

Program EVALPLOT  
(Version 2021-1)

by

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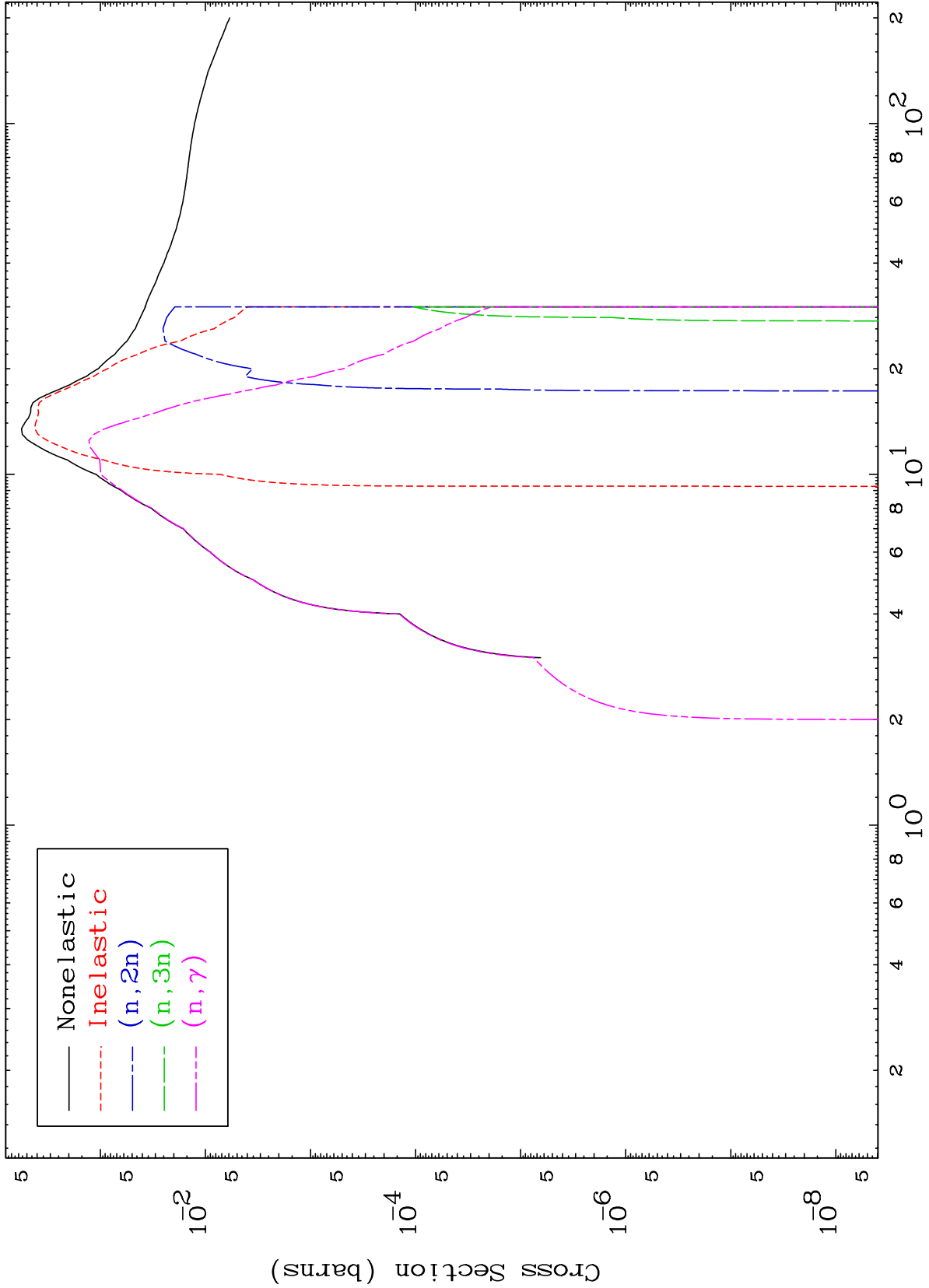
Press Mouse Button to Start

MAT 8290

Photon Major

83-Bi-197m

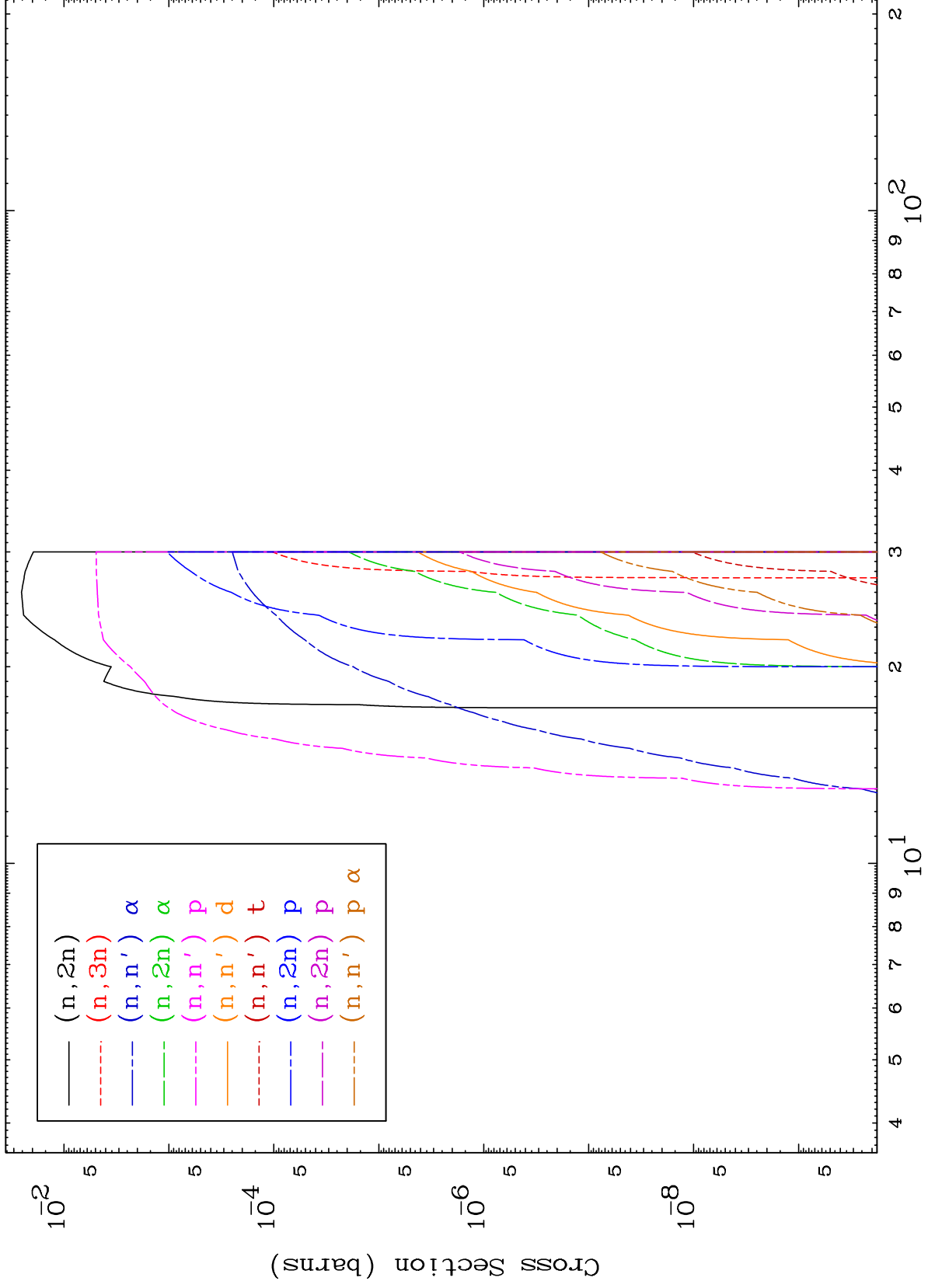
0 Kelvin Cross Sections



MAT 8290

Photon Neutron Absorption  
0 Kelvin Cross Sections

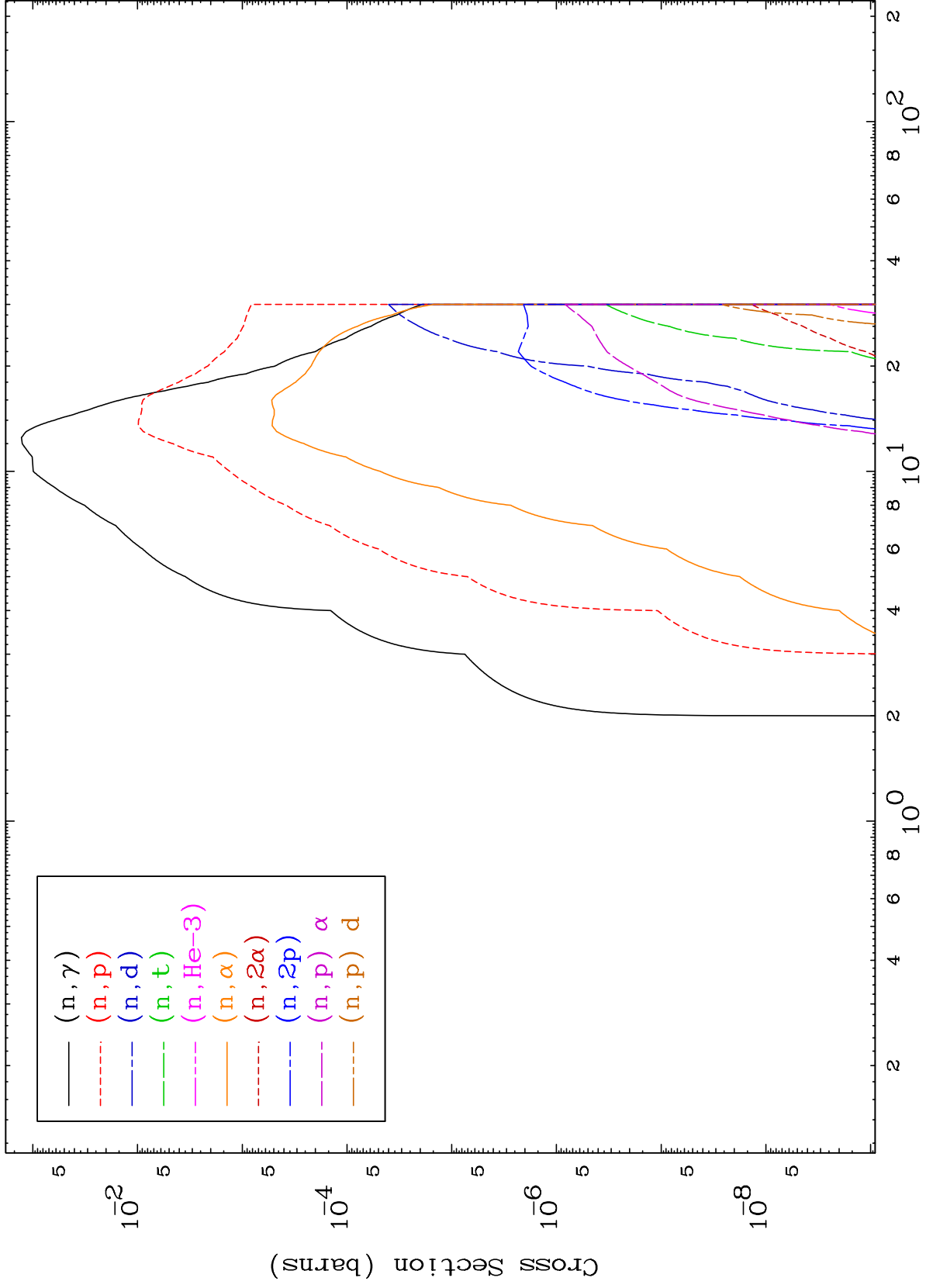
83-Bi-197m



2

Incident Energy (MeV)

