

Program EVALPLOT  
(Version 2021-1)

by

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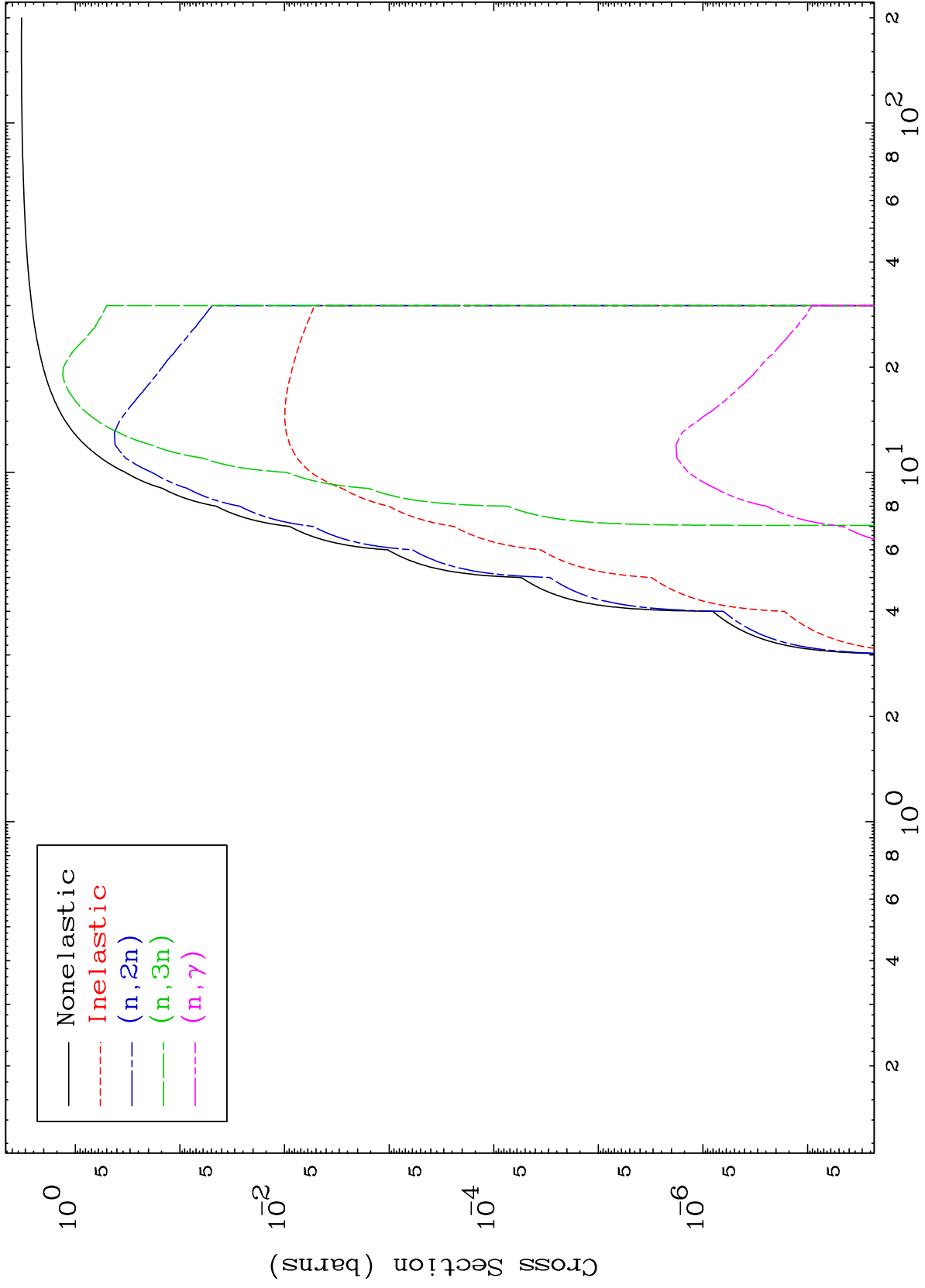
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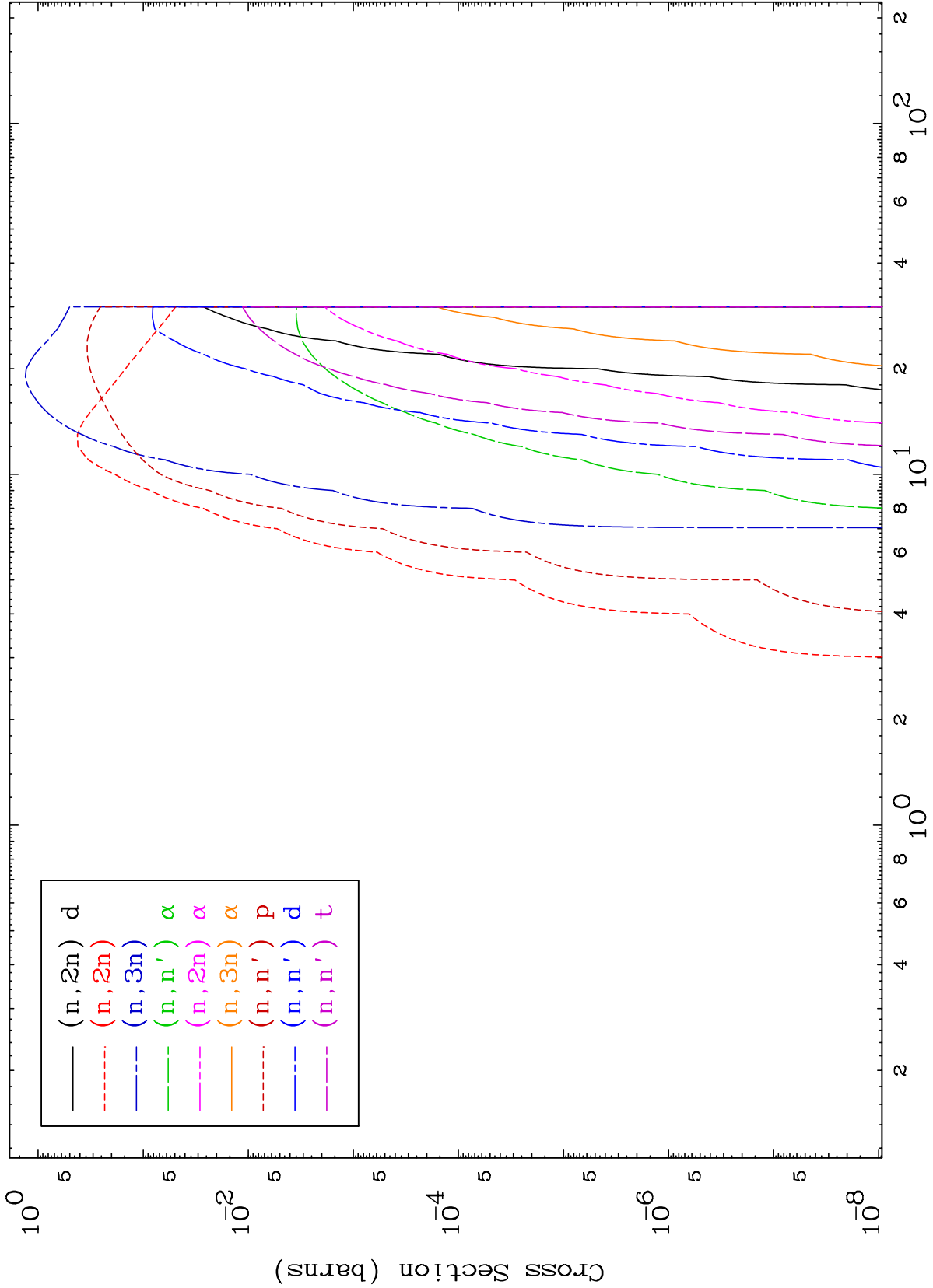
Press Mouse Button to Start

