

Program EVALPLOT  
(Version 2021-1)

by

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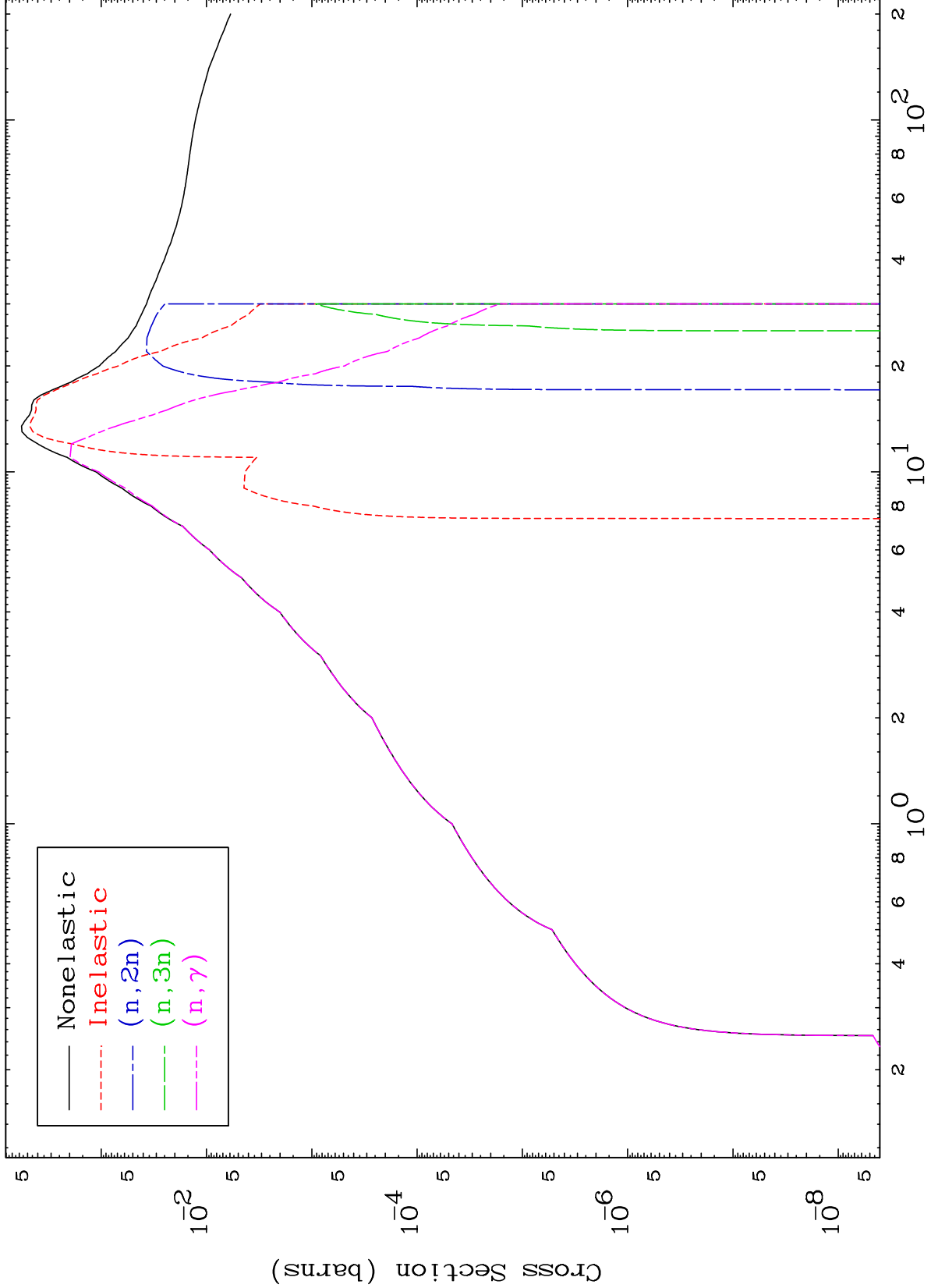
Press Mouse Button to Start

MAT 8293

Photon Major

83-Bi-198m

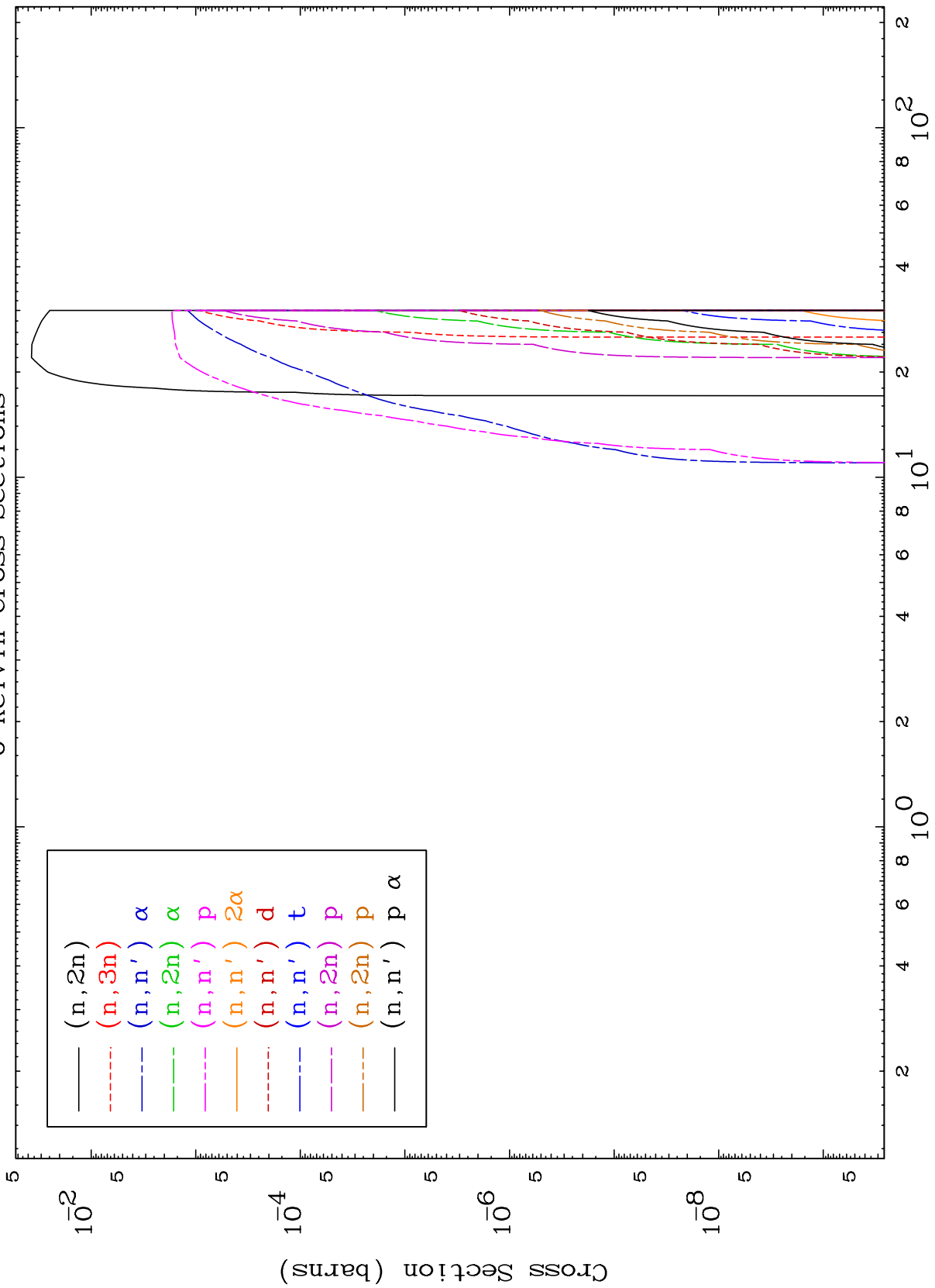
0 Kelvin Cross Sections



MAT 8293

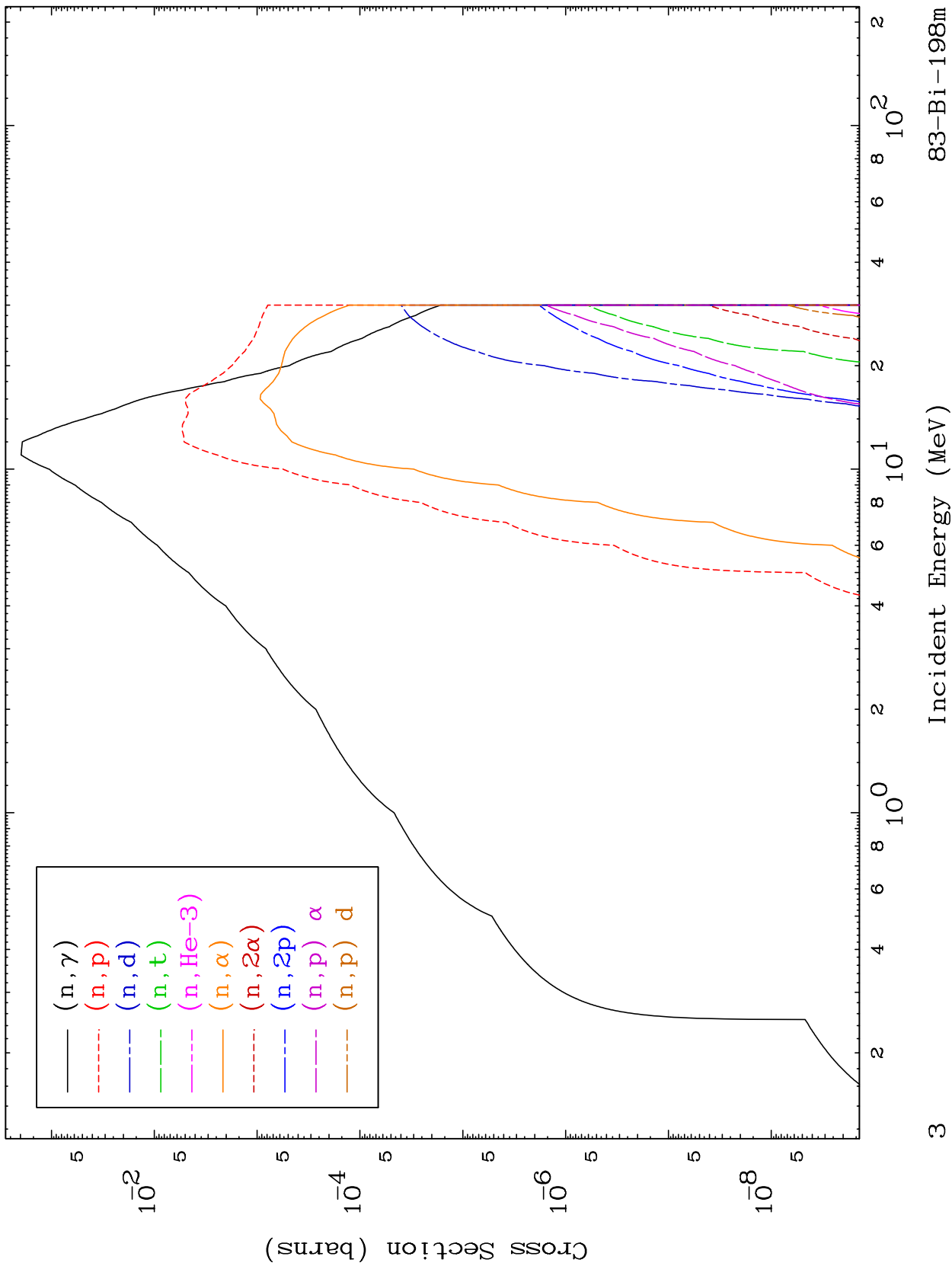
Photon Neutron Absorption  
0 Kelvin Cross Sections

83-Bi-198m



83-Bi-198m

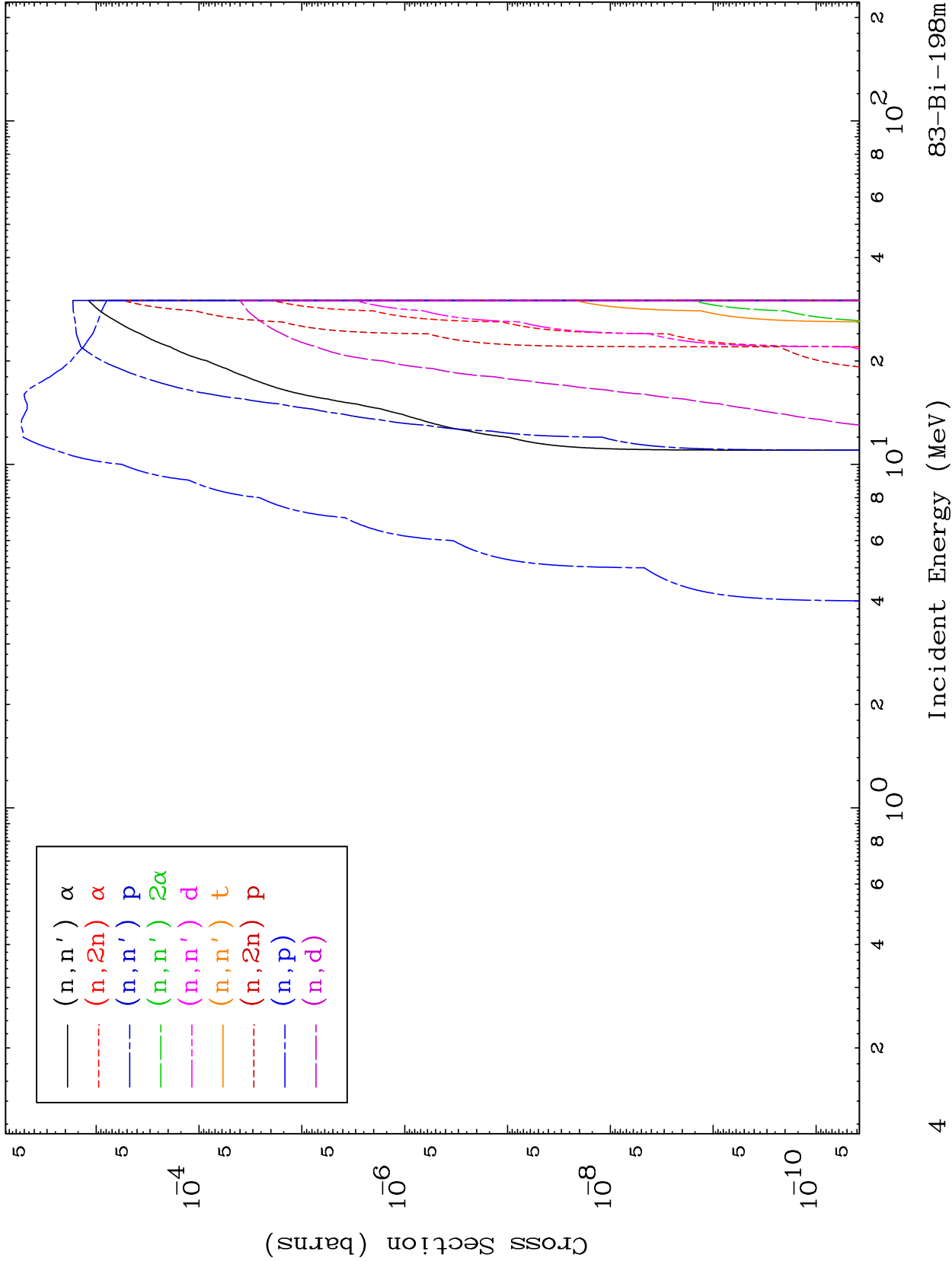
Incident Energy (MeV)