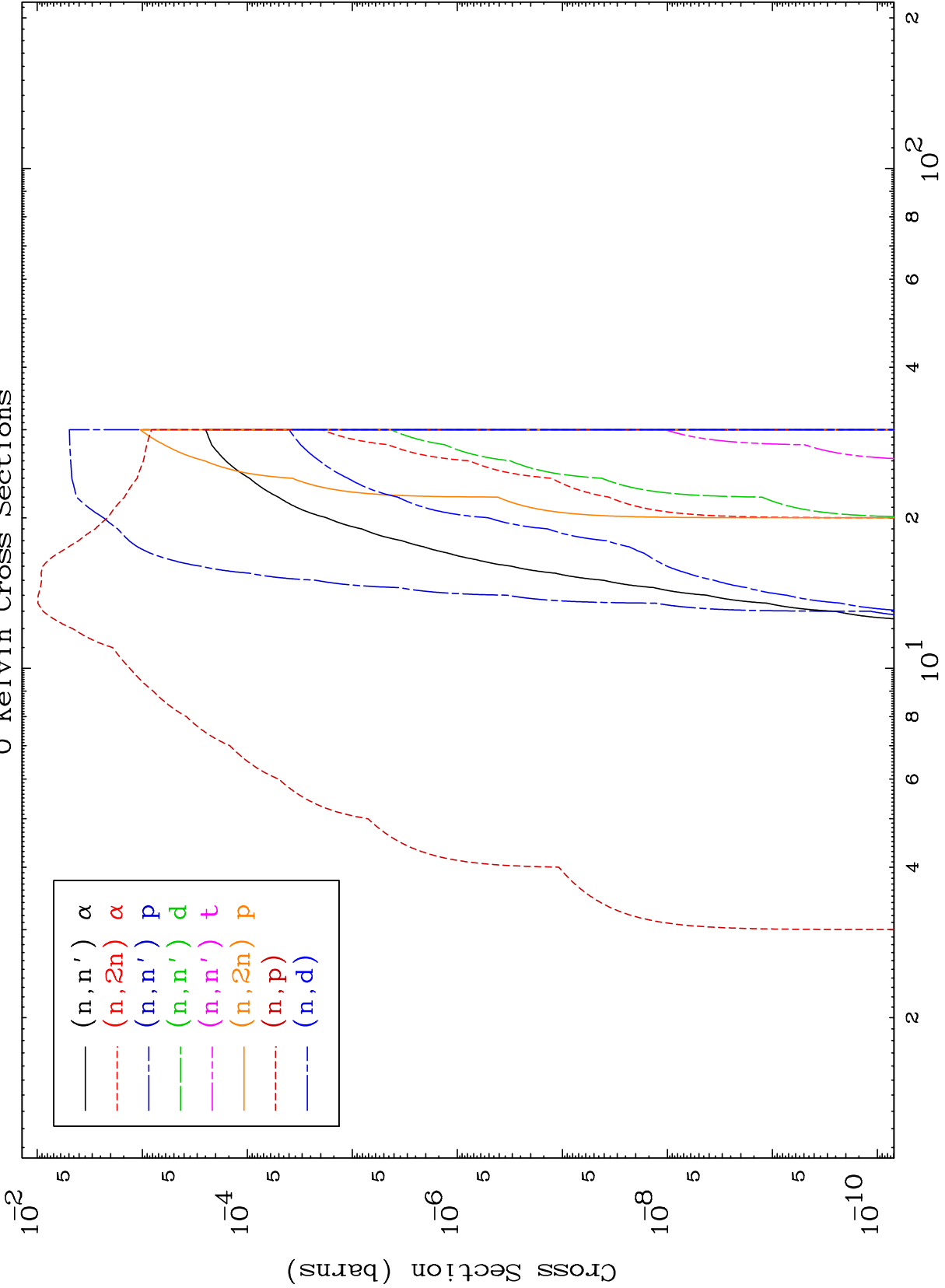
83-Bi-197m

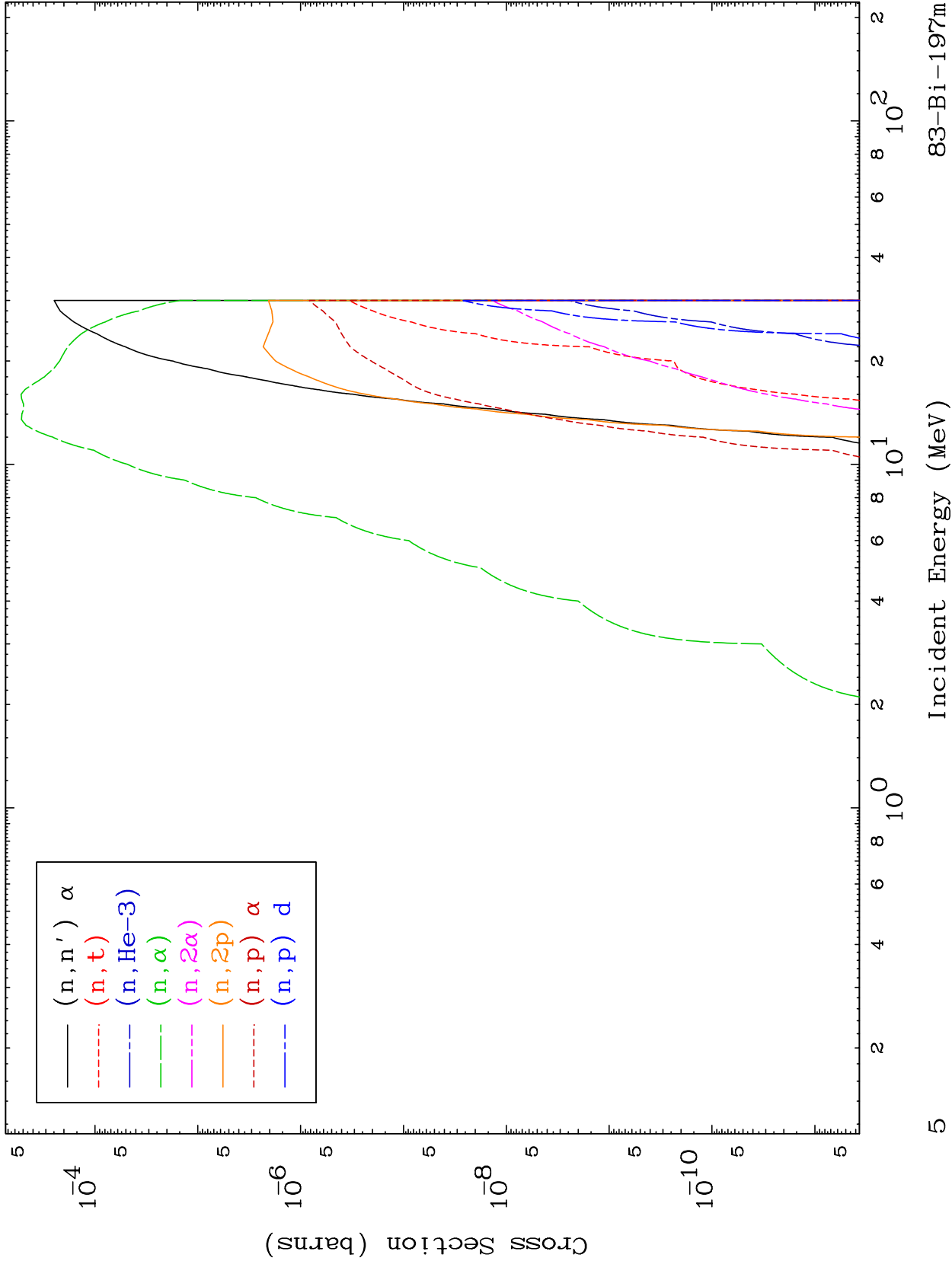


MAT 8290

Photon Charged Particle  
0 Kelvin Cross Sections

83-Bi-197m



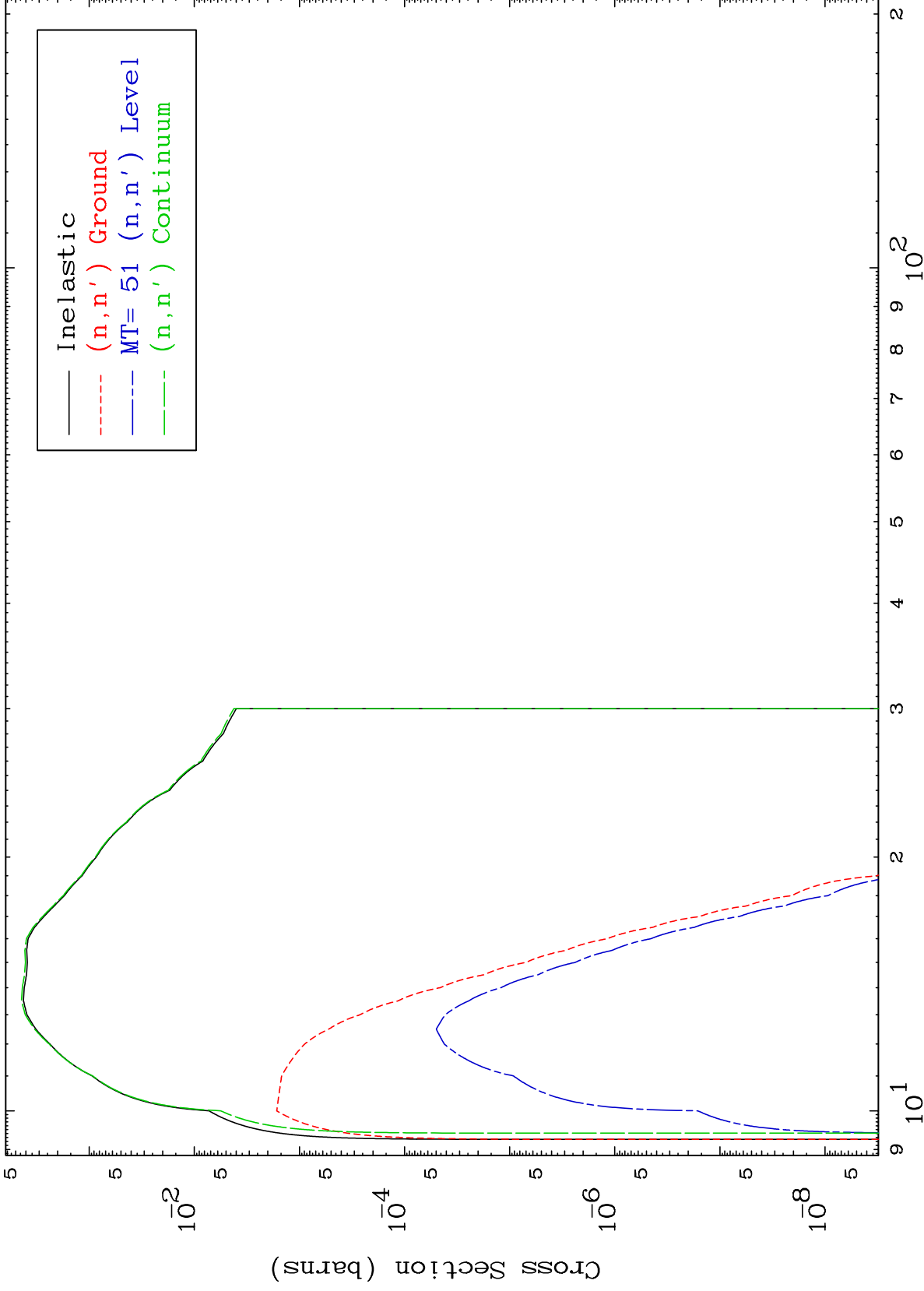


MAT 8290

( $\gamma, n'$ ) Levels

83-Bi-197m

0 Kelvin Cross Sections



Incident Energy (MeV)