MAT 8138

Deuteron Major  
0 Kelvin Cross Sections

81-Tl-207m

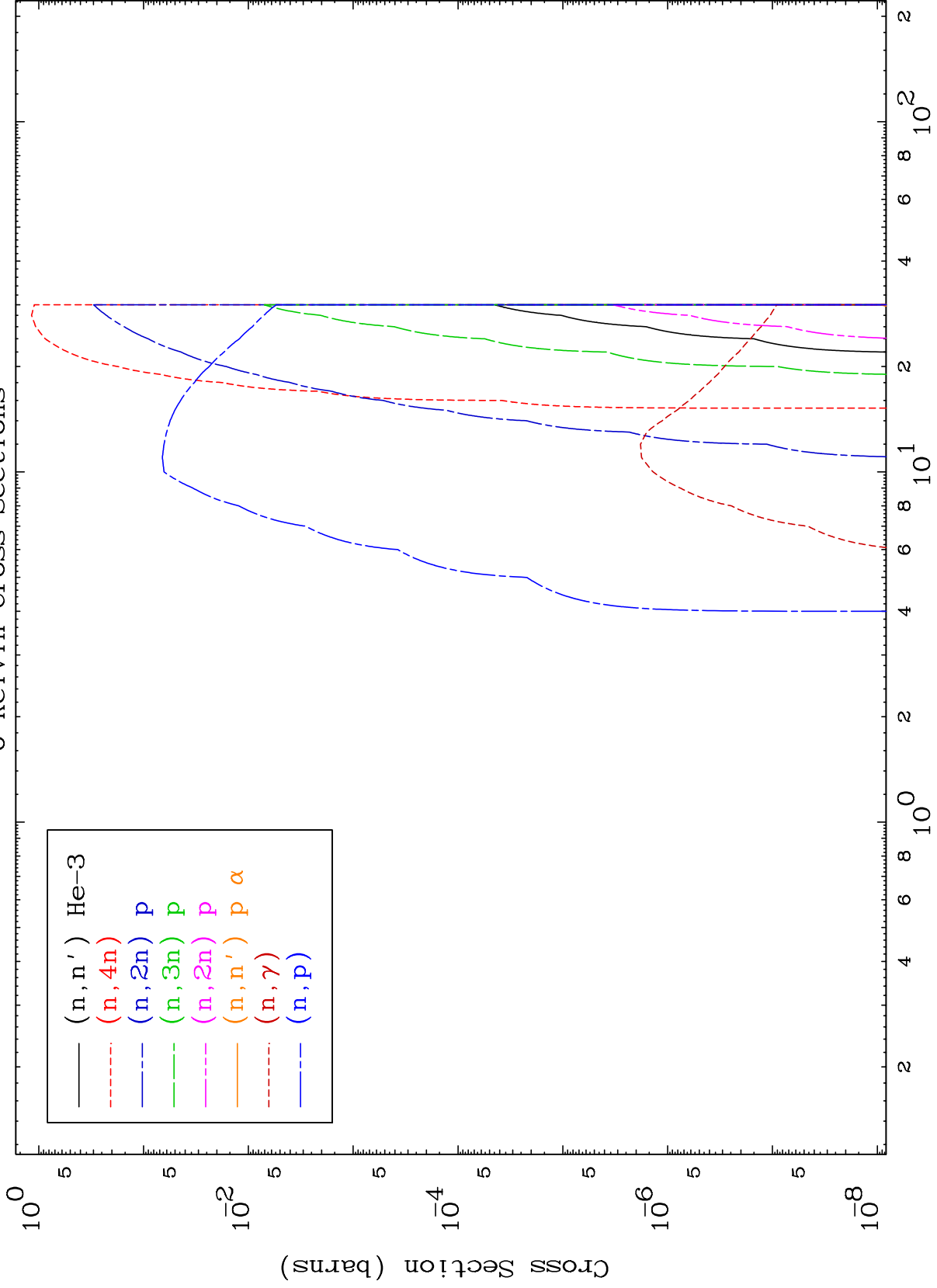




MAT 8138

Deuteron Neutron Absorption  
0 Kelvin Cross Sections

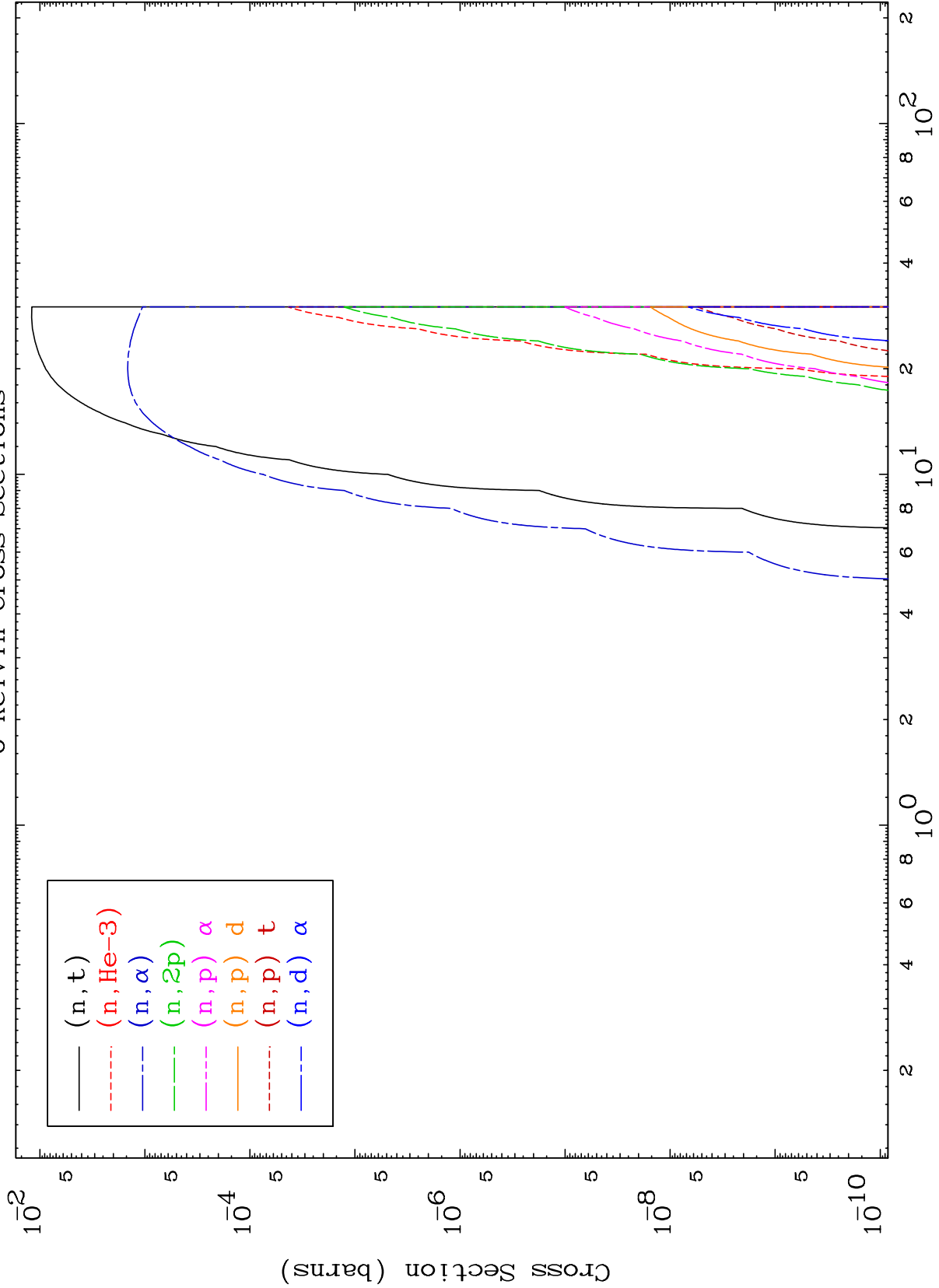
81-Tl-207m

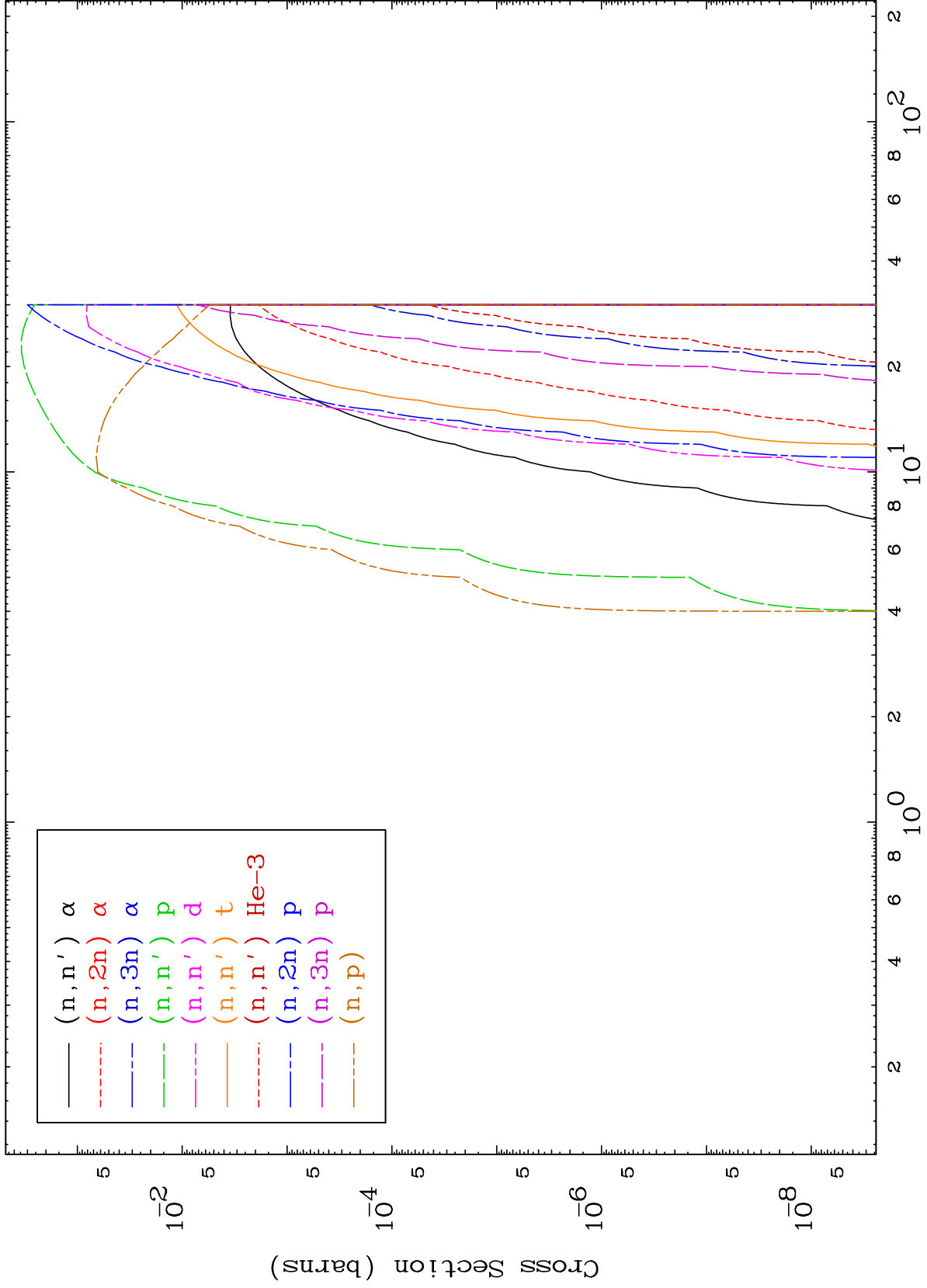


MAT 8138

