

Program EVALPLOT  
(Version 2021-1)

by

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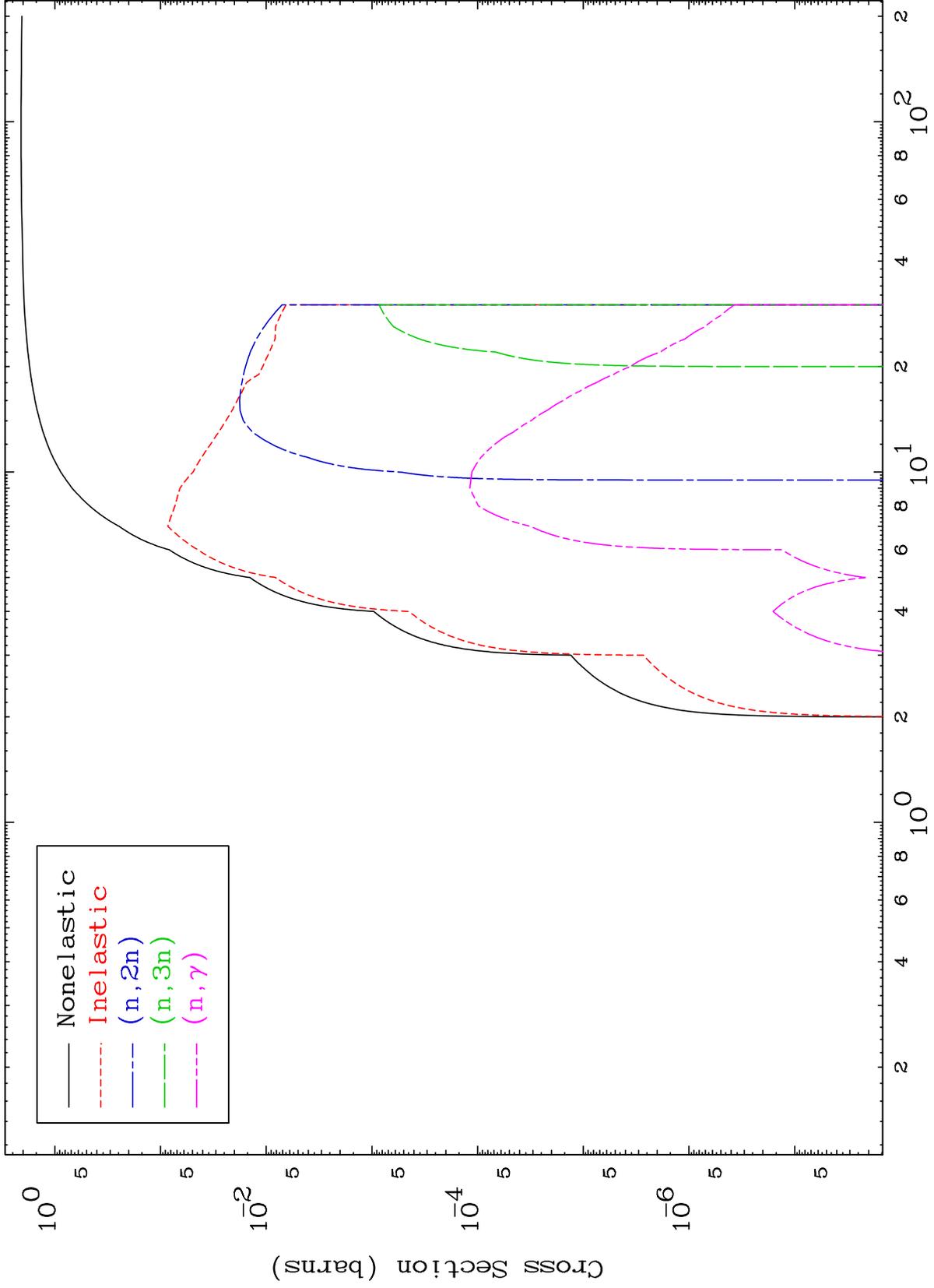
Press Mouse Button to Start

