

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

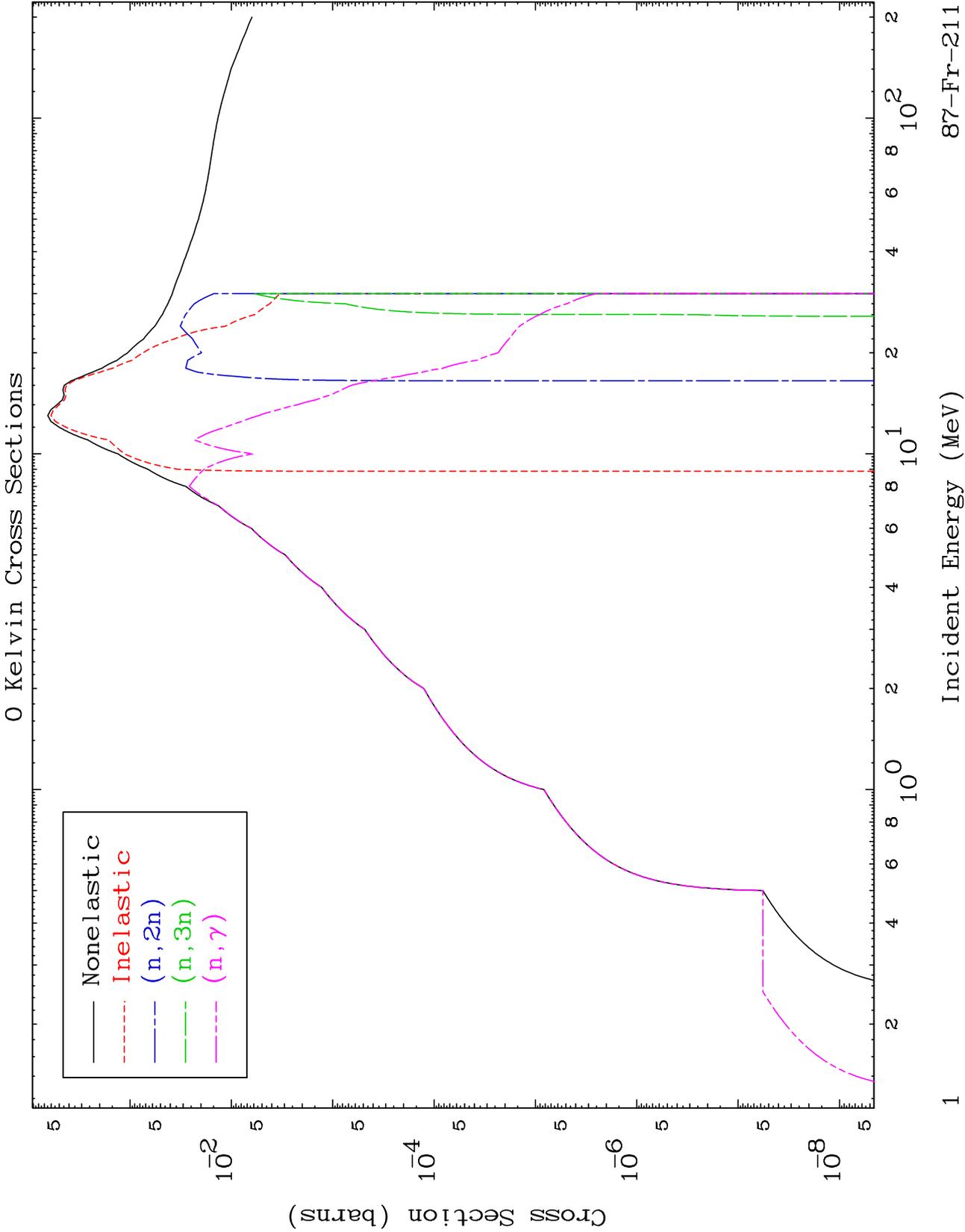
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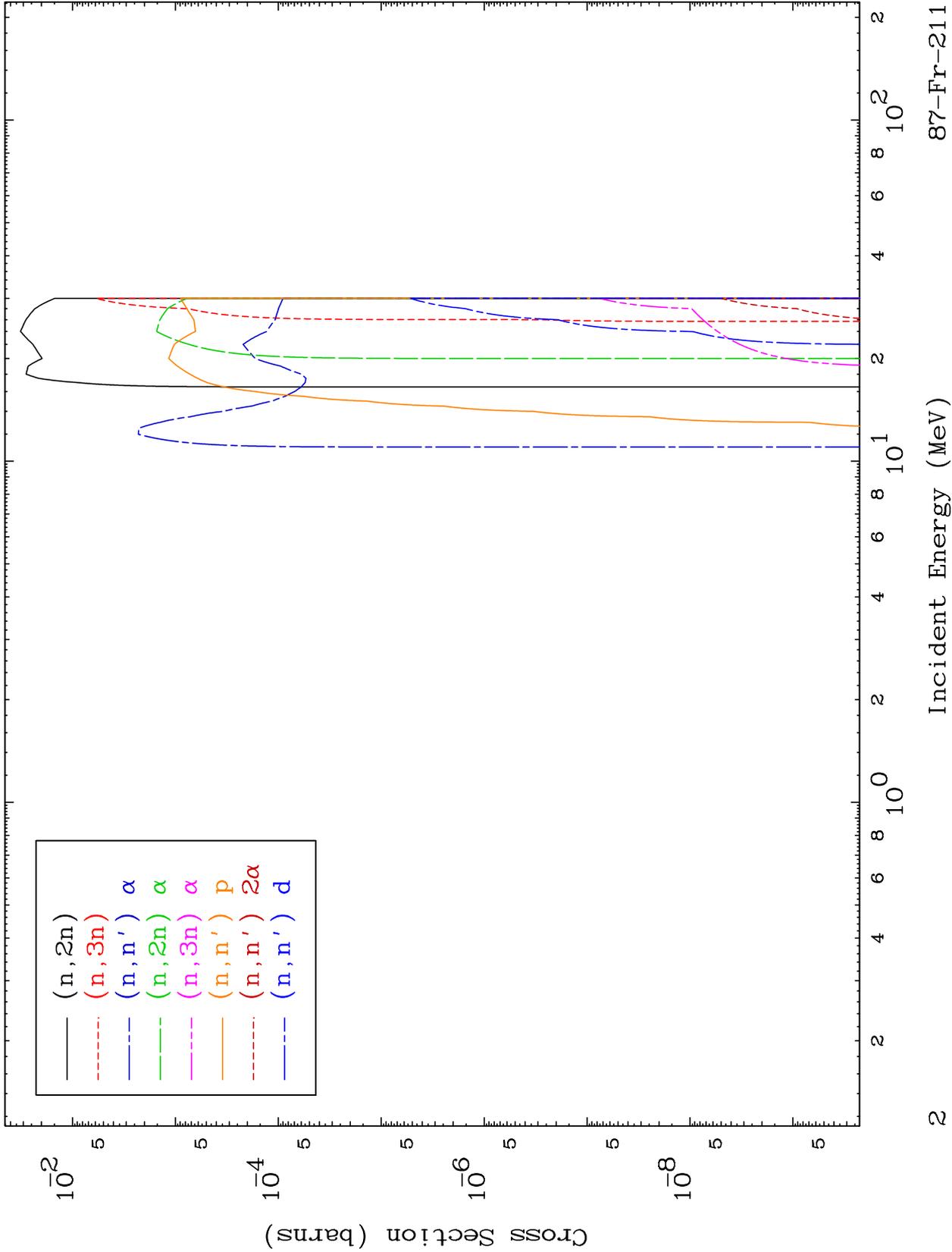
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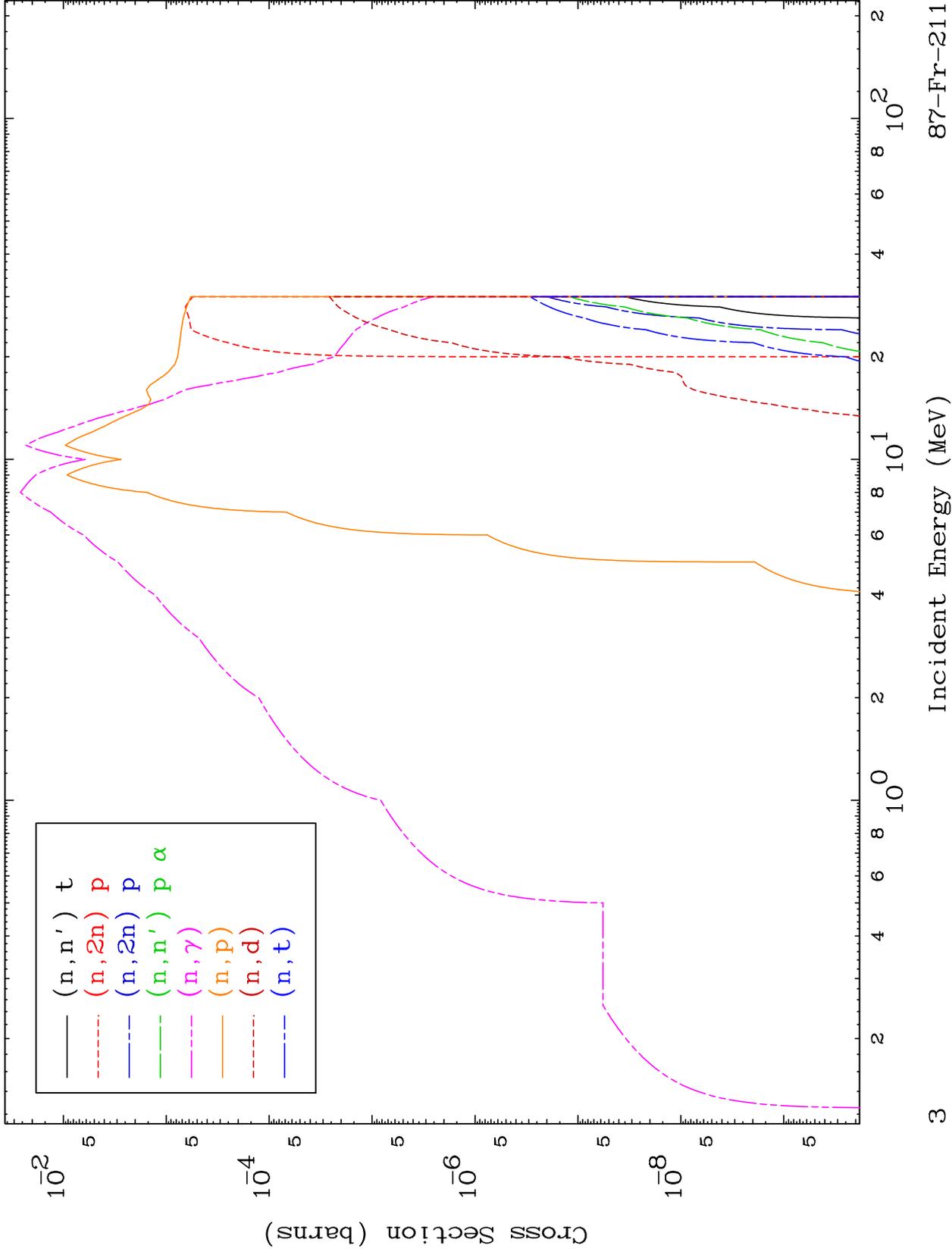