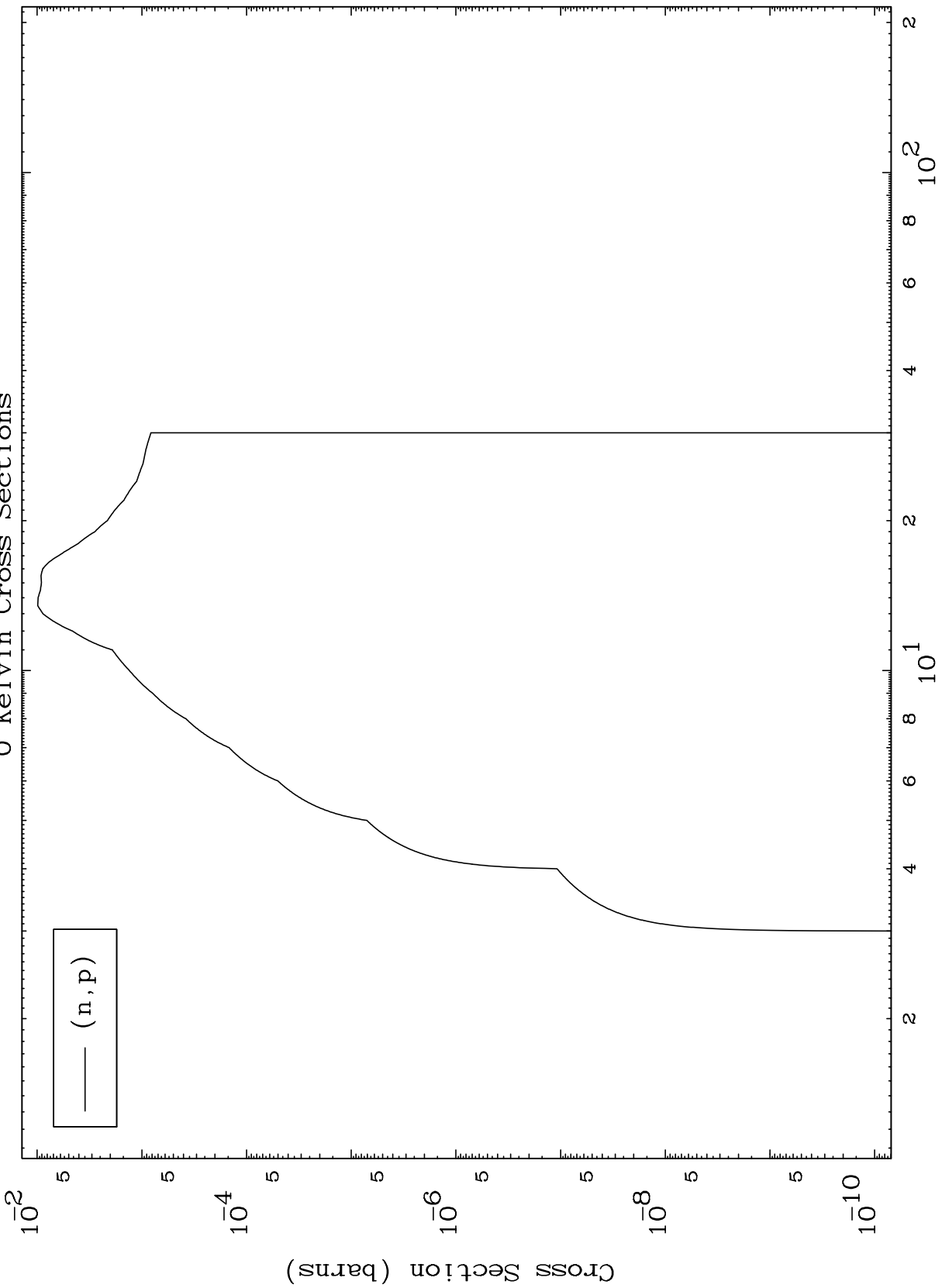
83-Bi-197m

MAT 8290

( $\gamma, p$ ) Levels

83-Bi-197m

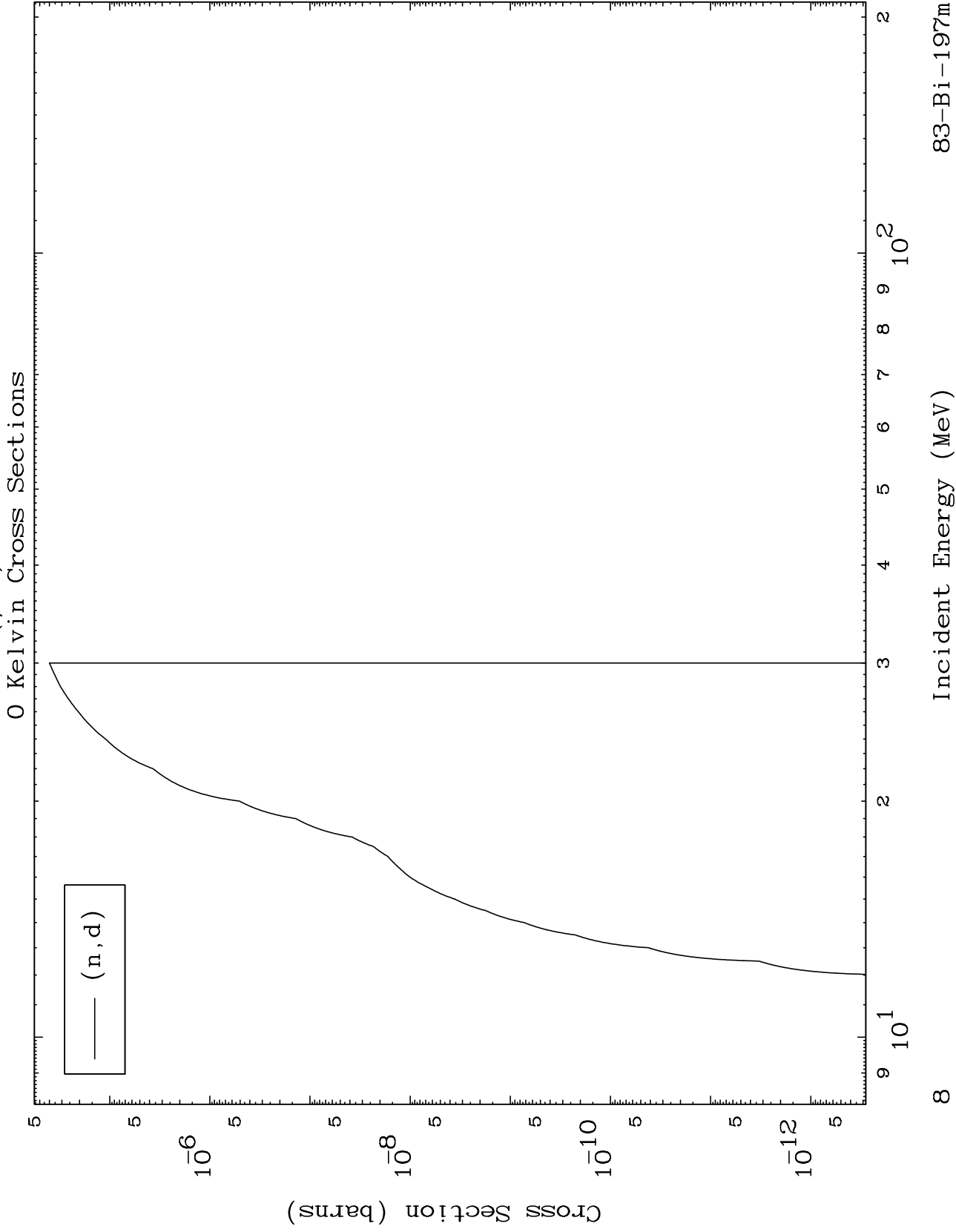
0 Kelvin Cross Sections



MAT 8290

( $\gamma, d$ ) Levels

$^{83}\text{Bi}$ -197m

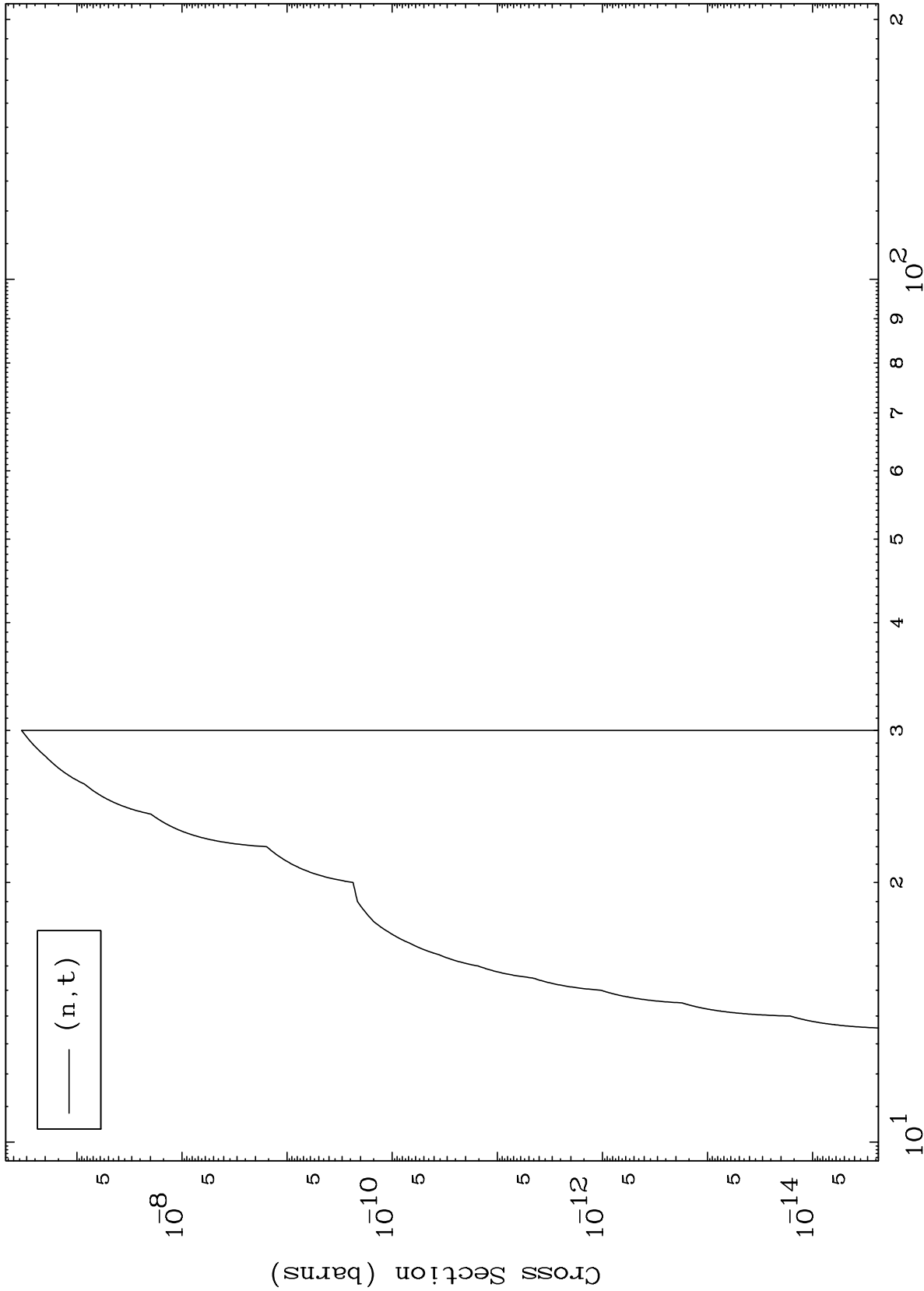


MAT 8290

( $\gamma, t$ ) Levels

83-Bi-197m

0 Kelvin Cross Sections



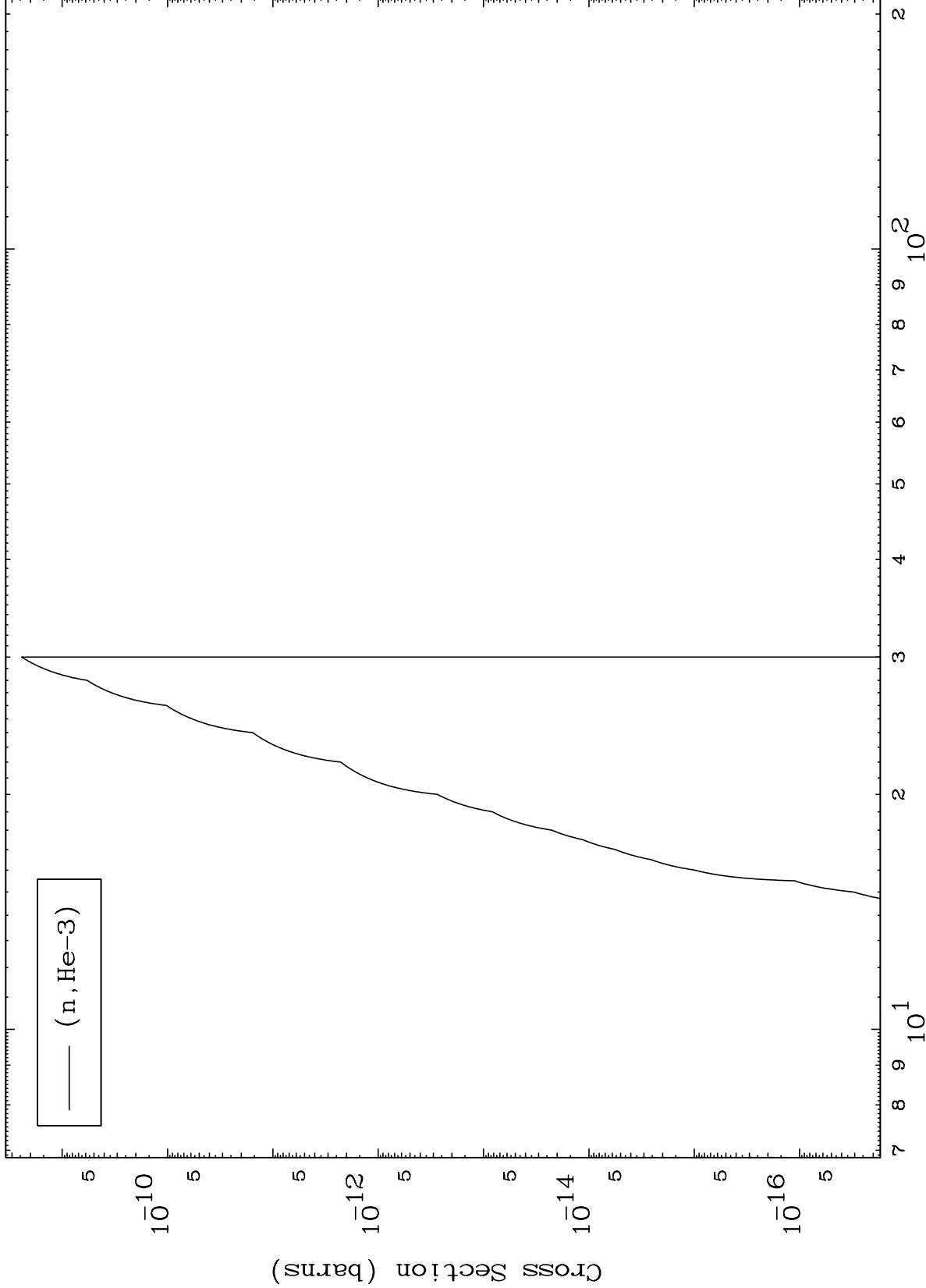
Incident Energy (MeV)

83-Bi-197m

MAT 8290

( $\gamma, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

83-Bi-197m



10

Incident Energy (MeV)