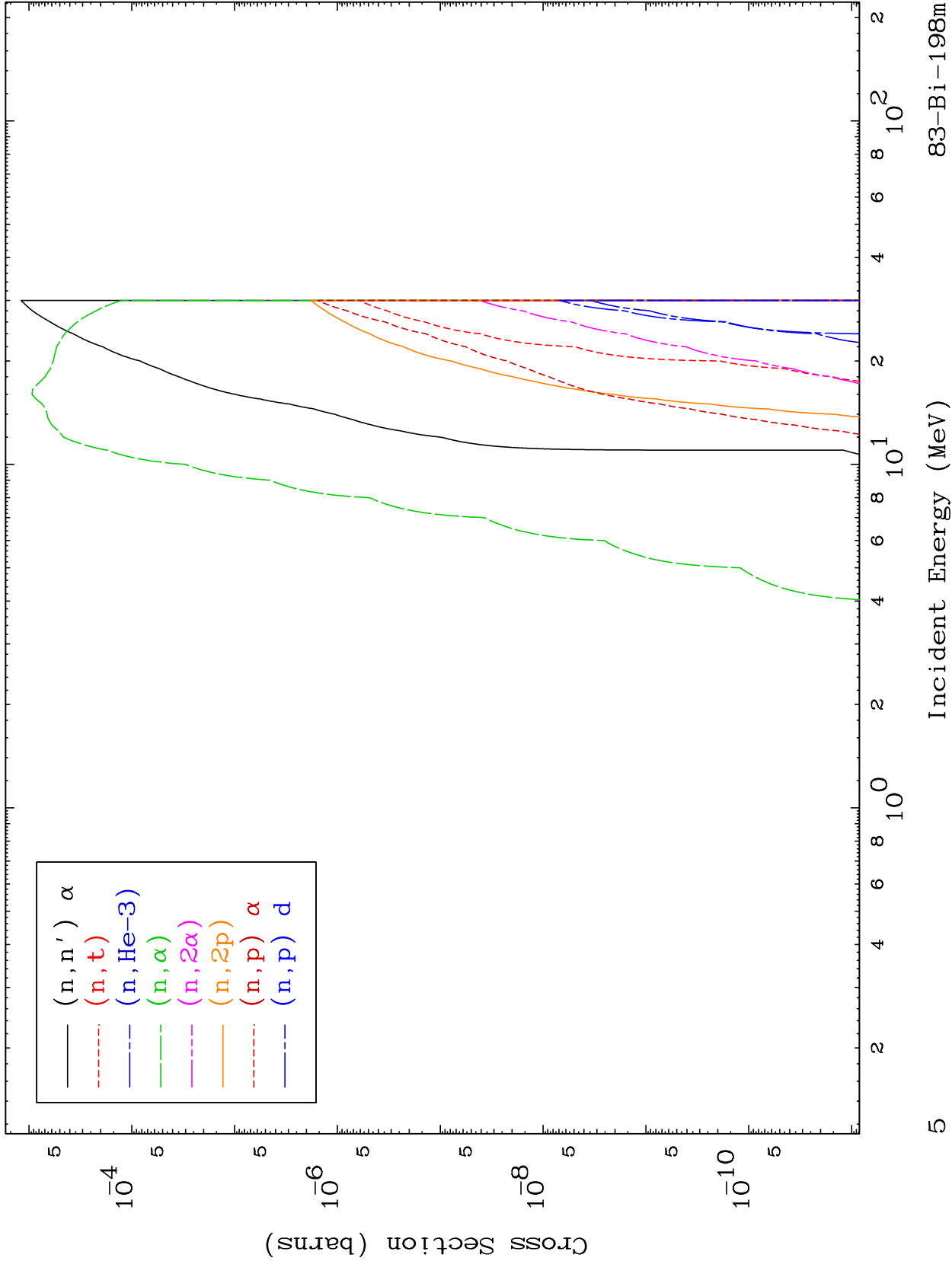


MAT 8293

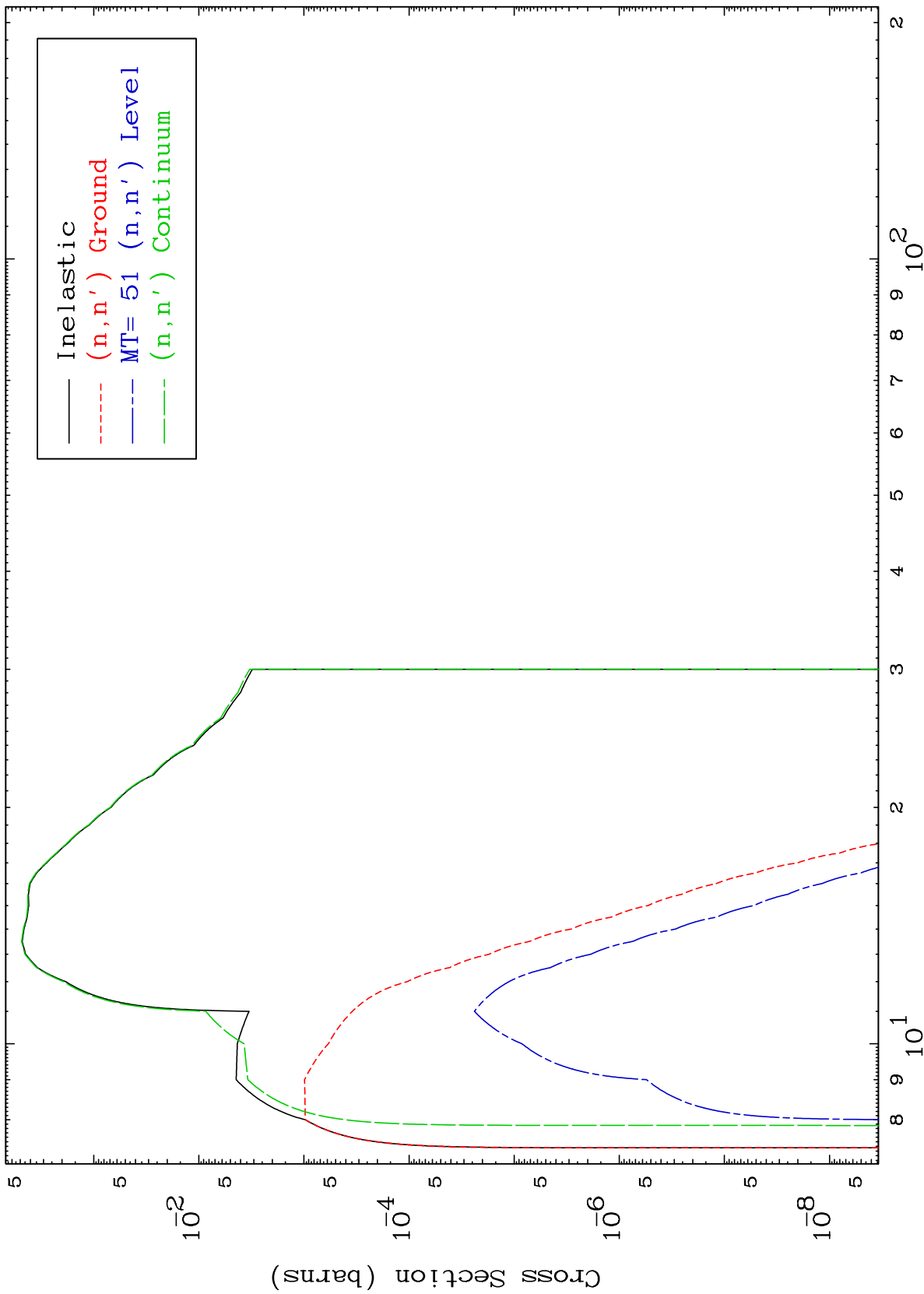
Photon Charged Particle  
0 Kelvin Cross Sections

83-Bi-198m





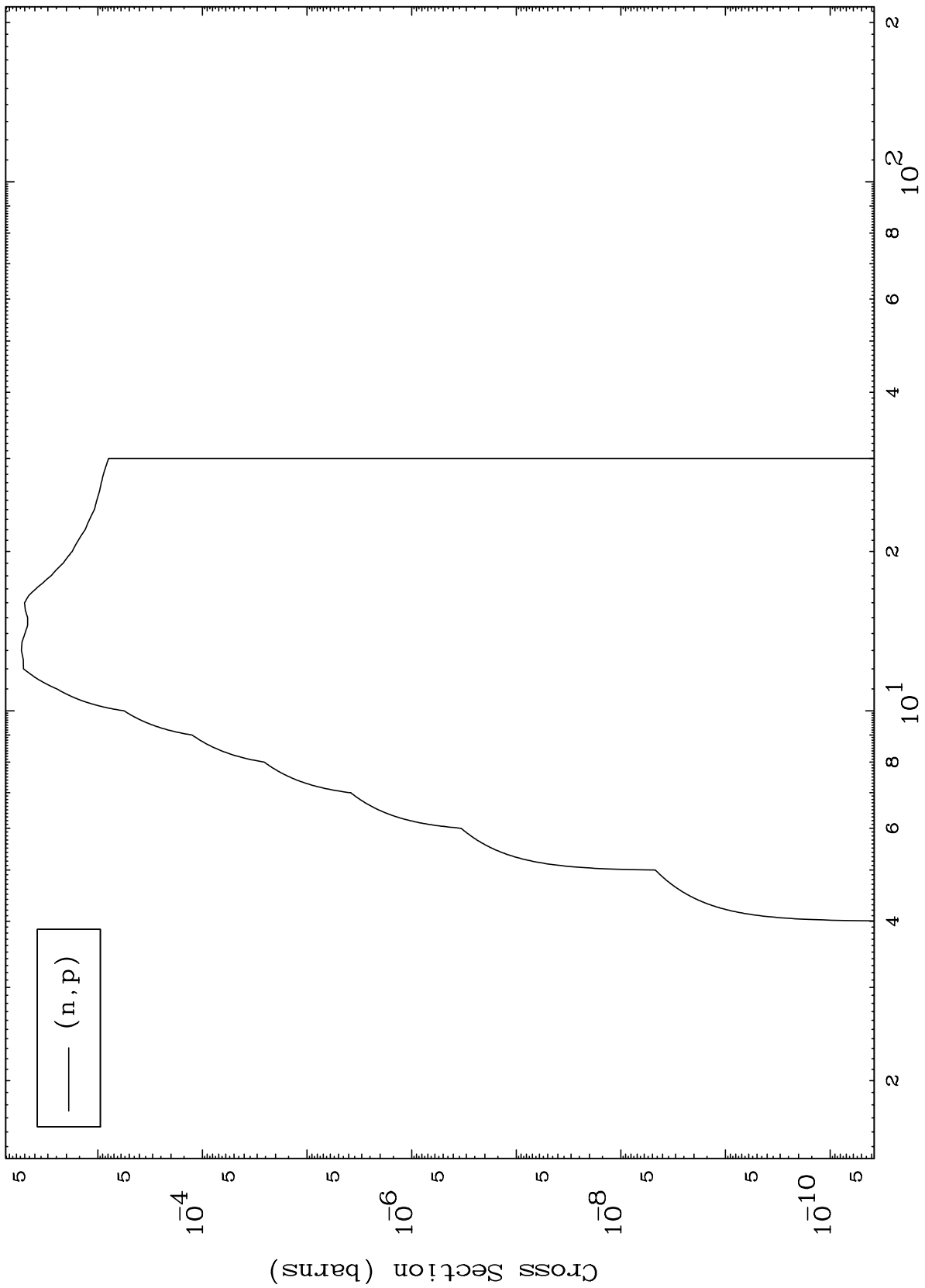
( $\gamma, n'$ ) Levels  
0 Kelvin Cross Sections



