

Program EVALPLOT  
(Version 2021-1)

by

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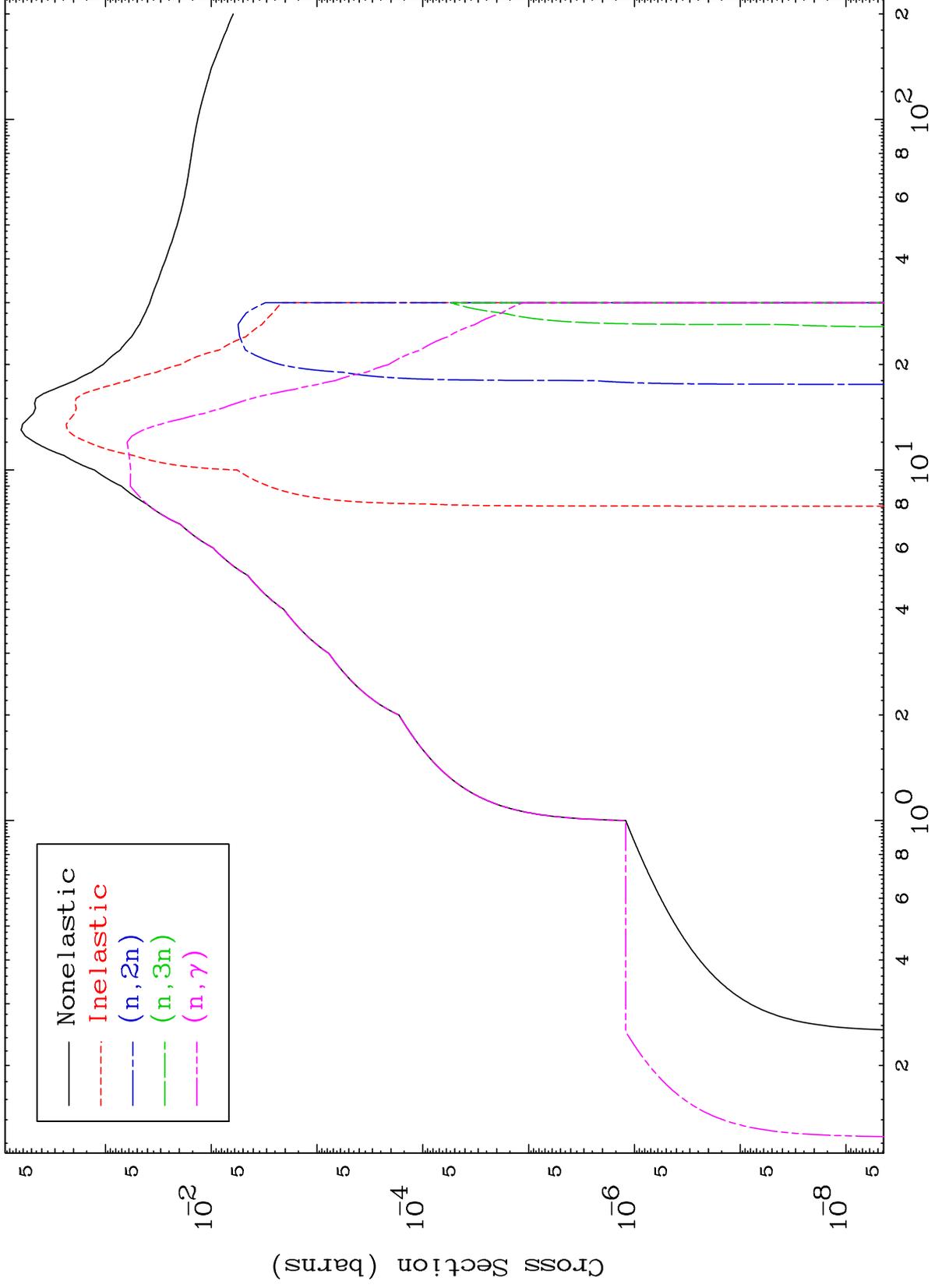
Press Mouse Button to Start

