

Program EVALPLOT  
(Version 2021-1)

by

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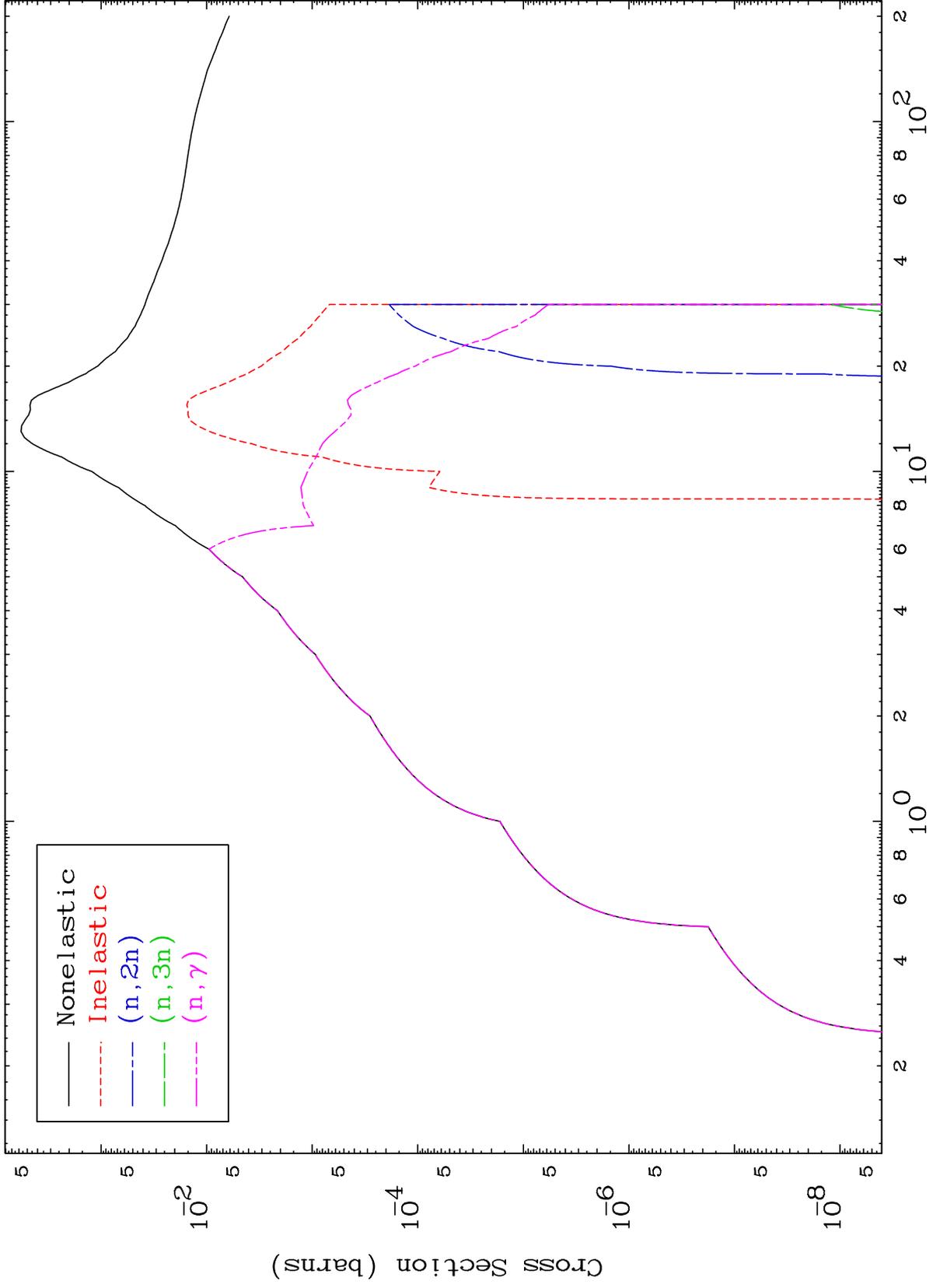
Press Mouse Button to Start

MAT 8701

Photon Major

87-Fr-204

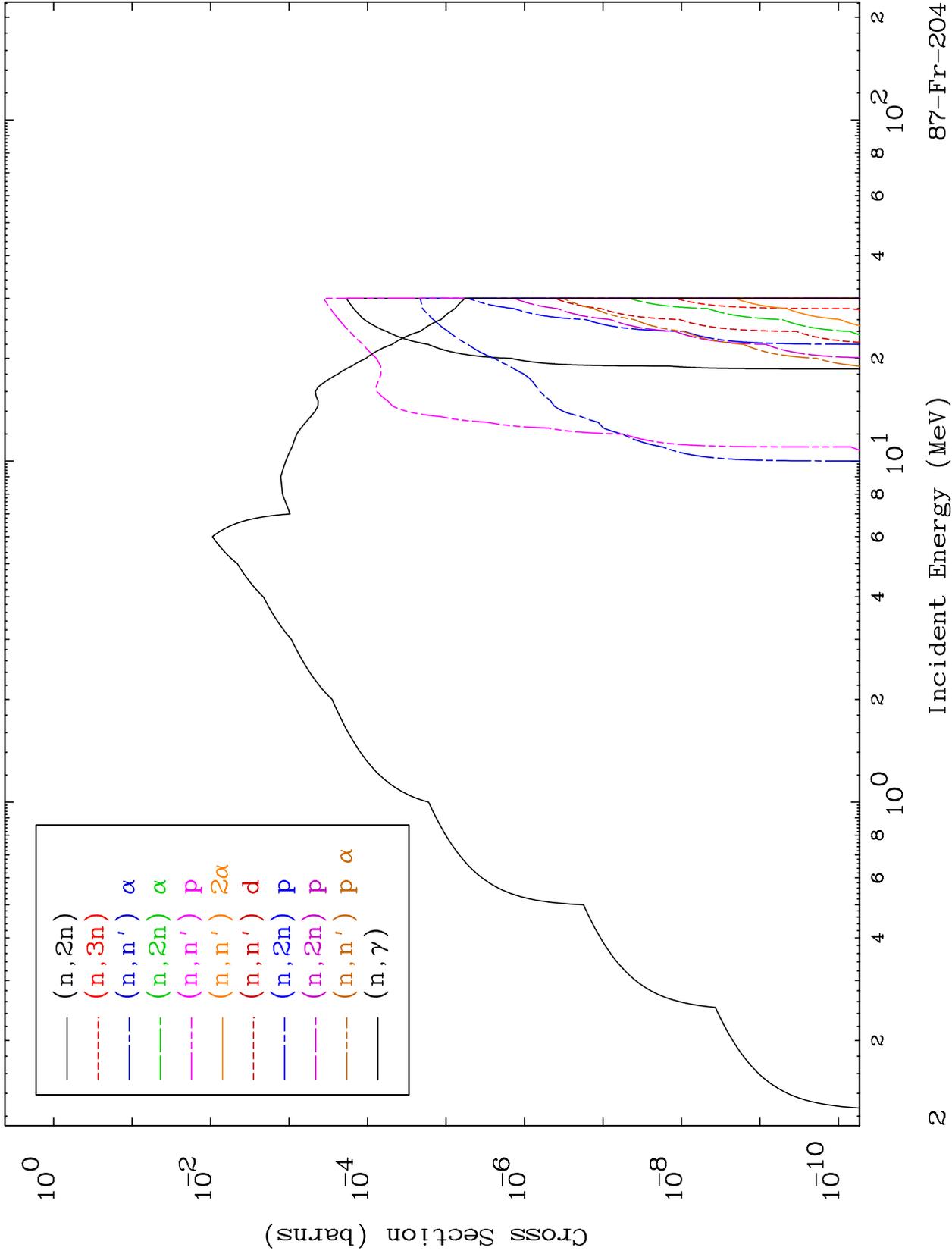
0 Kelvin Cross Sections



MAT 8701

Photon Neutron Absorption  
0 Kelvin Cross Sections

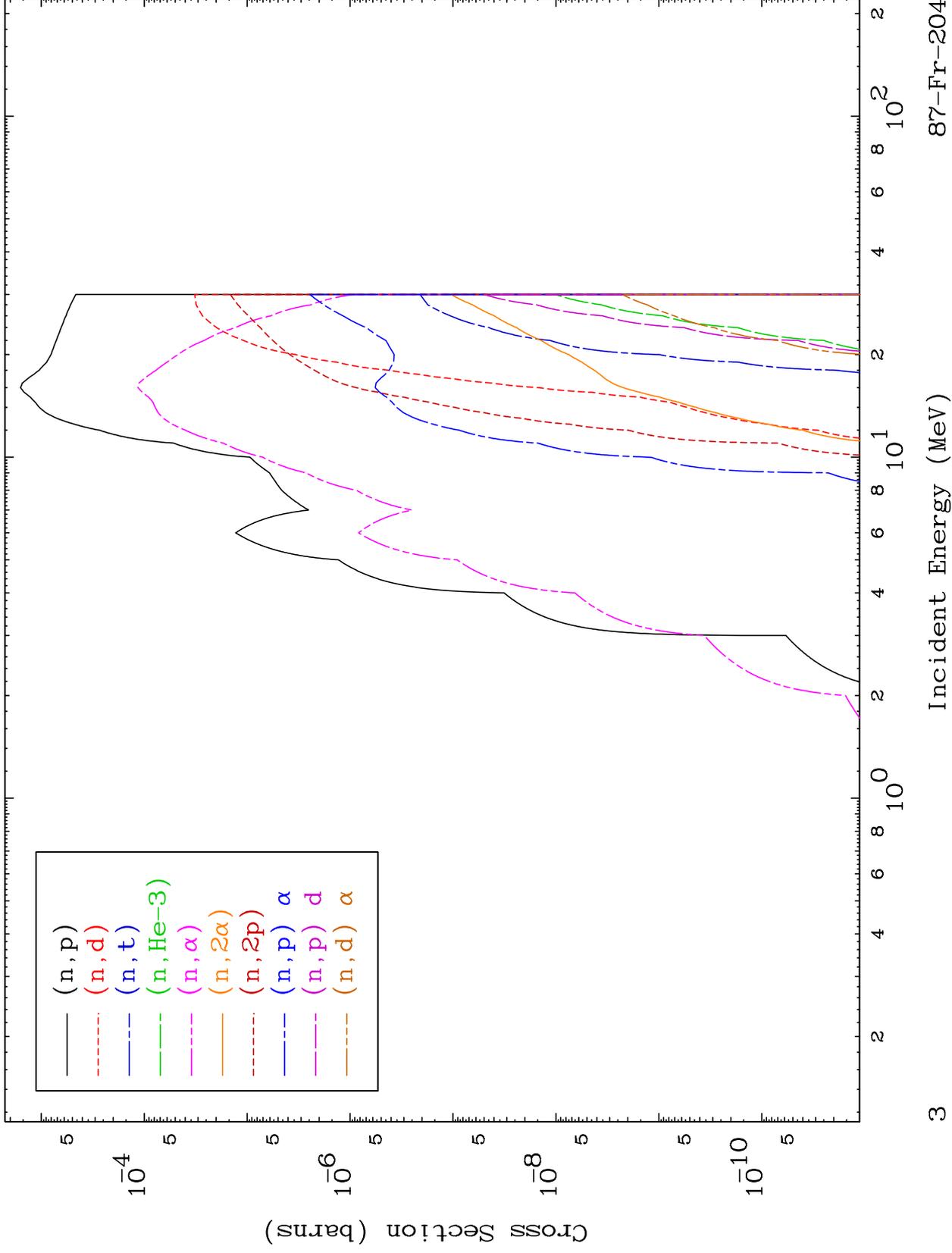
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MAT 8701