MAT 8293

83-Bi-198m

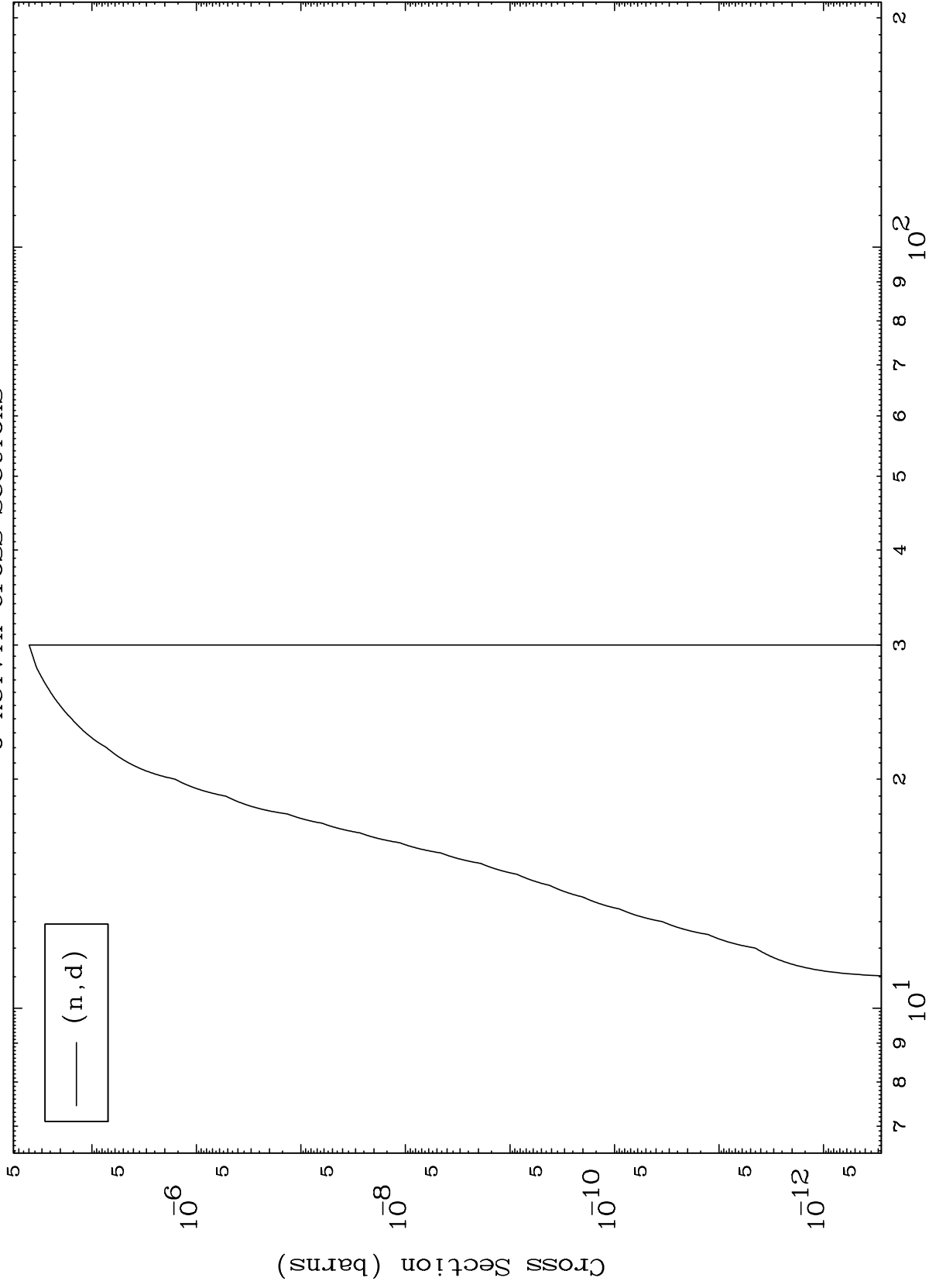
( $\gamma, p$ ) Levels  
0 Kelvin Cross Sections



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( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

83-Bi-198m



8

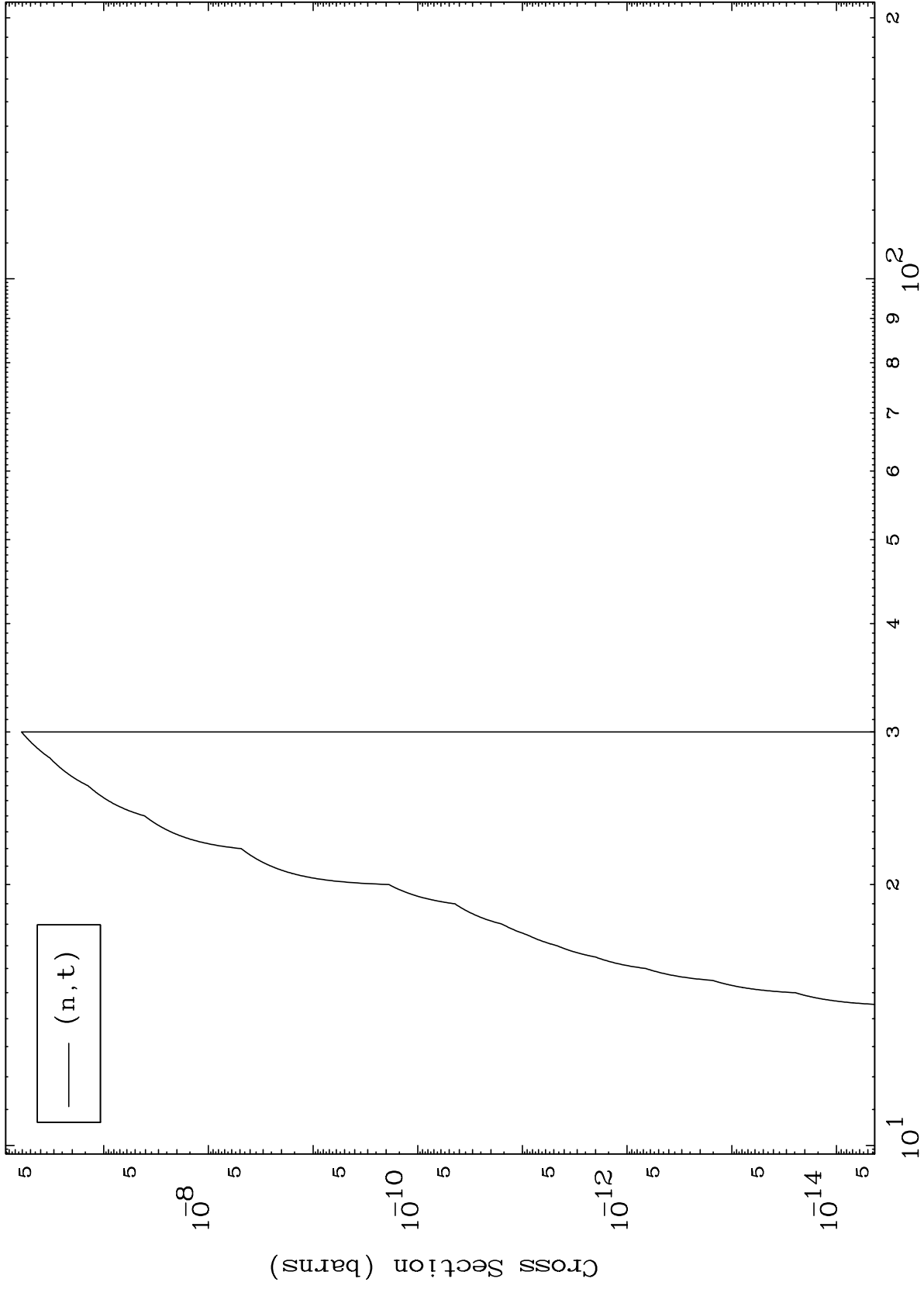
Incident Energy (MeV)

83-Bi-198m

MAT 8293

( $\gamma, t$ ) Levels  
0 Kelvin Cross Sections

83-Bi-198m



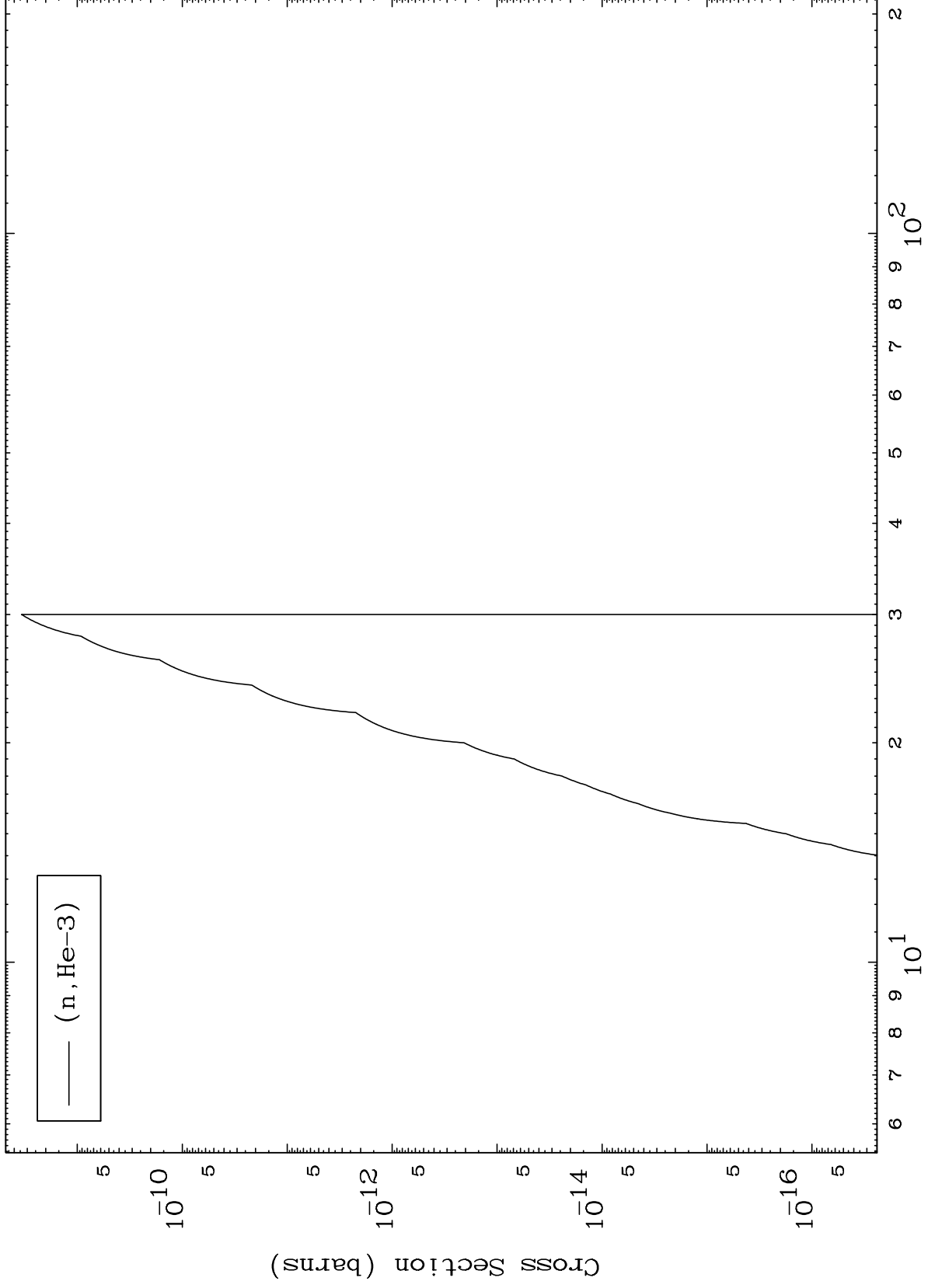
Incident Energy (MeV)

83-Bi-198m

MAT 8293

( $\gamma$ , He3) Levels  
0 Kelvin Cross Sections

83-Bi-198m



10

Incident Energy (MeV)

