

Program EVALPLOT
(Version 2021-1)

by

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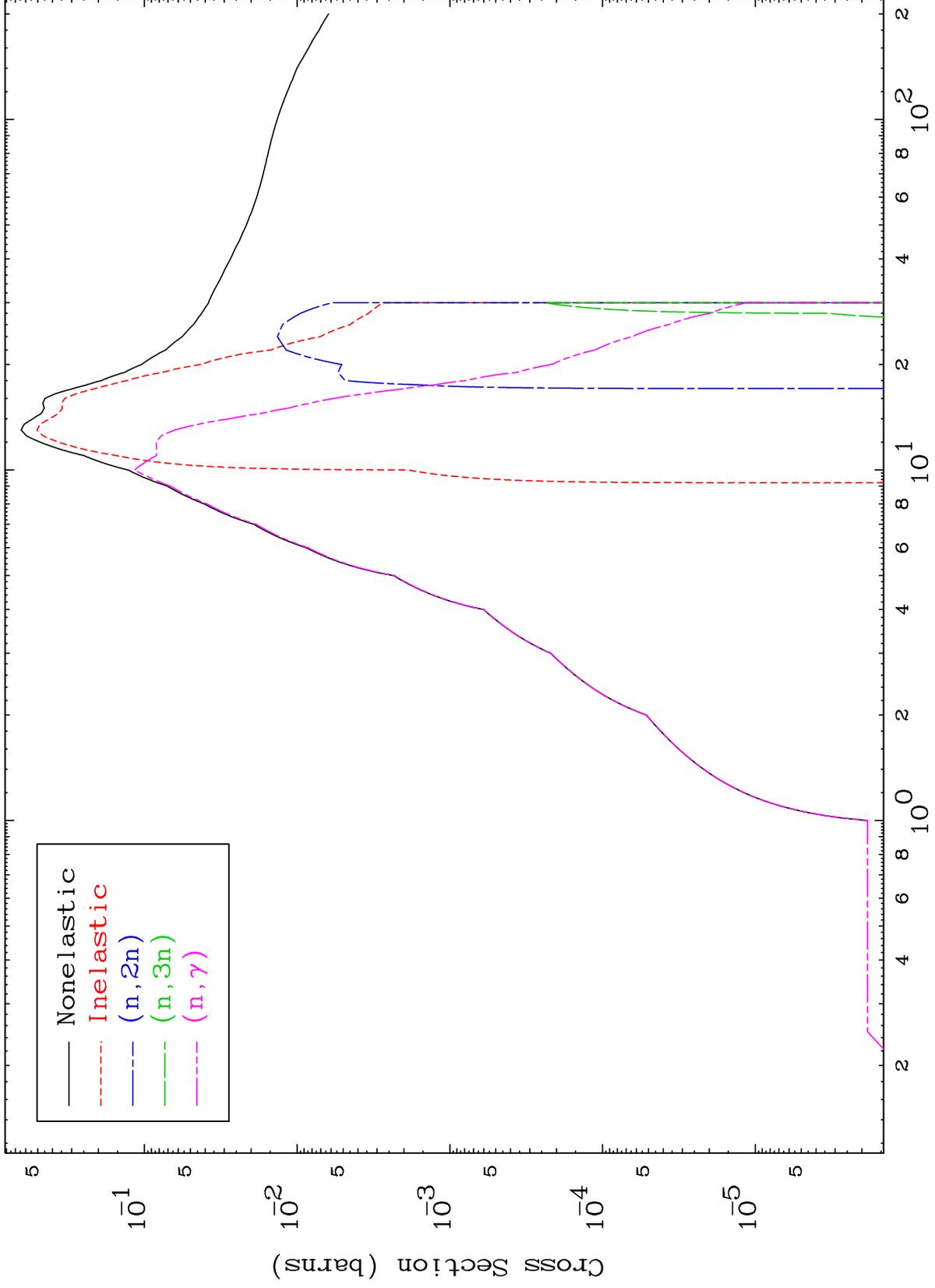
Press Mouse Button to Start