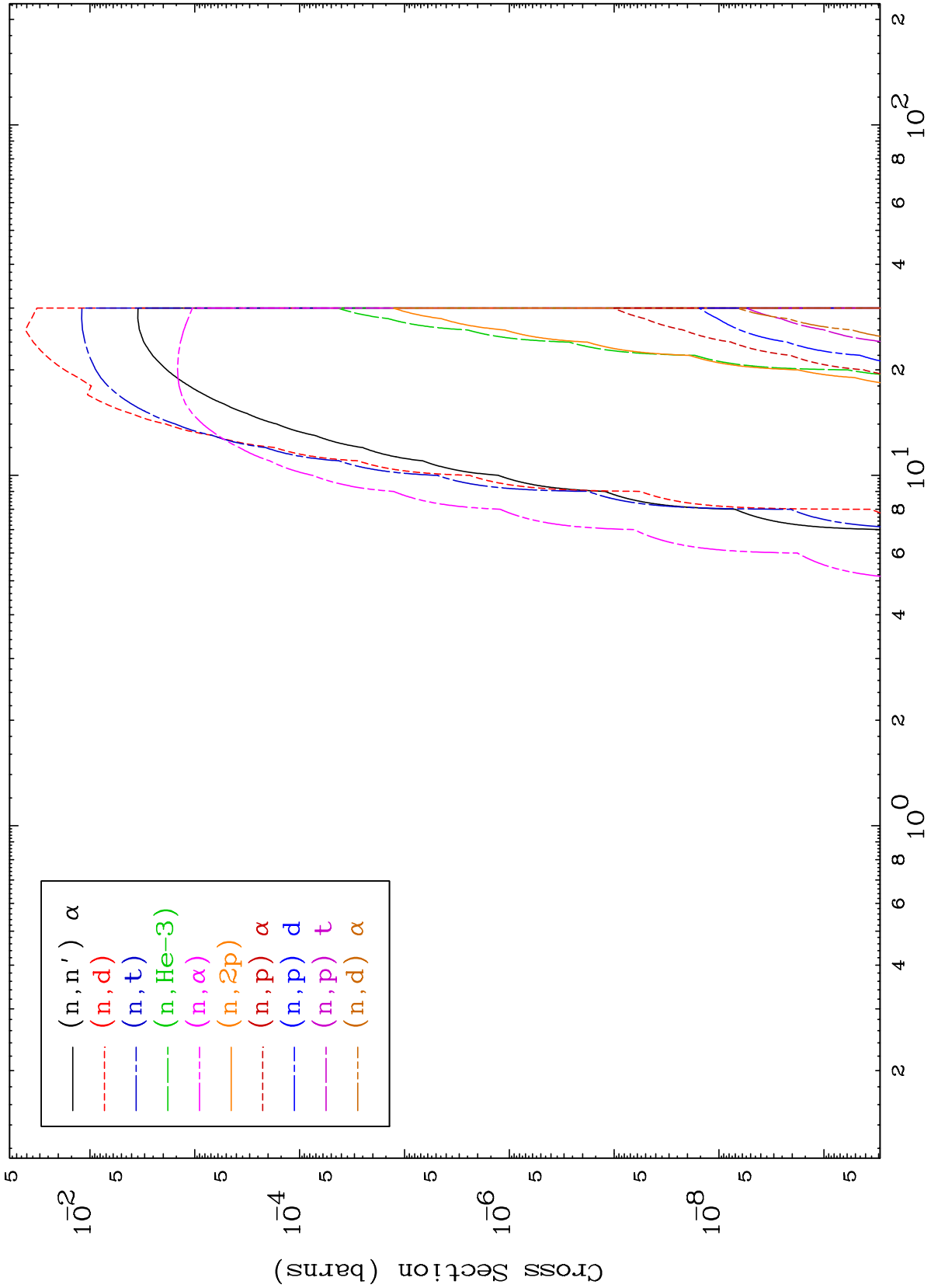
Deuteron Neutron Absorption  
0 Kelvin Cross Sections

81-Tl-207m





—	$(n, n')$	$\alpha$
- - -	$(n, 2n)$	$\alpha$
- · - ·	$(n, 3n)$	$\alpha$
—	$(n, n')$	p
- - -	$(n, n')$	d
—	$(n, n')$	t
- · - ·	$(n, n')$	He-3
- · - ·	$(n, 2n)$	p
- · - ·	$(n, 3n)$	p
- - -	$(n, p)$	