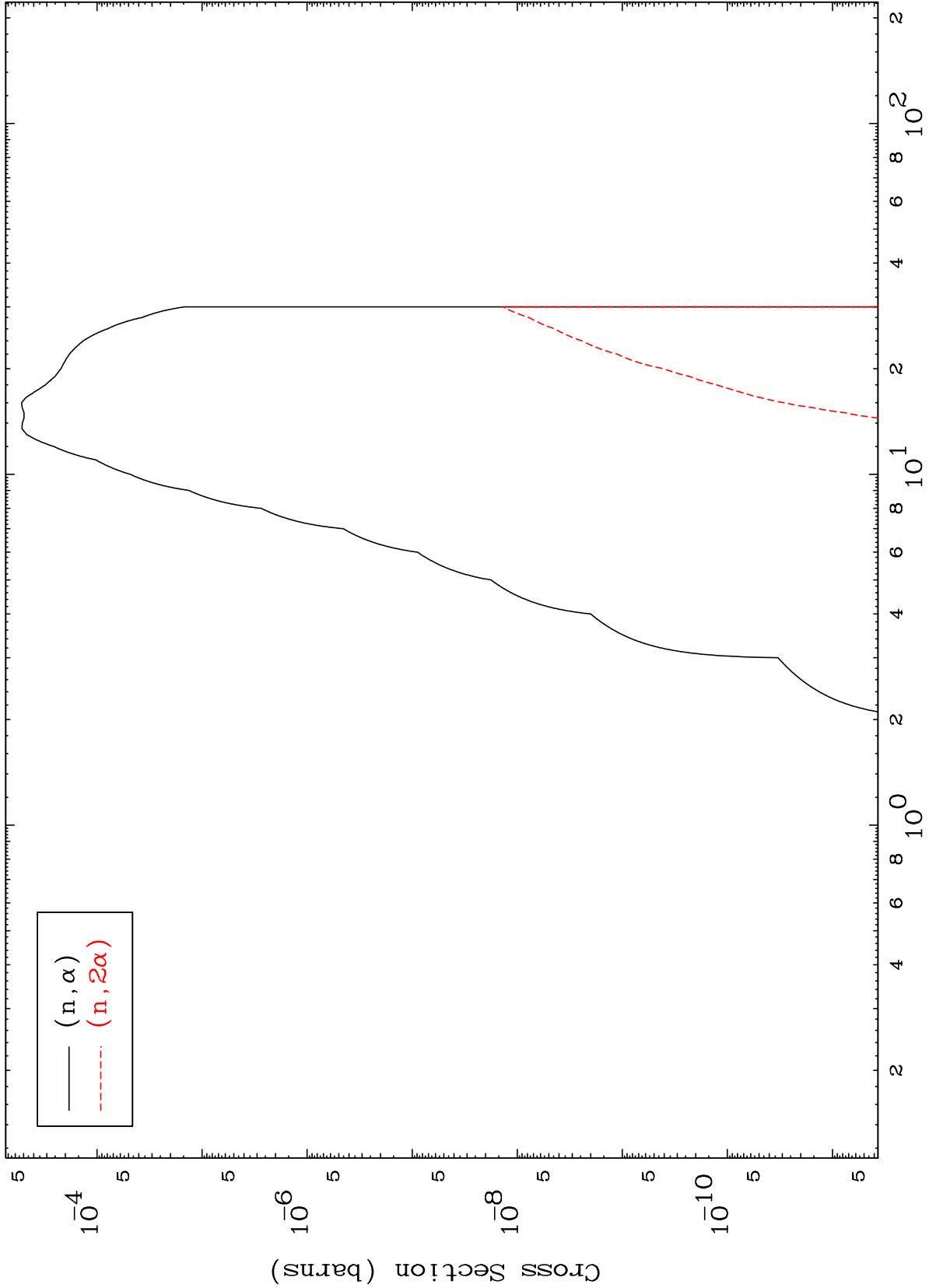
83-Bi-197m

MAT 8290

( $\gamma, \alpha$ ) Levels

83-Bi-197m

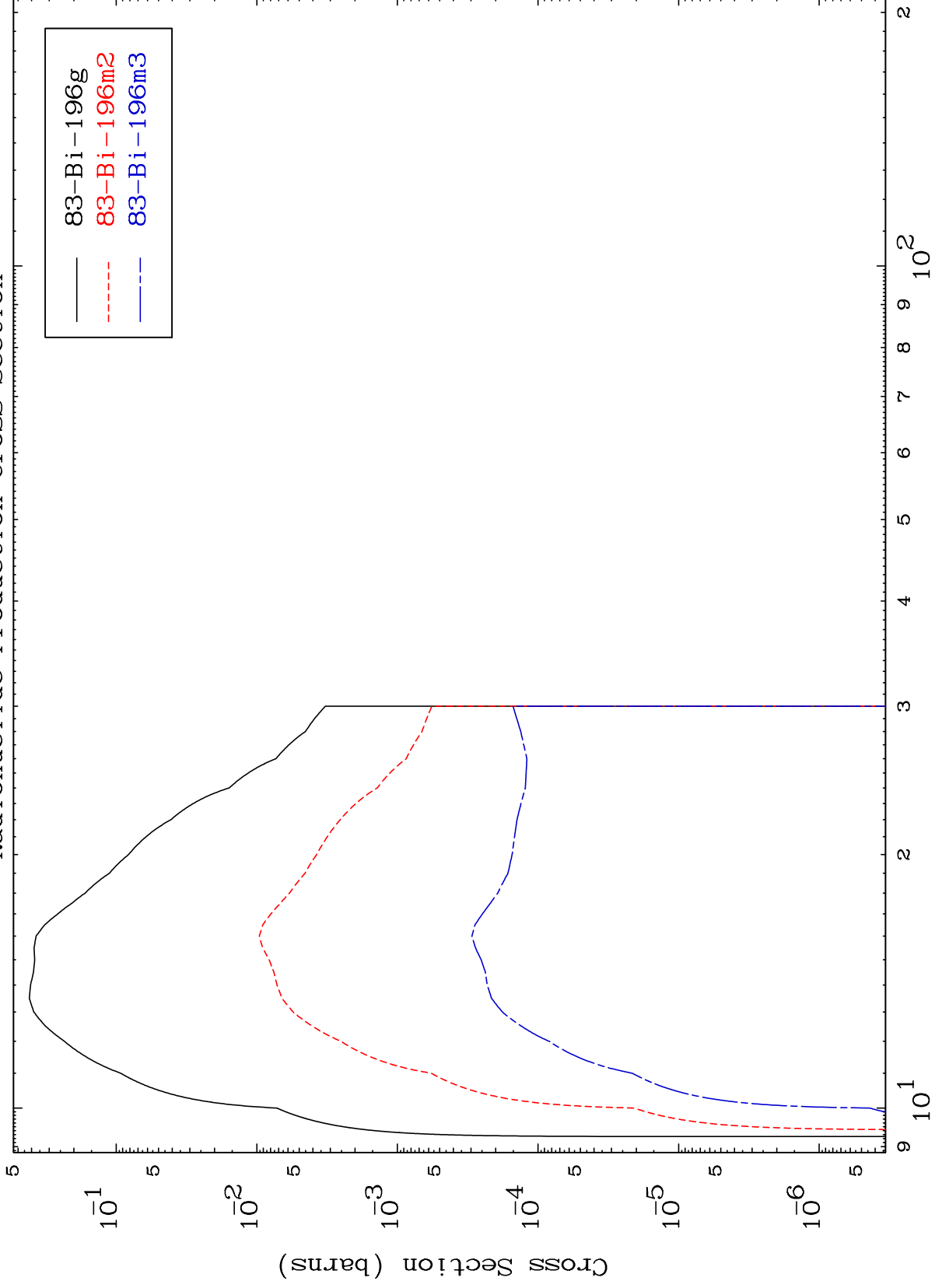
0 Kelvin Cross Sections



MAT 8290

83-Bi-197m

Inelastic  
Radionuclide Production Cross Section



83-Bi-197m

Incident Energy (MeV)

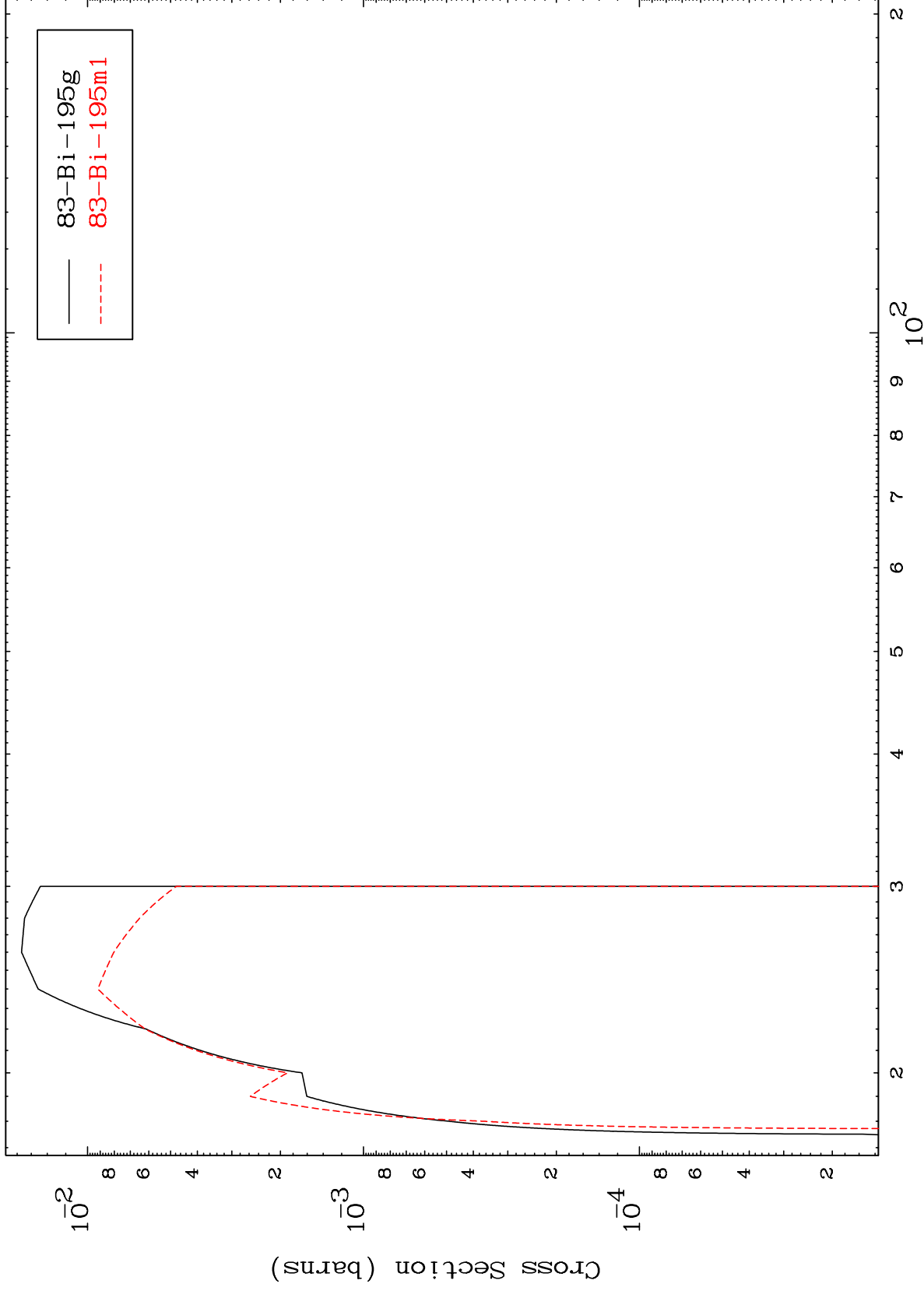
12

MAT 8290

(n,2n)

83-Bi-197m

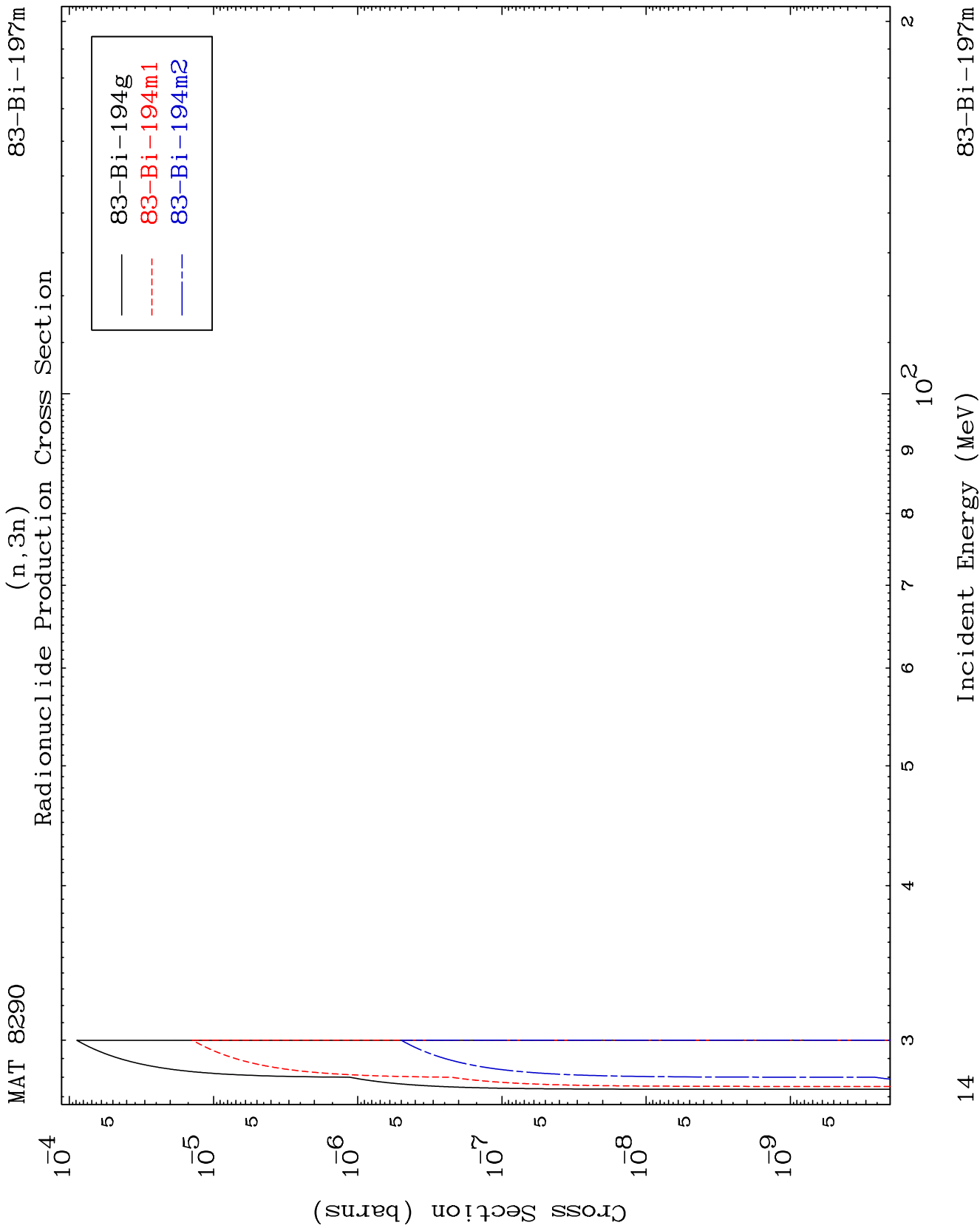
Radionuclide Production Cross Section



13

Incident Energy (MeV)

