

Program EVALPLOT
(Version 2021-1)

by

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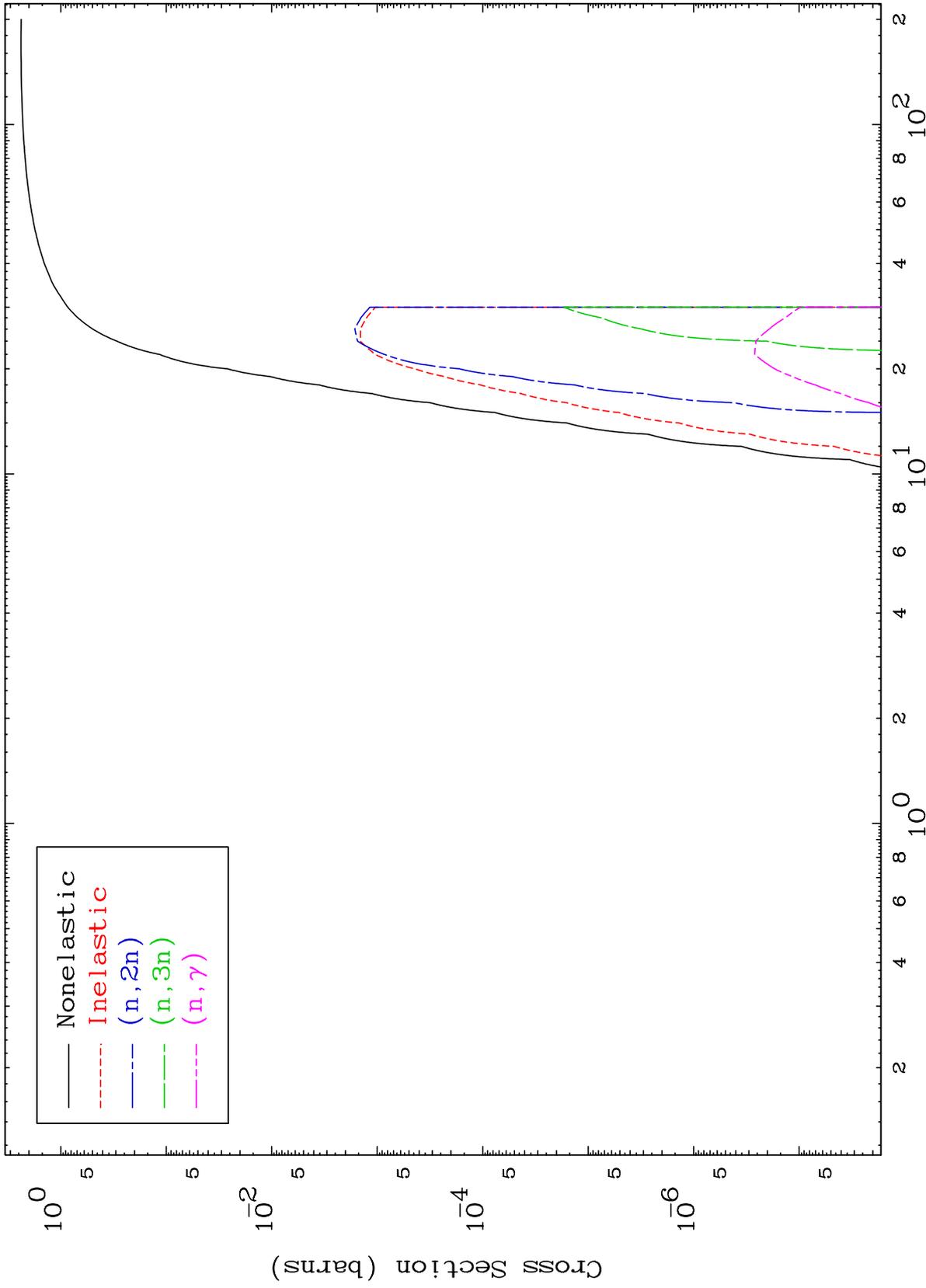
Press Mouse Button to Start

