

Program EVALPLOT
(Version 2021-1)

by

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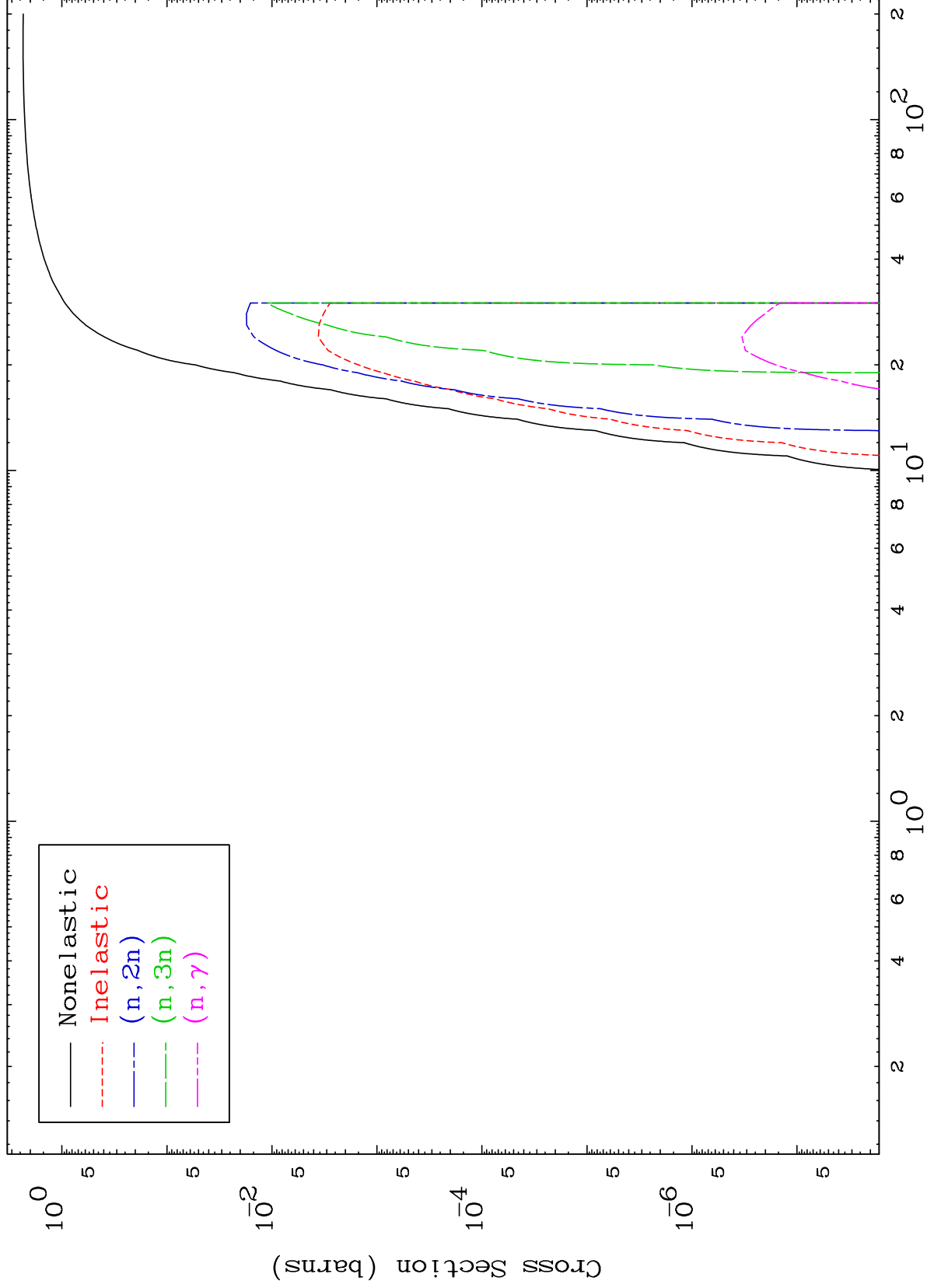
Press Mouse Button to Start

MAT 7901

He-3 Major

79-Au-189

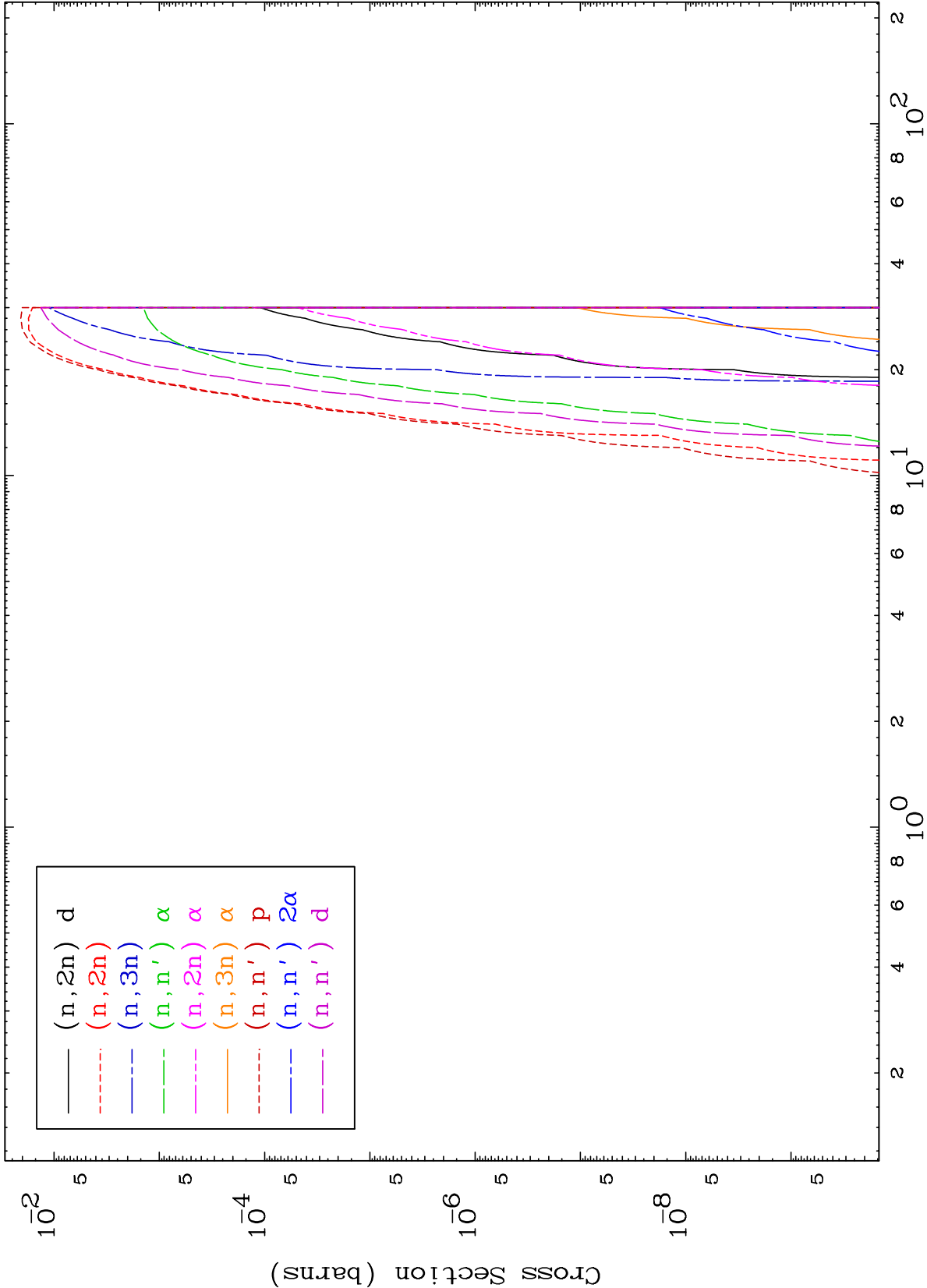
0 Kelvin Cross Sections

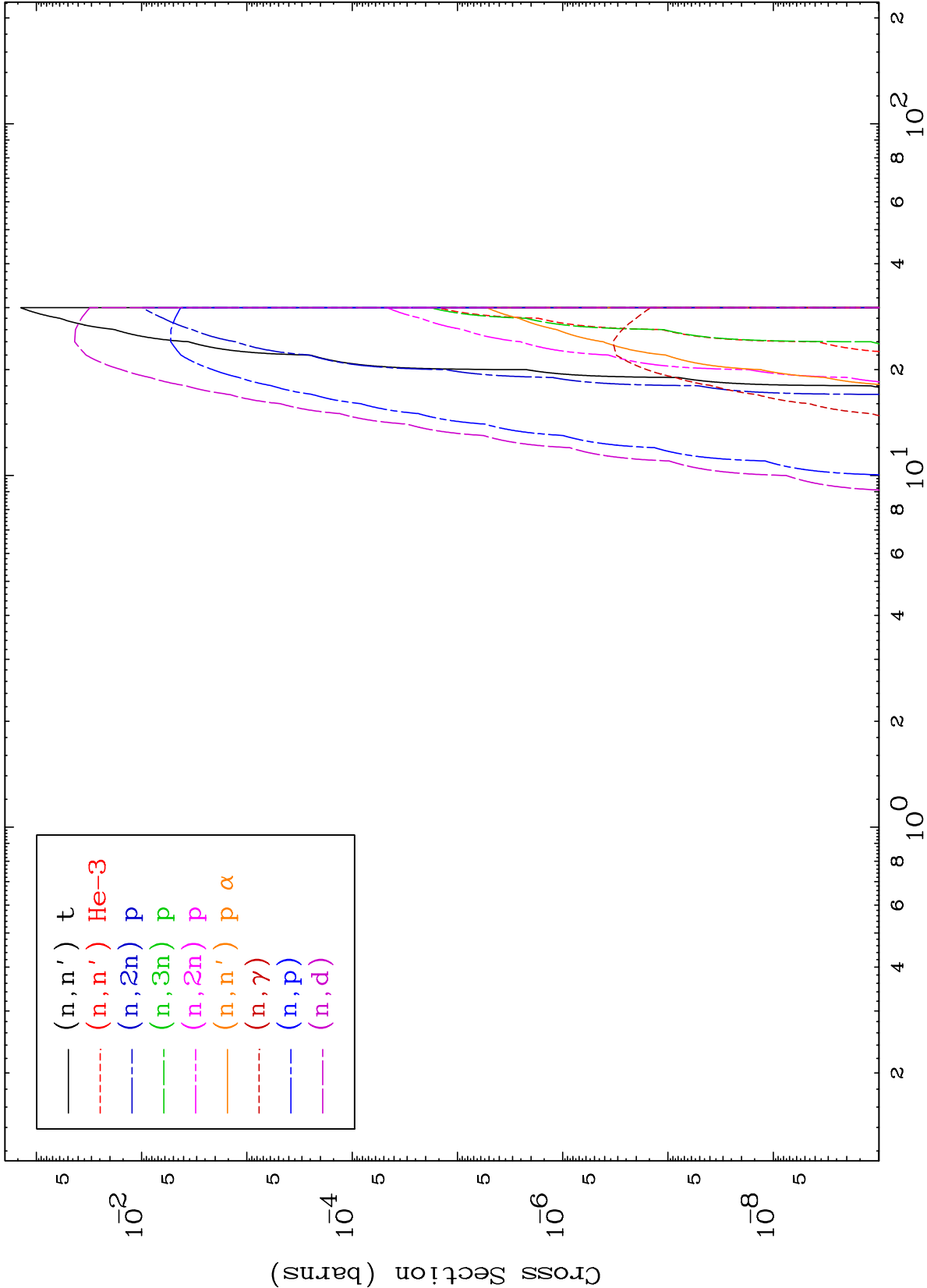


1

Incident Energy (MeV)