MAT 8713

Photon Major

87-Fr-208

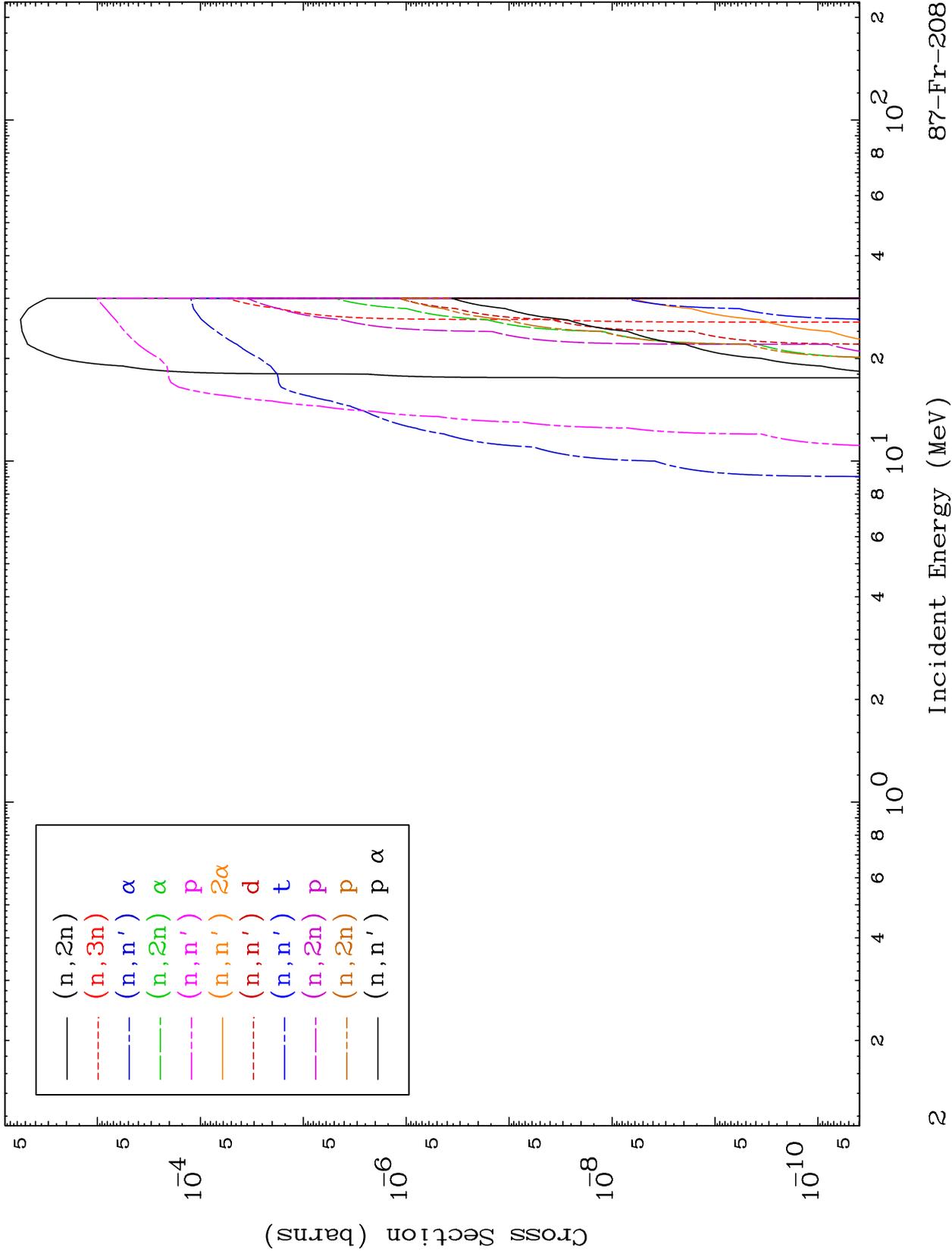
0 Kelvin Cross Sections

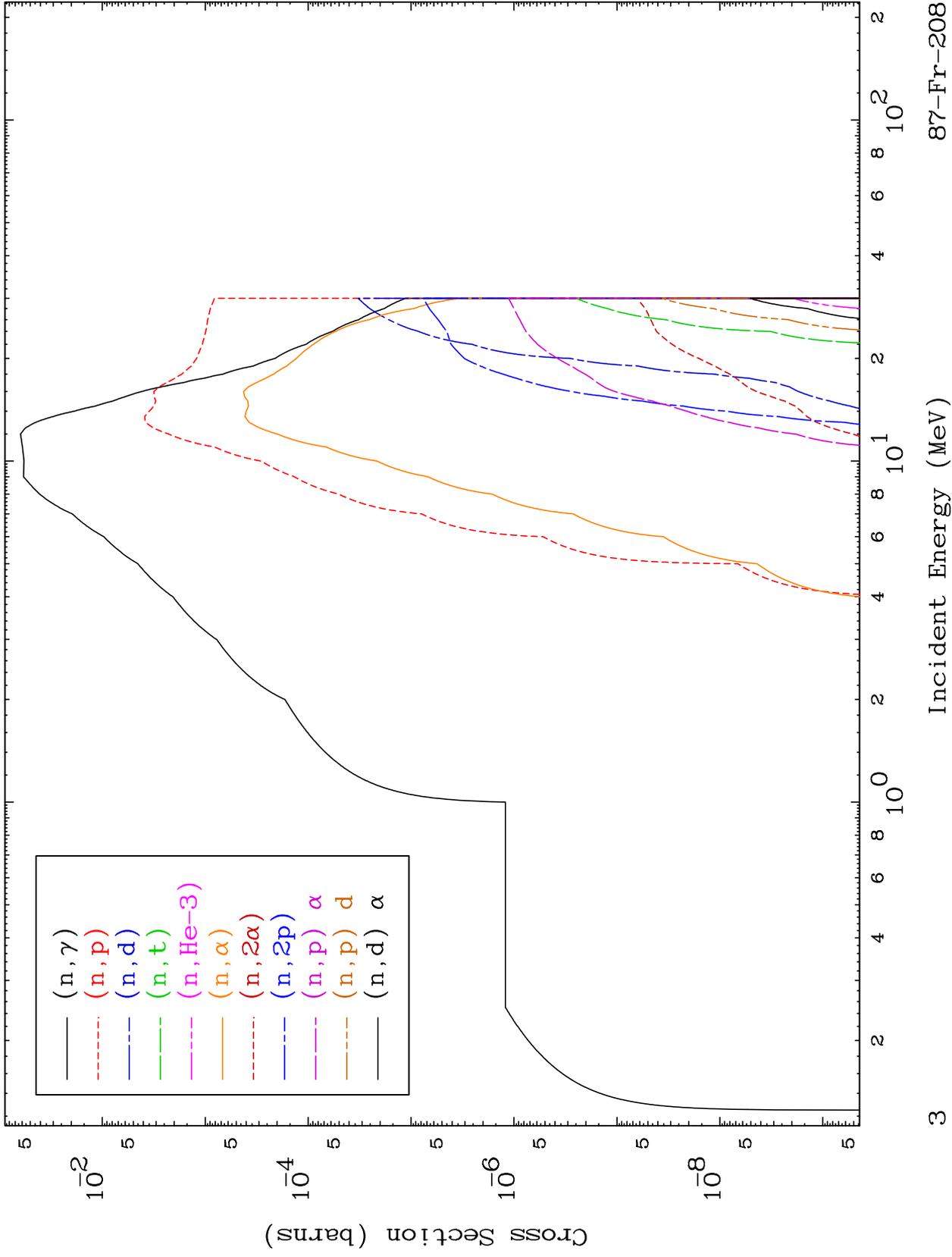


MAT 8713

Photon Neutron Absorption  
0 Kelvin Cross Sections

87-Fr-208

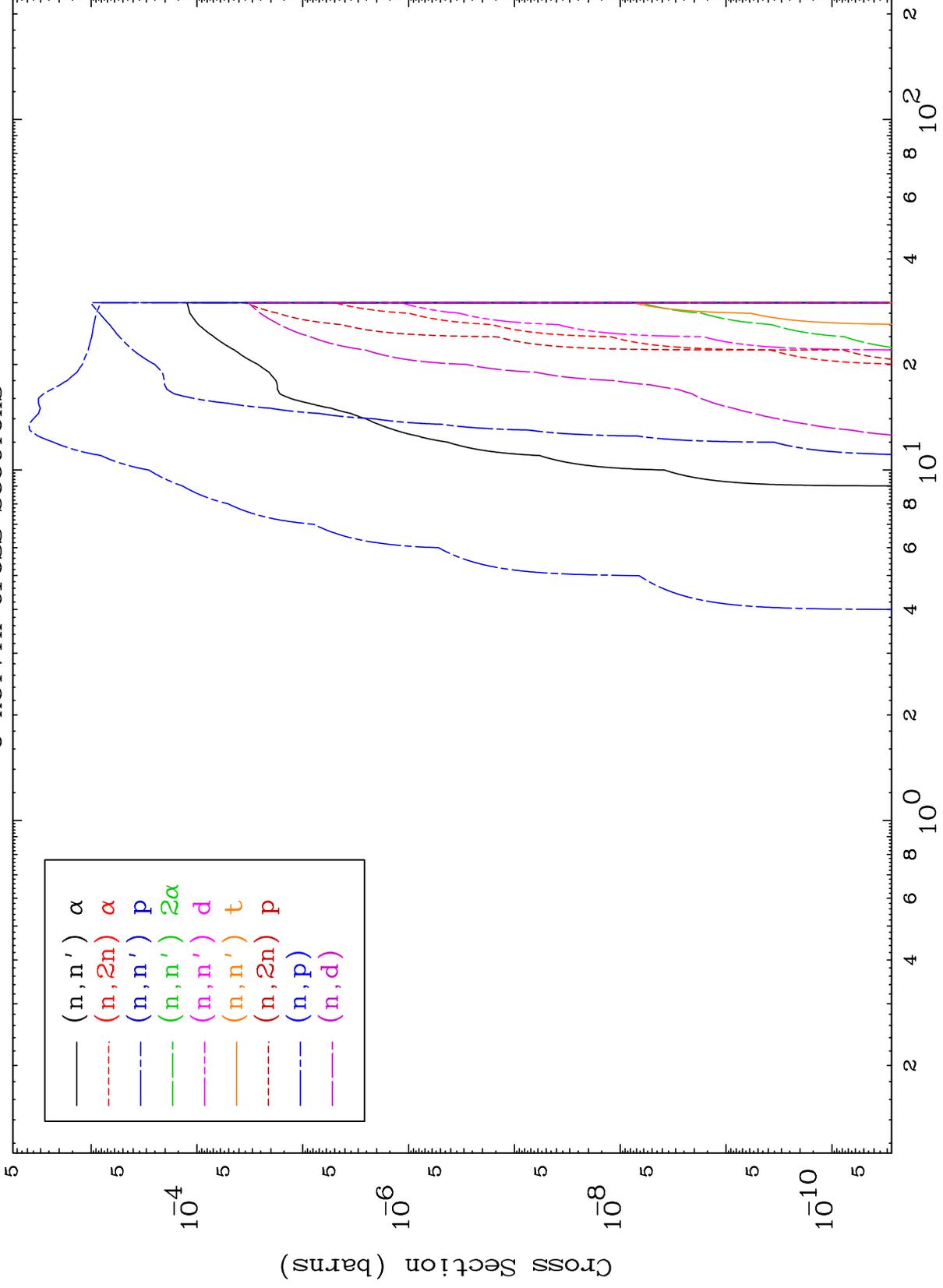