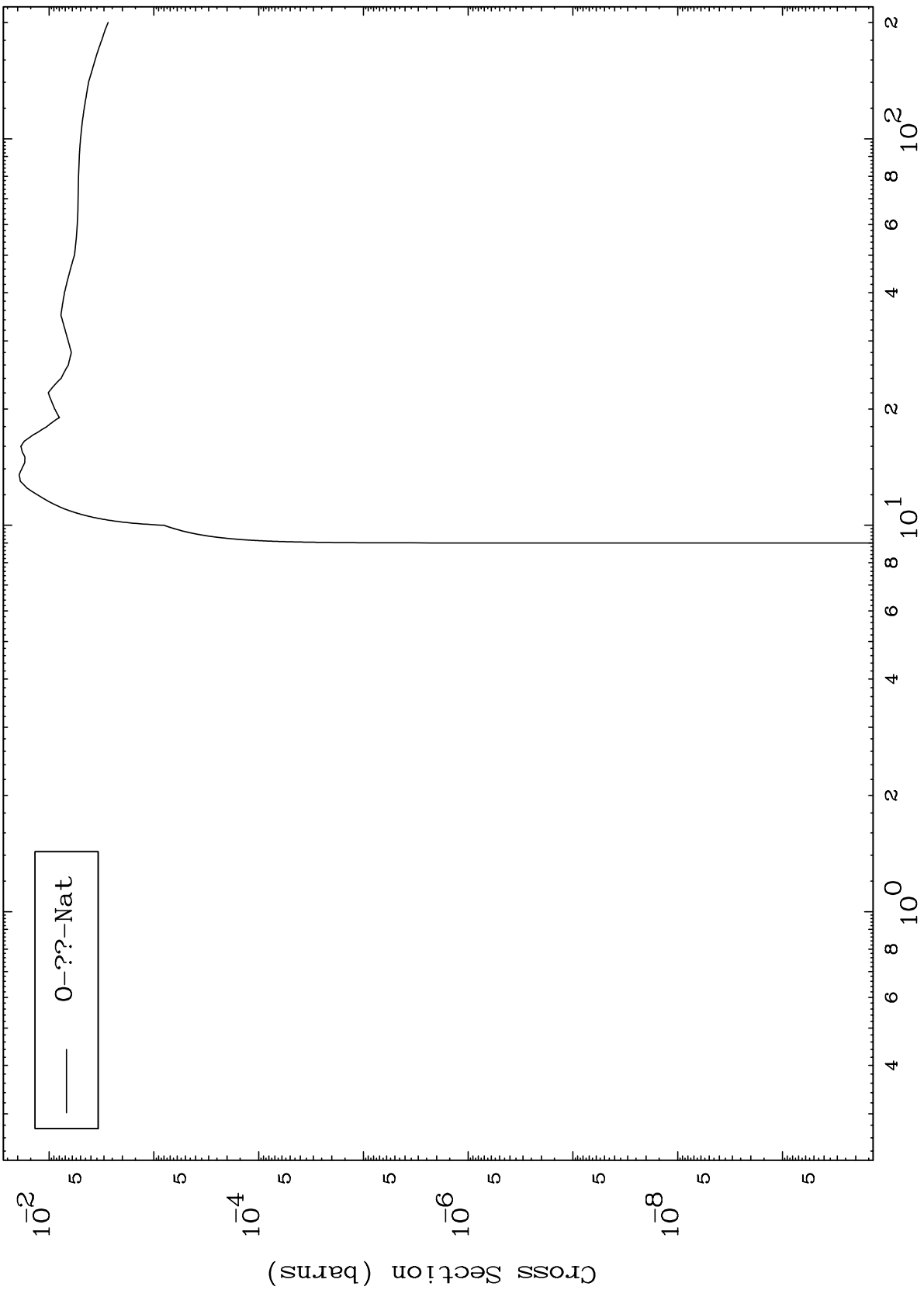
83-Bi-197m



MAT 8290

83-Bi-197m

Fission  
Radionuclide Production Cross Section

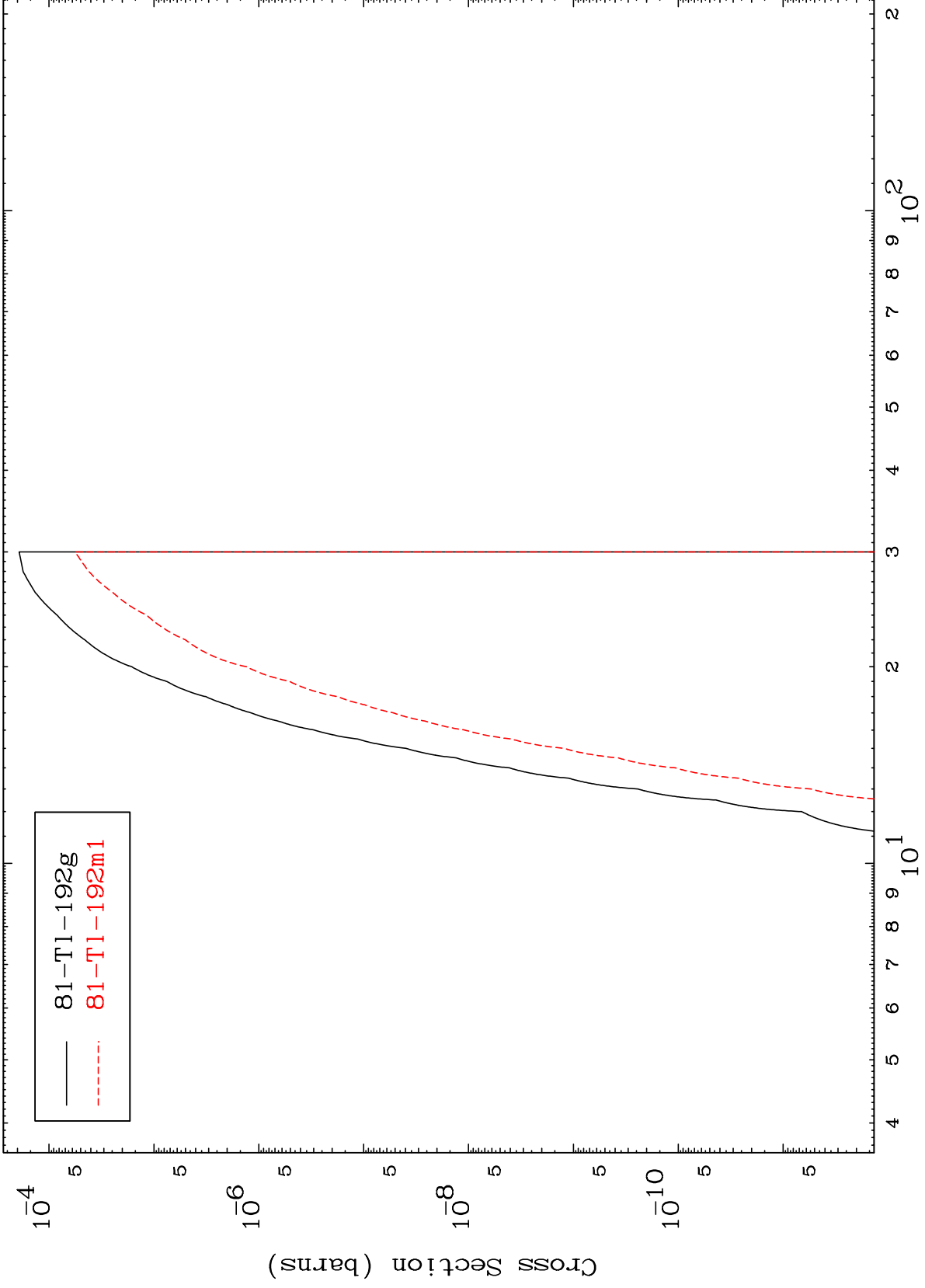


MAT 8290

$(n, n') \alpha$

83-Bi-197m

Radionuclide Production Cross Section



16

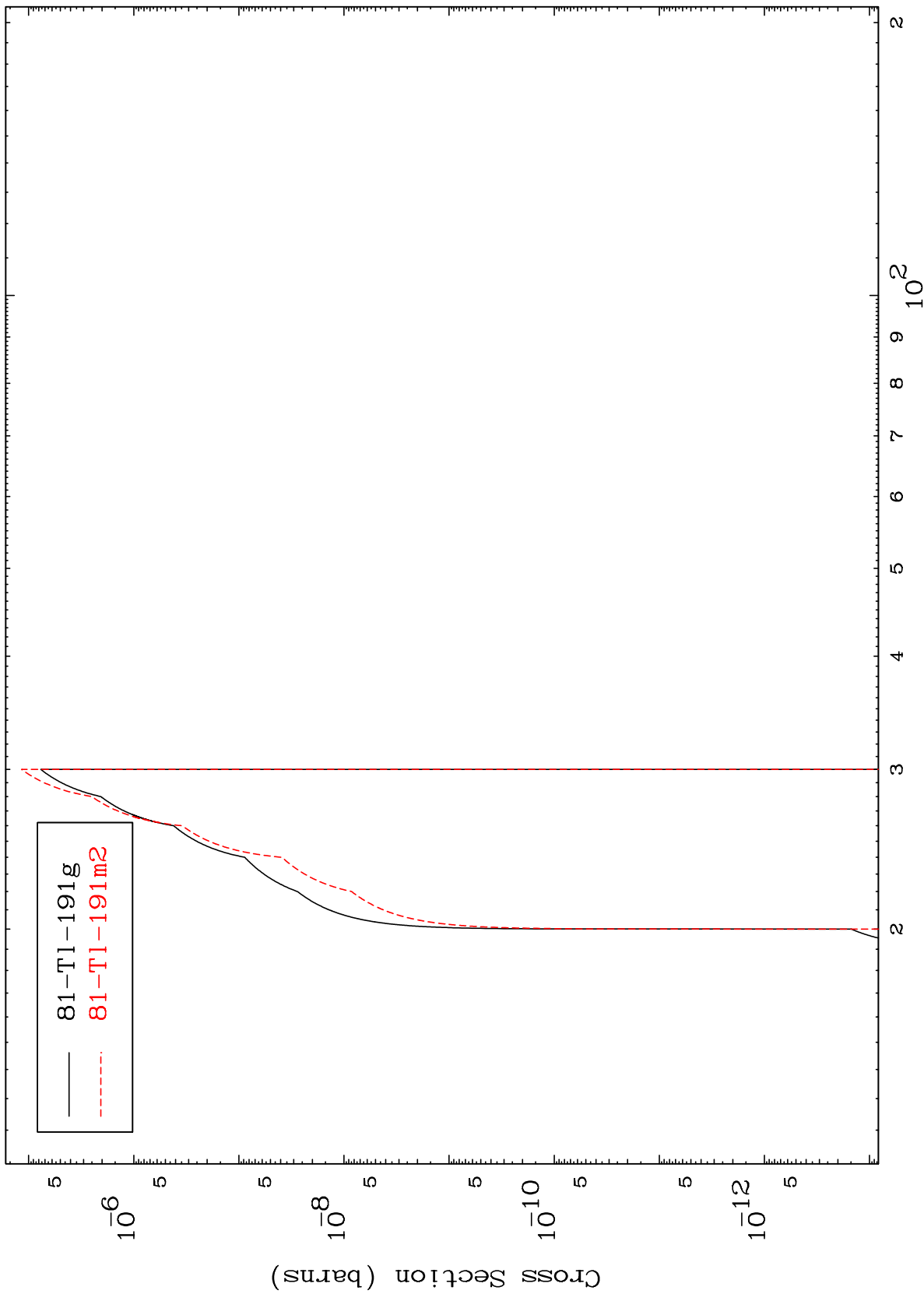
Incident Energy (MeV)