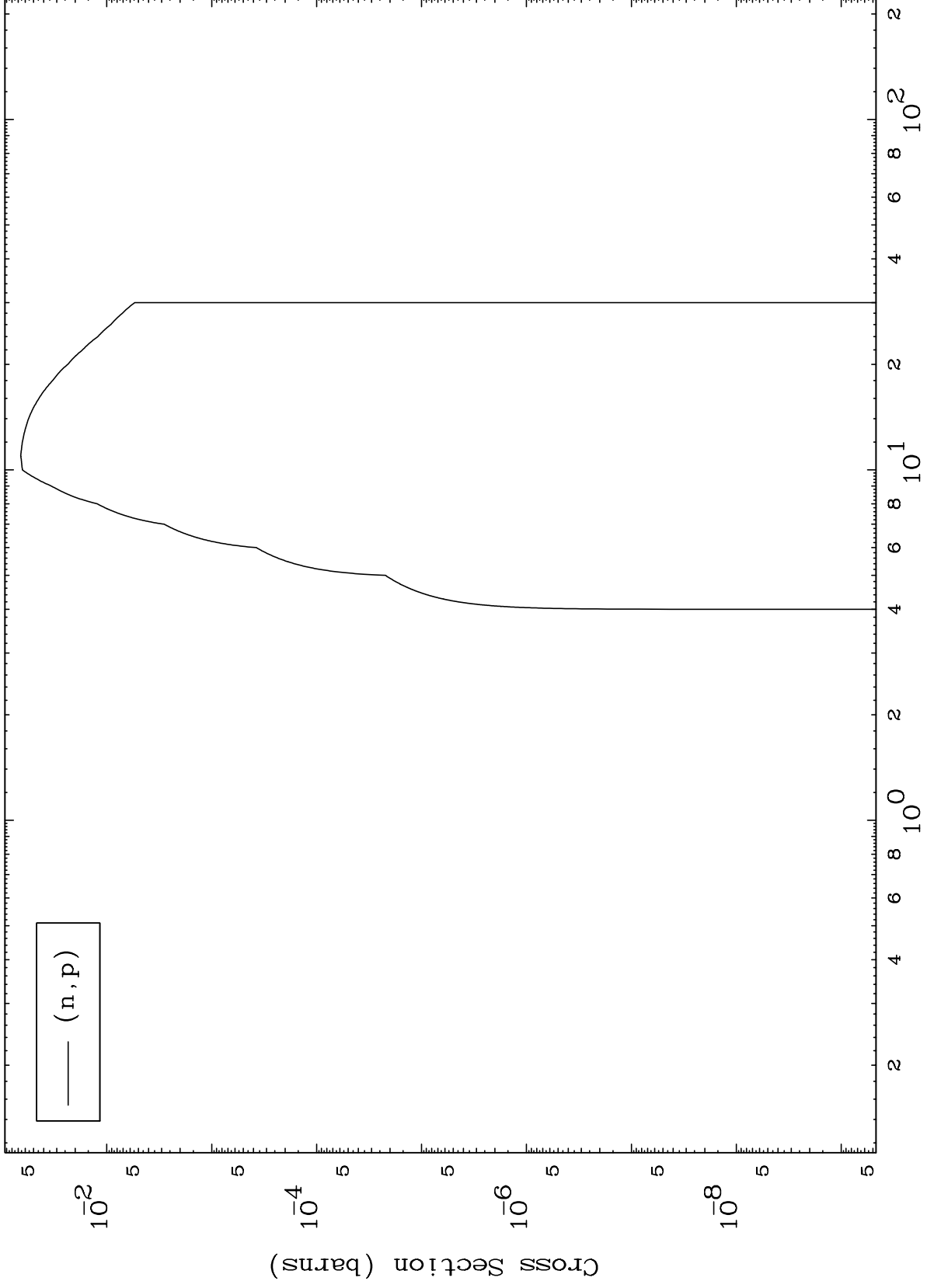


MAT 8138

(d,p) Levels

81-Tl-207m

0 Kelvin Cross Sections

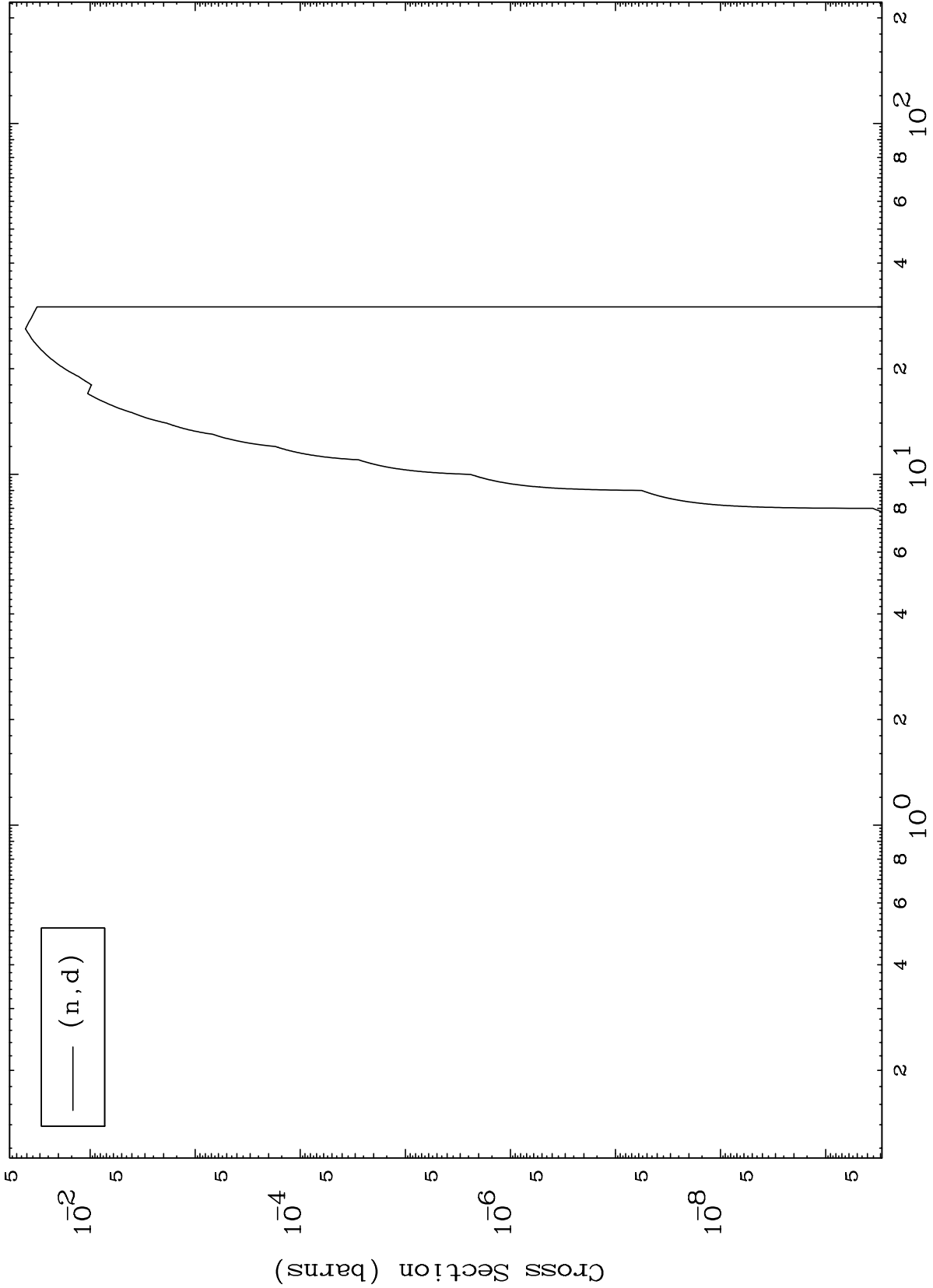


MAT 8138

(d,d) Levels

81-Tl-207m

0 Kelvin Cross Sections



(n,d)

Incident Energy (MeV)