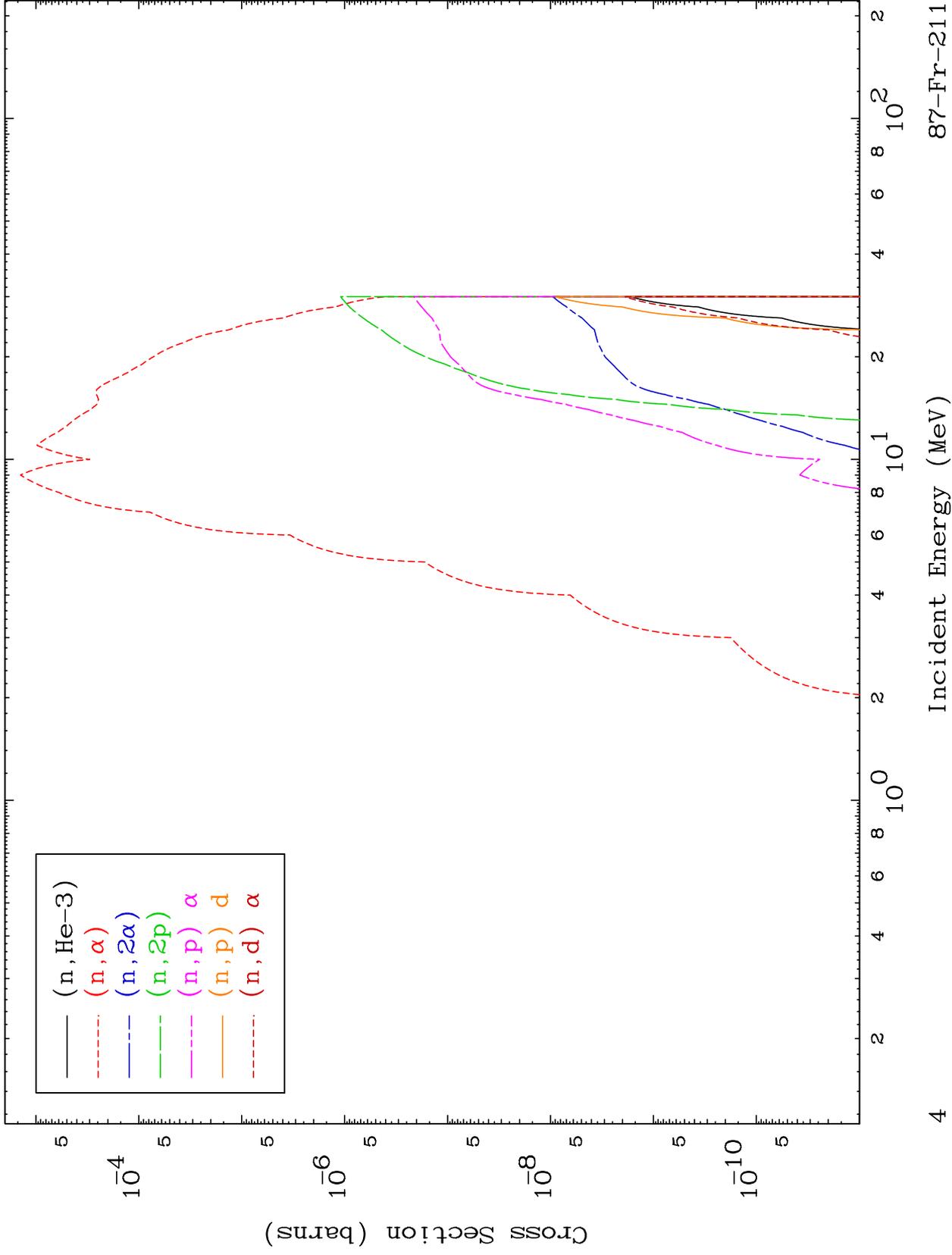
MAT 8722

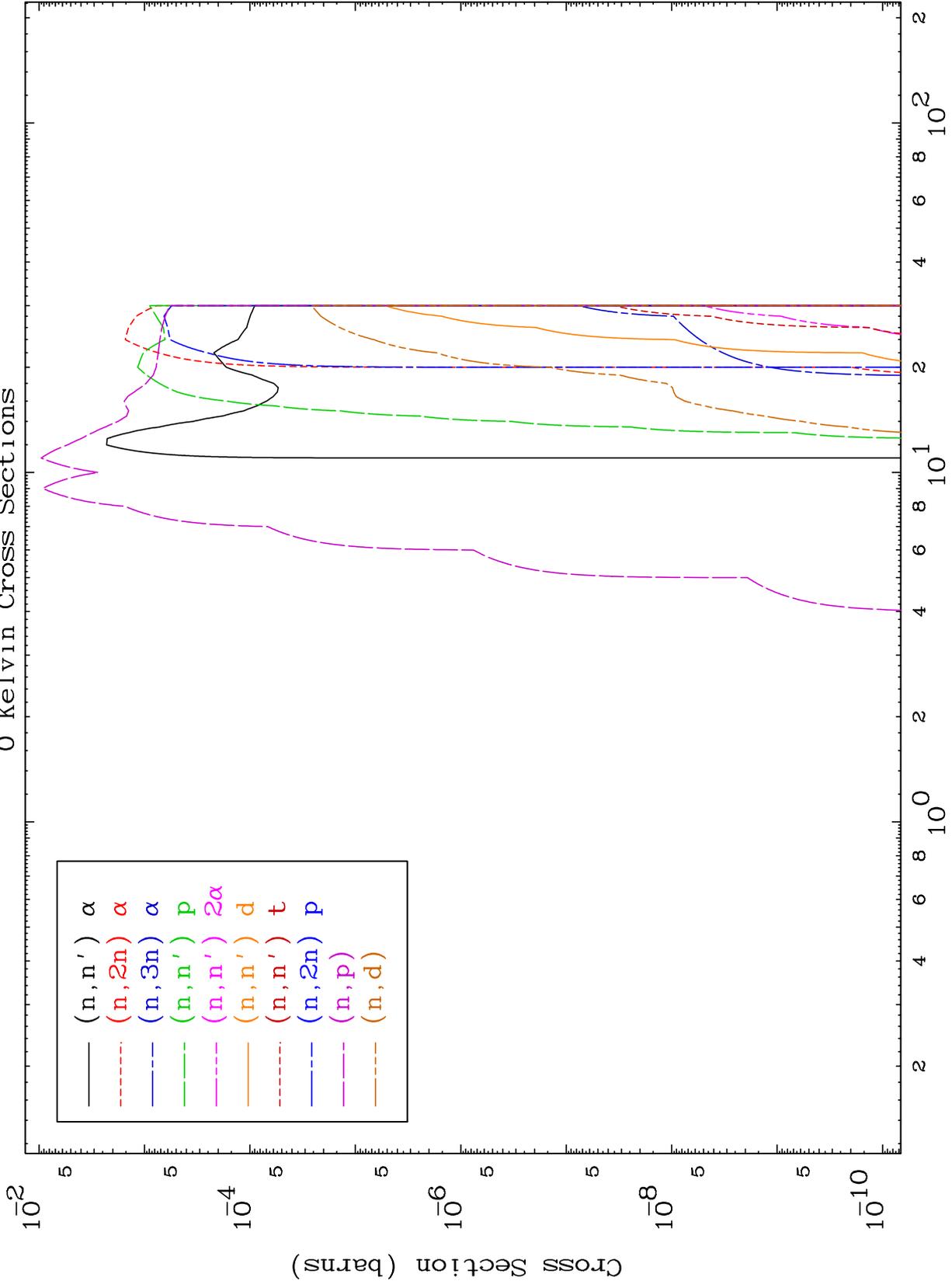
Photon Neutron Absorption
0 Kelvin Cross Sections