MAT 4901

Deuteron Major

49-In-105

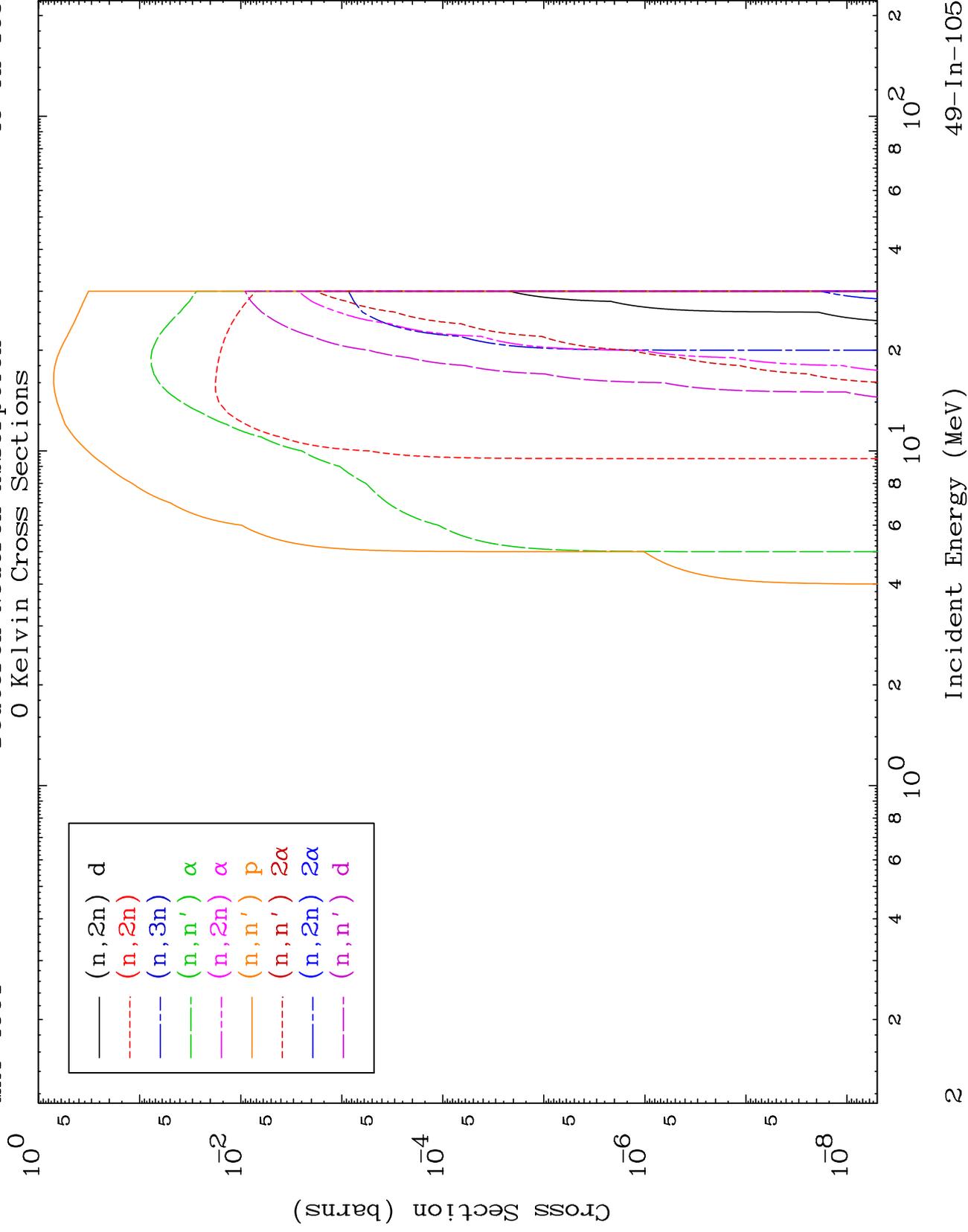
0 Kelvin Cross Sections



MAT 4901

Deuteron Neutron Absorption  
0 Kelvin Cross Sections

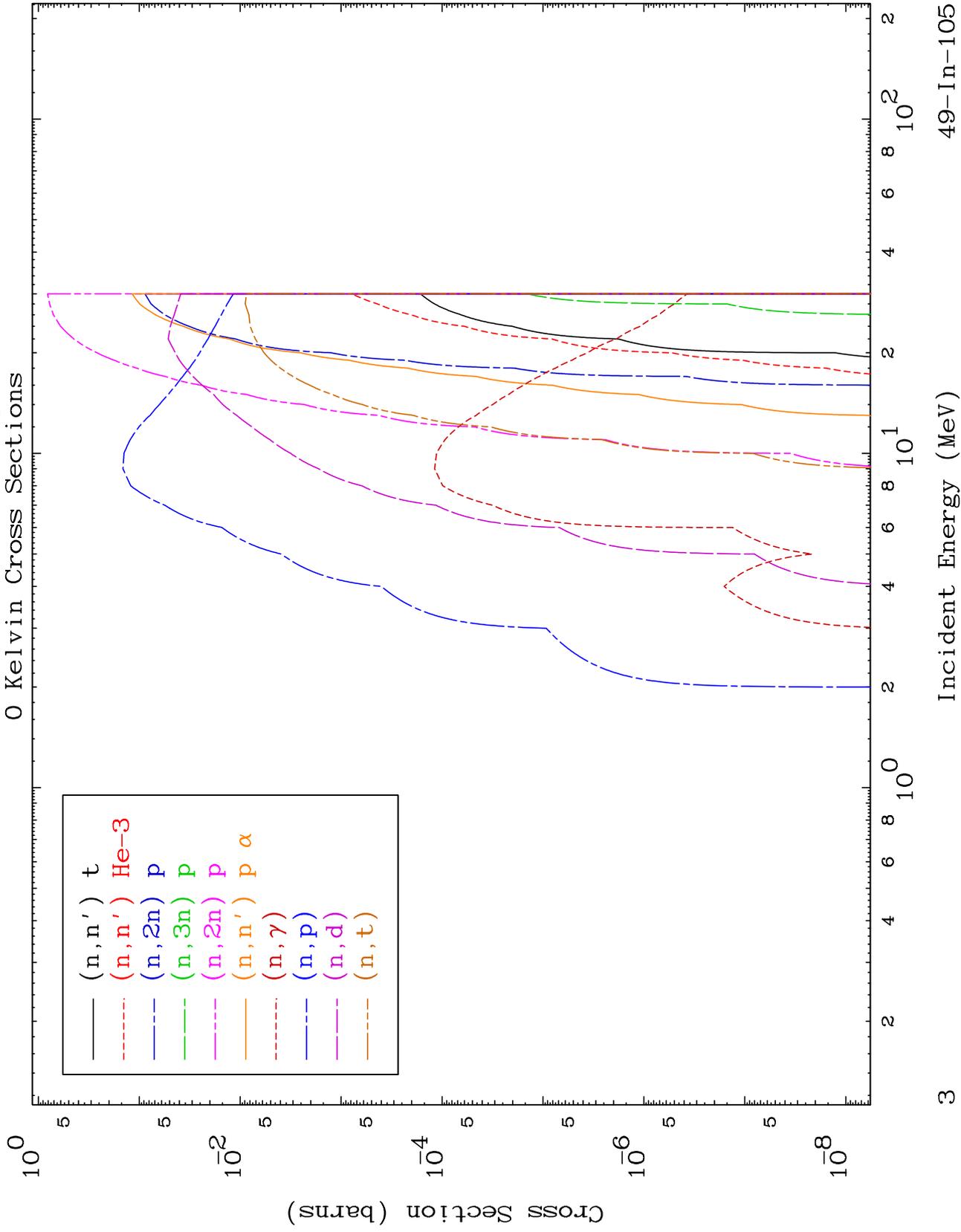
49-In-105



MAT 4901

Deuteron Neutron Absorption  
0 Kelvin Cross Sections

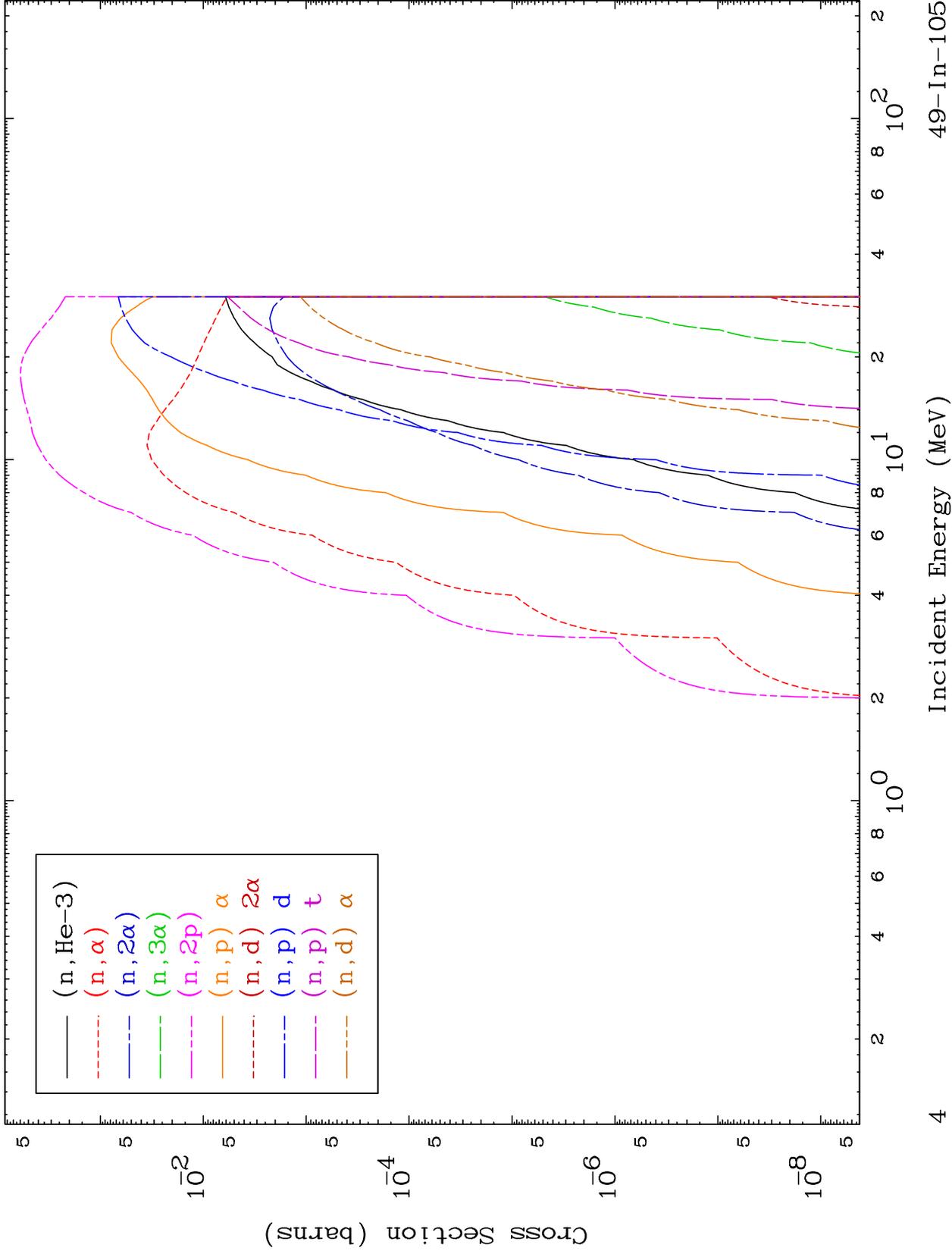
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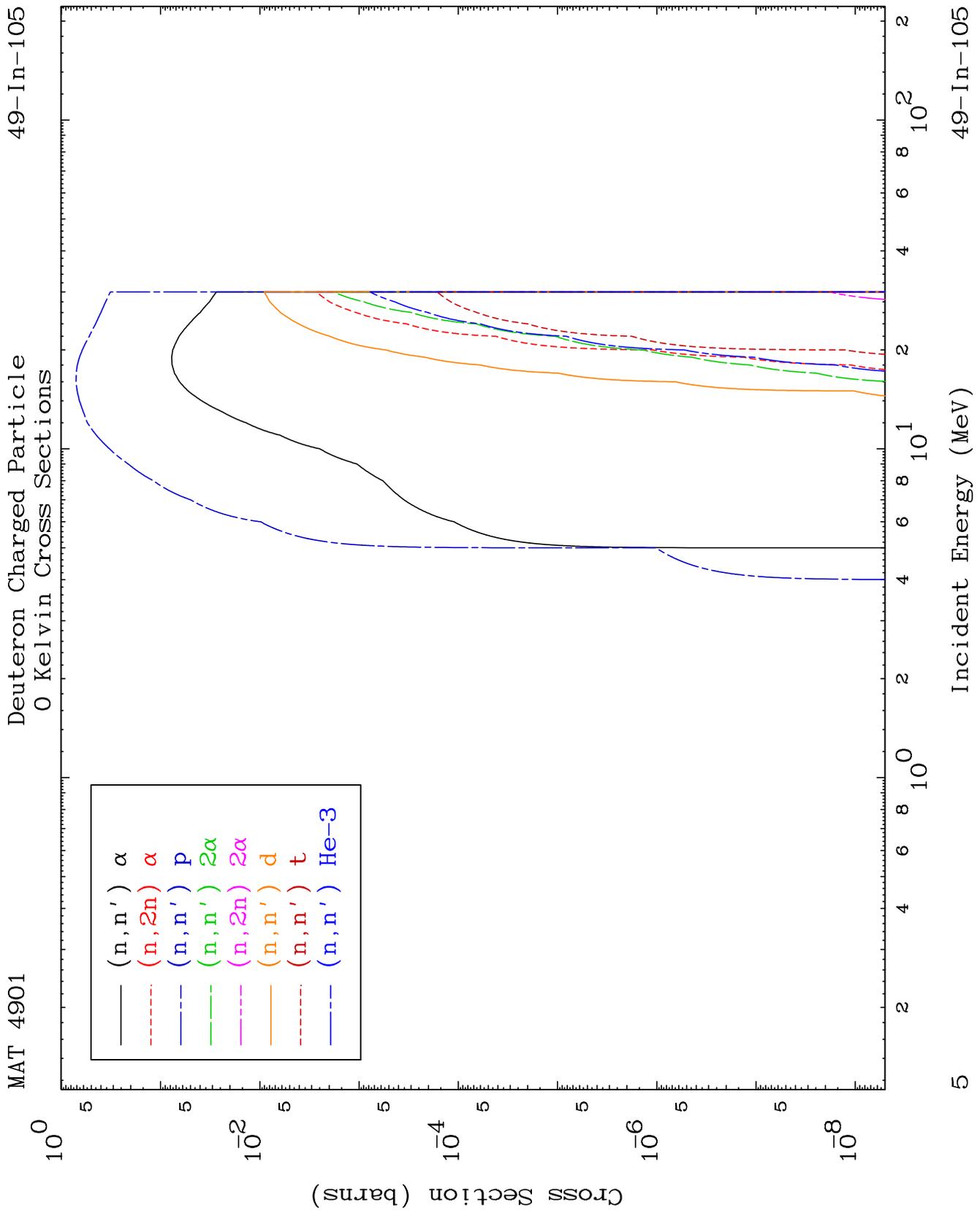


MAT 4901

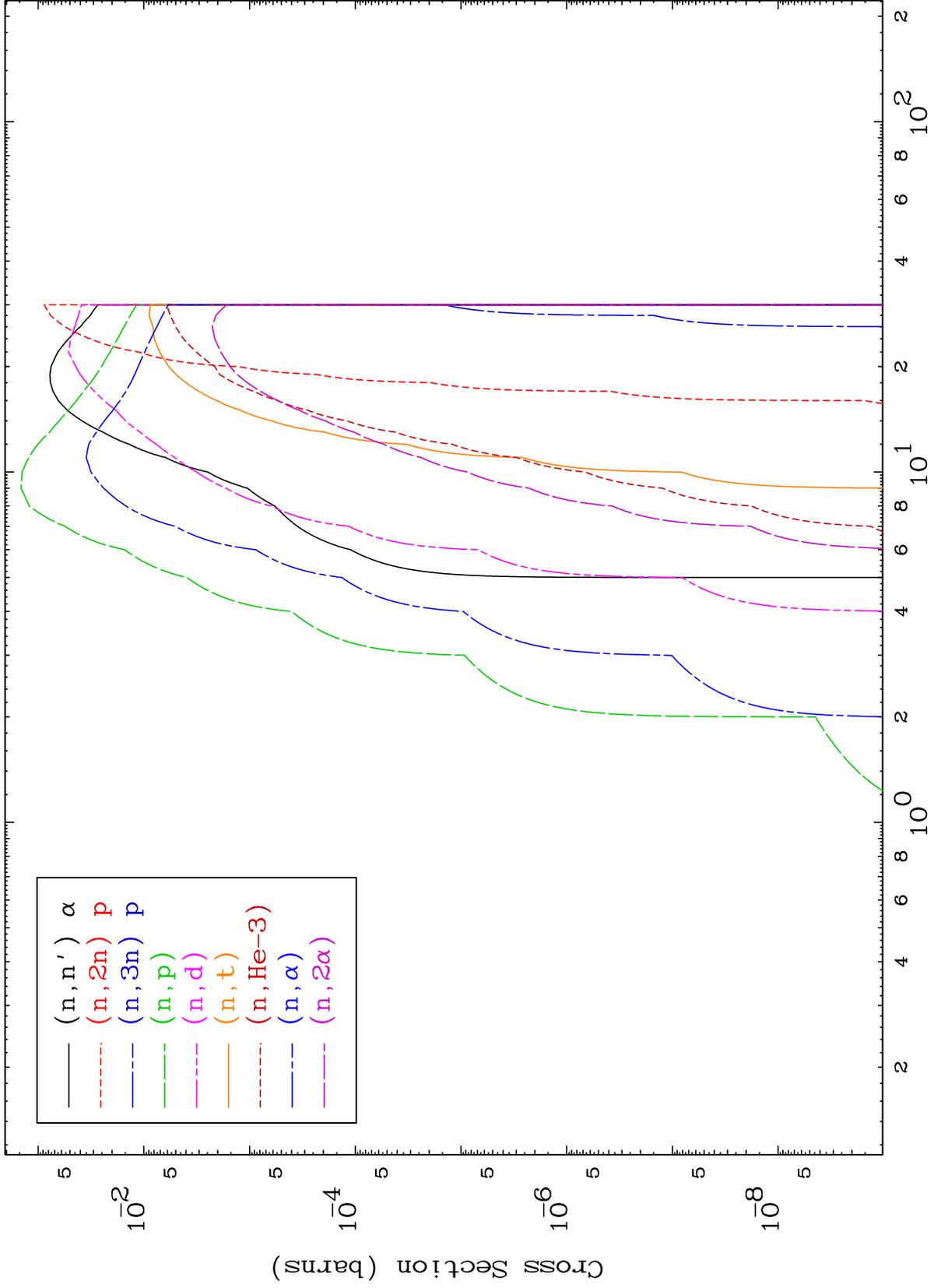
Deuteron Neutron Absorption  
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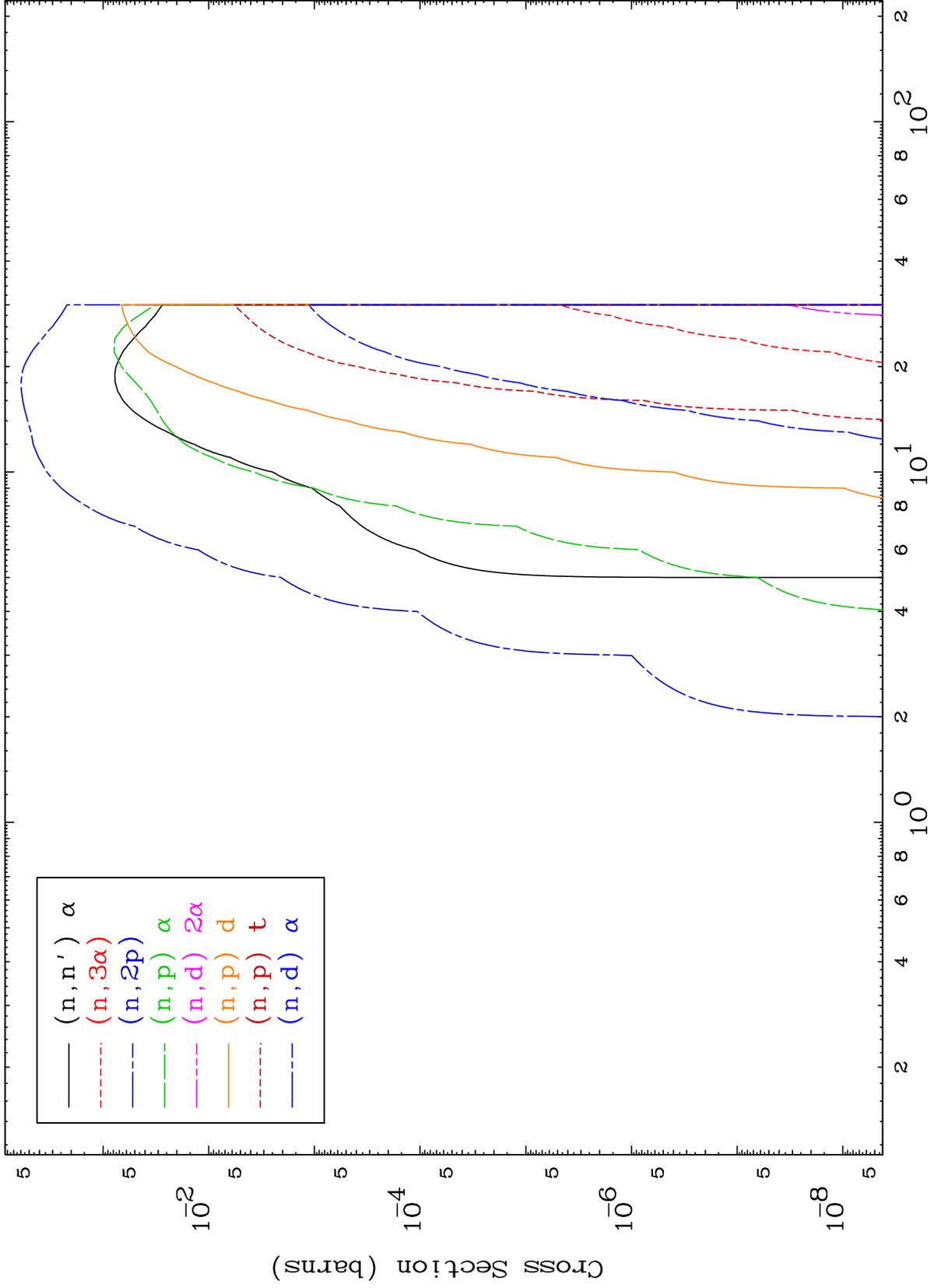
49-In-105