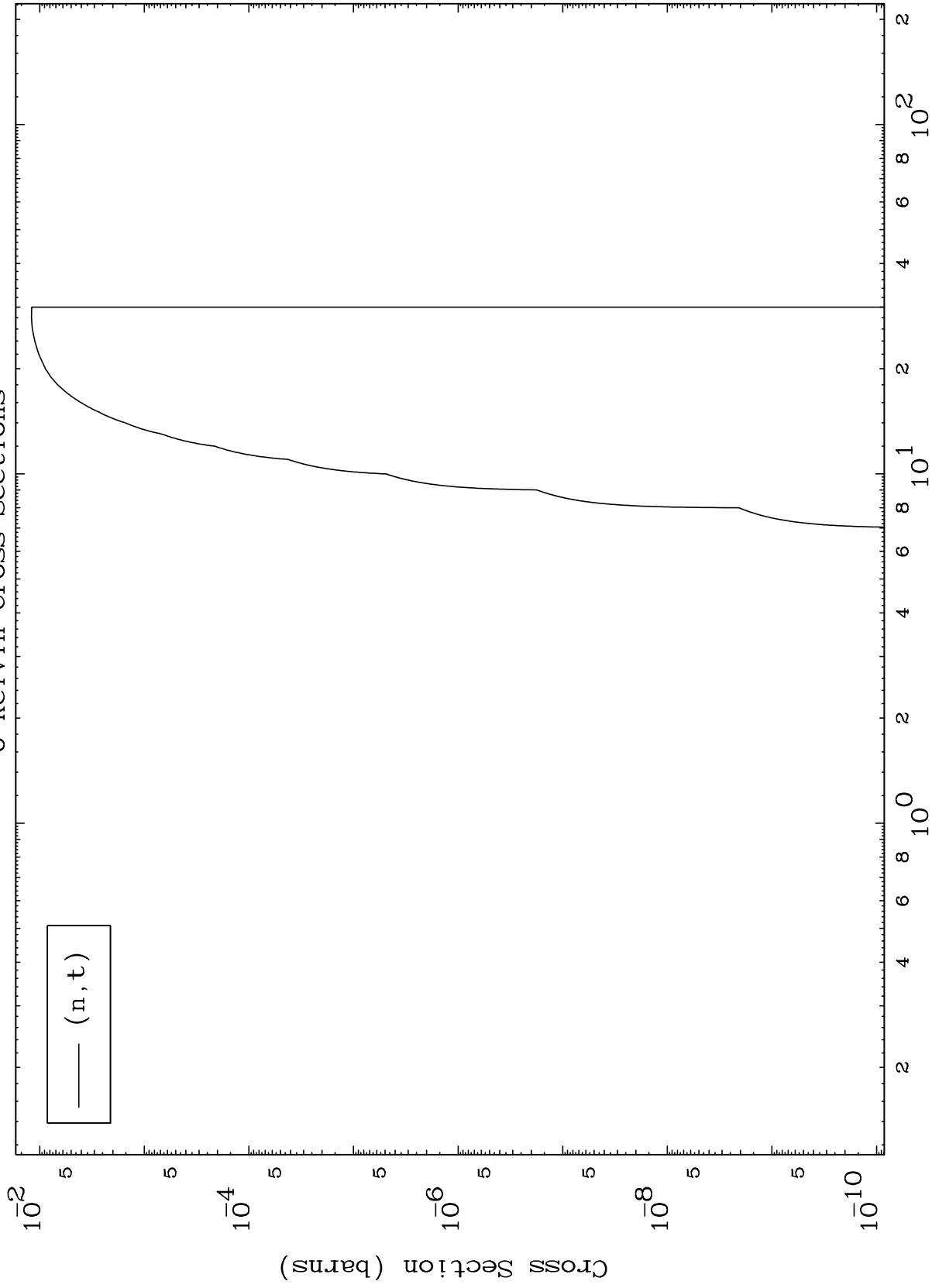
81-Tl-207m

MAT 8138

(d, t) Levels

81-Tl-207m

0 Kelvin Cross Sections

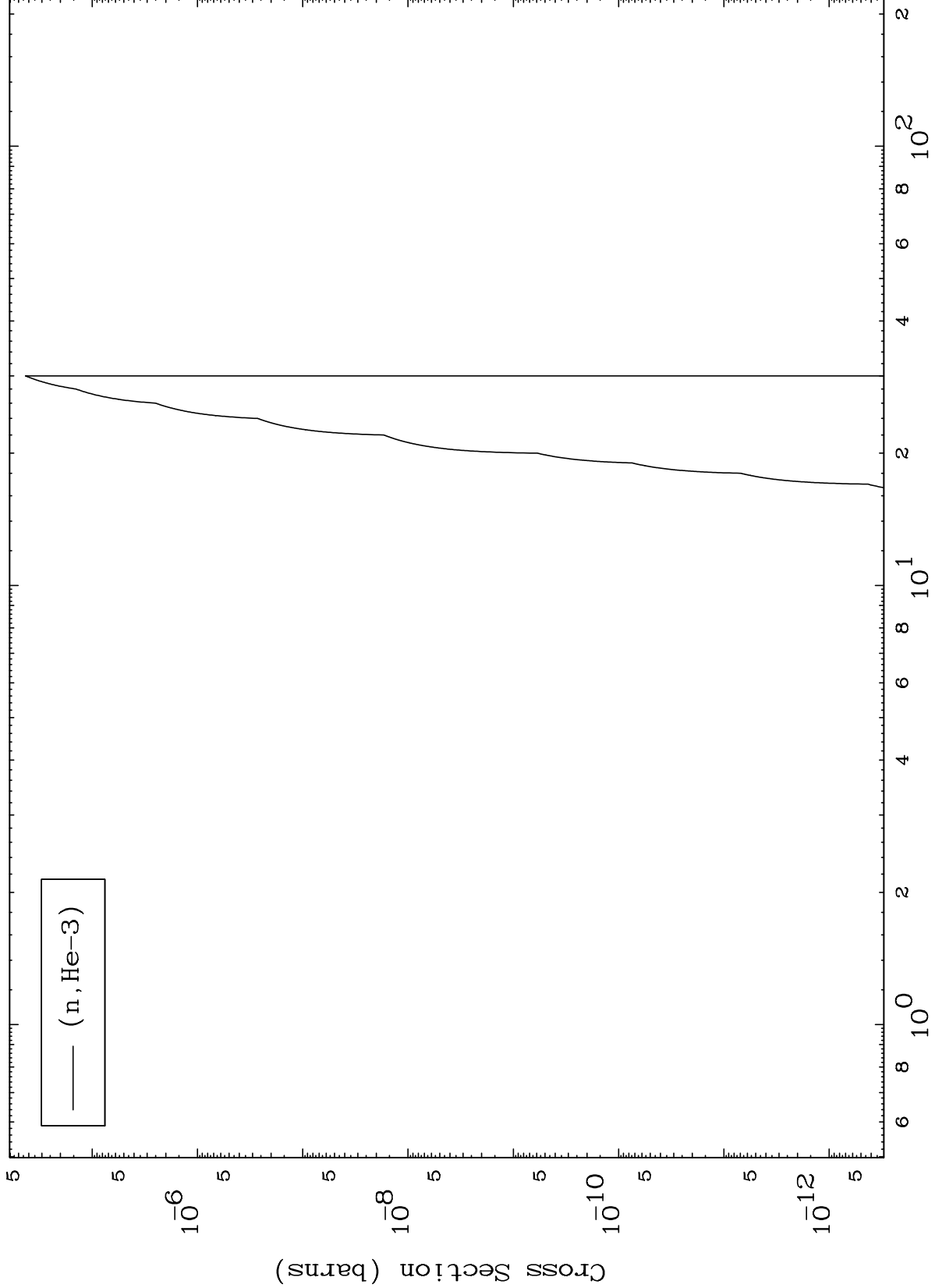


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(d,He3) Levels

81-Tl-207m

0 Kelvin Cross Sections



10

Incident Energy (MeV)