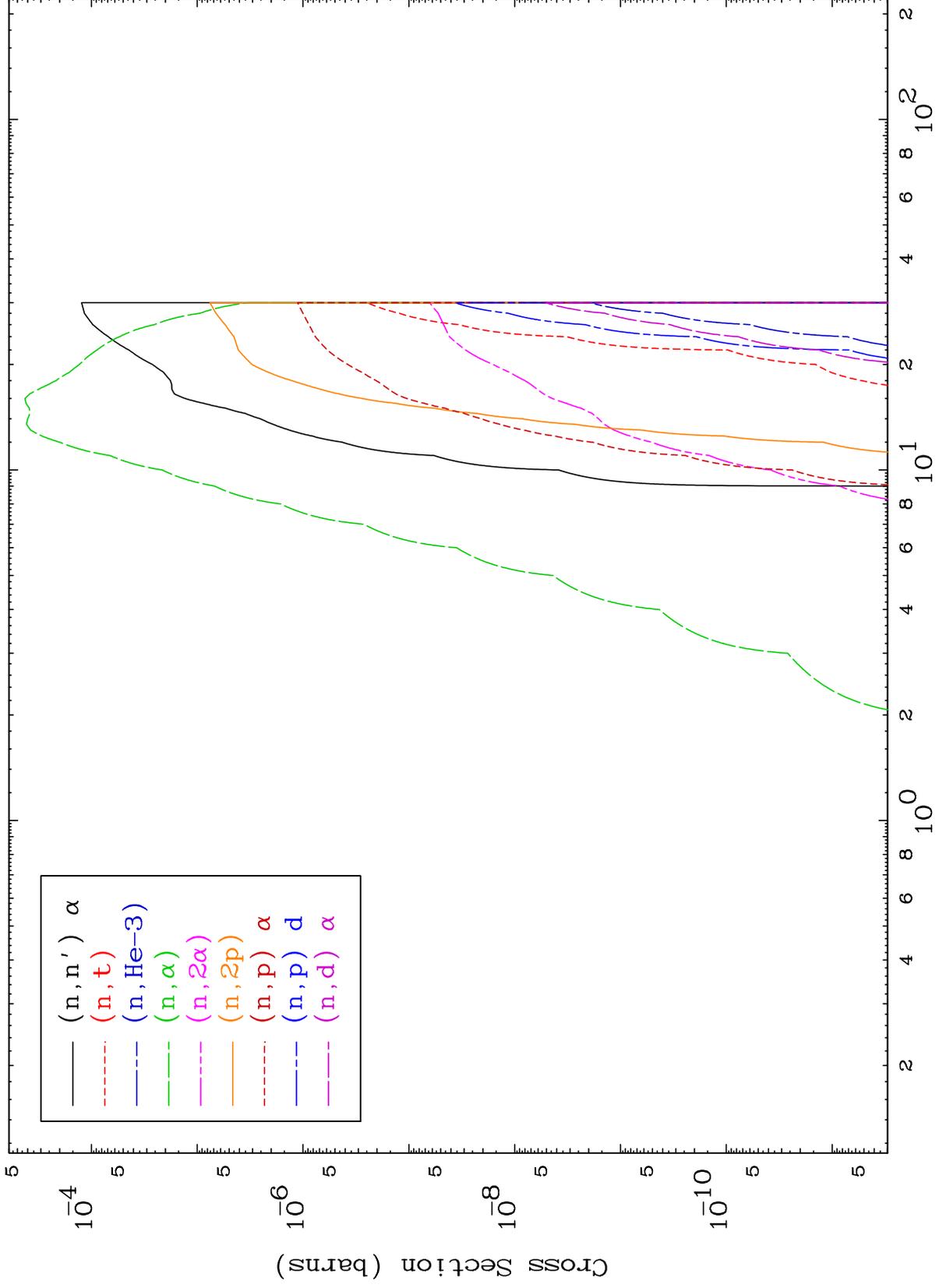


MAT 8713

Photon Charged Particle  
0 Kelvin Cross Sections

87-Fr-208



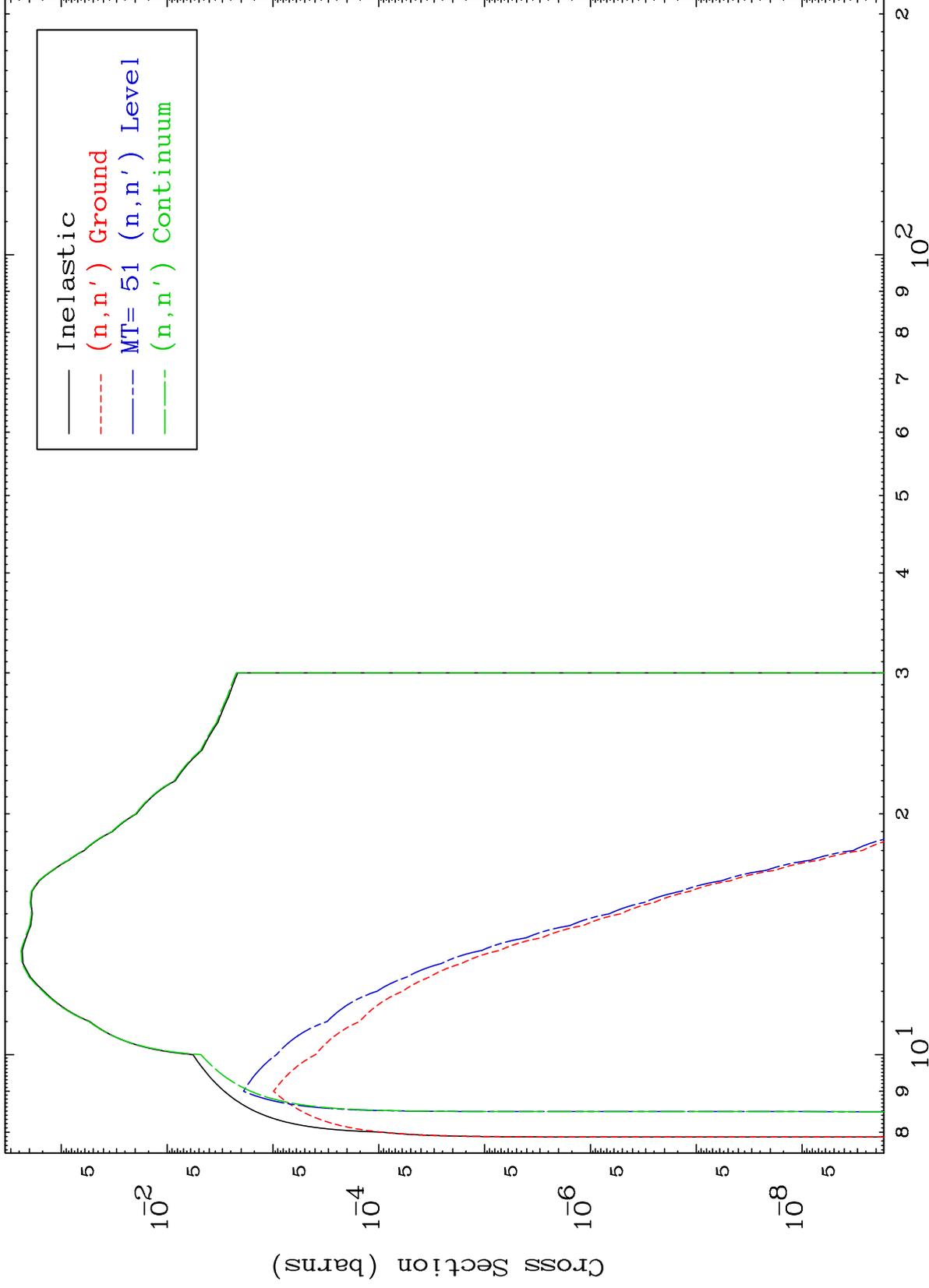


MAT 8713

( $\gamma, n'$ ) Levels

87-Fr-208

0 Kelvin Cross Sections



6

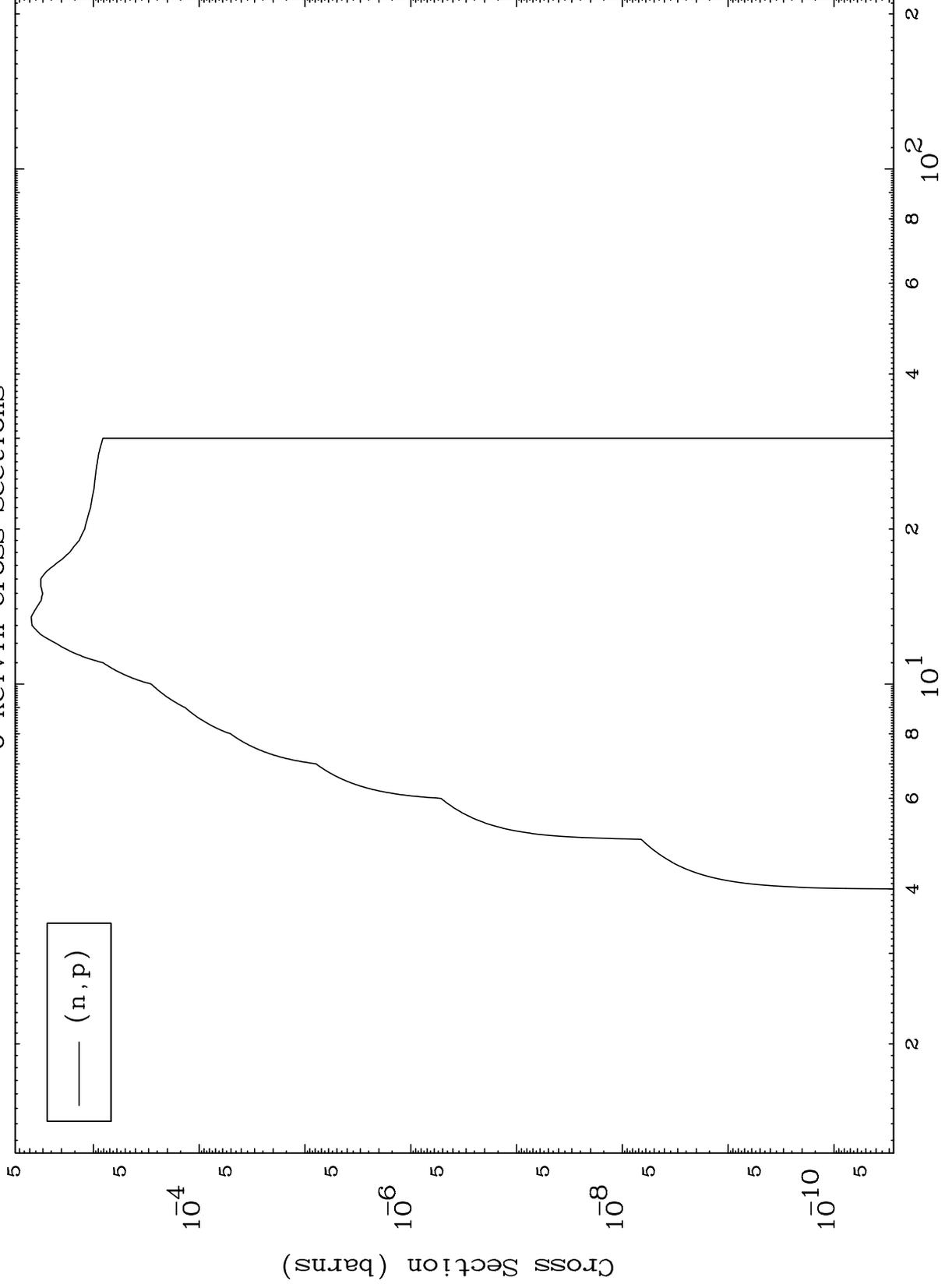
Incident Energy (MeV)