—	(n, n') $\alpha$
- - -	(n, 2n) $\alpha$
- · -	(n, n') p
- - -	(n, n') 2 $\alpha$
- - -	(n, 2n) 2 $\alpha$
—	(n, n') d
- · -	(n, n') t
- · -	(n, n') He-3

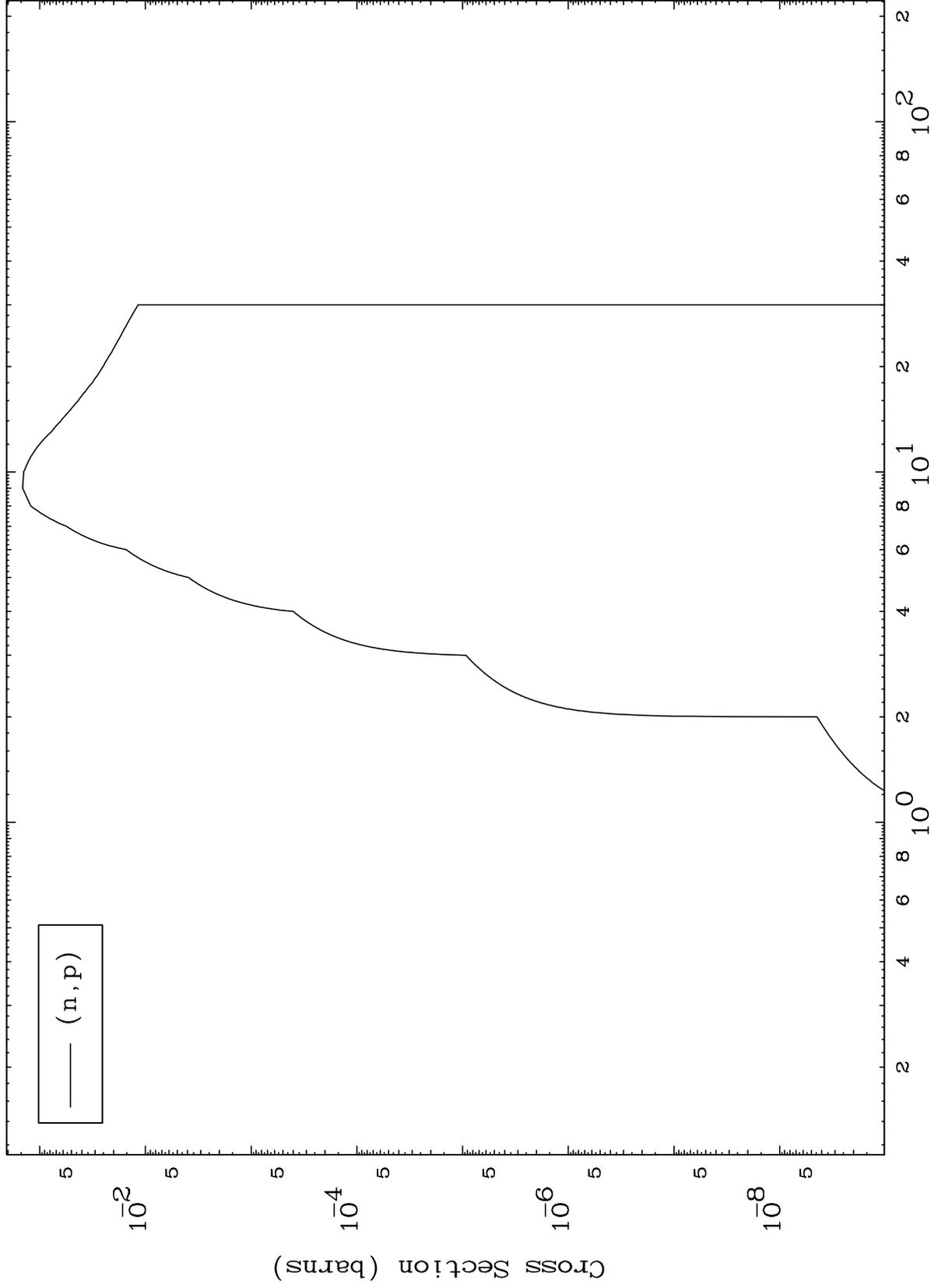




MAT 4901

49-In-105

(d,p) Levels  
0 Kelvin Cross Sections

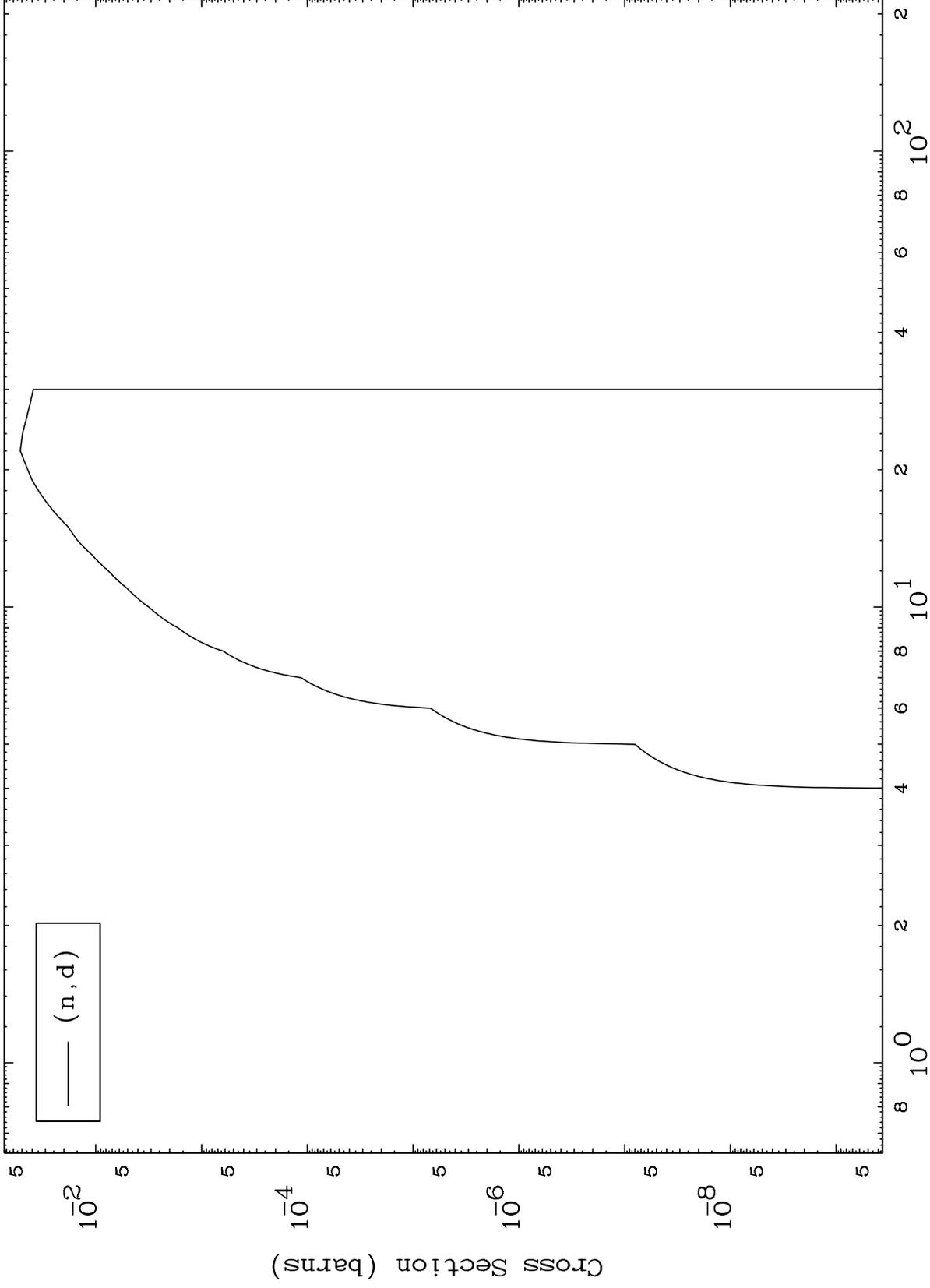


MAT 4901

(d,d) Levels

49-In-105

0 Kelvin Cross Sections



9

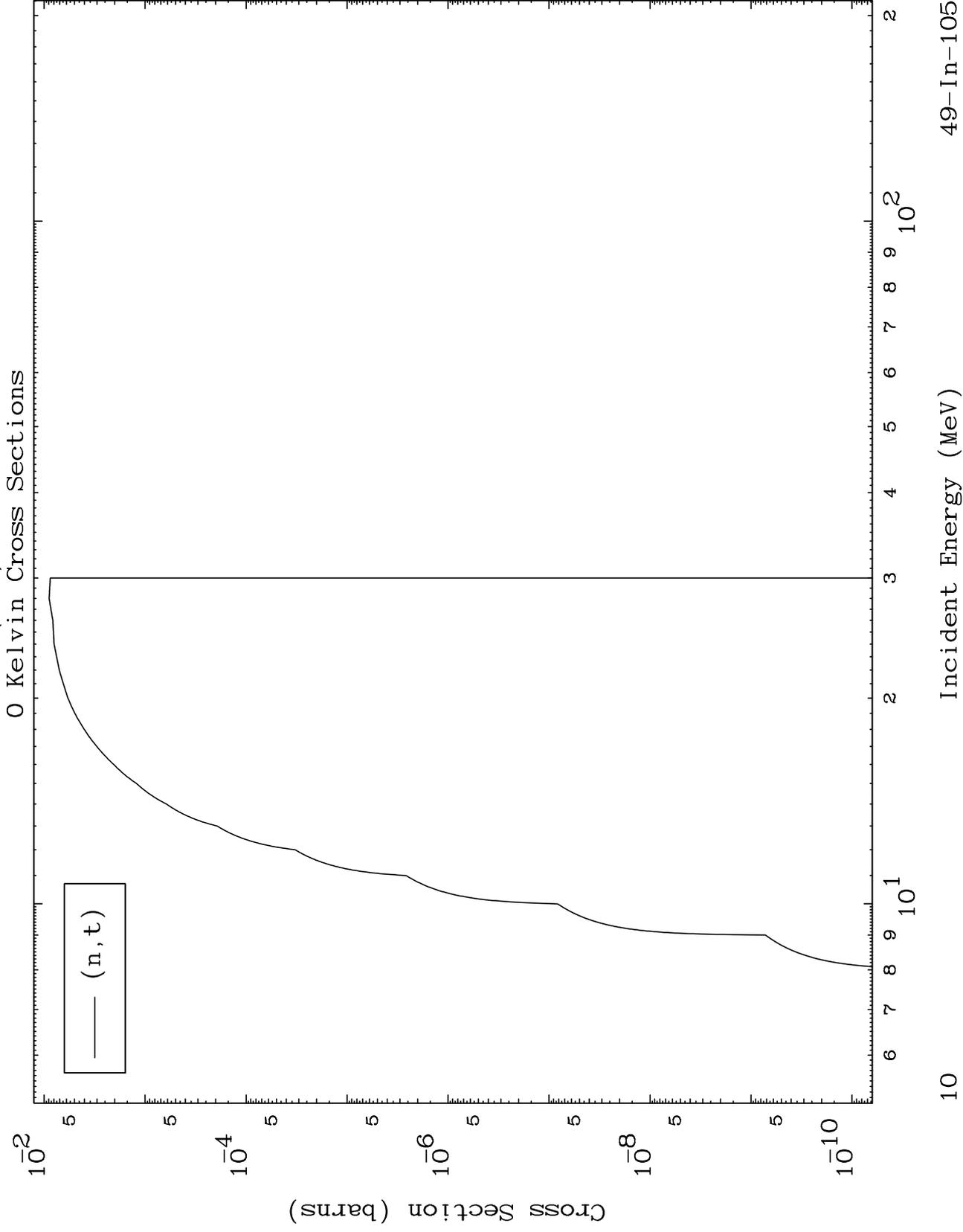
Incident Energy (MeV)

49-In-105

MAT 4901