79-Au-189

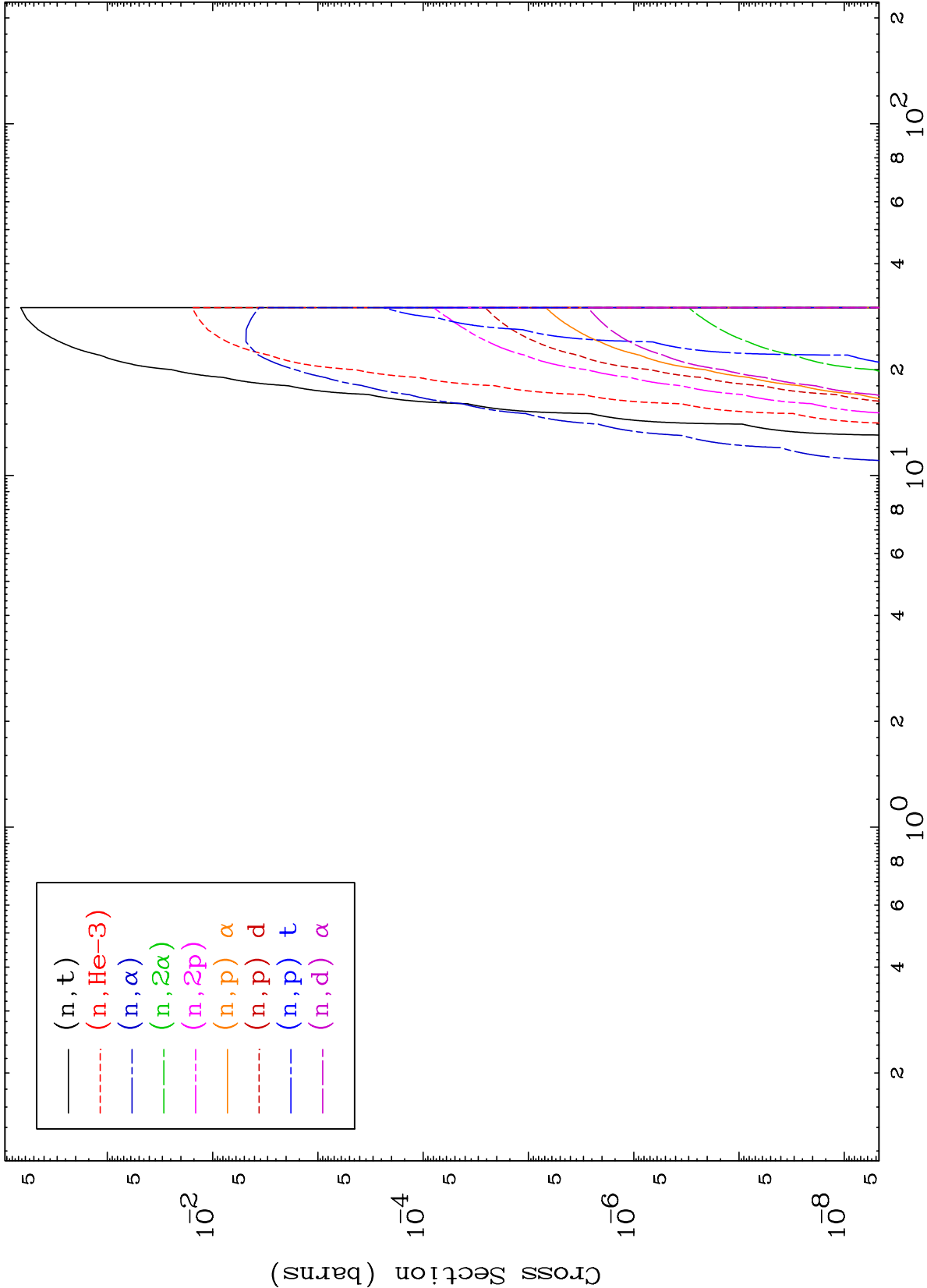


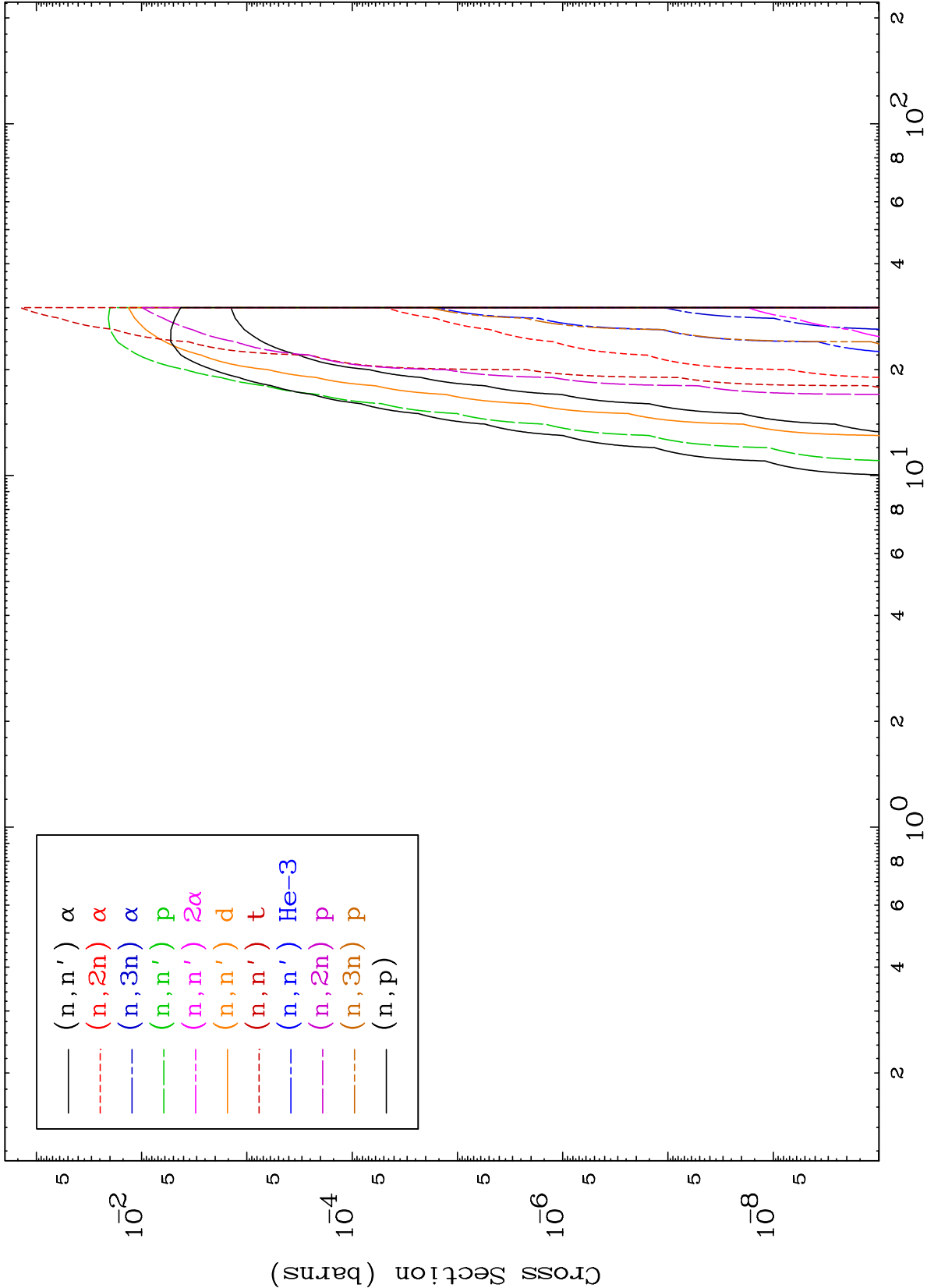


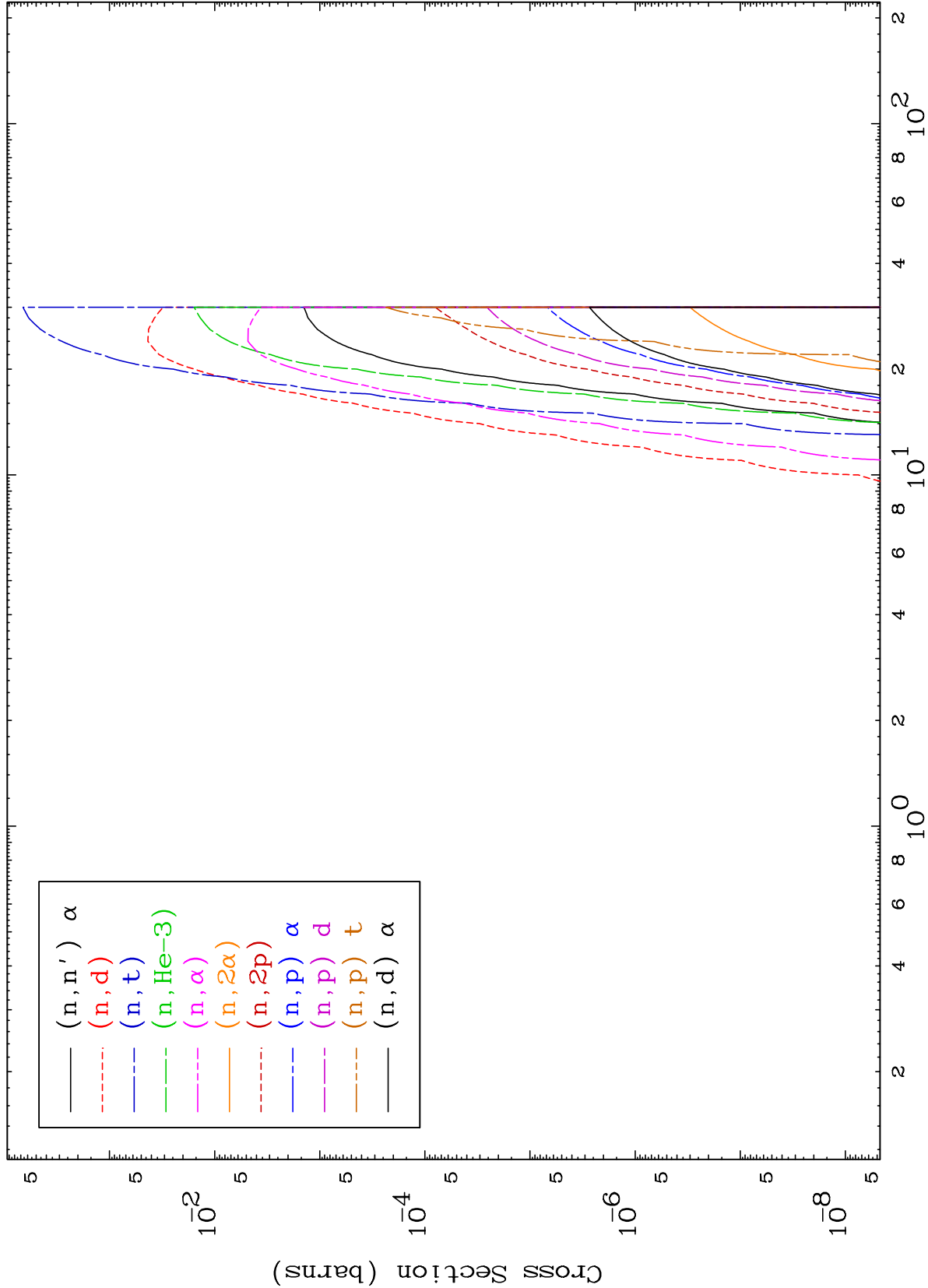
MAT 7901

He-3 Neutron Absorption
0 Kelvin Cross Sections

79-Au-189



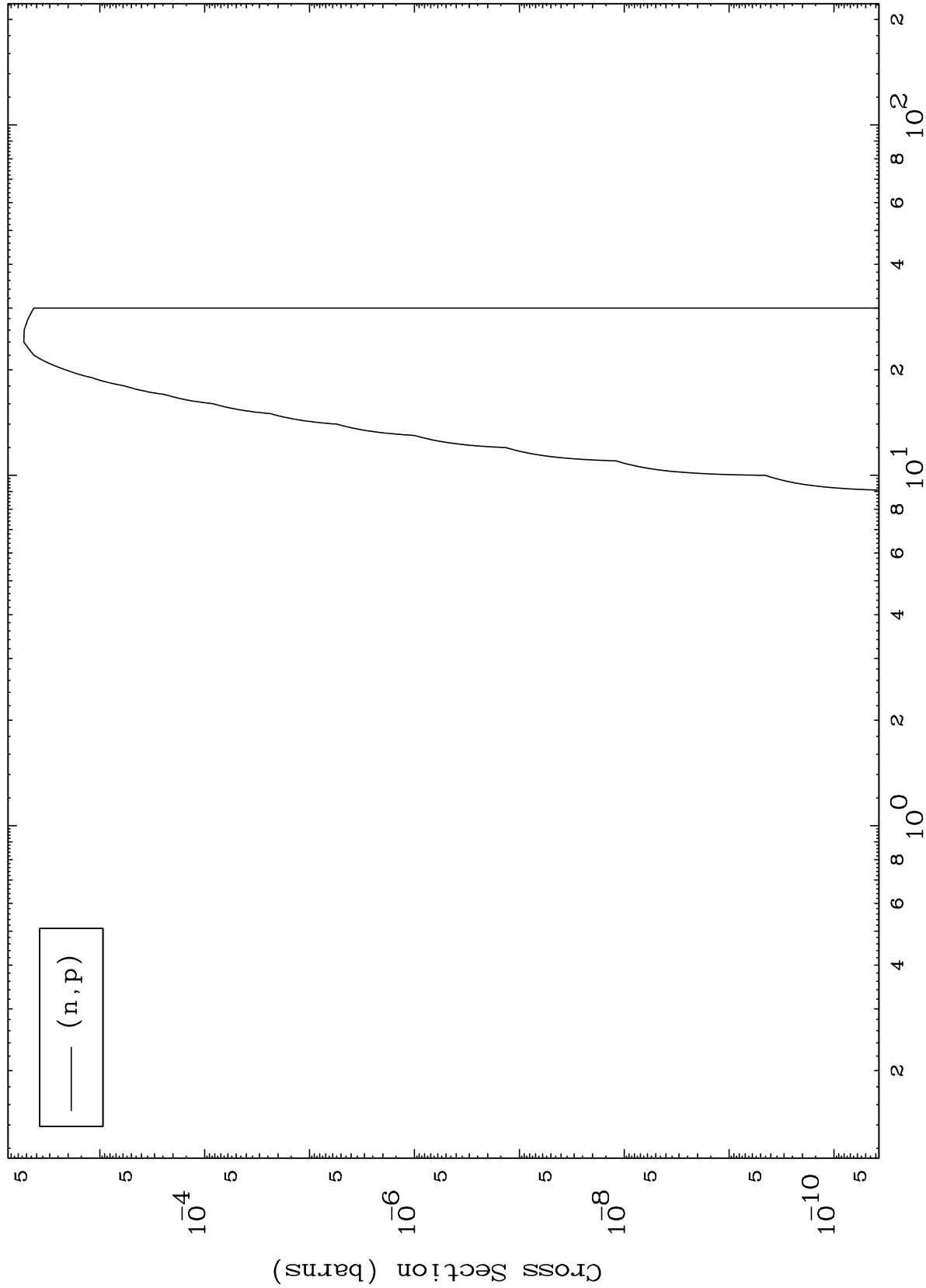




MAT 7901

79-Au-189

(He-3,p) Levels
0 Kelvin Cross Sections