87-Fr-208

MAT 8713

( $\gamma, p$ ) Levels  
0 Kelvin Cross Sections

87-Fr-208

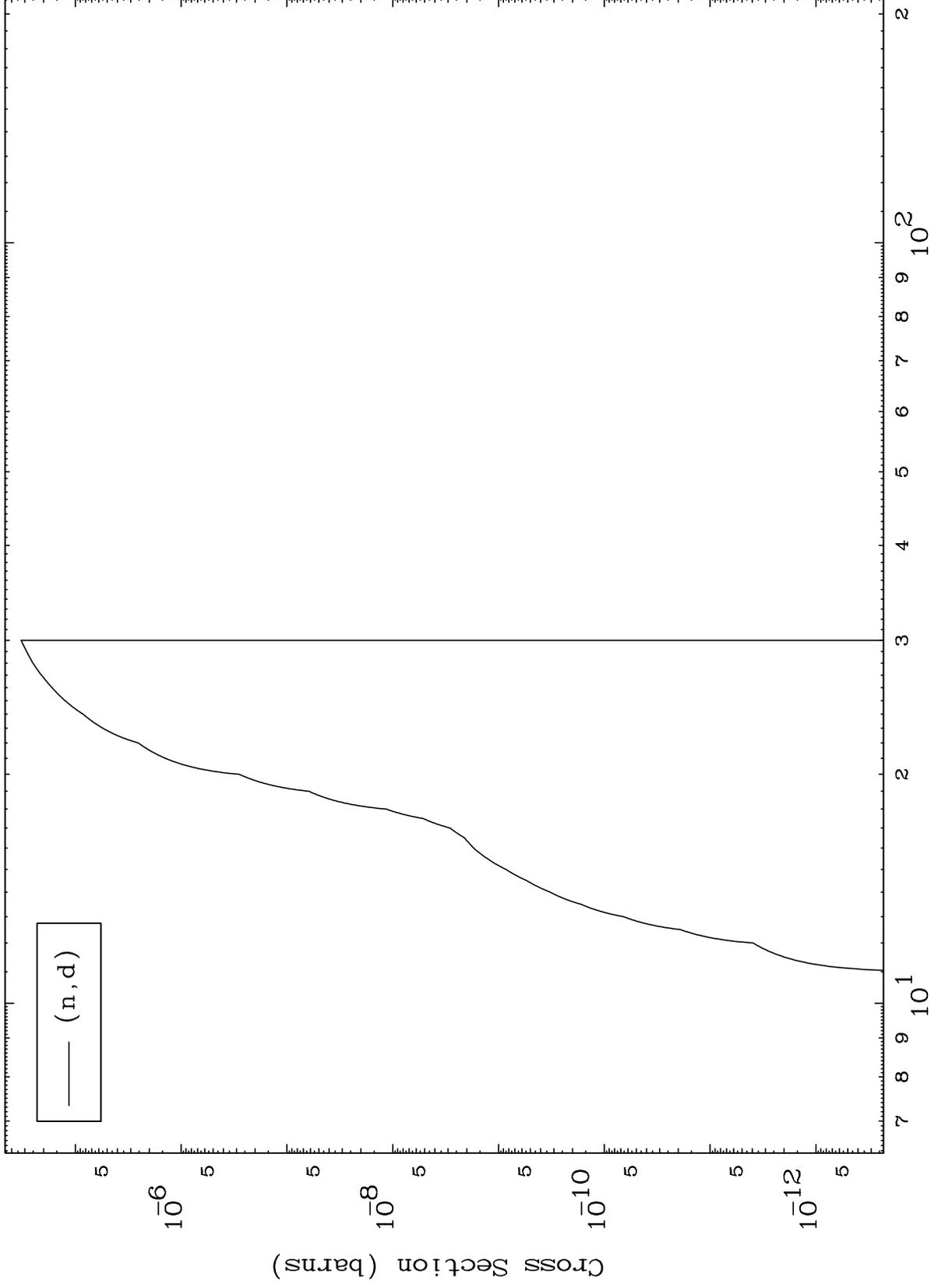


(n, p)

MAT 8713

( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

87-Fr-208



8

Incident Energy (MeV)

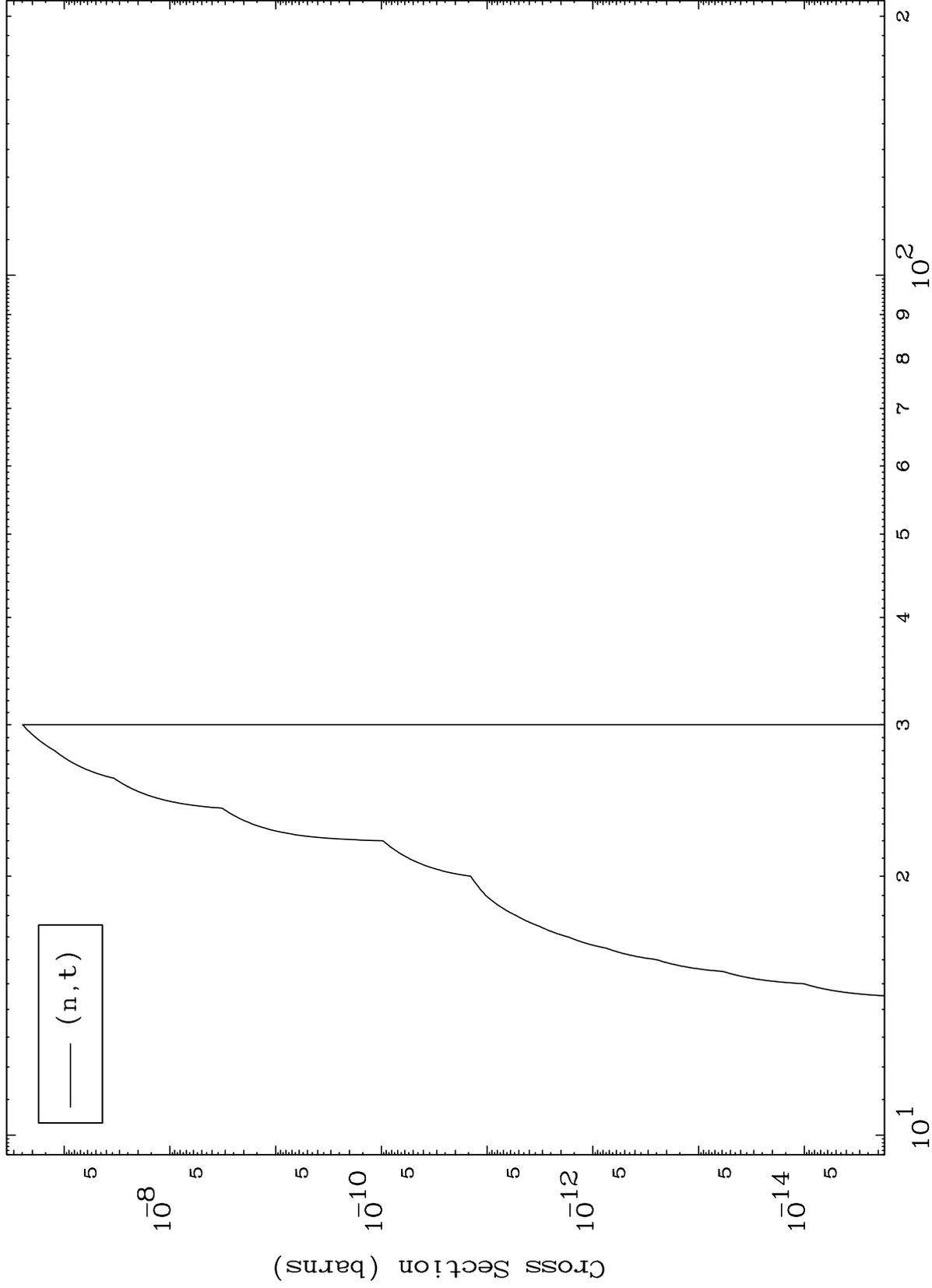
87-Fr-208

MAT 8713

( $\gamma, t$ ) Levels

87-Fr-208

0 Kelvin Cross Sections



Incident Energy (MeV)

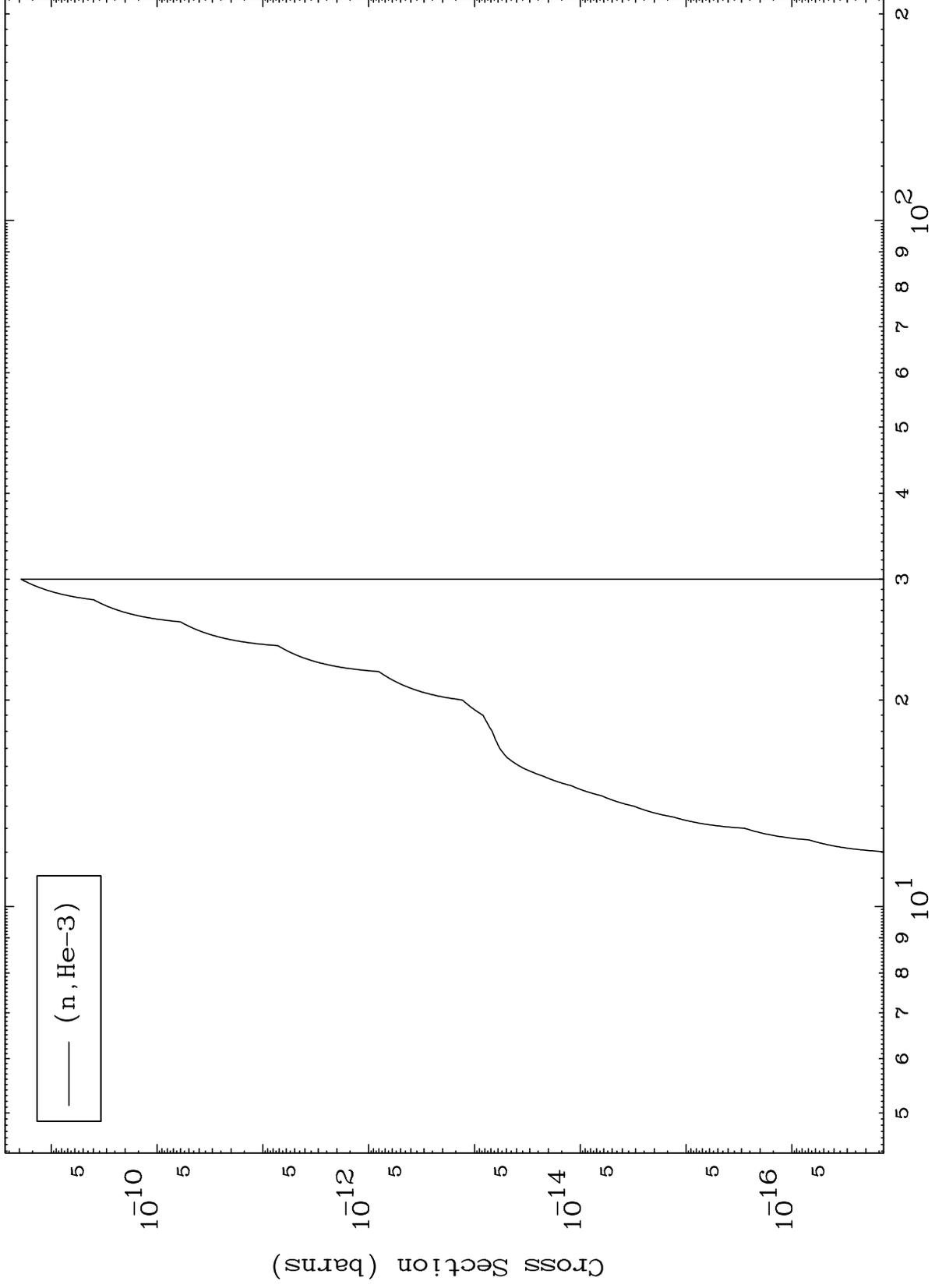
87-Fr-208

MAT 8713

( $\gamma, \text{He}3$ ) Levels

87-Fr-208

0 Kelvin Cross Sections



10

Incident Energy (MeV)