MAT 8290

He-3 Major

83-Bi-197m

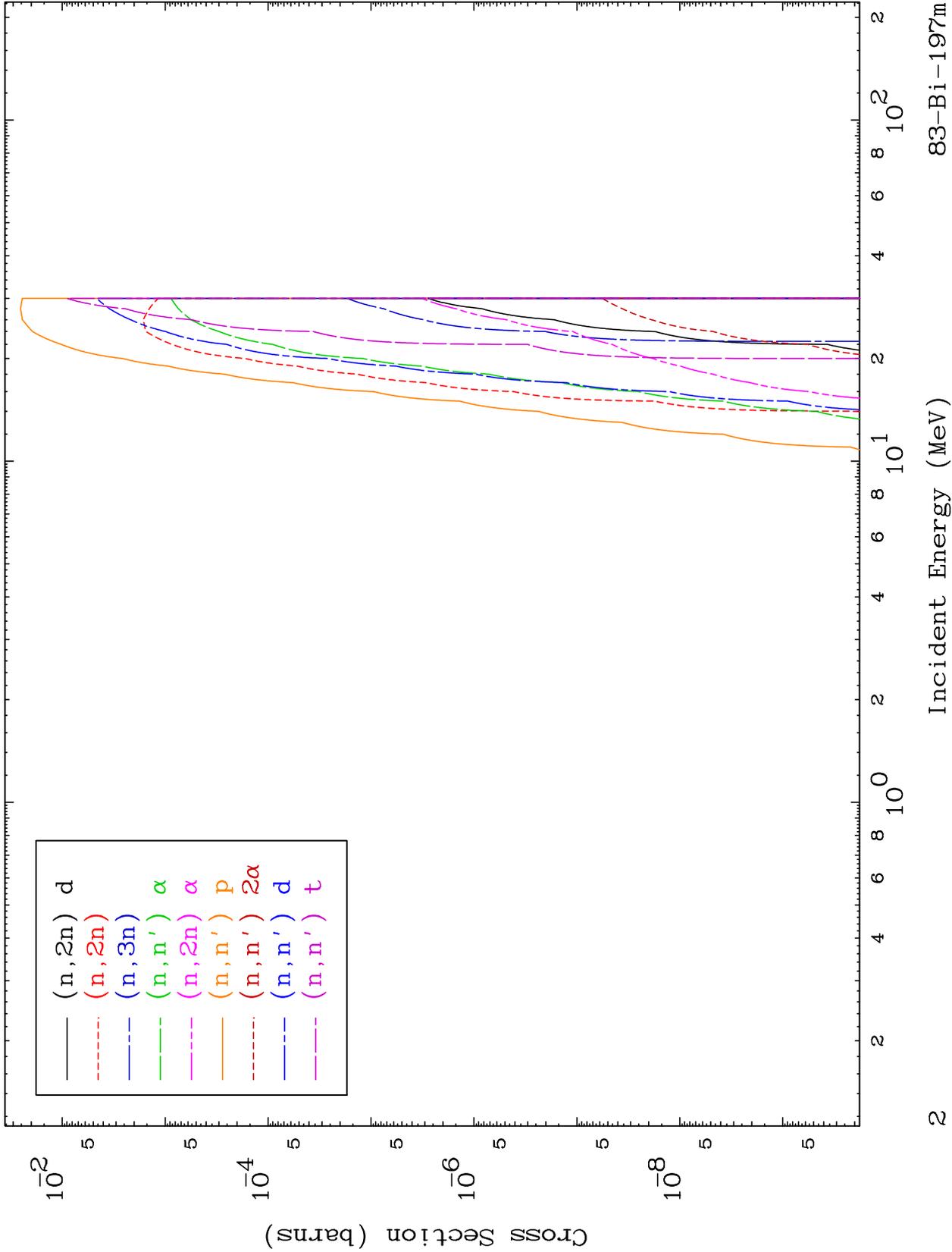
0 Kelvin Cross Sections



MAT 8290

He-3 Neutron Absorption
0 Kelvin Cross Sections

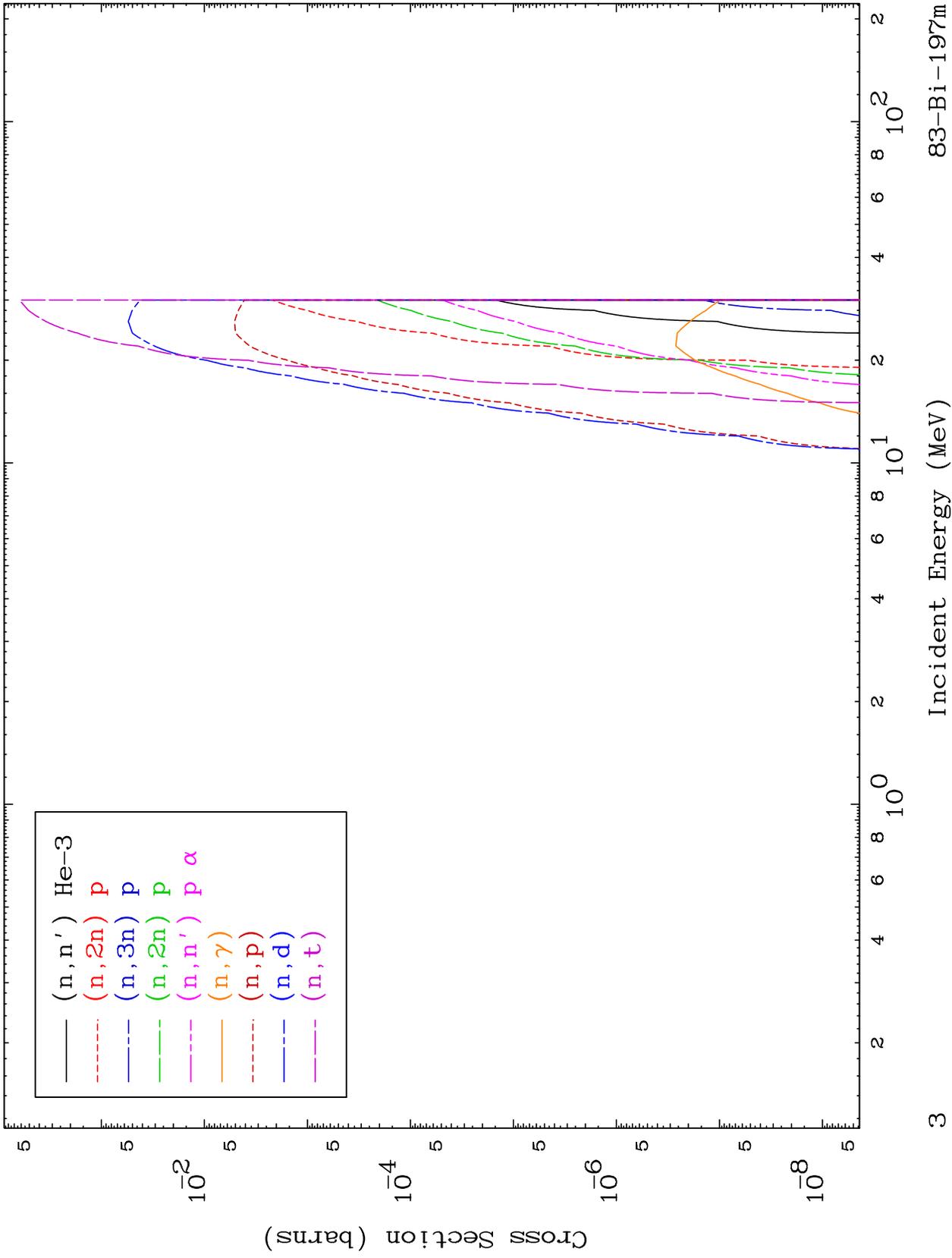
83-Bi-197m



MAT 8290

He-3 Neutron Absorption
0 Kelvin Cross Sections

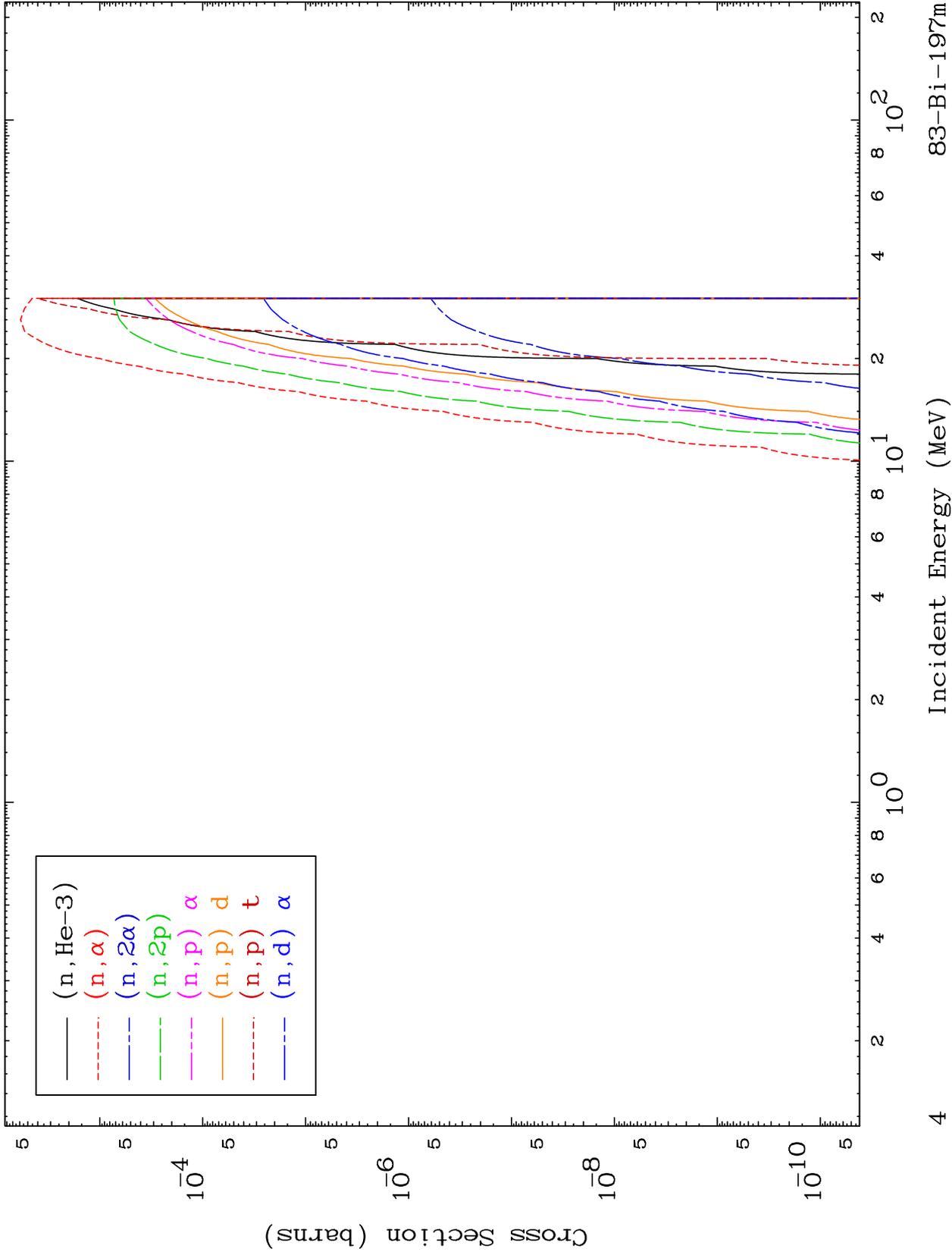
83-Bi-197m



MAT 8290

He-3 Neutron Absorption
0 Kelvin Cross Sections