(d,t) Levels

49-In-105

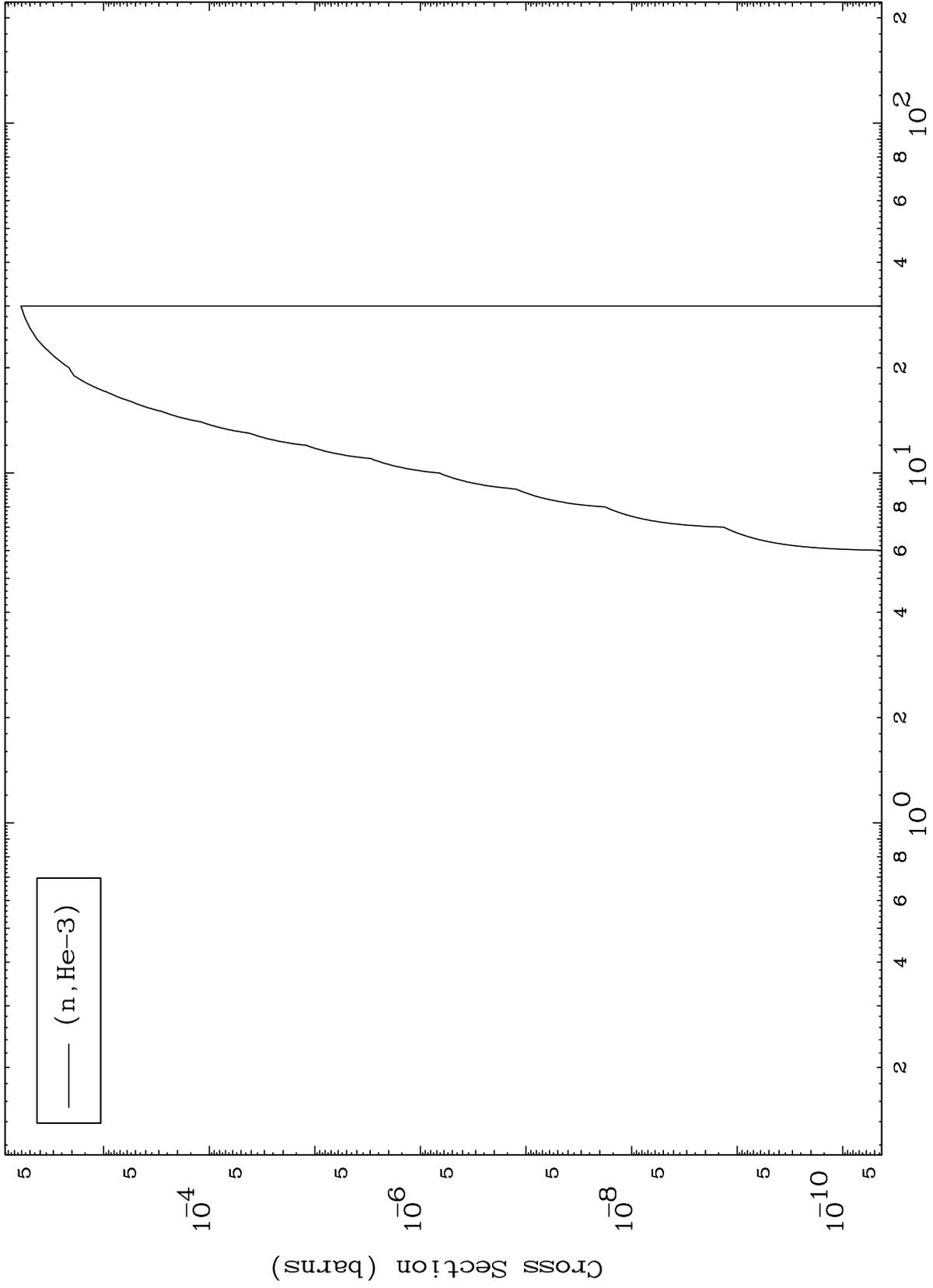


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(d,He3) Levels

49-In-105

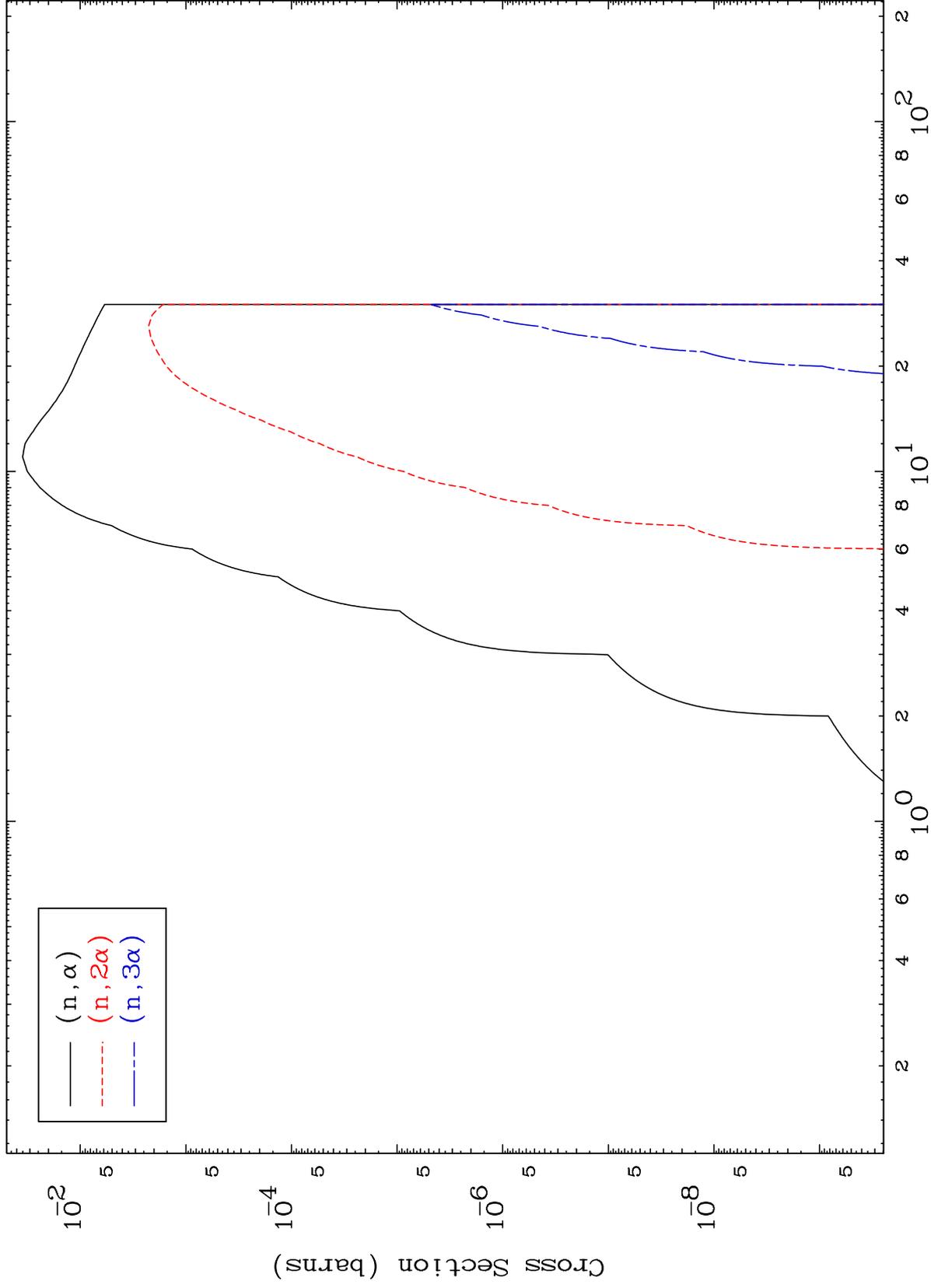
0 Kelvin Cross Sections



MAT 4901

49-In-105

(d,  $\alpha$ ) Levels  
0 Kelvin Cross Sections



49-In-105

Incident Energy (MeV)

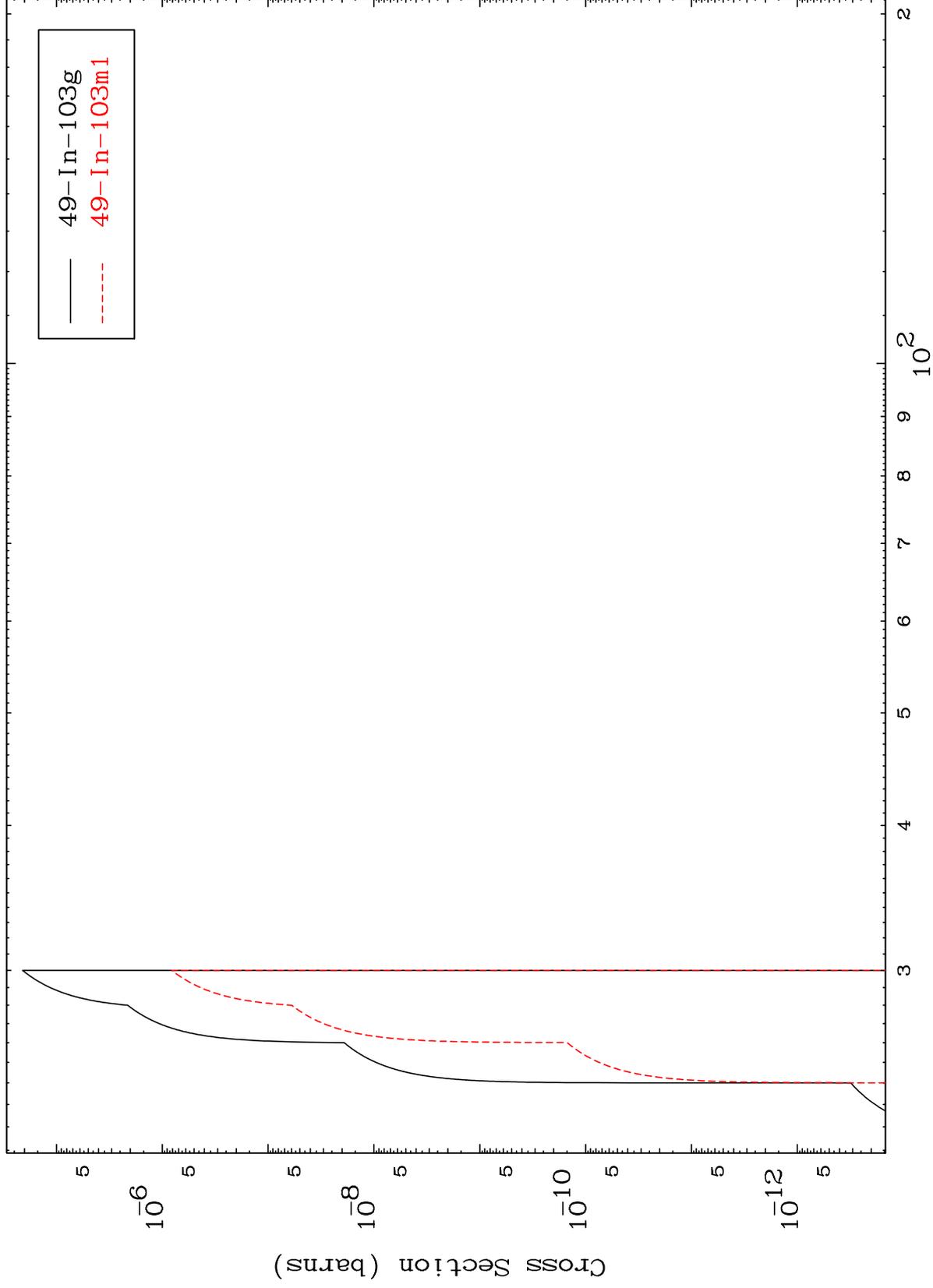
12

MAT 4901

(n,2n) d

49-In-105

Radionuclide Production Cross Section



13

Incident Energy (MeV)