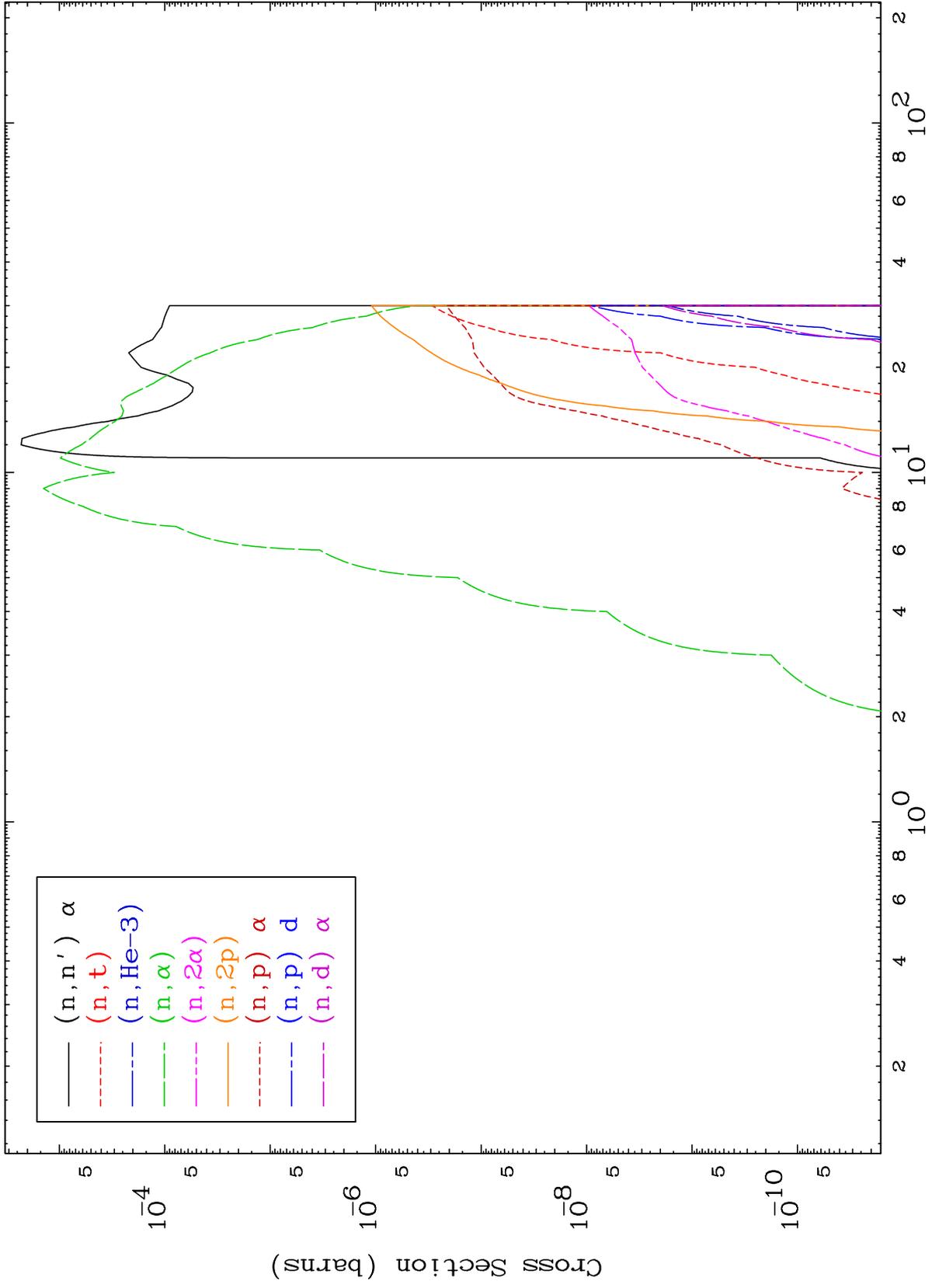
87-Fr-211









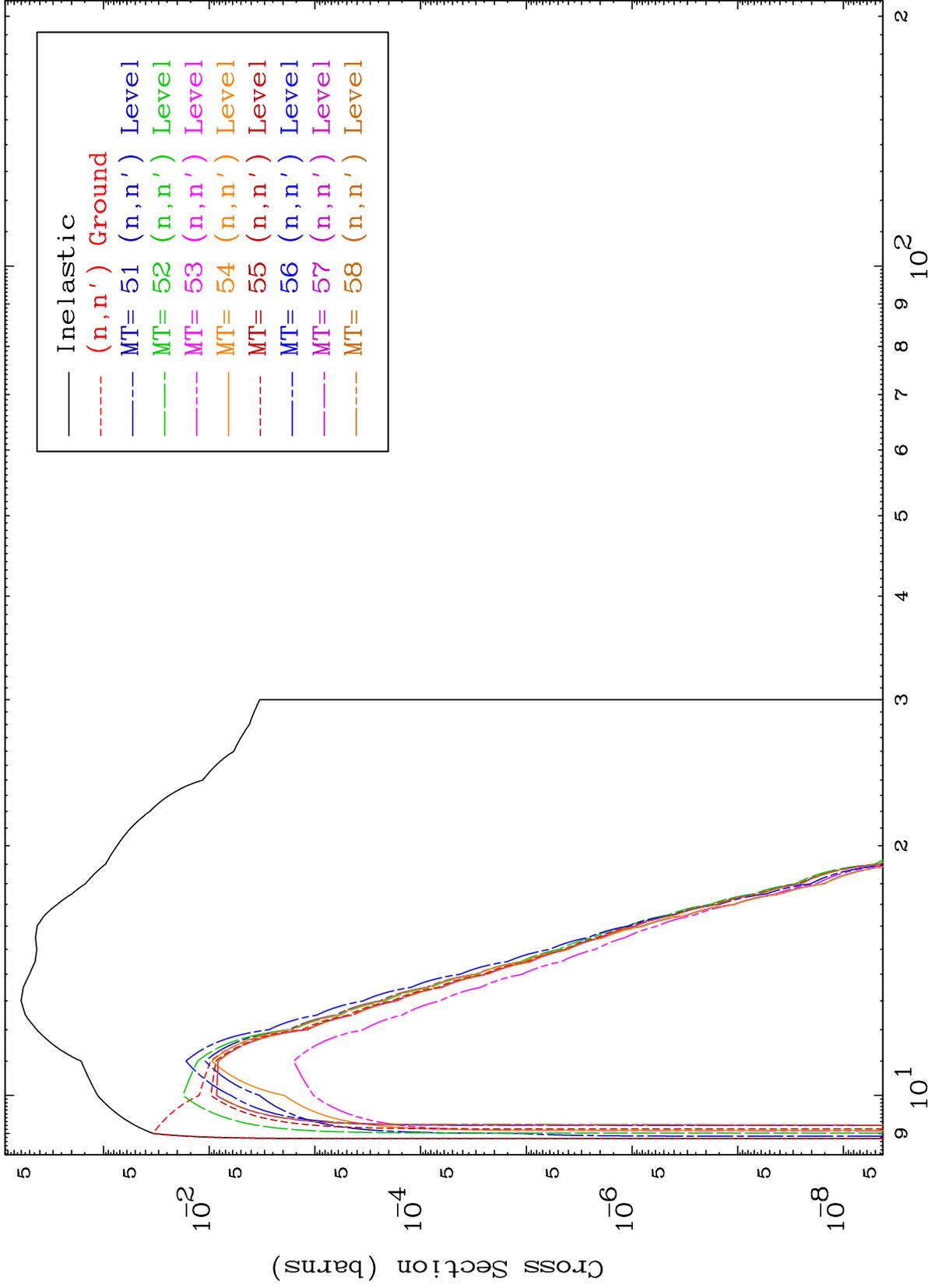


MAT 8722

(γ, n') Levels

87-Fr-211

0 Kelvin Cross Sections



Incident Energy (MeV)

