

Program EVALPLOT  
(Version 2021-1)

by

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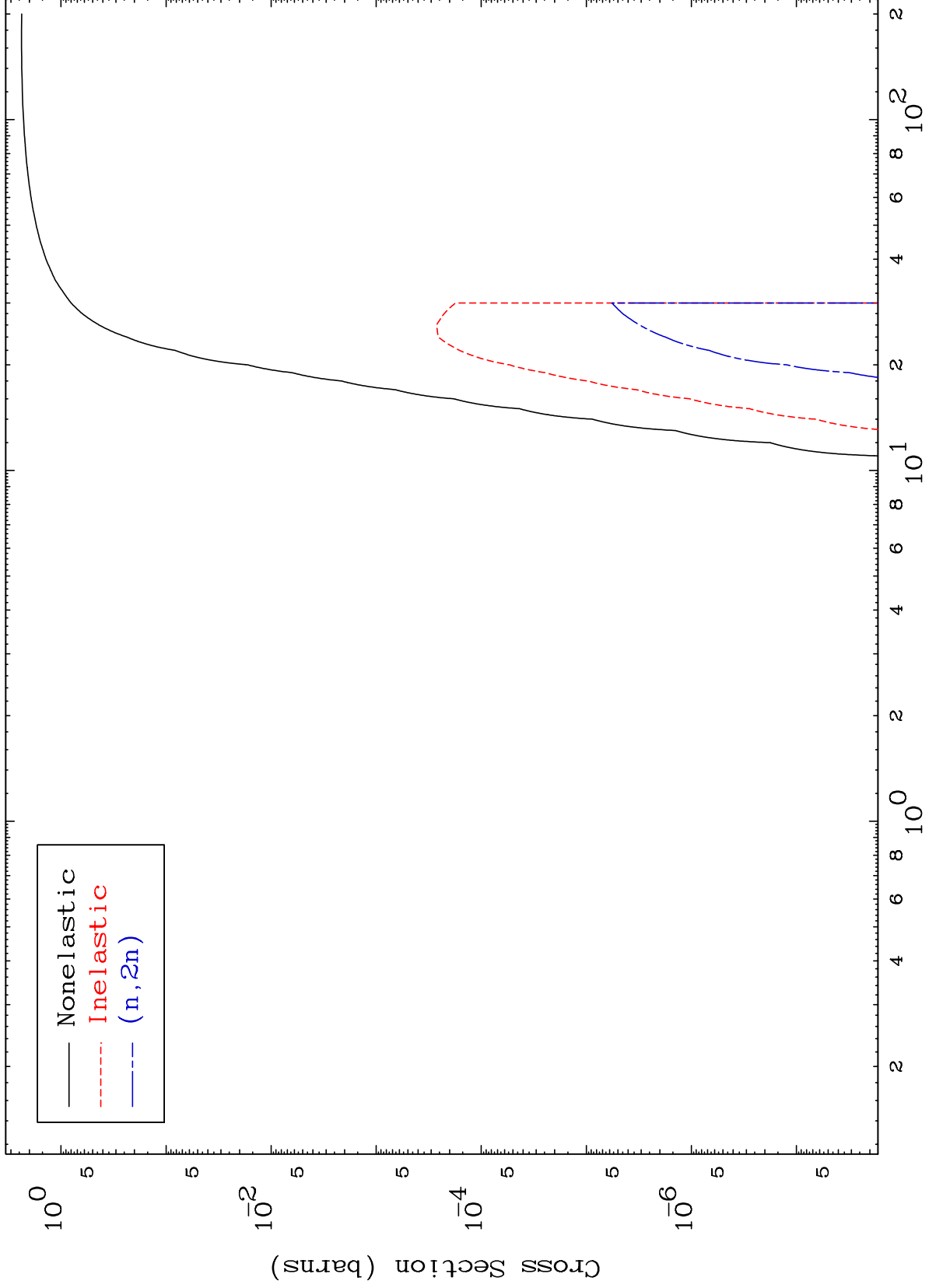
Press Mouse Button to Start

MAT 8511

He-3 Major

85-At-198m

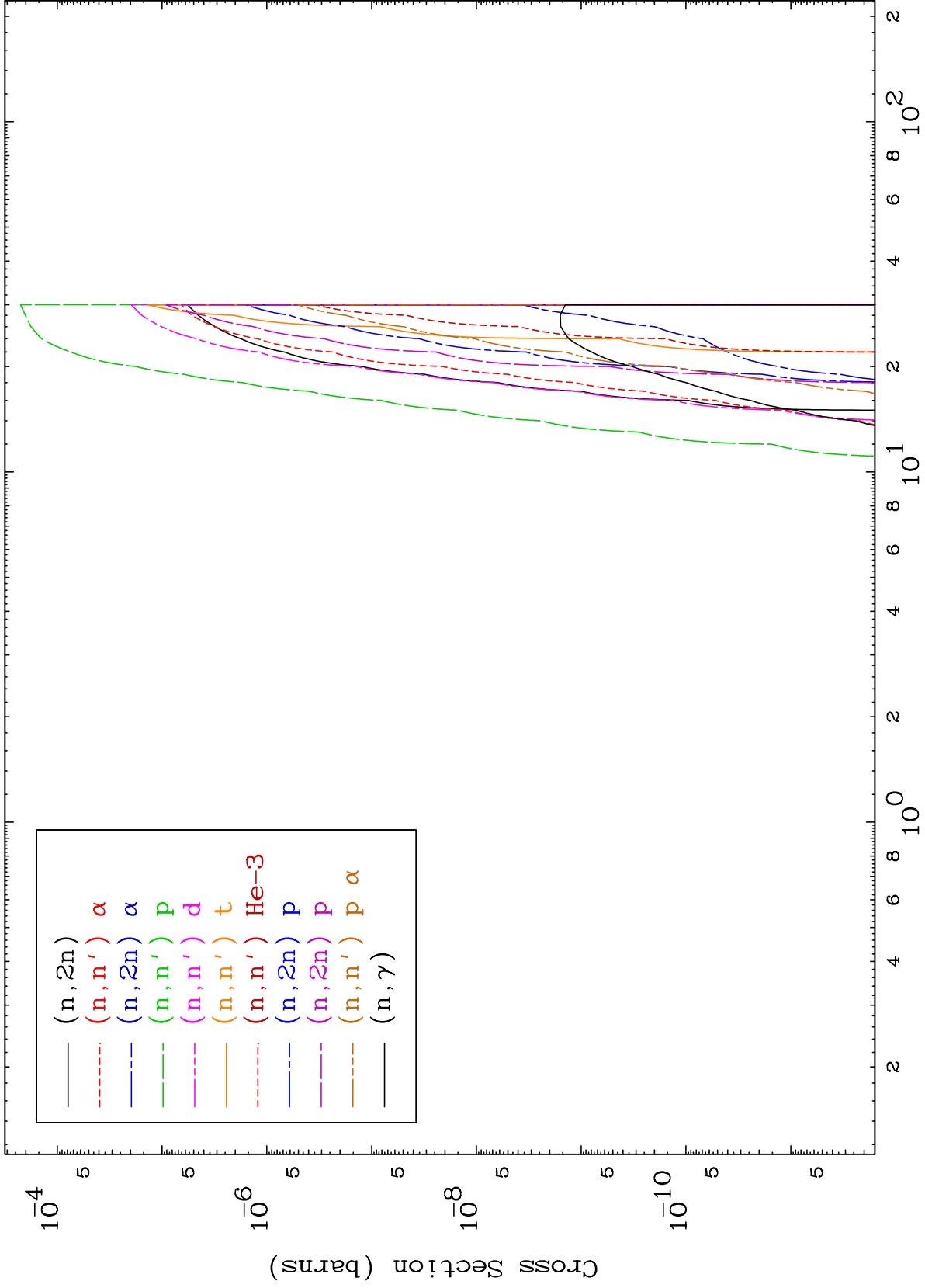
0 Kelvin Cross Sections



MAT 8511

He-3 Neutron Absorption  
0 Kelvin Cross Sections

85-At-198m



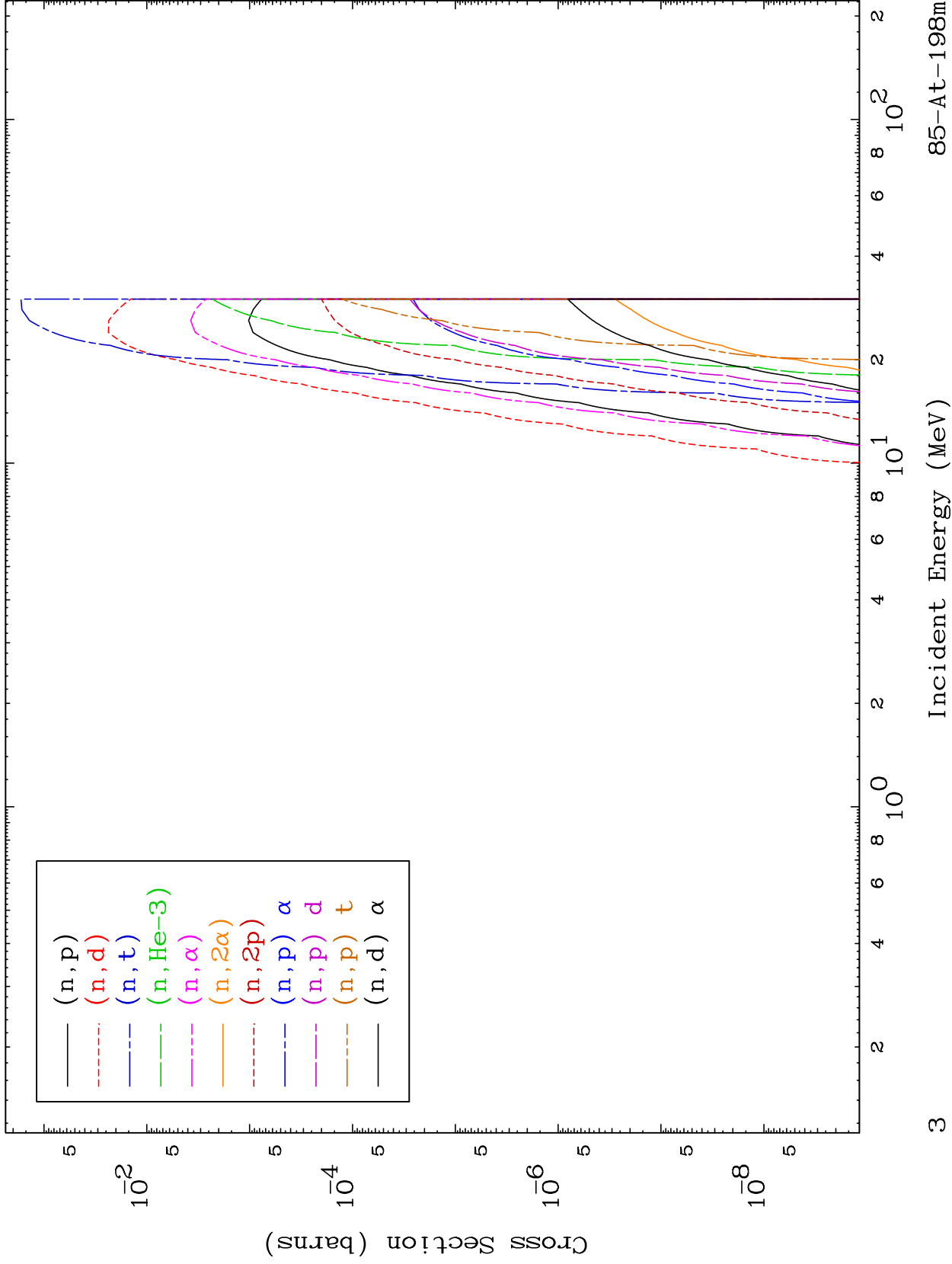
Incident Energy (MeV)