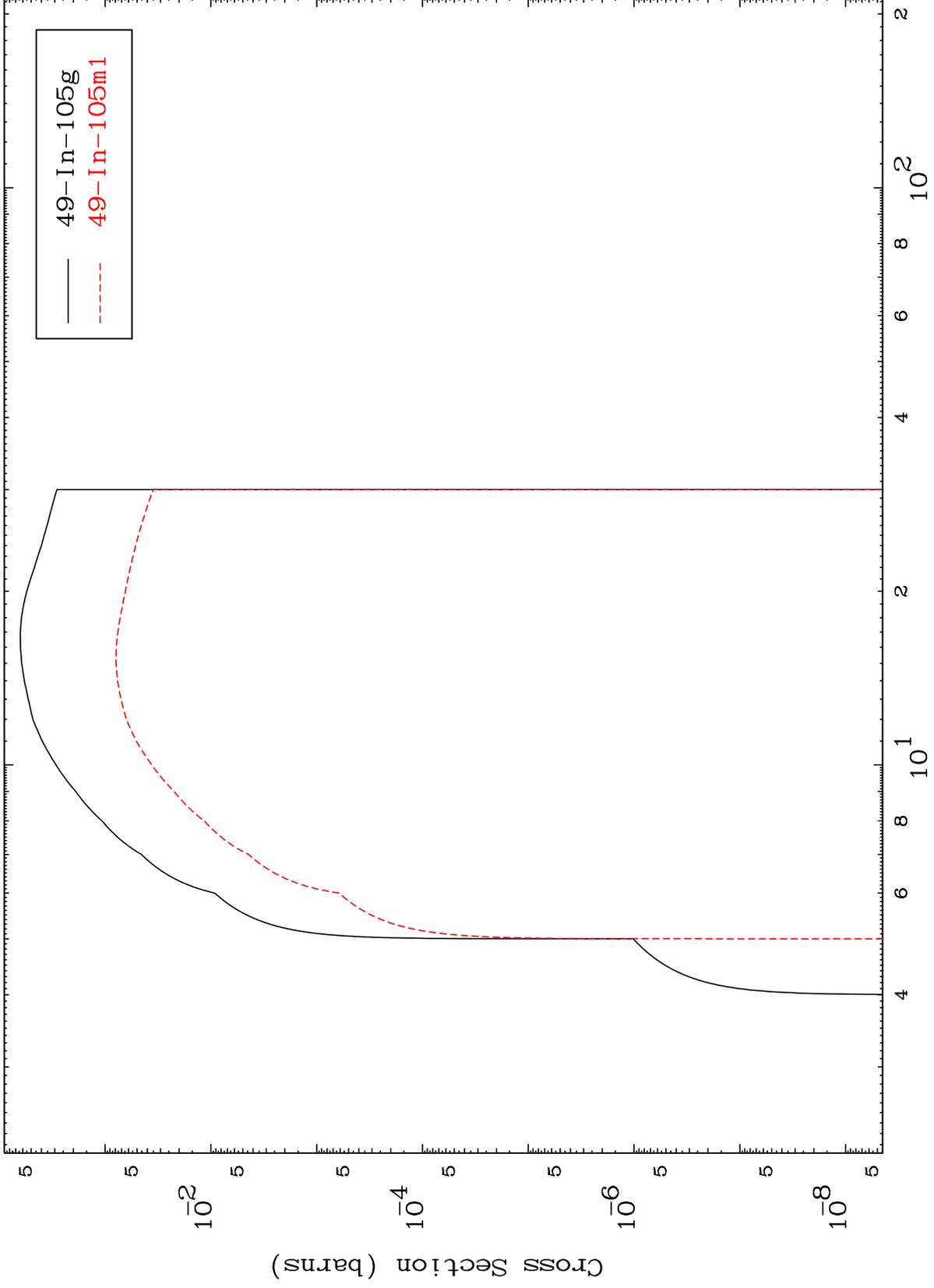
49-In-105

MAT 4901

49-In-105

(n,n') p

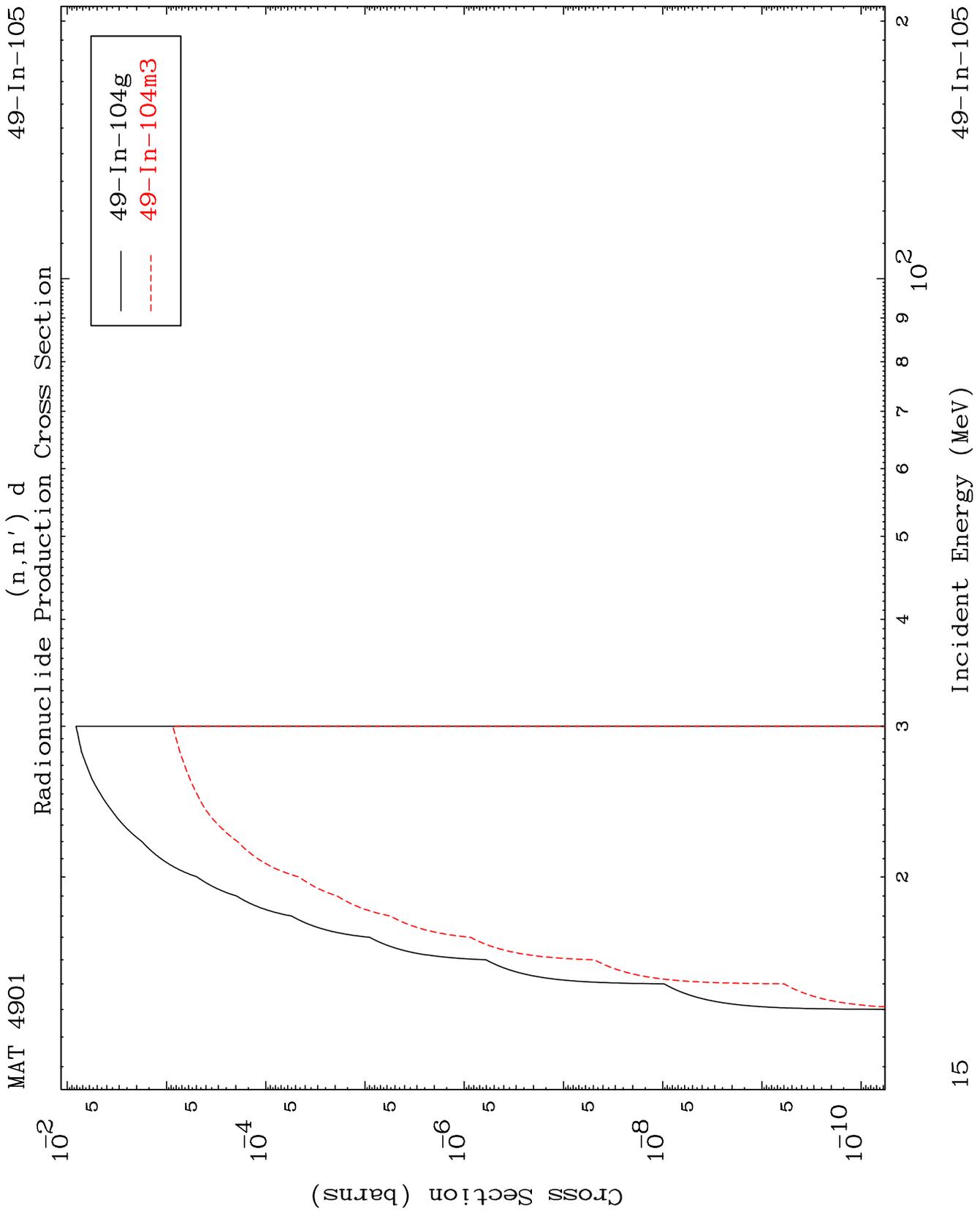
Radionuclide Production Cross Section



14

Incident Energy (MeV)

49-In-105

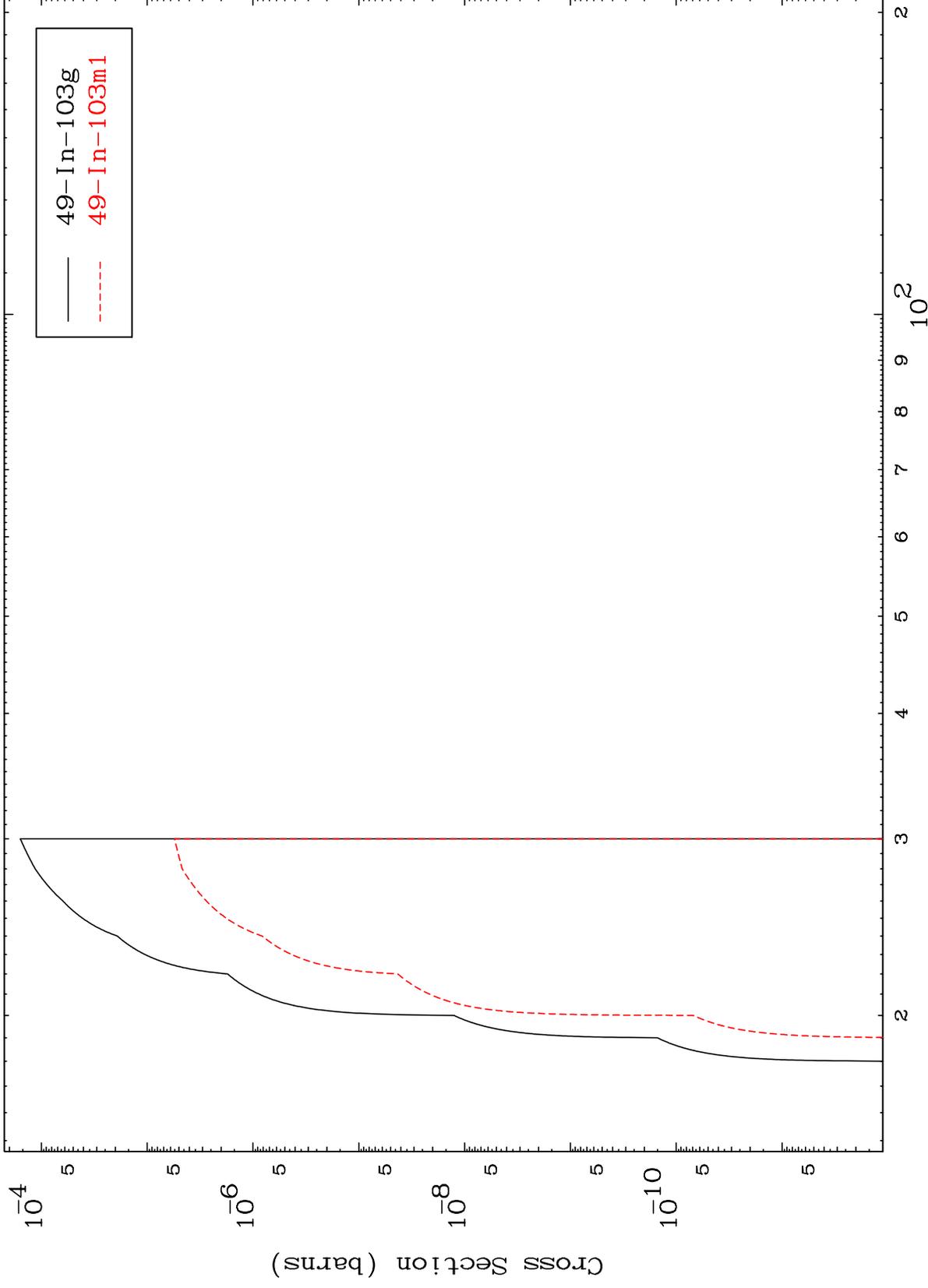


MAT 4901

(n,n') t

49-In-105

Radionuclide Production Cross Section



16

Incident Energy (MeV)