79-Au-189

Incident Energy (MeV)

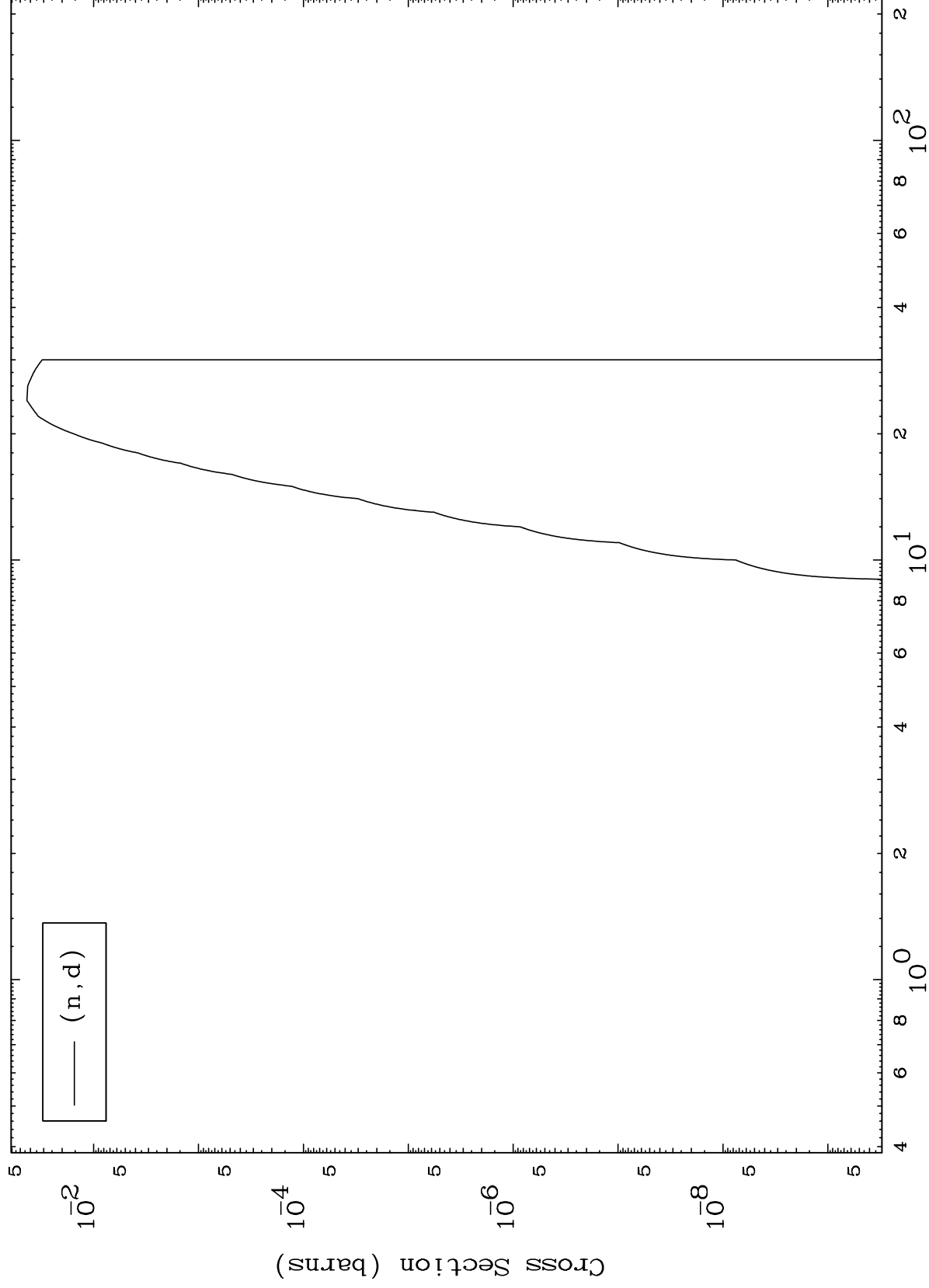
7

MAT 7901

(He-3,d) Levels

79-Au-189

0 Kelvin Cross Sections



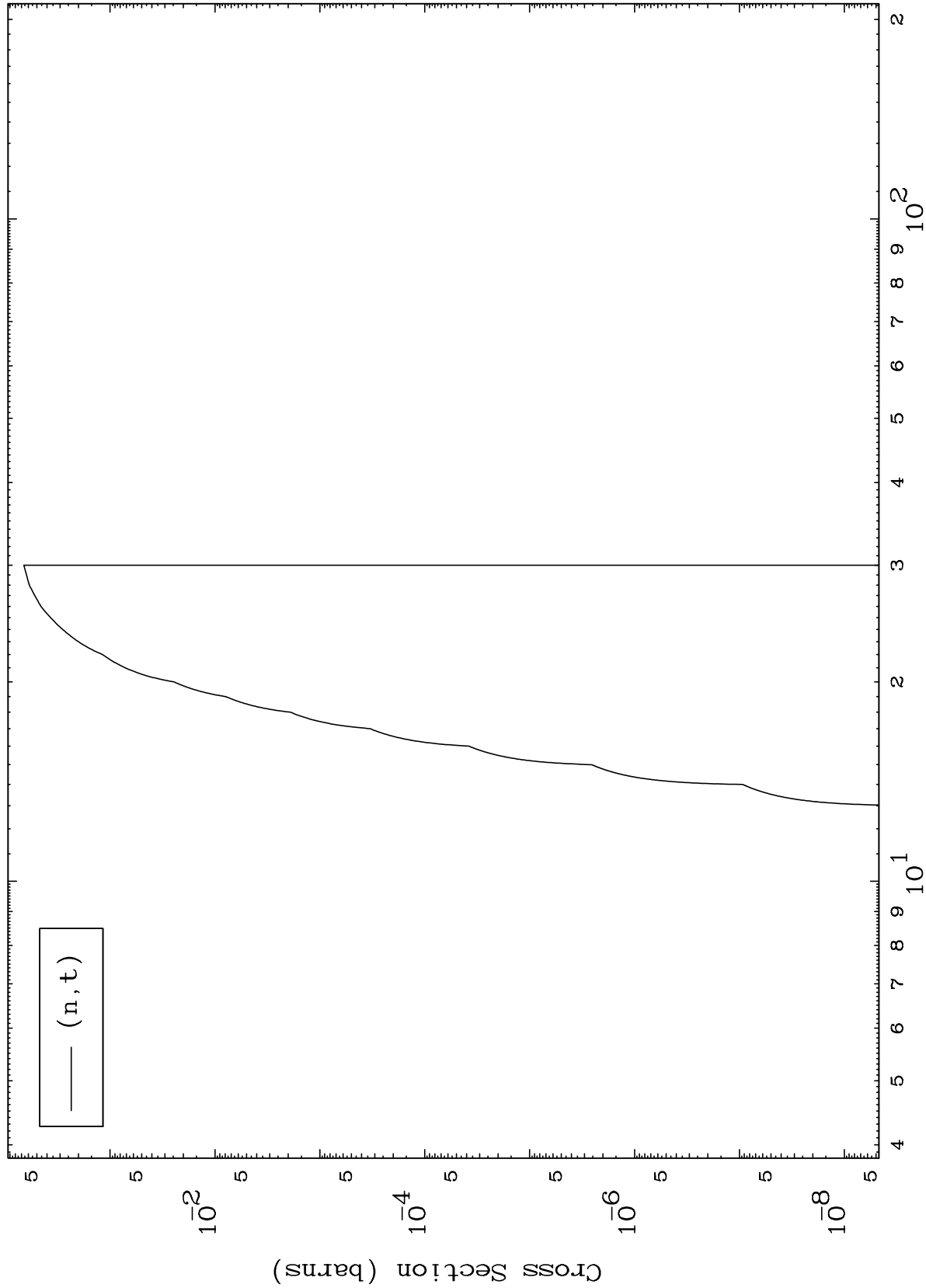
Incident Energy (MeV)

79-Au-189

MAT 7901

79-Au-189

(He-3,t) Levels
0 Kelvin Cross Sections



9

Incident Energy (MeV)