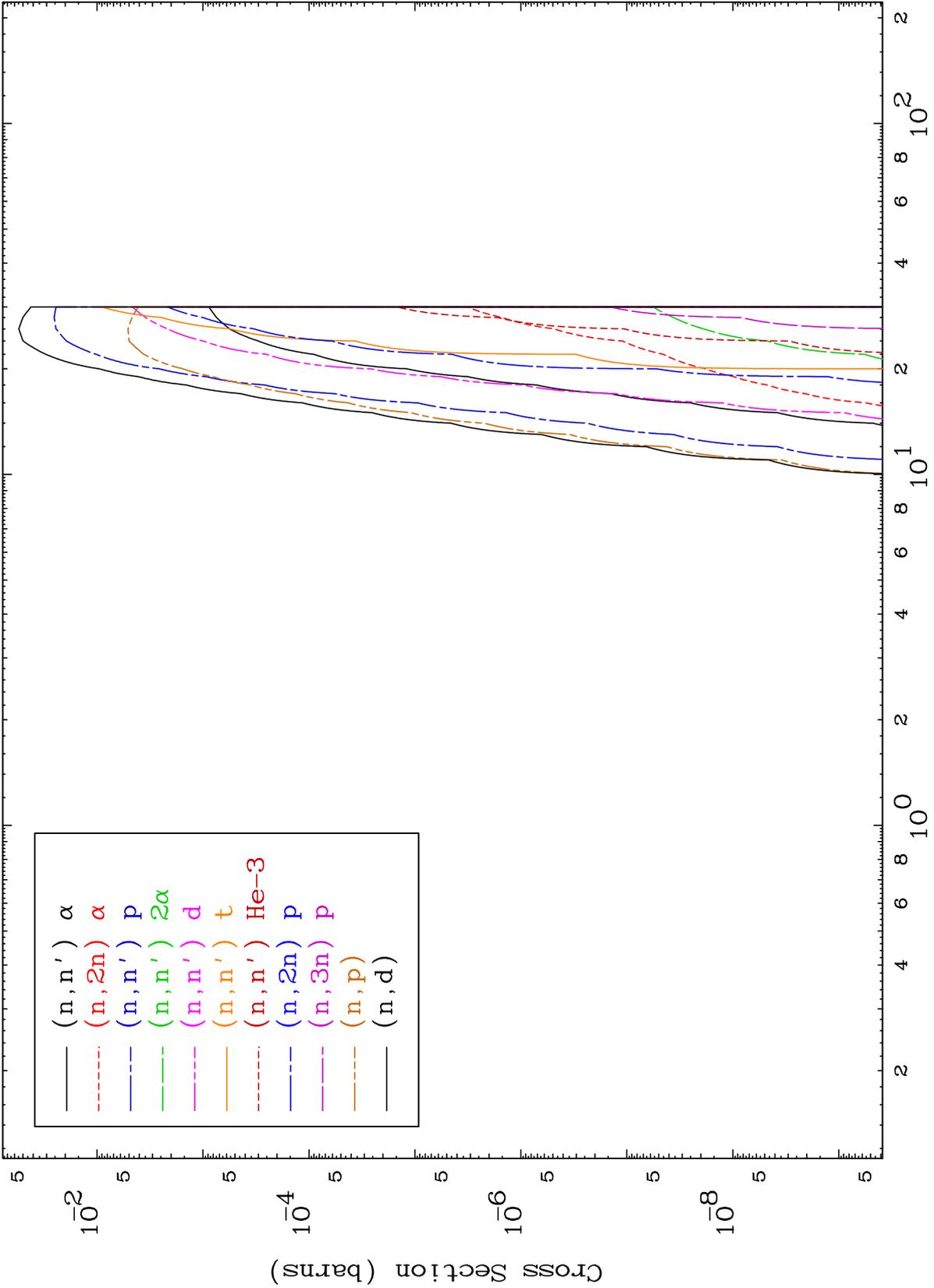
83-Bi-197m



MAT 8290

He-3 Charged Particle
0 Kelvin Cross Sections

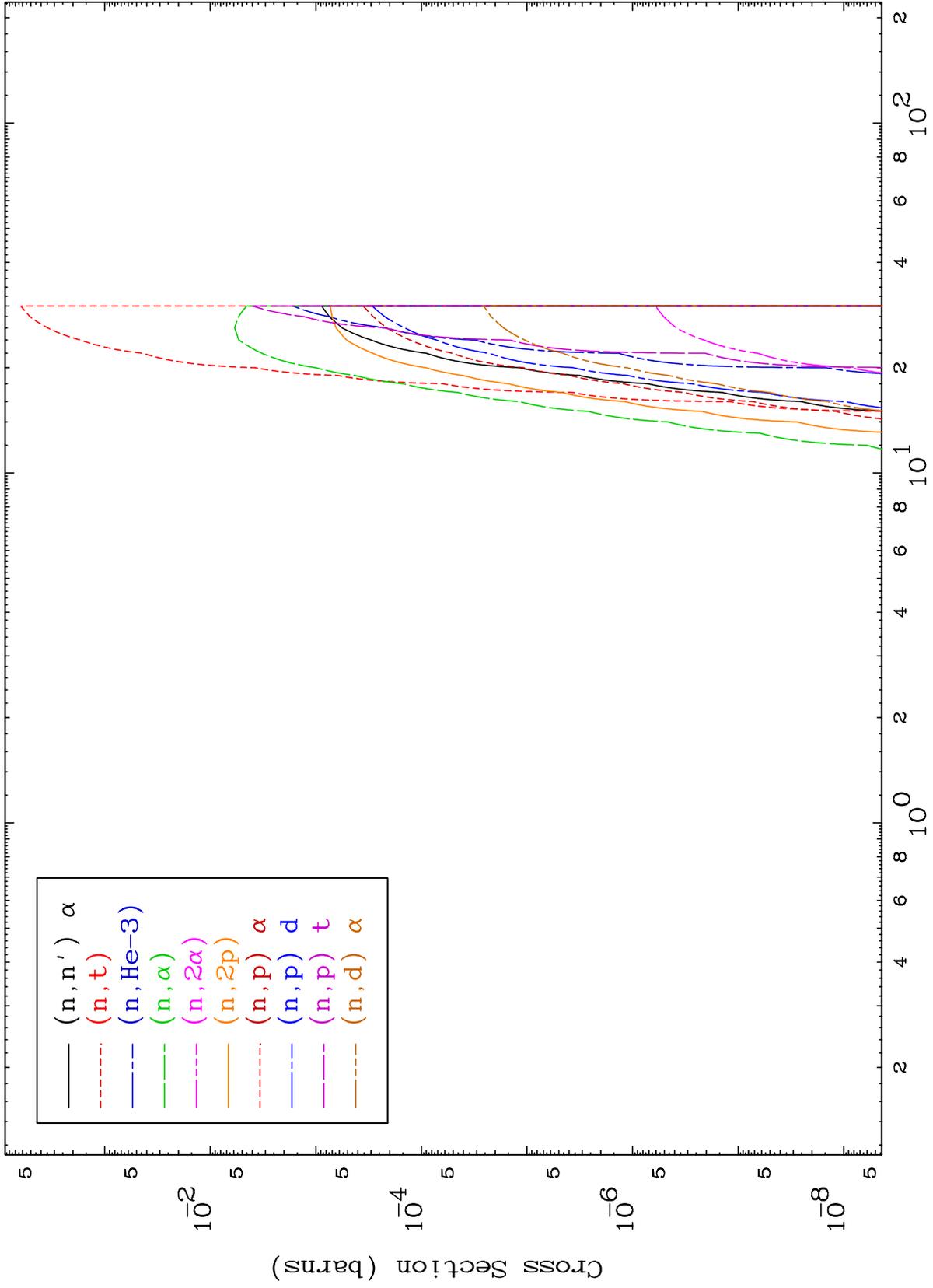
83-Bi-197m



MAT 8290

He-3 Charged Particle
0 Kelvin Cross Sections

83-Bi-197m

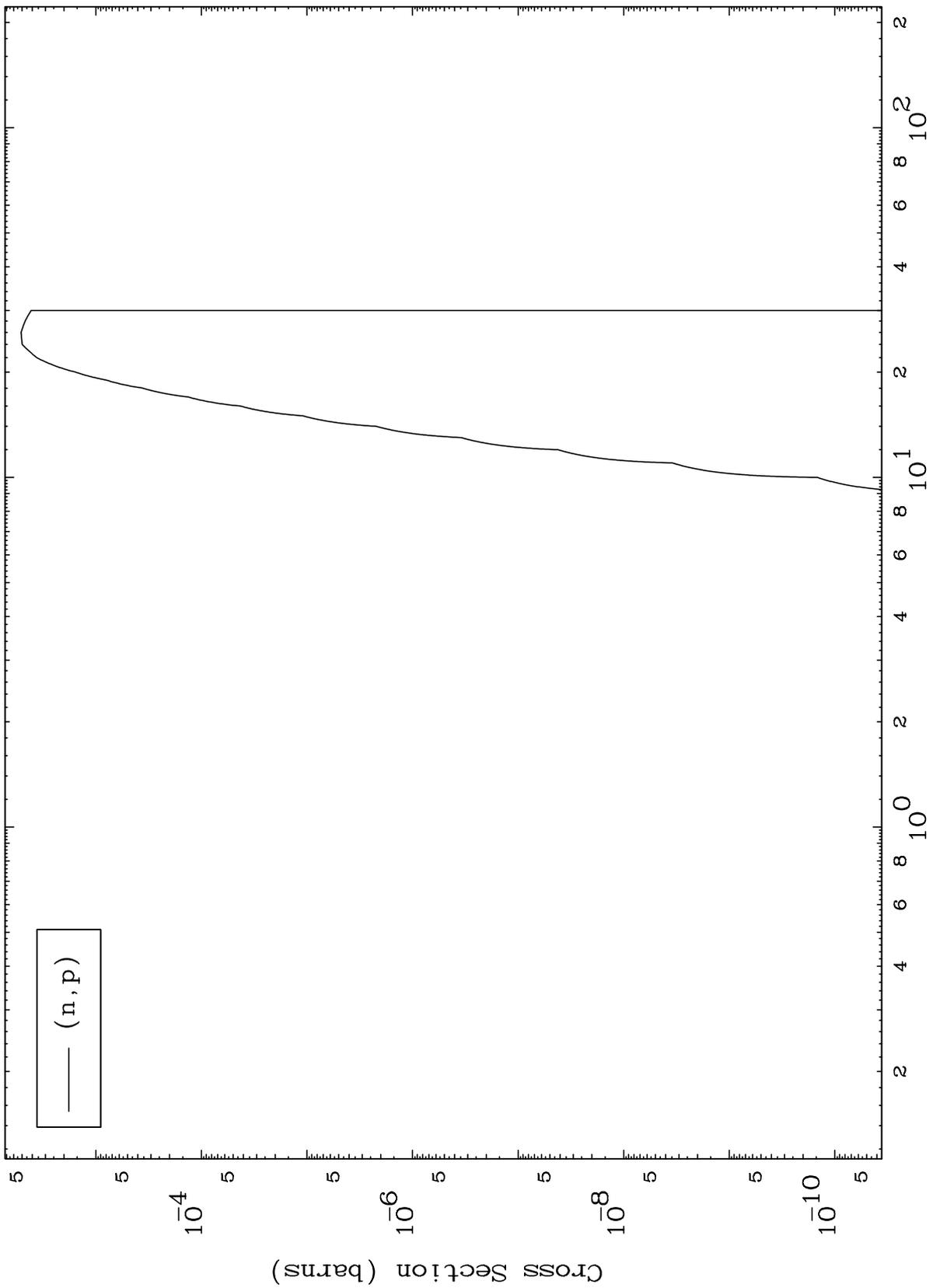


MAT 8290

(He-3,p) Levels

83-Bi-197m

0 Kelvin Cross Sections

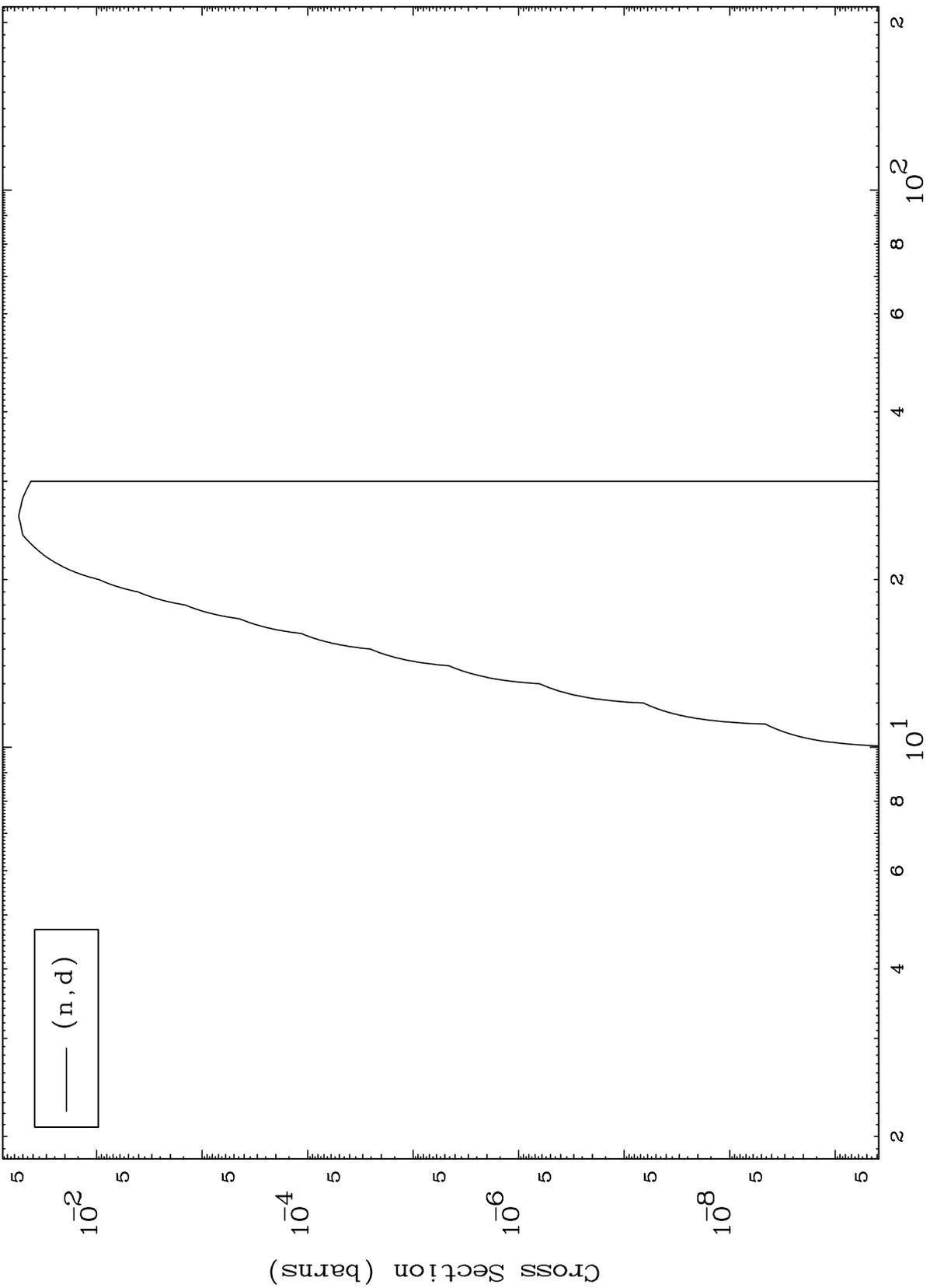