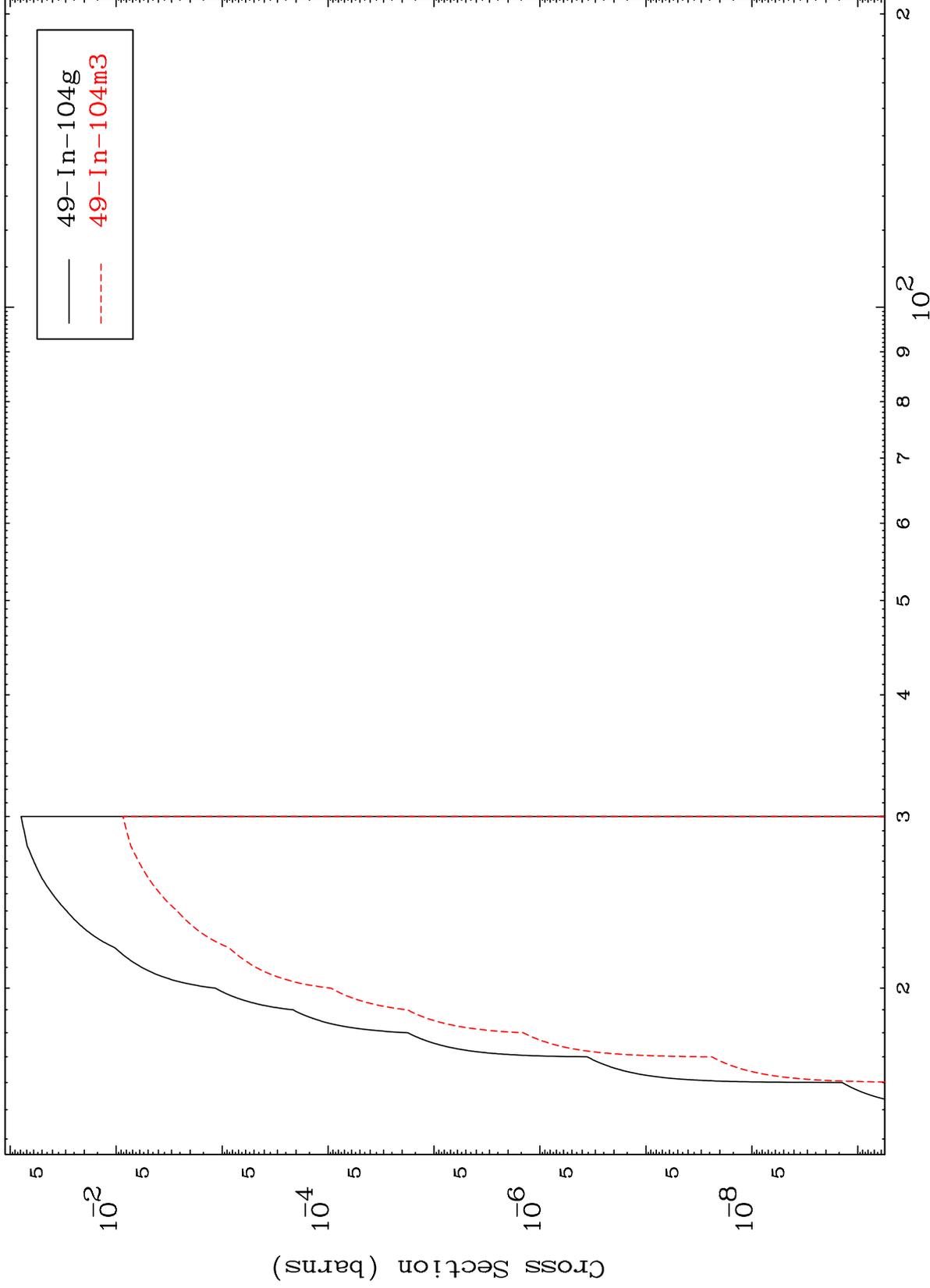
49-In-105

MAT 4901

(n,2n) p

49-In-105

Radionuclide Production Cross Section



17

Incident Energy (MeV)

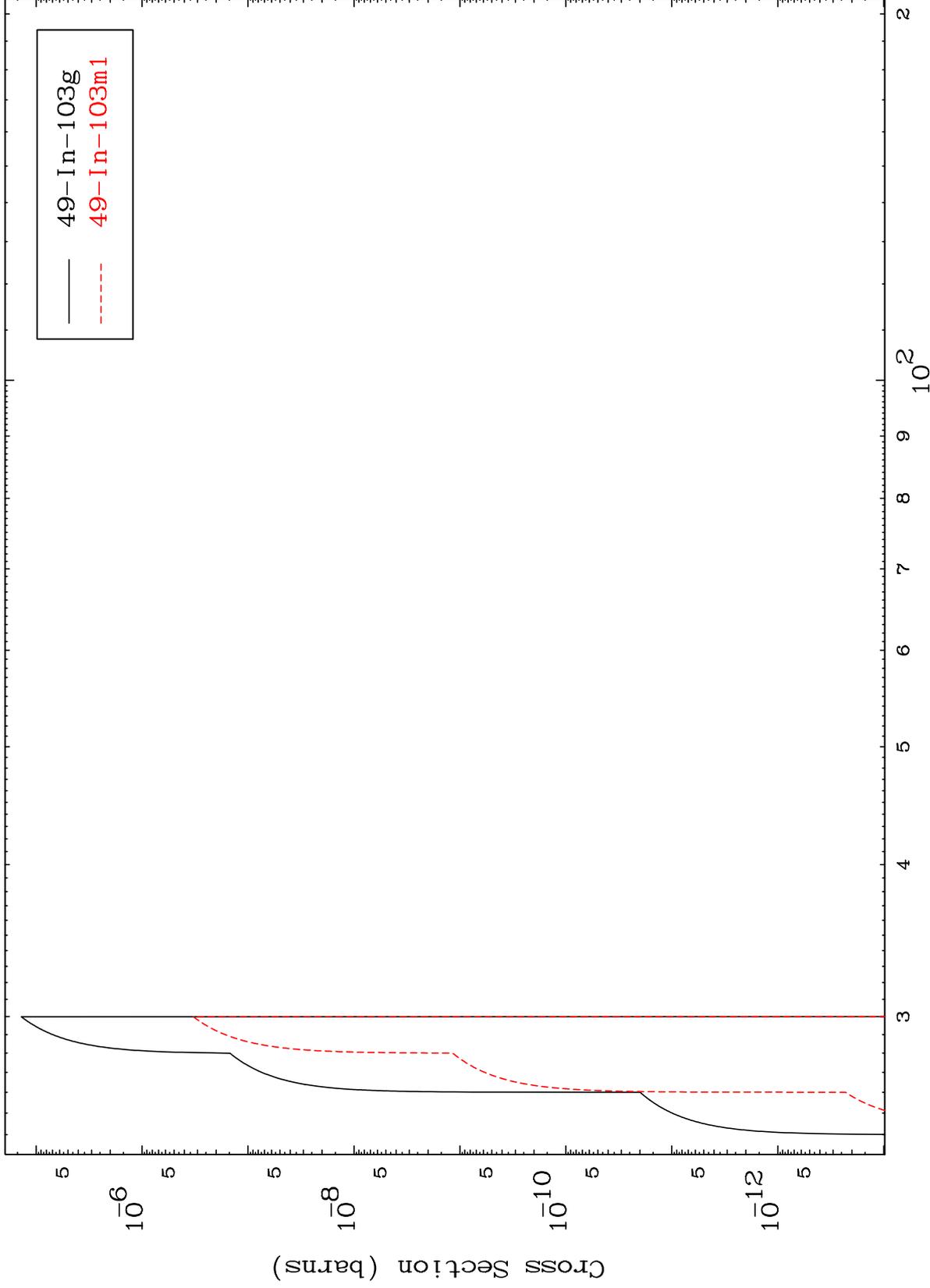
49-In-105

MAT 4901

(n,3n) p

49-In-105

Radionuclide Production Cross Section



18

Incident Energy (MeV)