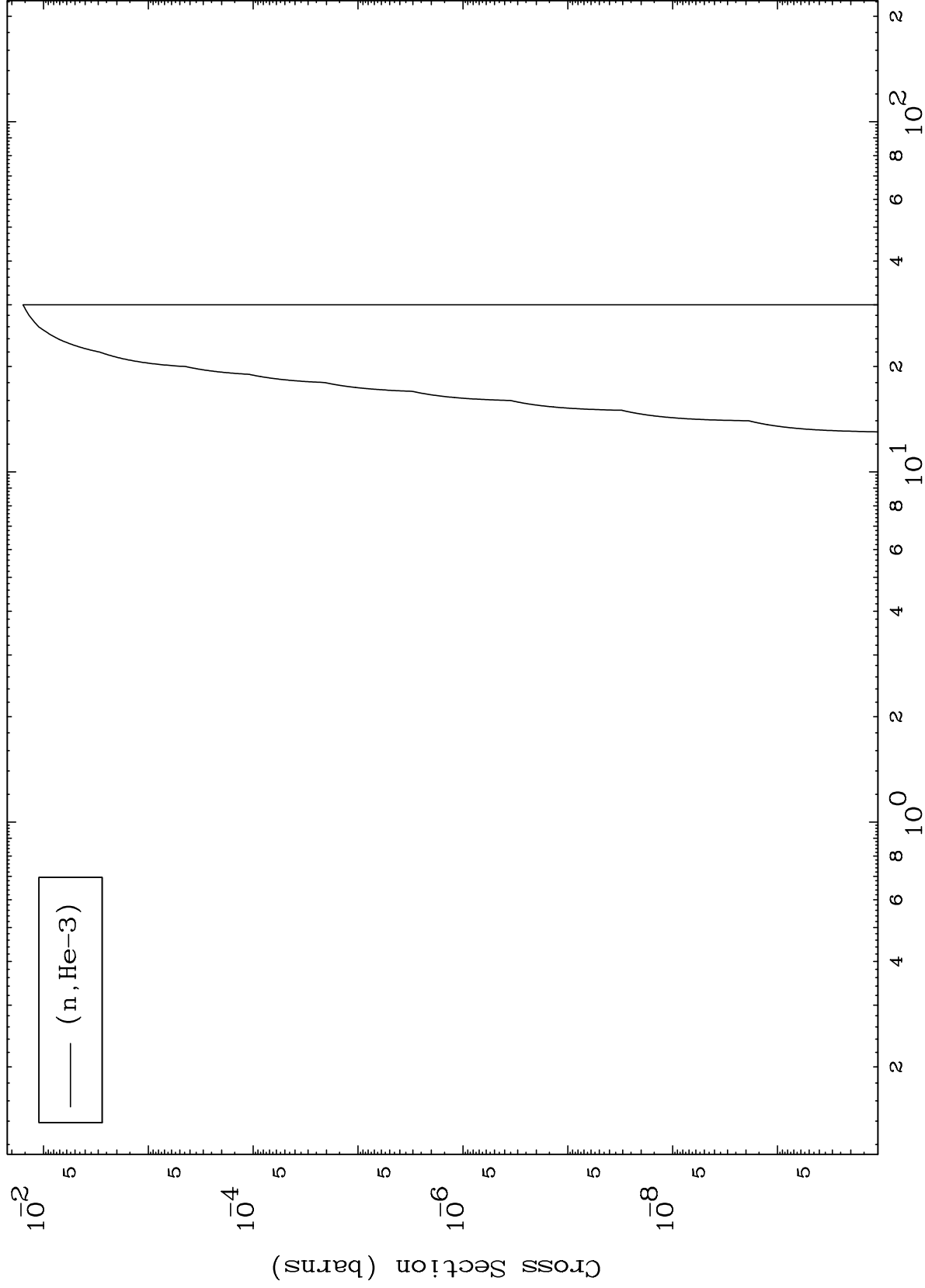
79-Au-189

MAT 7901

(He-3,He3) Levels

79-Au-189

0 Kelvin Cross Sections



10

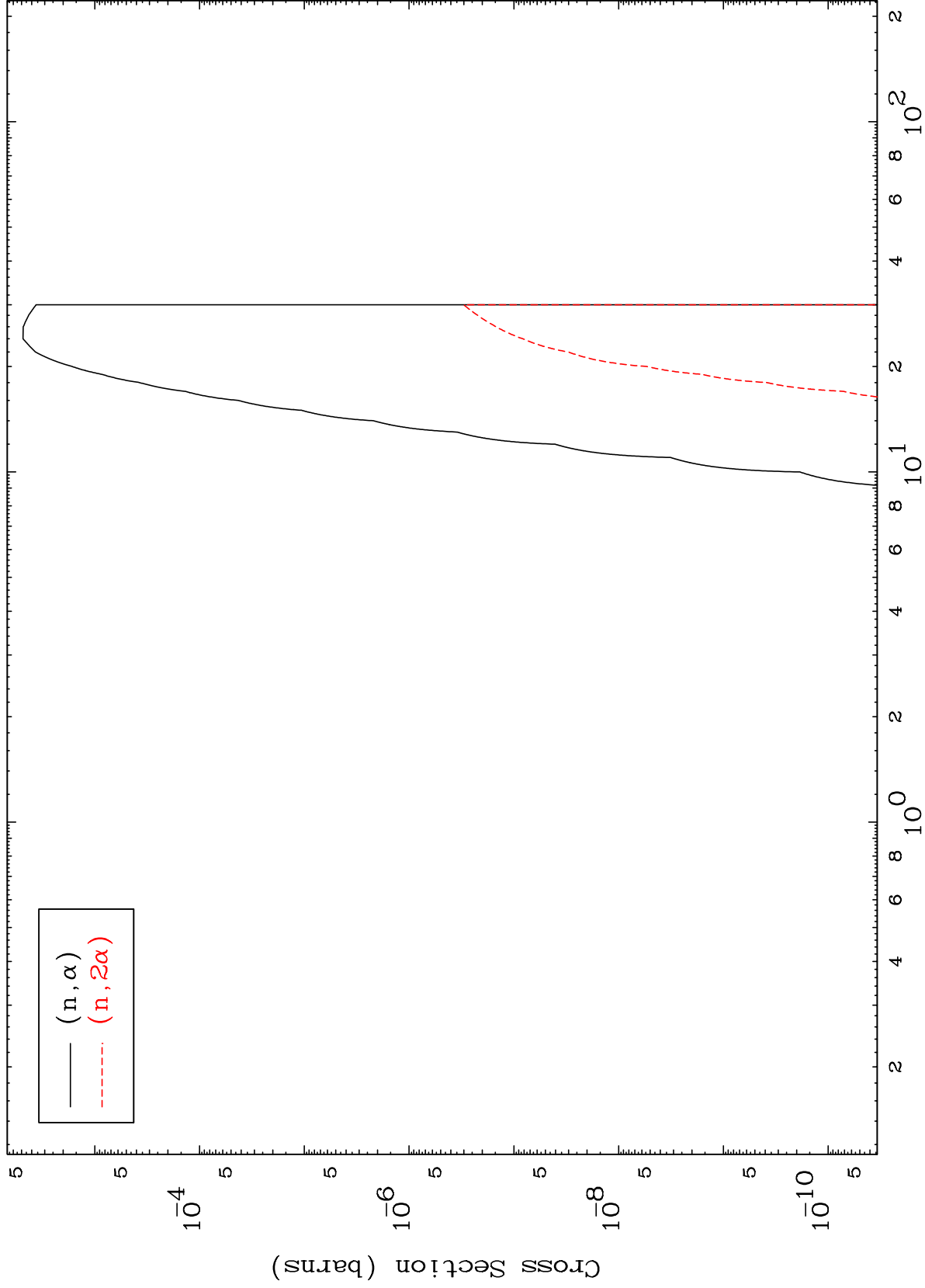
Incident Energy (MeV)