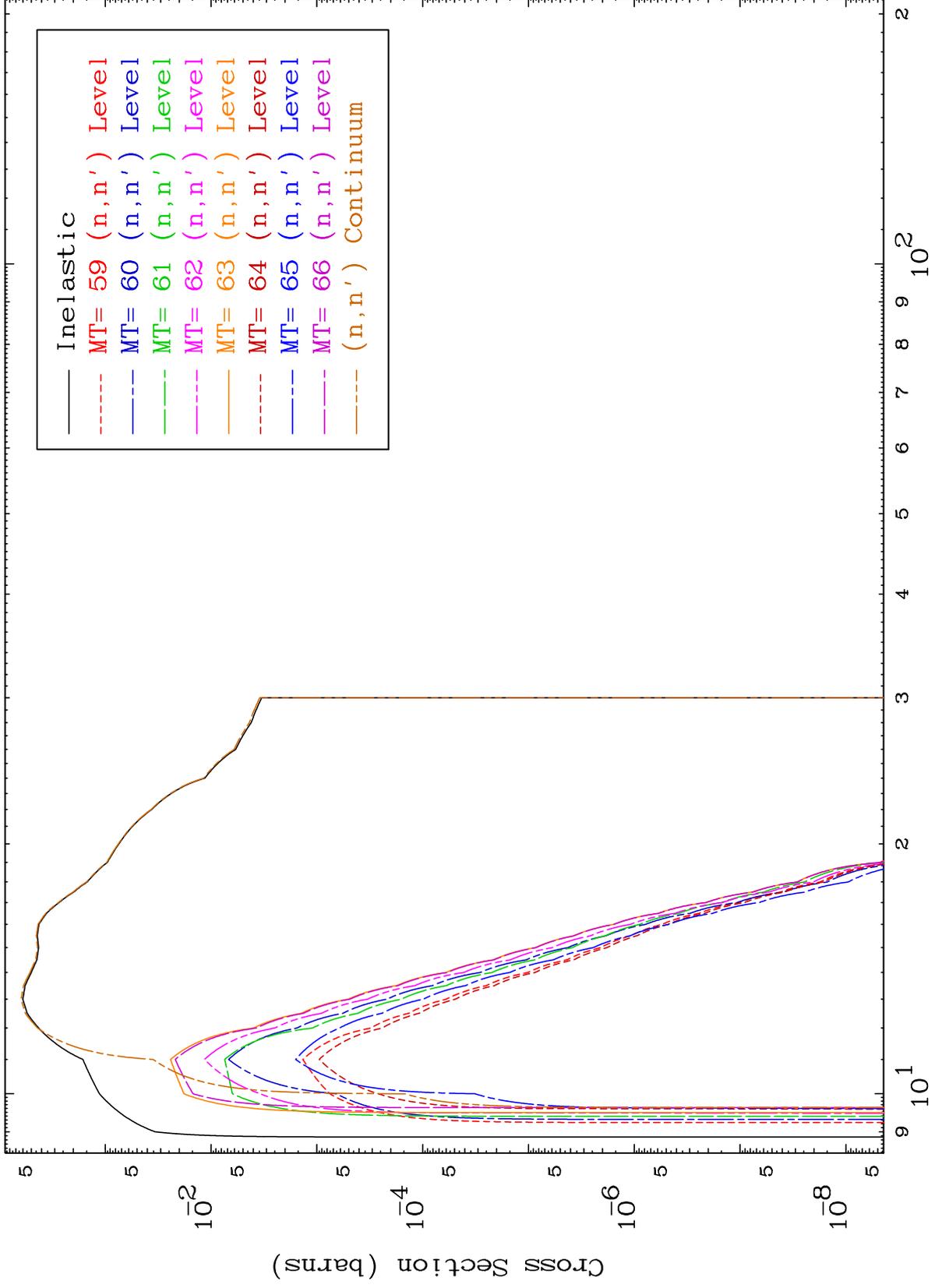
87-Fr-211

MAT 8722

(γ, n') Levels

87-Fr-211

0 Kelvin Cross Sections

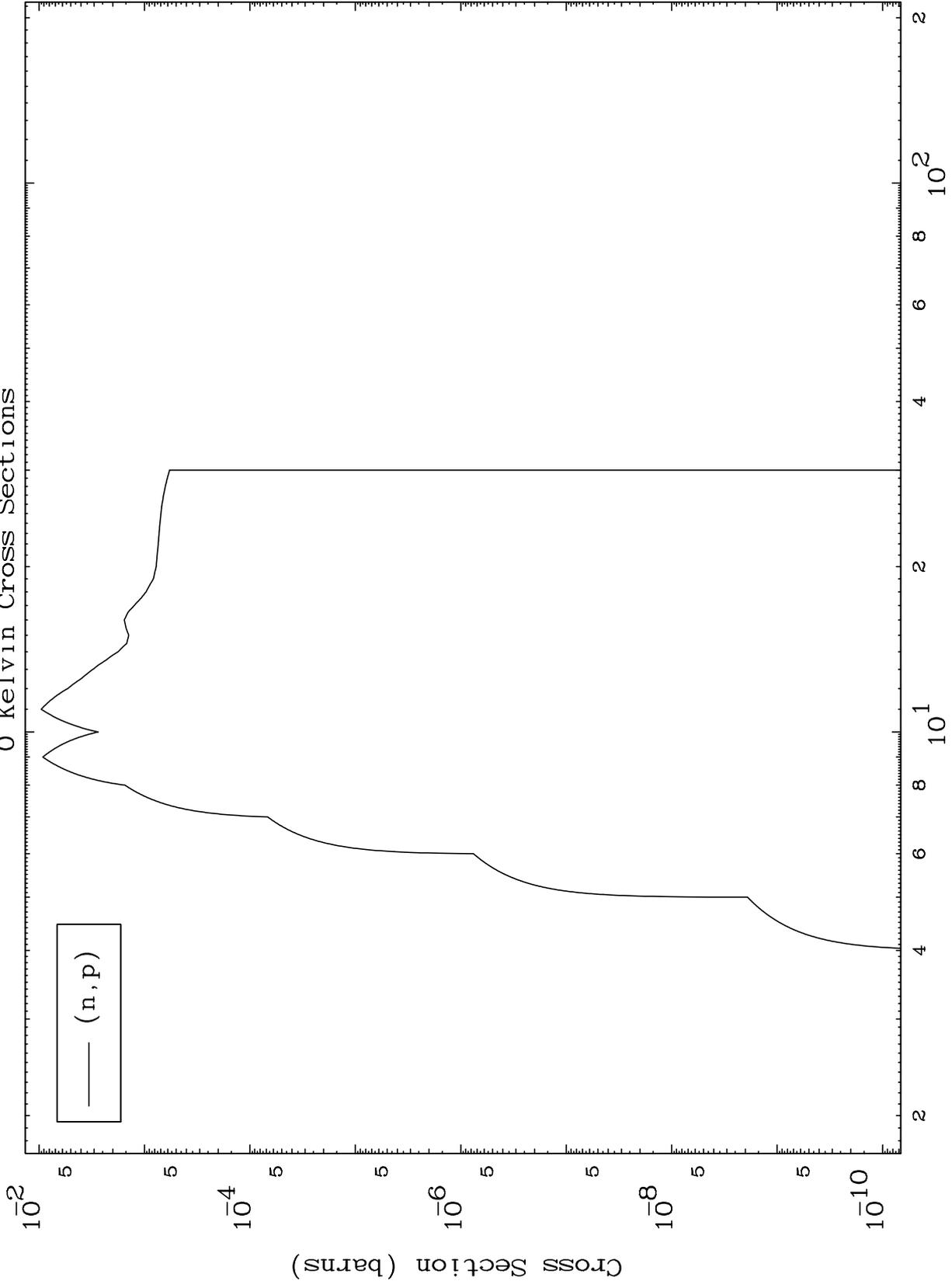


Incident Energy (MeV)