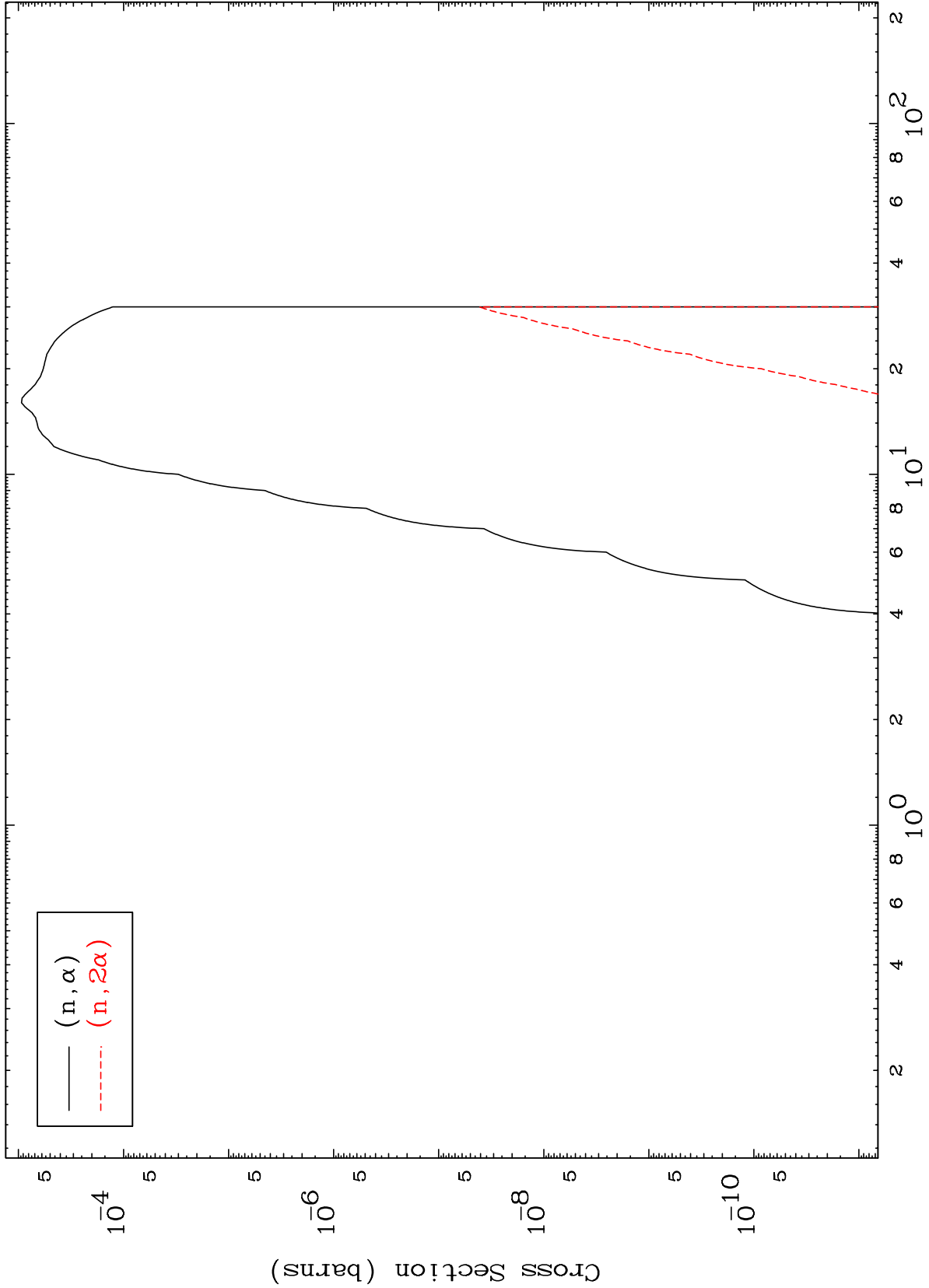
83-Bi-198m

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( $\gamma, \alpha$ ) Levels

$^{83}\text{Bi}-198\text{m}$

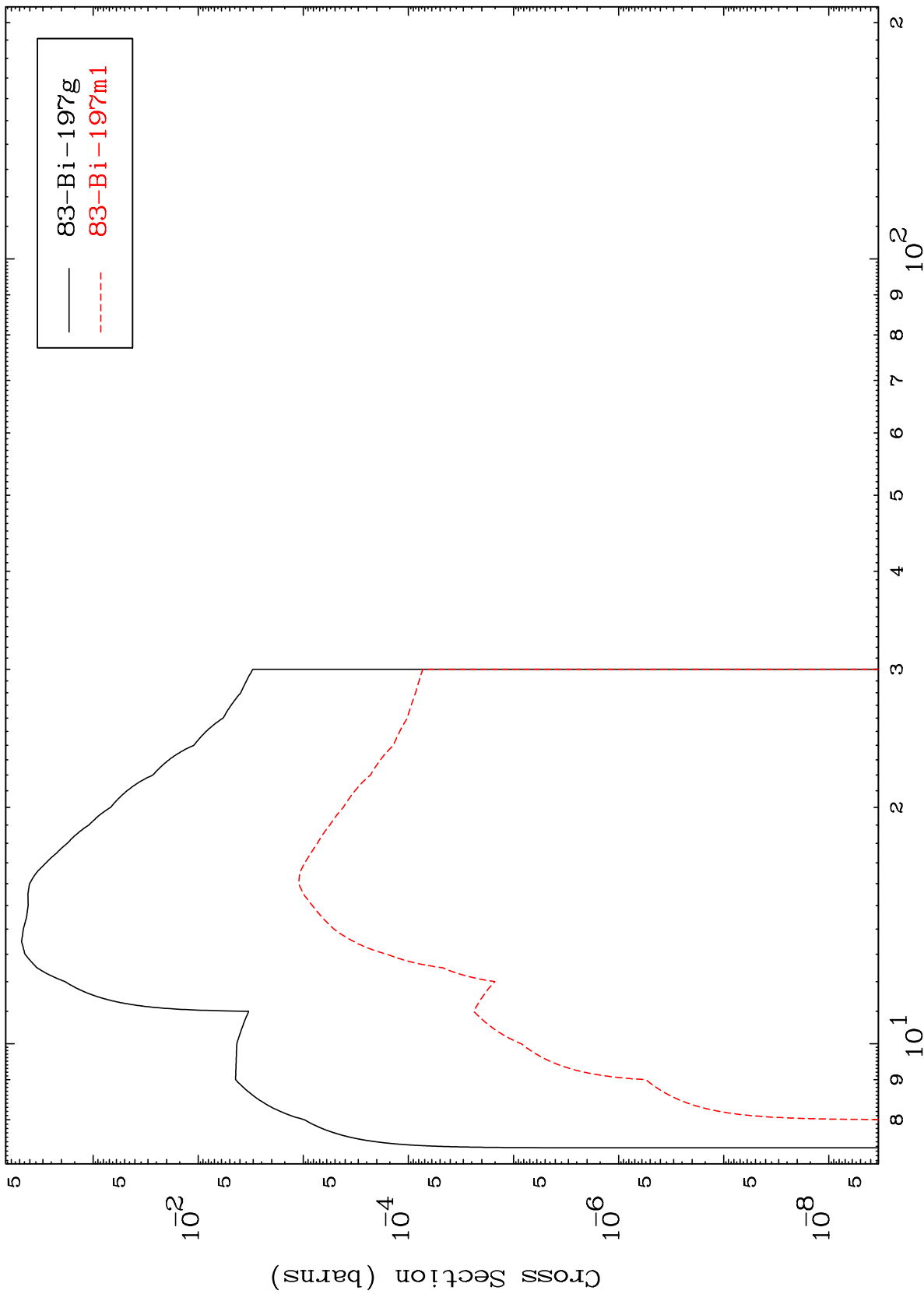
0 Kelvin Cross Sections



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83-Bi-198m

Inelastic  
Radionuclide Production Cross Section



12

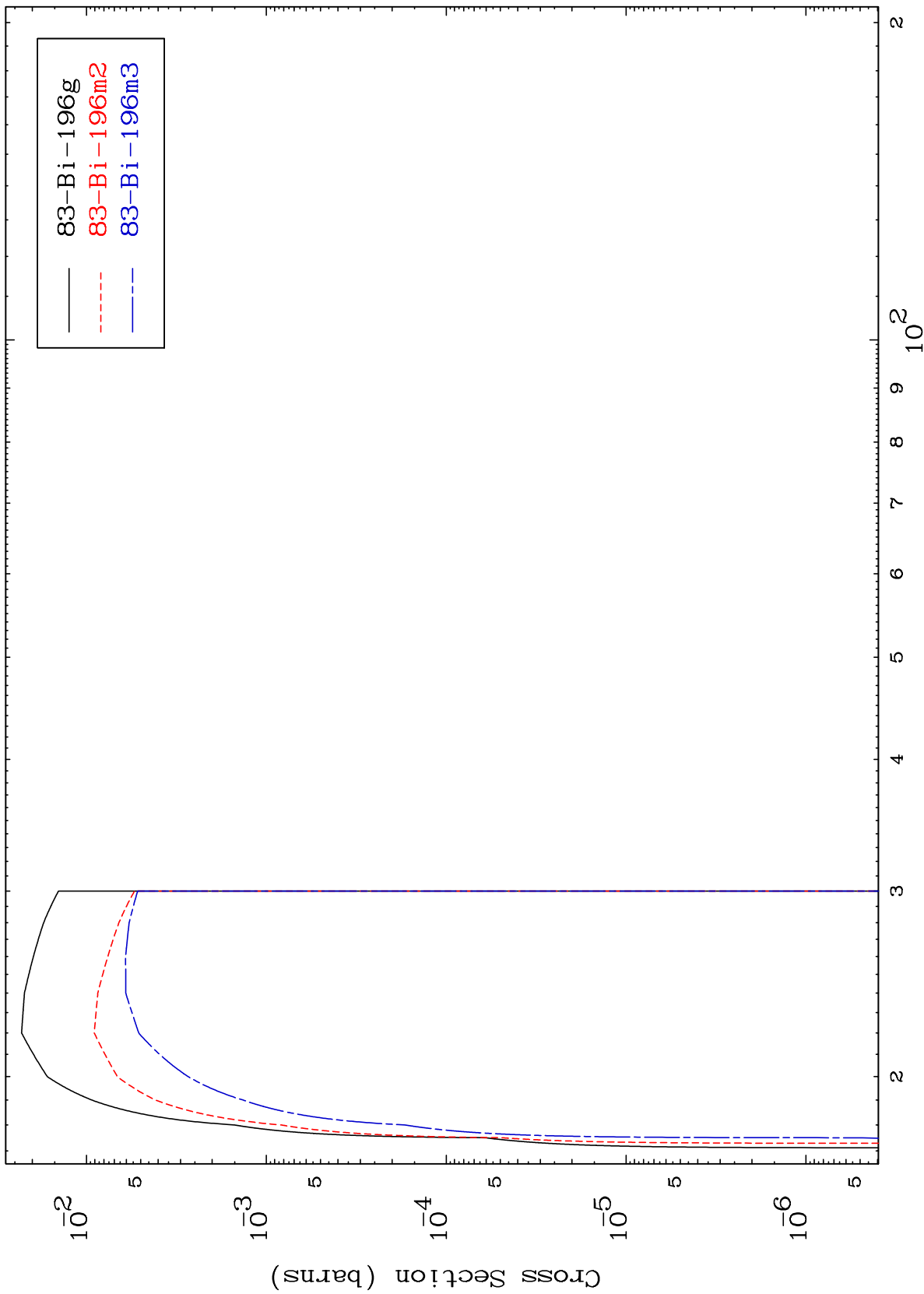
Incident Energy (MeV)

83-Bi-198m

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83-Bi-198m

(n,2n)  
Radionuclide Production Cross Section



83-Bi-198m

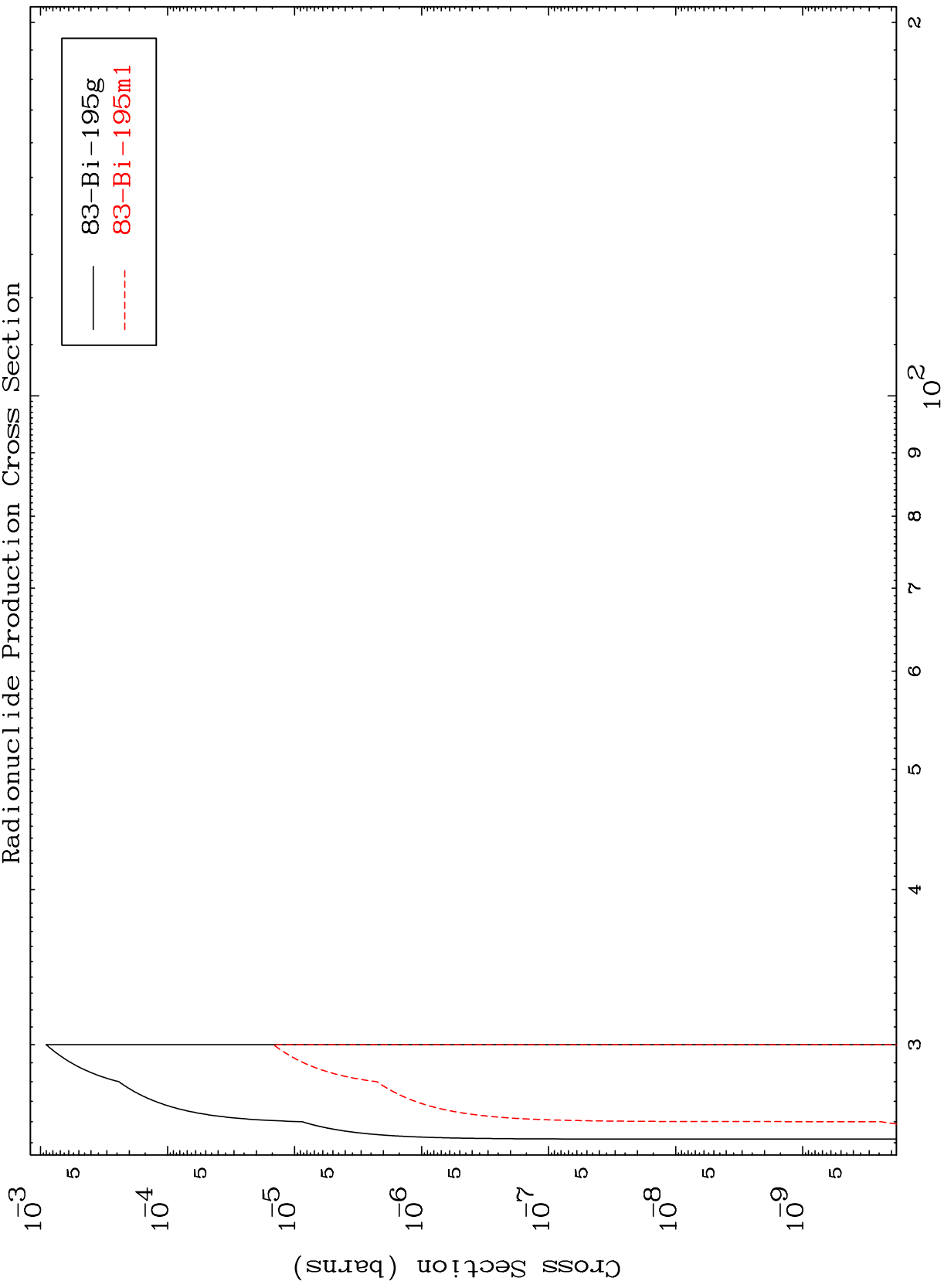
Incident Energy (MeV)

13

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83-Bi-198m

(n,3n)  
Radionuclide Production Cross Section



83-Bi-198m

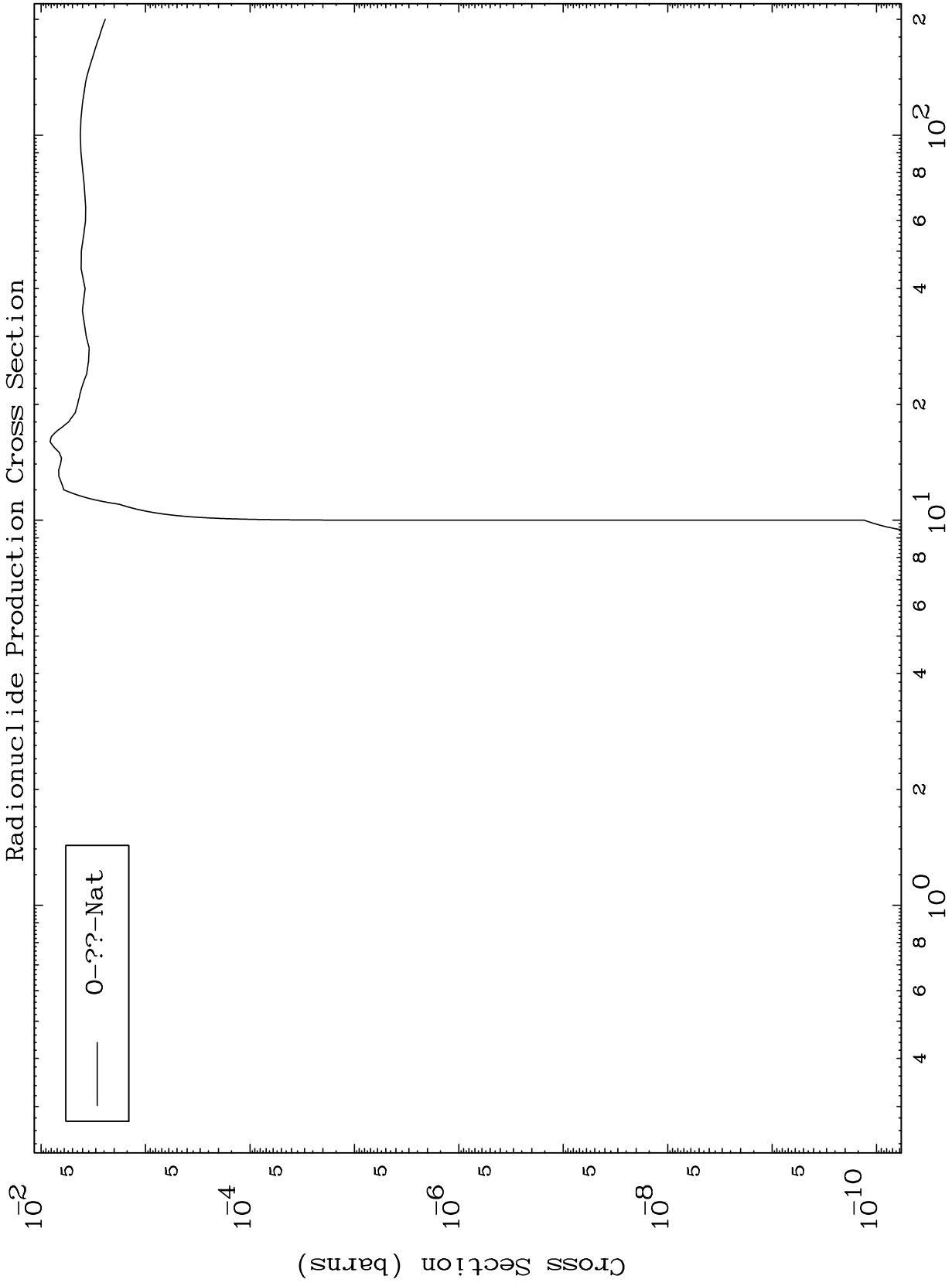
Incident Energy (MeV)