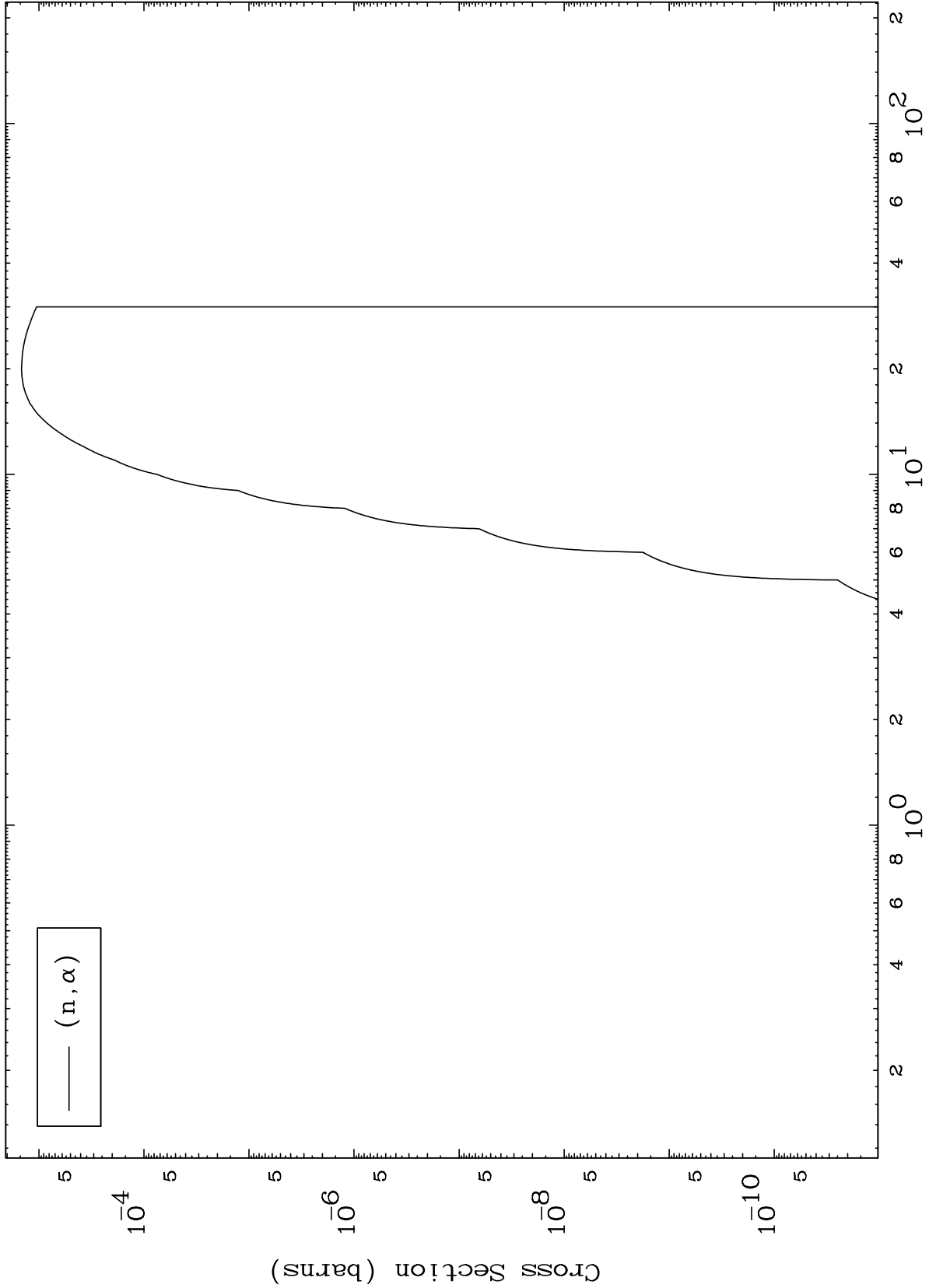
81-Tl-207m

MAT 8138

(d,  $\alpha$ ) Levels

81-Tl-207m

0 Kelvin Cross Sections

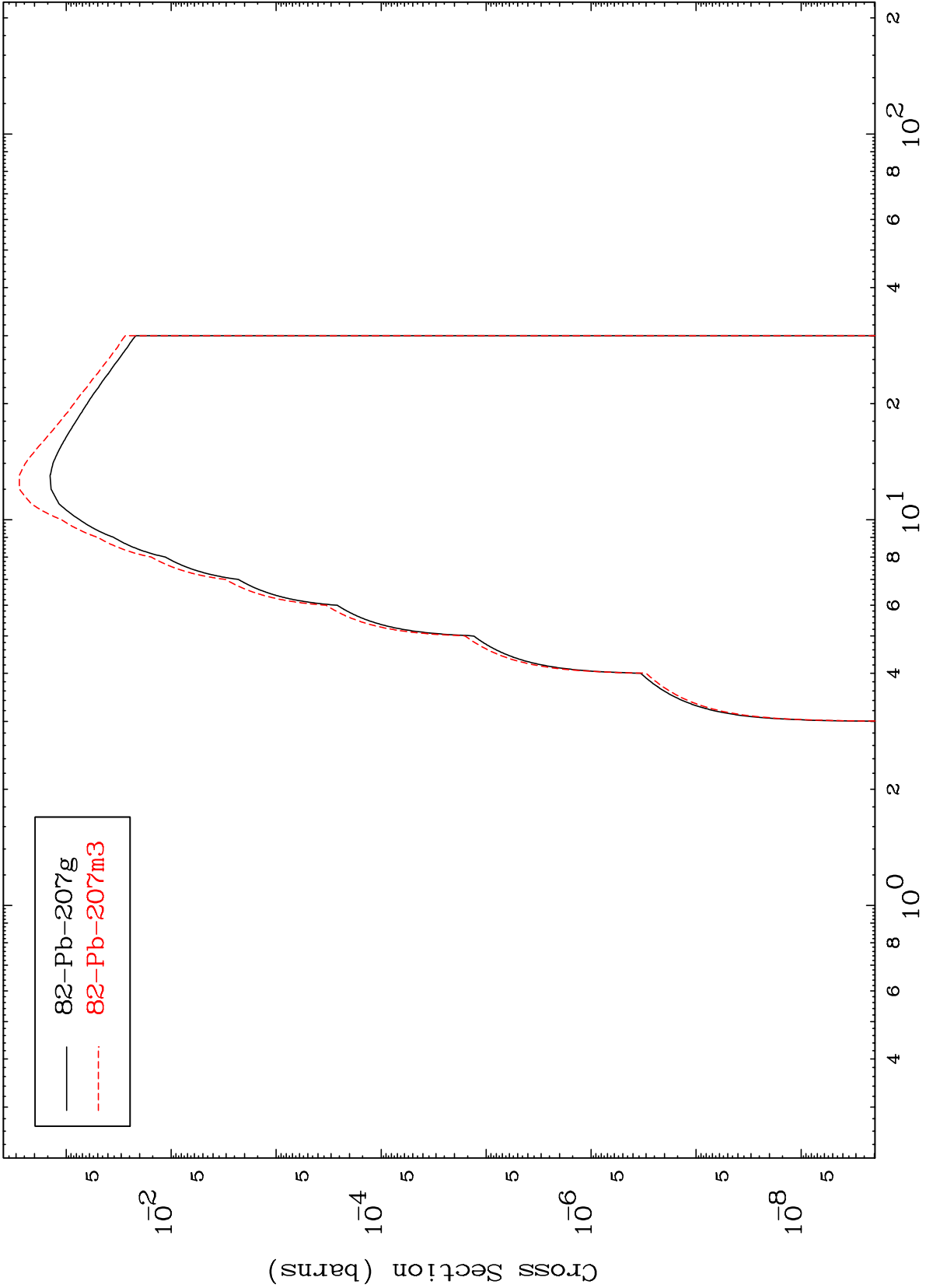


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(n,2n)