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MAT 8722

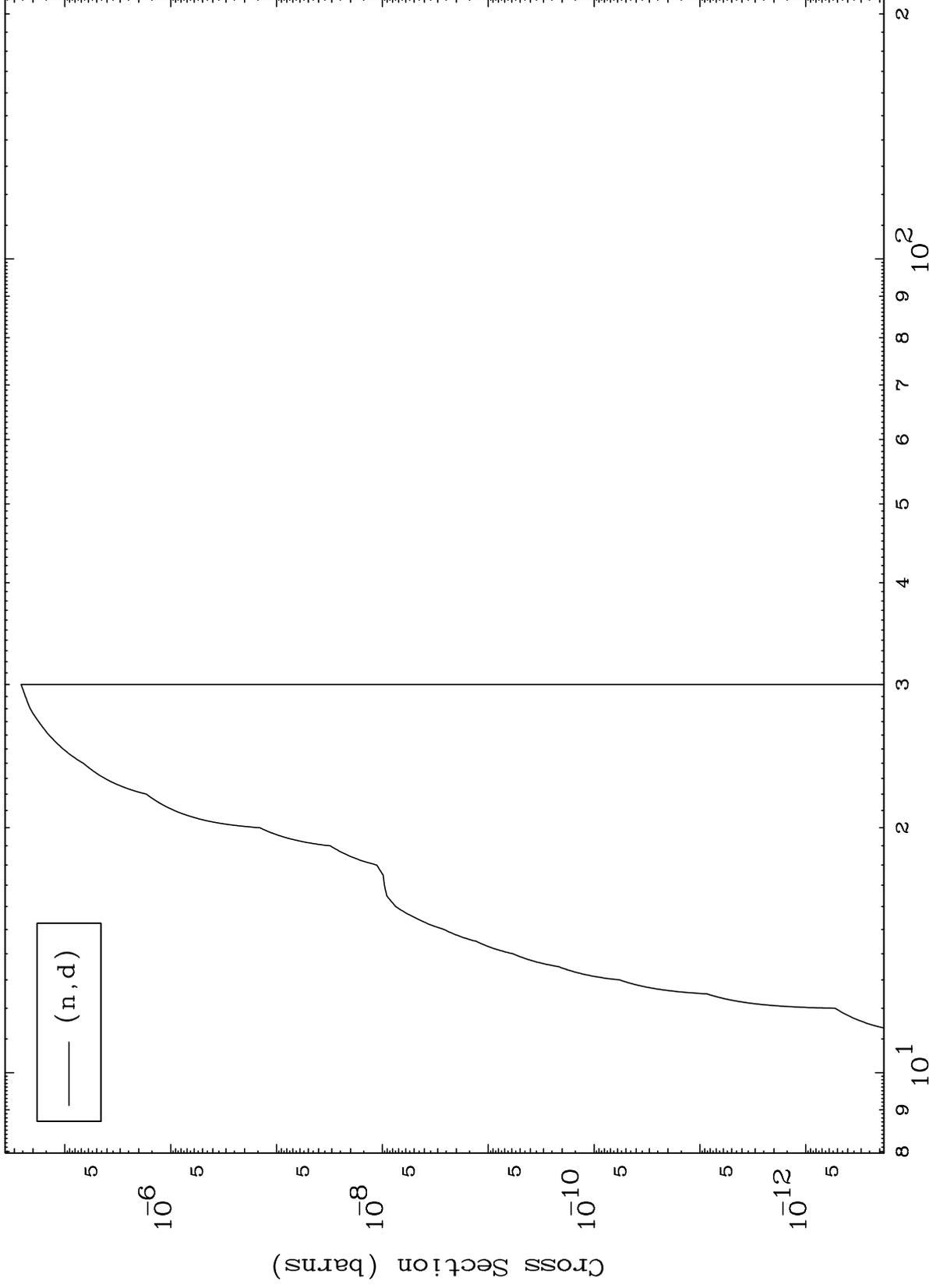
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(γ, d) Levels
0 Kelvin Cross Sections