MAT 8290

(He-3,d) Levels

83-Bi-197m

0 Kelvin Cross Sections



8

Incident Energy (MeV)

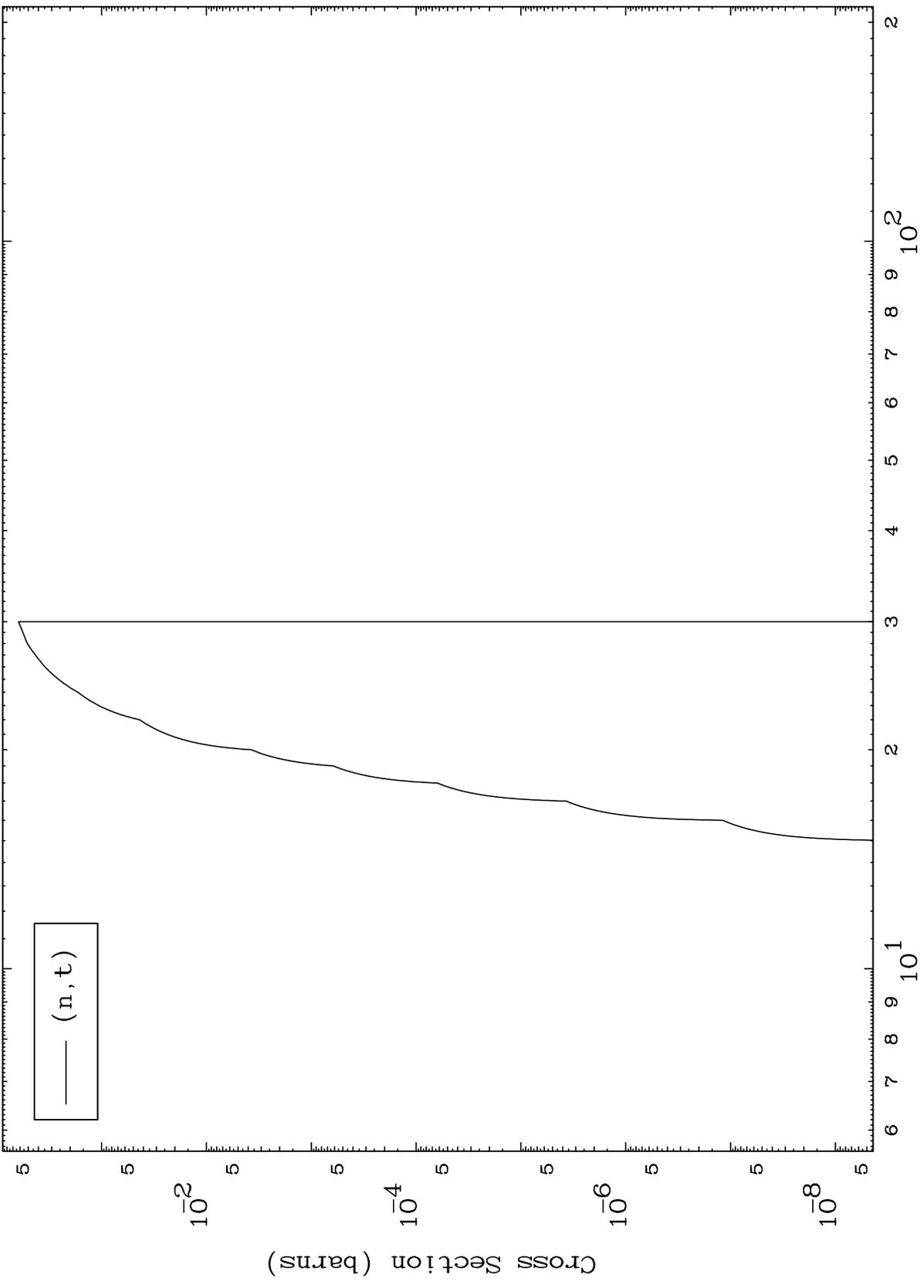
83-Bi-197m

MAT 8290

(He-3, t) Levels

83-Bi-197m

0 Kelvin Cross Sections



9

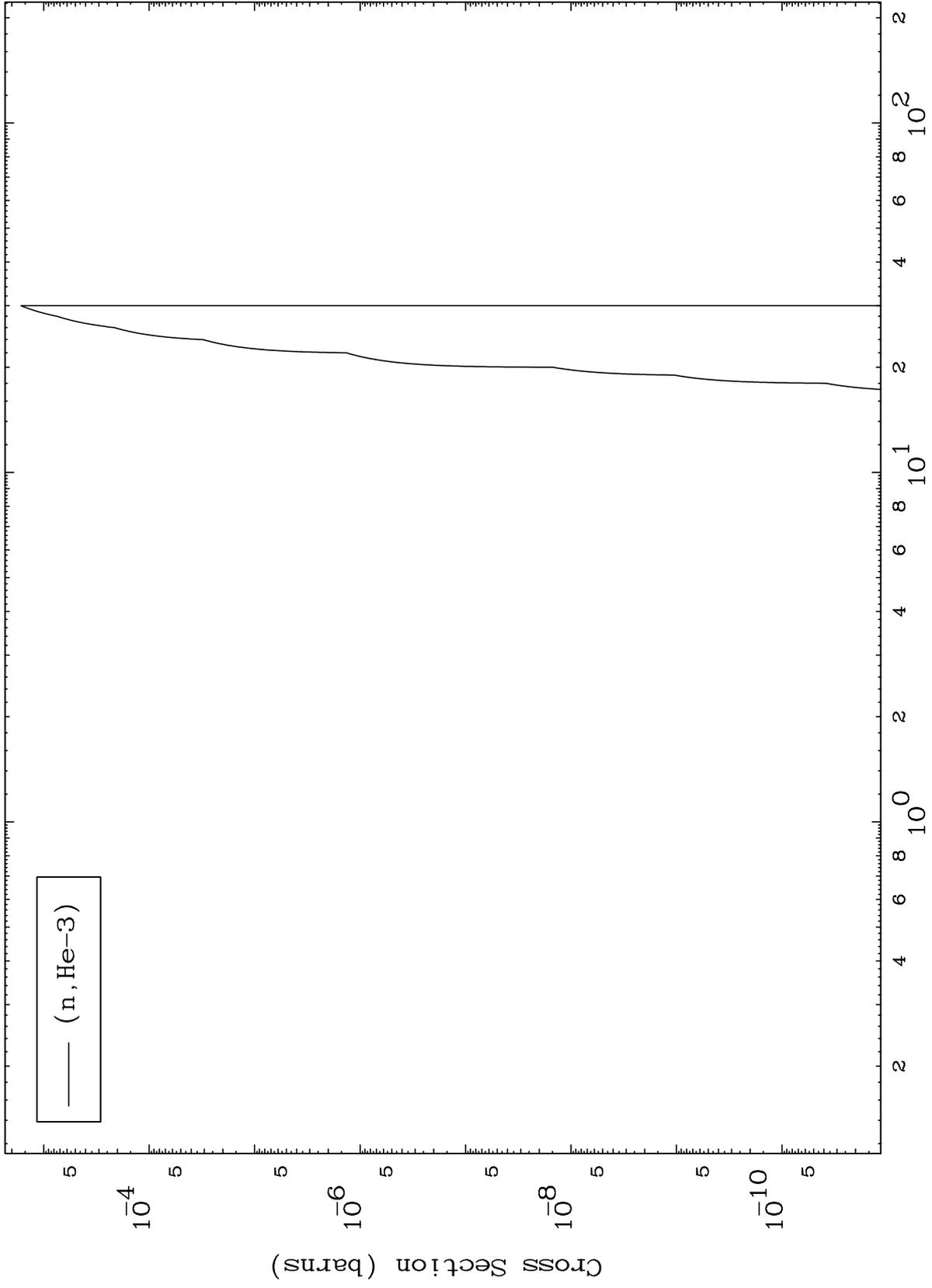
Incident Energy (MeV)

83-Bi-197m

MAT 8290

(He-3, He3) Levels
0 Kelvin Cross Sections

83-Bi-197m



10

Incident Energy (MeV)