Photon Neutron Absorption  
0 Kelvin Cross Sections

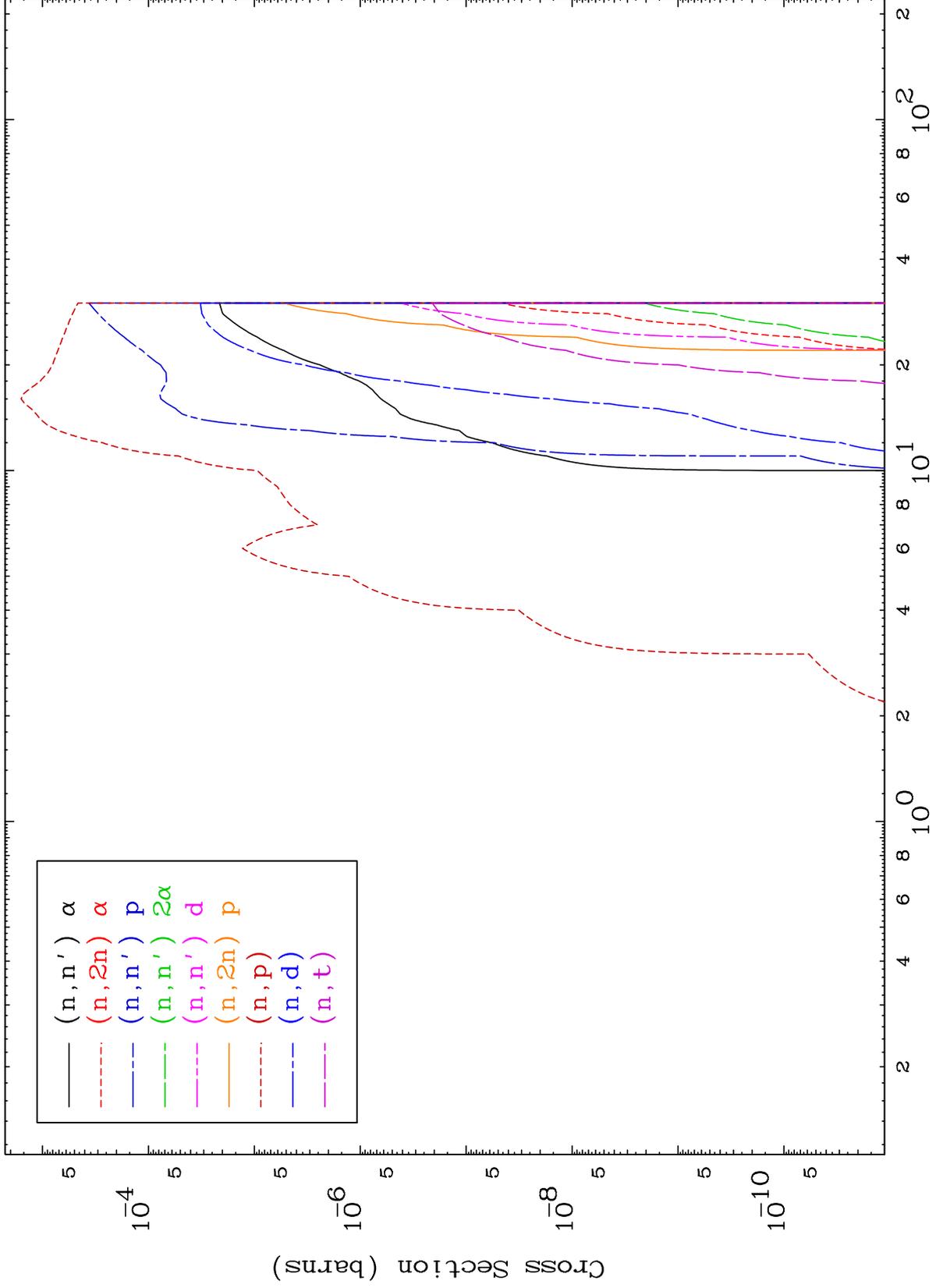
87-Fr-204



MAT 8701

Photon Charged Particle  
0 Kelvin Cross Sections

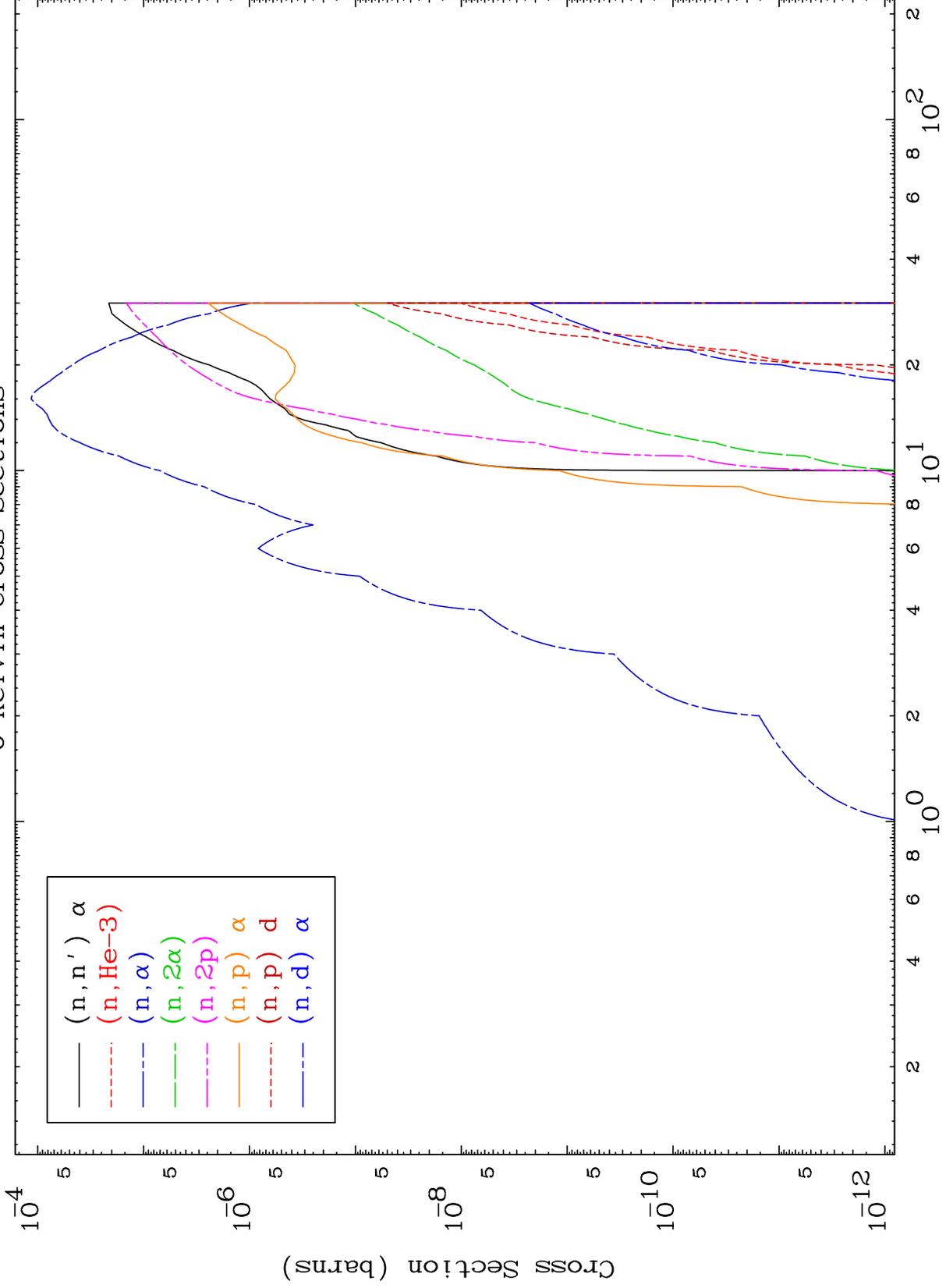
87-Fr-204



MAT 8701

Photon Charged Particle  
0 Kelvin Cross Sections

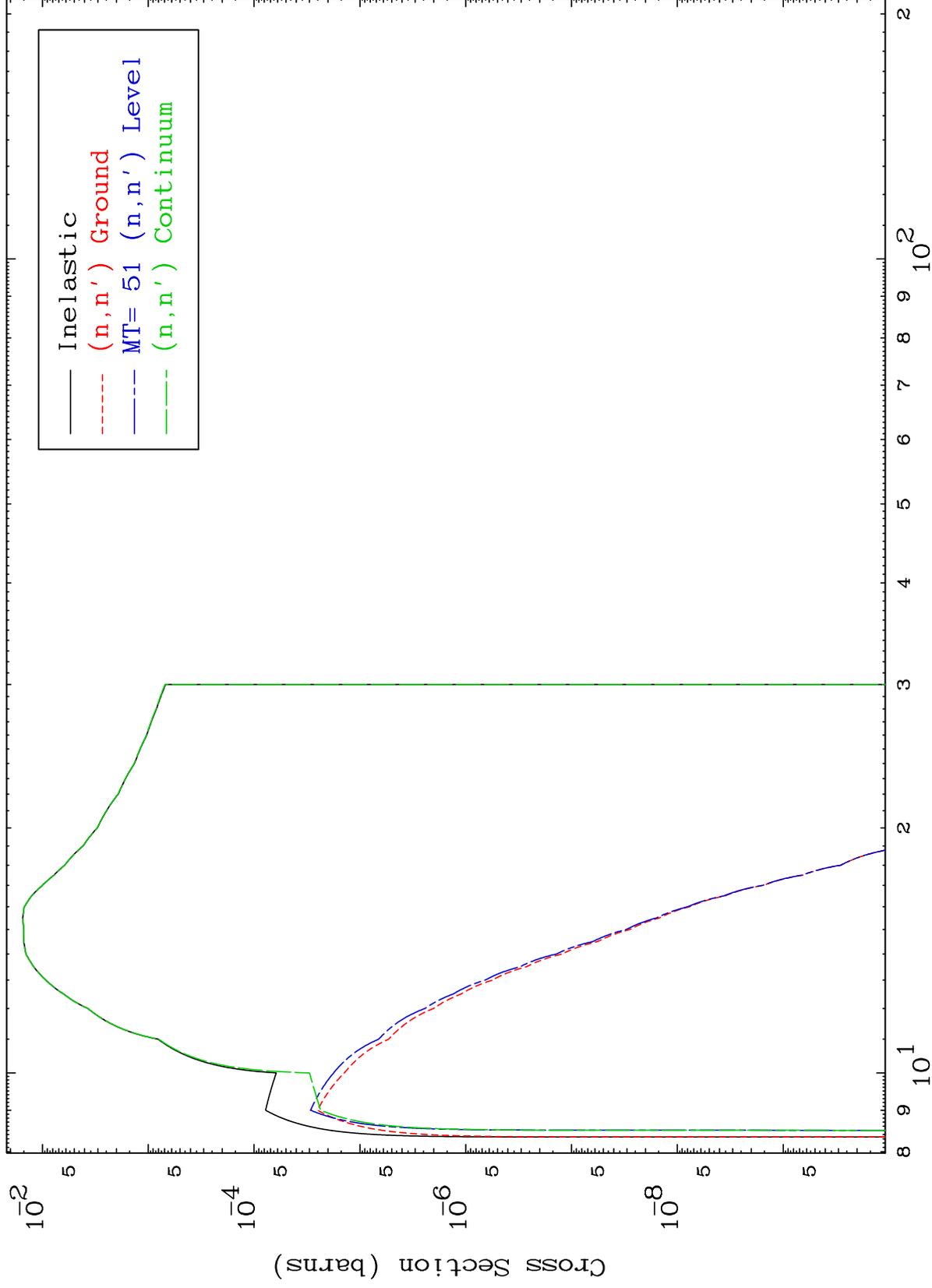
87-Fr-204



MAT 8701

87-Fr-204

( $\gamma, n'$ ) Levels  
0 Kelvin Cross Sections



87-Fr-204

Incident Energy (MeV)