MAT 8716

Photon Major

87-Fr-209

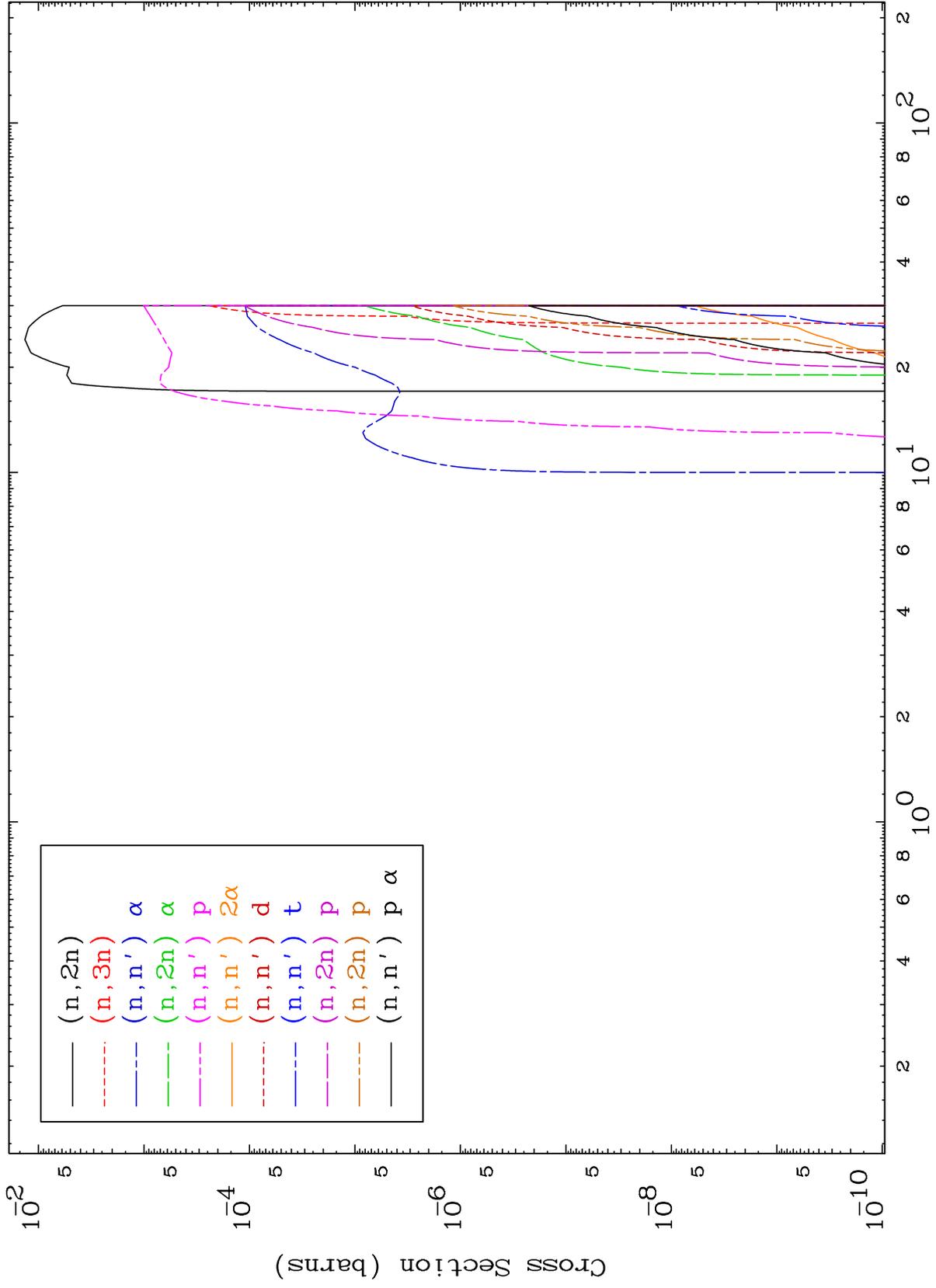
0 Kelvin Cross Sections



MAT 8716

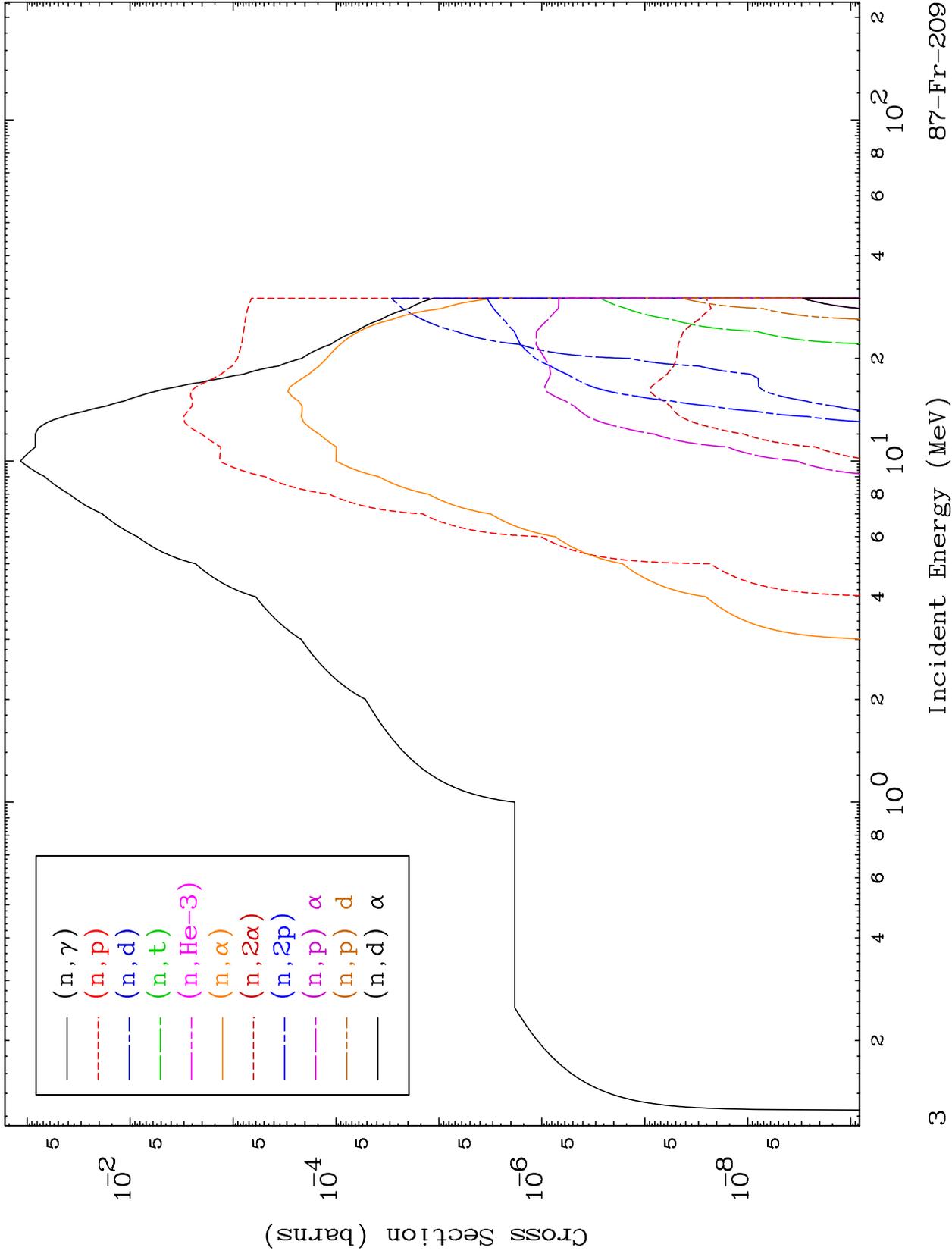
Photon Neutron Absorption
0 Kelvin Cross Sections

87-Fr-209



87-Fr-209

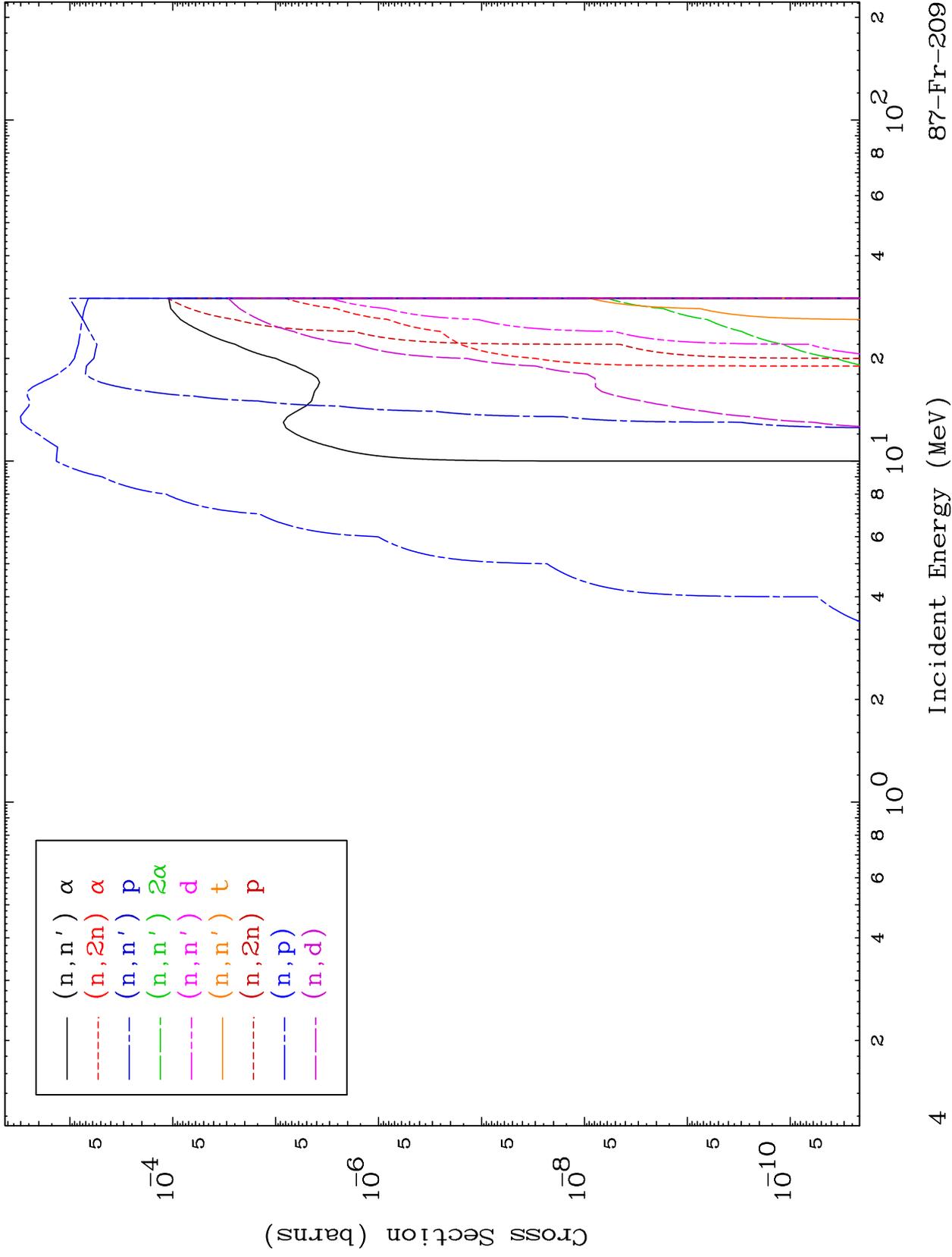
Incident Energy (MeV)