79-Au-189

MAT 7901

79-Au-189

(He-3, α) Levels
0 Kelvin Cross Sections



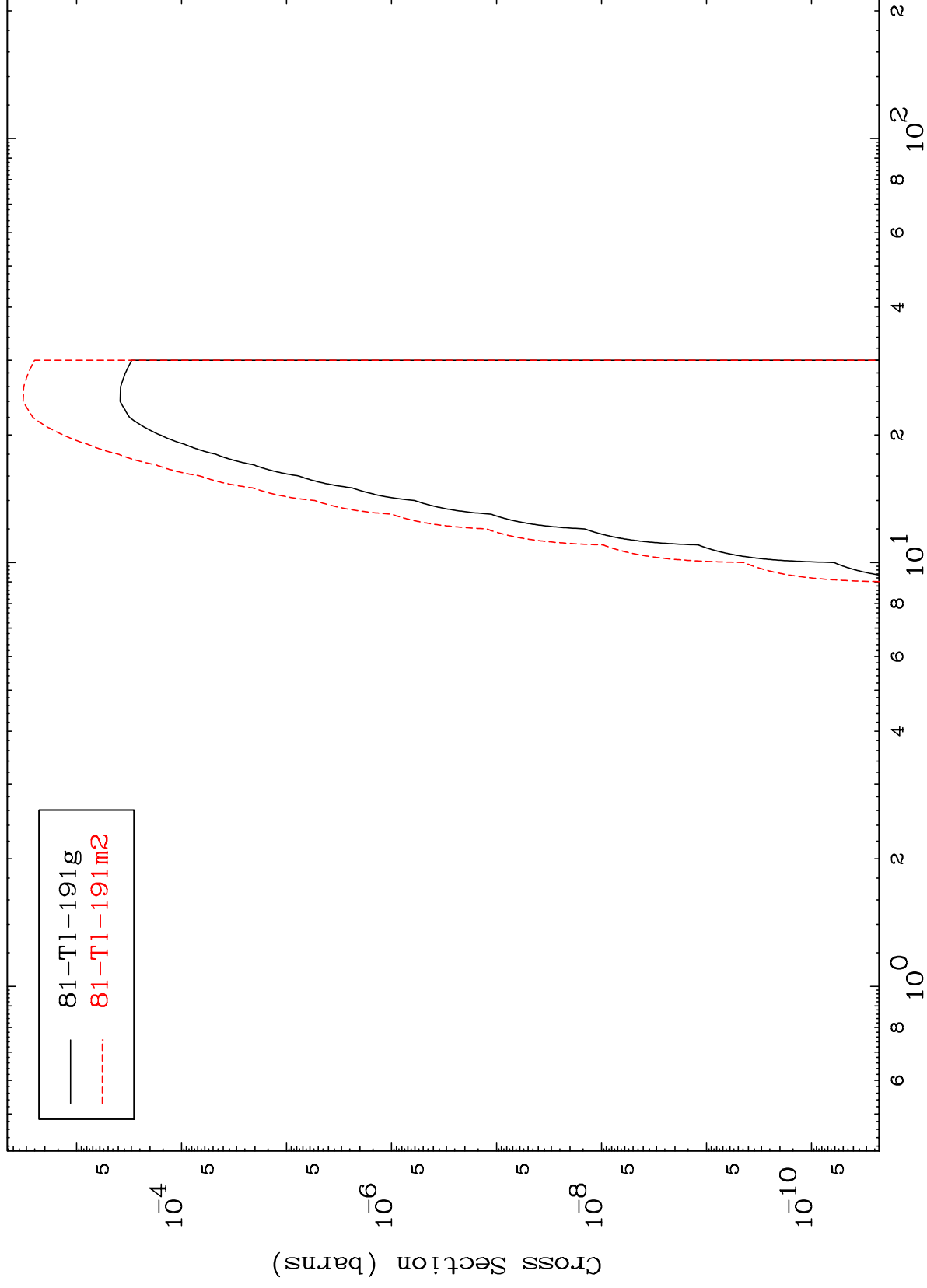
11

79-Au-189

MAT 7901

Radionuclide Production Cross Section

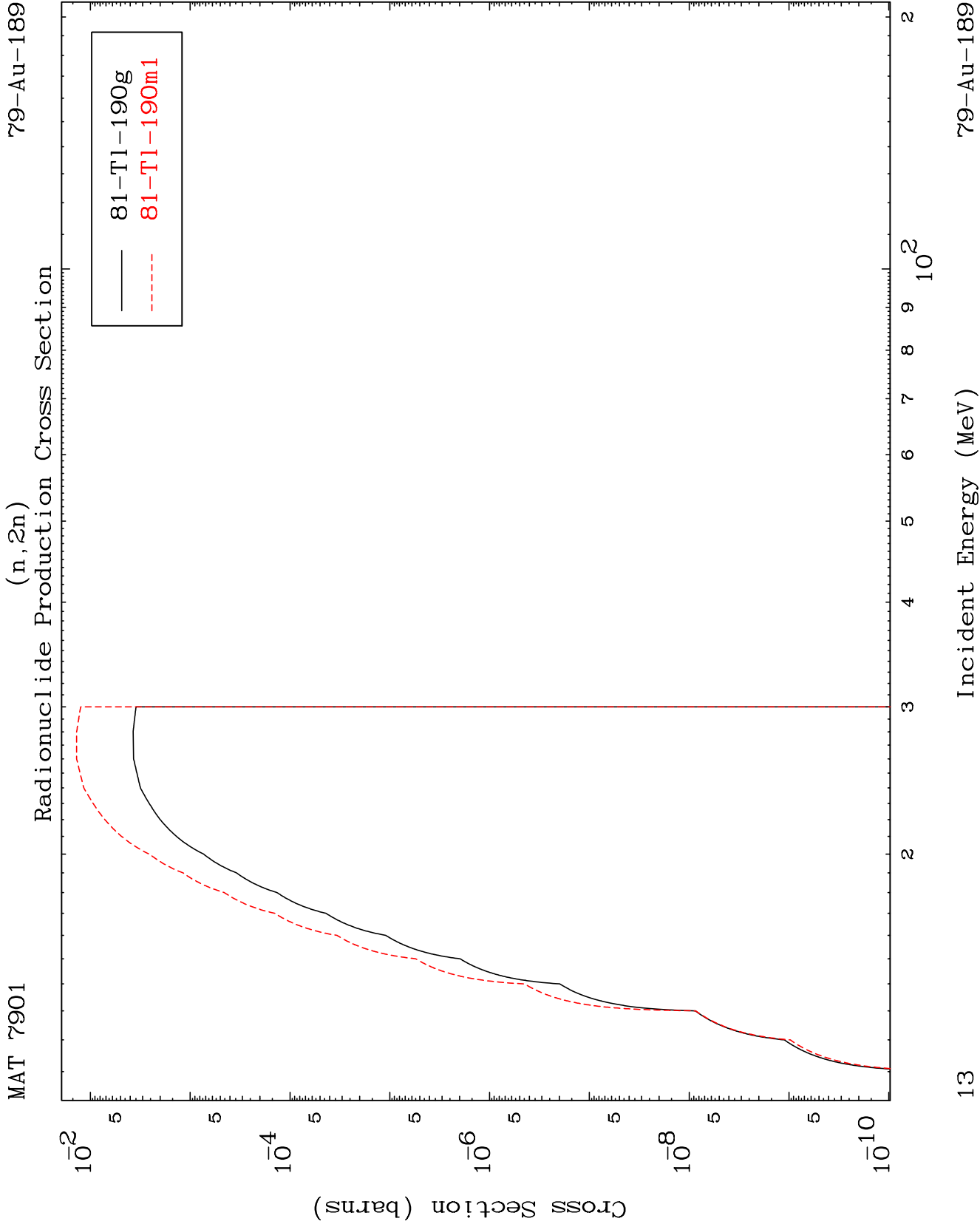
79-Au-189

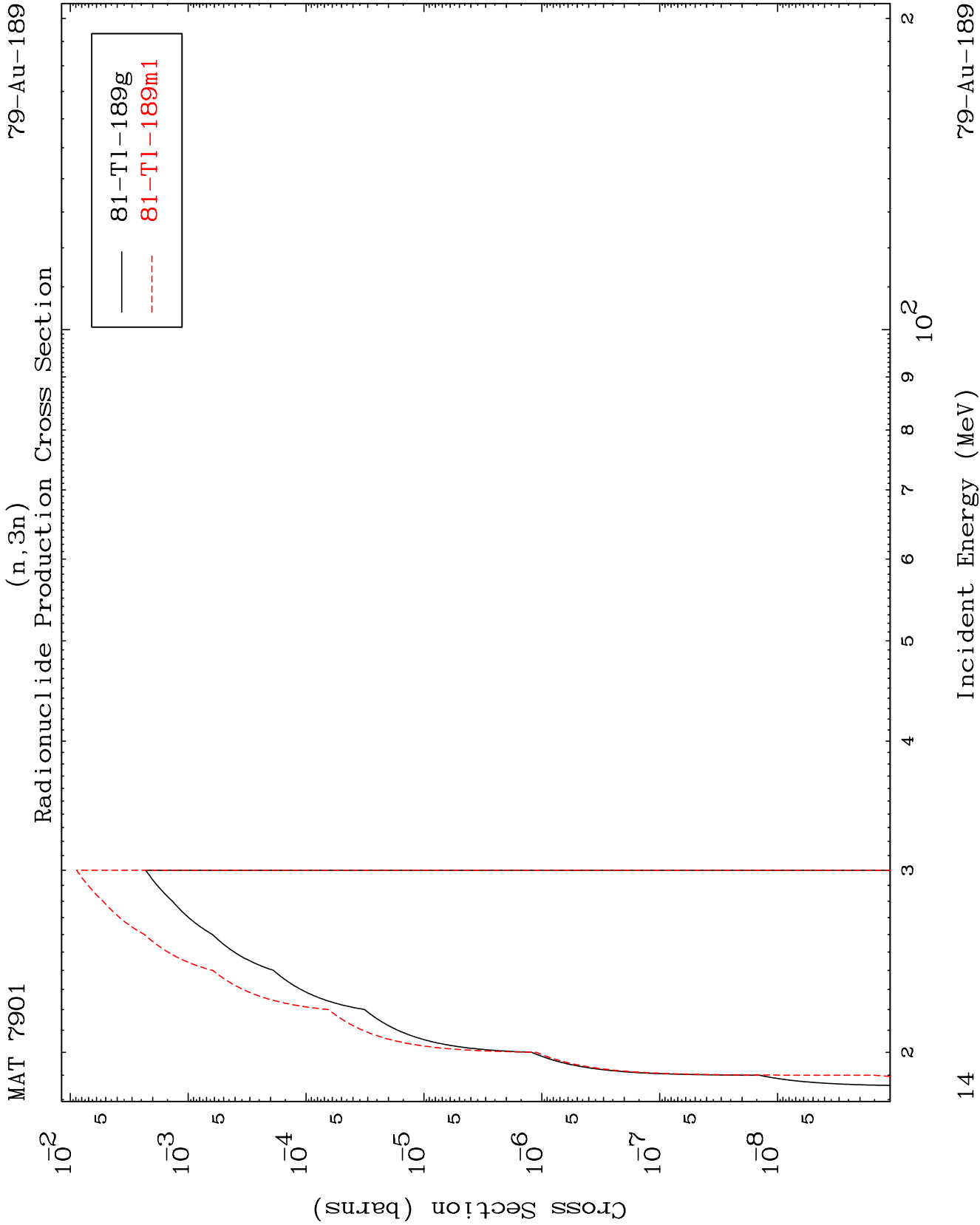


12

Incident Energy (MeV)

79-Au-189



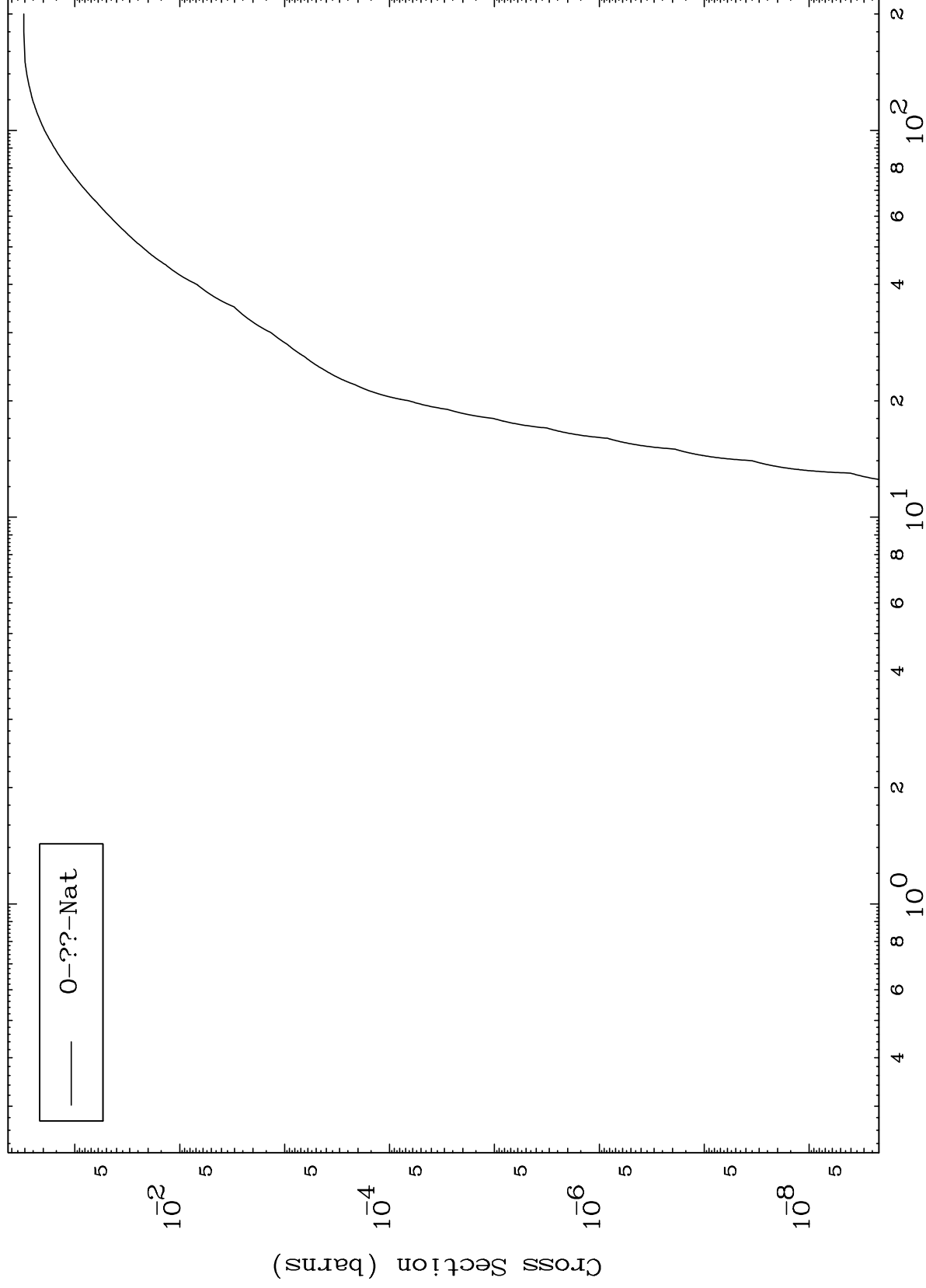


MAT 7901

Fission

⁷⁹Au-189

Radionuclide Production Cross Section



15

Incident Energy (MeV)

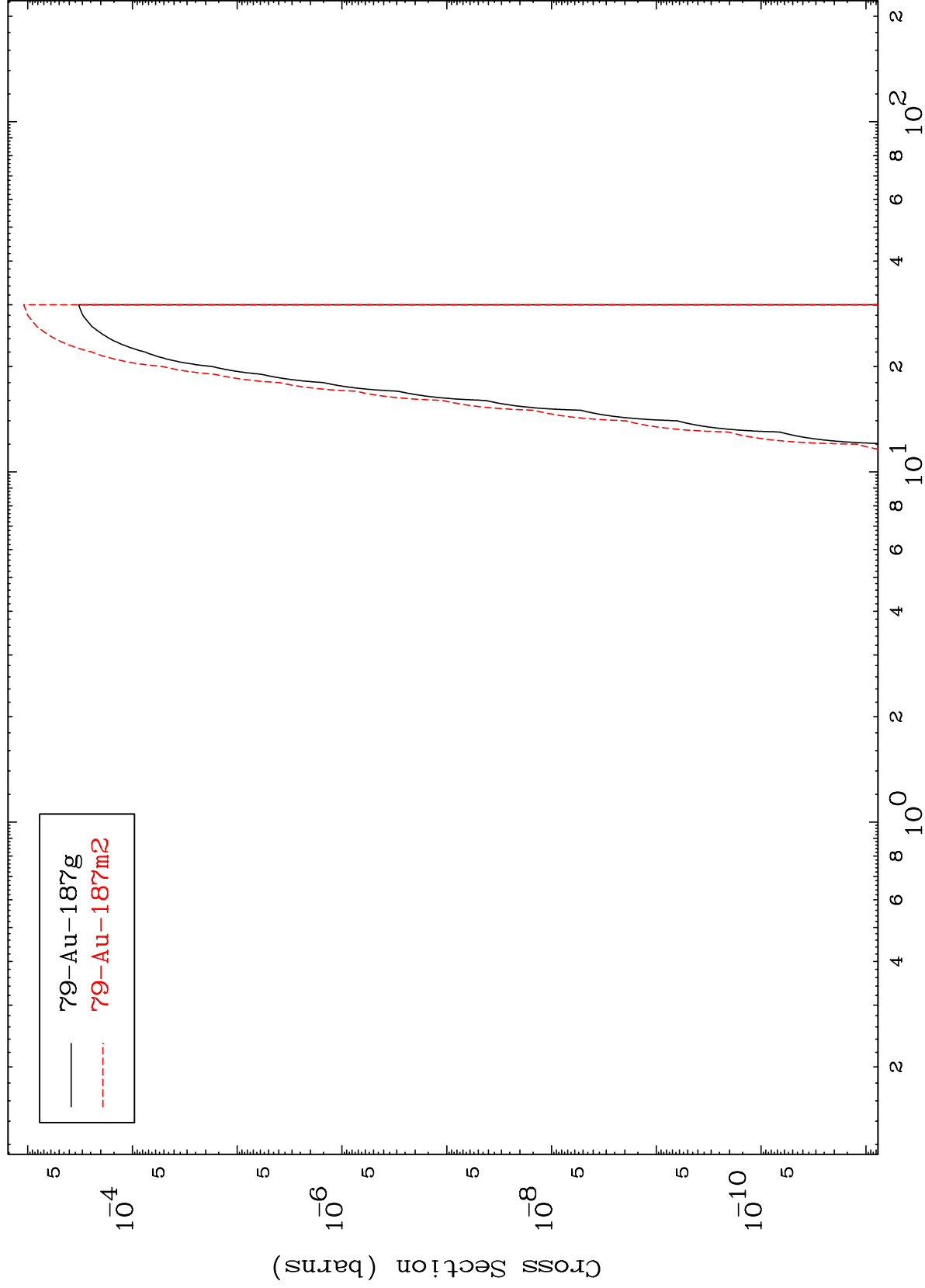
⁷⁹Au-189

MAT 7901

$(n,n') \alpha$

$^{79}\text{Au-189}$

Radionuclide Production Cross Section



16

Incident Energy (MeV)