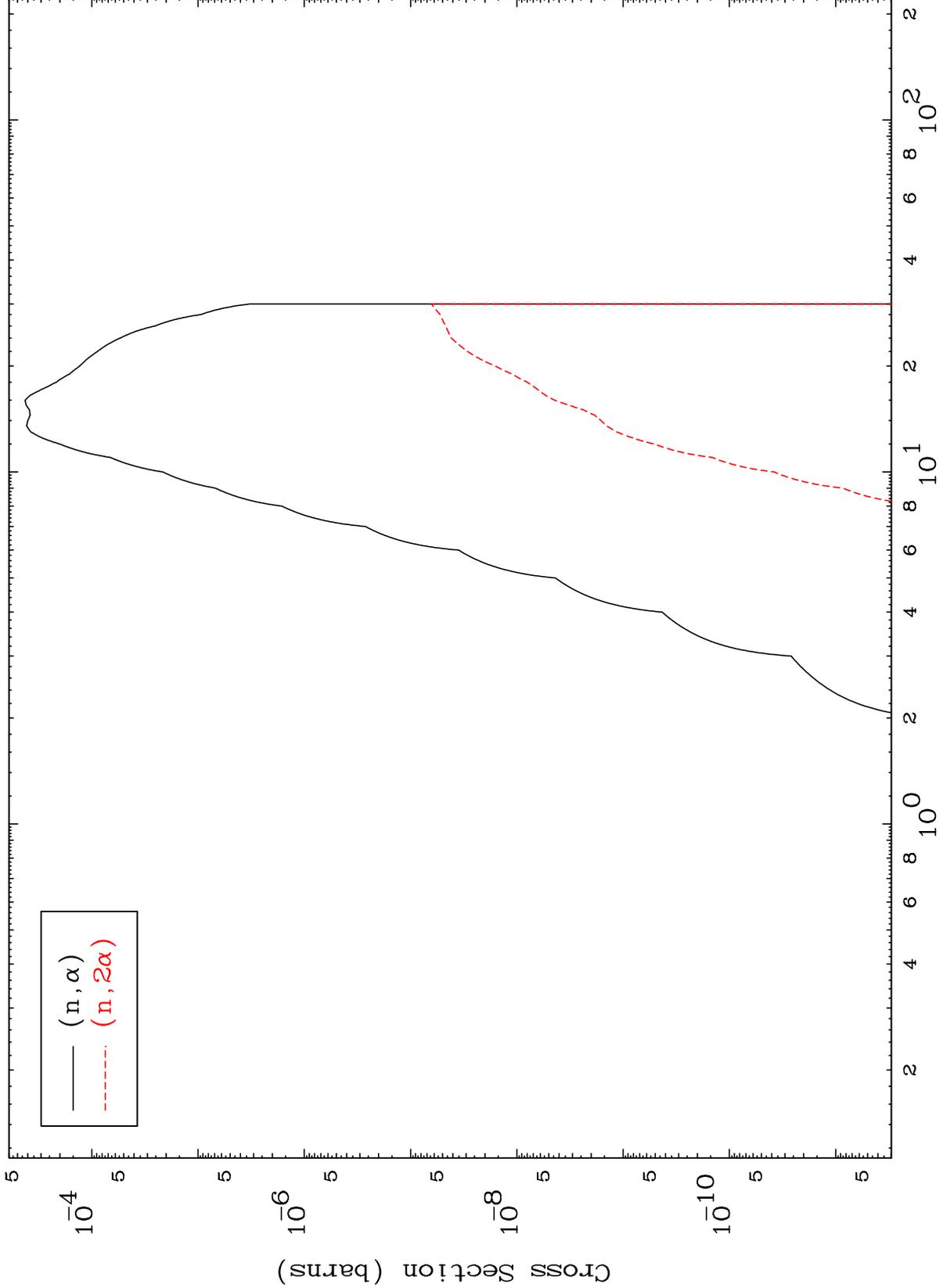
87-Fr-208

MAT 8713

( $\gamma, \alpha$ ) Levels

87-Fr-208

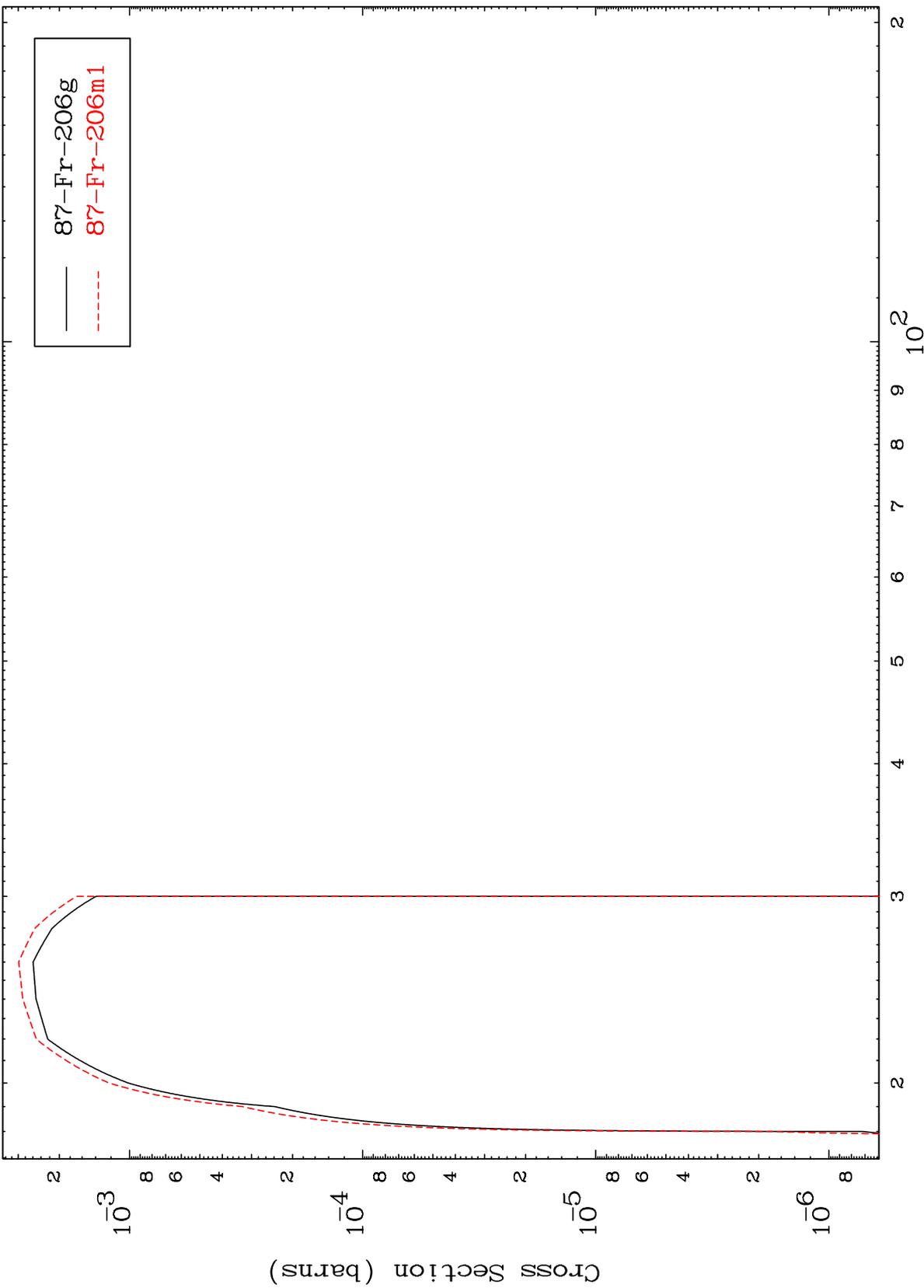
0 Kelvin Cross Sections



MAT 8713

87-Fr-208

(n,2n)  
Radionuclide Production Cross Section



12

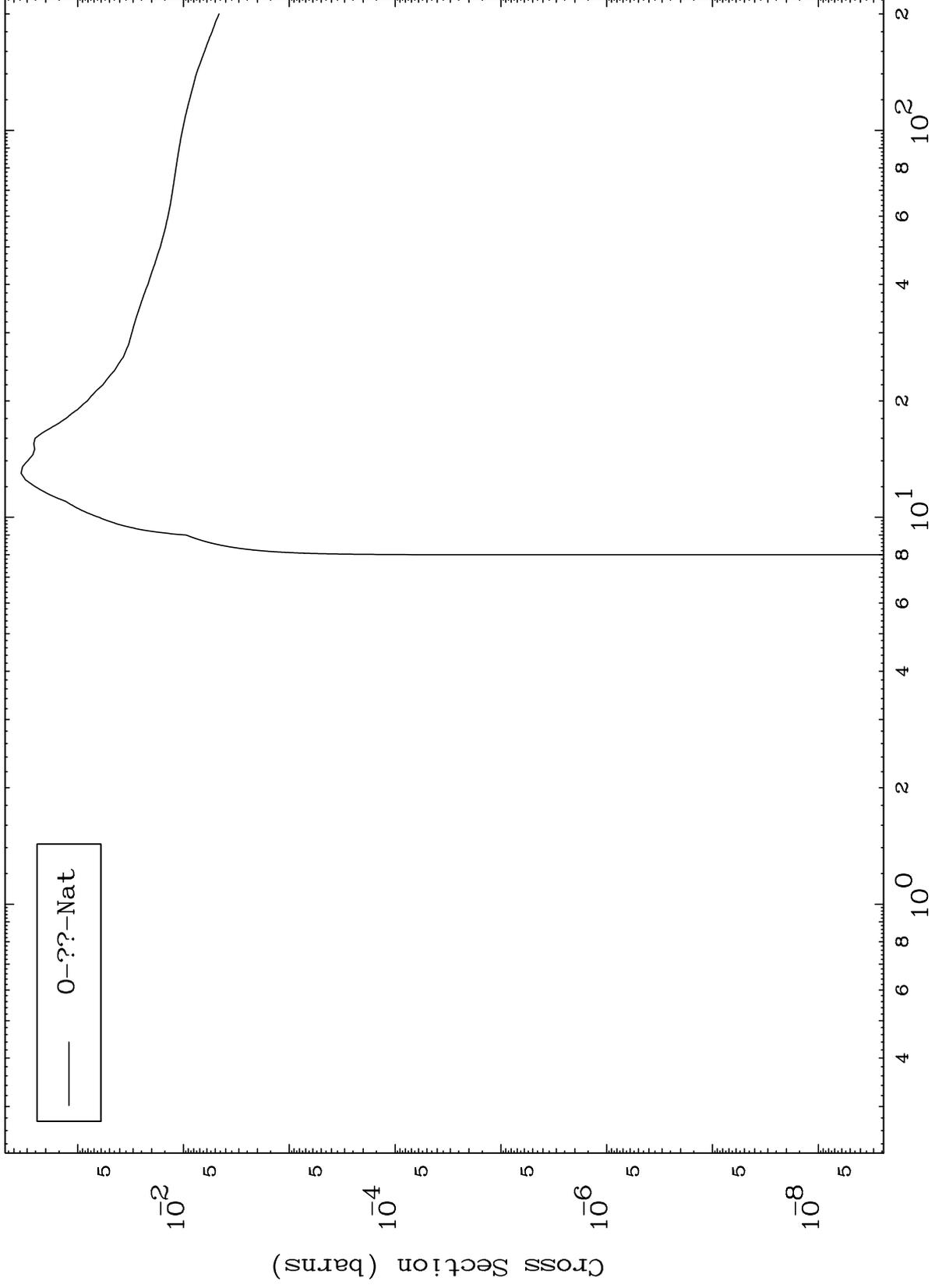
Incident Energy (MeV)