85-At-198m

MAT 8511

He-3 Neutron Absorption  
0 Kelvin Cross Sections

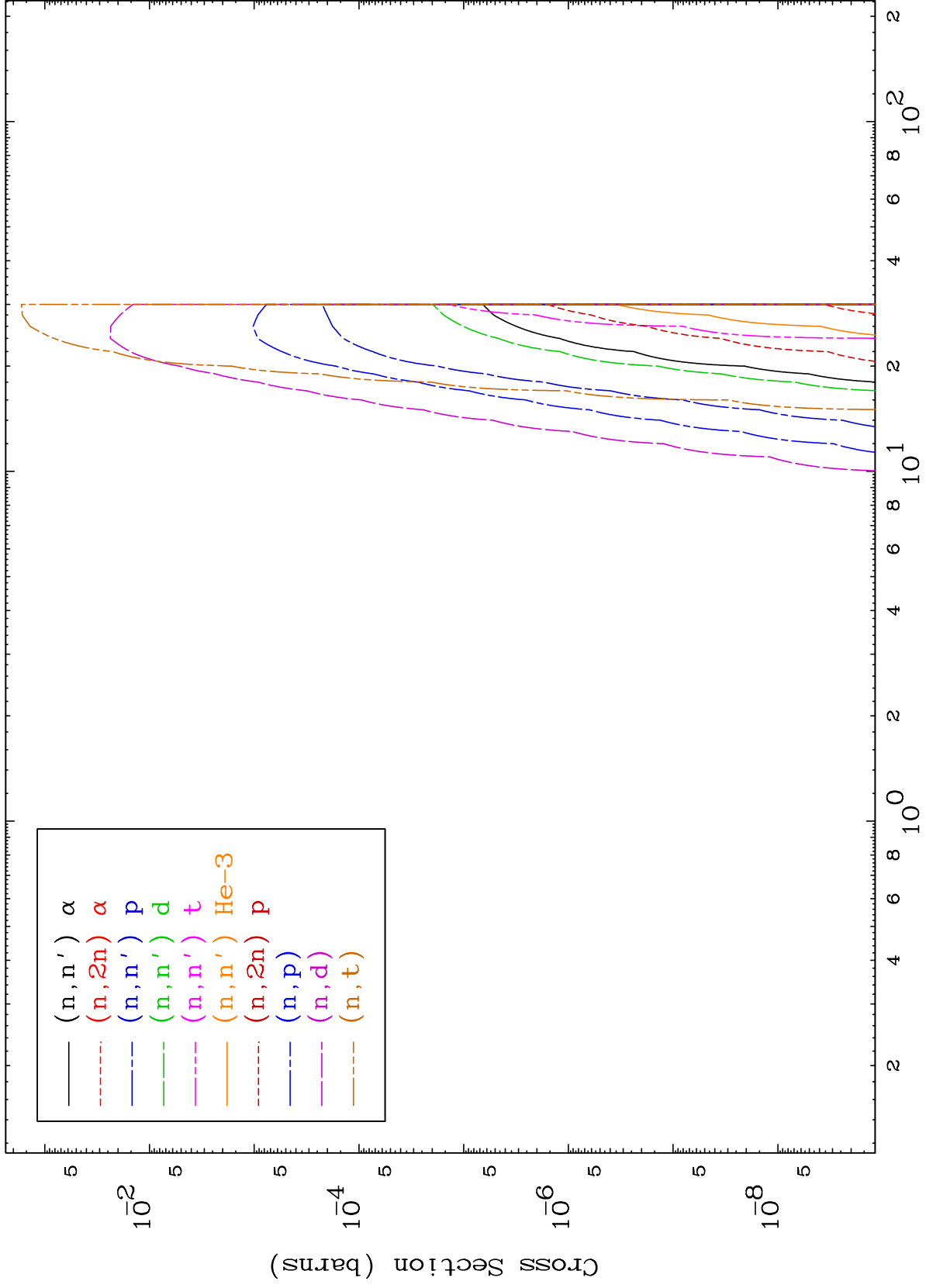
85-At-198m



MAT 8511

He-3 Charged Particle  
0 Kelvin Cross Sections

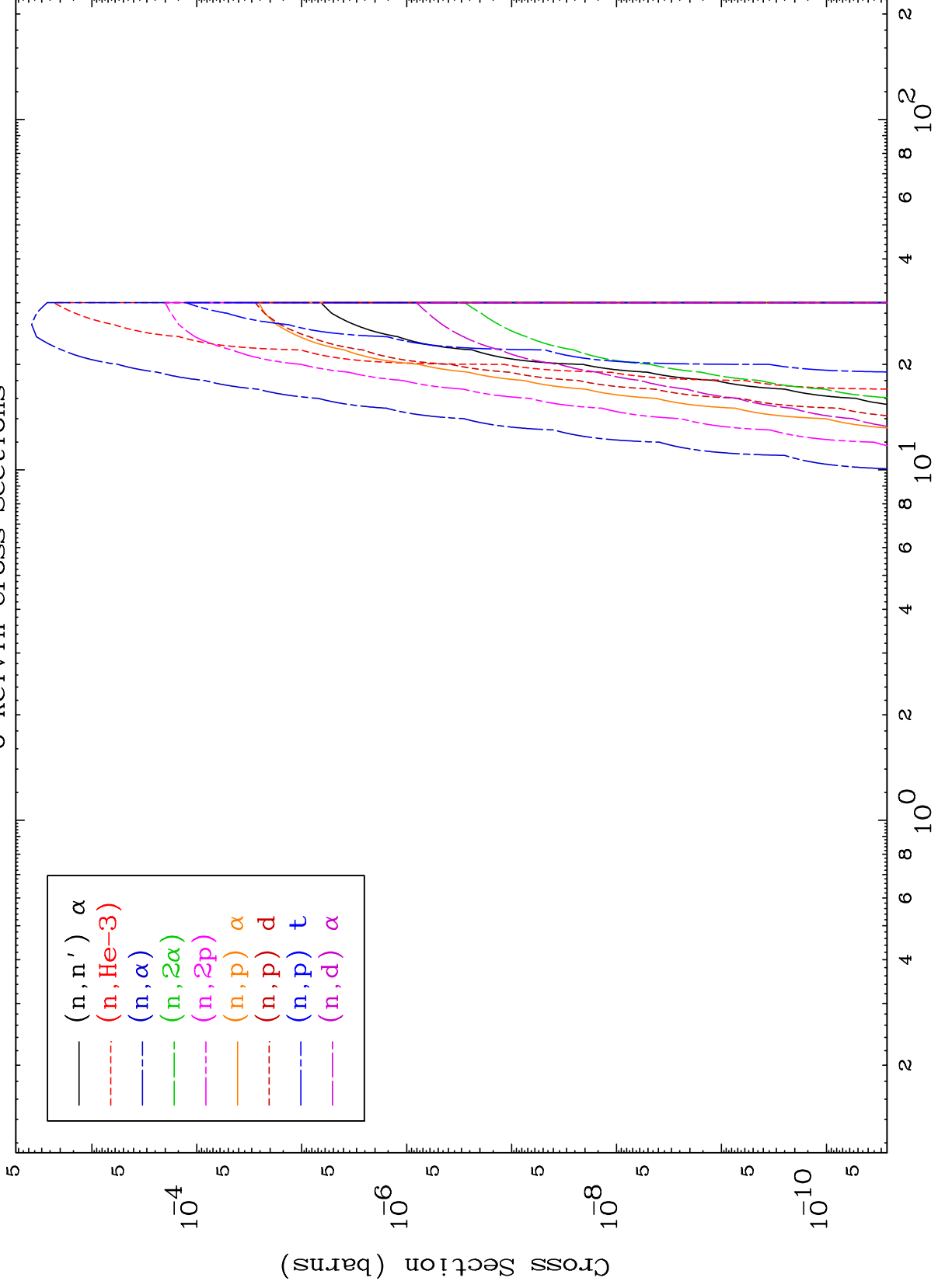
85-At-198m



MAT 8511

He-3 Charged Particle  