14

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Fission

83-Bi-198m

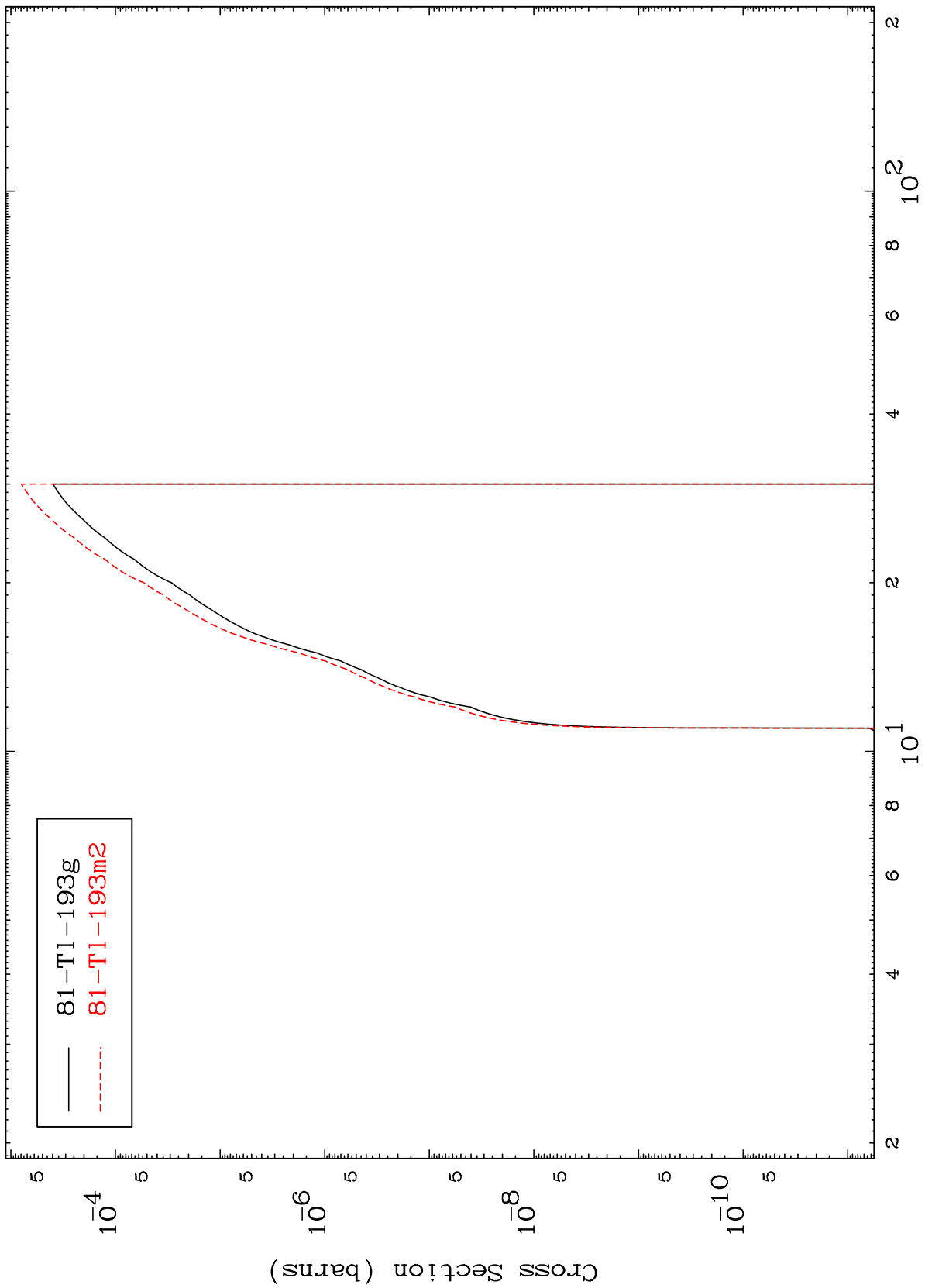


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$(n, n') \alpha$

$^{83}\text{Bi}-198\text{m}$

Radionuclide Production Cross Section



—  $^{81}\text{Tl}-193\text{g}$   
- - -  $^{81}\text{Tl}-193\text{m}2$

16

Incident Energy (MeV)