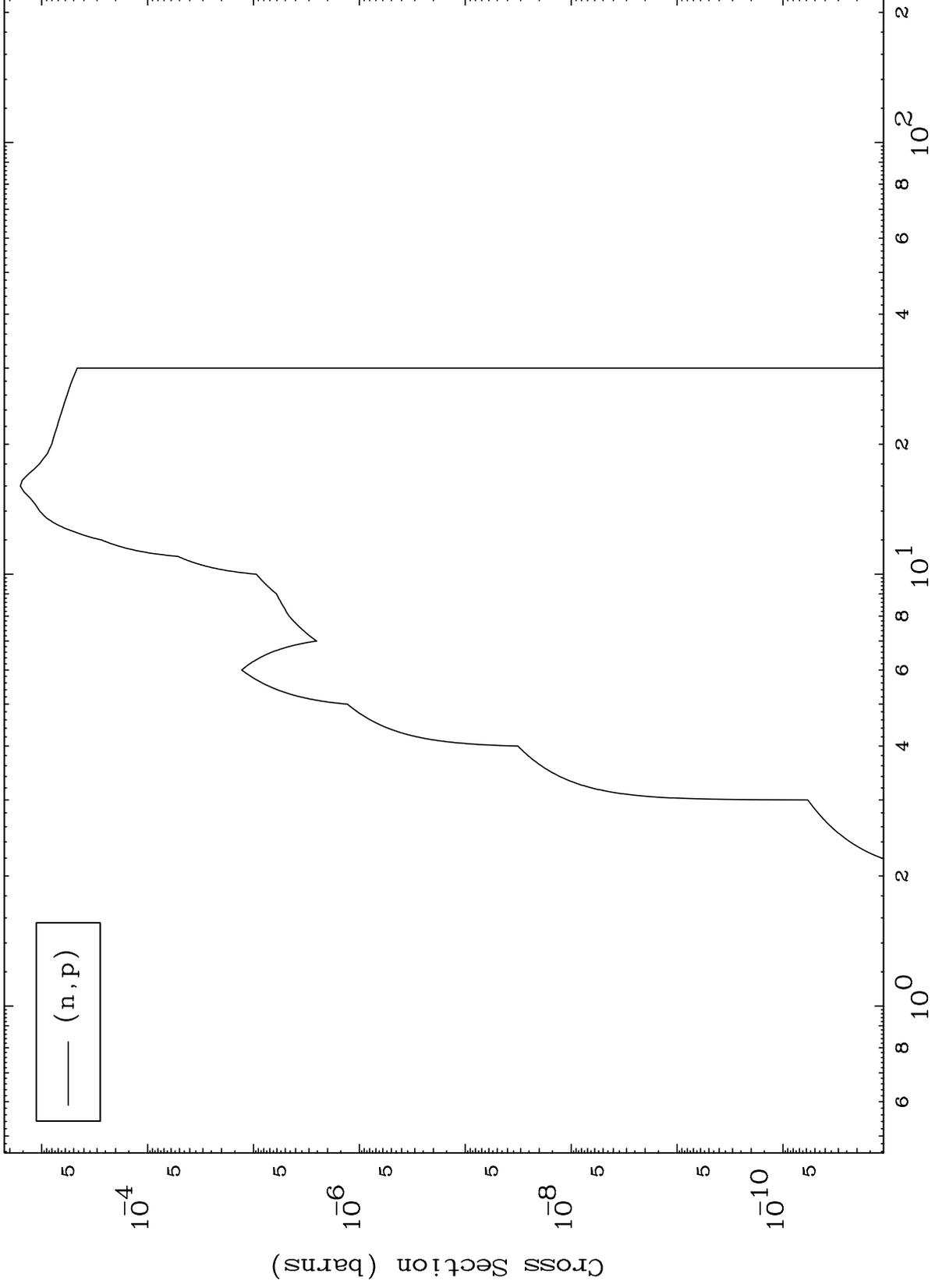
6

MAT 8701

( $\gamma, p$ ) Levels

87-Fr-204

0 Kelvin Cross Sections



7

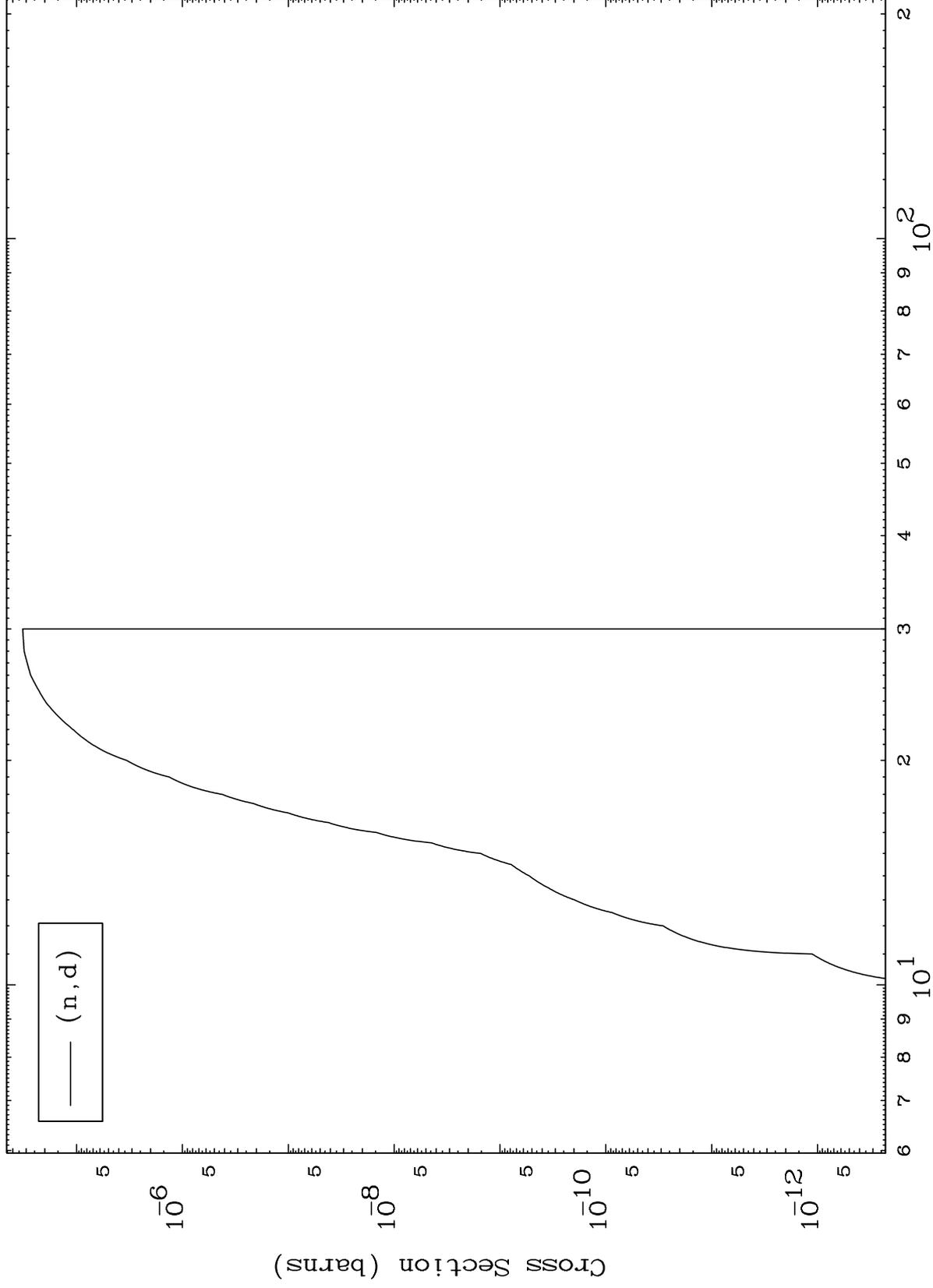
Incident Energy (MeV)

87-Fr-204

MAT 8701

( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

87-Fr-204



8

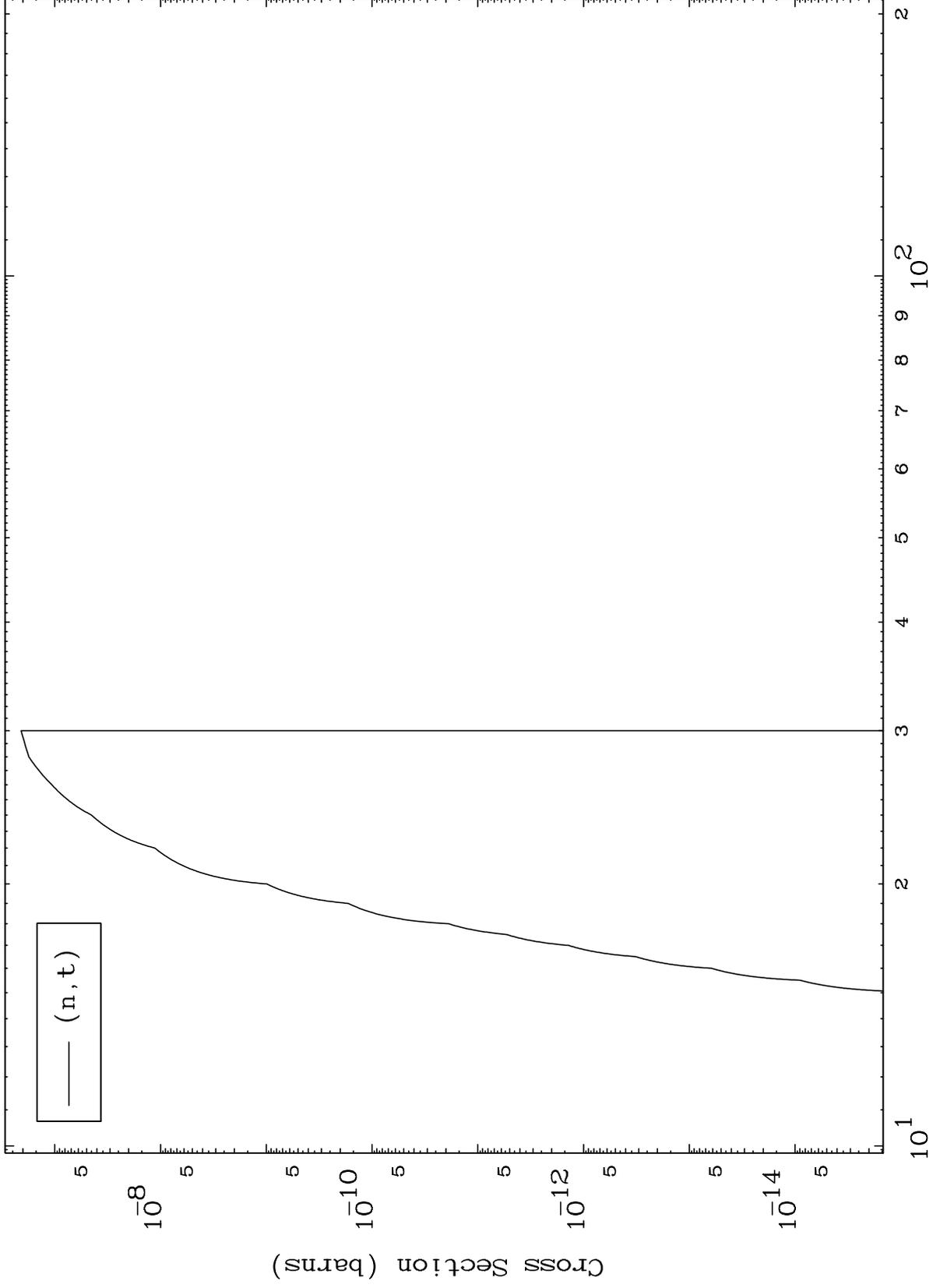
Incident Energy (MeV)