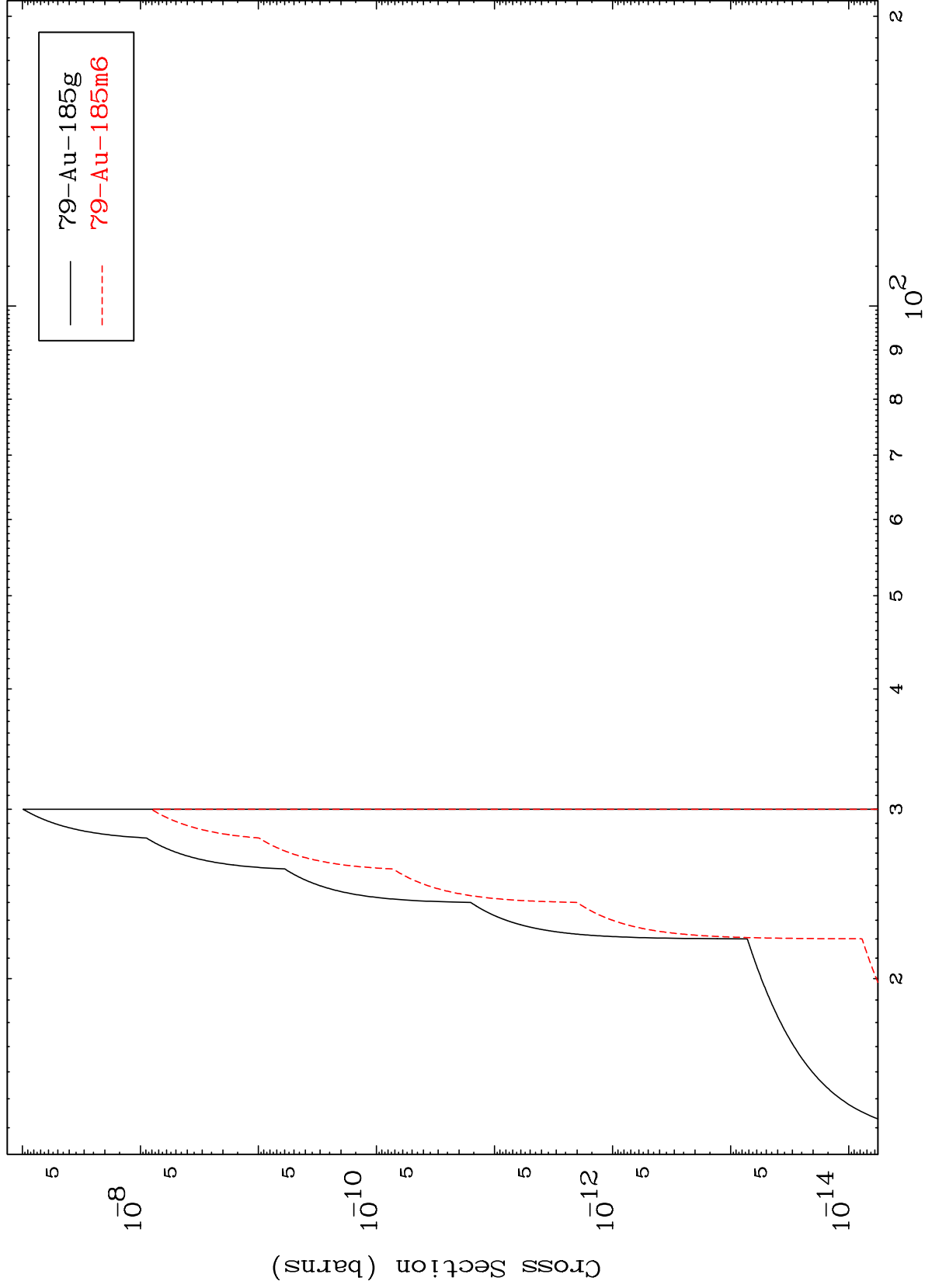
$^{79}\text{Au-189}$

MAT 7901

79-Au-189

(n,3n) α

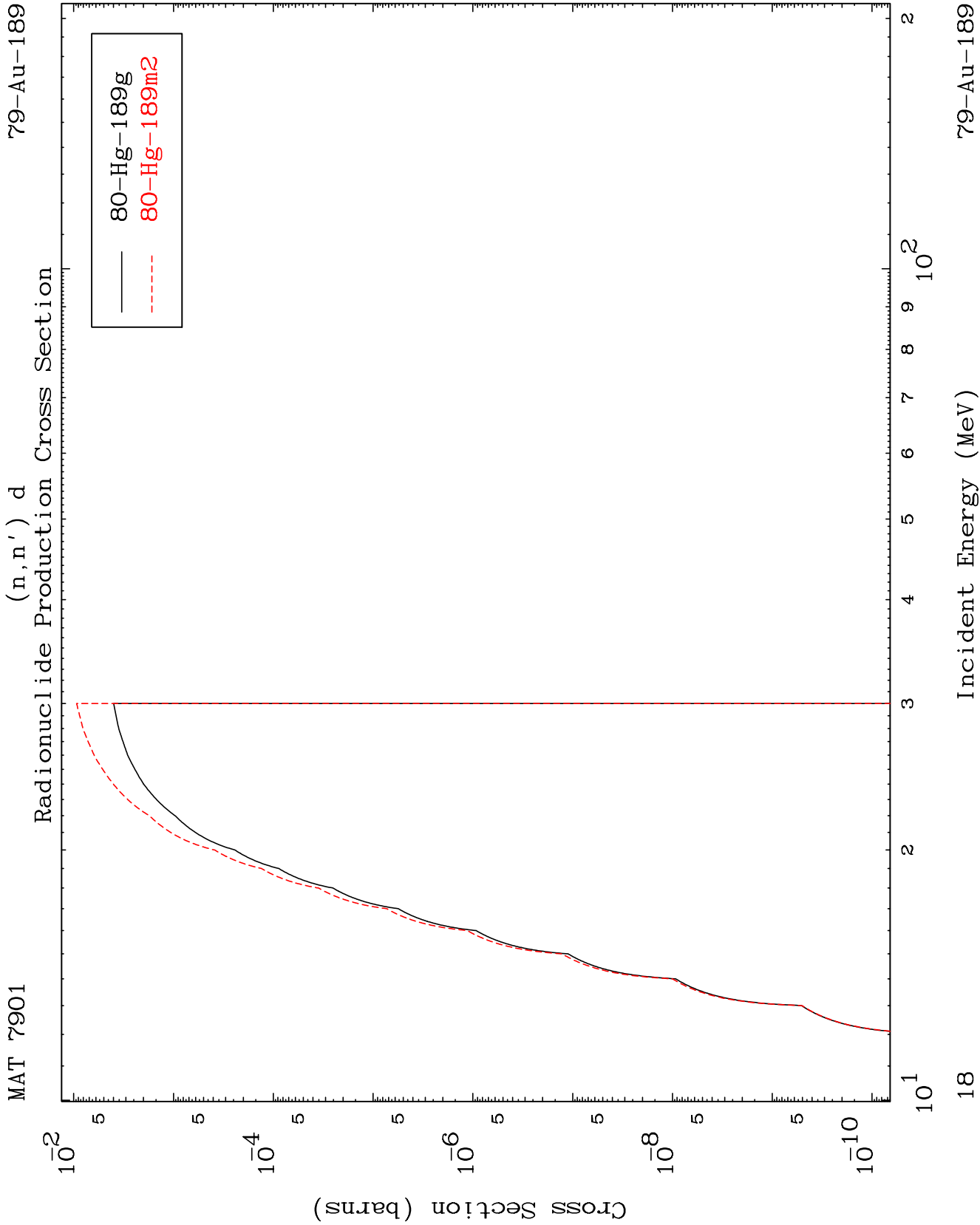
Radionuclide Production Cross Section



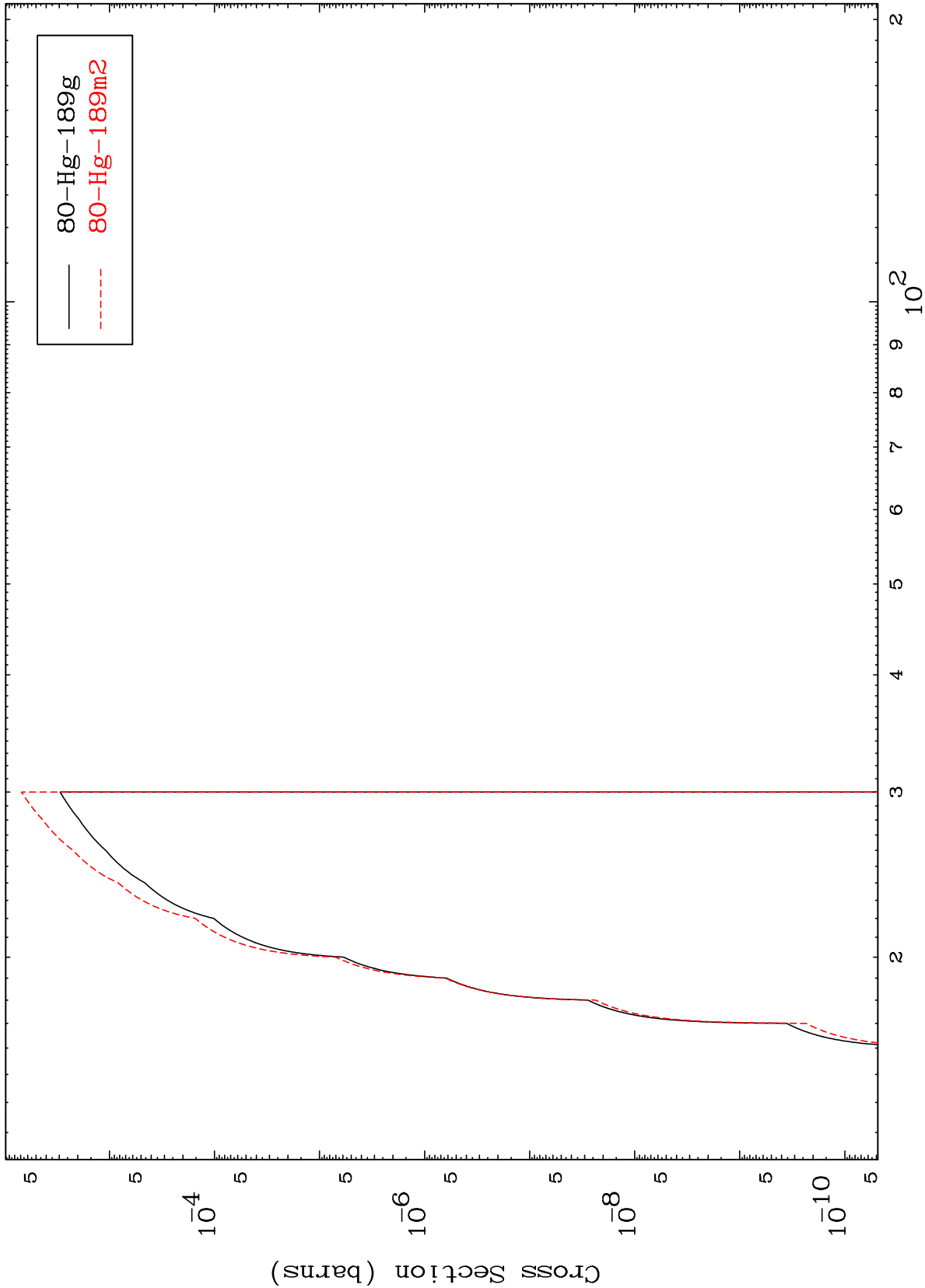
79-Au-189

Incident Energy (MeV)