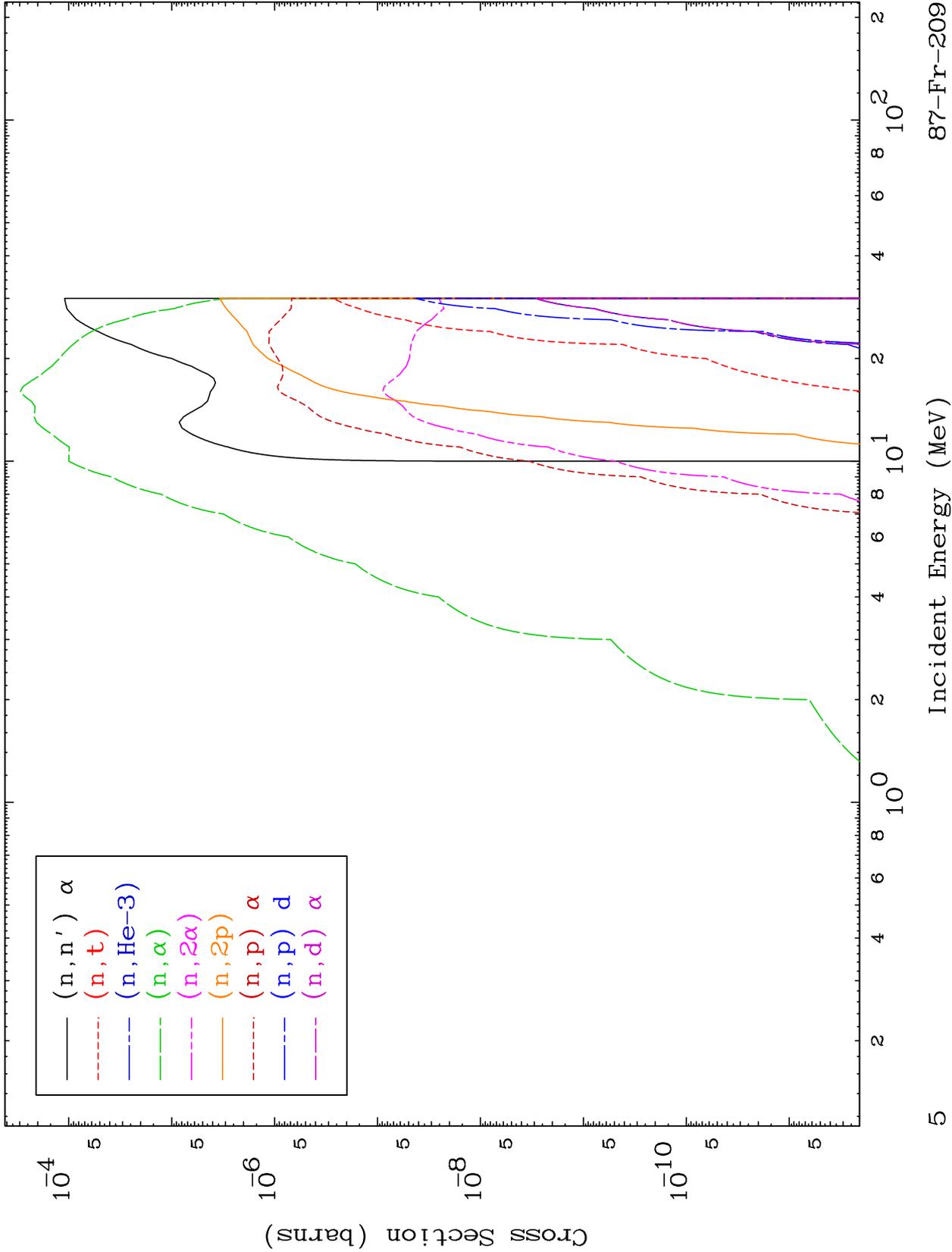


MAT 8716

Photon Charged Particle
0 Kelvin Cross Sections

87-Fr-209



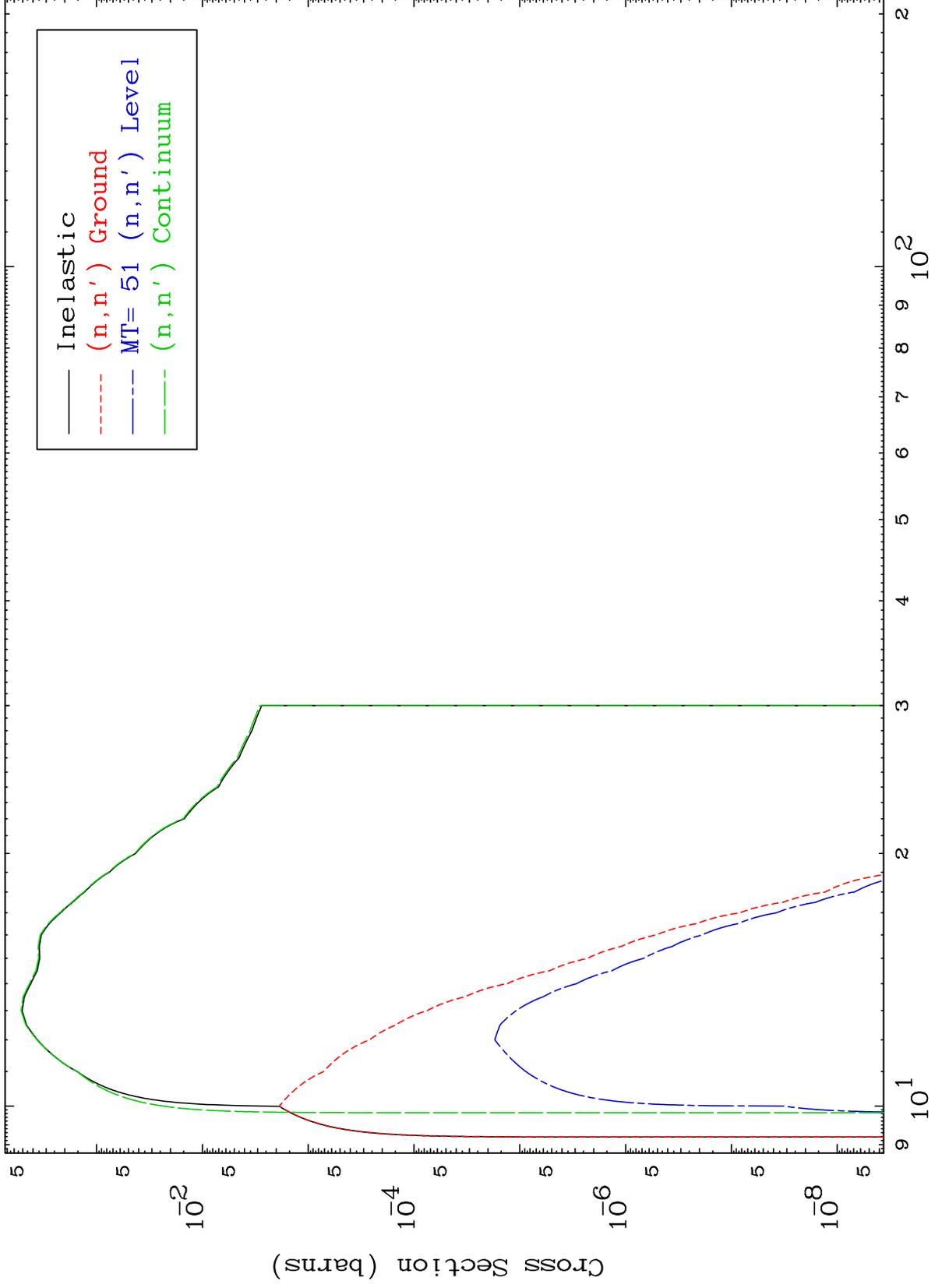


MAT 8716

(γ, n') Levels

87-Fr-209

0 Kelvin Cross Sections



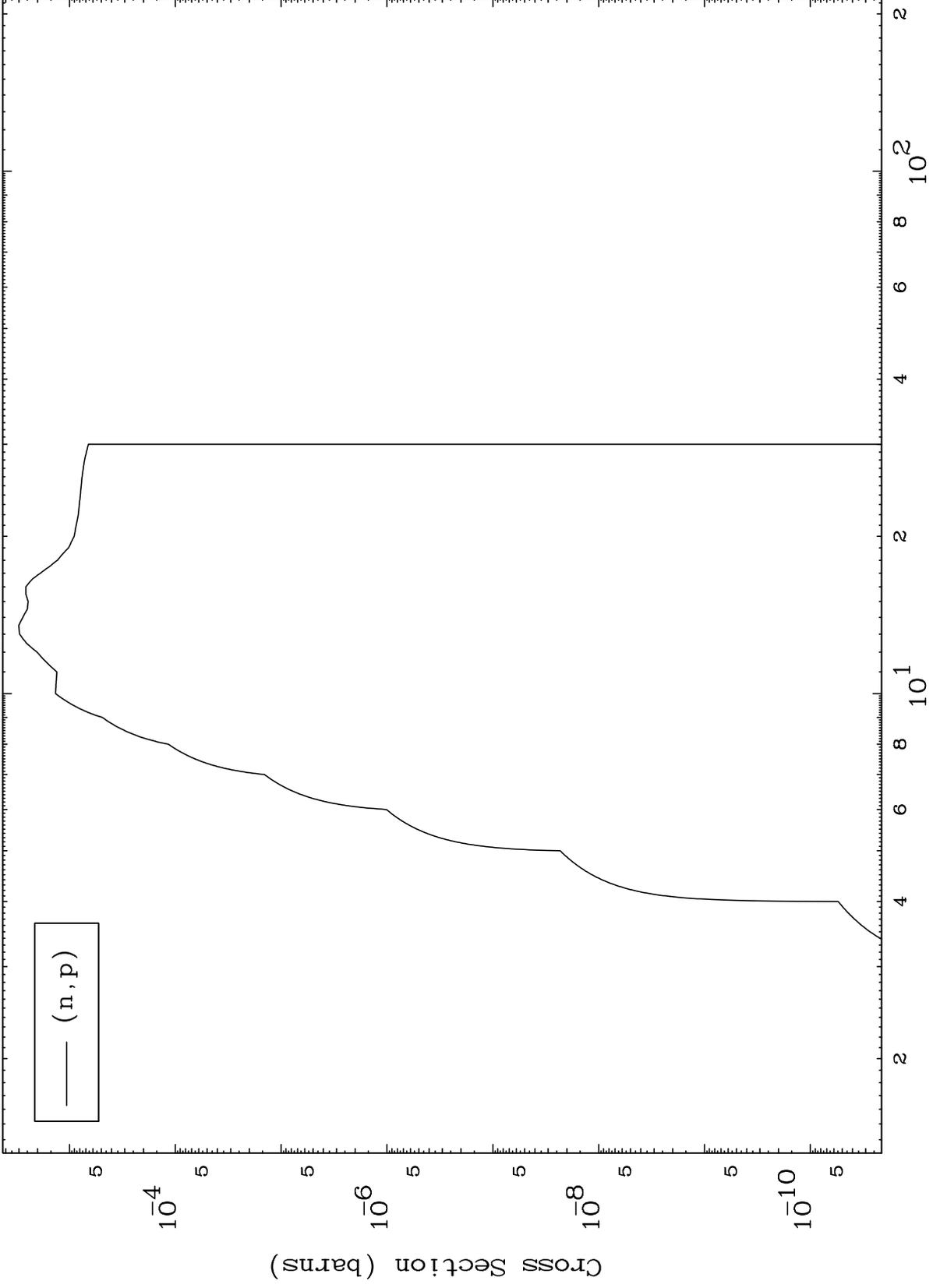
Incident Energy (MeV)