87-Fr-204

MAT 8701

( $\gamma, t$ ) Levels  
0 Kelvin Cross Sections

87-Fr-204



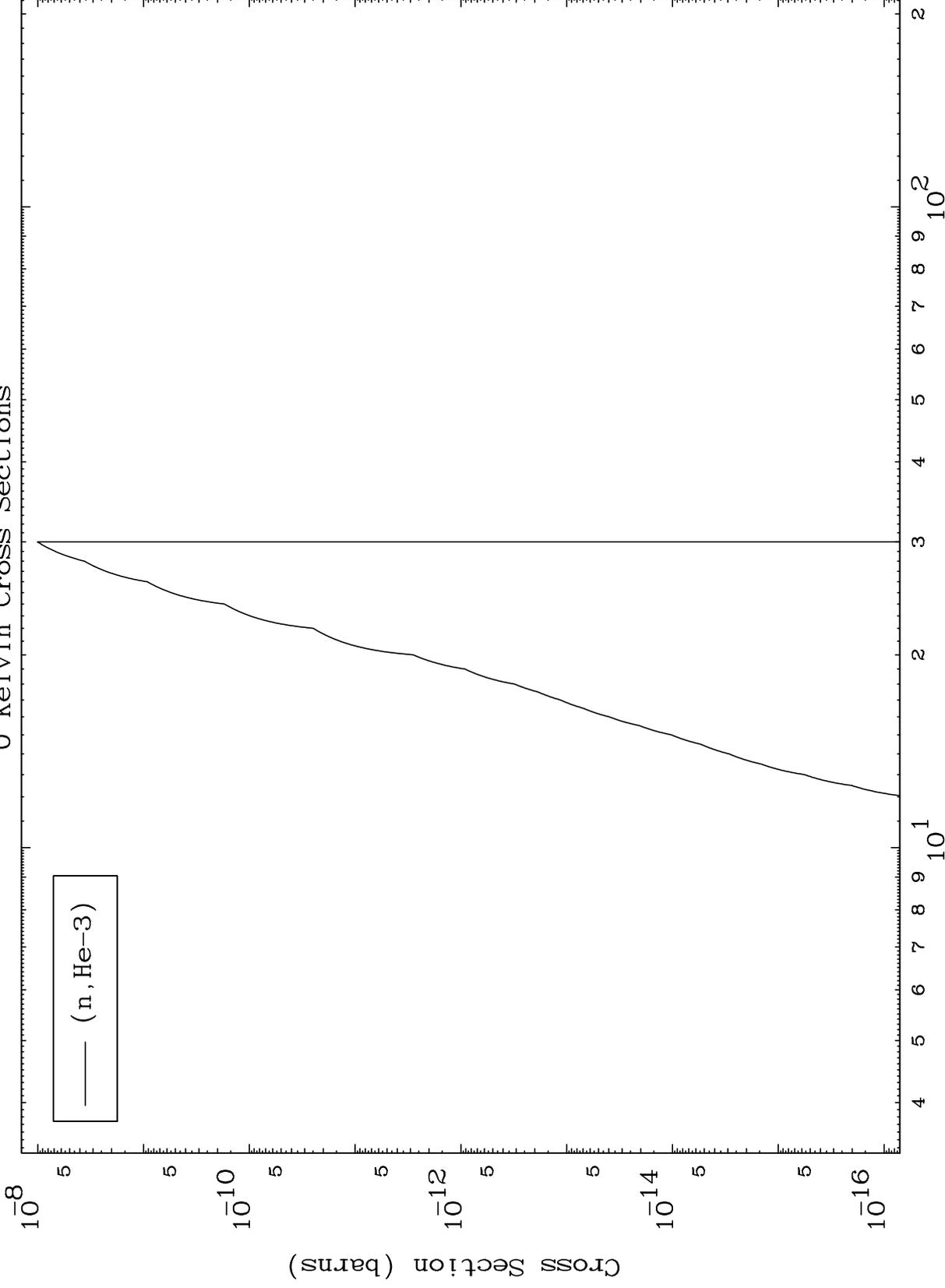
Incident Energy (MeV)

87-Fr-204

MAT 8701

( $\gamma, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

87-Fr-204



10

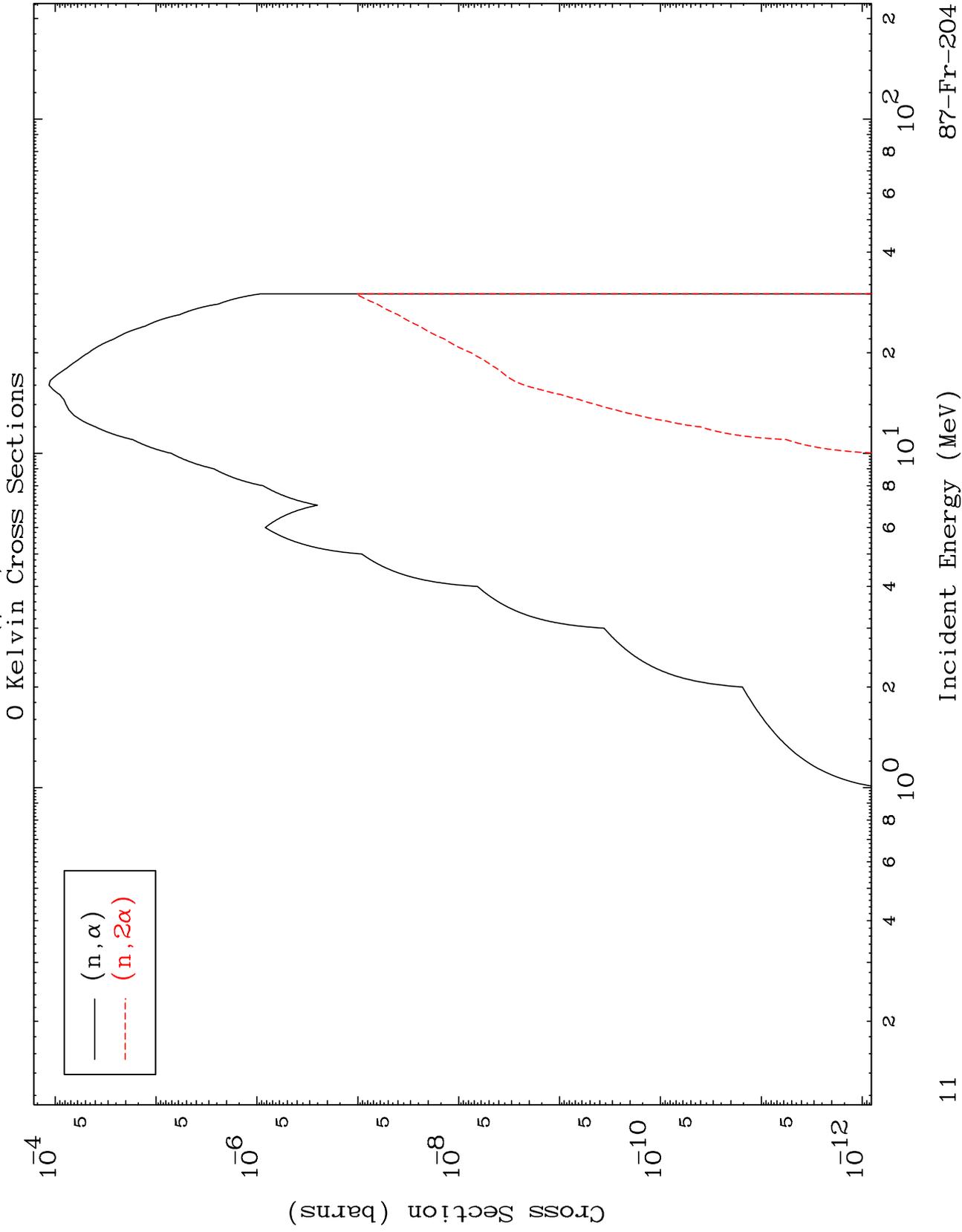
Incident Energy (MeV)