83-Bi-197m

MAT 8290

83-Bi-197m

(n,2n)  $\alpha$   
Radionuclide Production Cross Section

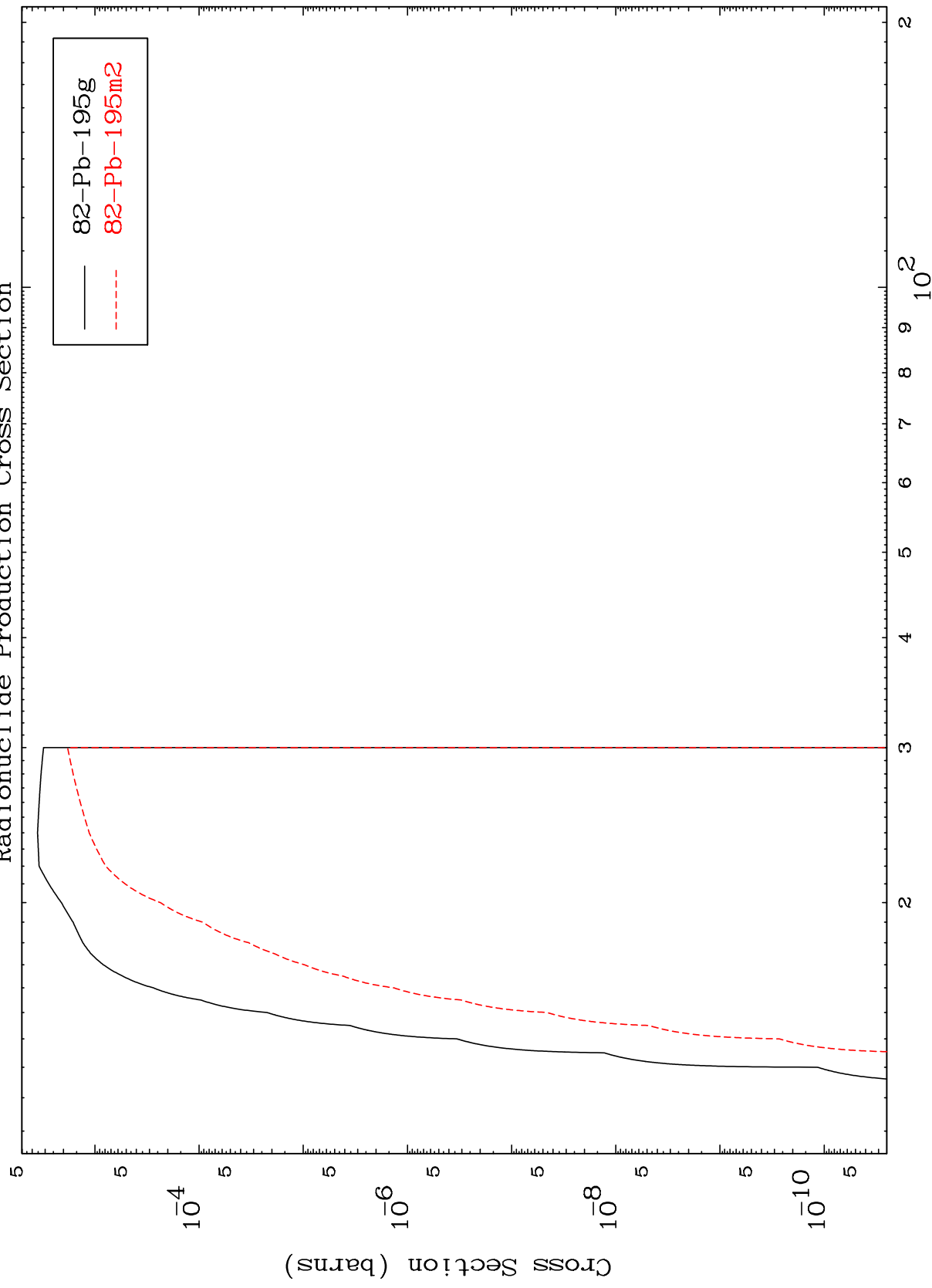


MAT 8290

$(n, n')$  p

83-Bi-197m

Radionuclide Production Cross Section



18

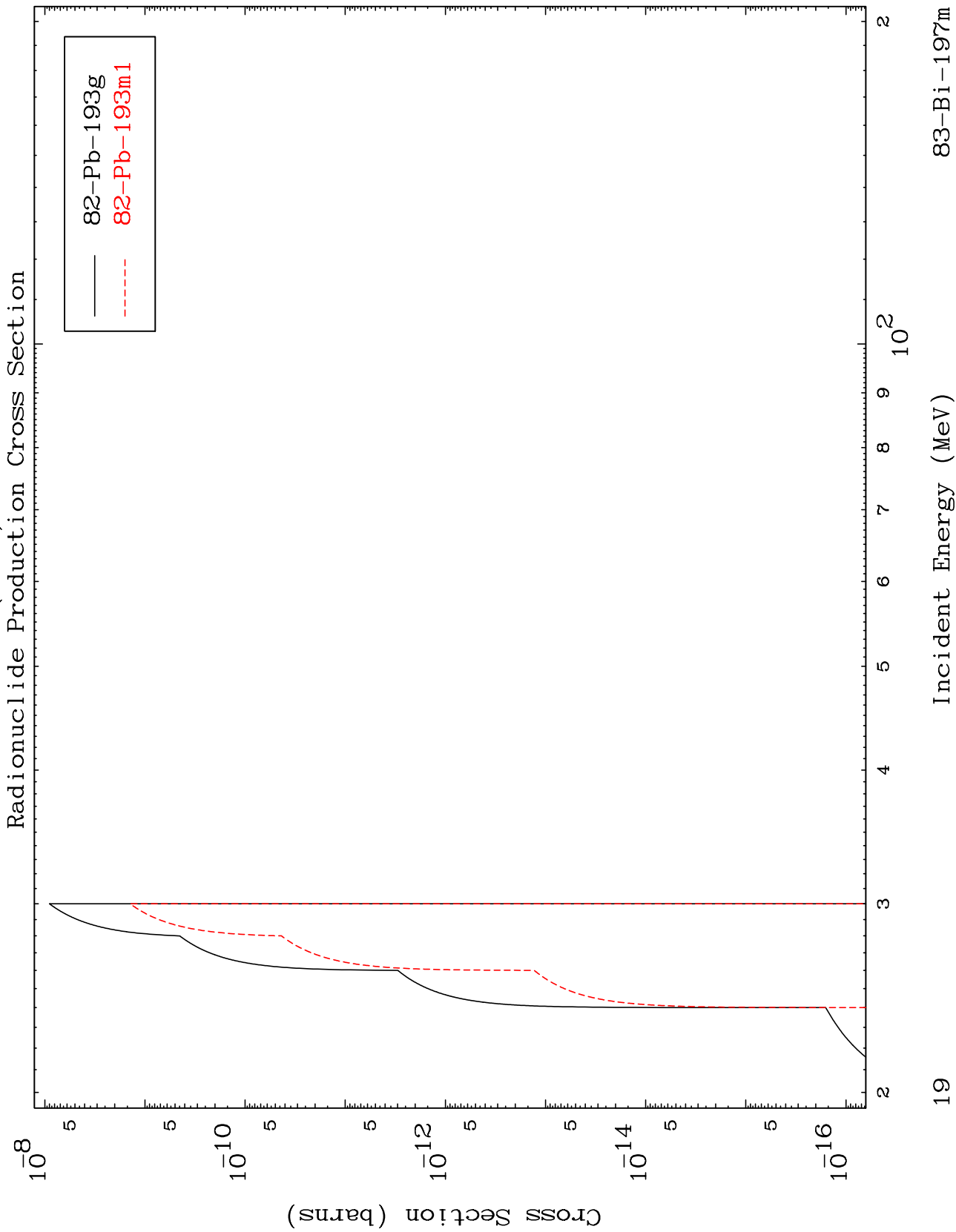
Incident Energy (MeV)

83-Bi-197m

MAT 8290

(n,n') t

83-Bi-197m

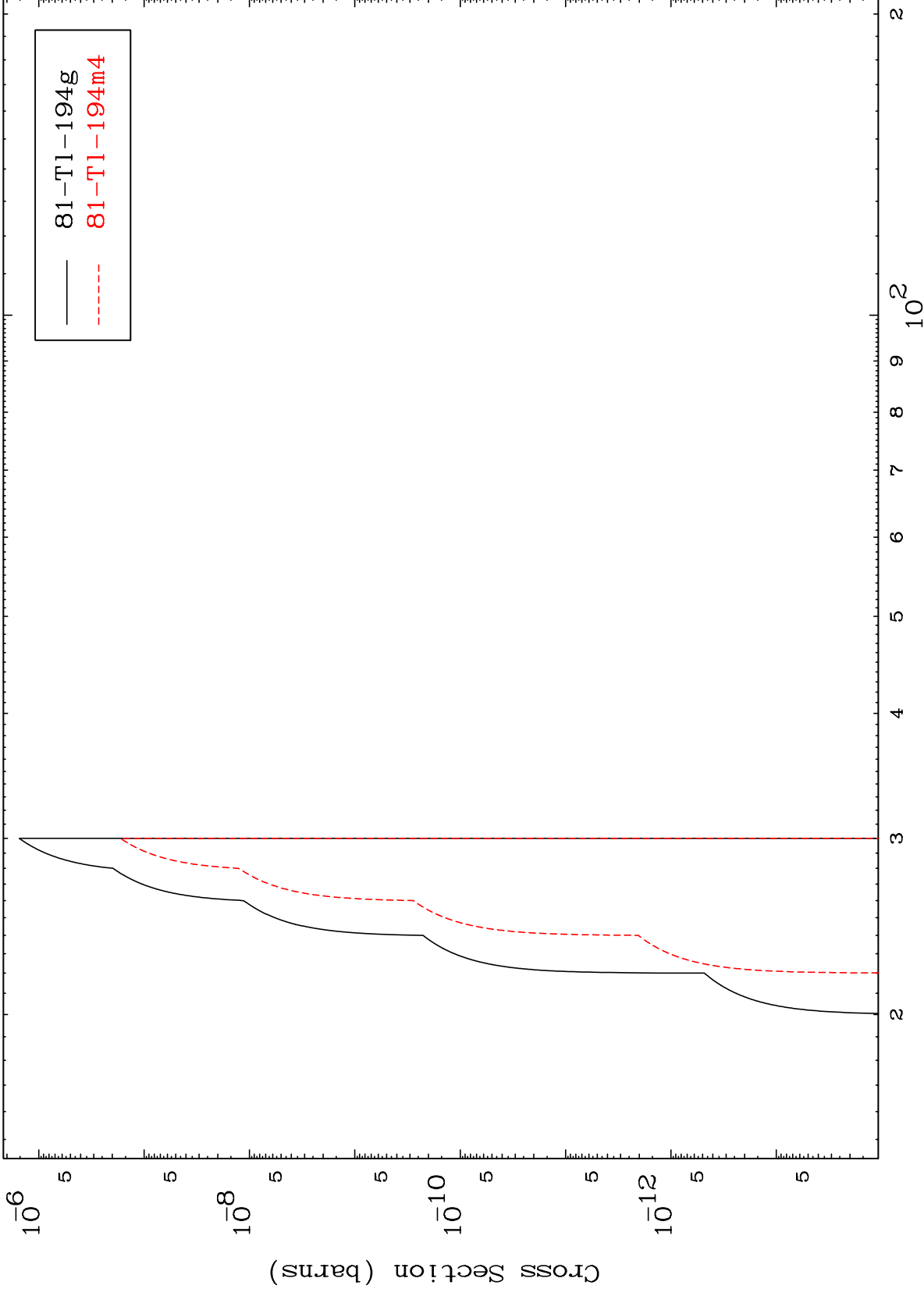


19

Incident Energy (MeV)