87-Fr-209

MAT 8716

87-Fr-209

(γ, p) Levels
0 Kelvin Cross Sections



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87-Fr-209

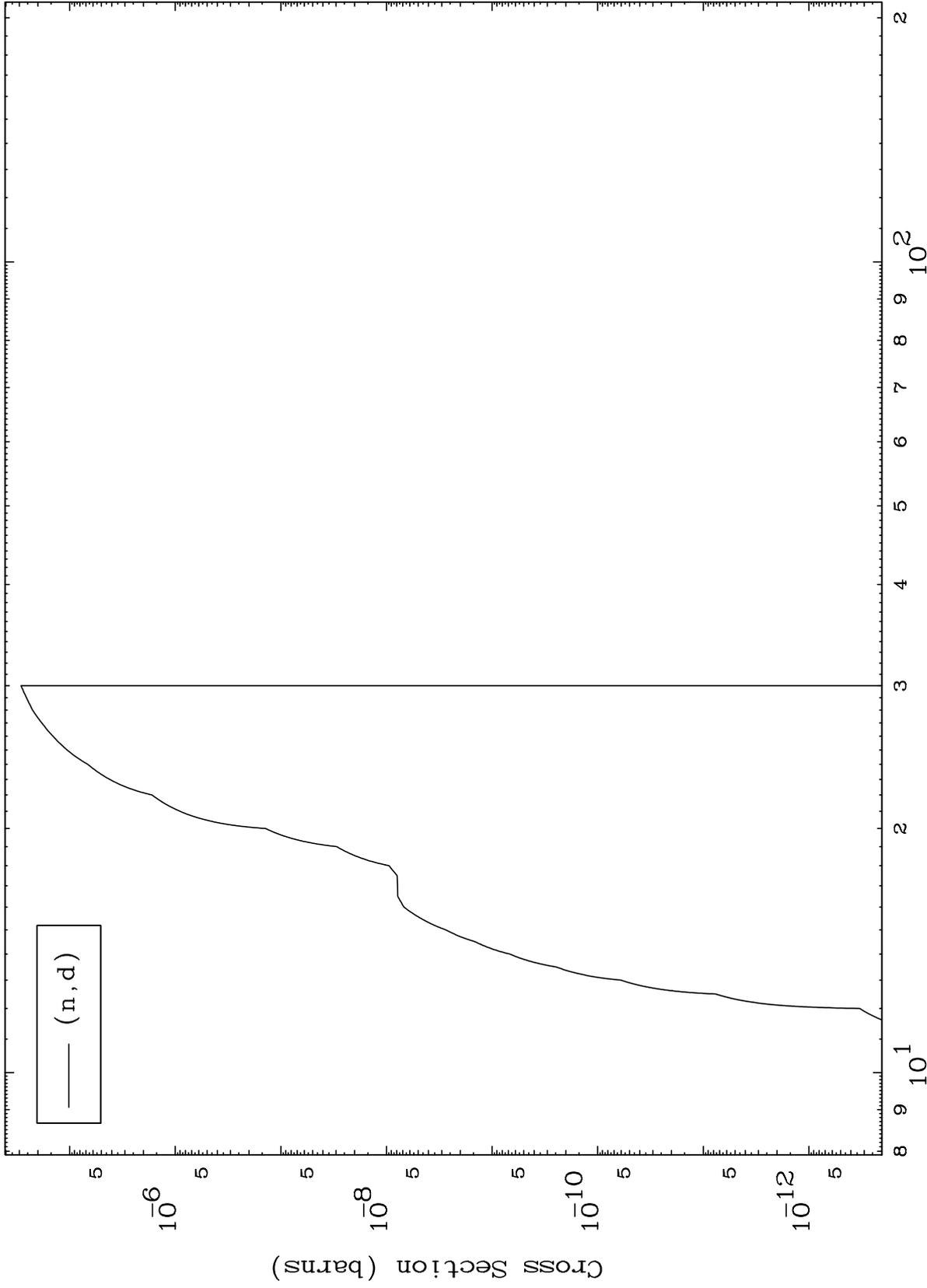
Incident Energy (MeV)

MAT 8716

(γ, d) Levels

87-Fr-209

0 Kelvin Cross Sections



8

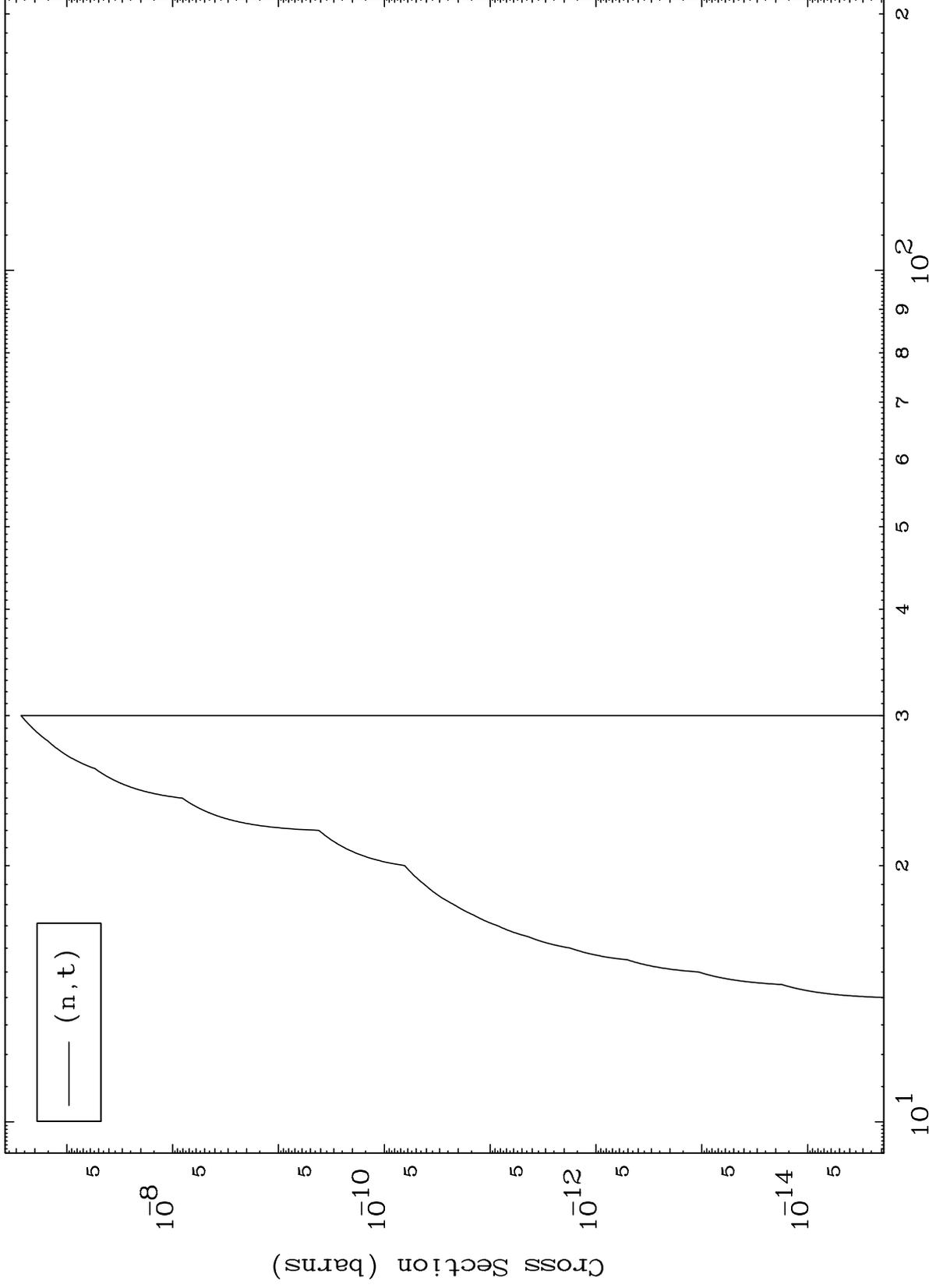
Incident Energy (MeV)

87-Fr-209

MAT 8716

(γ, t) Levels
0 Kelvin Cross Sections

87-Fr-209



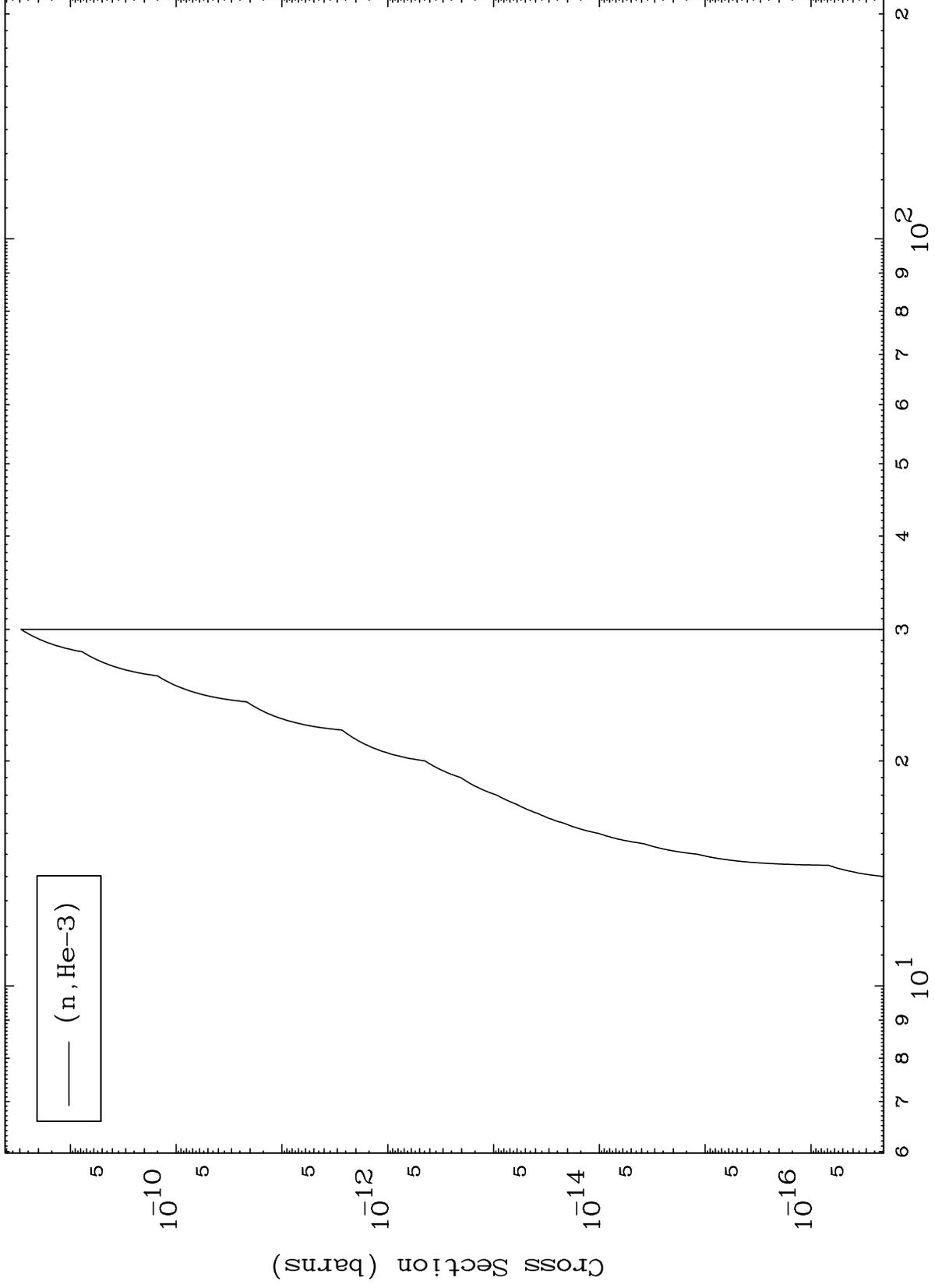
Incident Energy (MeV)