87-Fr-204

MAT 8701

( $\gamma, \alpha$ ) Levels

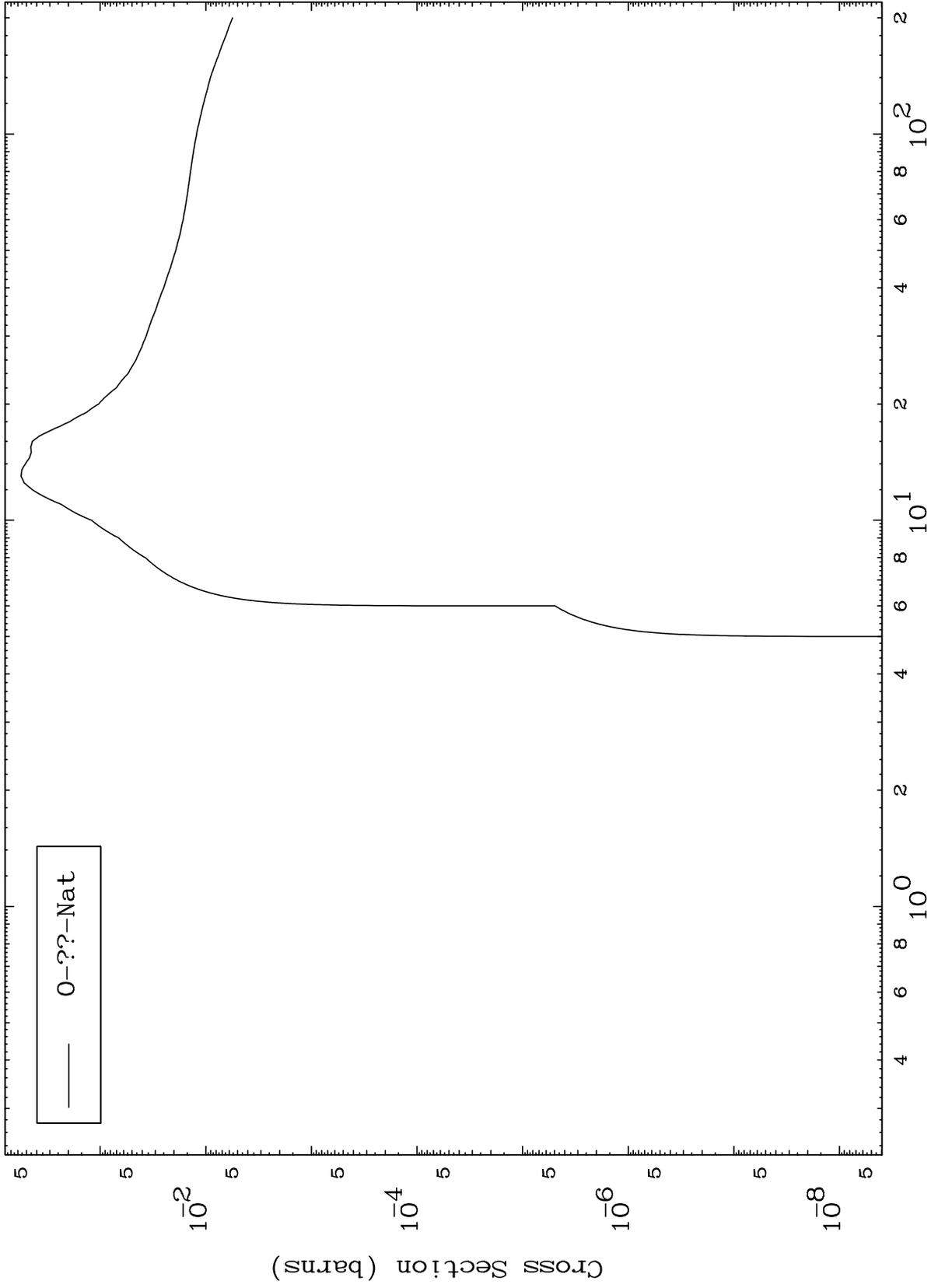
87-Fr-204



MAT 8701

87-Fr-204

Fission  
Radionuclide Production Cross Section



12

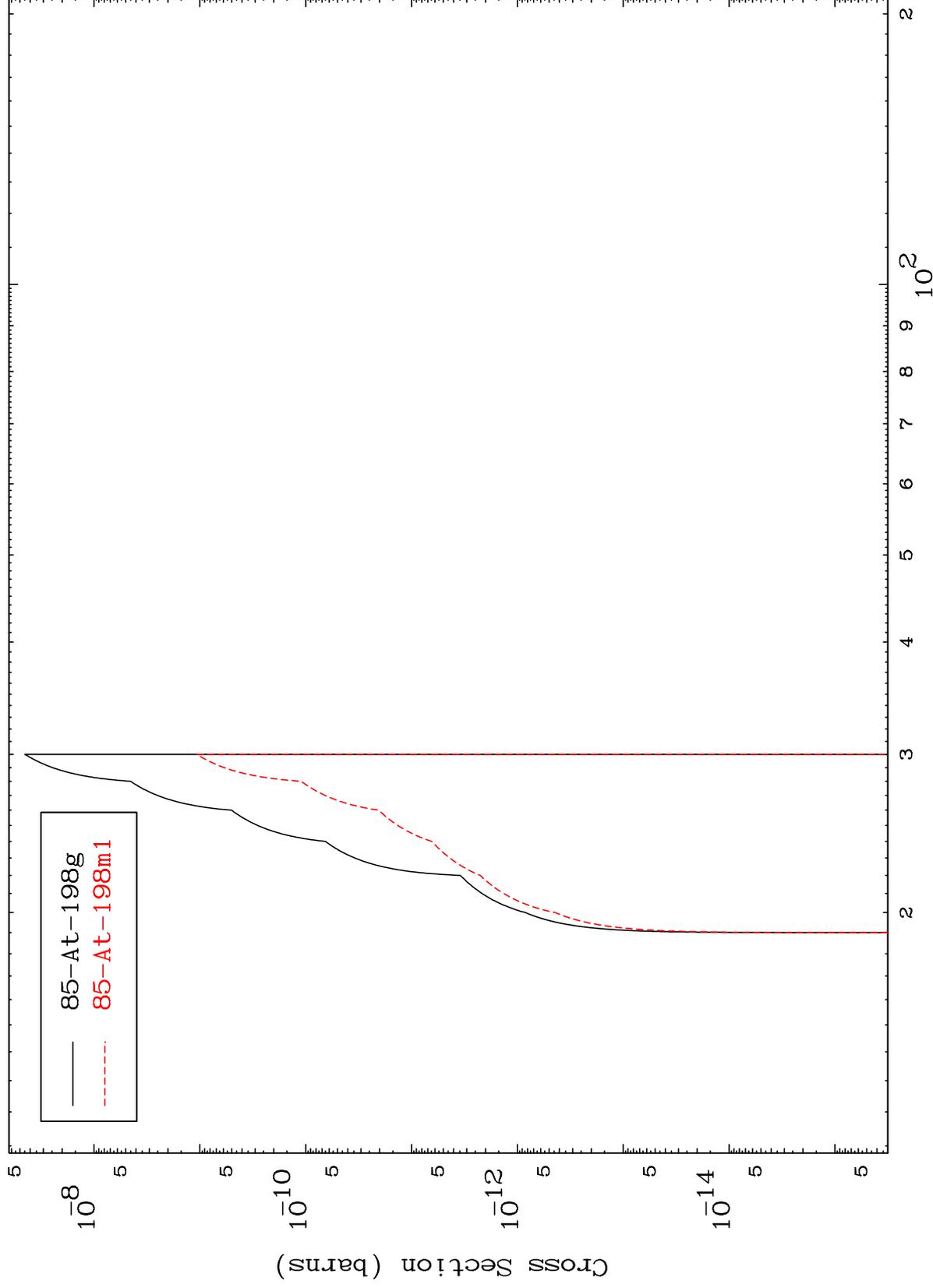
87-Fr-204

Incident Energy (MeV)

MAT 8701

87-Fr-204

(n,2n)  $\alpha$   
Radionuclide Production Cross Section



13

Incident Energy (MeV)