87-Fr-211



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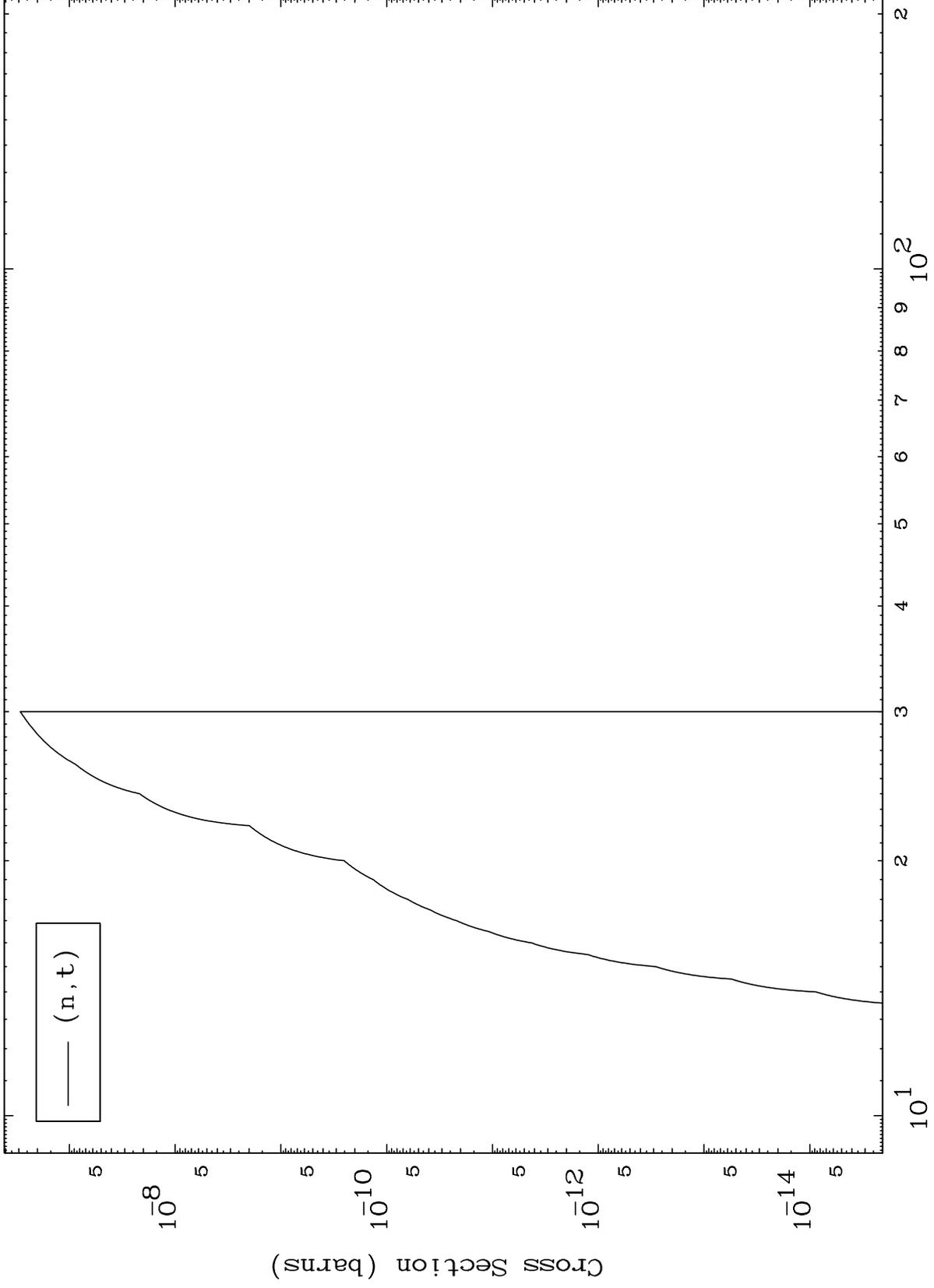
Incident Energy (MeV)

87-Fr-211

MAT 8722

(γ, t) Levels
0 Kelvin Cross Sections

87-Fr-211

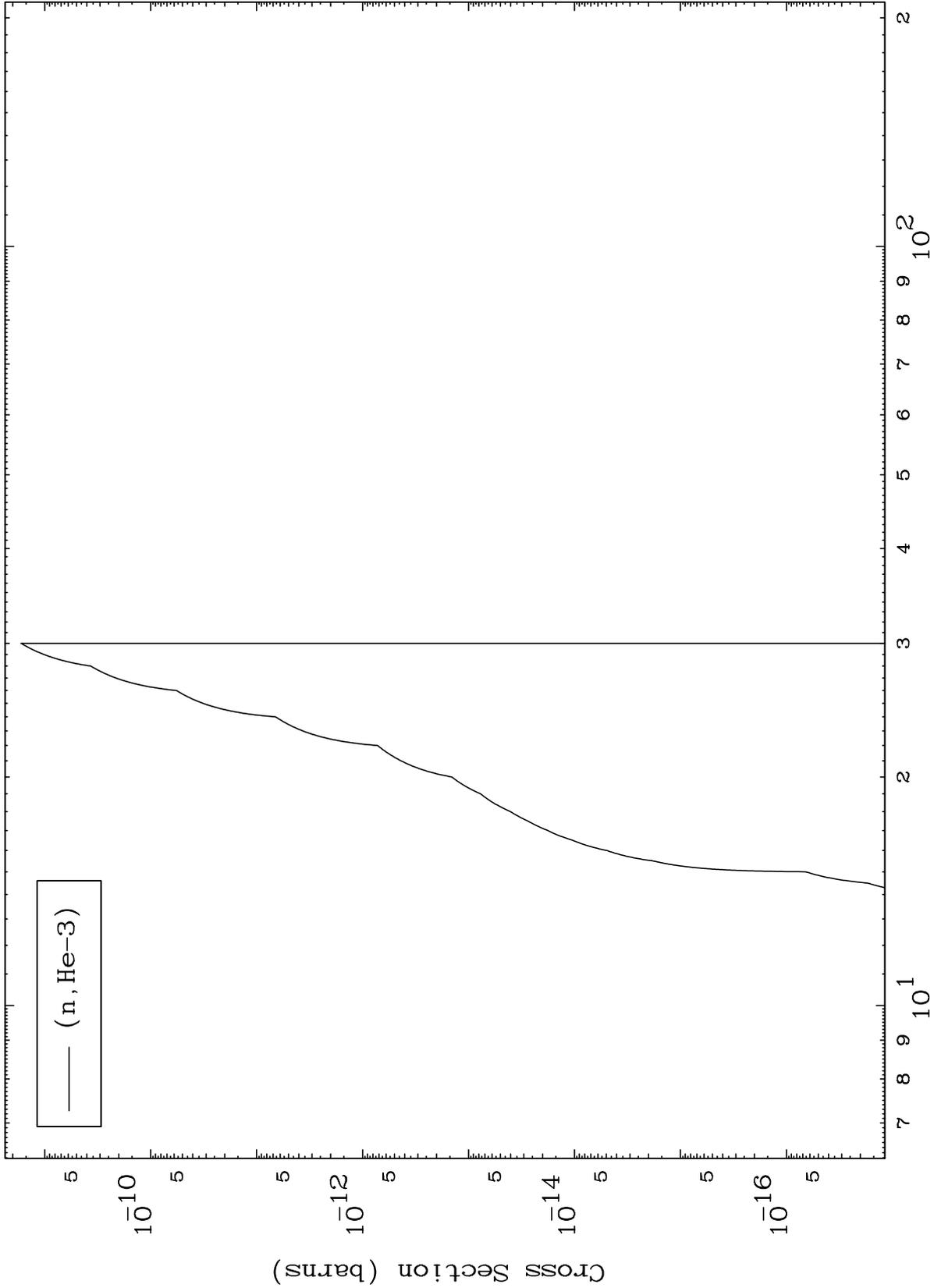


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Incident Energy (MeV)