81-Tl-207m

Radionuclide Production Cross Section

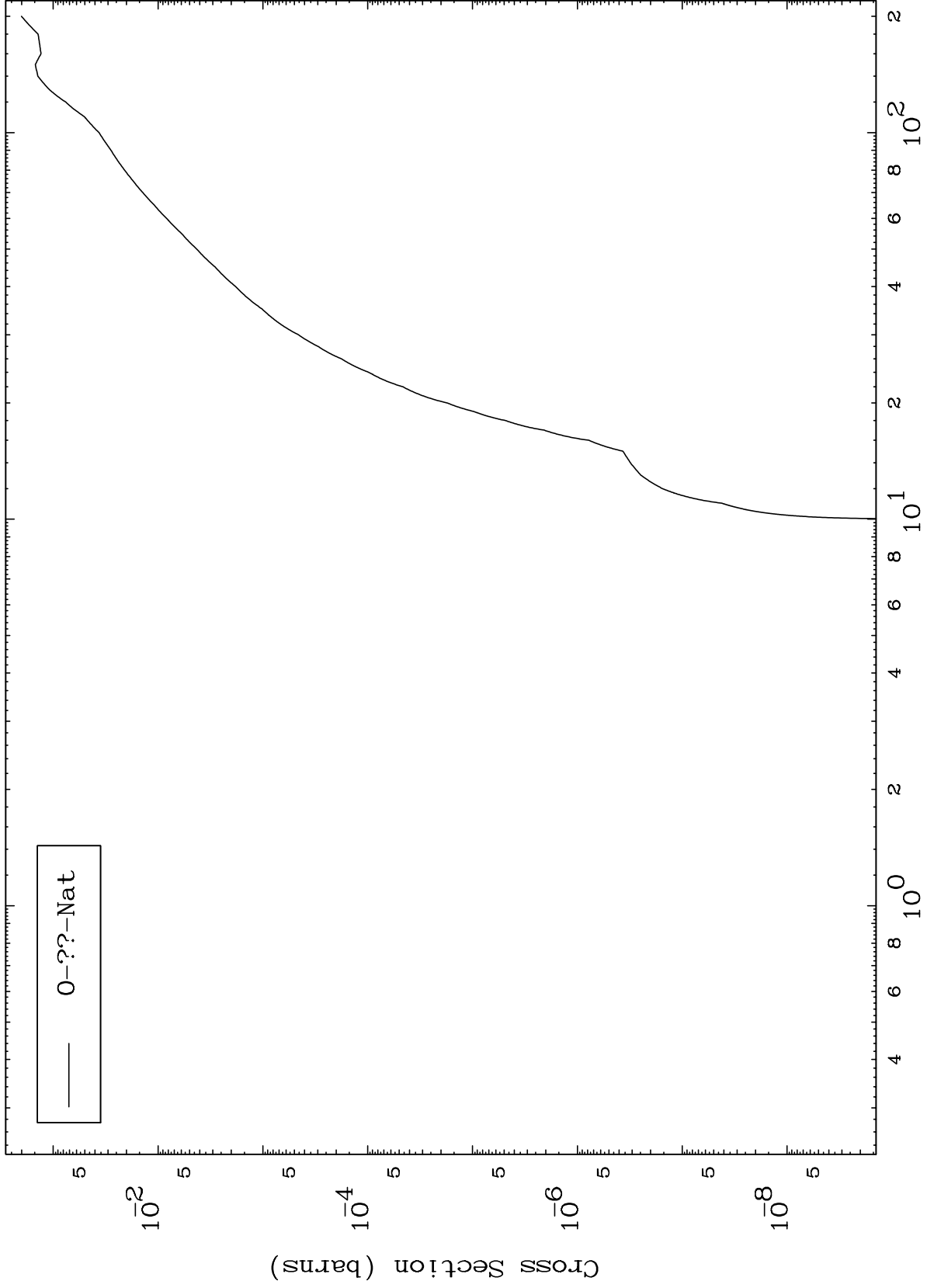


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Fission

81-Tl-207m

Radionuclide Production Cross Section

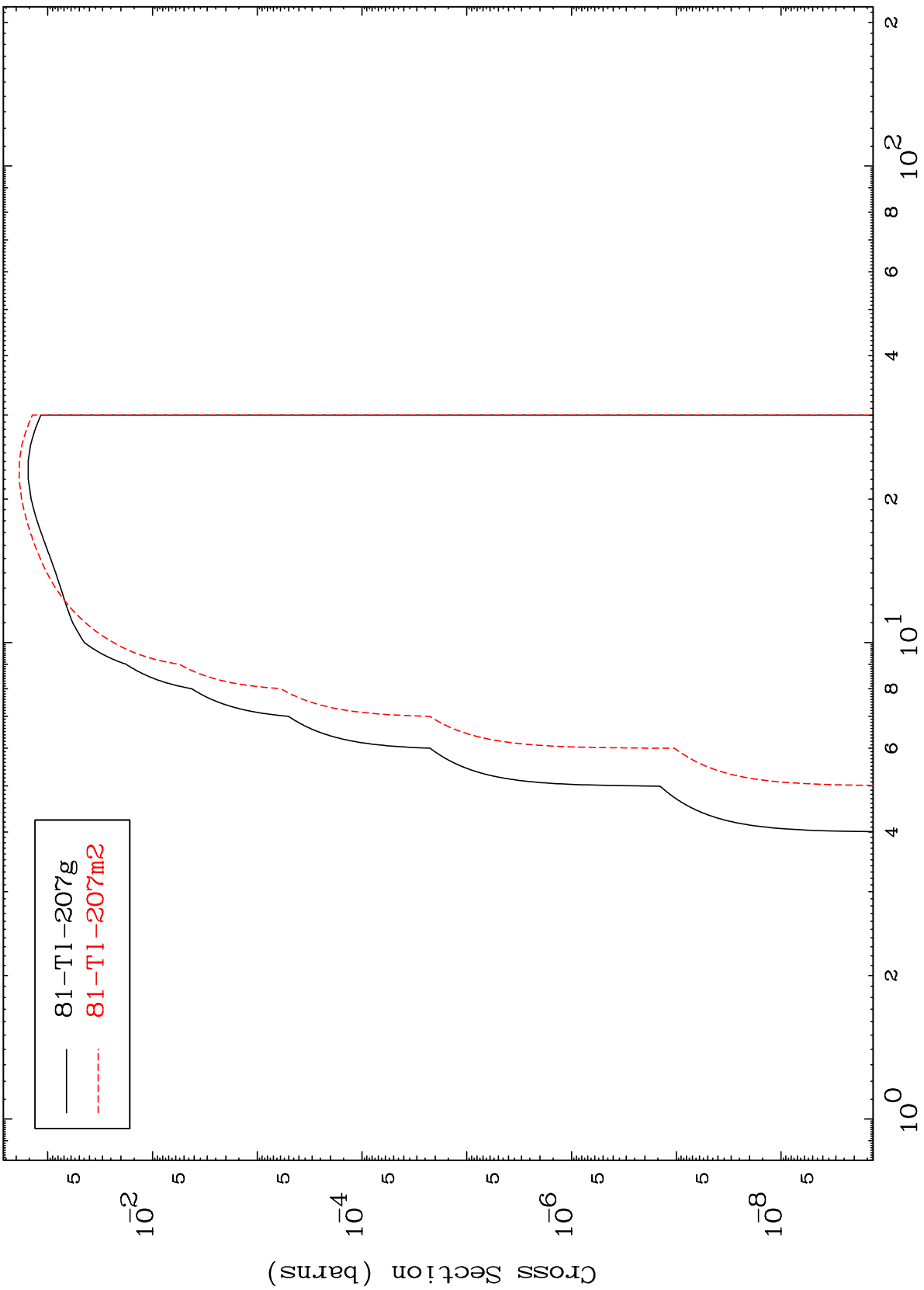


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(n,n') p

81-Tl-207m

Radionuclide Production Cross Section



14

Incident Energy (MeV)

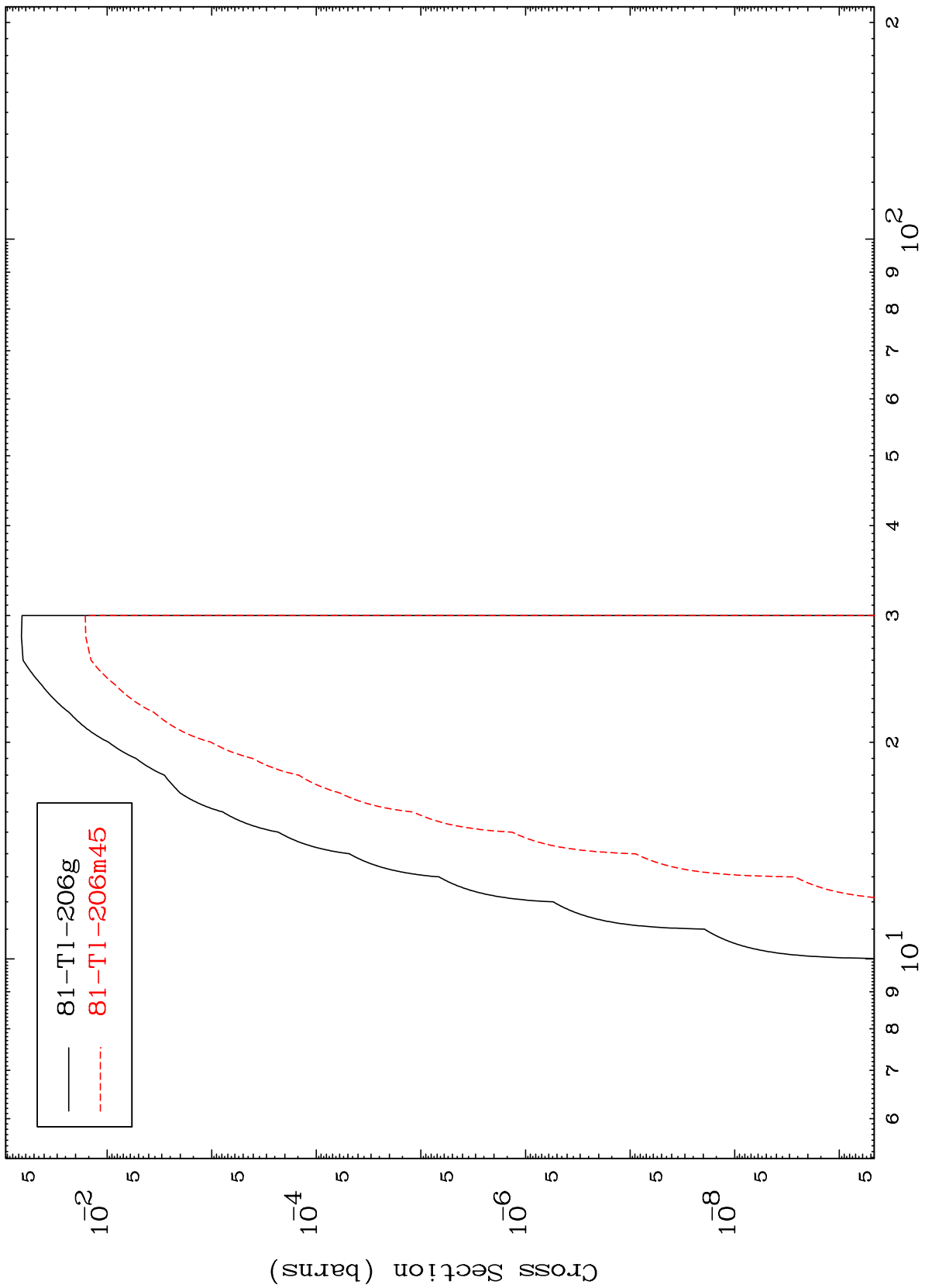
81-Tl-207m

MAT 8138

(n,n') d

81-Tl-207m

Radionuclide Production Cross Section



15

Incident Energy (MeV)

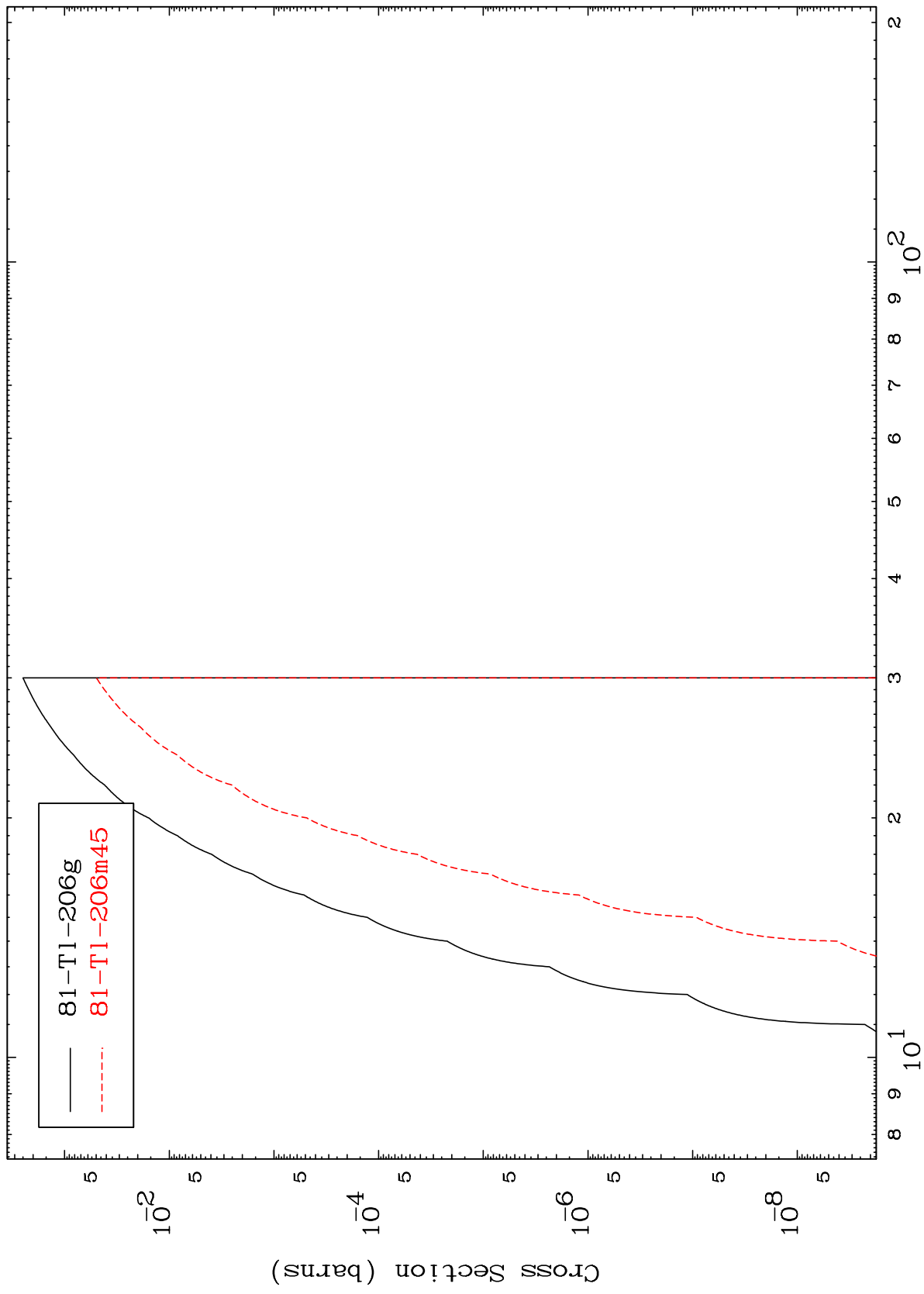
81-Tl-207m

MAT 8138

(n,2n) p

81-Tl-207m

Radionuclide Production Cross Section



81-Tl-206g  
81-Tl-206m45

16

Incident Energy (MeV)