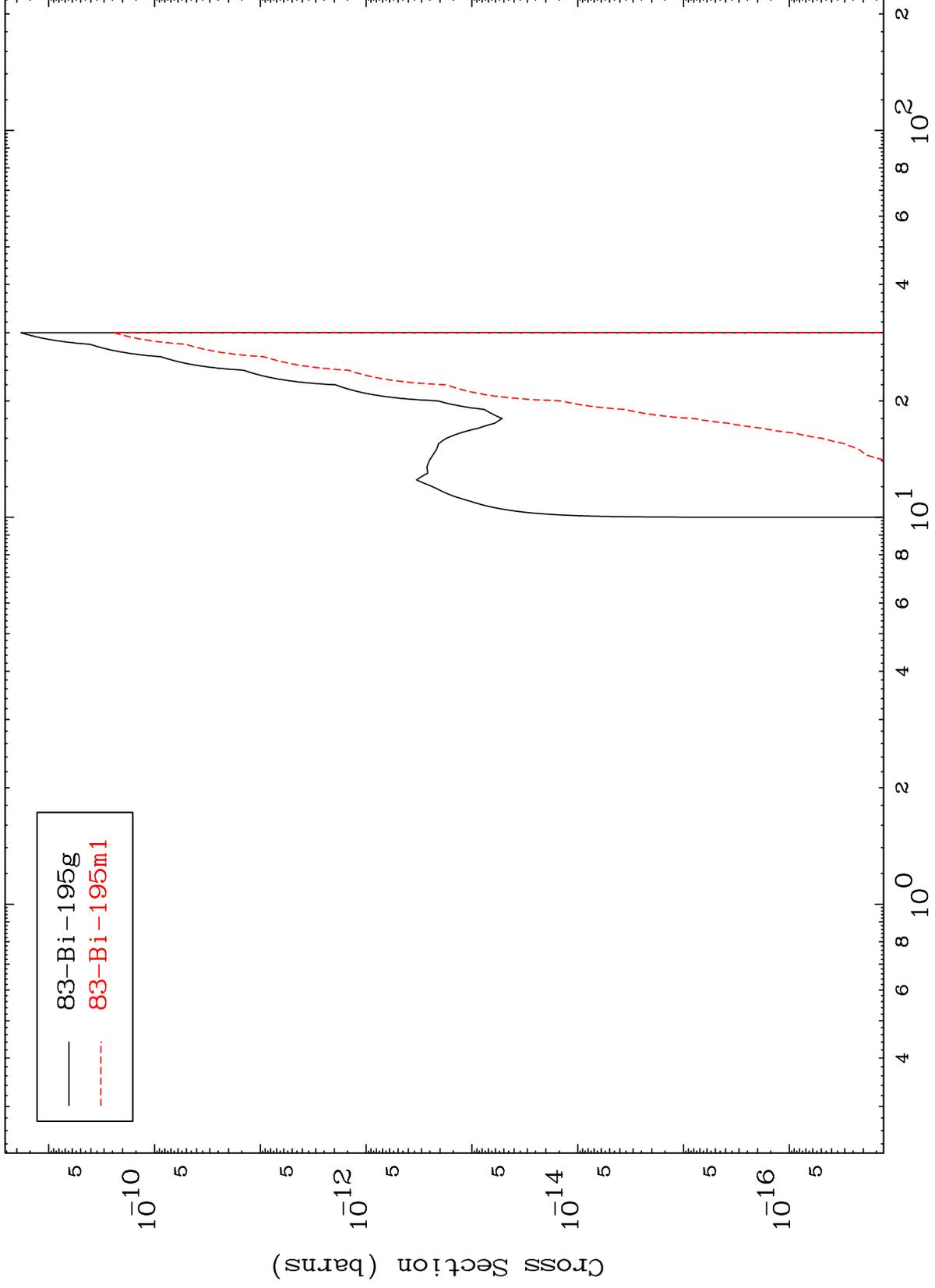
87-Fr-204

MAT 8701

(n,n') 2 $\alpha$

87-Fr-204

Radionuclide Production Cross Section

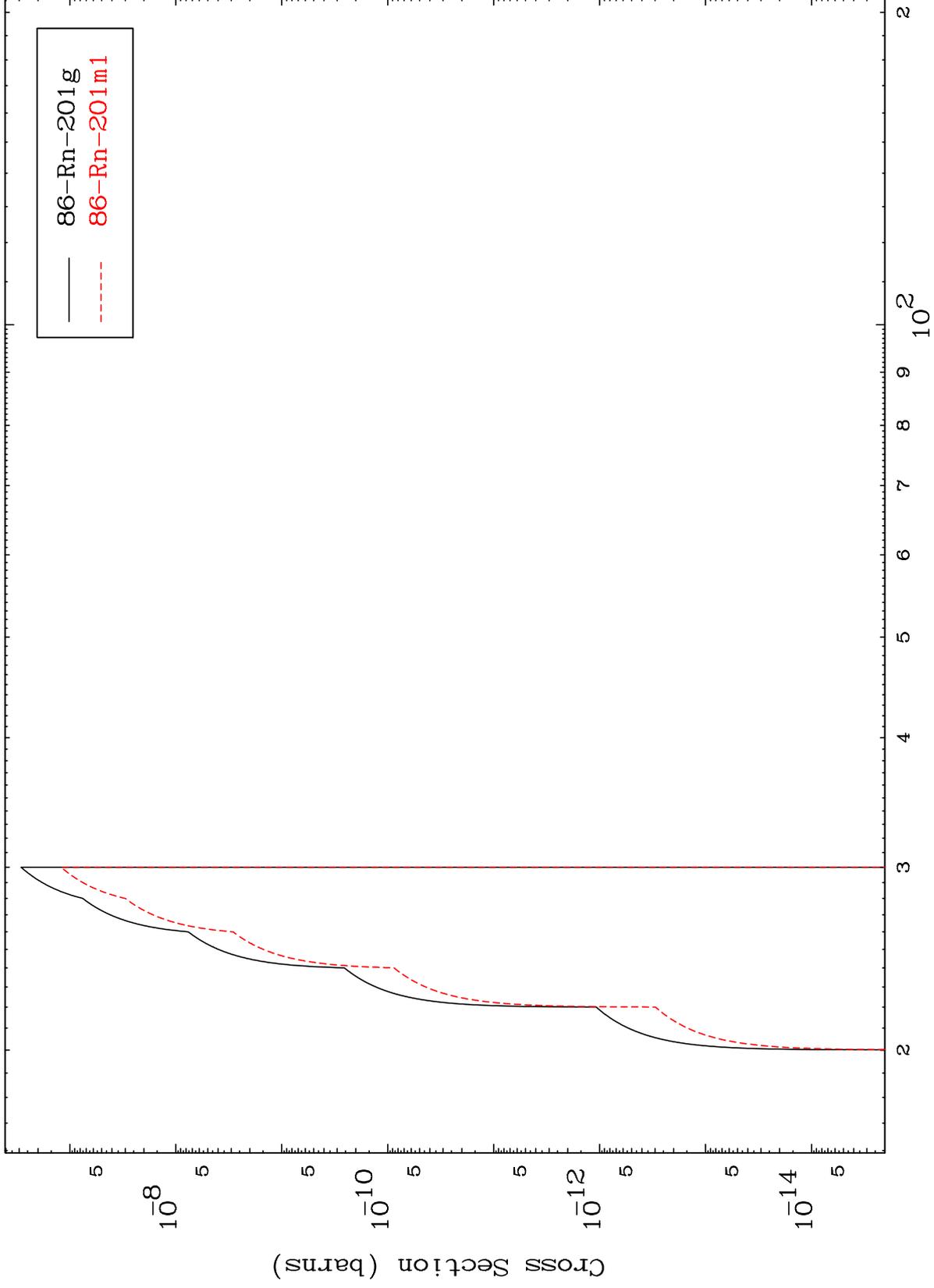


MAT 8701

(n,n') d

87-Fr-204

Radionuclide Production Cross Section



15

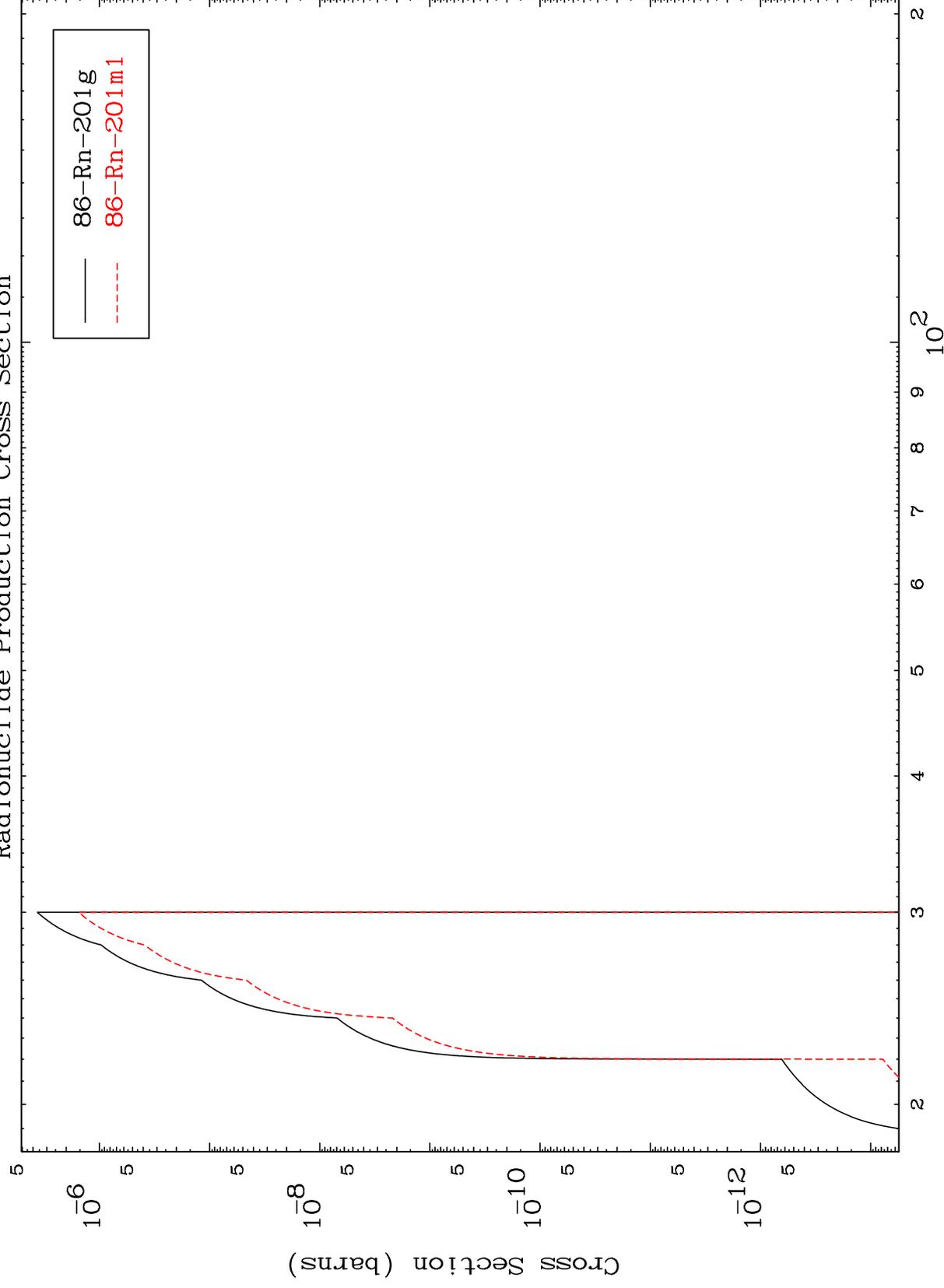
Incident Energy (MeV)

87-Fr-204

MAT 8701

87-Fr-204

(n,2n) p  
Radionuclide Production Cross Section



16

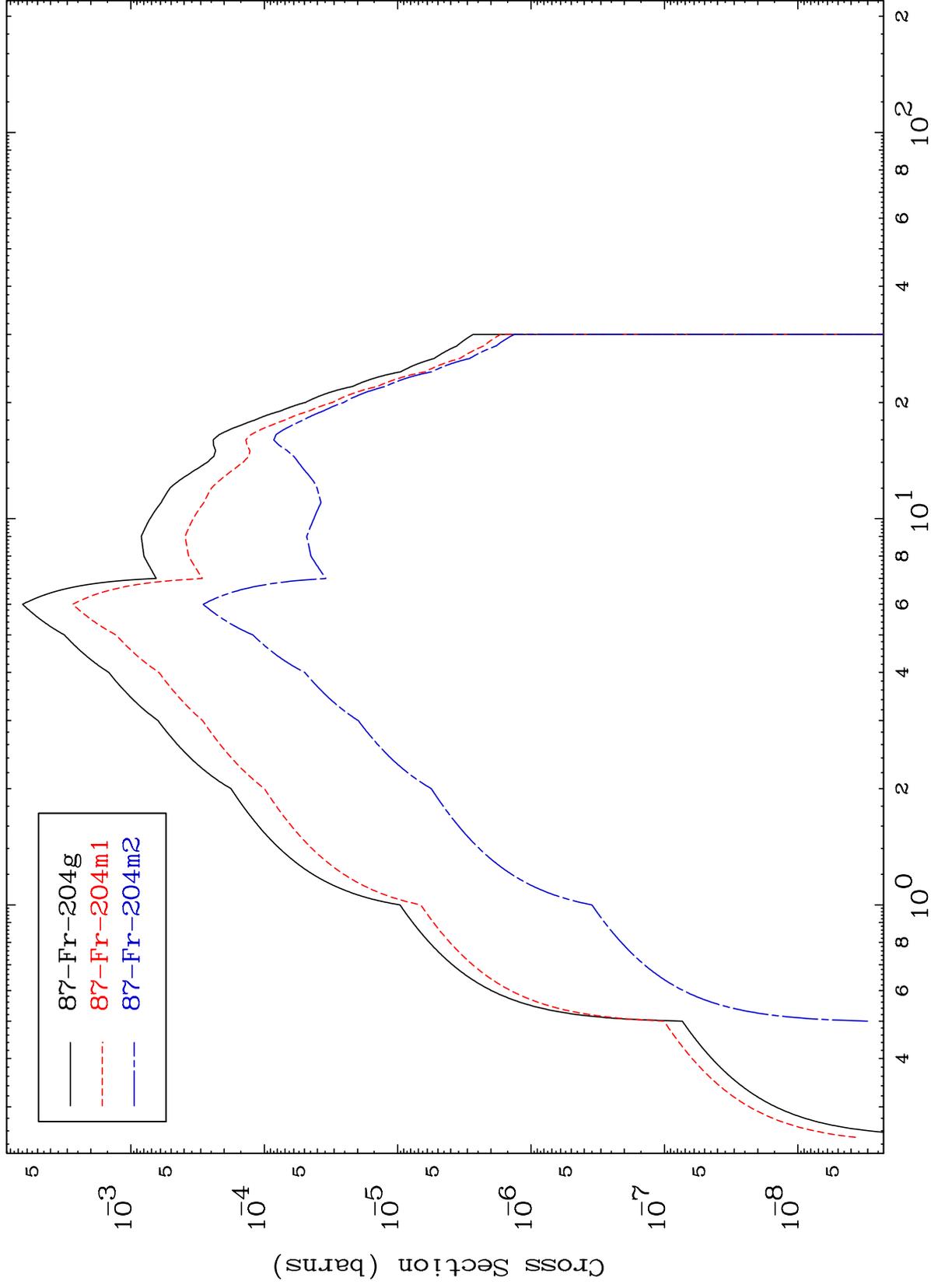
Incident Energy (MeV)