87-Fr-209

MAT 8716

(γ , He3) Levels
0 Kelvin Cross Sections

87-Fr-209



10

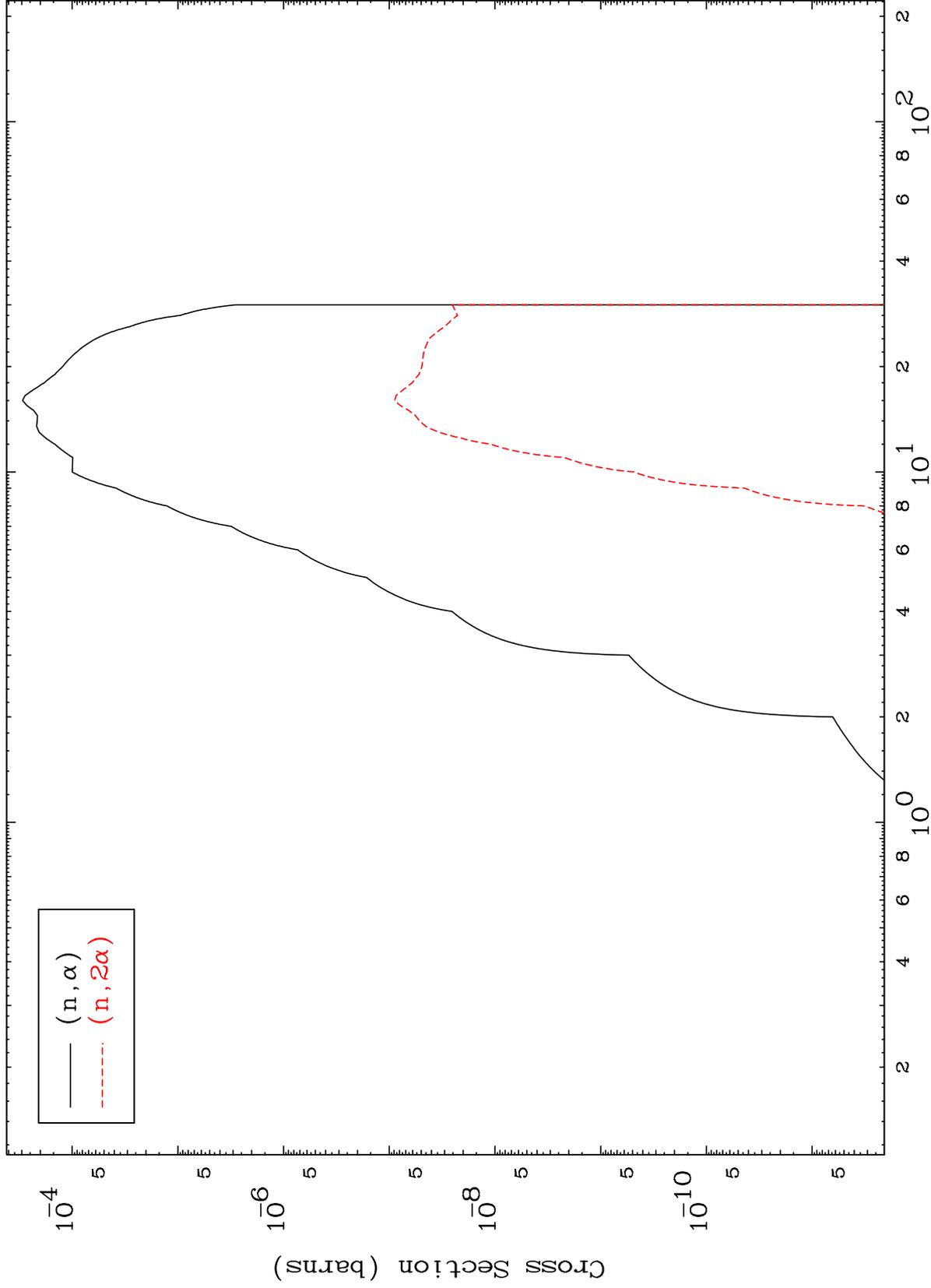
Incident Energy (MeV)

87-Fr-209

MAT 8716

87-Fr-209

(γ, α) Levels
0 Kelvin Cross Sections



11

87-Fr-209

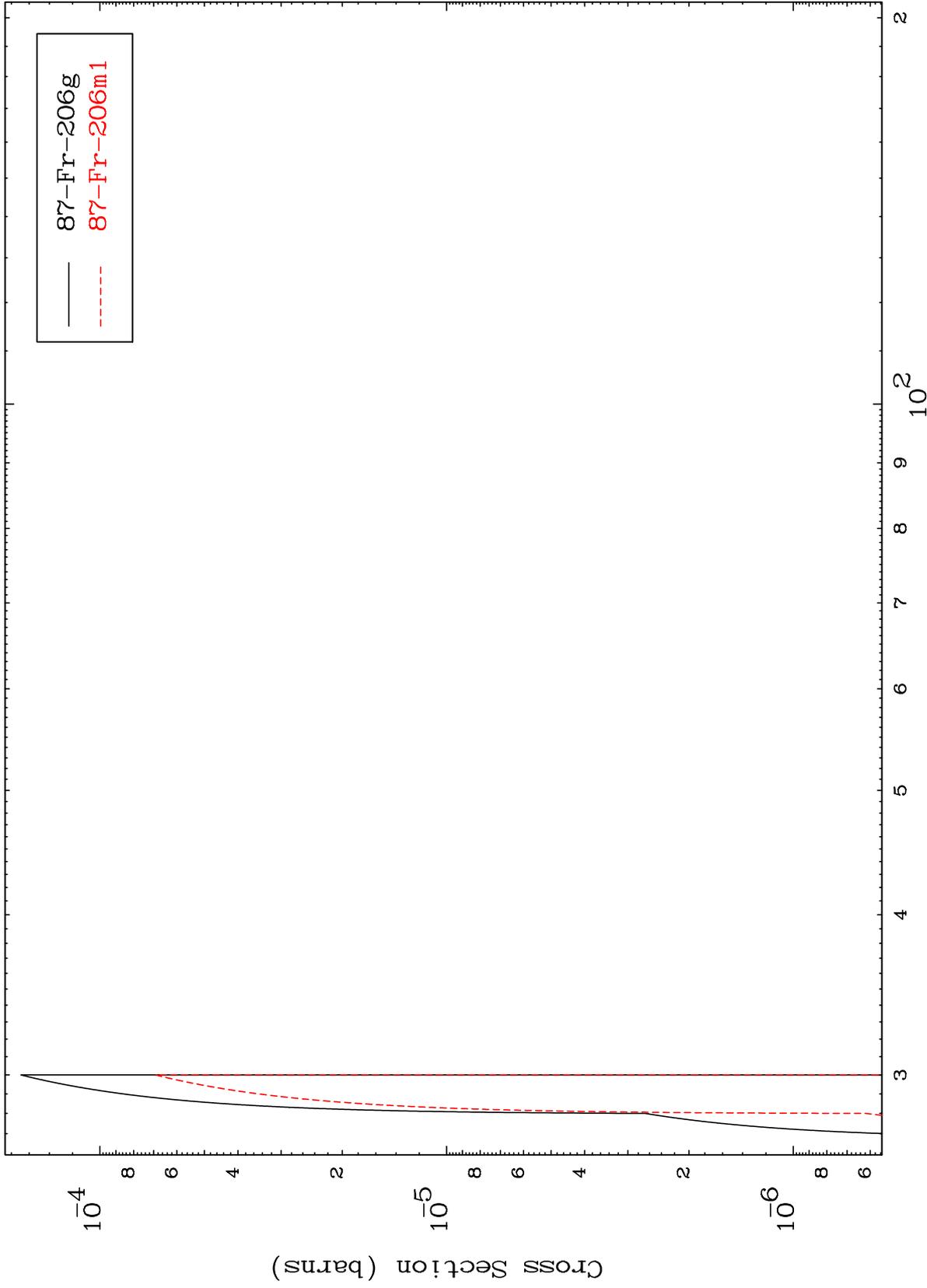
Incident Energy (MeV)