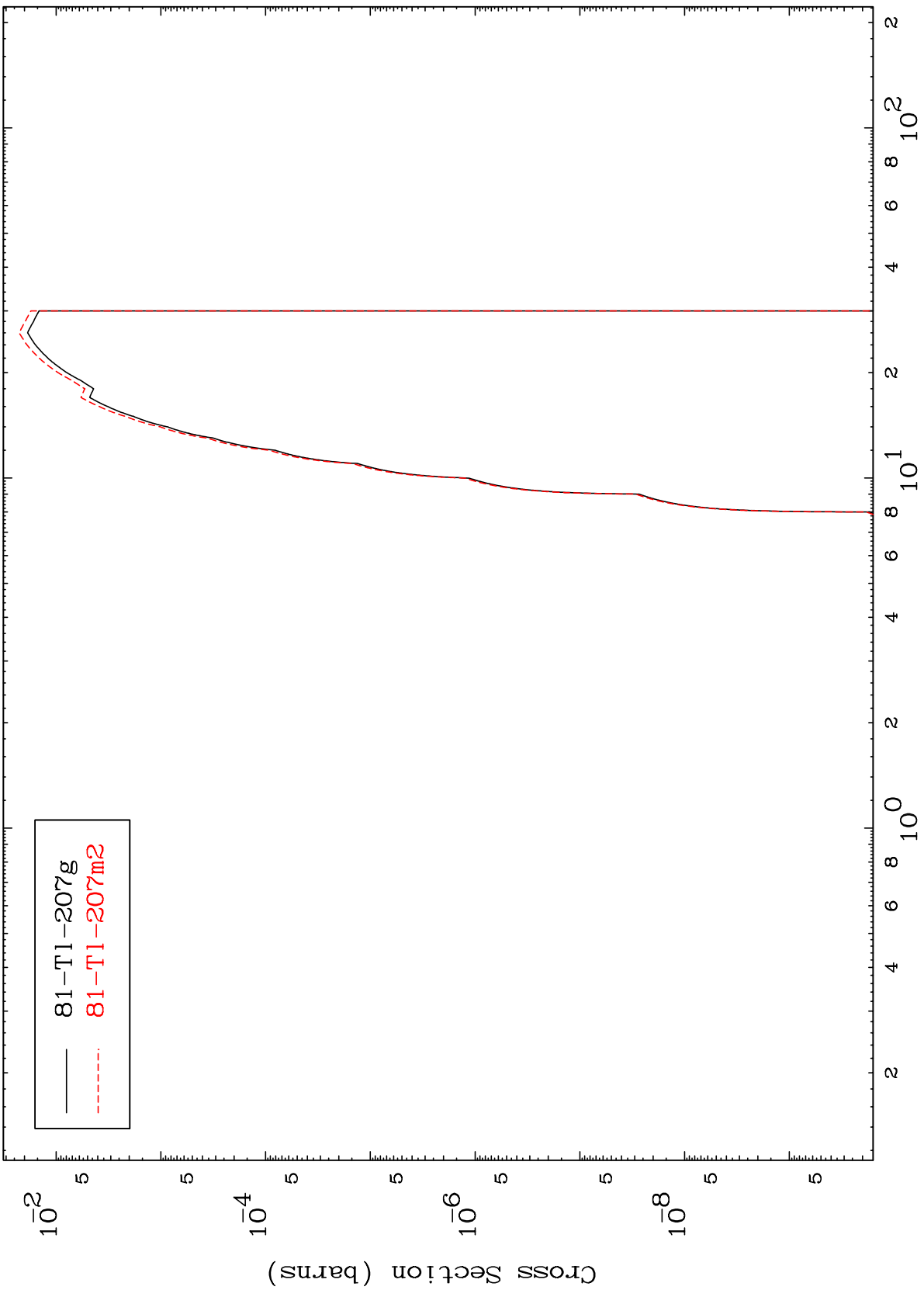
81-Tl-207m

MAT 8138

(n,d)

81-Tl-207m

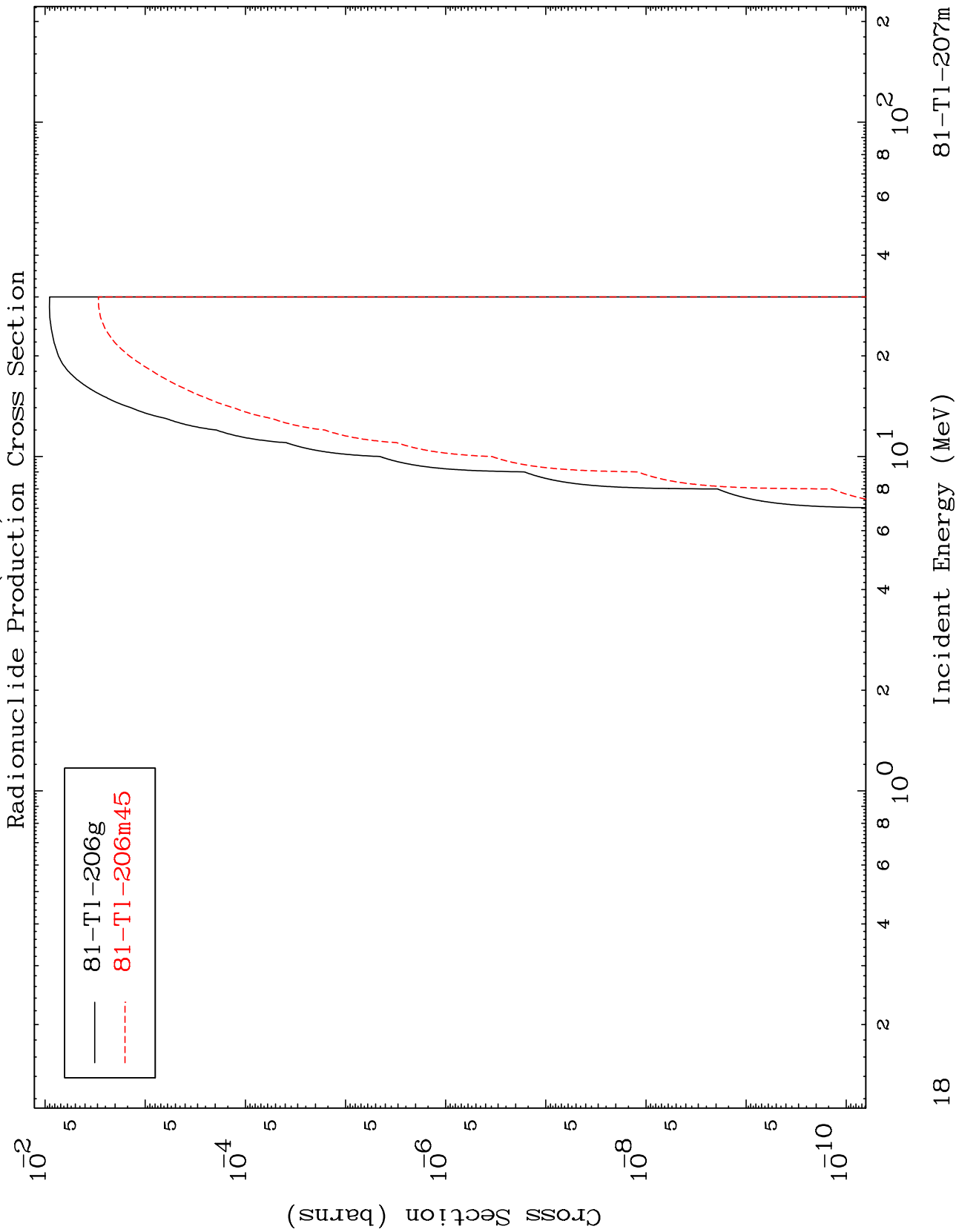
Radionuclide Production Cross Section



81-Tl-207g  
81-Tl-207m2

MAT 8138

81-Tl-207m



18

81-Tl-207m