87-Fr-204

MAT 8701

87-Fr-204

(n,  $\gamma$ )  
Radionuclide Production Cross Section



87-Fr-204

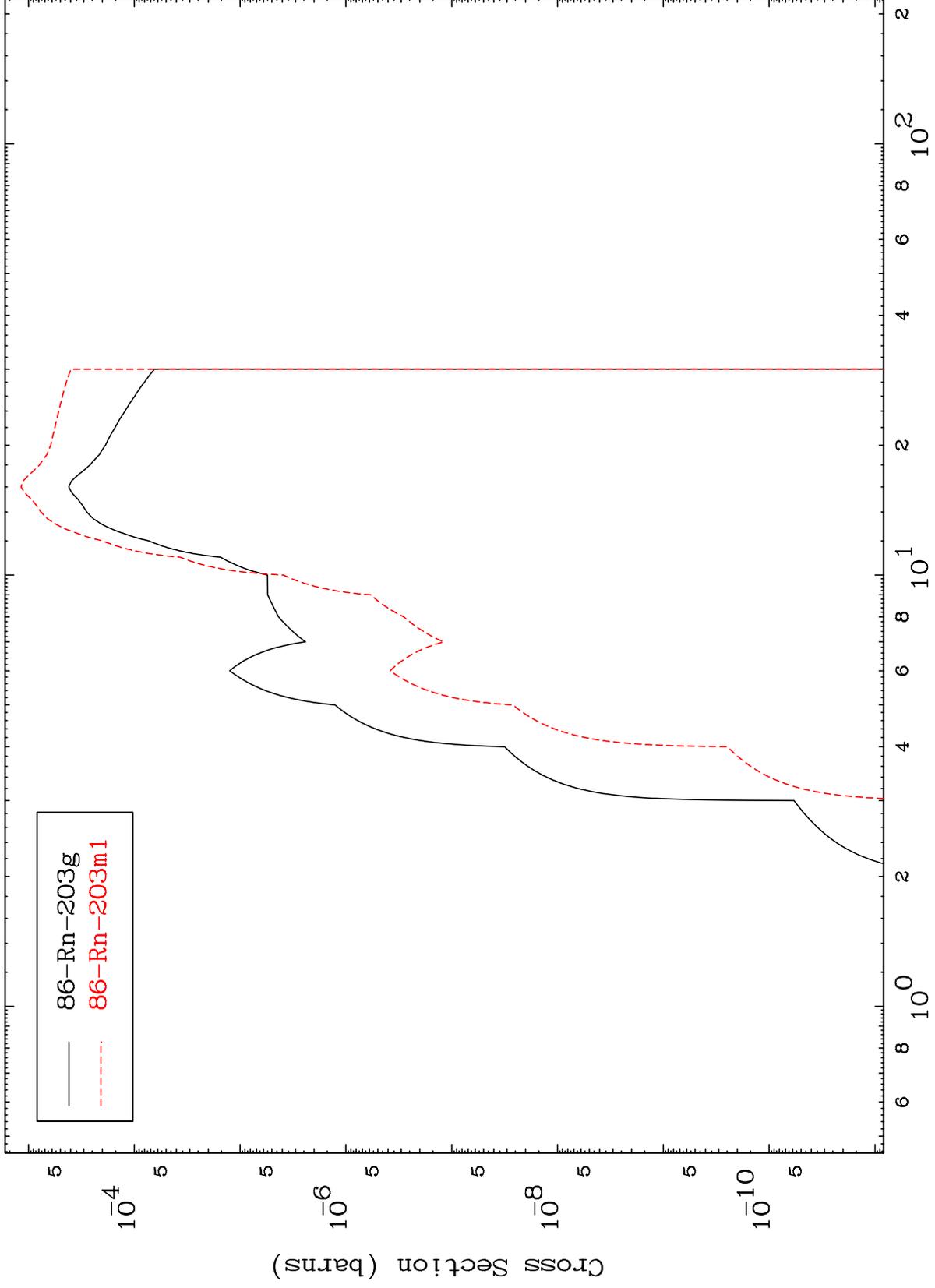
Incident Energy (MeV)

17

MAT 8701

87-Fr-204

(n,p)  
Radionuclide Production Cross Section



18

Incident Energy (MeV)

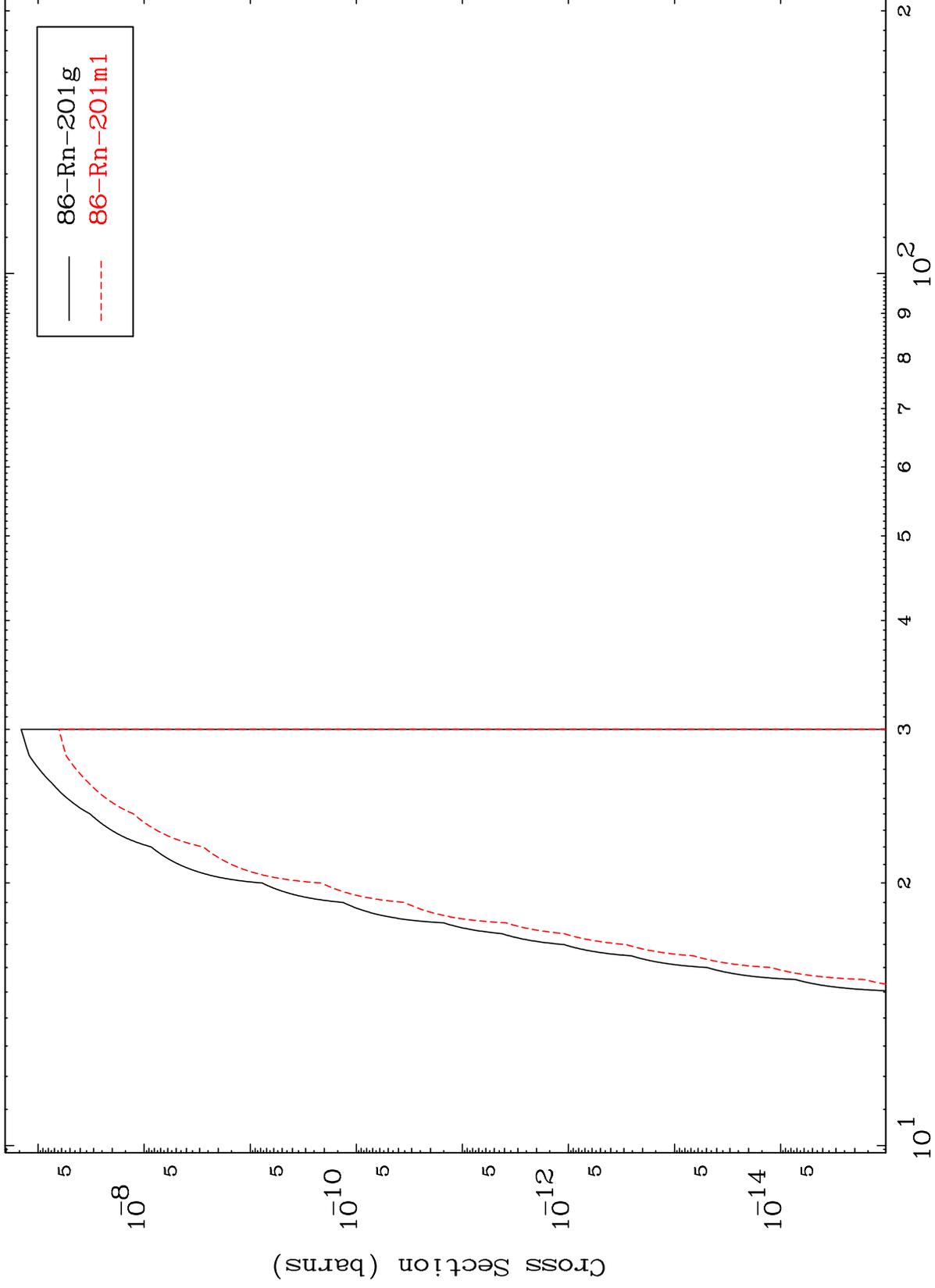
87-Fr-204

MAT 8701

(n, t)

87-Fr-204

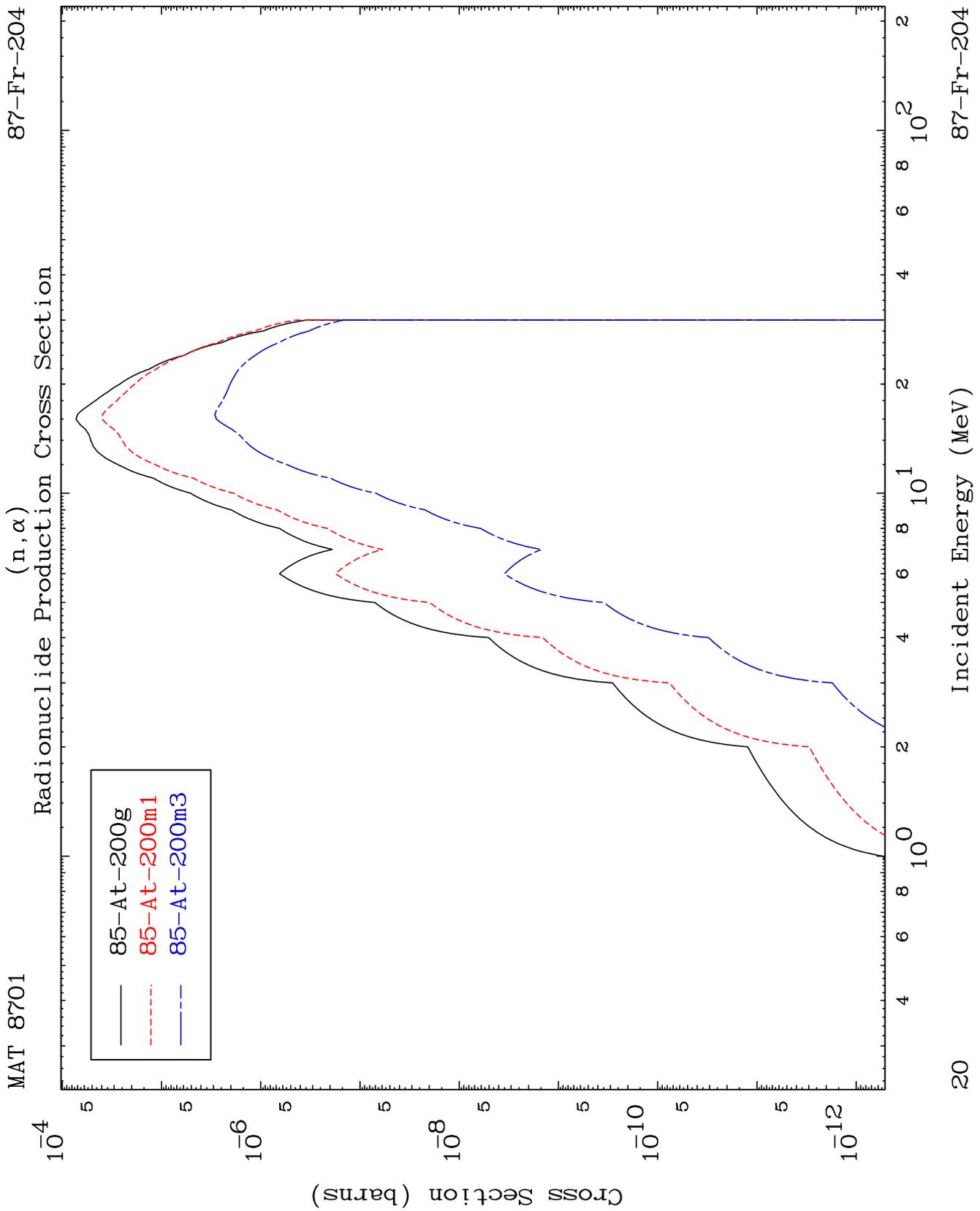
Radionuclide Production Cross Section



10<sup>1</sup>

Incident Energy (MeV)