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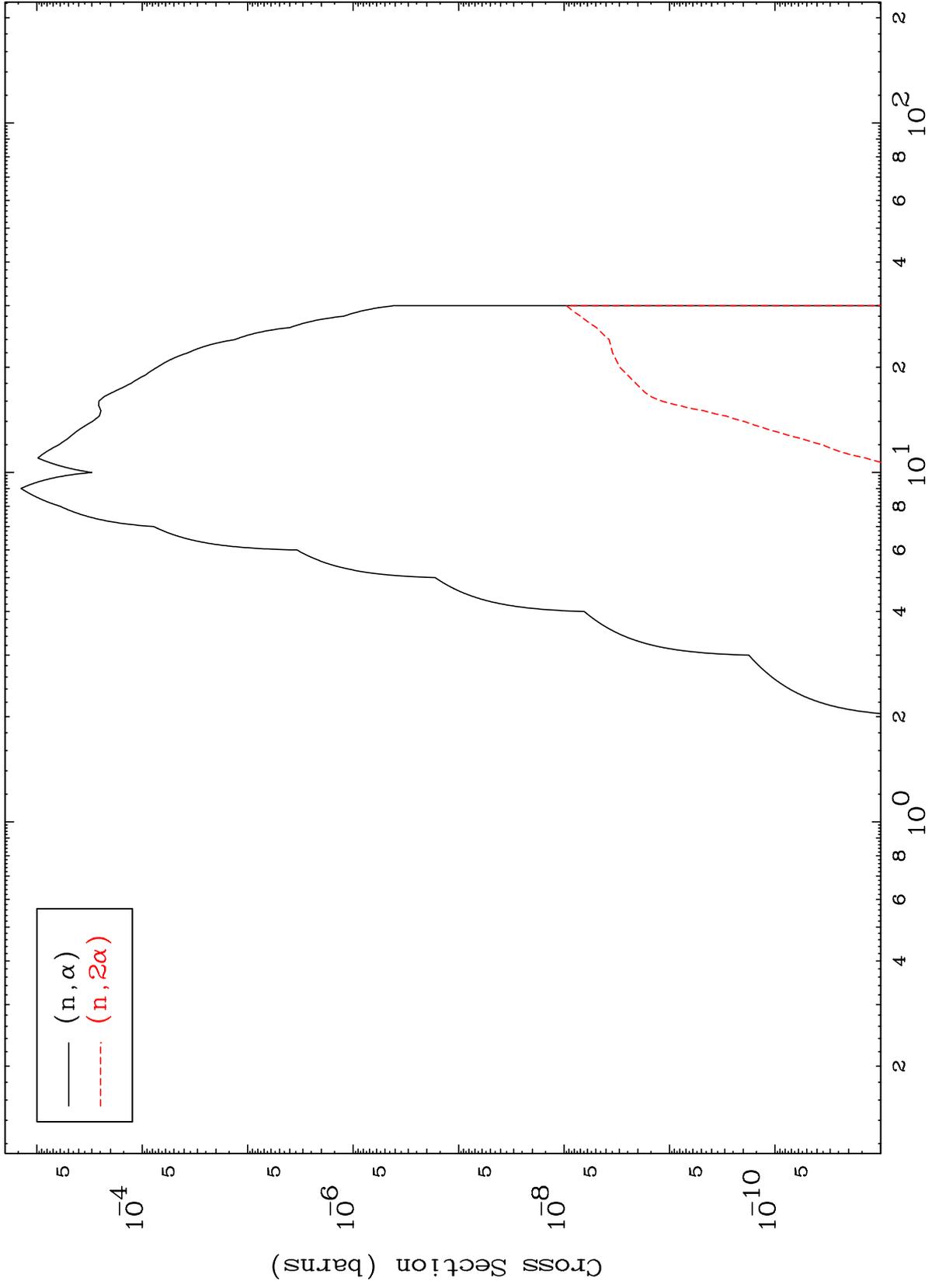
($\gamma, \text{He}3$) Levels
0 Kelvin Cross Sections



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(γ, α) Levels
0 Kelvin Cross Sections

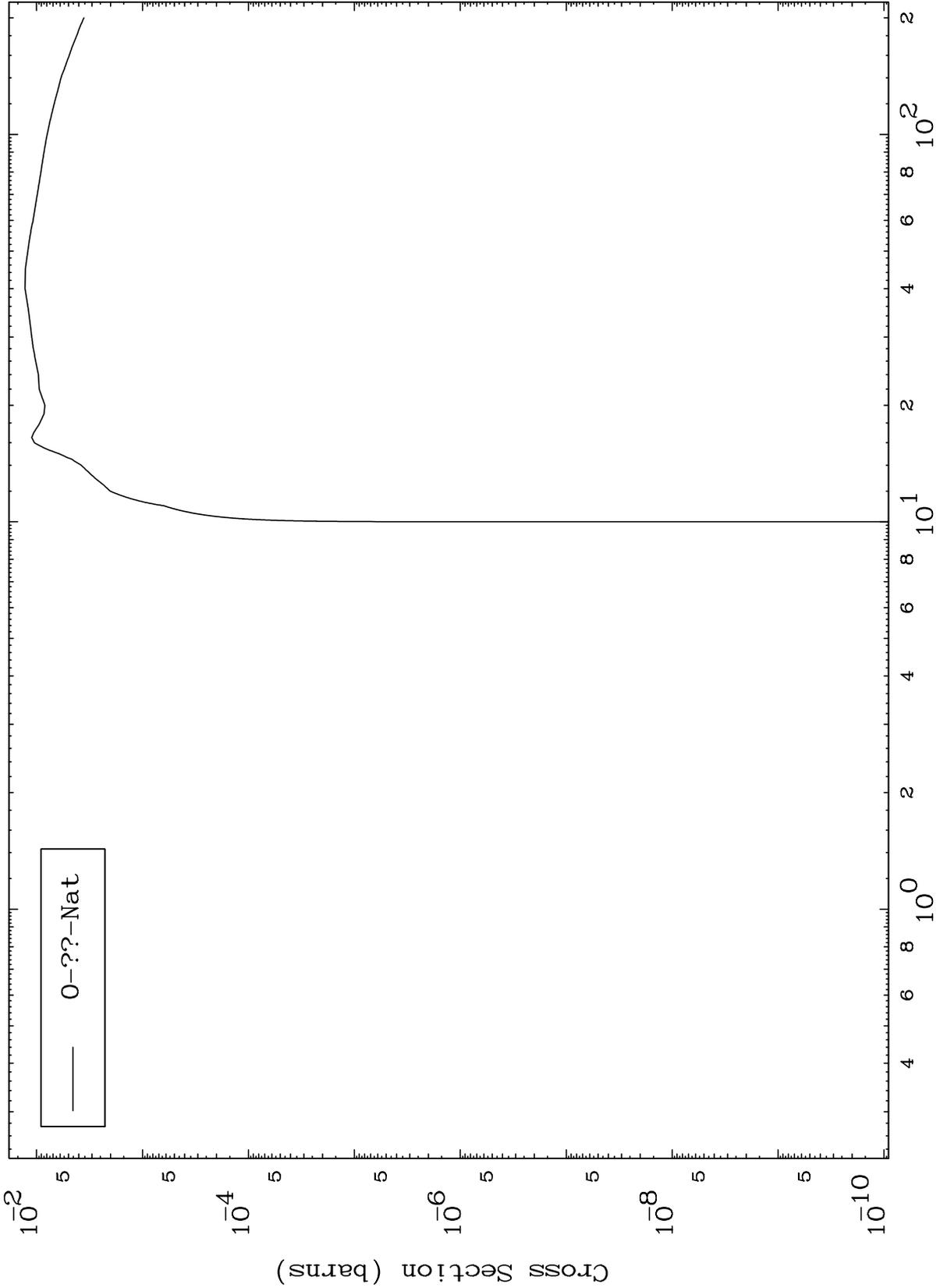
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Fission
Radionuclide Production Cross Section