17



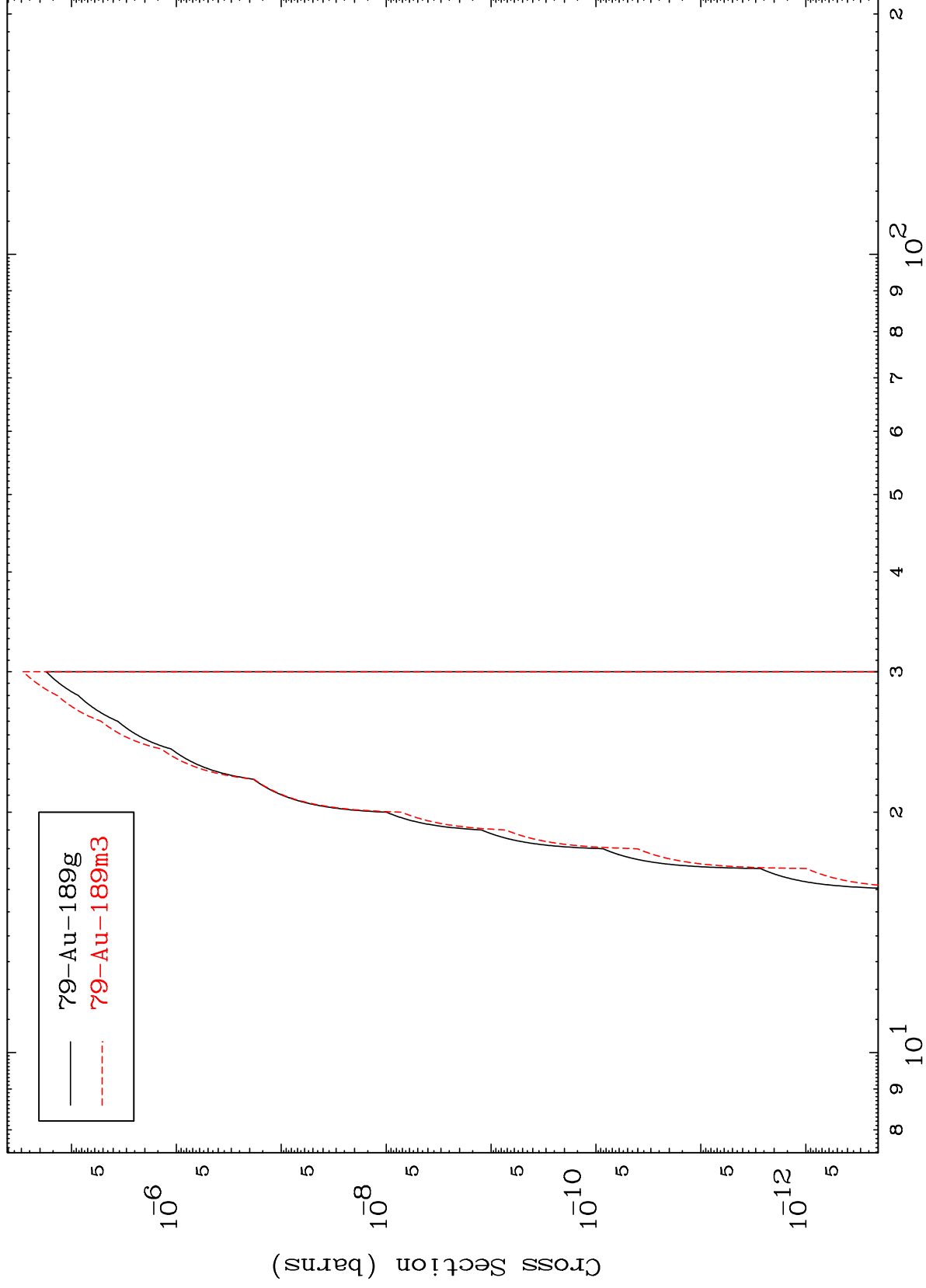
(n,2n) p
Radionuclide Production Cross Section



MAT 7901

79-Au-189

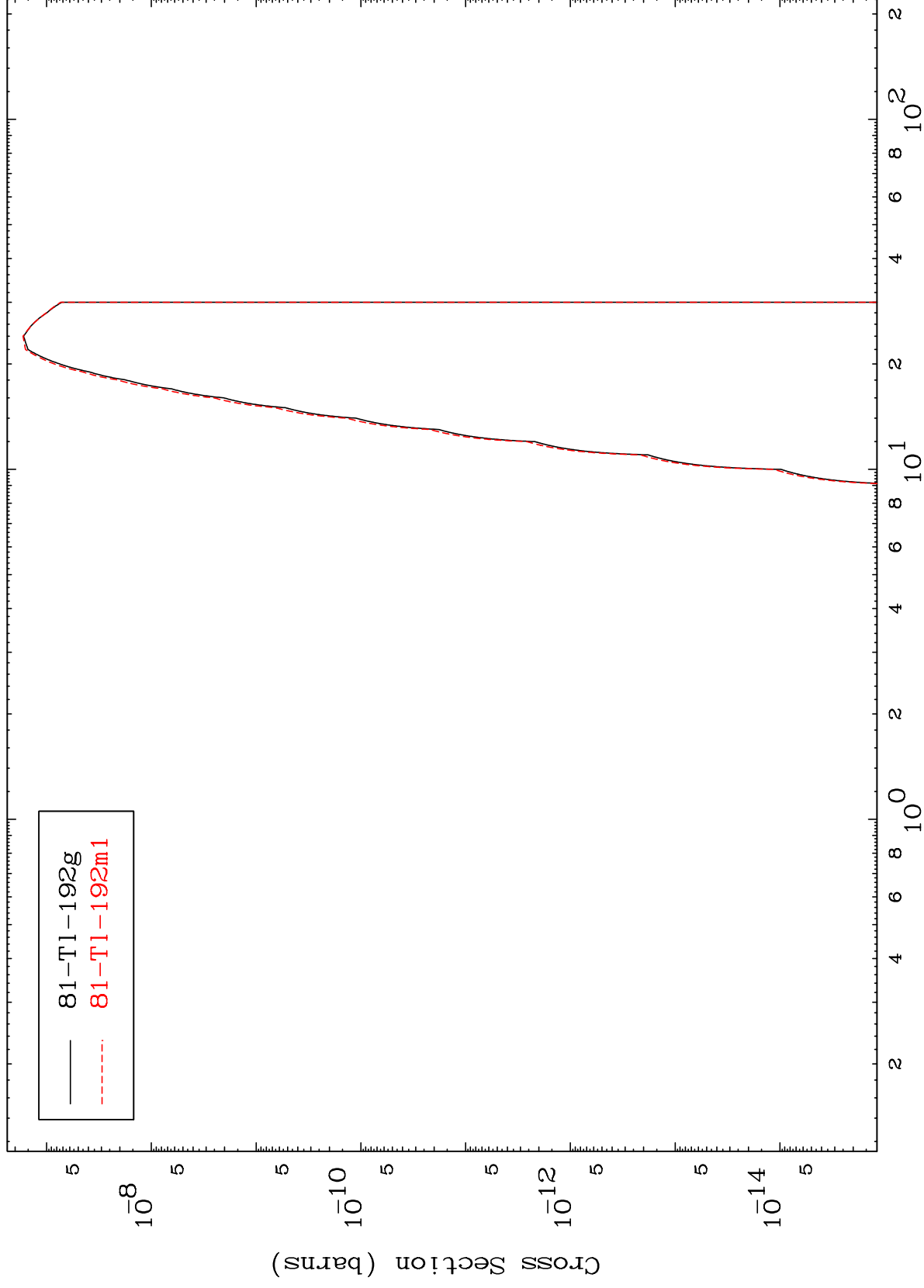
(n,2n) p
Radionuclide Production Cross Section



79-Au-189

Incident Energy (MeV)

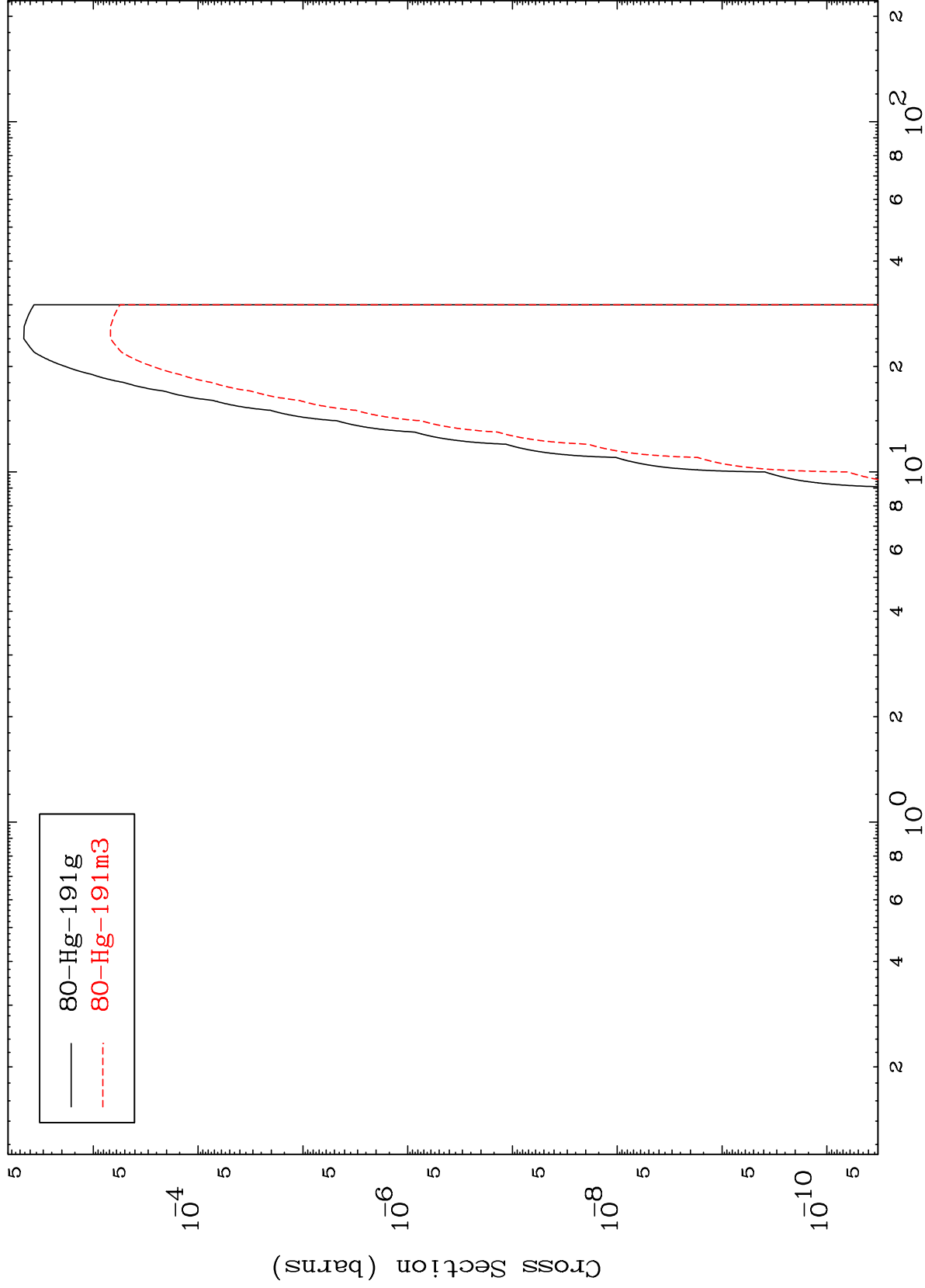
20

Radionuclide Production Cross Section (n, γ) 