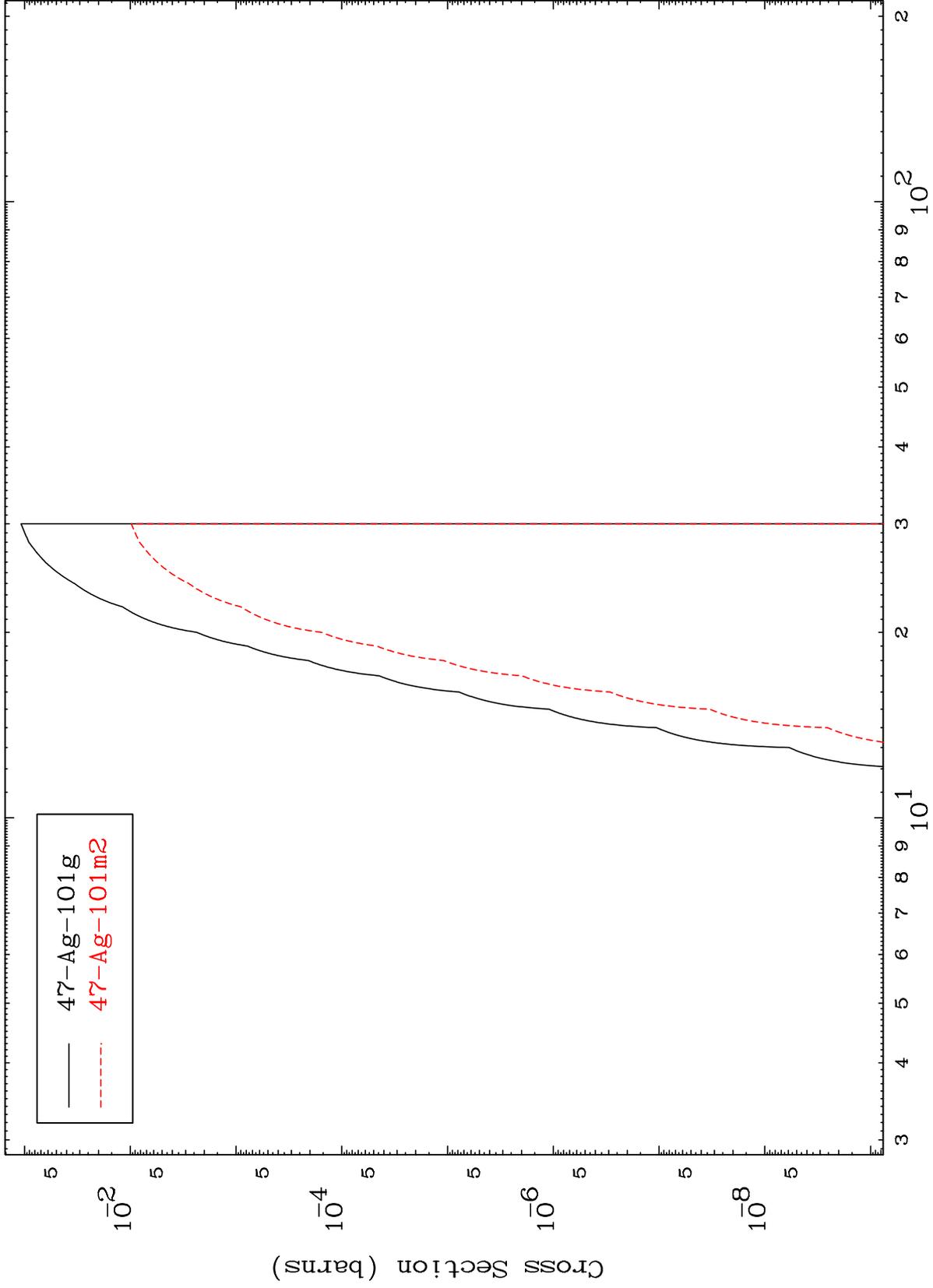
49-In-105

MAT 4901

(n,n') p  $\alpha$

49-In-105

Radionuclide Production Cross Section



19

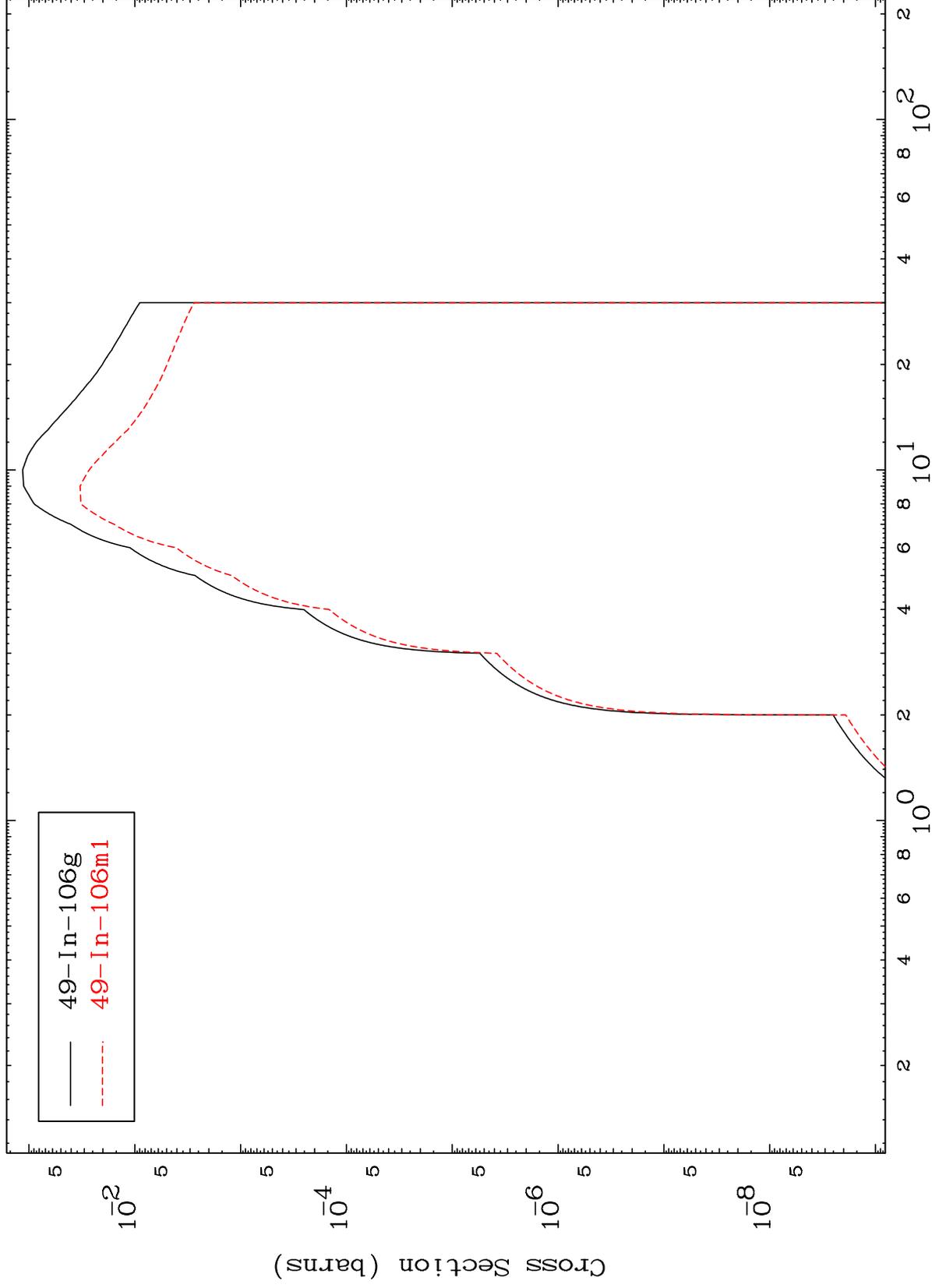
Incident Energy (MeV)

49-In-105

MAT 4901

49-In-105

(n,p)  
Radionuclide Production Cross Section



49-In-106g  
49-In-106m1

20

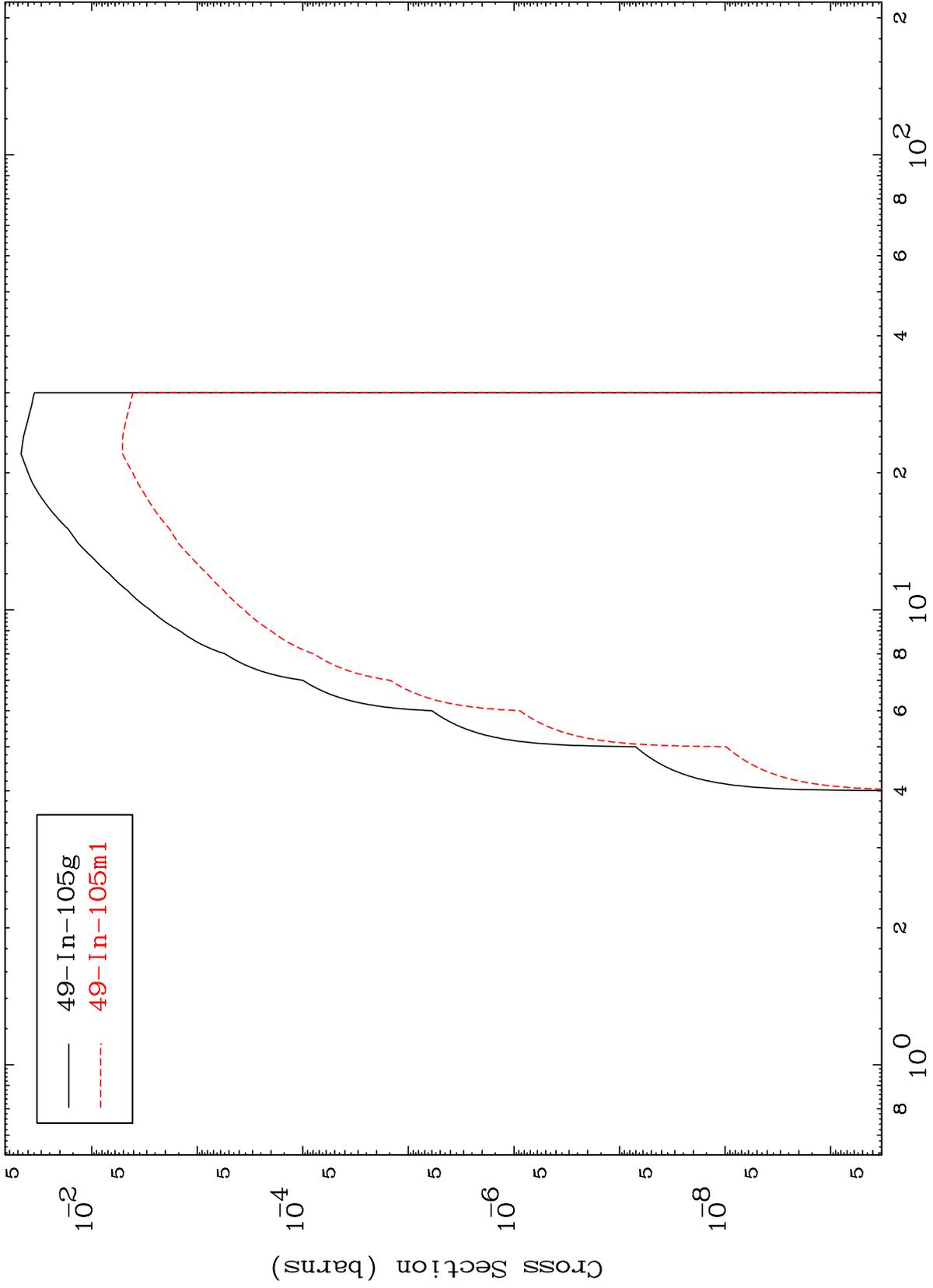
Incident Energy (MeV)

49-In-105

MAT 4901

49-In-105

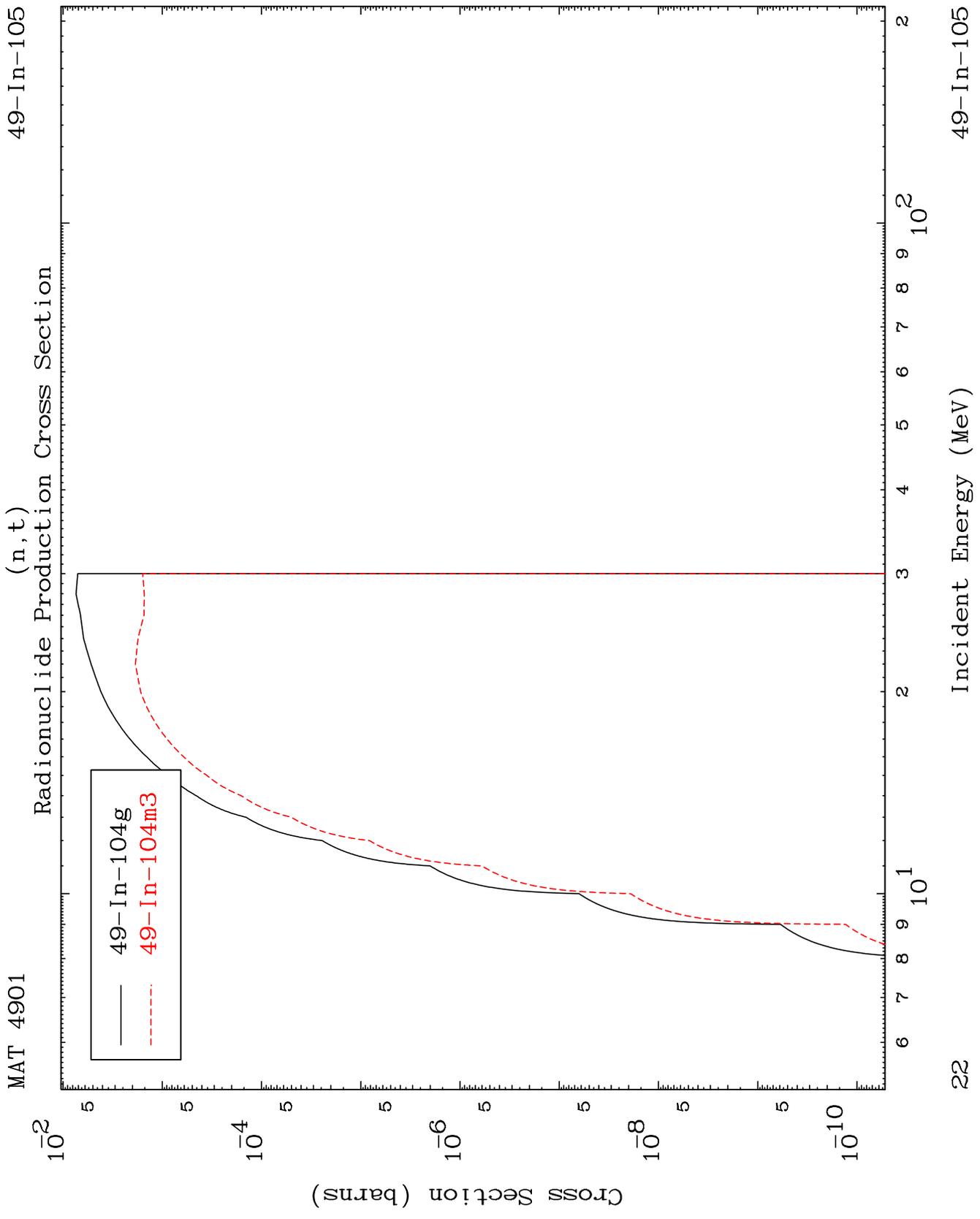
(n,d)  
Radionuclide Production Cross Section