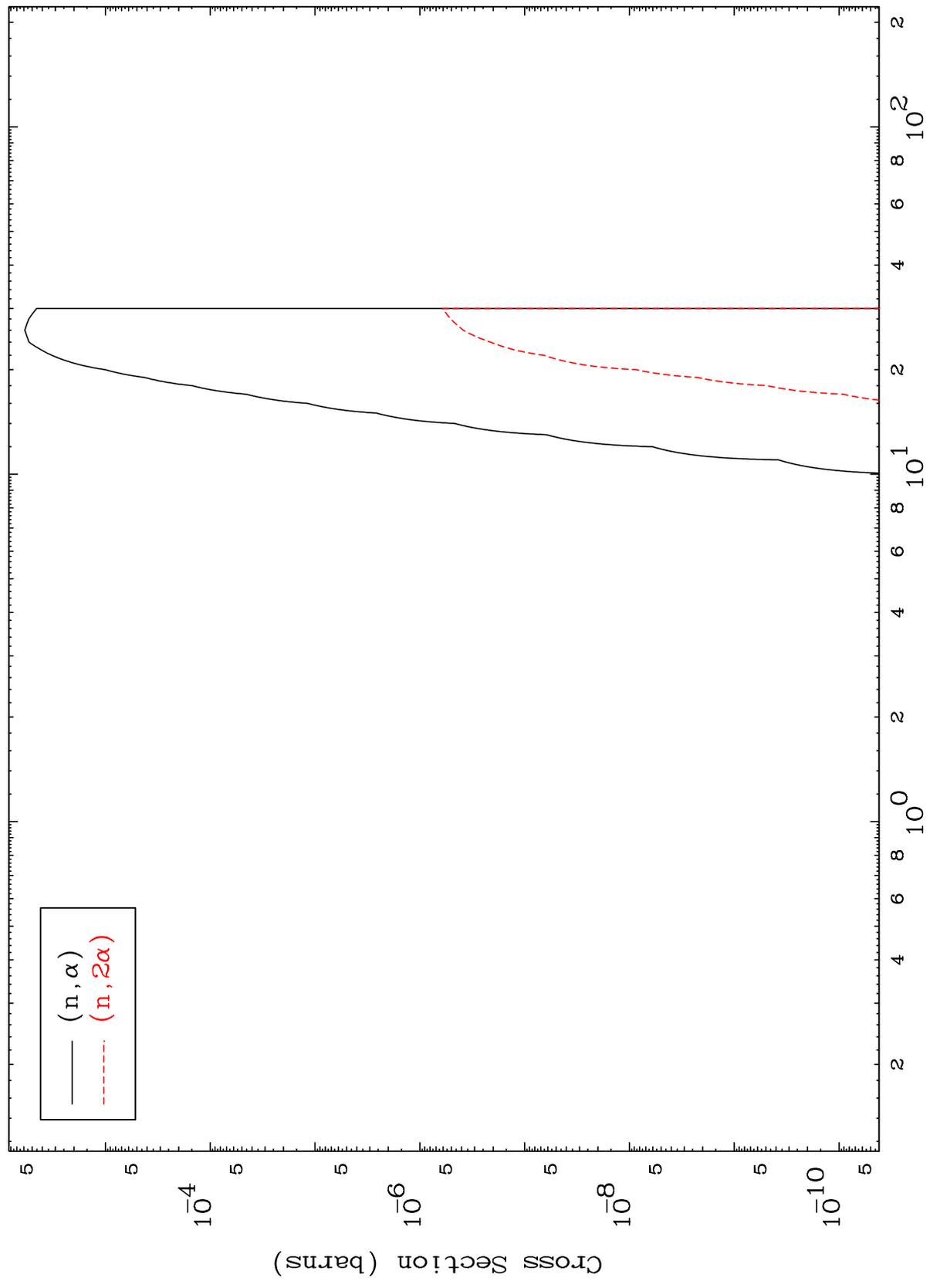
83-Bi-197m

MAT 8290

(He-3, α) Levels

83-Bi-197m

0 Kelvin Cross Sections

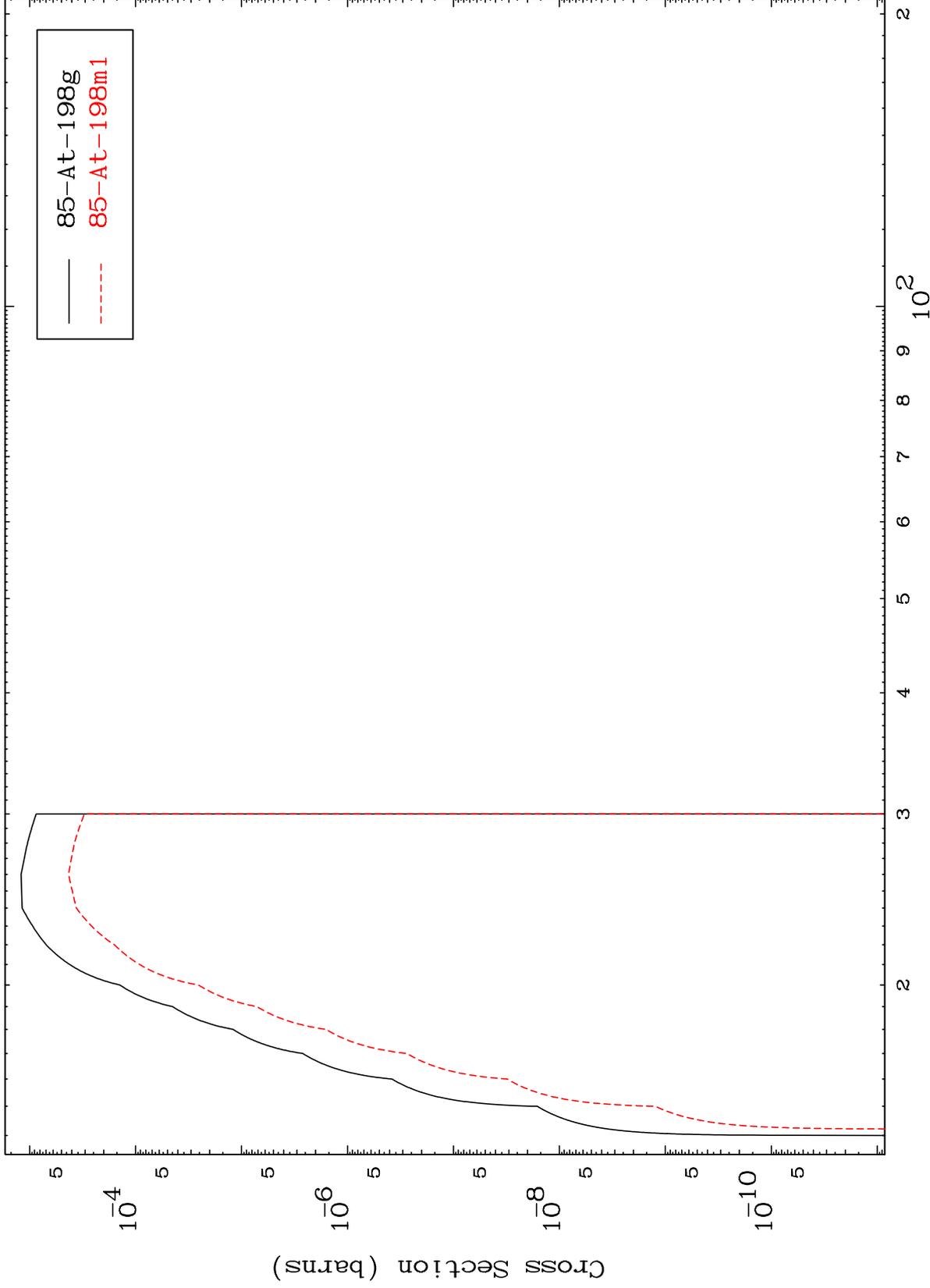


MAT 8290

(n,2n)

83-Bi-197m

Radionuclide Production Cross Section



12

Incident Energy (MeV)