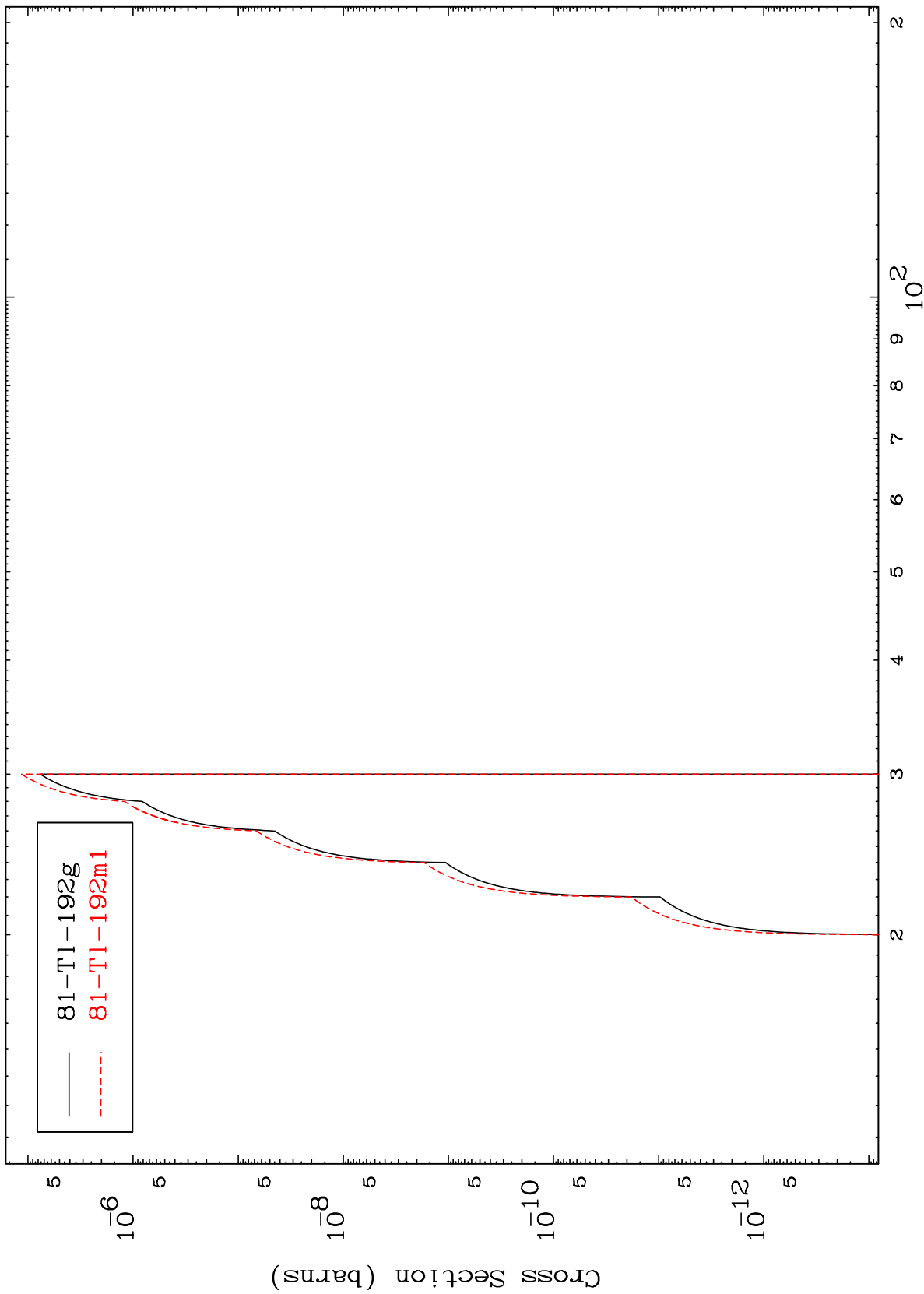
$^{83}\text{Bi}-198\text{m}$

MAT 8293

$(n,2n) \alpha$

83-Bi-198m

Radionuclide Production Cross Section

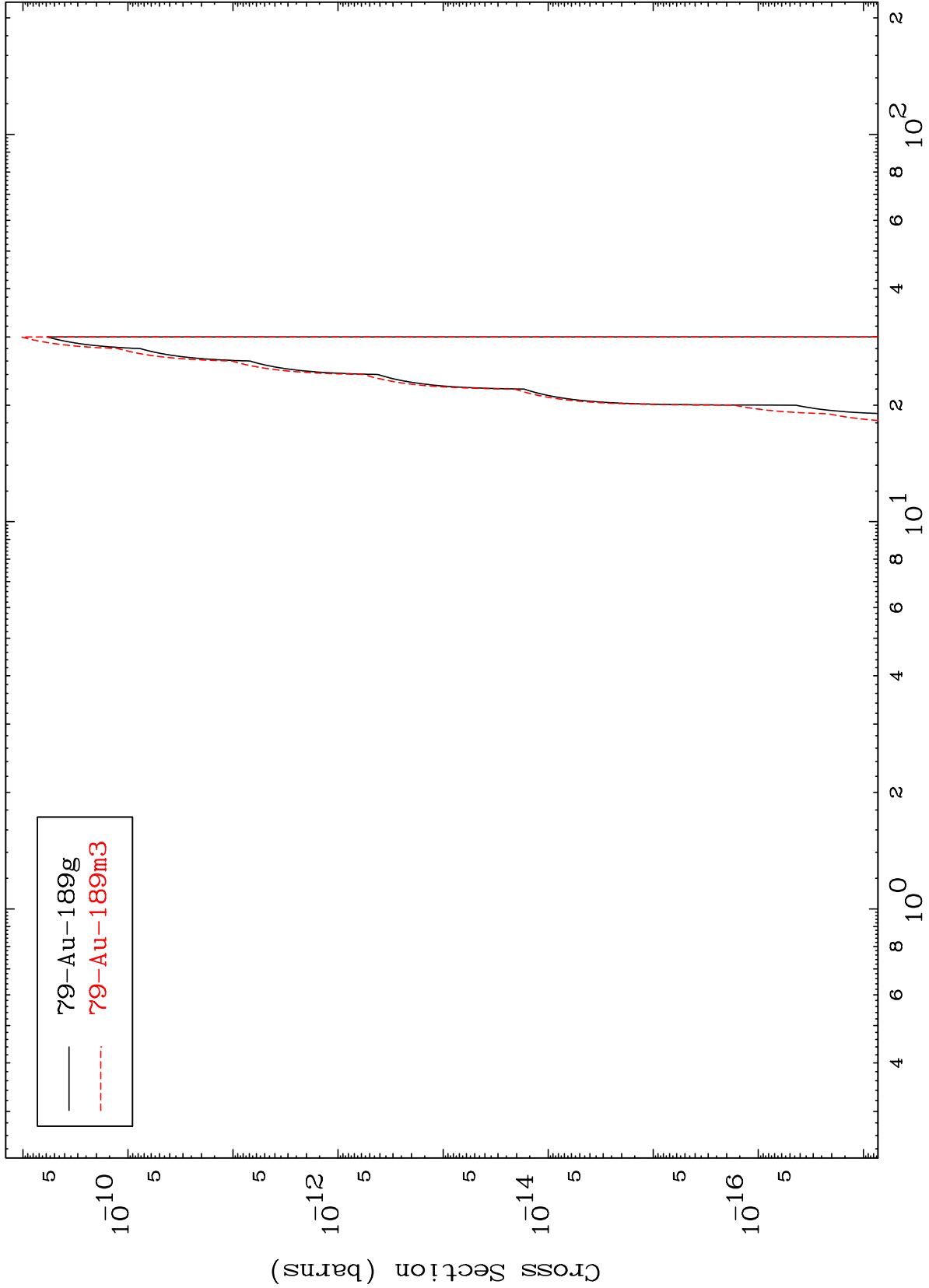


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(n,n') 2α

83-Bi-198m

Radionuclide Production Cross Section

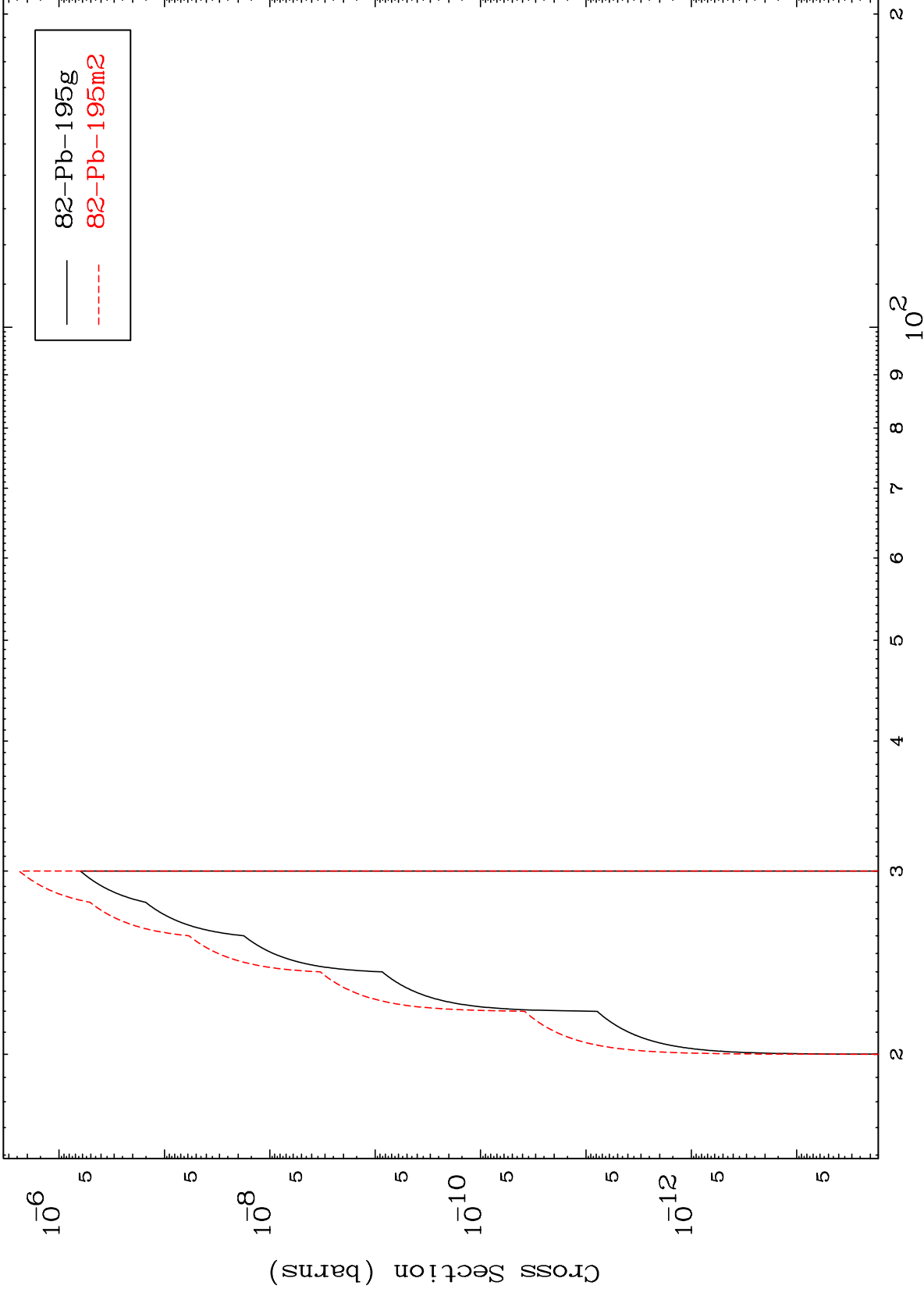


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(n,n') d

83-Bi-198m

Radionuclide Production Cross Section



19

Incident Energy (MeV)

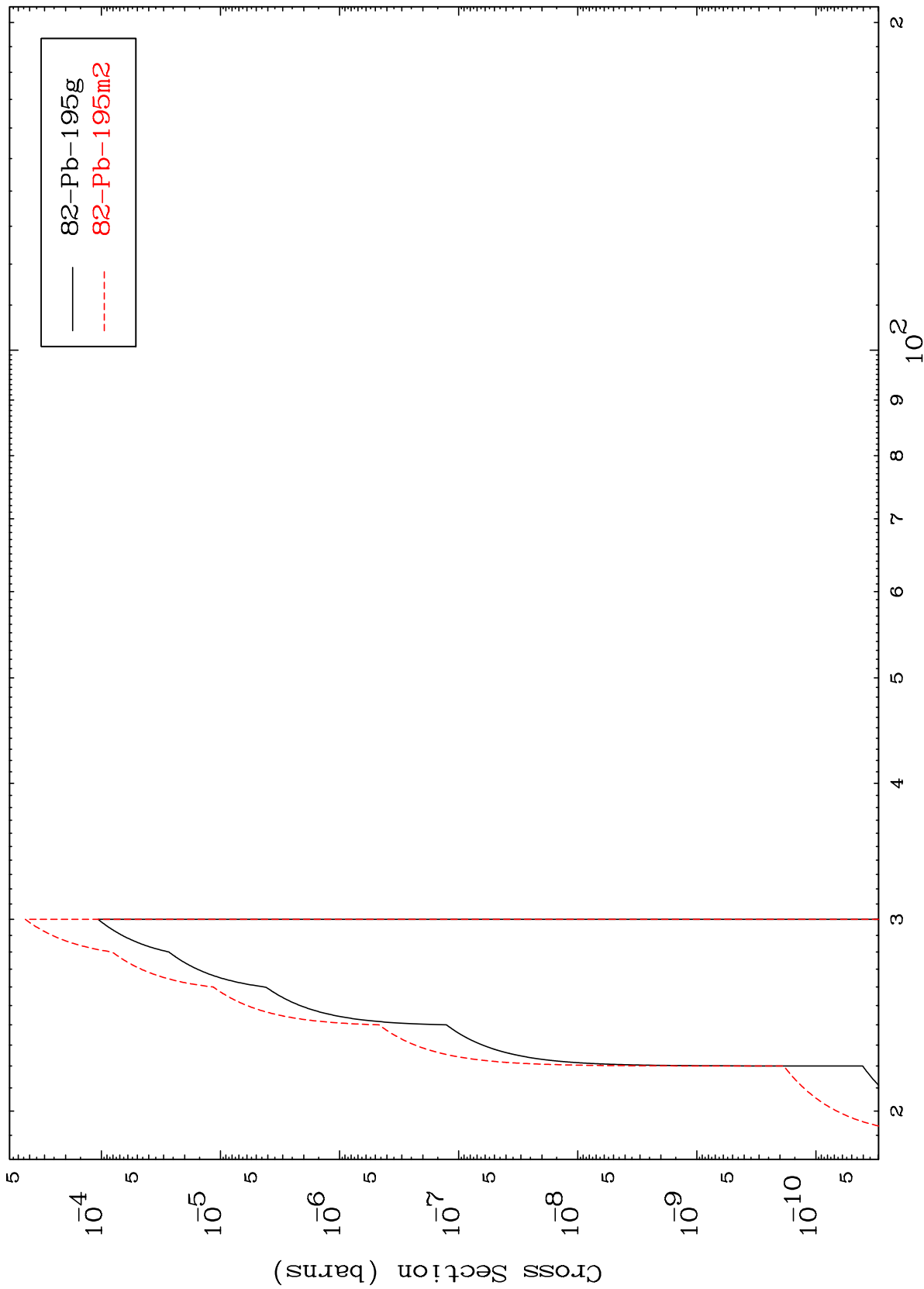
83-Bi-198m

MAT 8293

(n,2n) p

83-Bi-198m

Radionuclide Production Cross Section



82-Pb-195g  
82-Pb-195m2

20

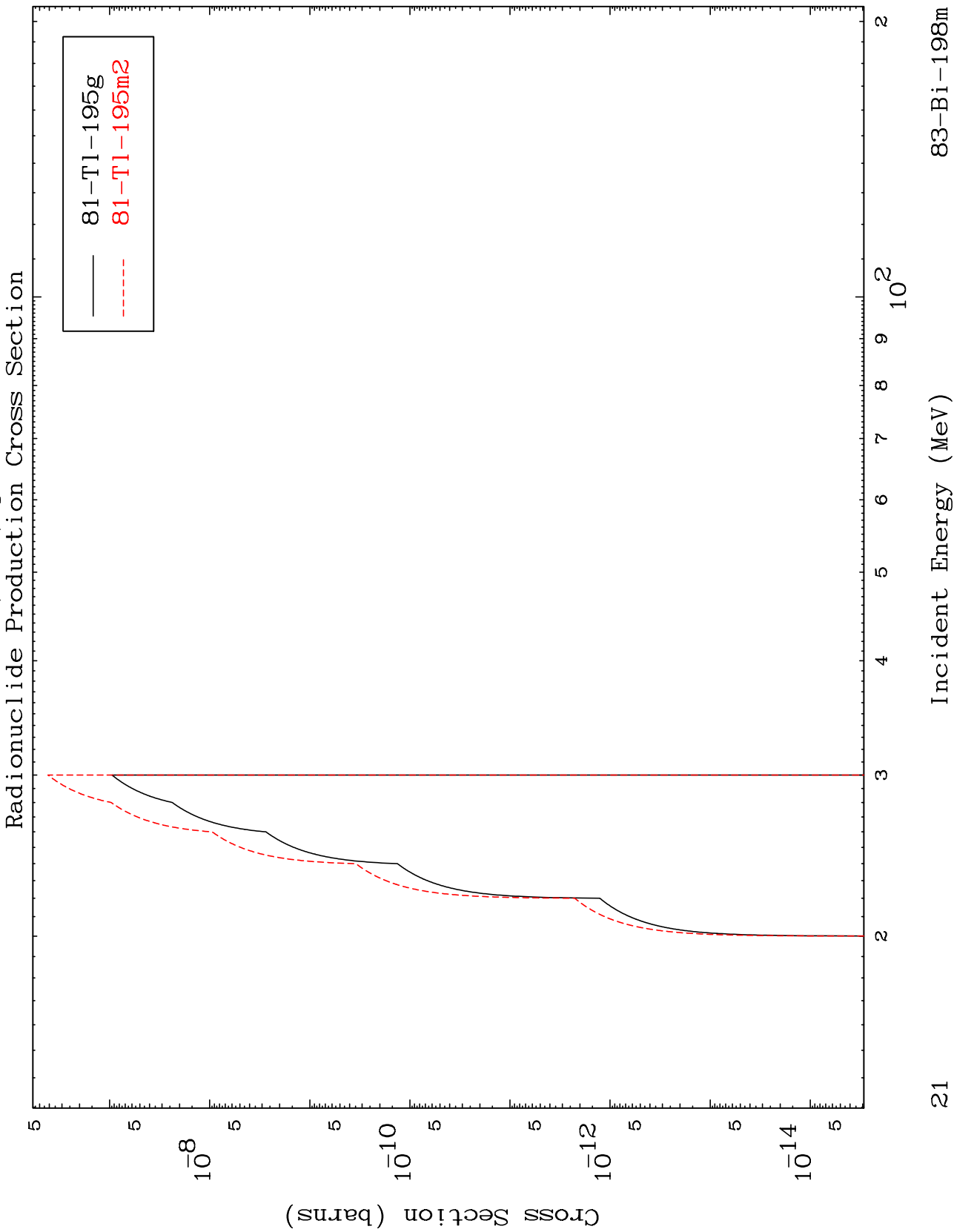
Incident Energy (MeV)

83-Bi-198m

MAT 8293

(n,2n) p

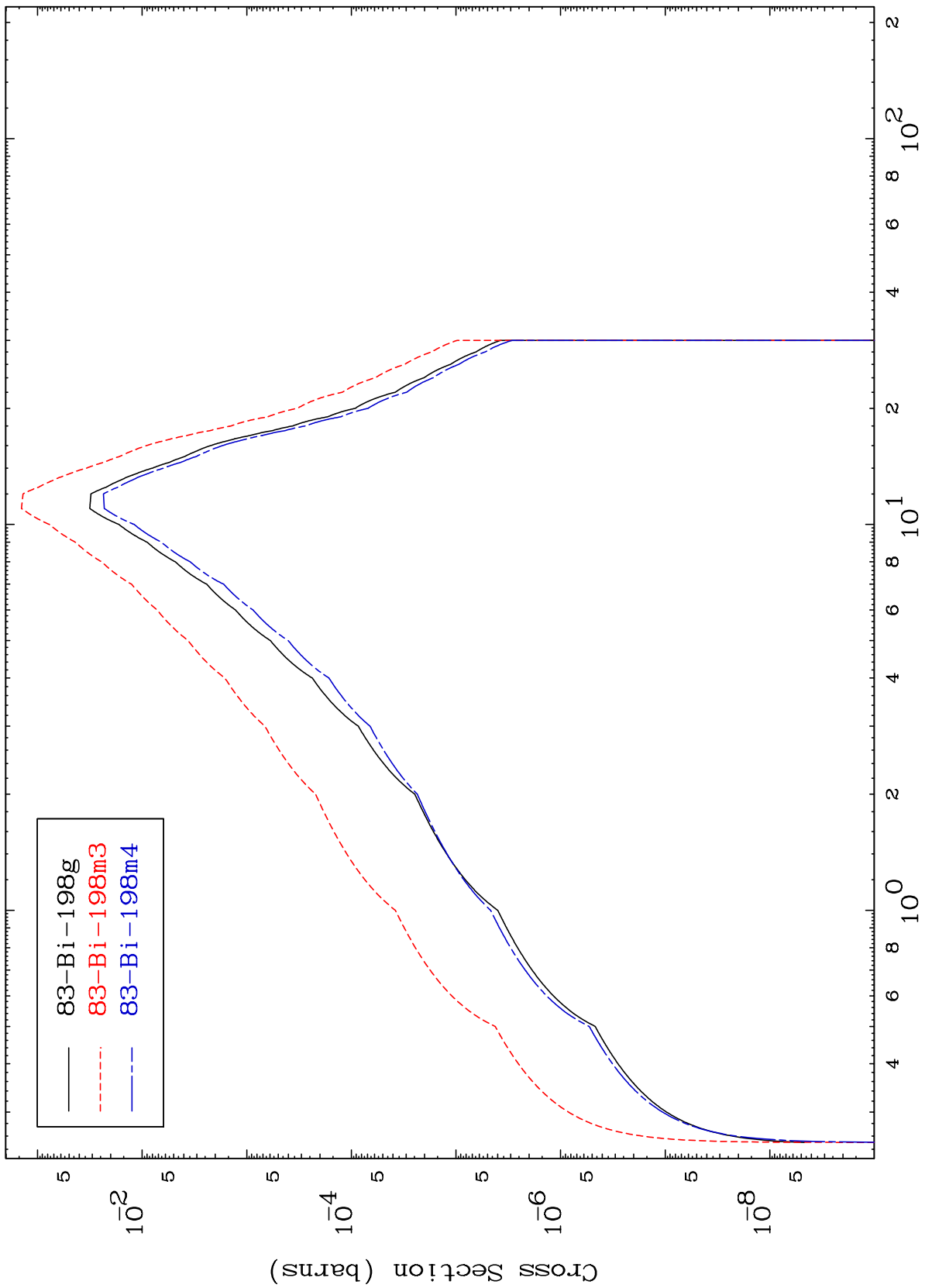
83-Bi-198m



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$^{83}\text{Bi}-198\text{m}$