0 Kelvin Cross Sections

85-At-198m

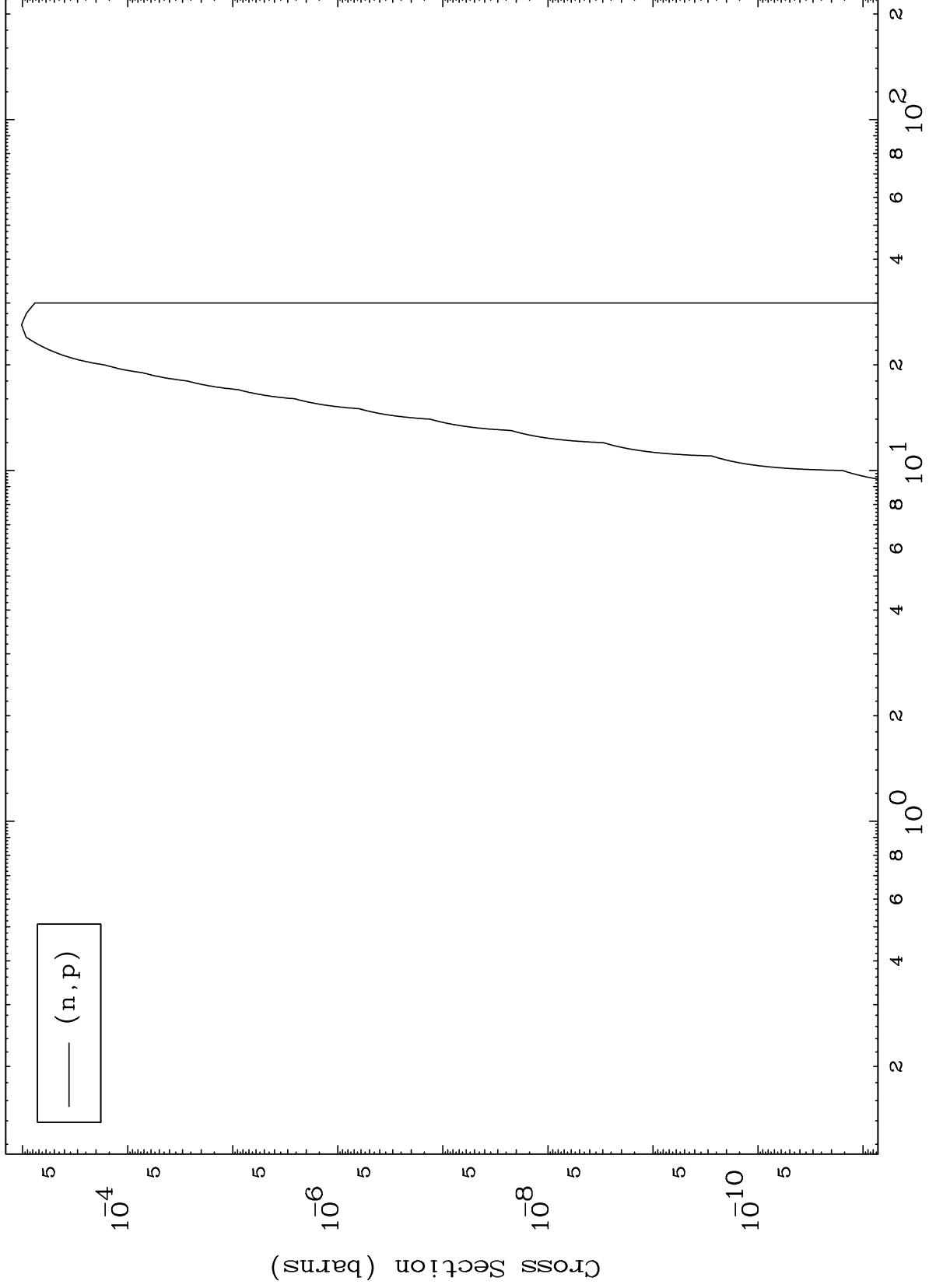


MAT 85111

(He-3,p) Levels

85-At-198m

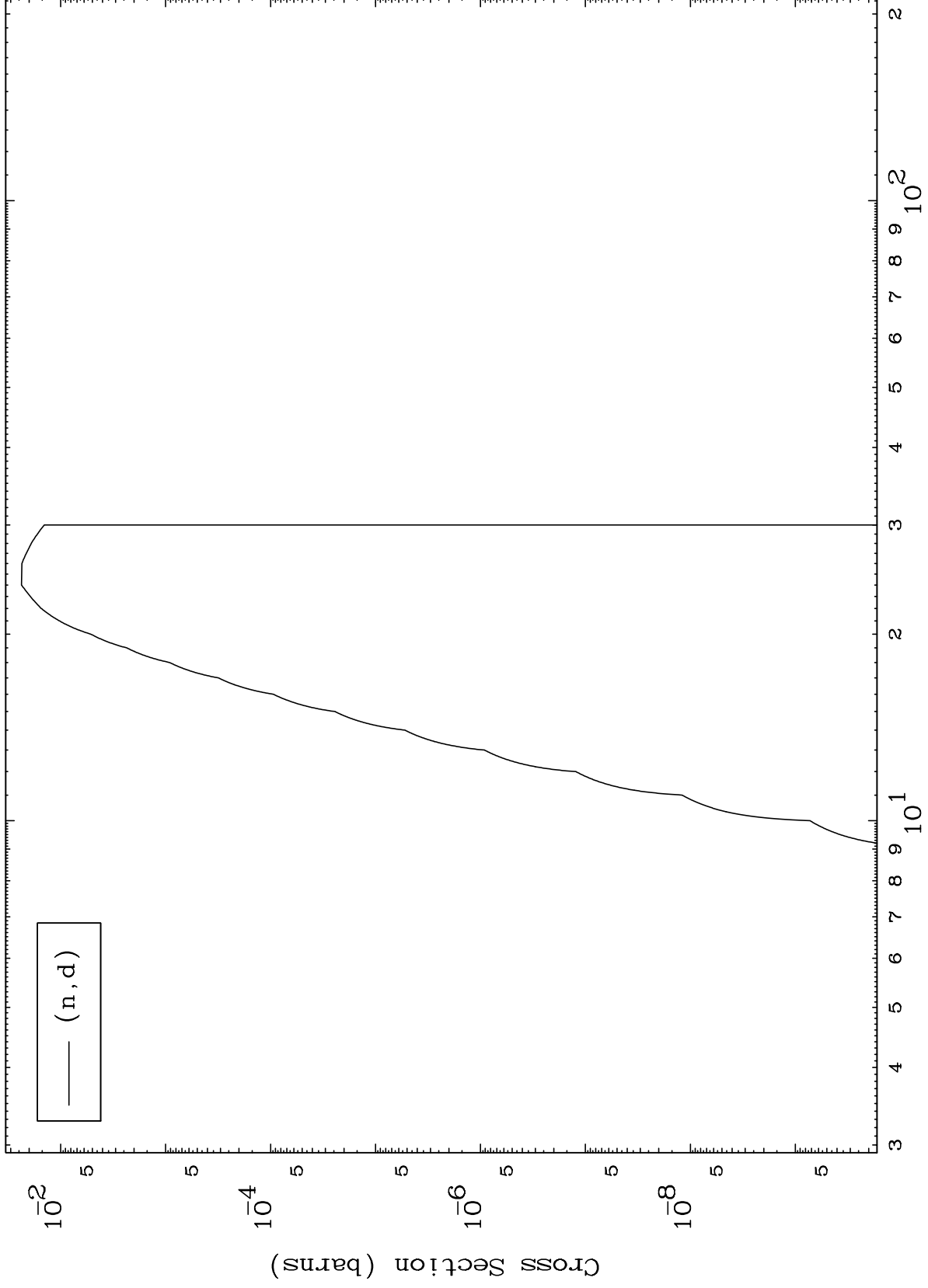
0 Kelvin Cross Sections



MAT 85111

(He-3,d) Levels  
0 Kelvin Cross Sections

85-At-198m



7

Incident Energy (MeV)