21

49-In-105

Incident Energy (MeV)

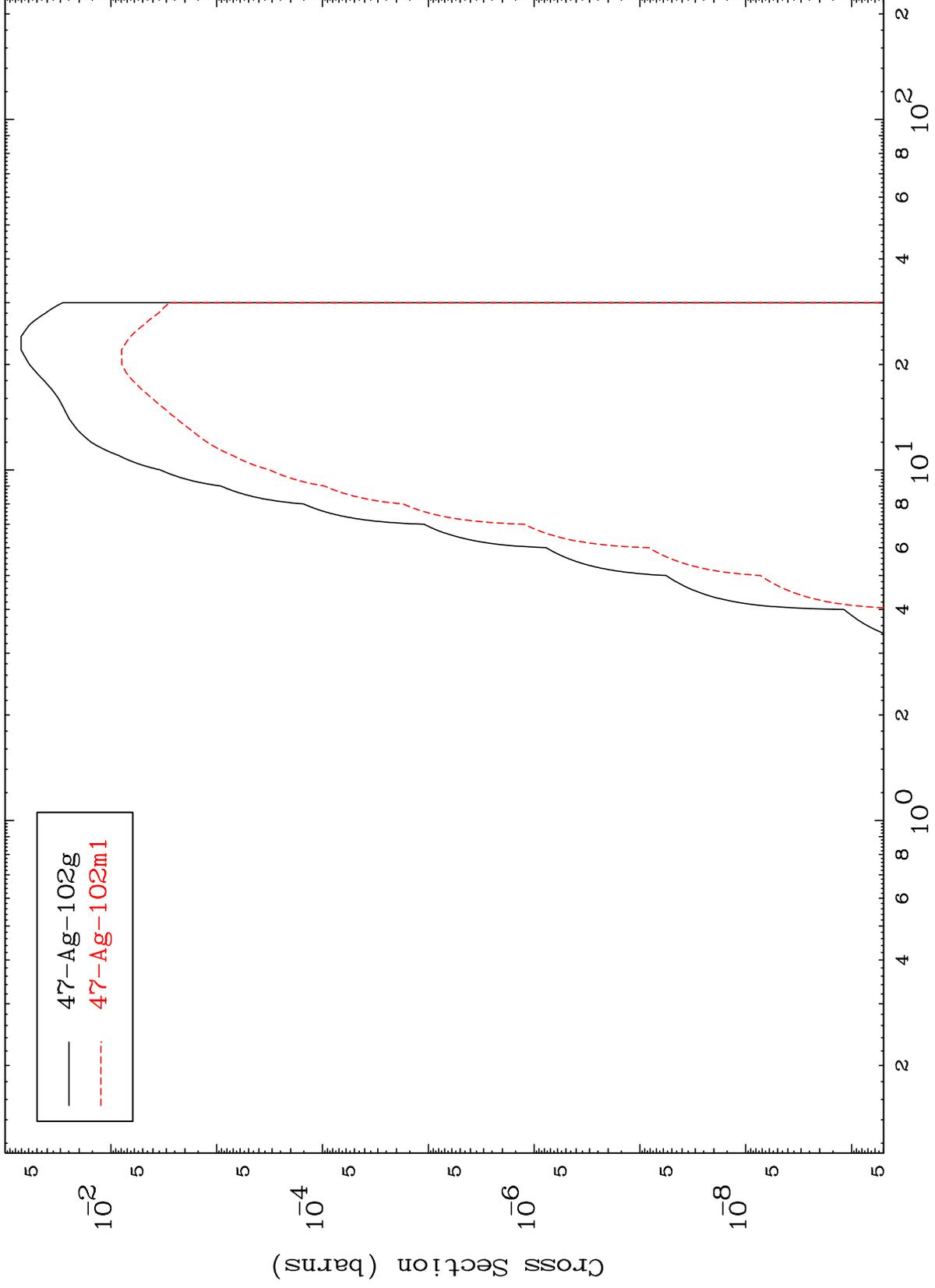


MAT 4901

(n,p)  $\alpha$

49-In-105

Radionuclide Production Cross Section

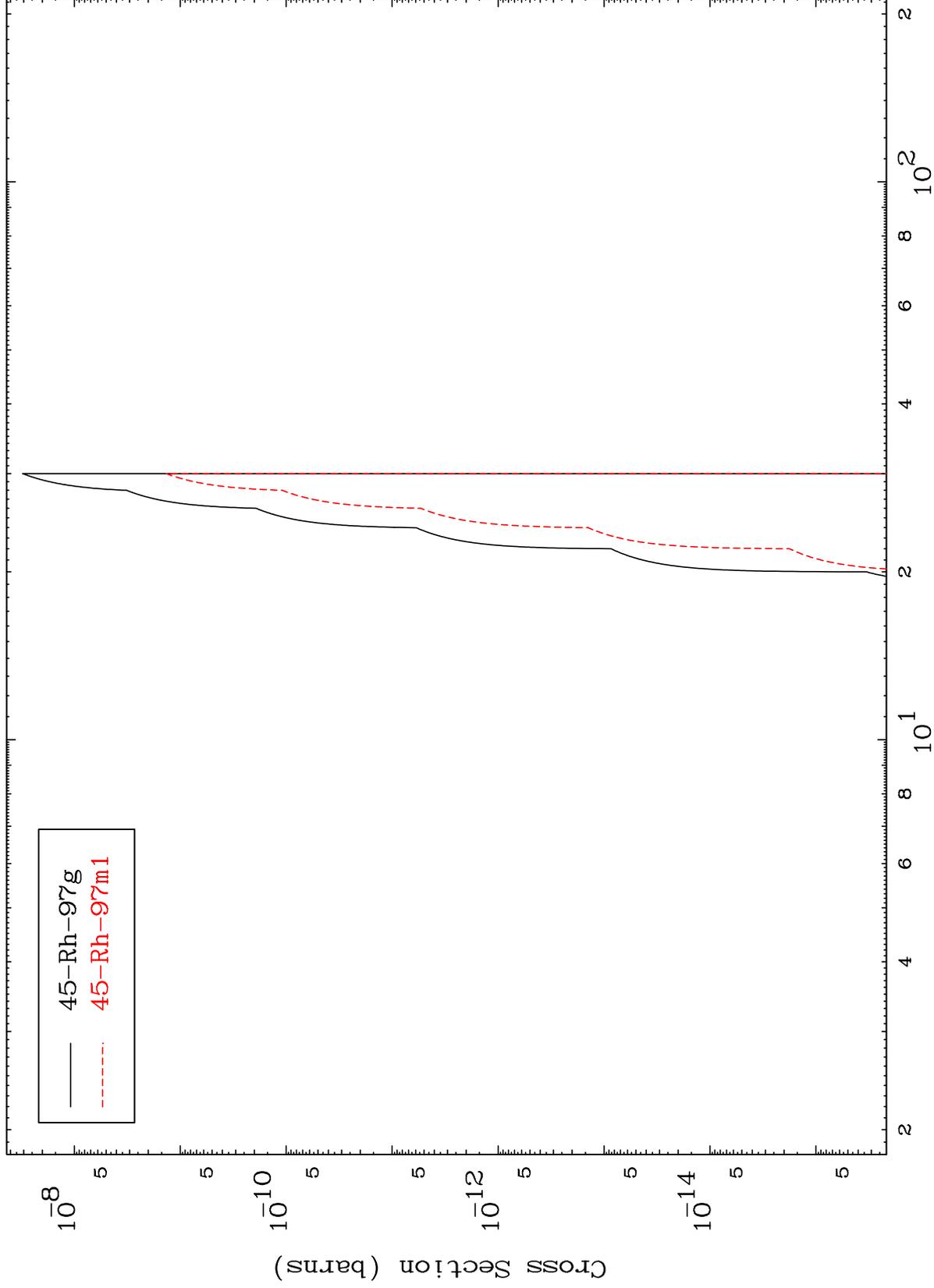


MAT 4901

(n,d) 2 $\alpha$

49-In-105

Radionuclide Production Cross Section



24

Incident Energy (MeV)

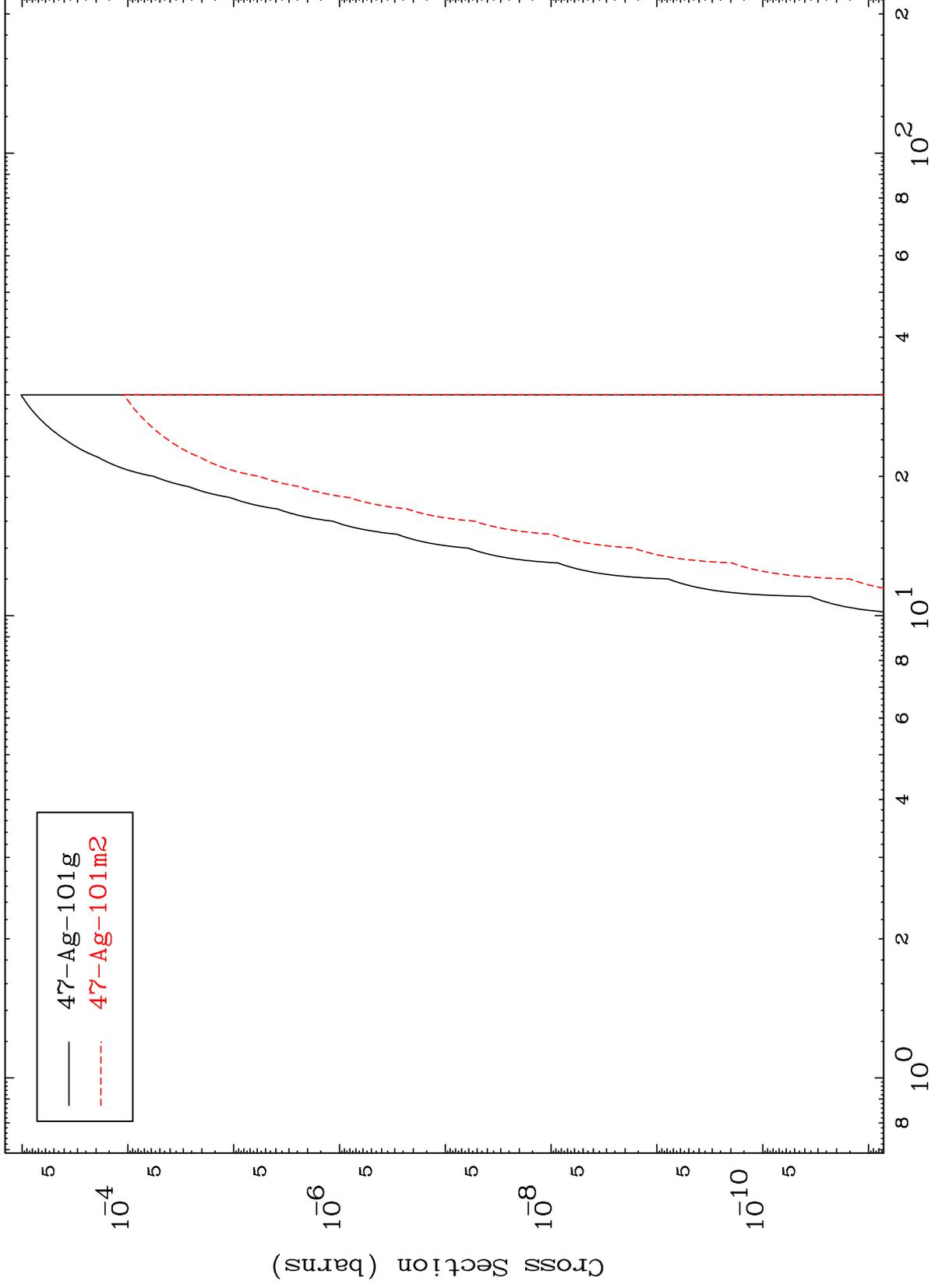
49-In-105

MAT 4901

(n,d)  $\alpha$

49-In-105

Radionuclide Production Cross Section



25

Incident Energy (MeV)

49-In-105