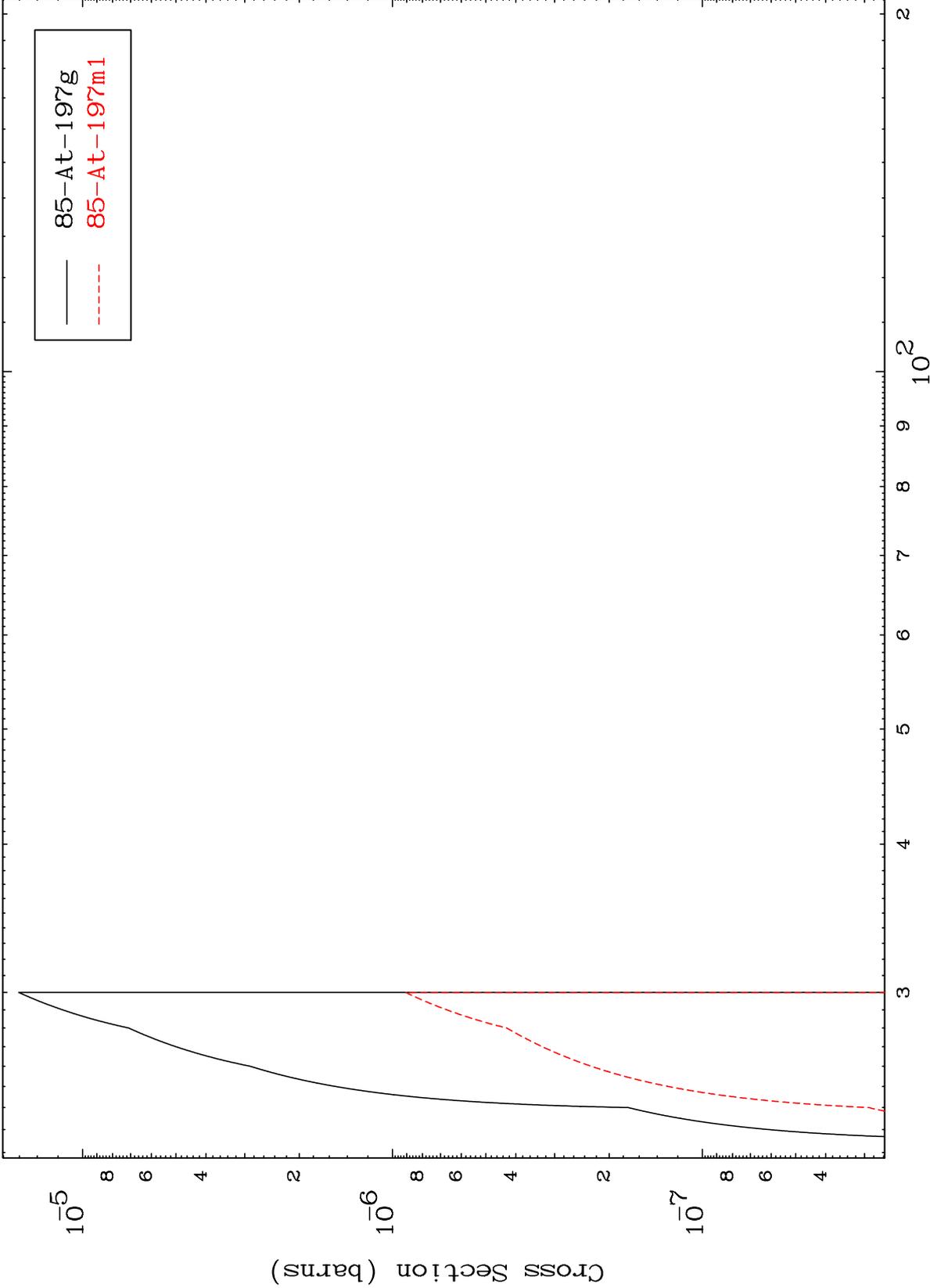
83-Bi-197m

MAT 8290

(n,3n)

83-Bi-197m

Radionuclide Production Cross Section



13

Incident Energy (MeV)