MAT 8716

(n,3n)

87-Fr-209

Radionuclide Production Cross Section



12

Incident Energy (MeV)

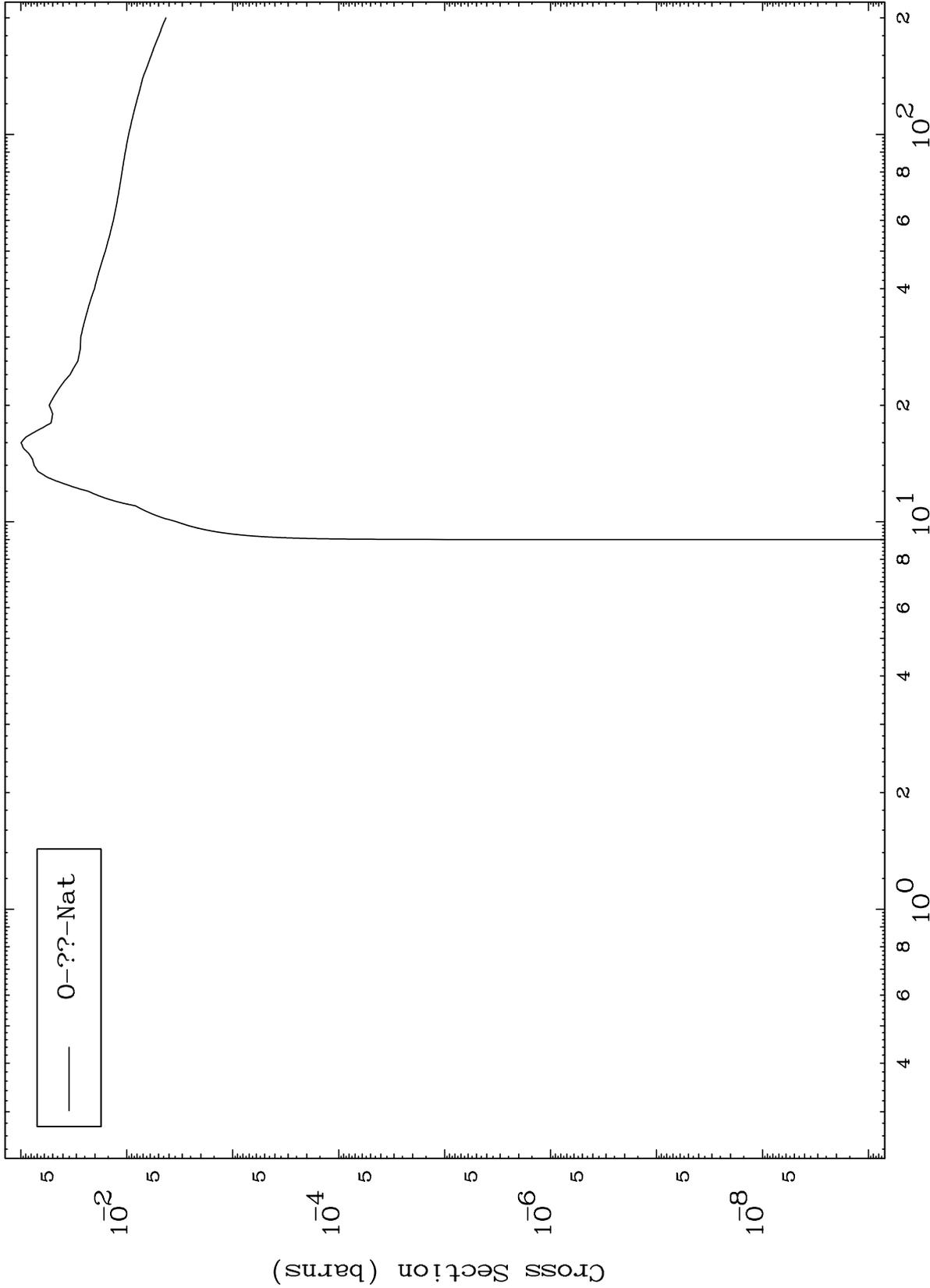
87-Fr-209

MAT 8716

Fission

87-Fr-209

Radionuclide Production Cross Section

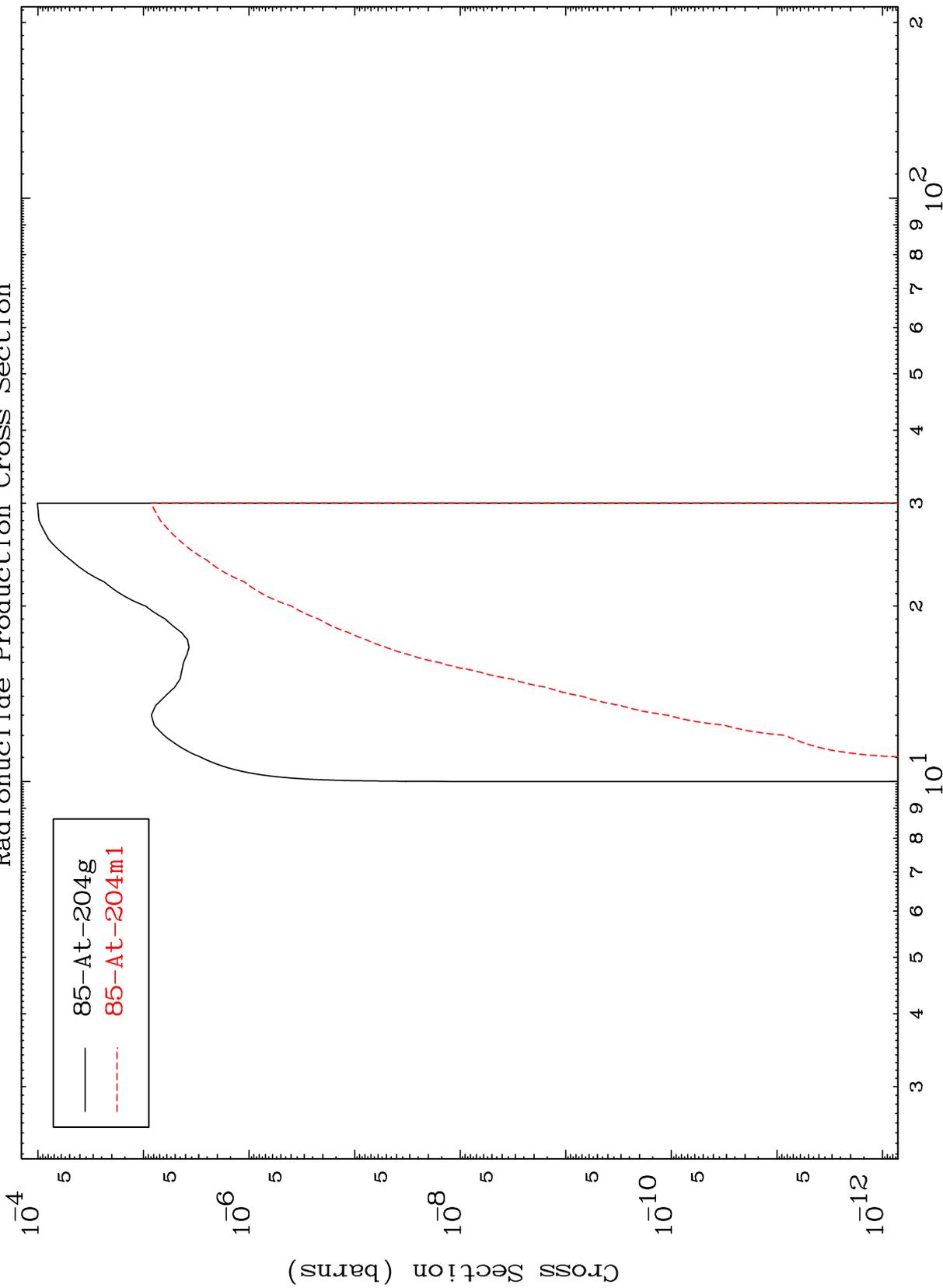


MAT 8716

$(n, n') \alpha$

87-Fr-209

Radionuclide Production Cross Section



14

Incident Energy (MeV)