87-Fr-208

MAT 8713

87-Fr-208

Fission  
Radionuclide Production Cross Section



13

87-Fr-208

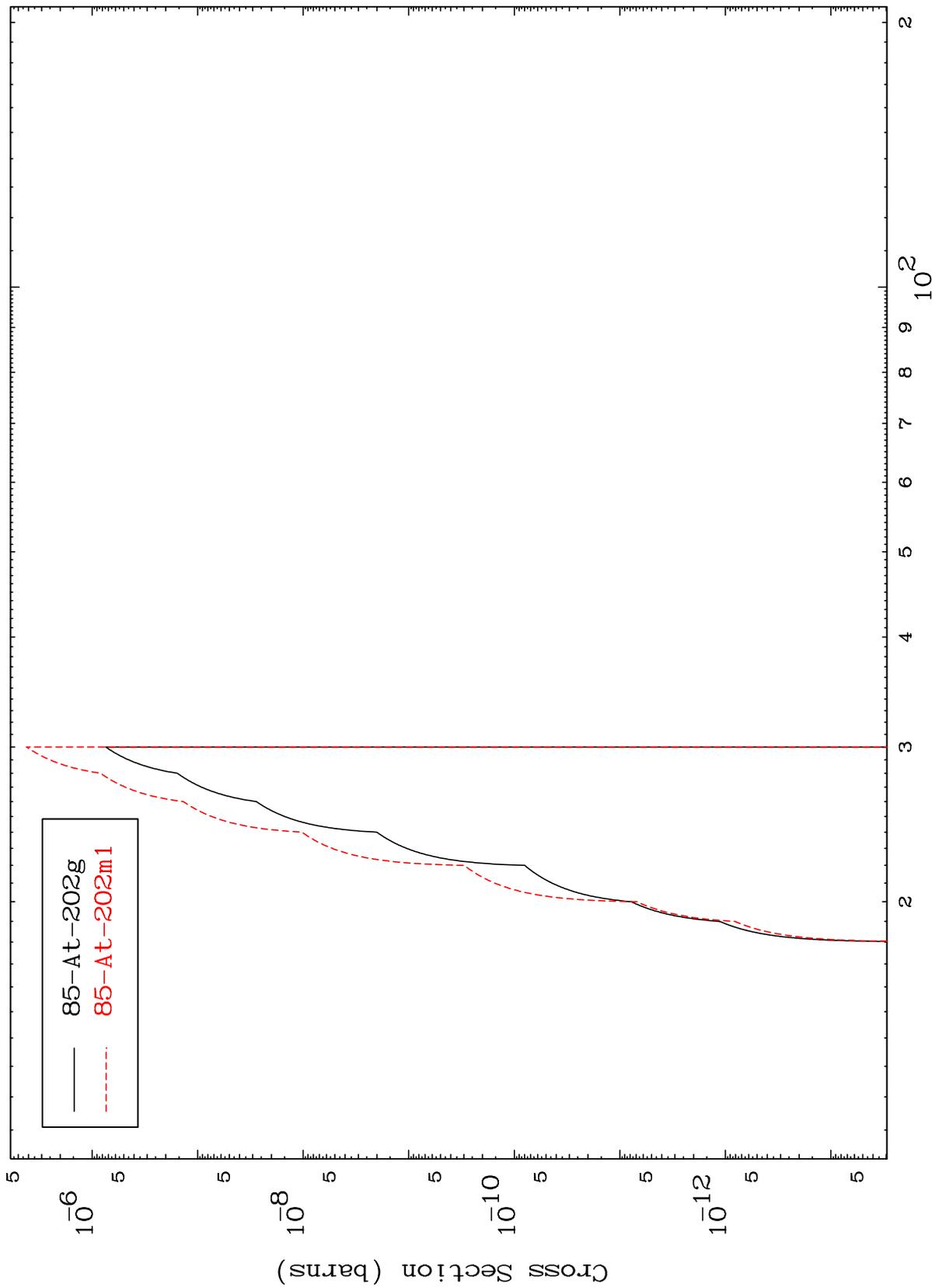
Incident Energy (MeV)

MAT 8713

(n,2n)  $\alpha$

87-Fr-208

Radionuclide Production Cross Section



Incident Energy (MeV)

87-Fr-208

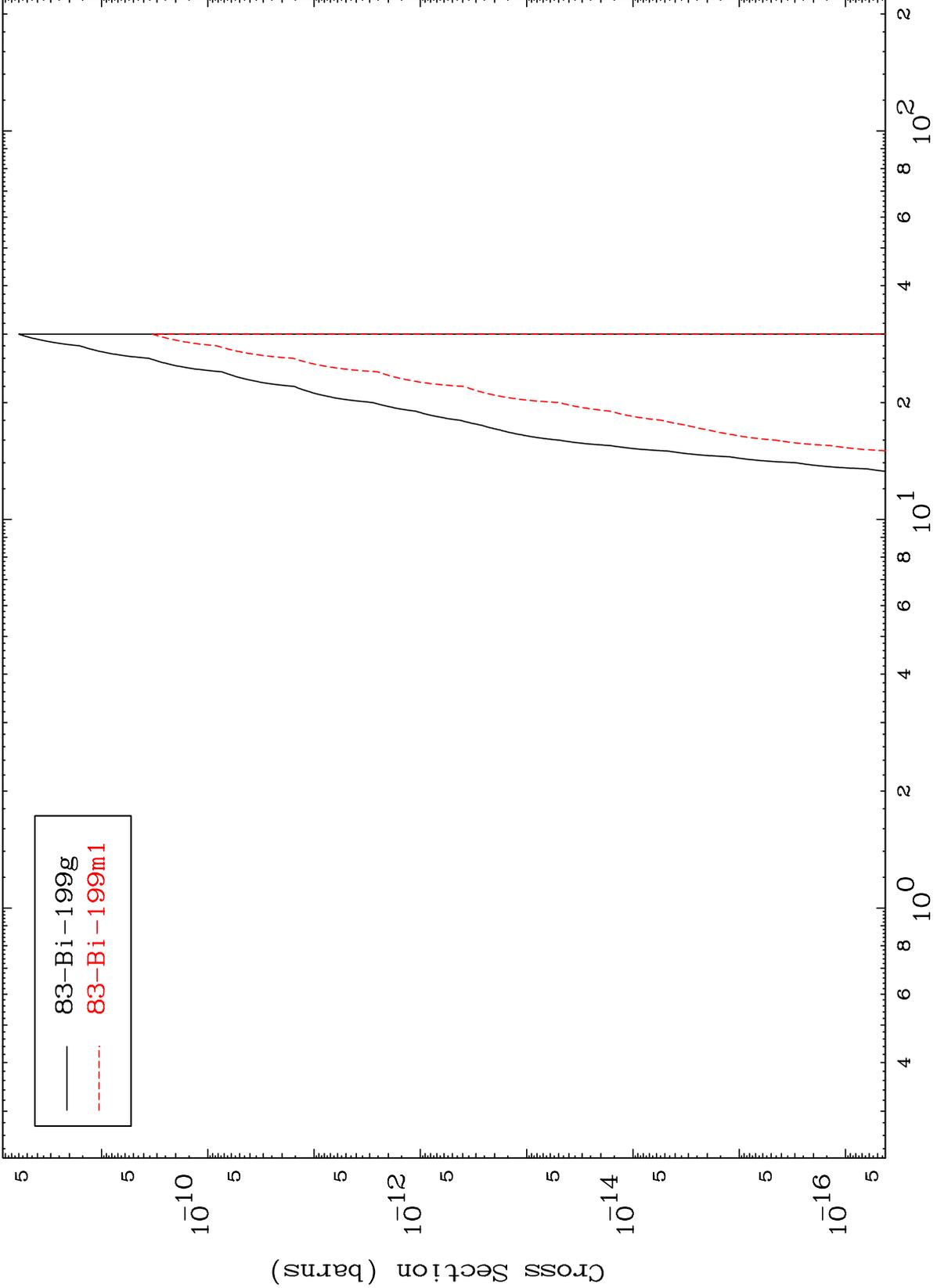
14

MAT 8713

(n,n') 2 $\alpha$

87-Fr-208

Radionuclide Production Cross Section



83-Bi-199g  
83-Bi-199m1

15

Incident Energy (MeV)