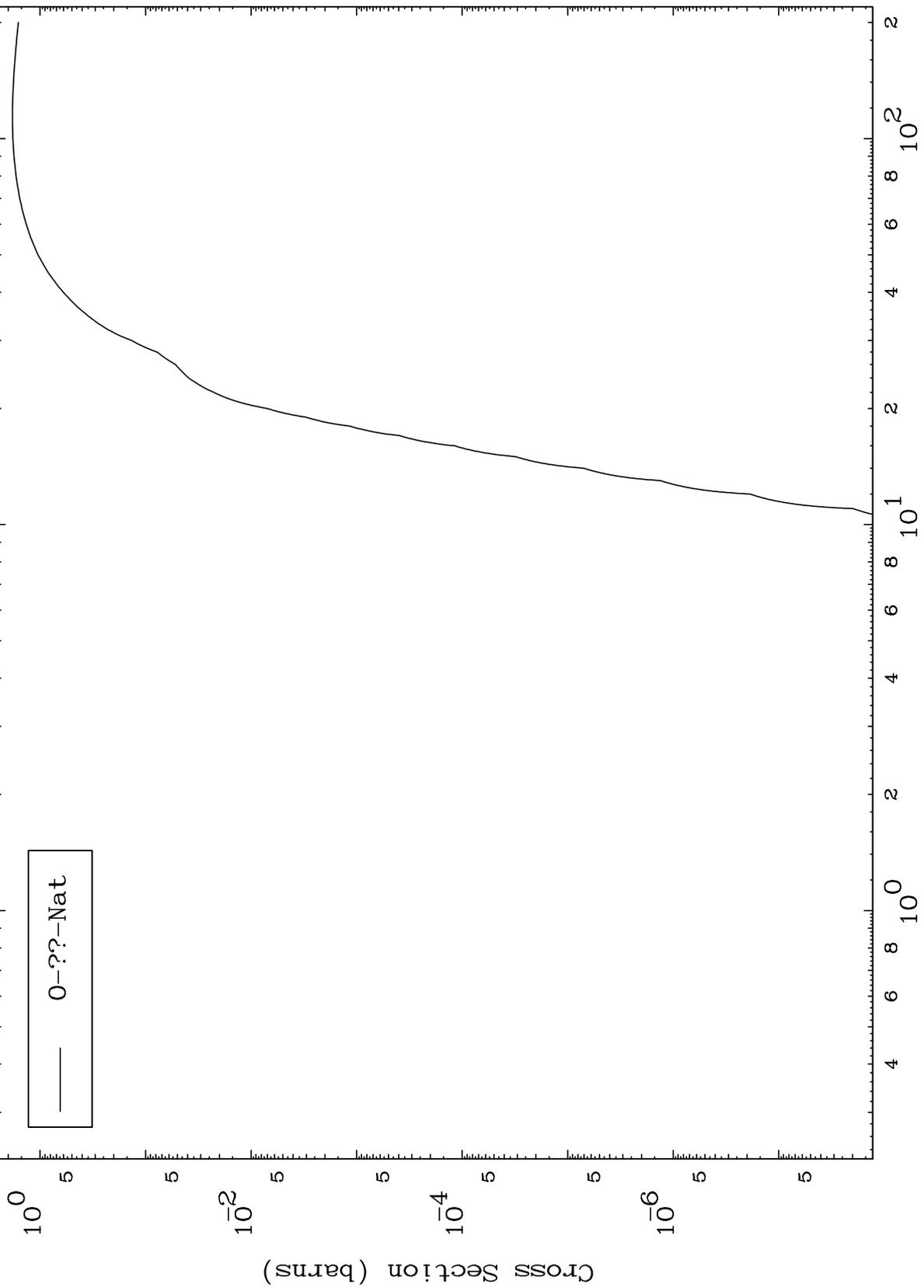
83-Bi-197m

MAT 8290

Fission

83-Bi-197m

Radionuclide Production Cross Section

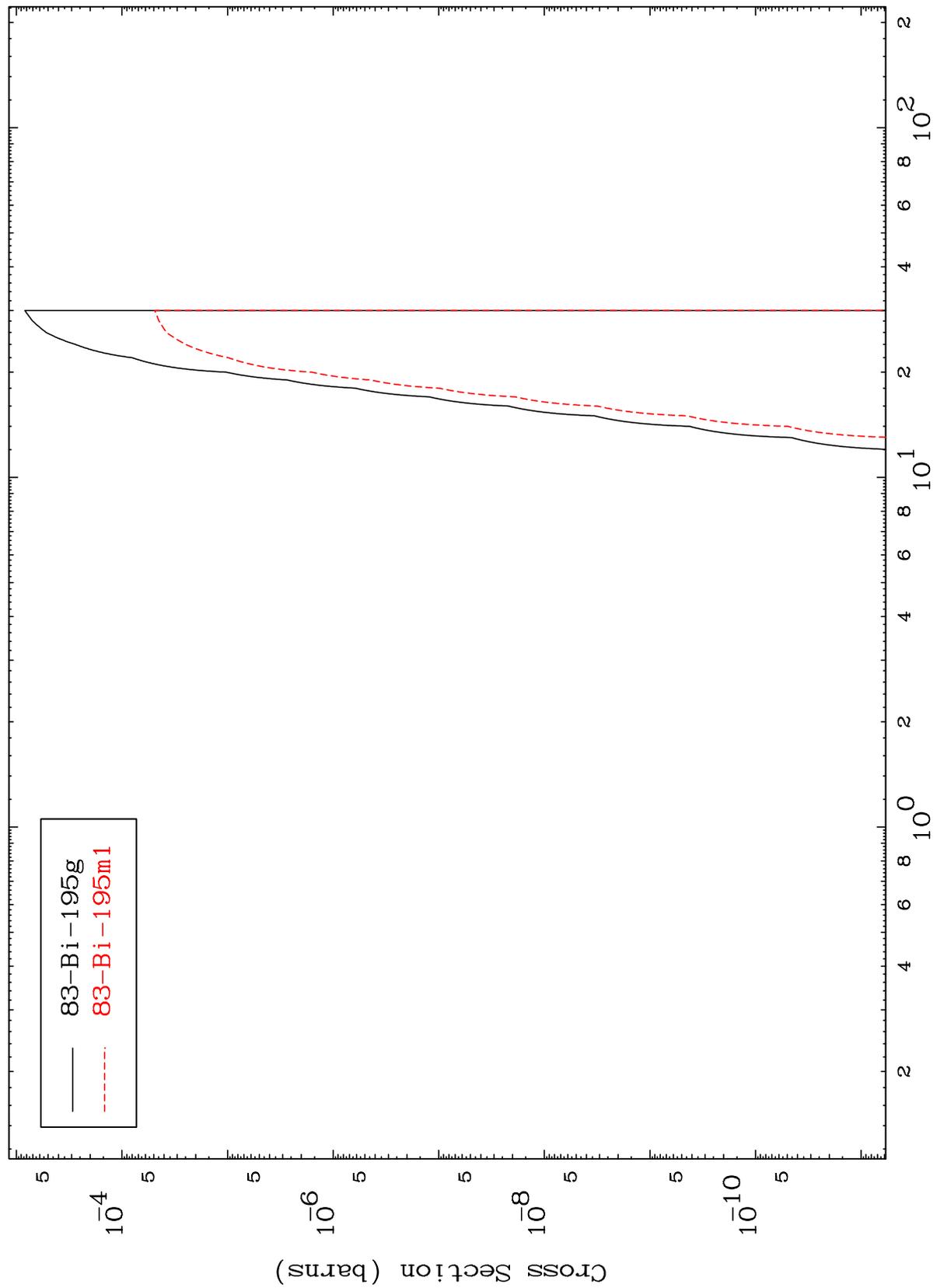


MAT 8290

$(n, n') \alpha$

$^{83}\text{Bi}-197\text{m}$

Radionuclide Production Cross Section

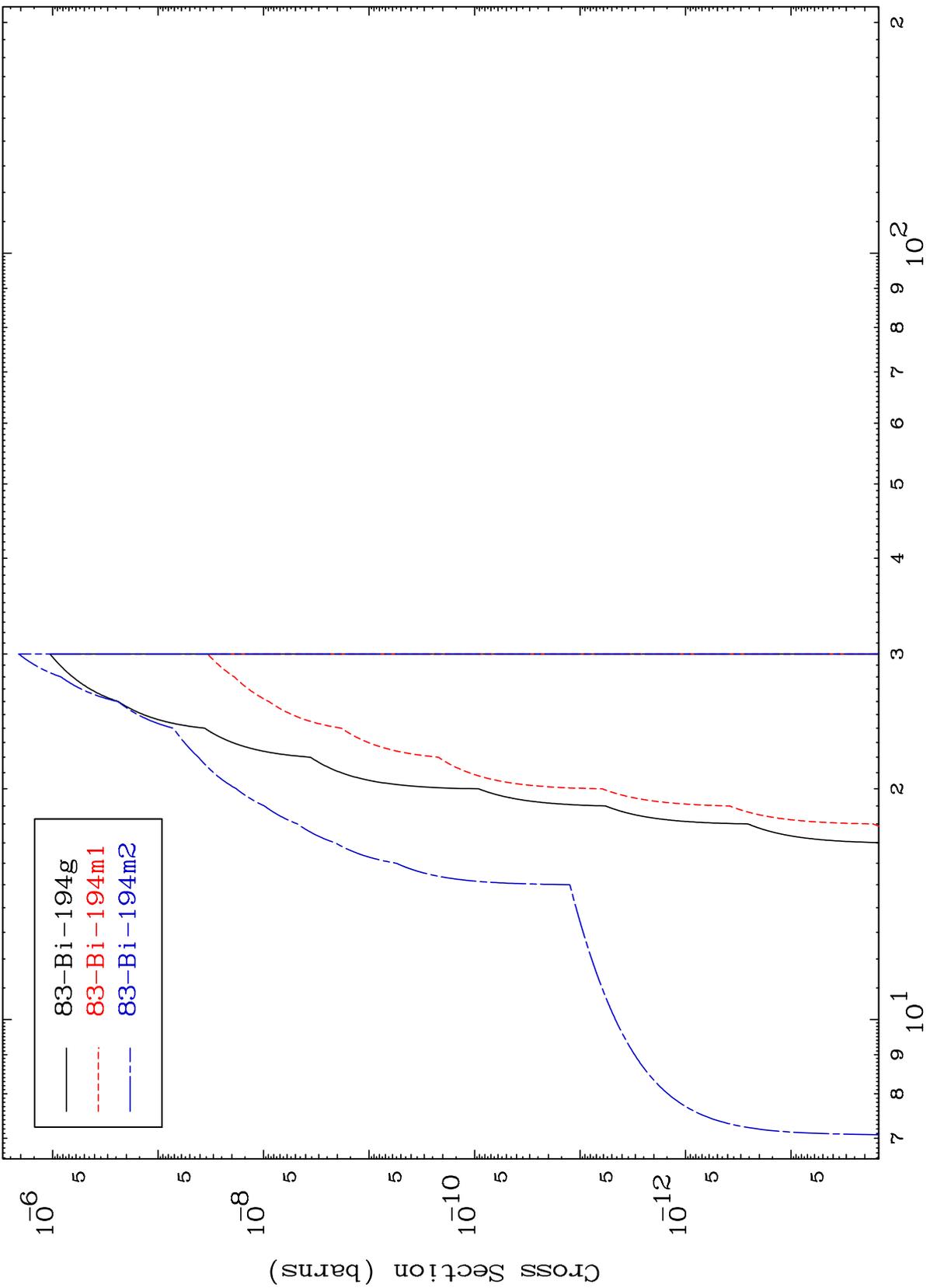


MAT 8290

$(n,2n) \alpha$

$^{83}\text{Bi}-197\text{m}$

Radionuclide Production Cross Section



16

Incident Energy (MeV)