83-Bi-197m

Radionuclide Production Cross Section

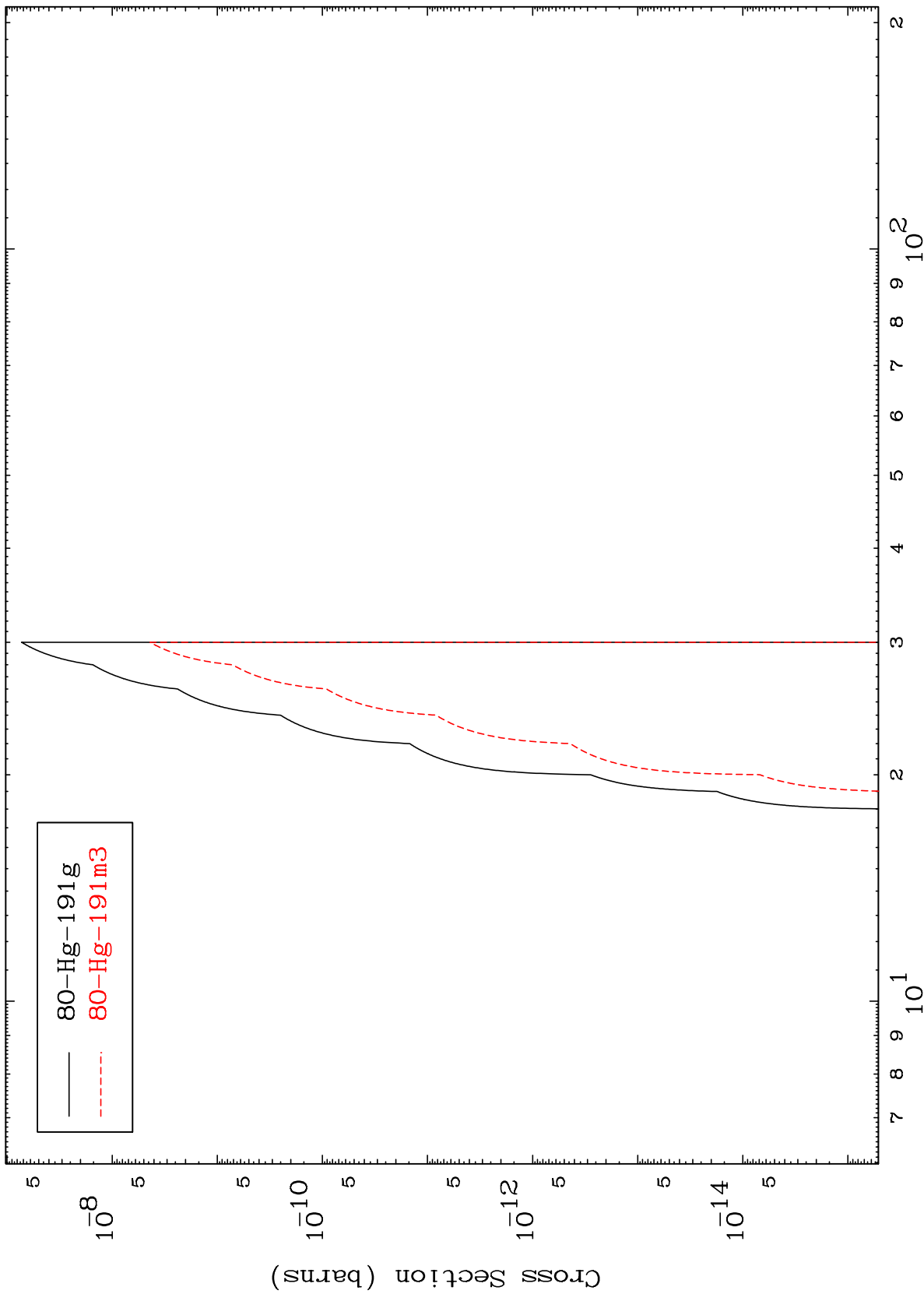


MAT 8290

(n,n') p  $\alpha$

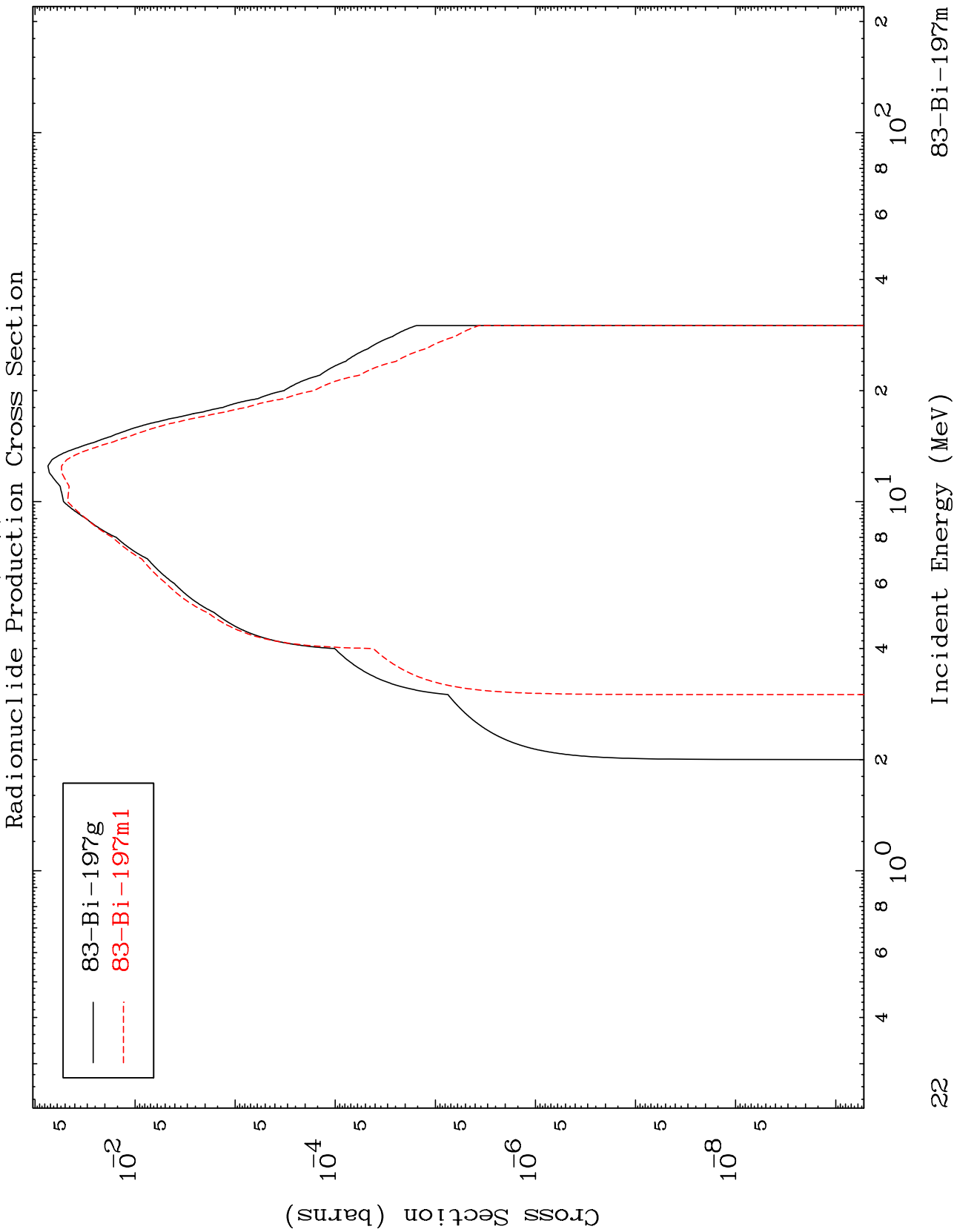
83-Bi-197m

Radionuclide Production Cross Section



MAT 8290

<sup>83</sup>Bi-197m

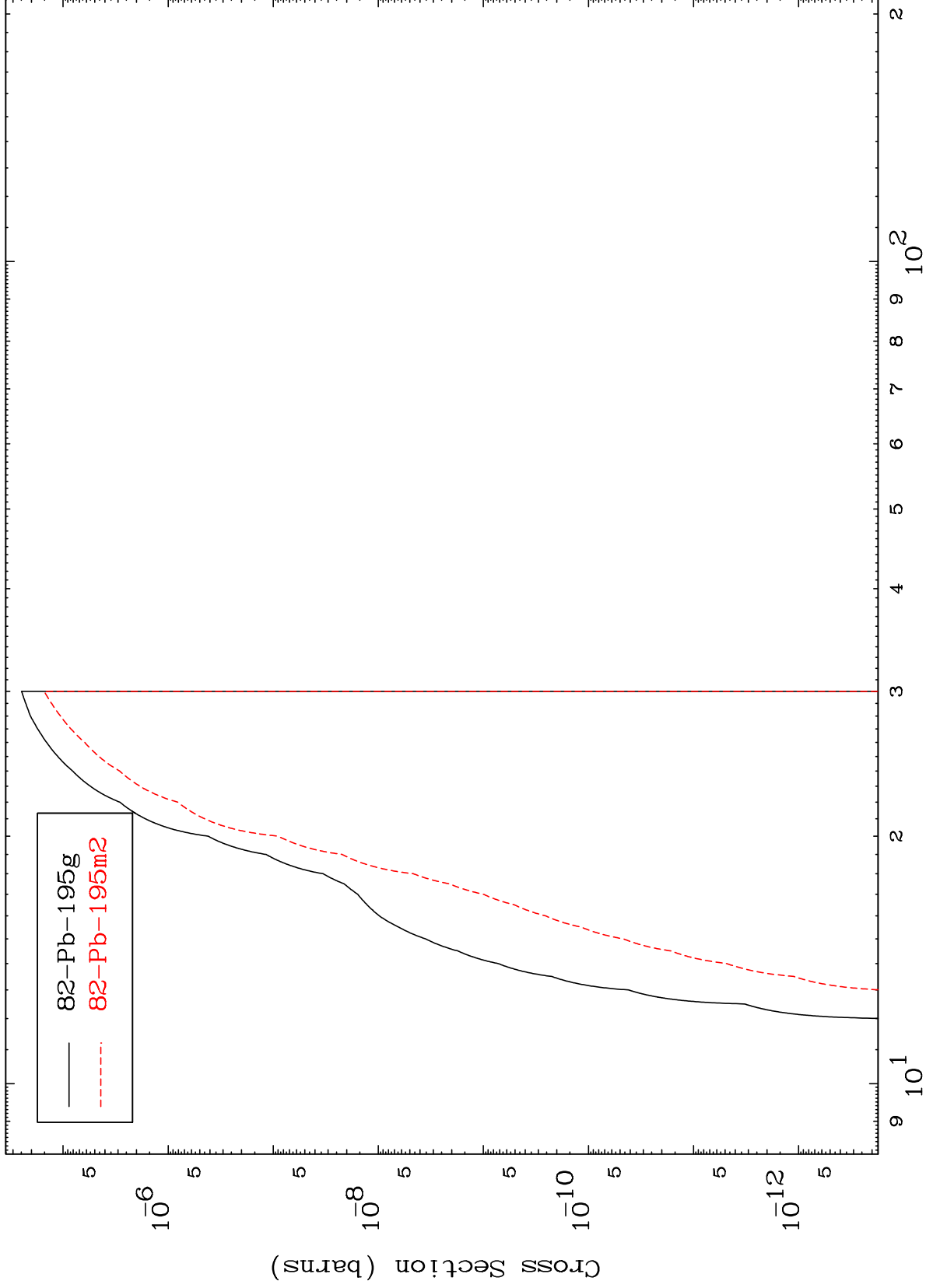


MAT 8290

(n,d)

<sup>83</sup>Bi-197m

Radionuclide Production Cross Section



23

Incident Energy (MeV)

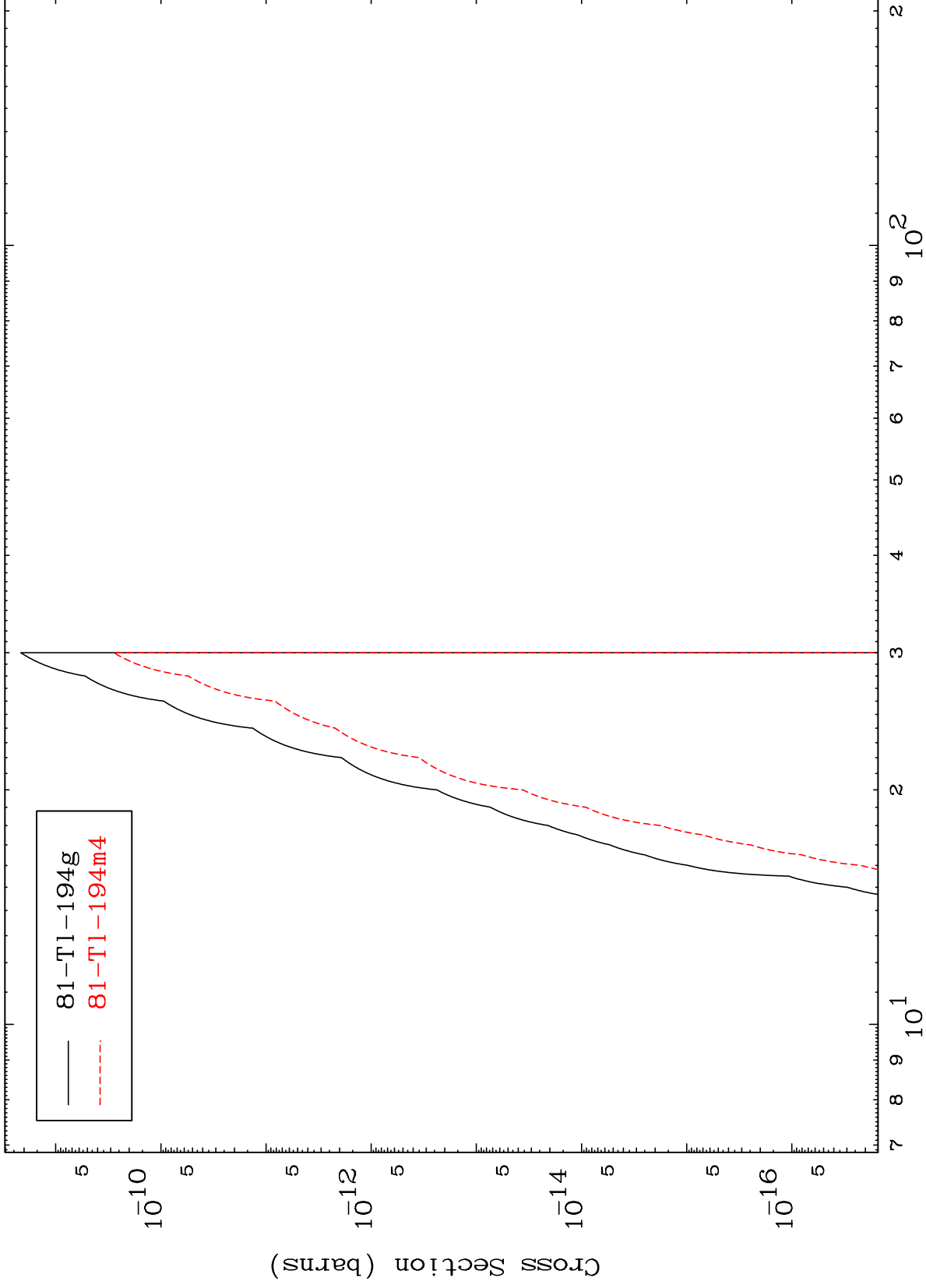
<sup>83</sup>Bi-197m

MAT 8290

(n,He-3)