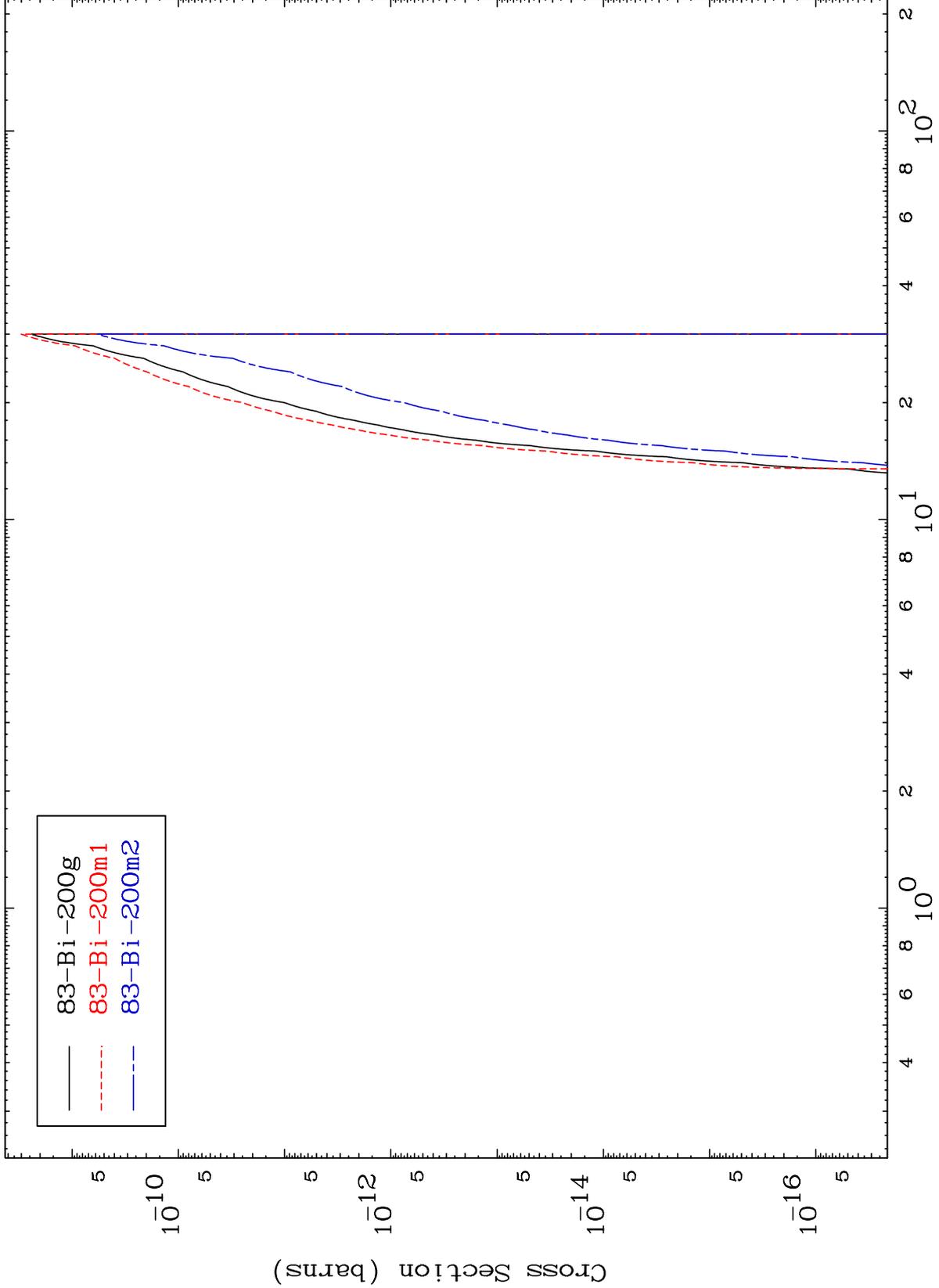
87-Fr-209

MAT 8716

(n,n') 2α

87-Fr-209

Radionuclide Production Cross Section



15

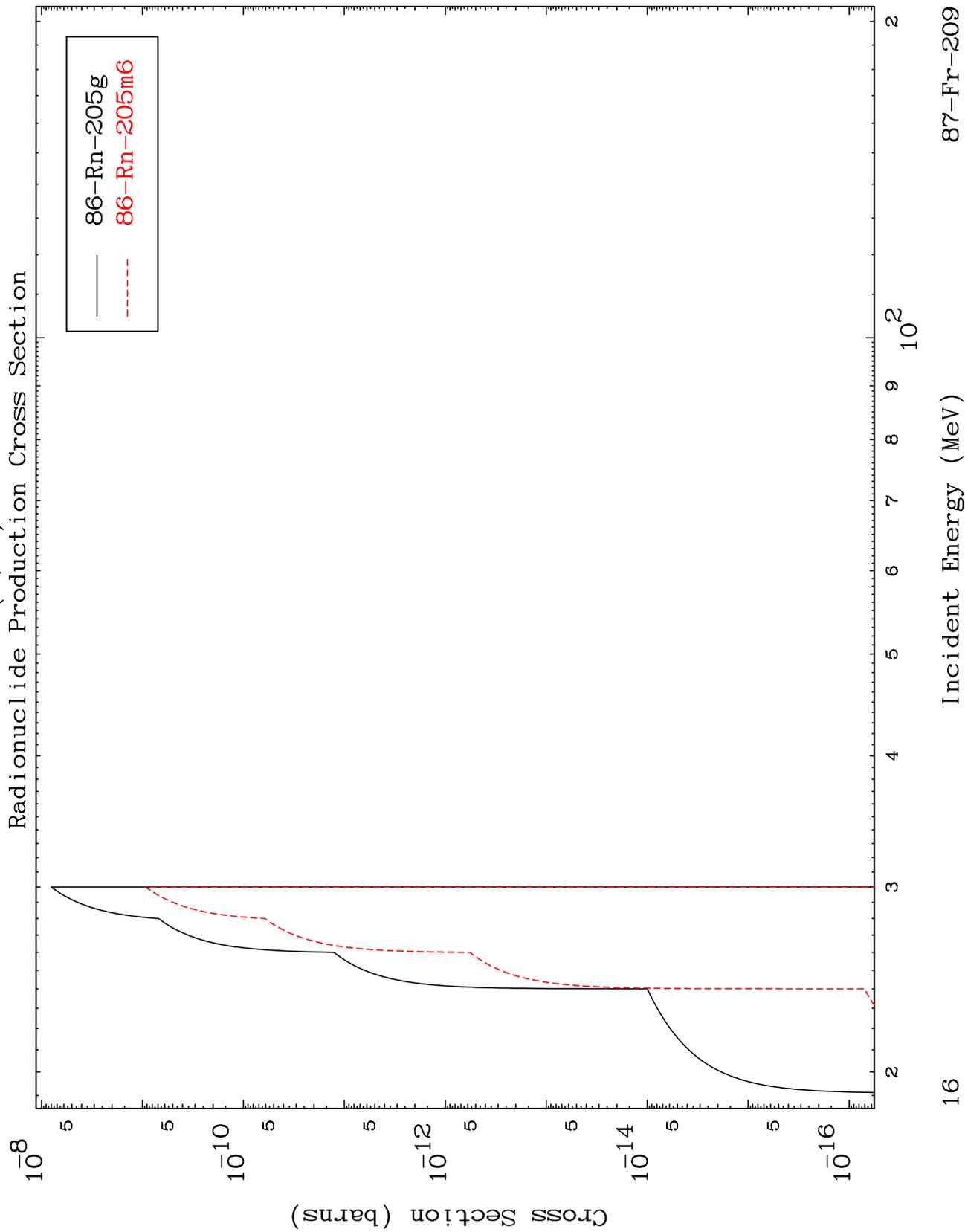
Incident Energy (MeV)

87-Fr-209

MAT 8716

(n,n') t

87-Fr-209



16

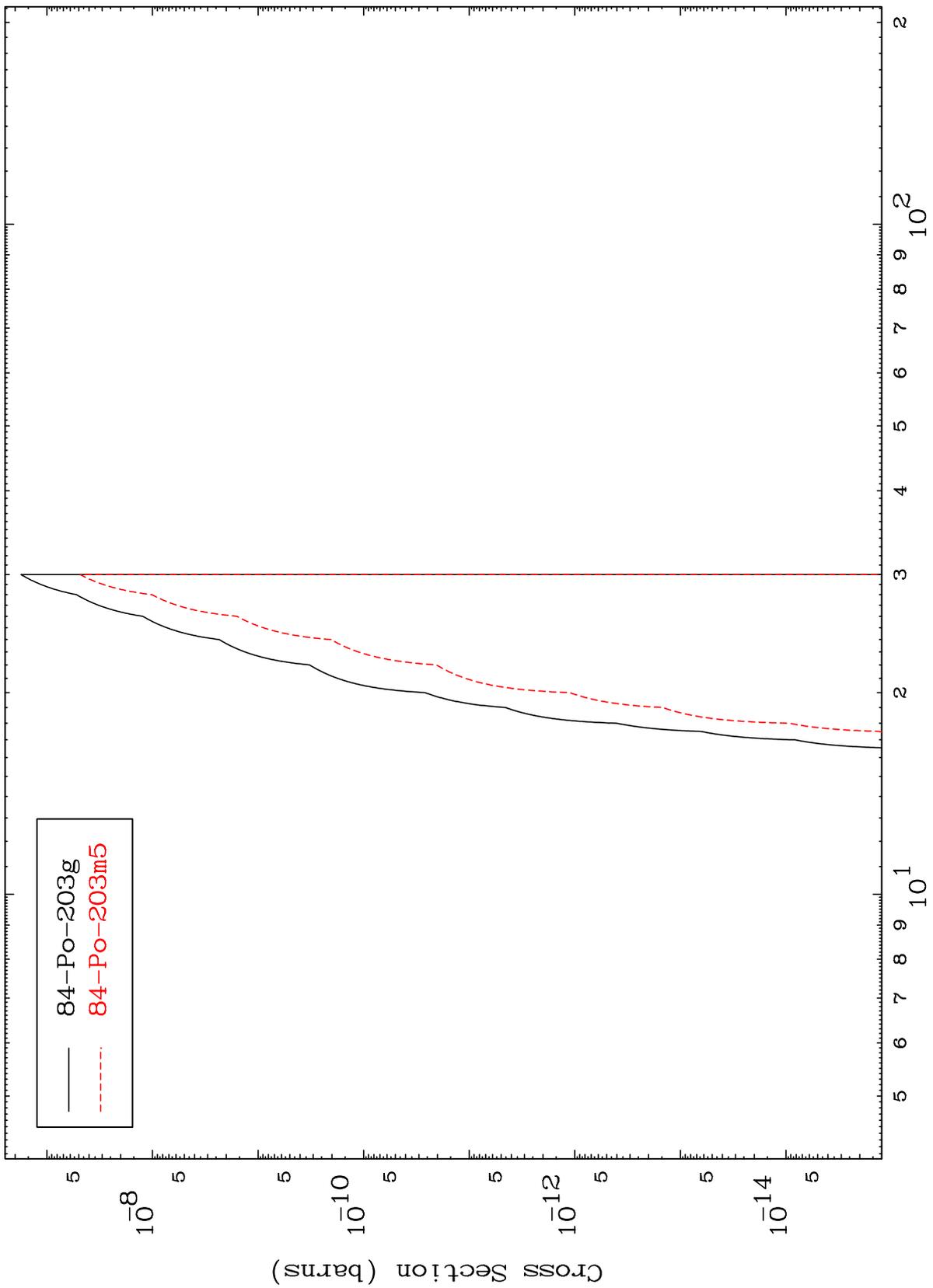
87-Fr-209

MAT 8716

(n,n') p α

87-Fr-209

Radionuclide Production Cross Section



84-Po-203g
84-Po-203m5

17

Incident Energy (MeV)