Radionuclide Production Cross Section  
( $n, \gamma$ )



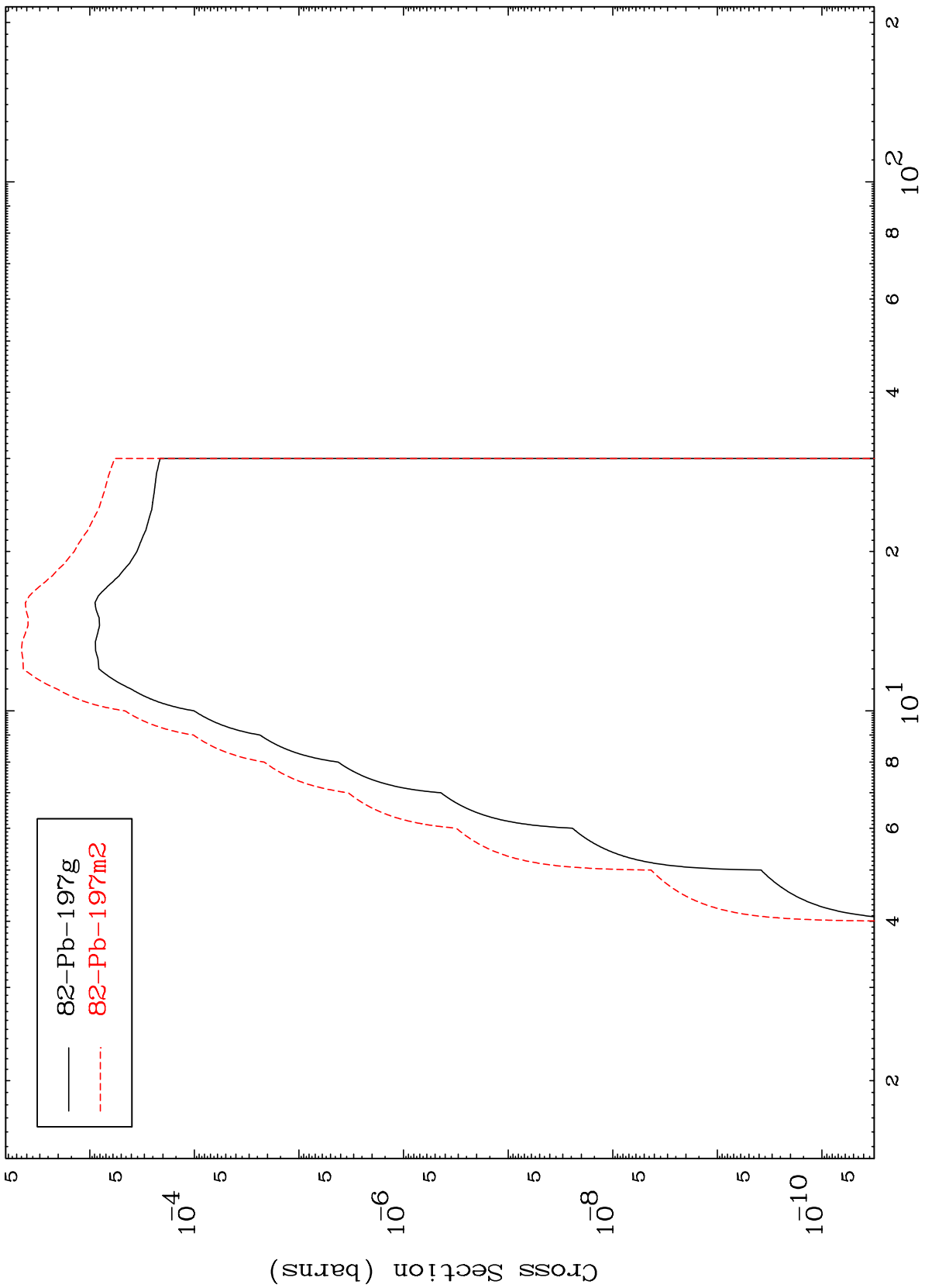
$^{83}\text{Bi}-198\text{m}$

Incident Energy (MeV)

MAT 8293

83-Bi-198m

(n,p)  
Radionuclide Production Cross Section



— 82-Pb-197g  
- - - 82-Pb-197m2

23

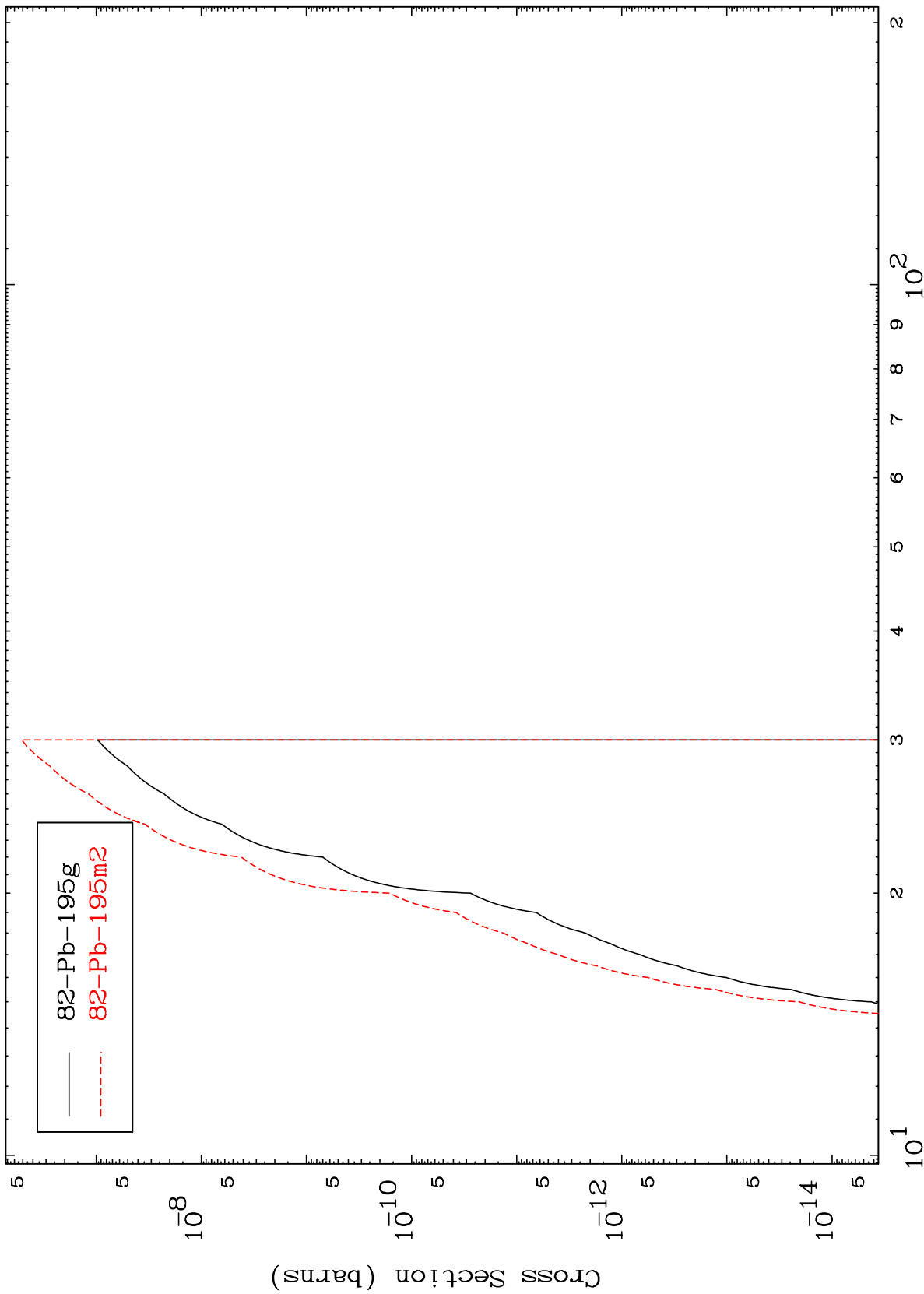
83-Bi-198m

Incident Energy (MeV)

MAT 8293

83-Bi-198m

(n,t)  
Radionuclide Production Cross Section



83-Bi-198m

Incident Energy (MeV)

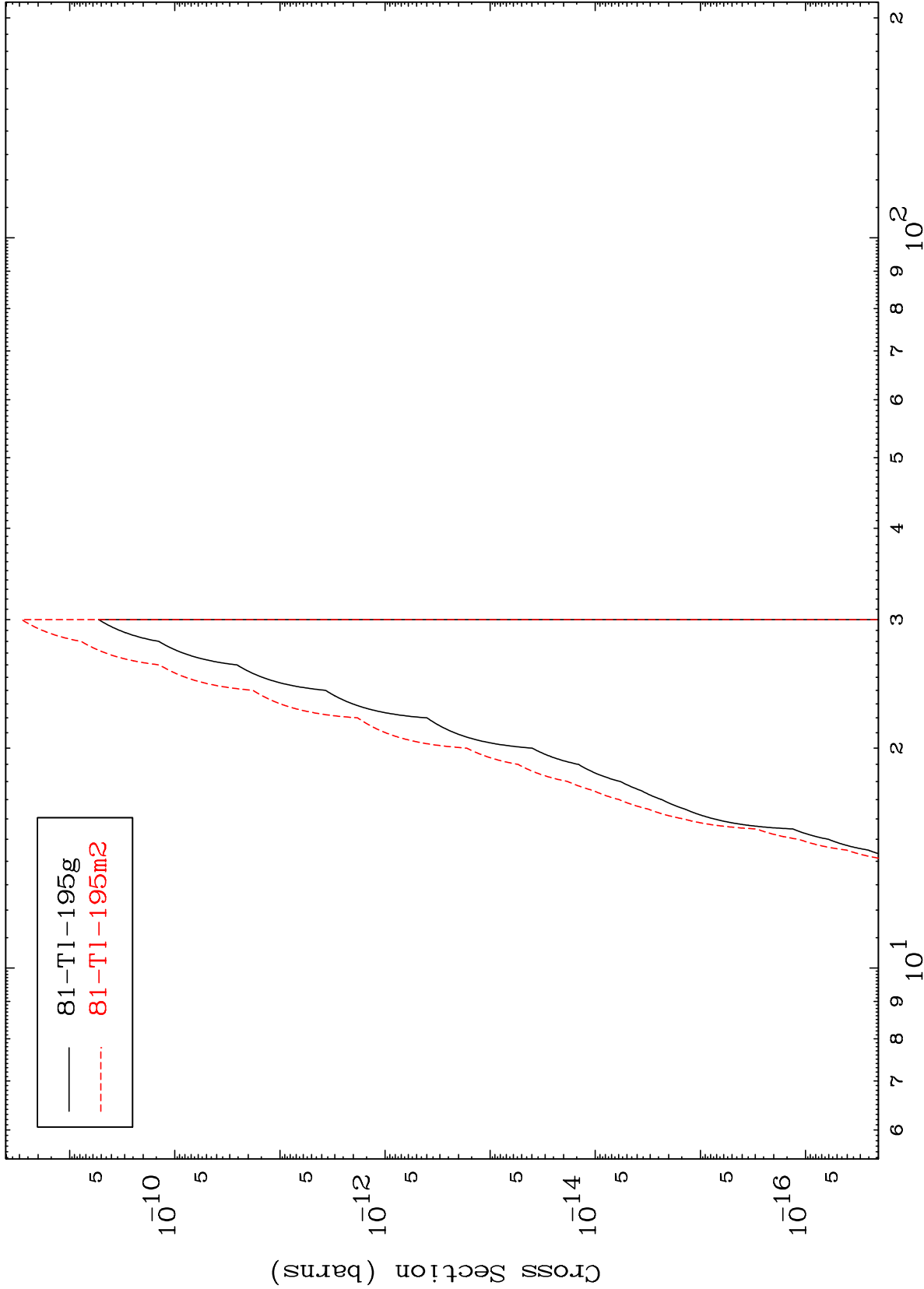
24

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(n,He-3)