87-Fr-204

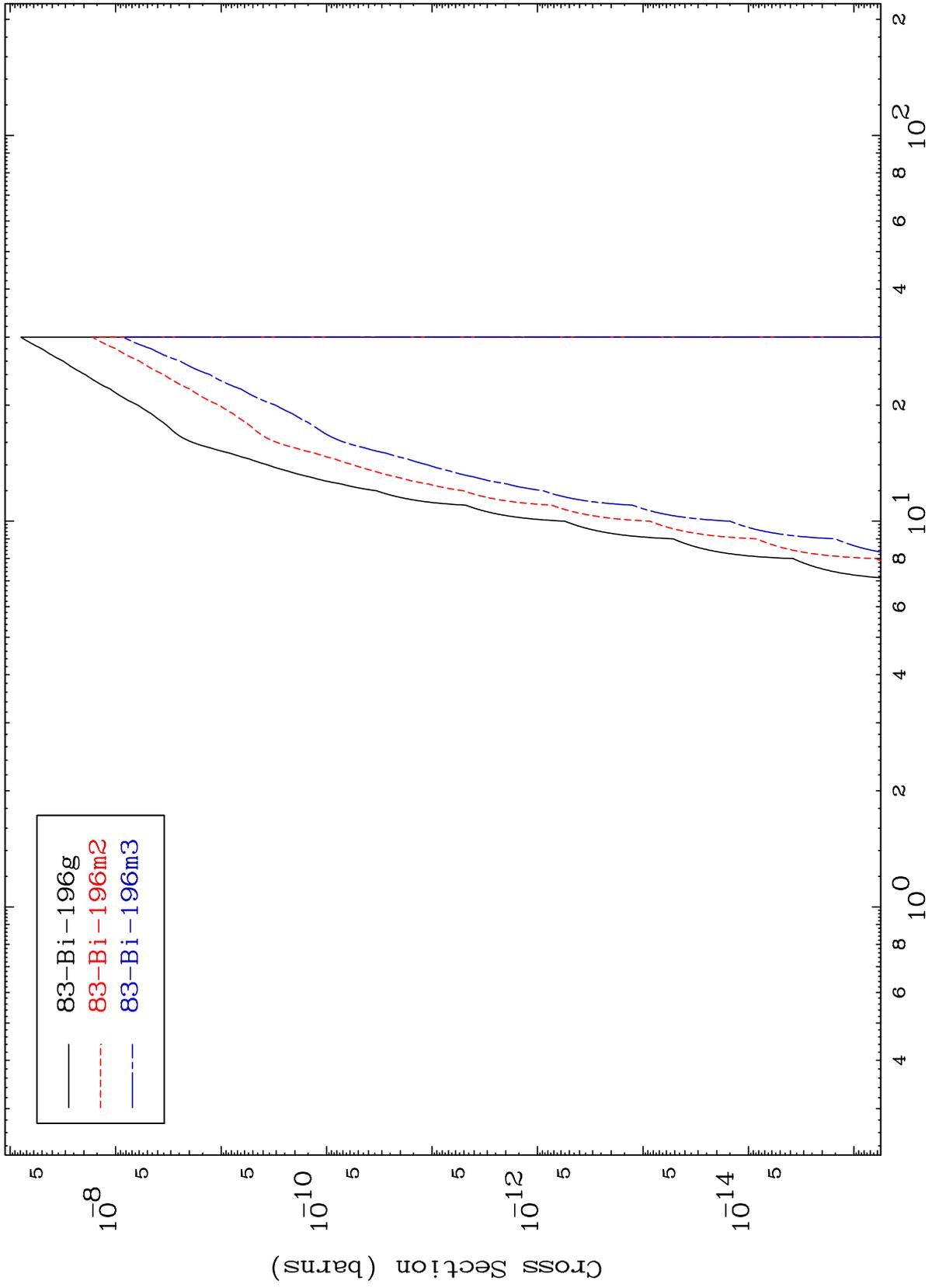


MAT 8701

(n,2α)

87-Fr-204

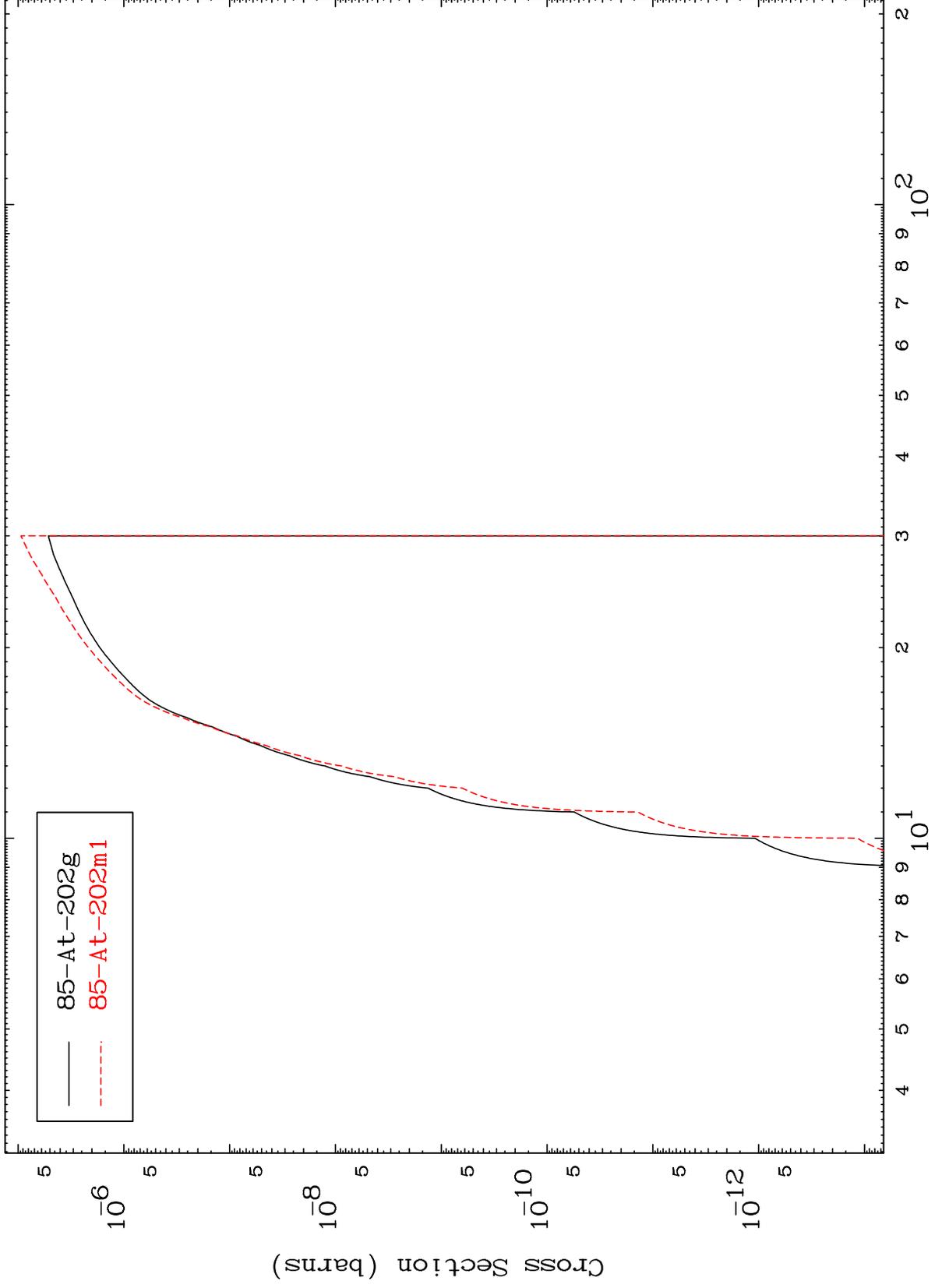
Radionuclide Production Cross Section



MAT 8701

87-Fr-204

(n,2p)  
Radionuclide Production Cross Section



22

Incident Energy (MeV)

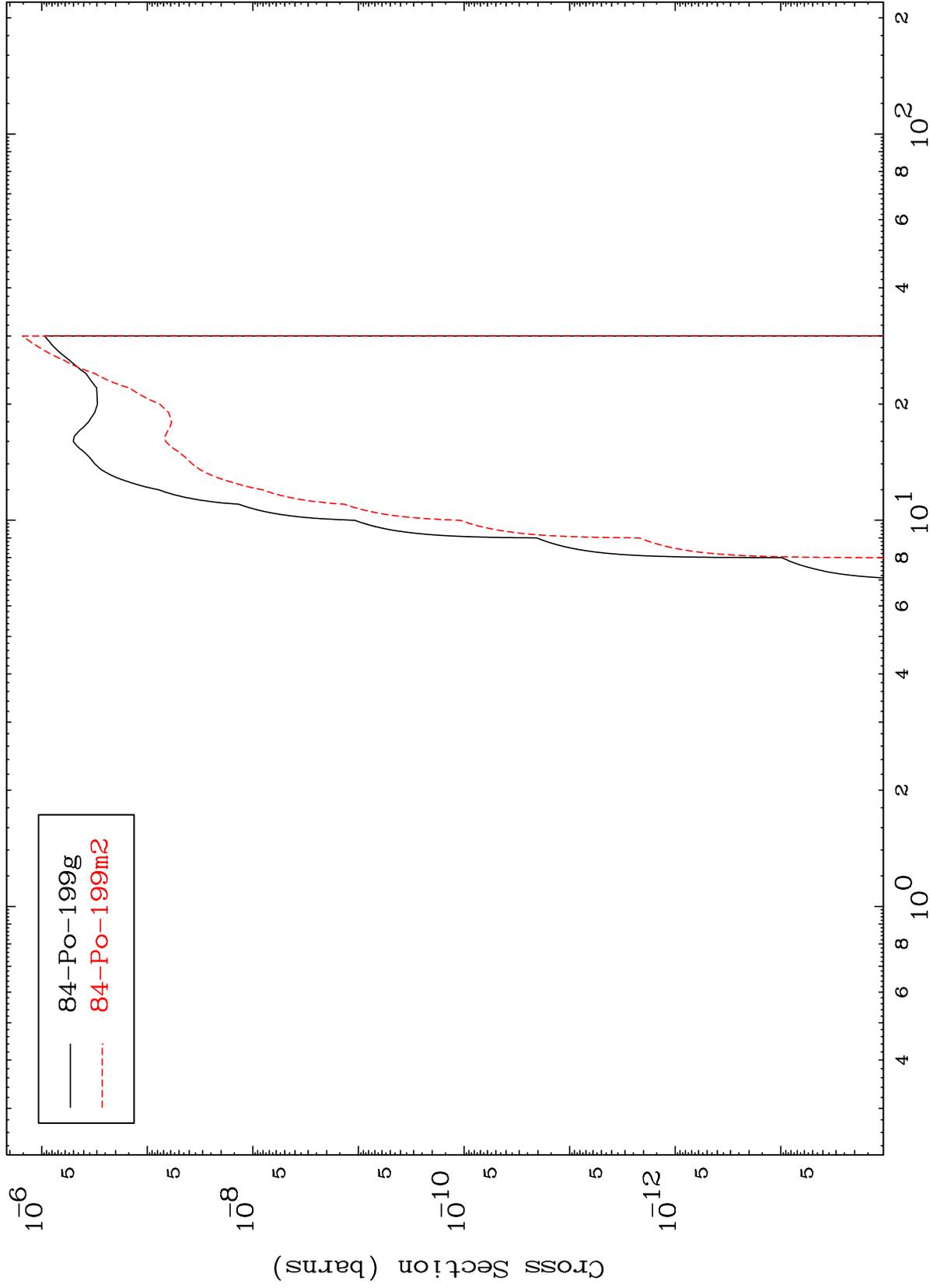
87-Fr-204

MAT 8701

(n,p)  $\alpha$

87-Fr-204

Radionuclide Production Cross Section



23

Incident Energy (MeV)

87-Fr-204