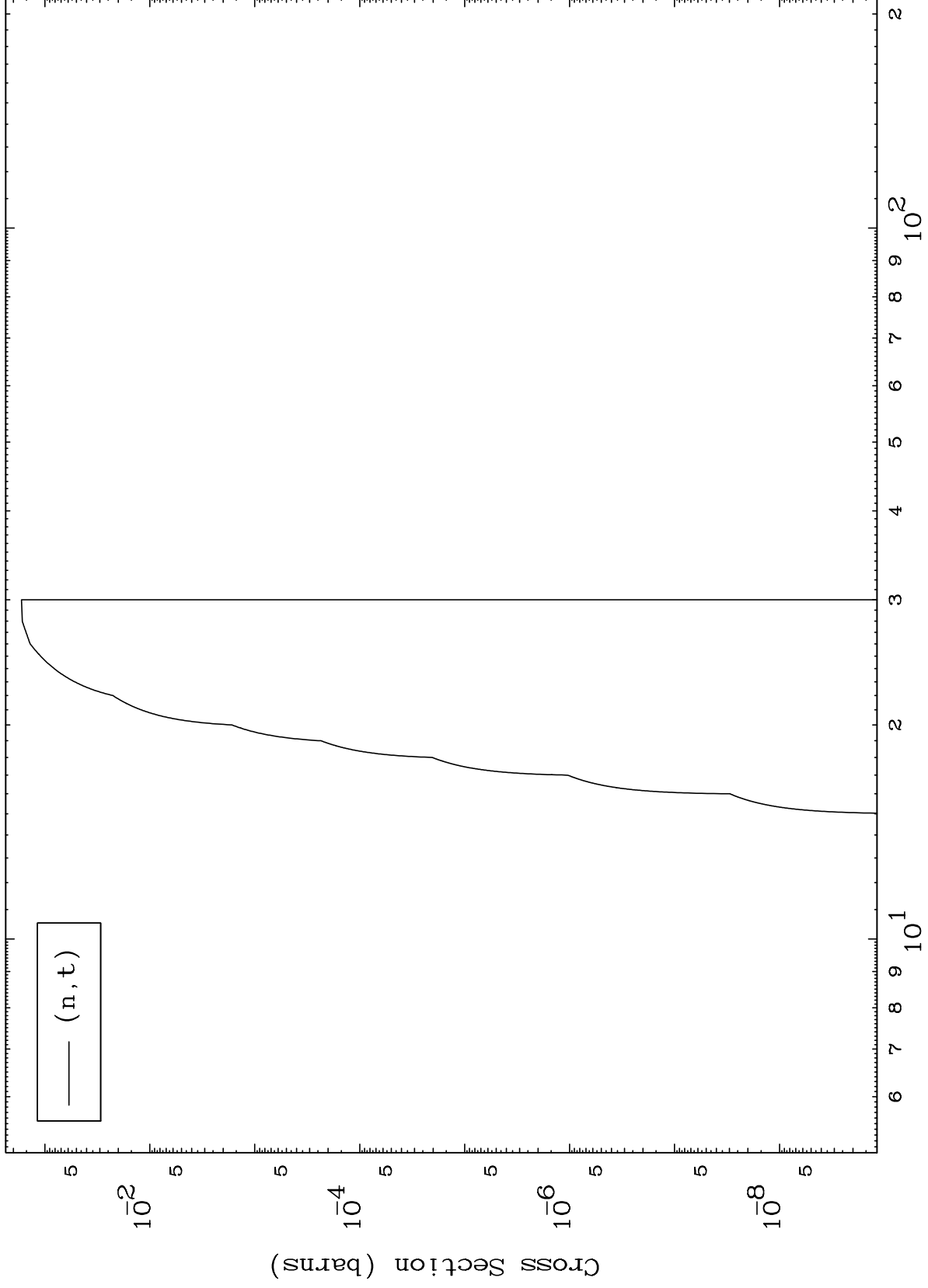
85-At-198m

MAT 85111

(He-3,t) Levels

85-At-198m

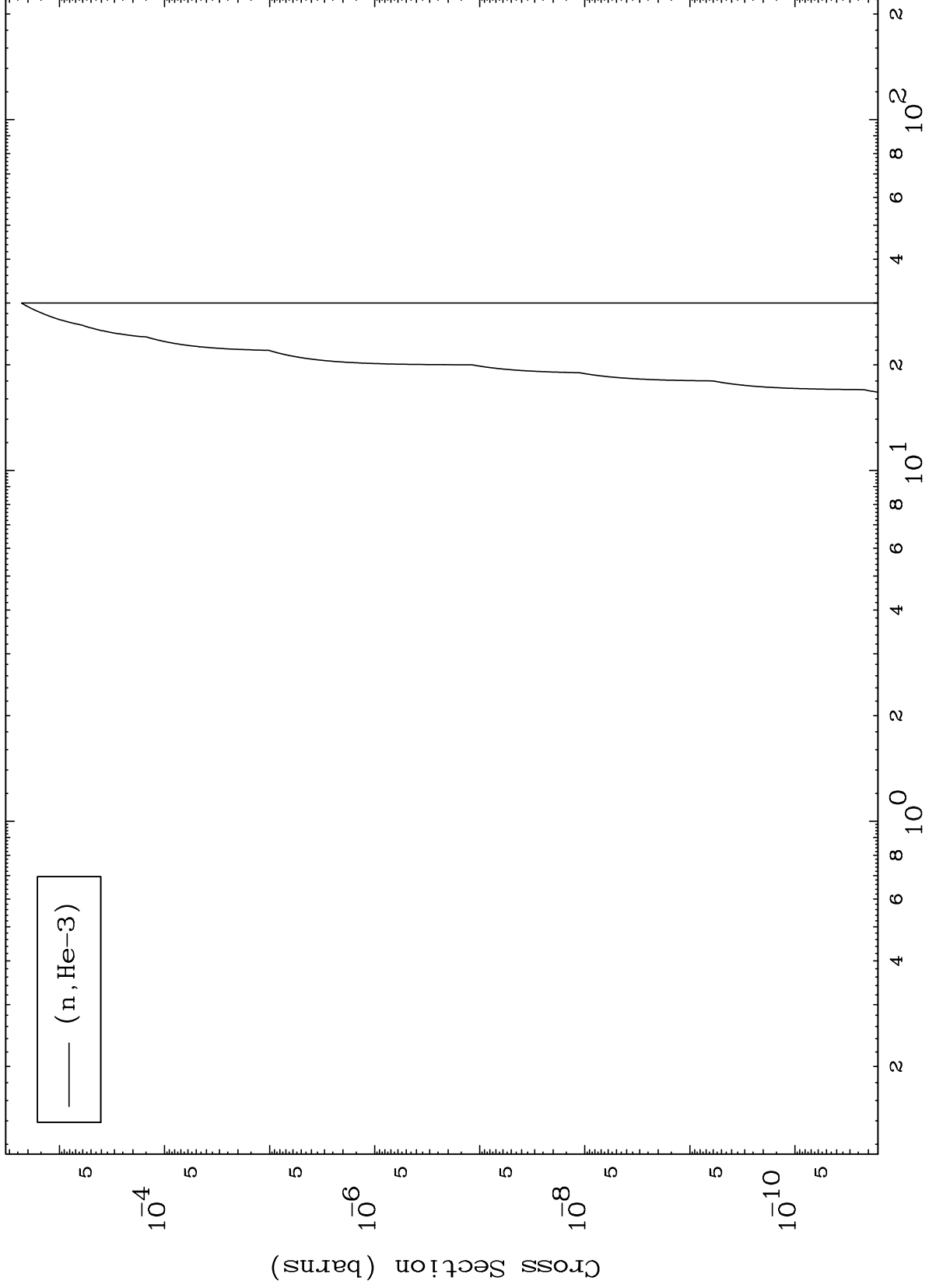
0 Kelvin Cross Sections



MAT 85111

(He-3, He3) Levels  
0 Kelvin Cross Sections

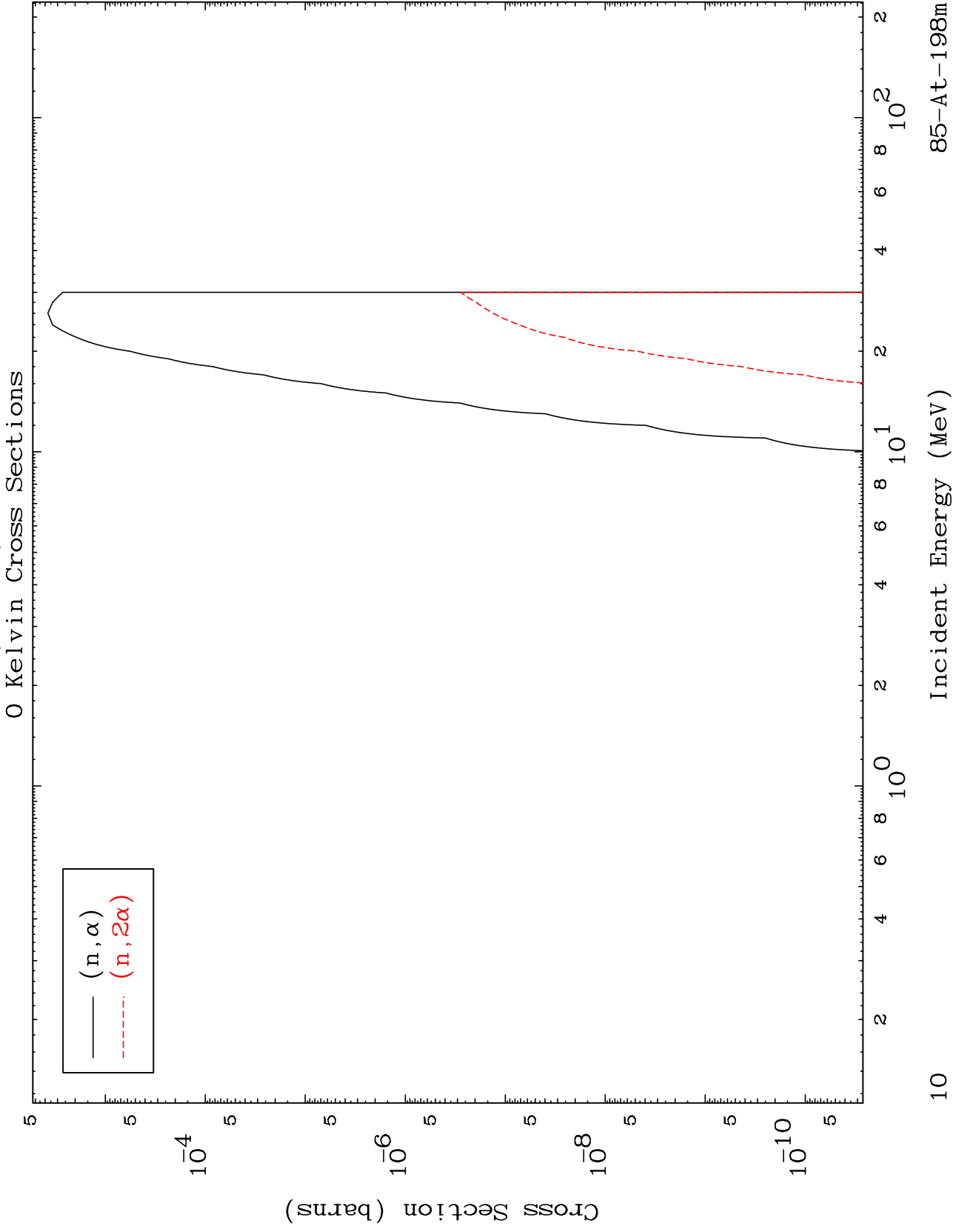
85-At-198m



MAT 8511

(He-3,  $\alpha$ ) Levels

85-At-198m

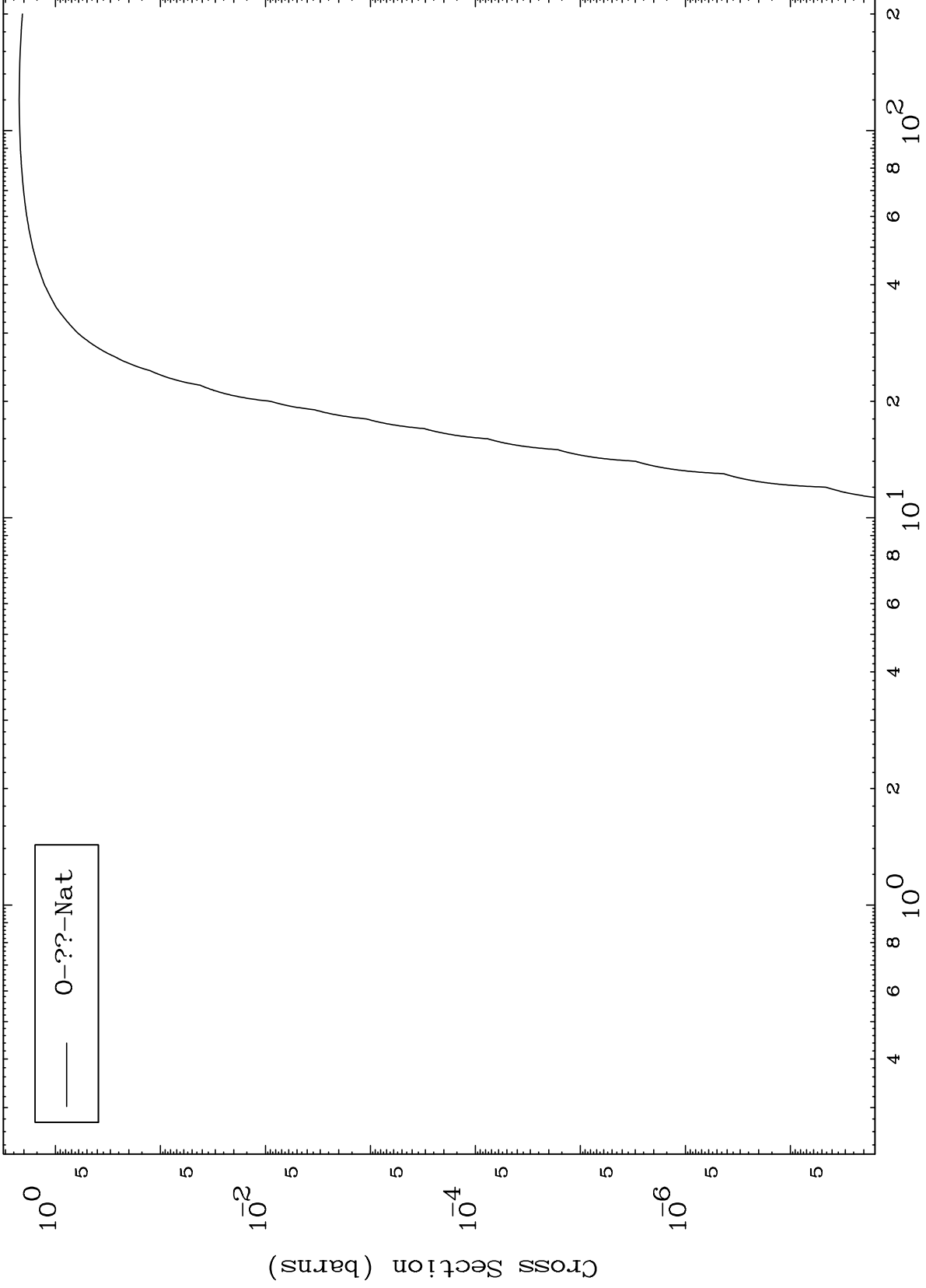
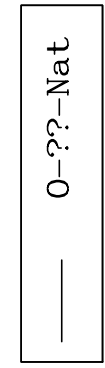