14

Incident Energy (MeV)

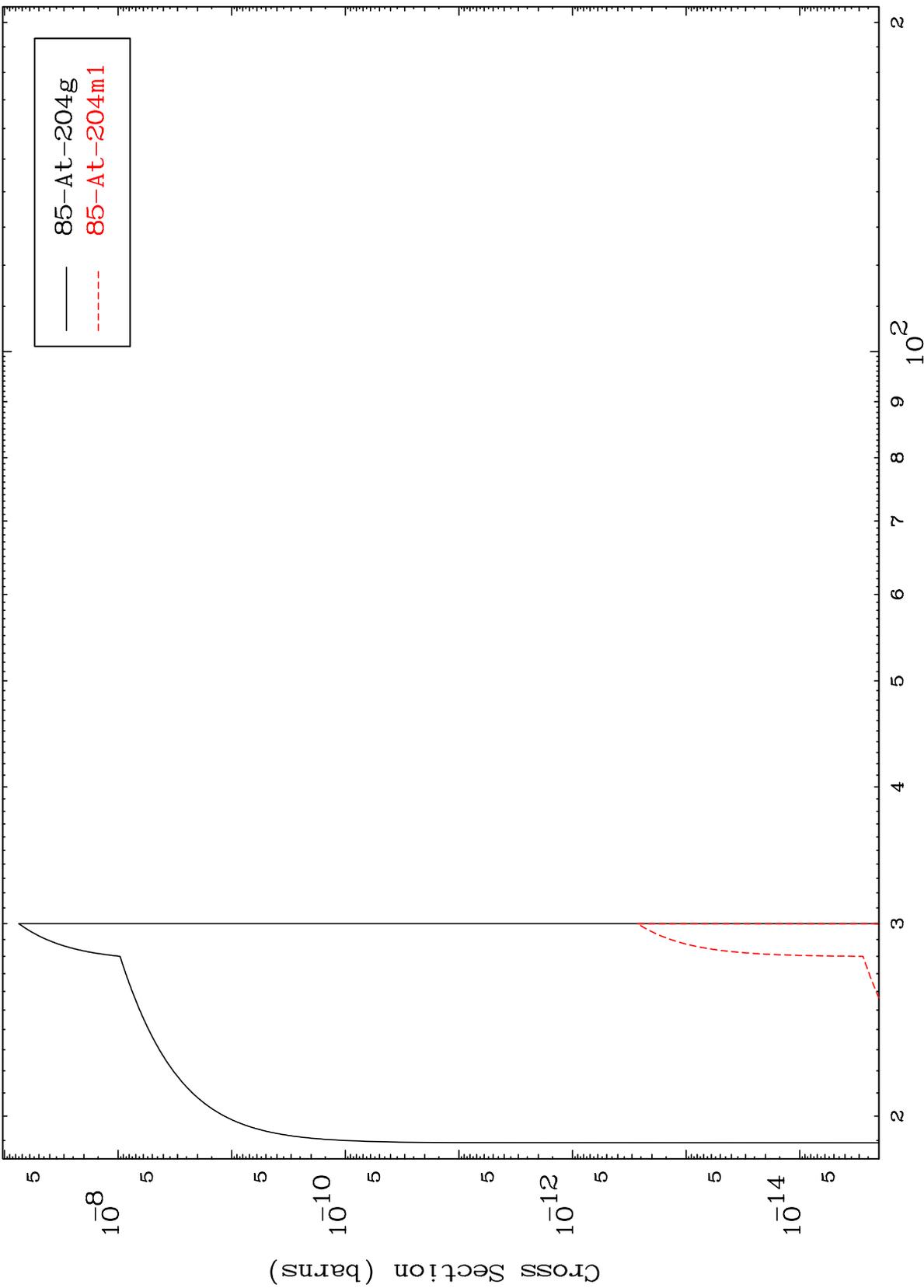
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(n,3n) α

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Radionuclide Production Cross Section



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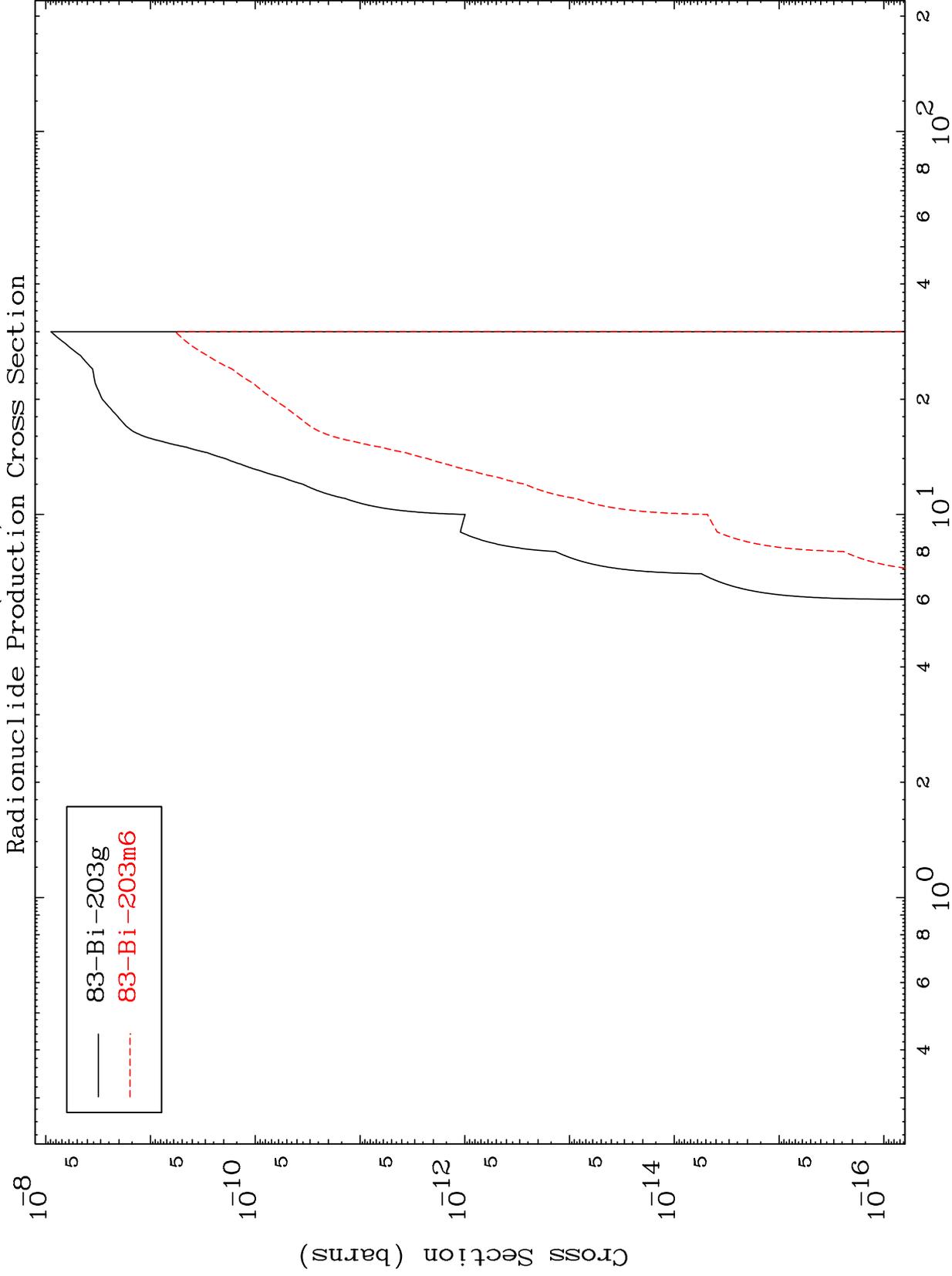
Incident Energy (MeV)

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(n,2α)

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Incident Energy (MeV)

87-Fr-211