83-Bi-197m

Radionuclide Production Cross Section



24

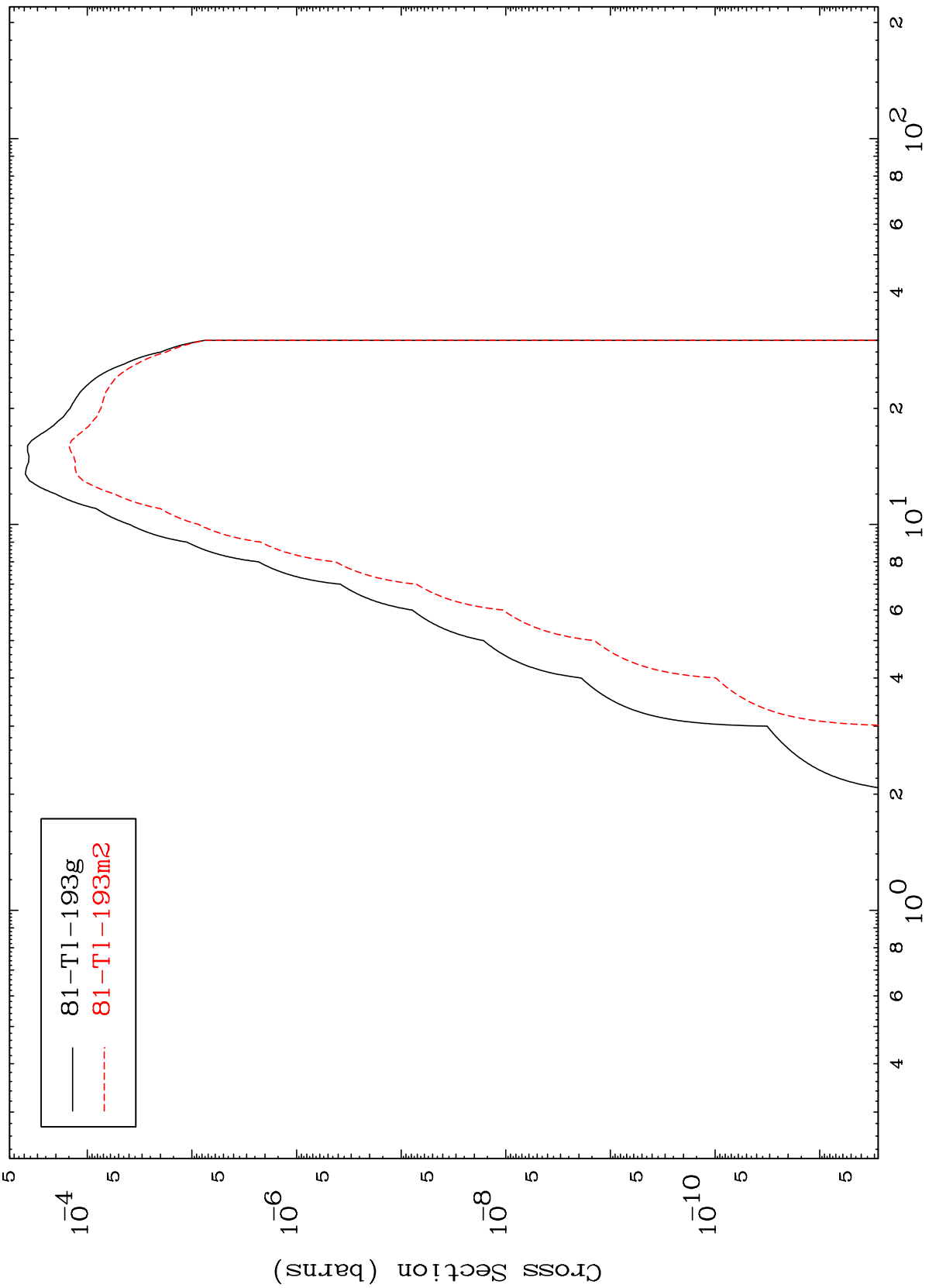
Incident Energy (MeV)

83-Bi-197m

MAT 8290

<sup>83</sup>Bi-197m

(n,α)  
Radionuclide Production Cross Section



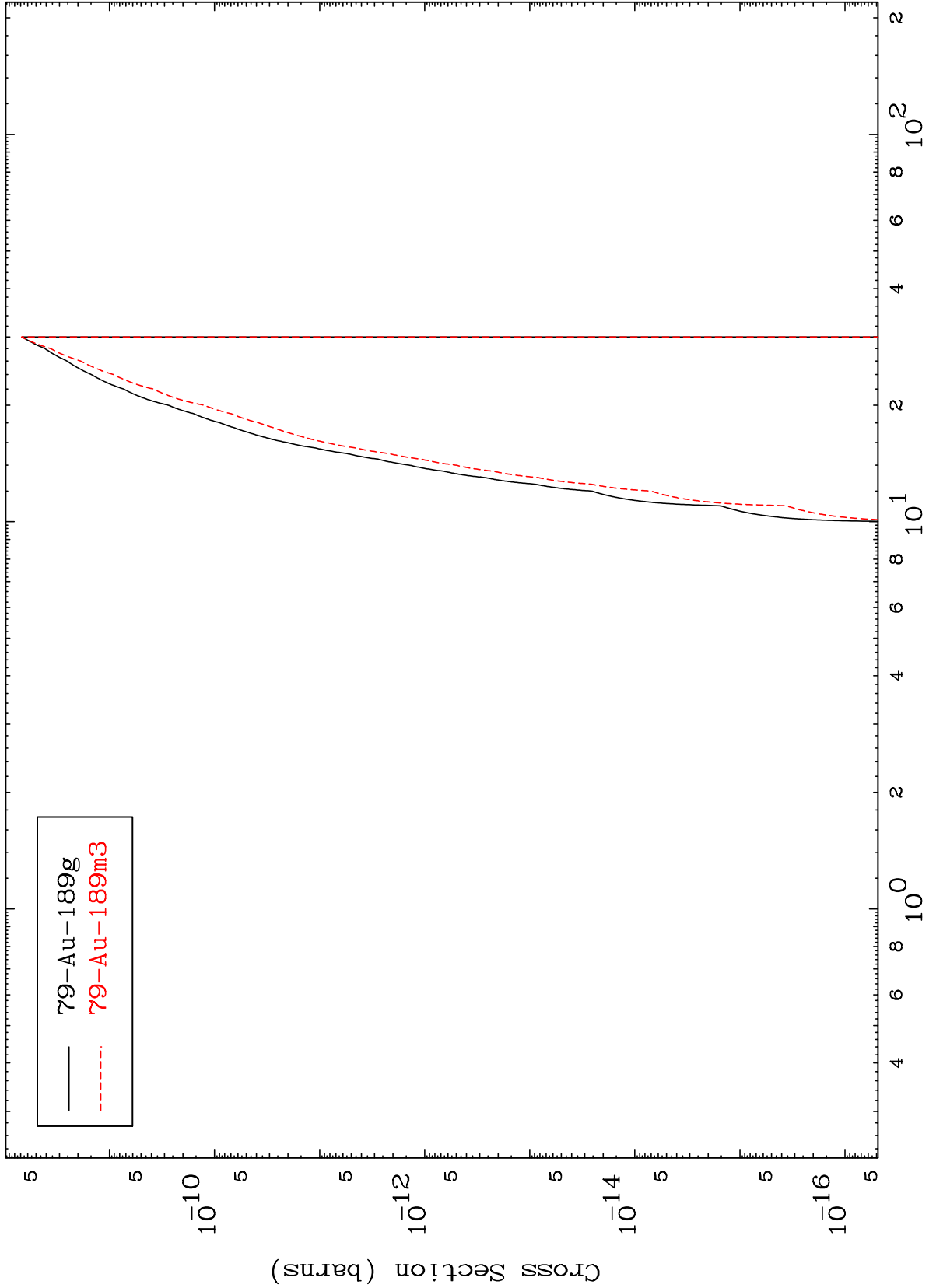
— 81-Tl-193g  
- - - 81-Tl-193m2

MAT 8290

(n,2α)

83-Bi-197m

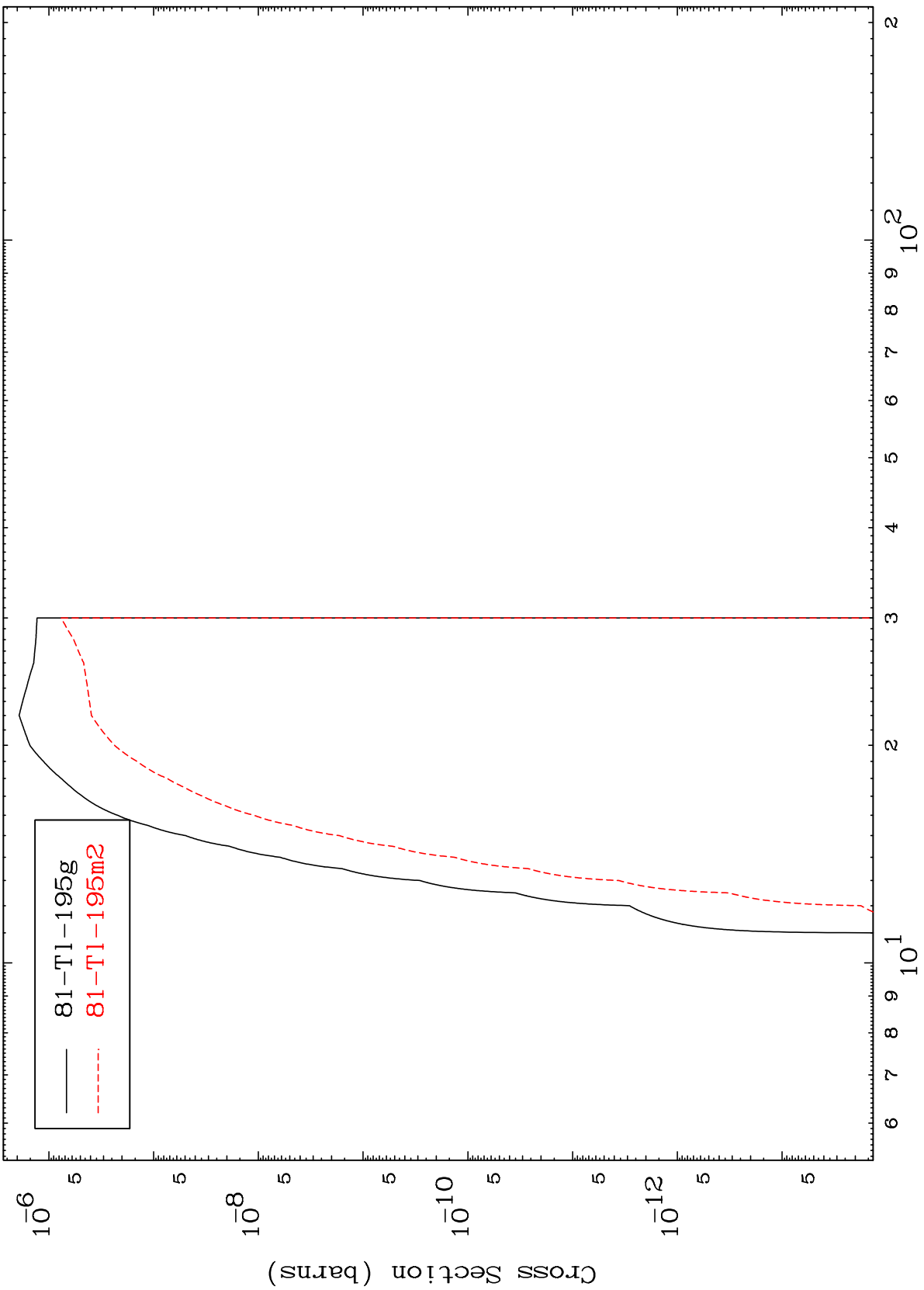
Radionuclide Production Cross Section



MAT 8290

<sup>83</sup>Bi-197m

Radionuclide Production Cross Section (n,2p)



27

Incident Energy (MeV)

<sup>83</sup>Bi-197m

Radionuclide Production Cross Section