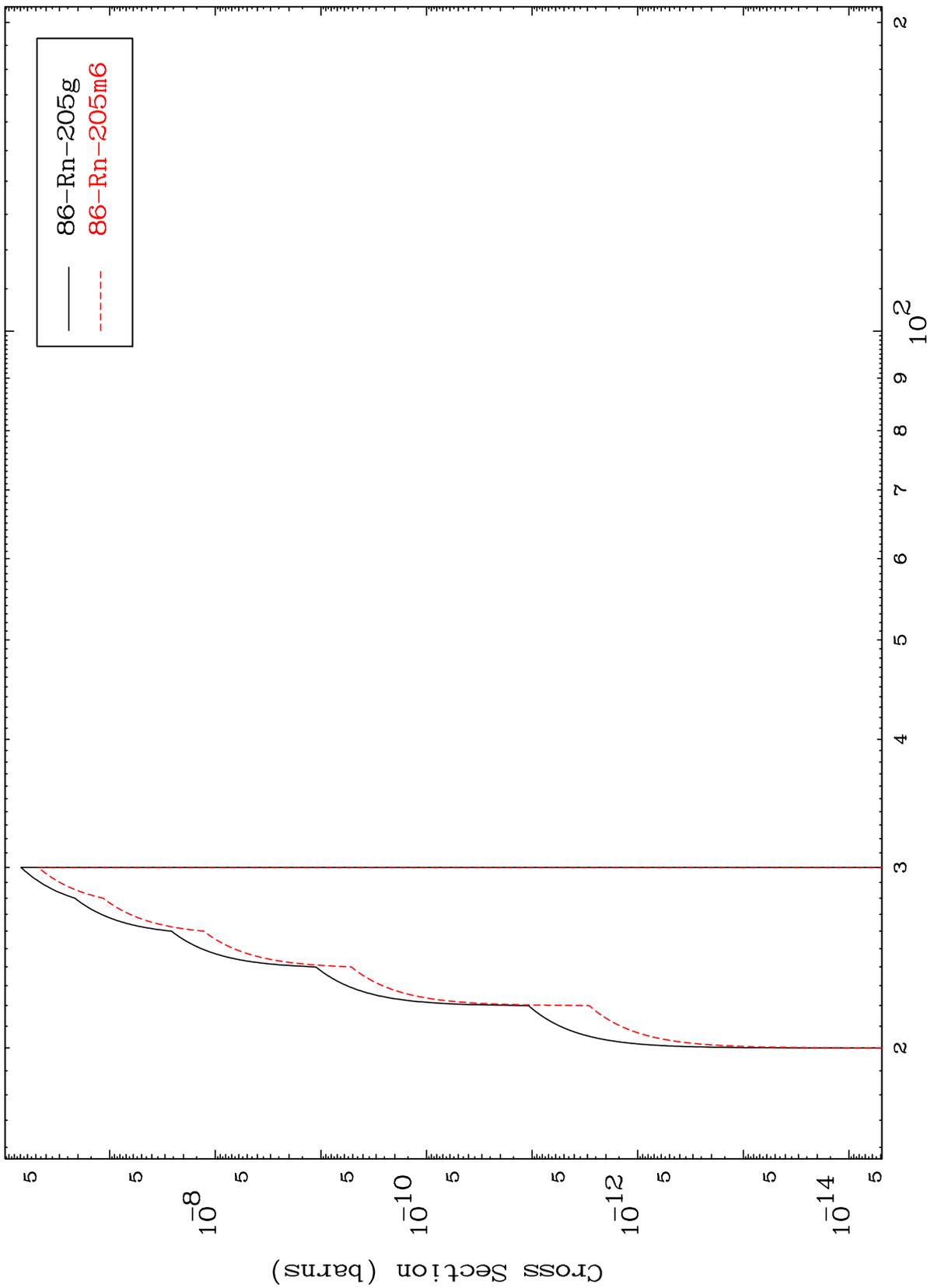
87-Fr-208

MAT 8713

(n,n') d

87-Fr-208

Radionuclide Production Cross Section



16

Incident Energy (MeV)

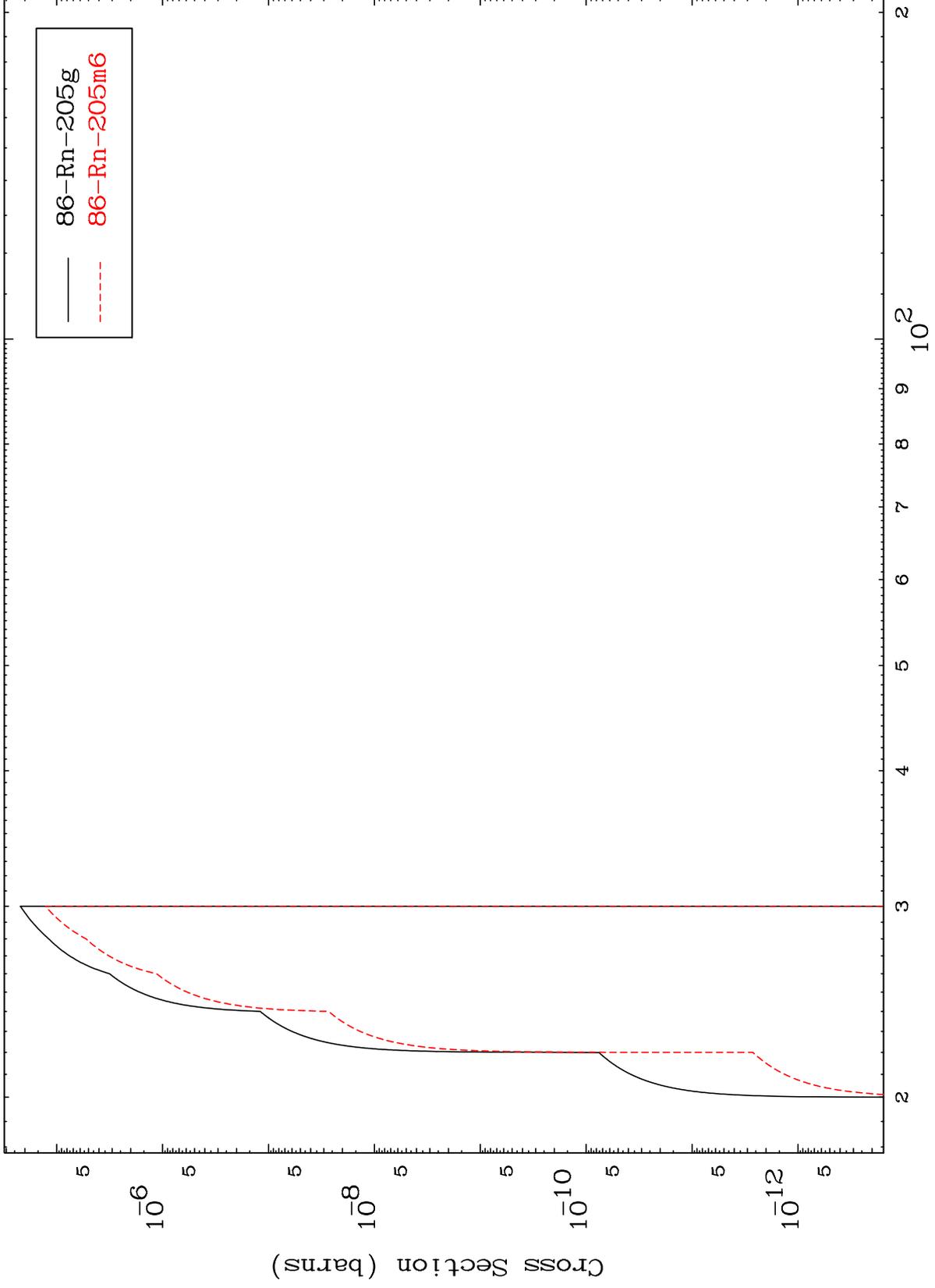
87-Fr-208

MAT 8713

(n,2n) p

87-Fr-208

Radionuclide Production Cross Section



17

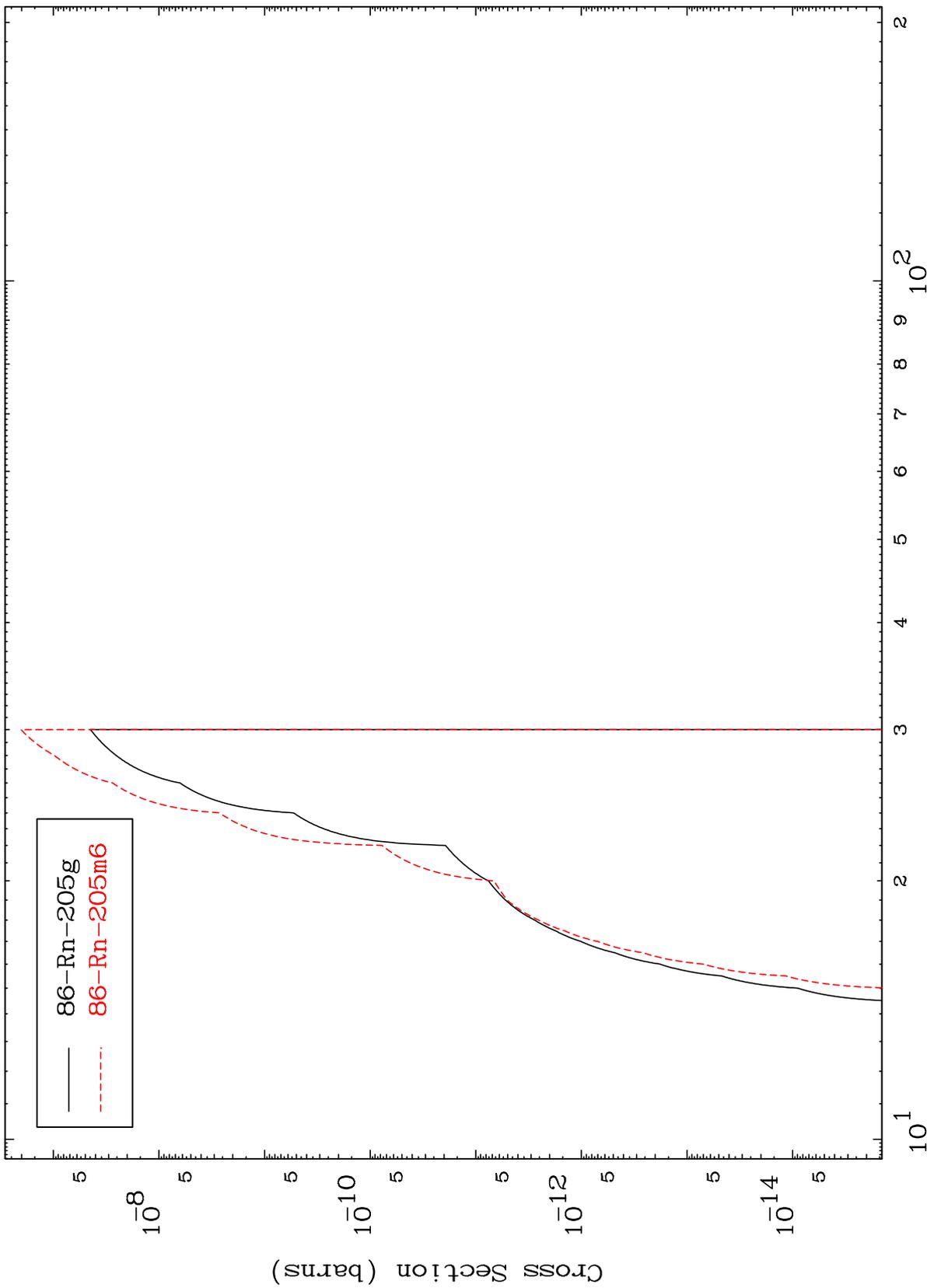
Incident Energy (MeV)