MAT 8511

Fission

85-At-198m

Radionuclide Production Cross Section

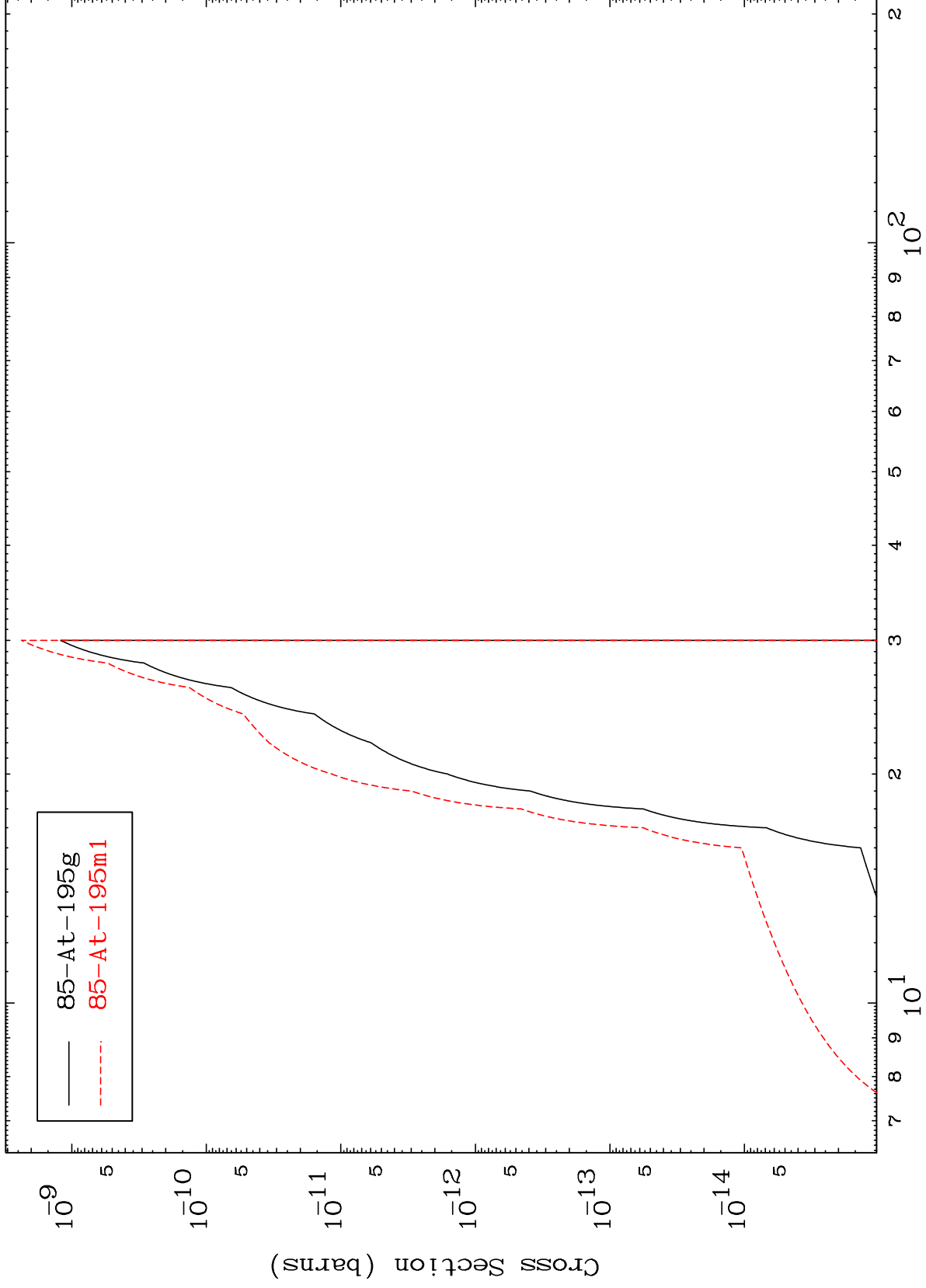


MAT 8511

(n,2n)  $\alpha$

85-At-198m

Radionuclide Production Cross Section



12

Incident Energy (MeV)