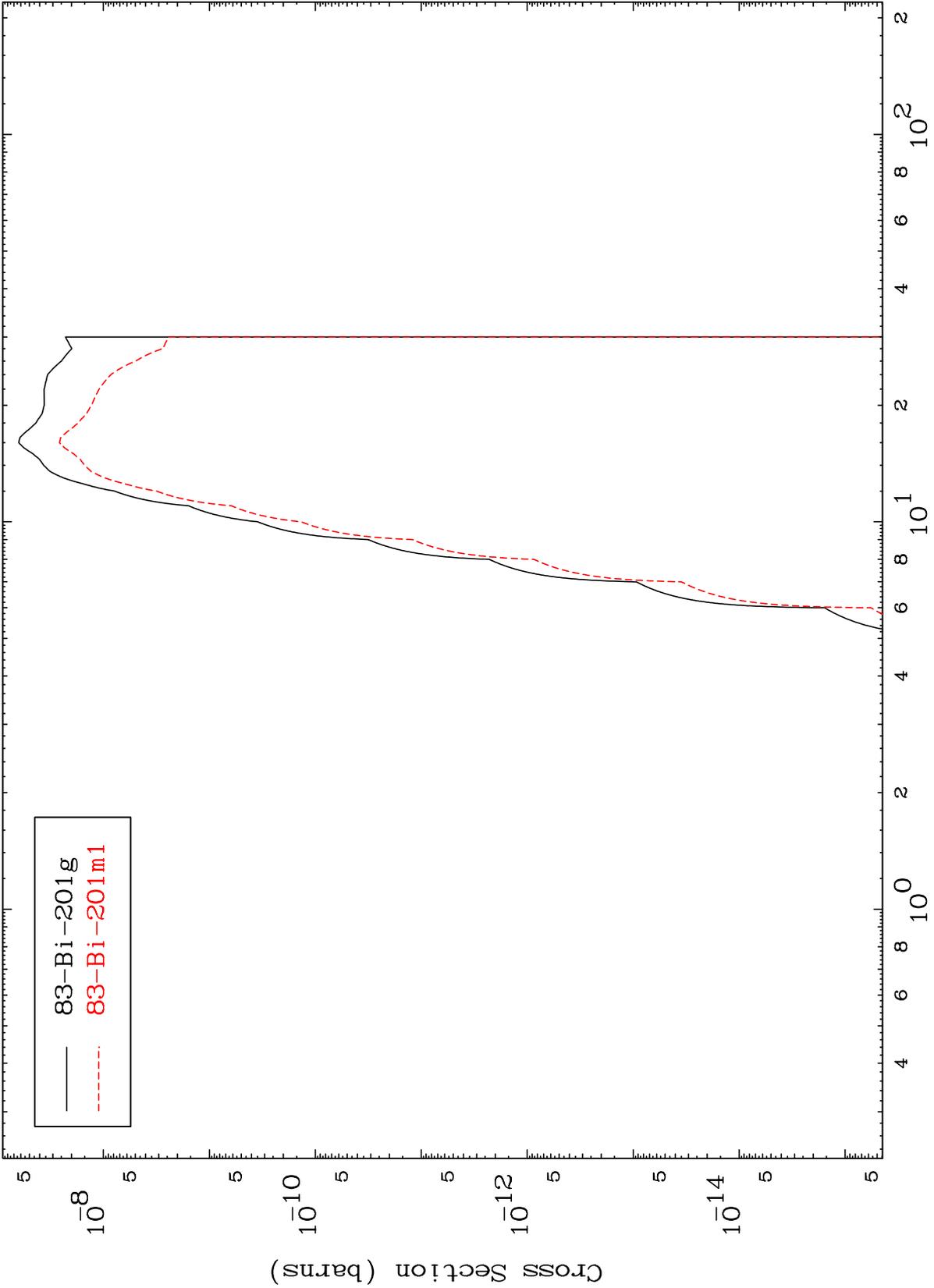
87-Fr-209

MAT 8716

(n,2α)

87-Fr-209

Radionuclide Production Cross Section



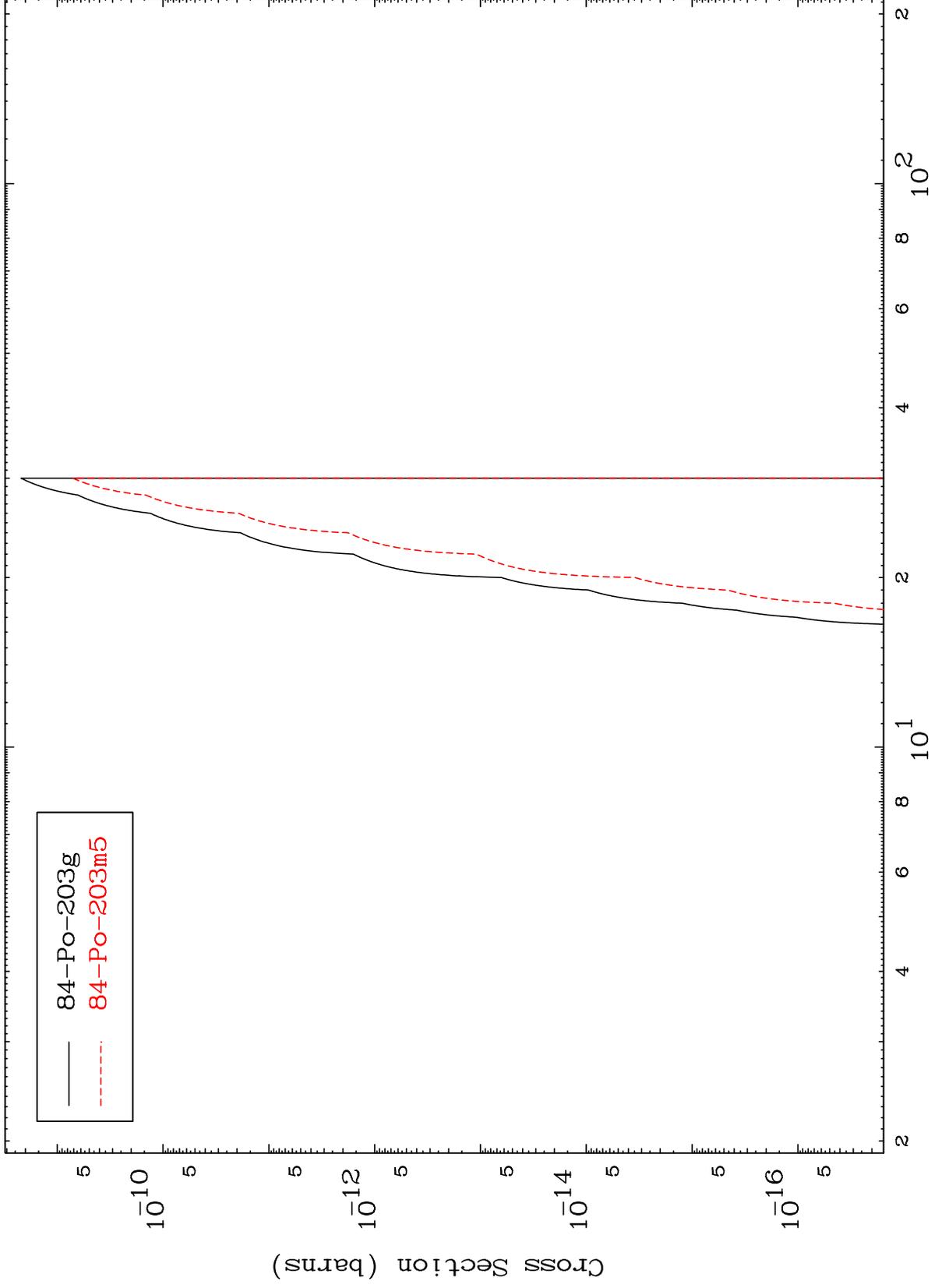
83-Bi-201g
83-Bi-201m1

MAT 8716

(n,d) α

87-Fr-209

Radionuclide Production Cross Section



19

Incident Energy (MeV)

87-Fr-209