87-Fr-208

MAT 8713

87-Fr-208

(n,t)  
Radionuclide Production Cross Section



87-Fr-208

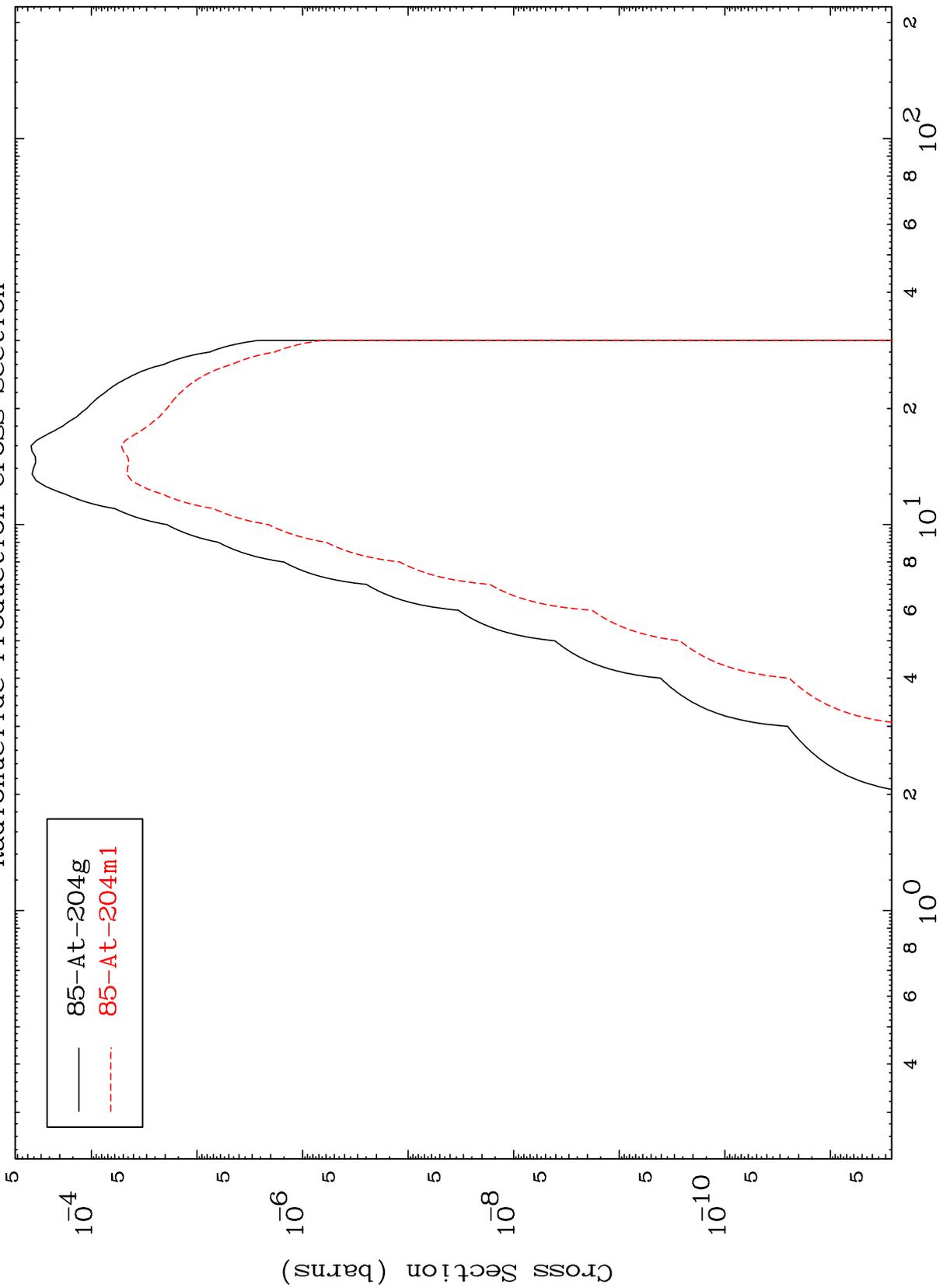
Incident Energy (MeV)

18

MAT 8713

87-Fr-208

(n,  $\alpha$ )  
Radionuclide Production Cross Section



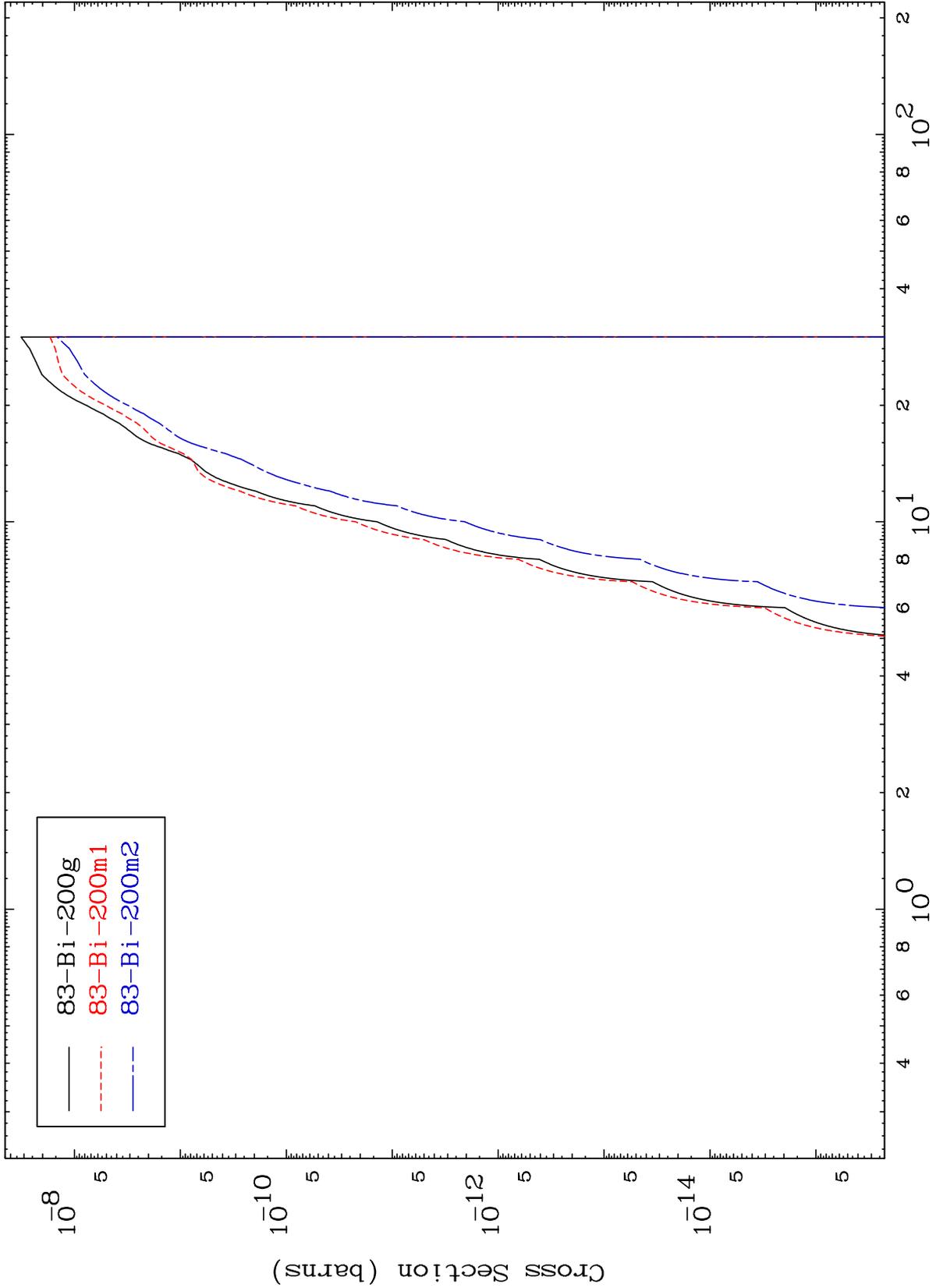
— 85-At-204g  
- - - 85-At-204m1

MAT 8713

(n,2α)

87-Fr-208

Radionuclide Production Cross Section



20

Incident Energy (MeV)

87-Fr-208