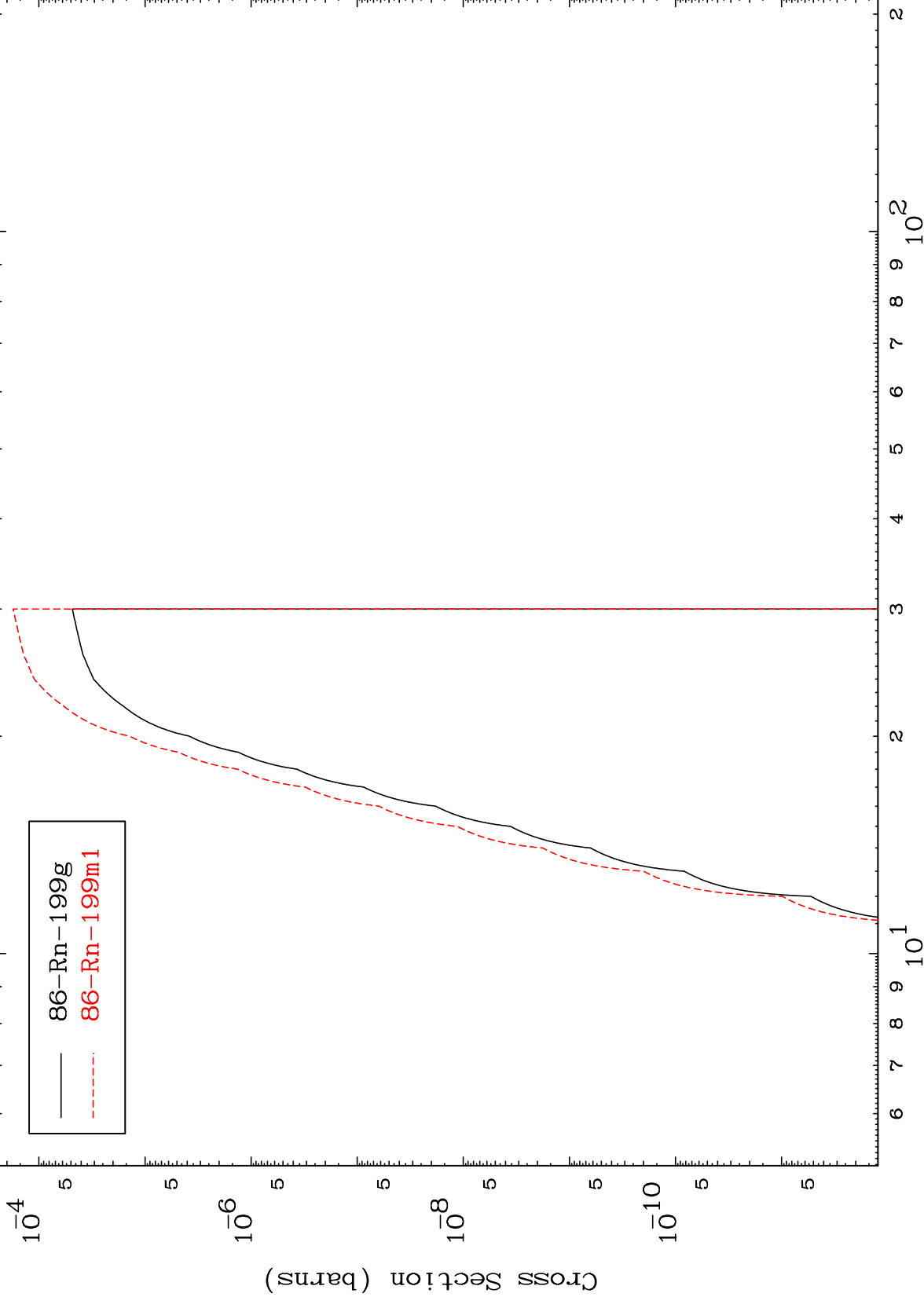
85-At-198m

MAT 8511

(n,n') p

85-At-198m

Radionuclide Production Cross Section

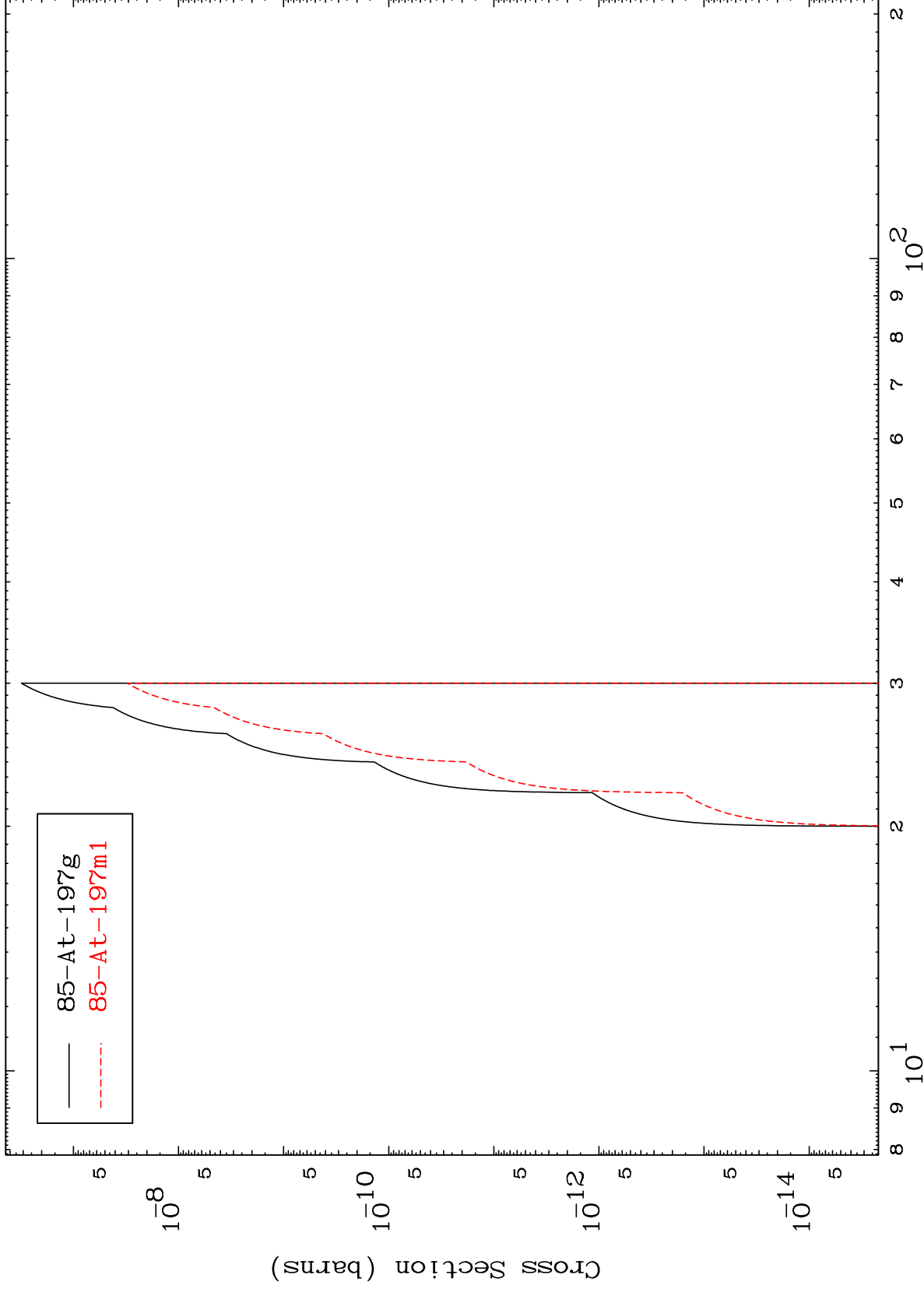


MAT 8511

(n,n') He-3

85-At-198m

Radionuclide Production Cross Section



14

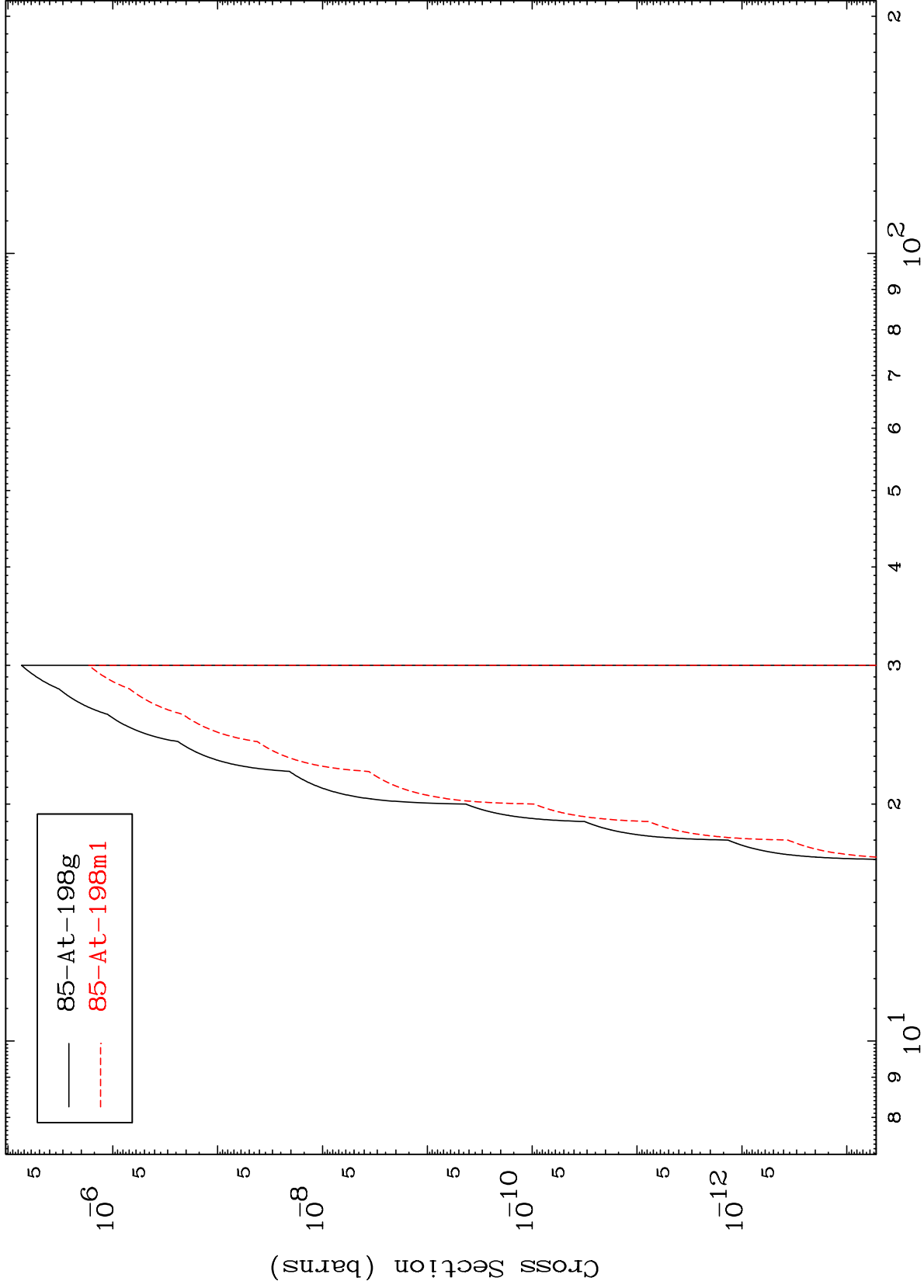
Incident Energy (MeV)

85-At-198m

MAT 8511

85-At-198m

(n,2n) p  
Radionuclide Production Cross Section



15

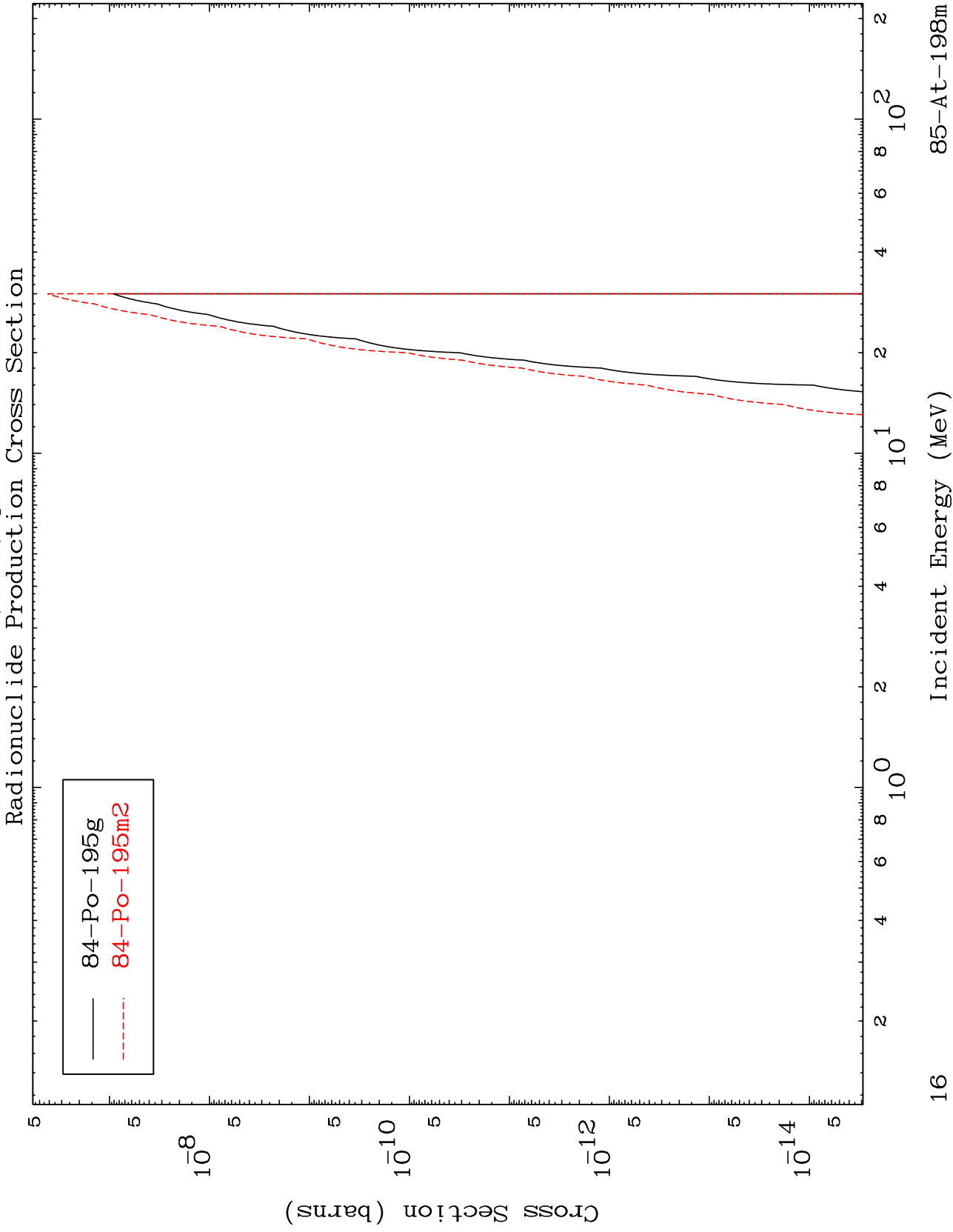
Incident Energy (MeV)

