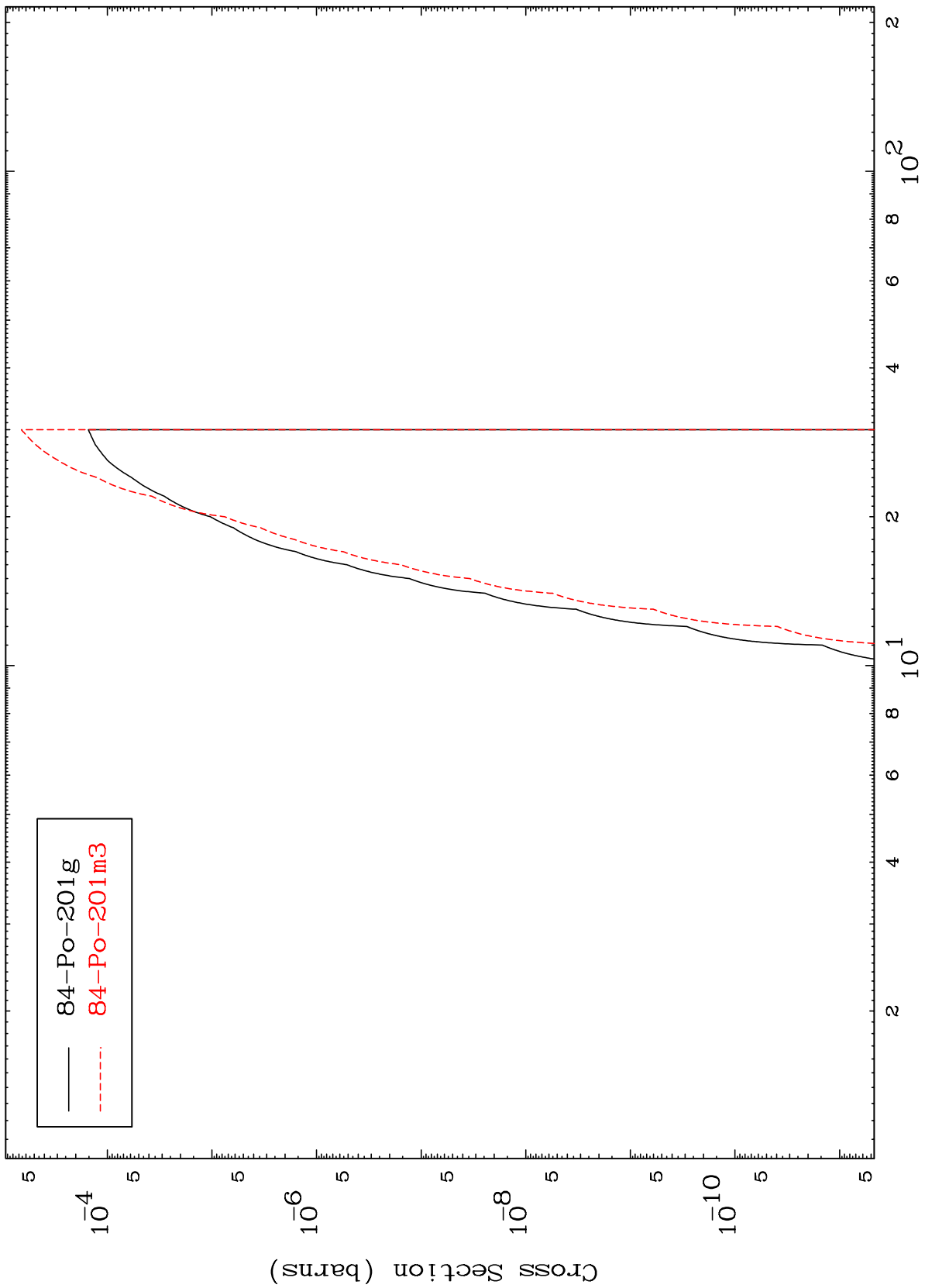
85-At-202m

MAT 8523

(n,p) d

85-At-202m

Radionuclide Production Cross Section



MAT 8523

(n,d) α

85-At-202m