83-Bi-198m

Radionuclide Production Cross Section



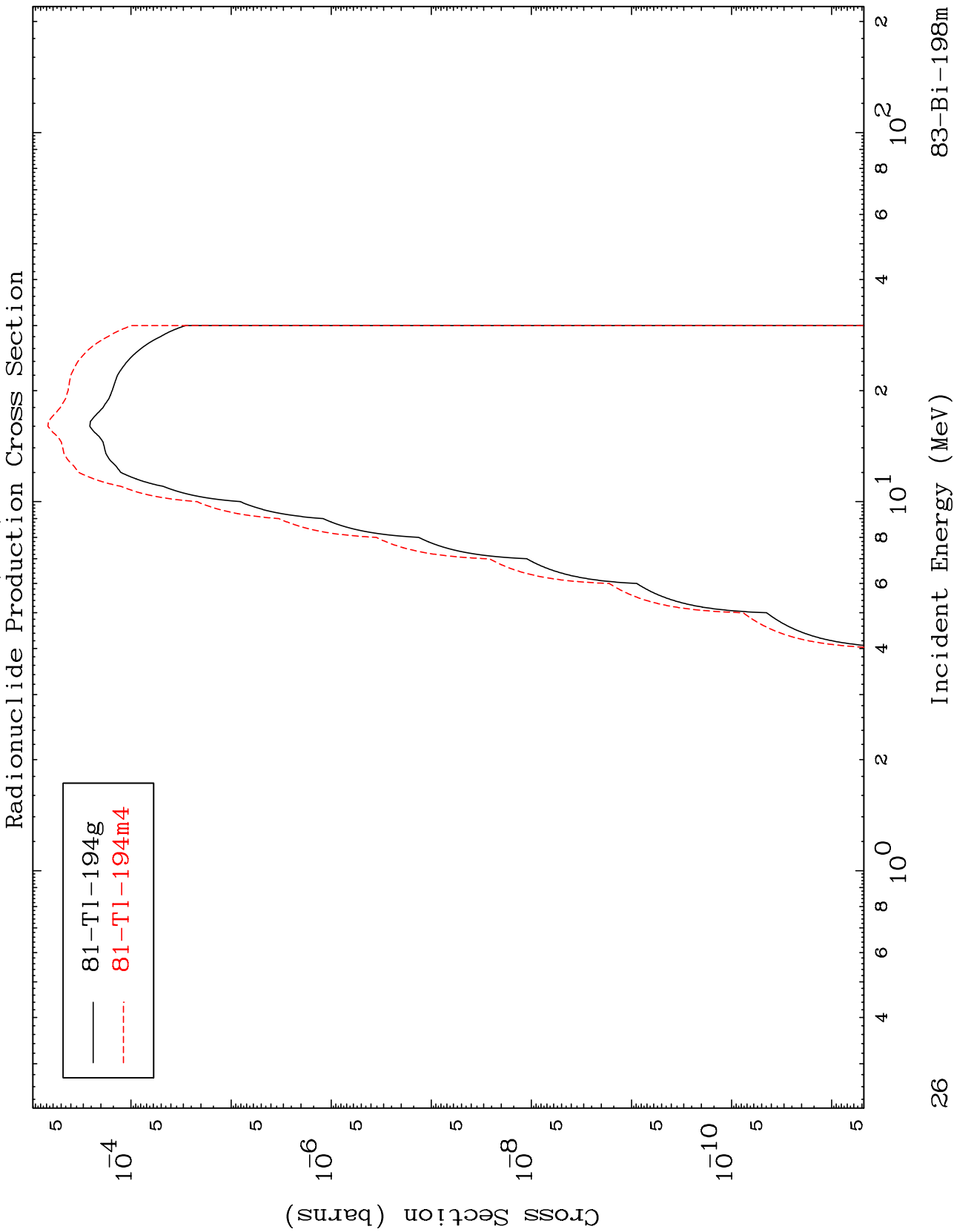
25

Incident Energy (MeV)

83-Bi-198m

MAT 8293

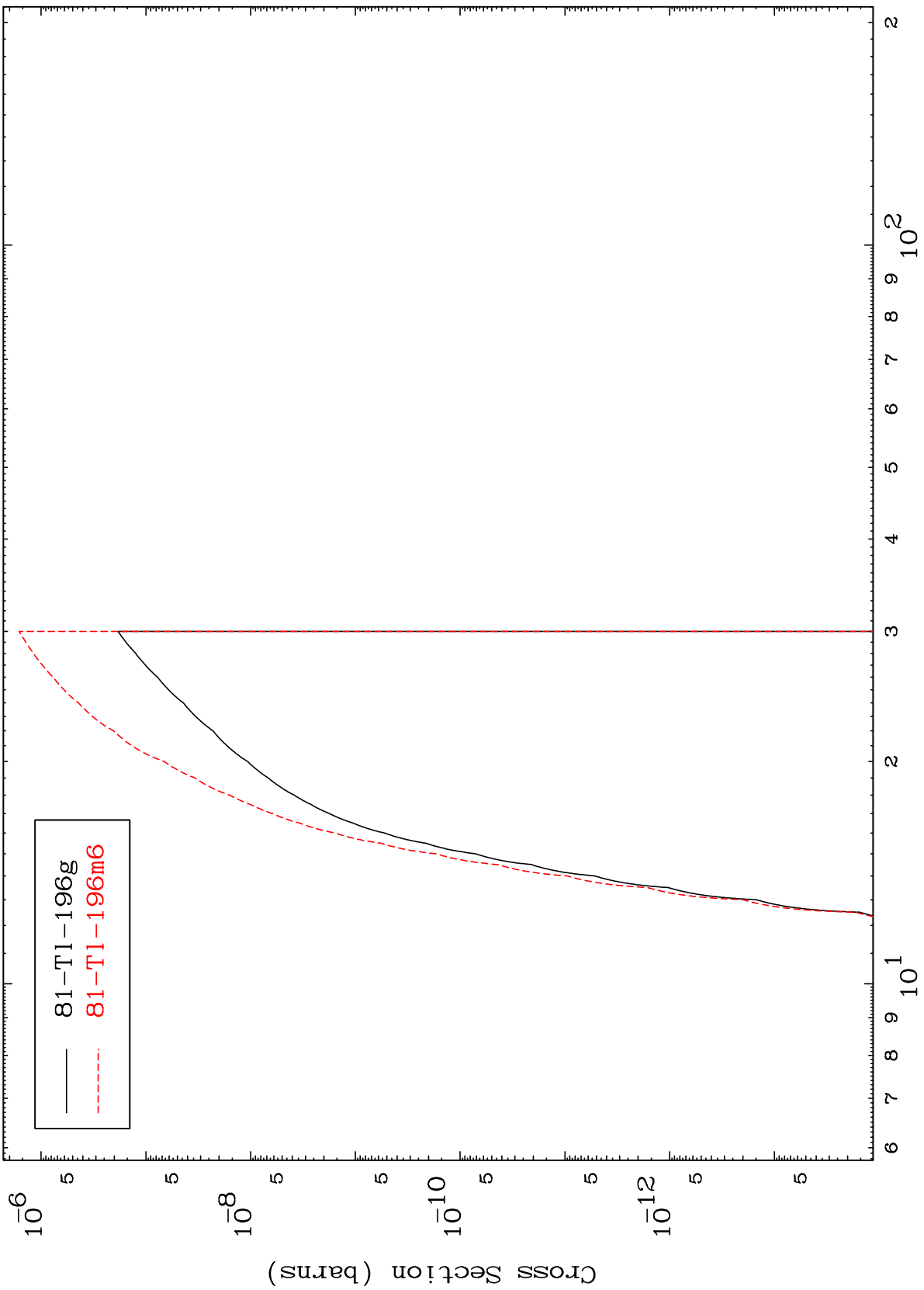
<sup>83</sup>Bi-198m



MAT 8293

83-Bi-198m

(n,2p)  
Radionuclide Production Cross Section



27

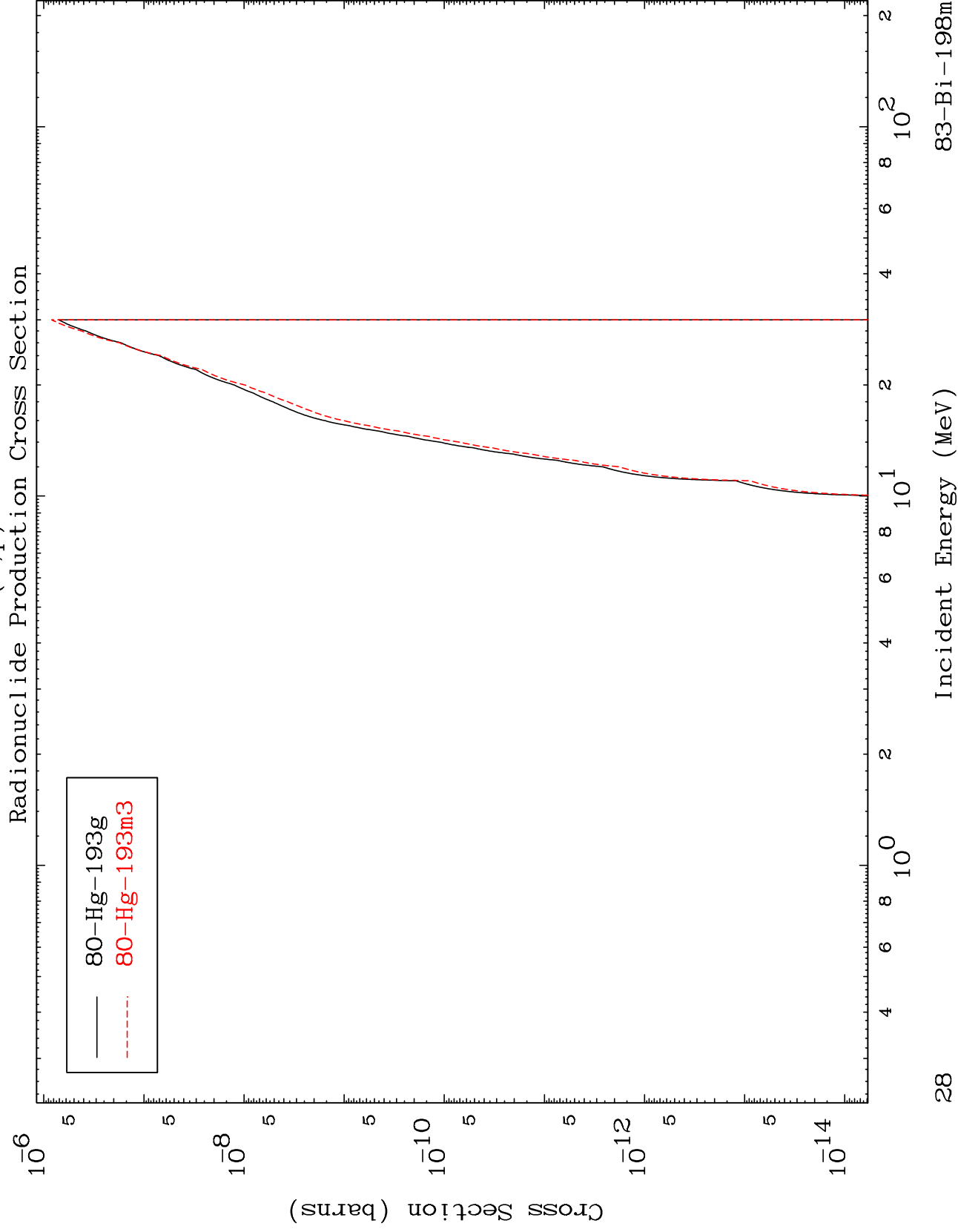
Incident Energy (MeV)

83-Bi-198m

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(n,p)  $\alpha$

$^{83}\text{Bi-198m}$

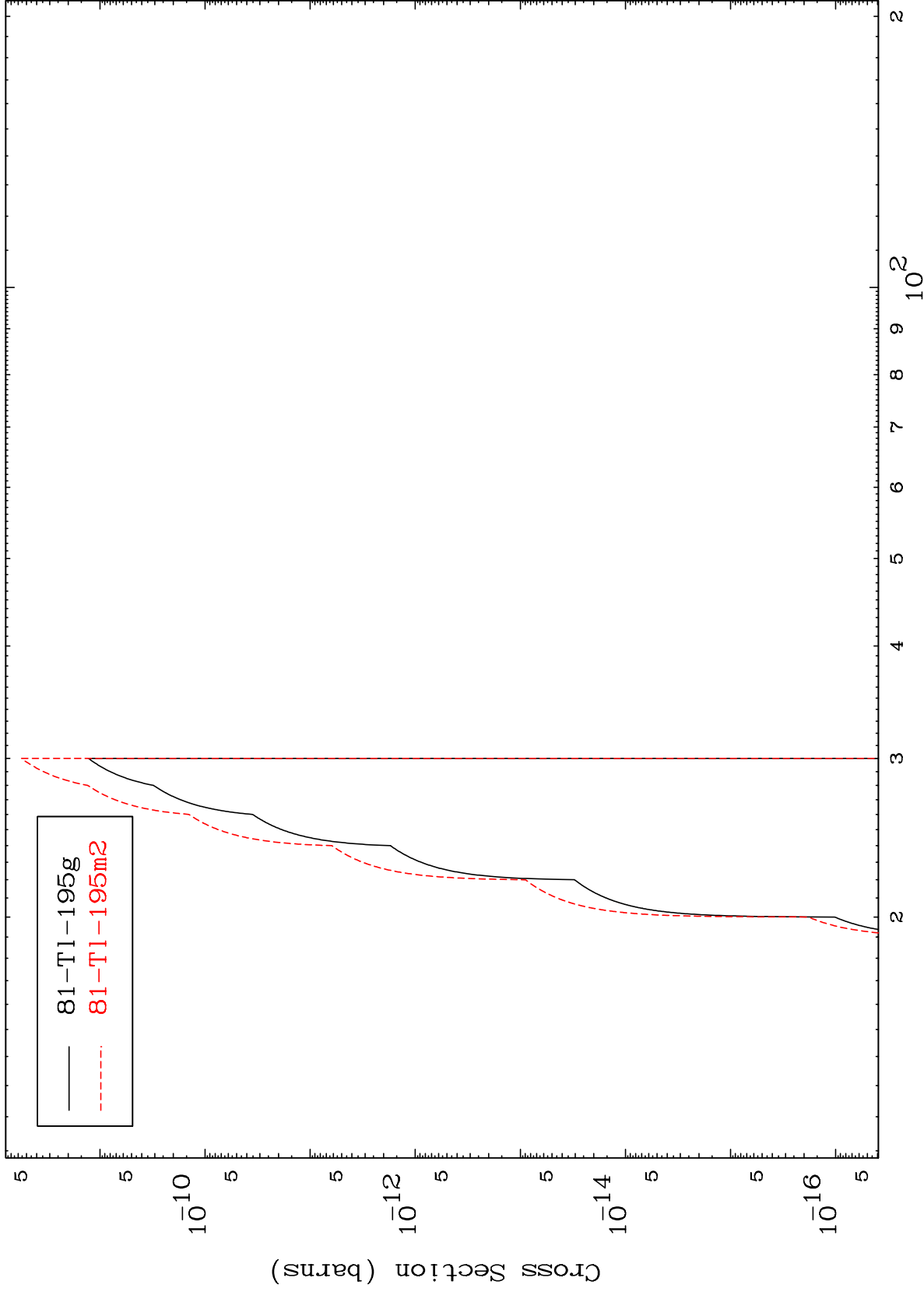


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(n,p) d

83-Bi-198m

Radionuclide Production Cross Section



29

Incident Energy (MeV)

83-Bi-198m