85-At-198m

MAT 8511

(n,n') p  $\alpha$

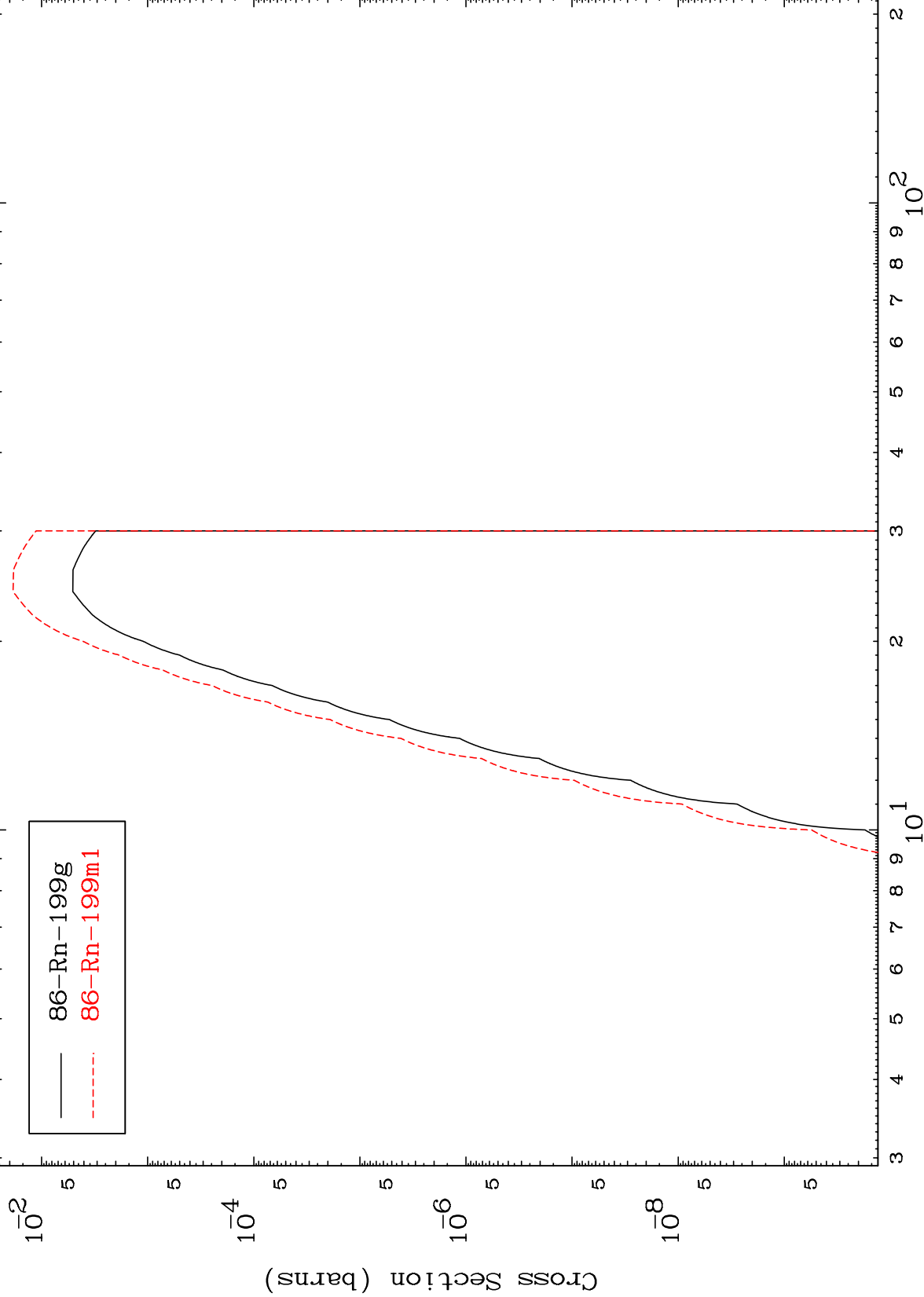
85-At-198m



MAT 8511

85-At-198m

(n,d)  
Radionuclide Production Cross Section



17

85-At-198m

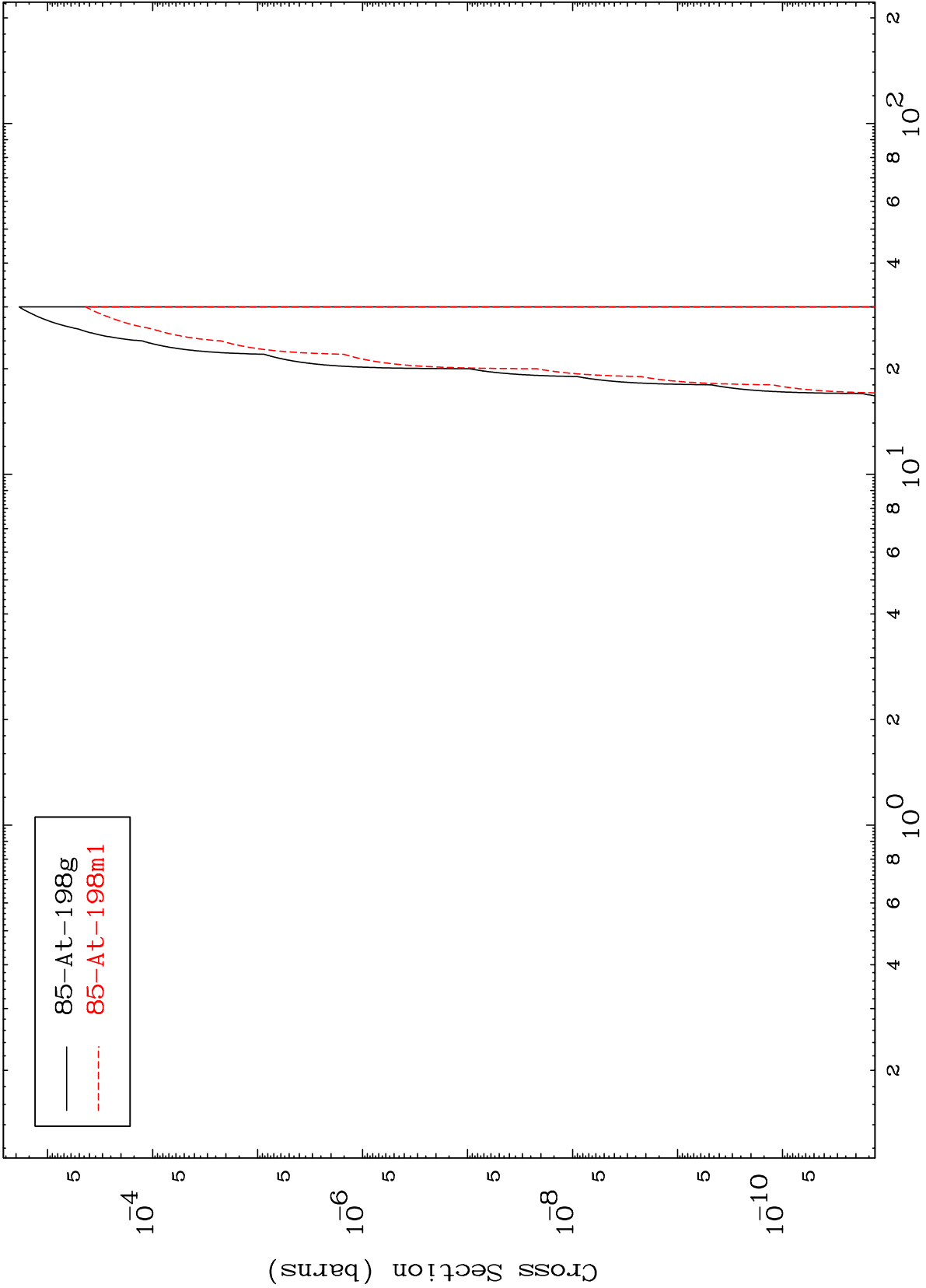
Incident Energy (MeV)

MAT 8511

(n,He-3)