MAT 7901

79-Au-189

Radionuclide Production Cross Section



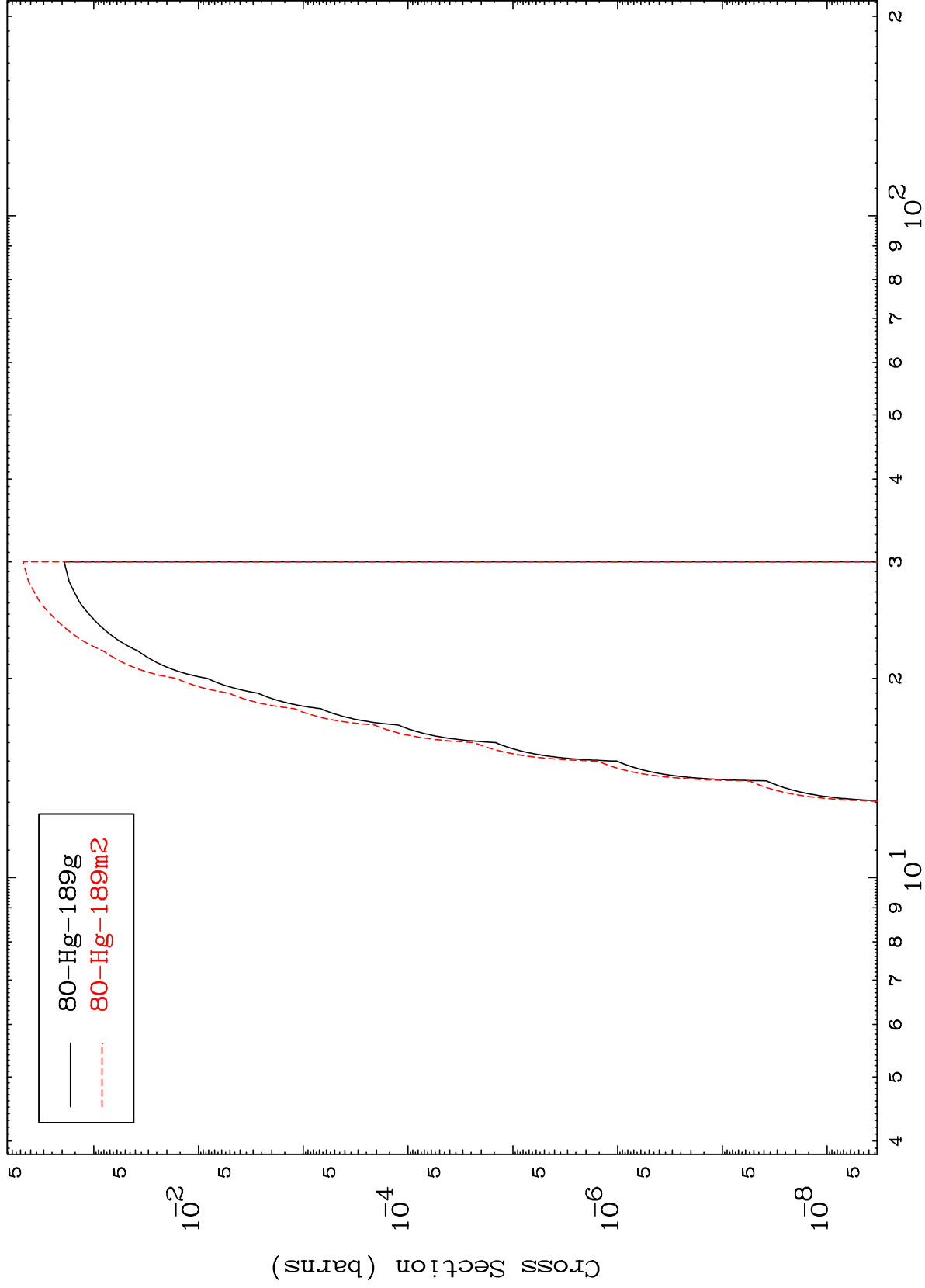
22

79-Au-189

MAT 7901

79-Au-189

(n,t)
Radionuclide Production Cross Section



23

Incident Energy (MeV)

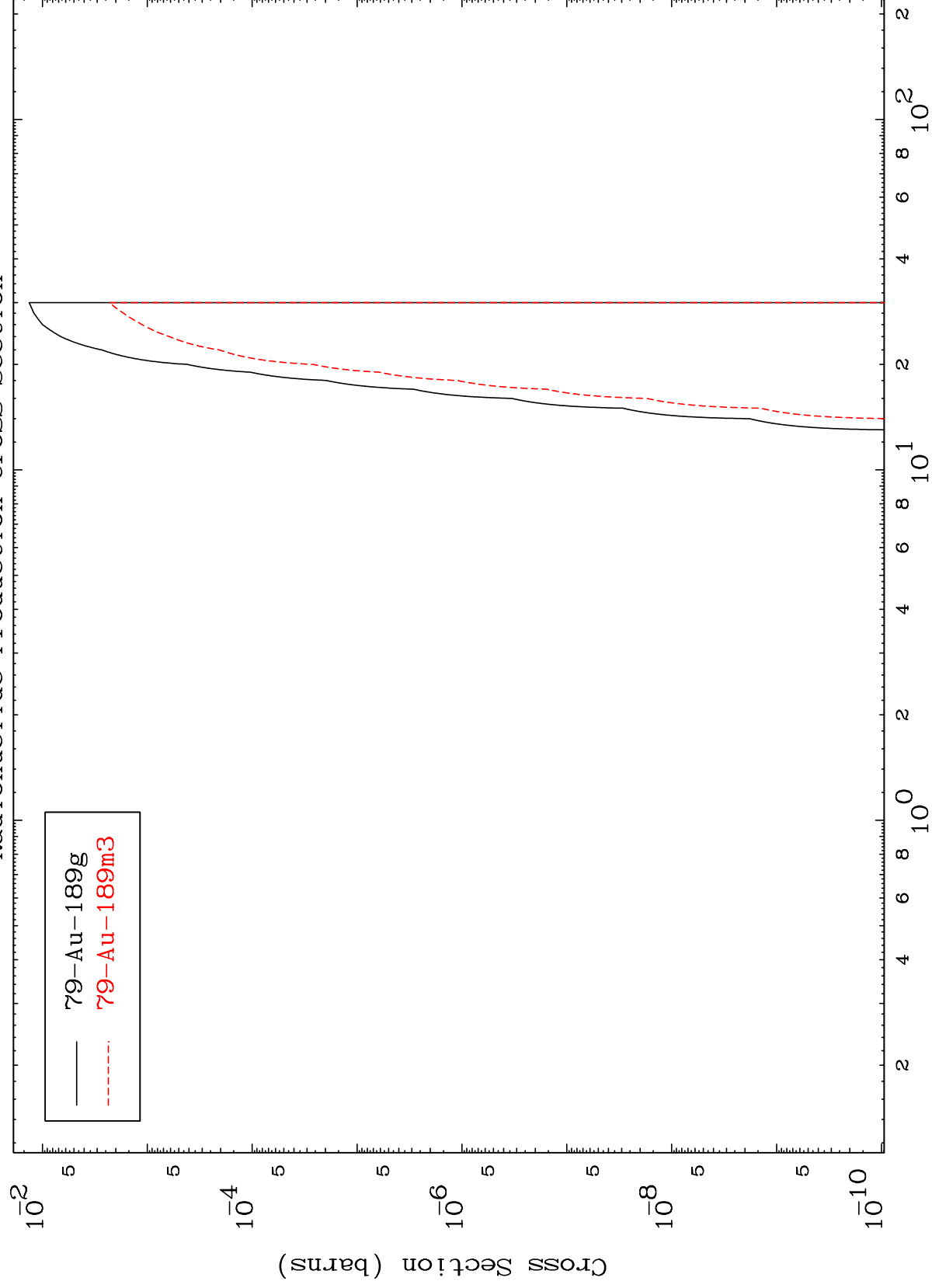
79-Au-189

MAT 7901

(n,He-3)

79-Au-189

Radionuclide Production Cross Section



24

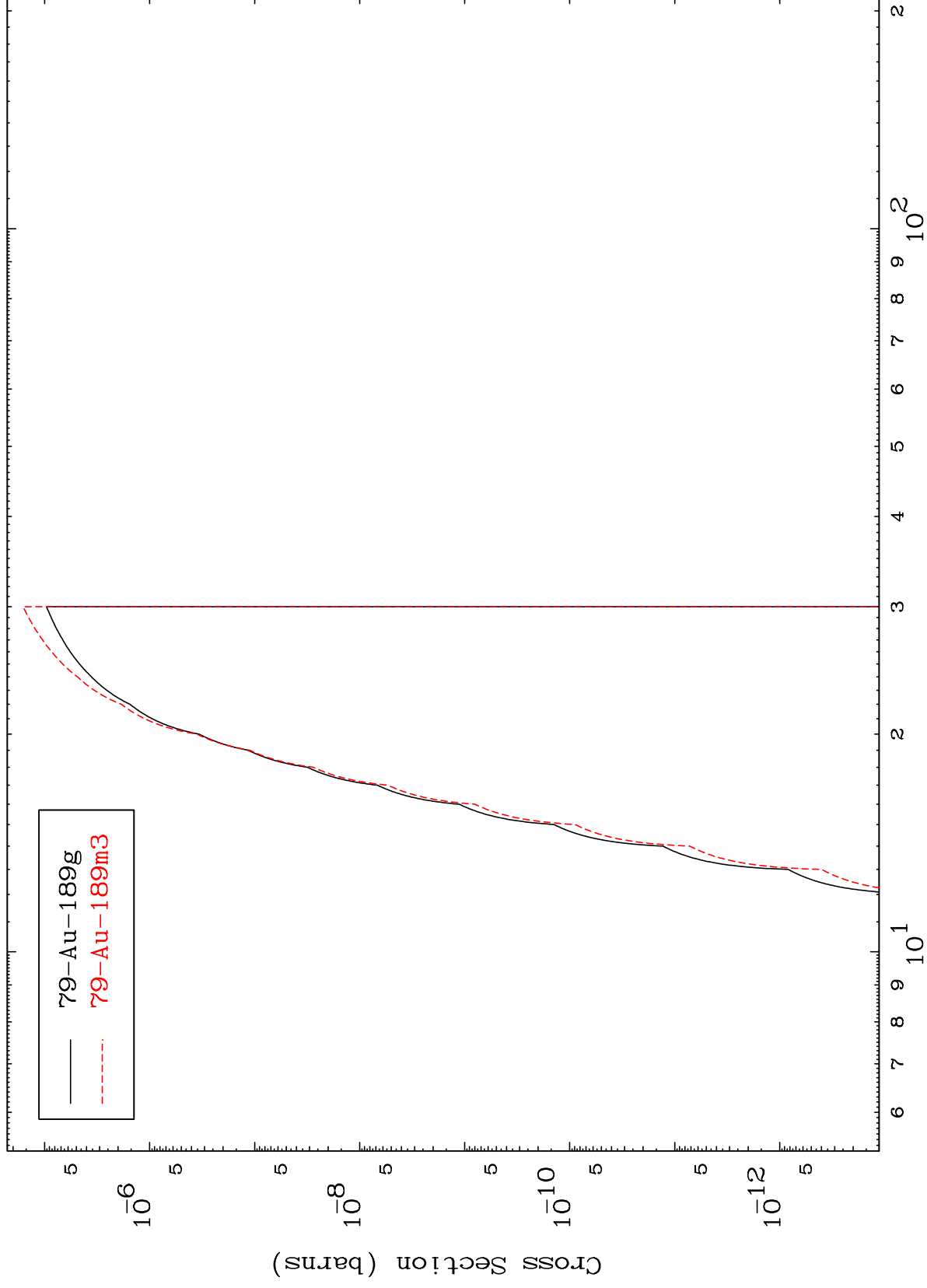
Incident Energy (MeV)

79-Au-189

MAT 7901

79-Au-189

(n,p) d
Radionuclide Production Cross Section



25

Incident Energy (MeV)

79-Au-189