85-At-198m

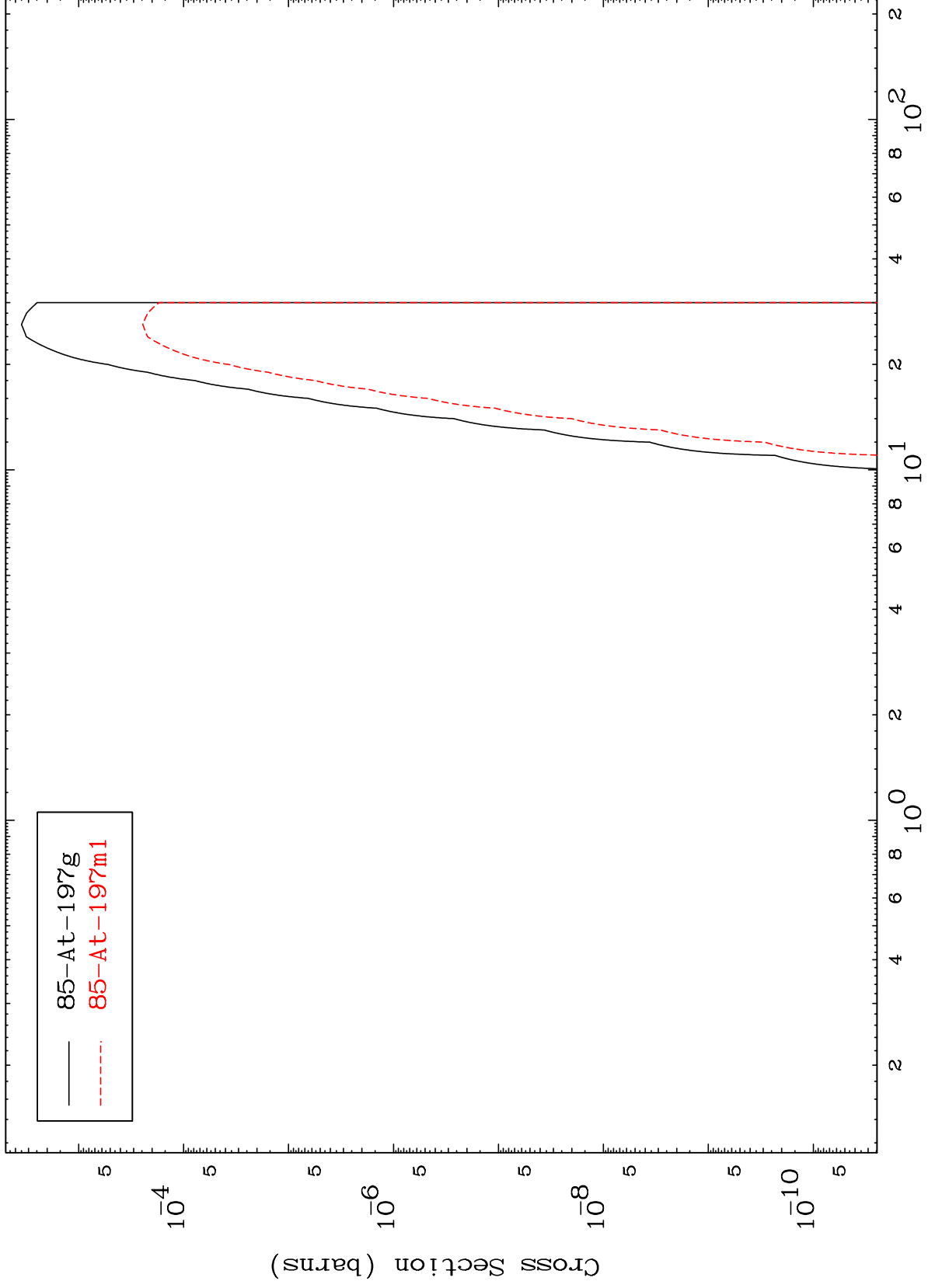
Radionuclide Production Cross Section



MAT 8511

85-At-198m

Radionuclide Production Cross Section

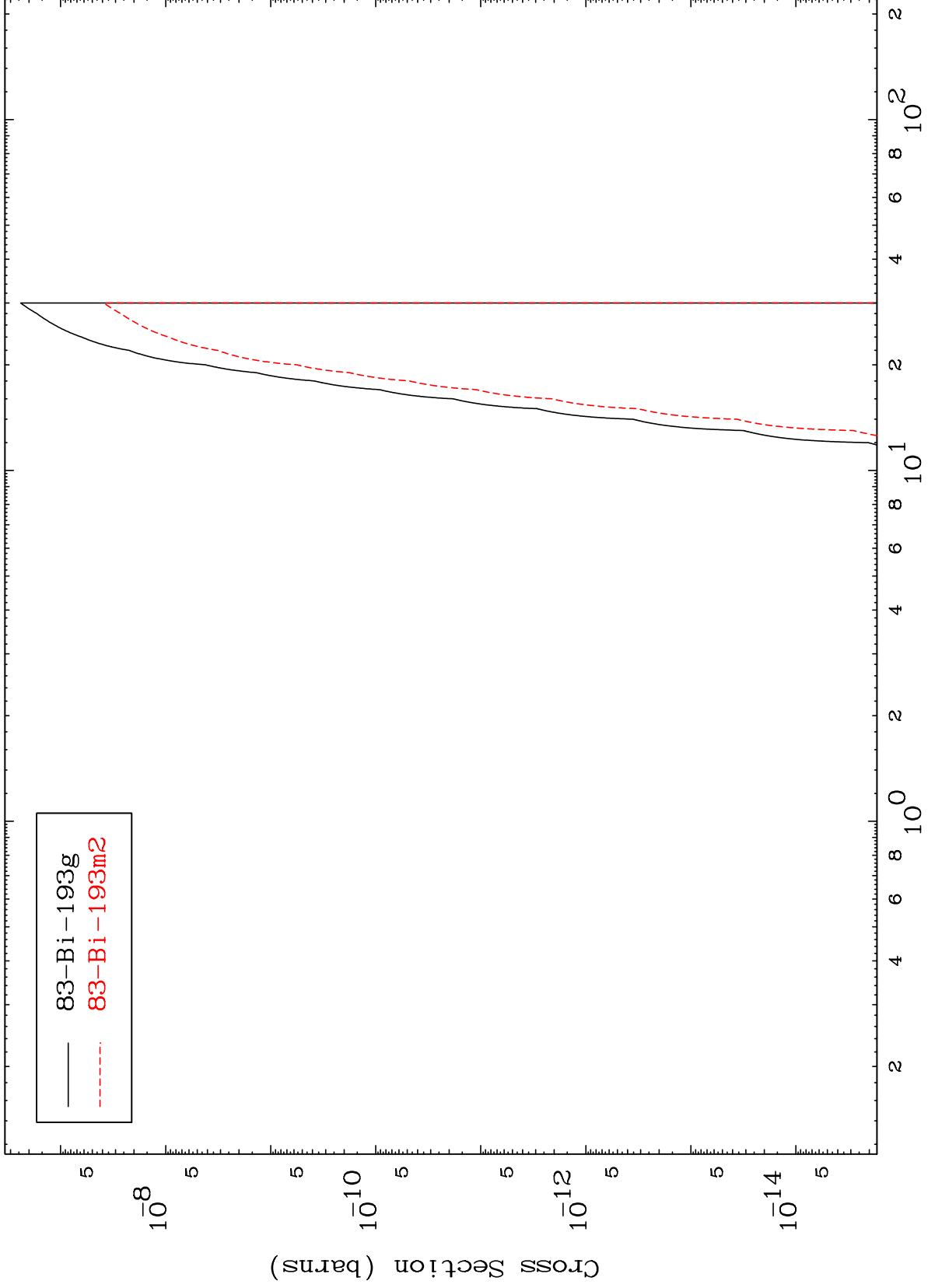


MAT 8511

(n,2α)

85-At-198m

Radionuclide Production Cross Section



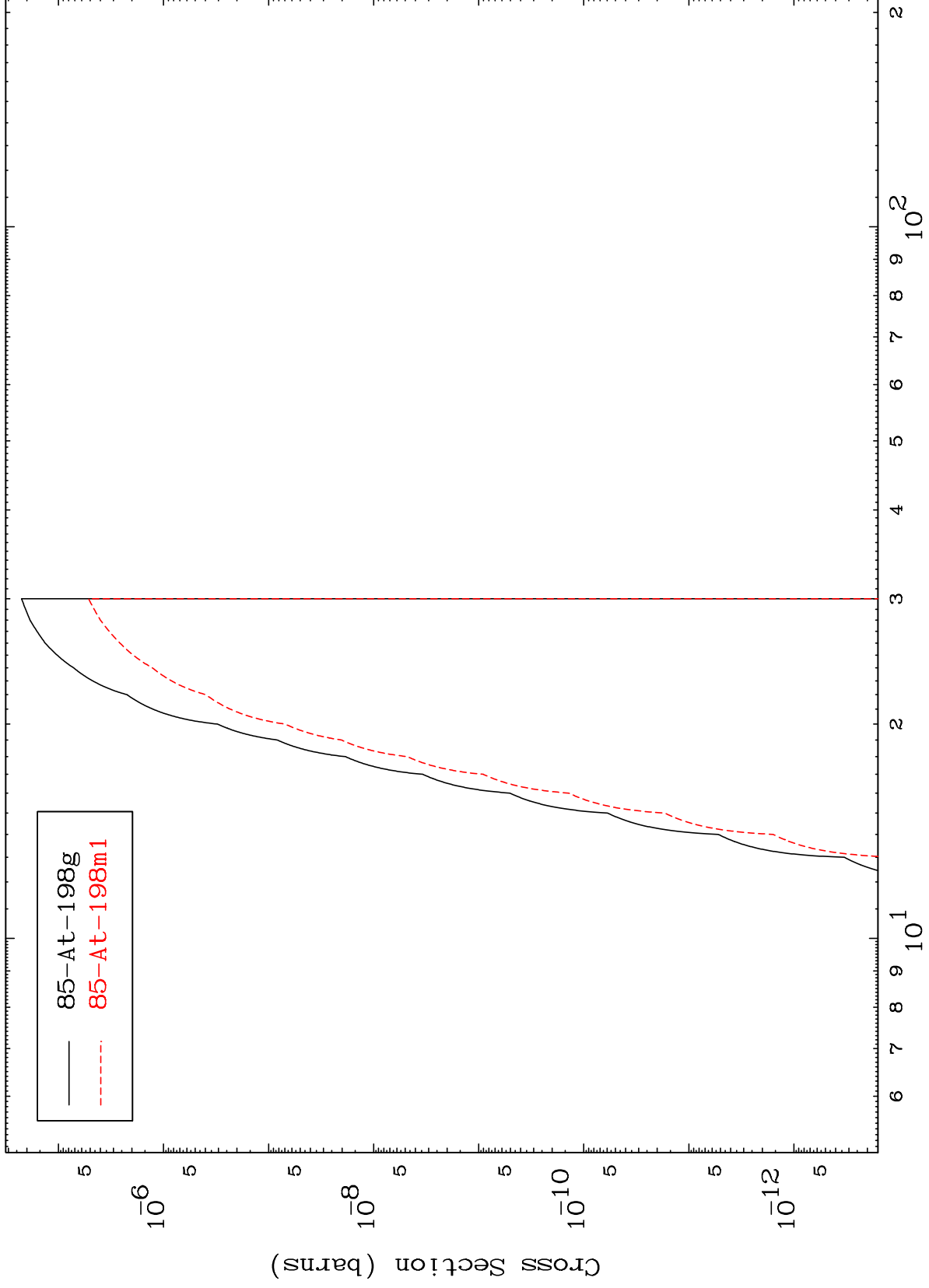
83-Bi-193g  
83-Bi-193m2

MAT 8511

(n,p) d

85-At-198m

Radionuclide Production Cross Section

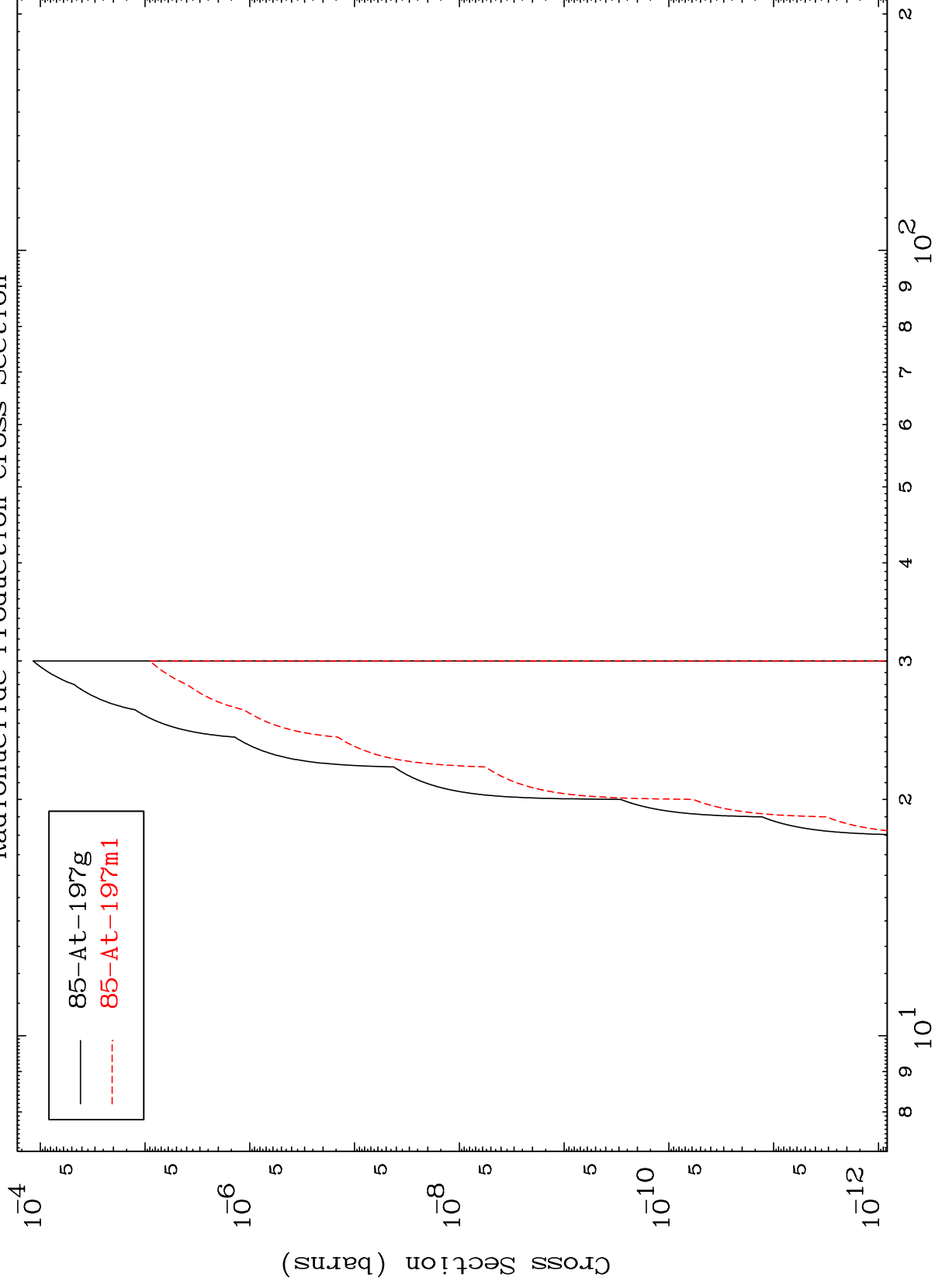


MAT 8511

(n,p) t

85-At-198m

Radionuclide Production Cross Section



Incident Energy (MeV)

85-At-198m

22

MAT 8511

(n,d)  $\alpha$

85-At-198m