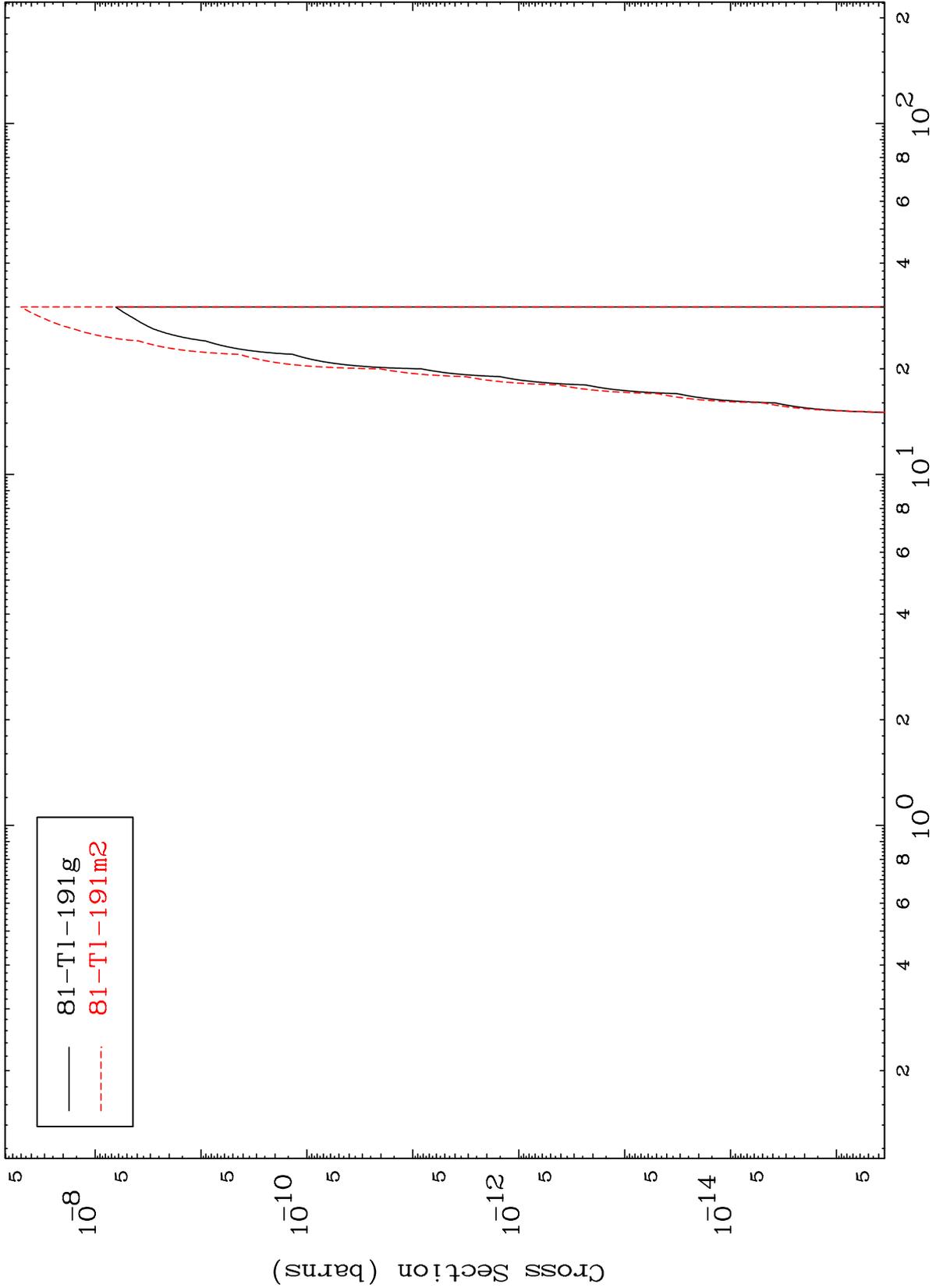
$^{83}\text{Bi}-197\text{m}$

MAT 8290

(n,n') 2 α

83-Bi-197m

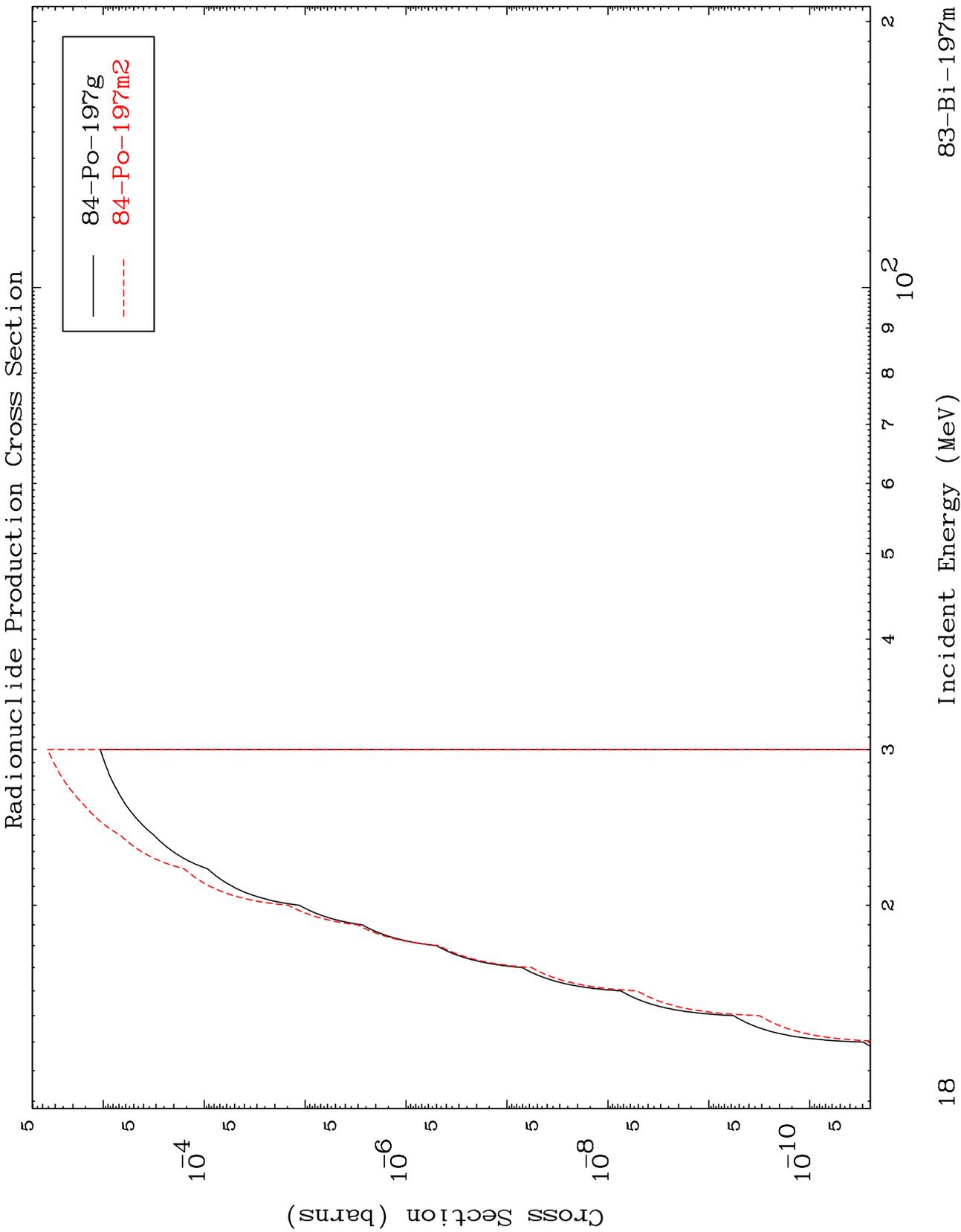
Radionuclide Production Cross Section



MAT 8290

(n,n') d

83-Bi-197m



18

Incident Energy (MeV)

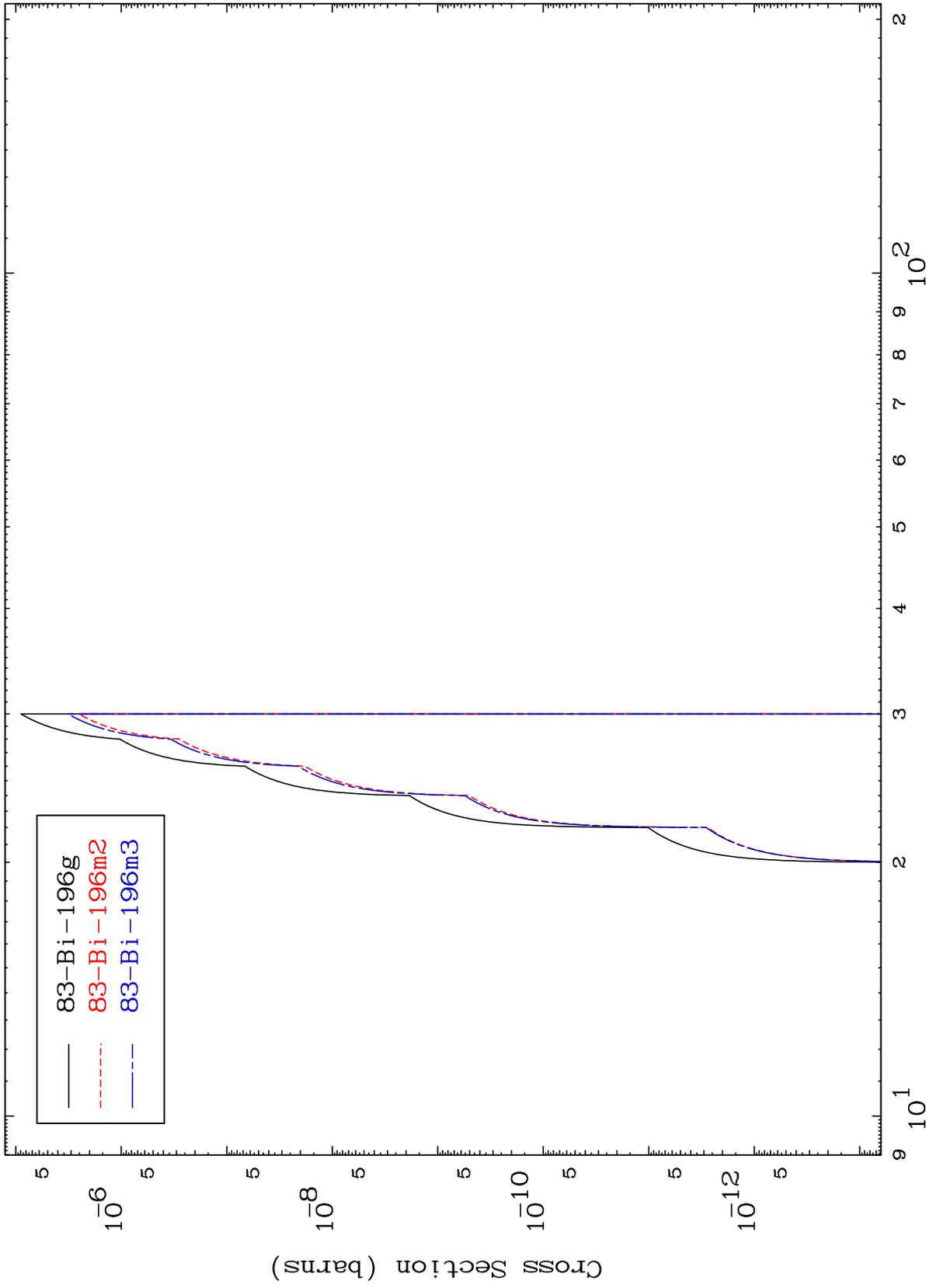
83-Bi-197m

MAT 8290

(n,n') He-3

83-Bi-197m

Radionuclide Production Cross Section



19

Incident Energy (MeV)

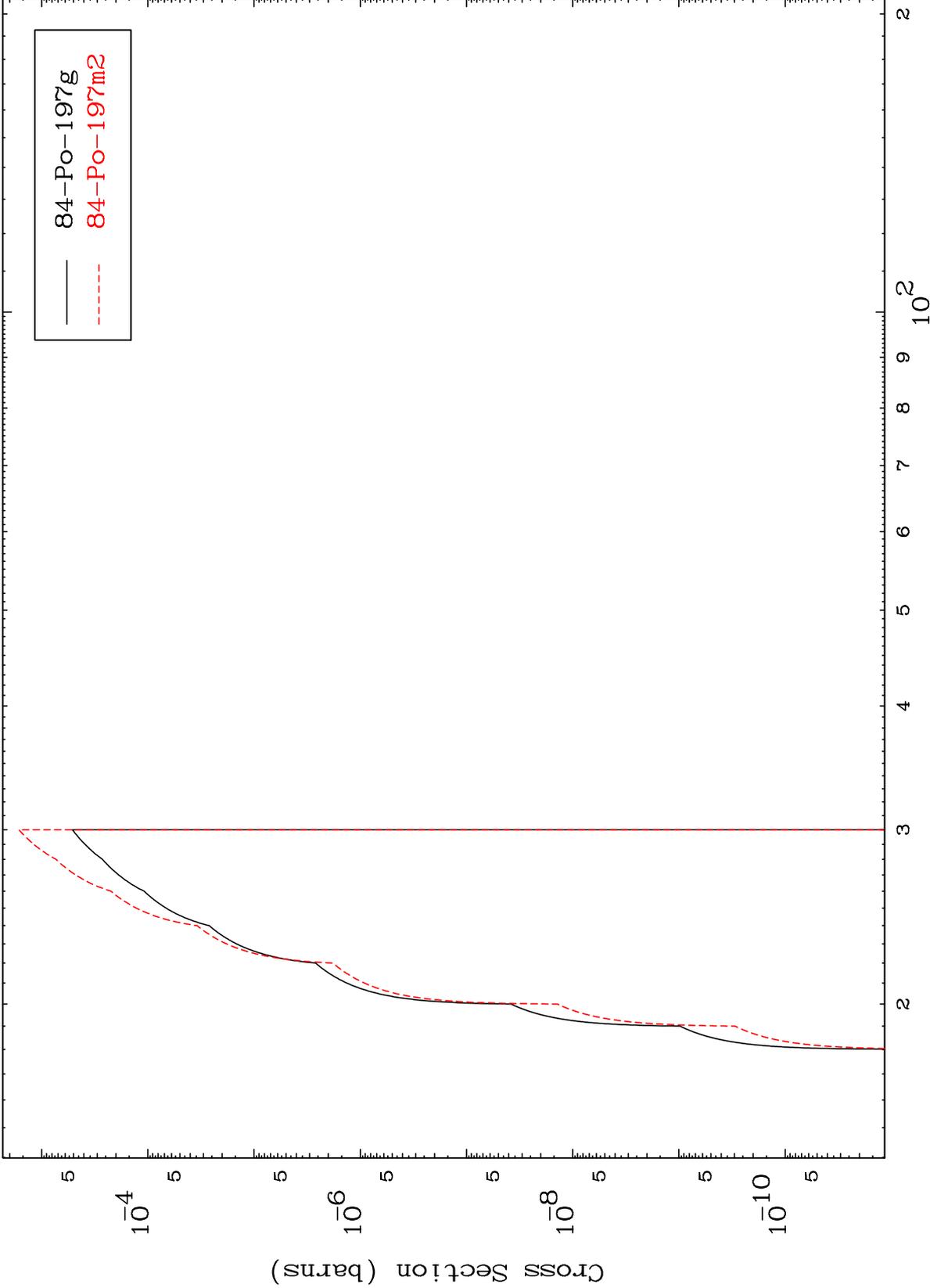
83-Bi-197m

MAT 8290

(n,2n) p

83-Bi-197m

Radionuclide Production Cross Section



20

Incident Energy (MeV)

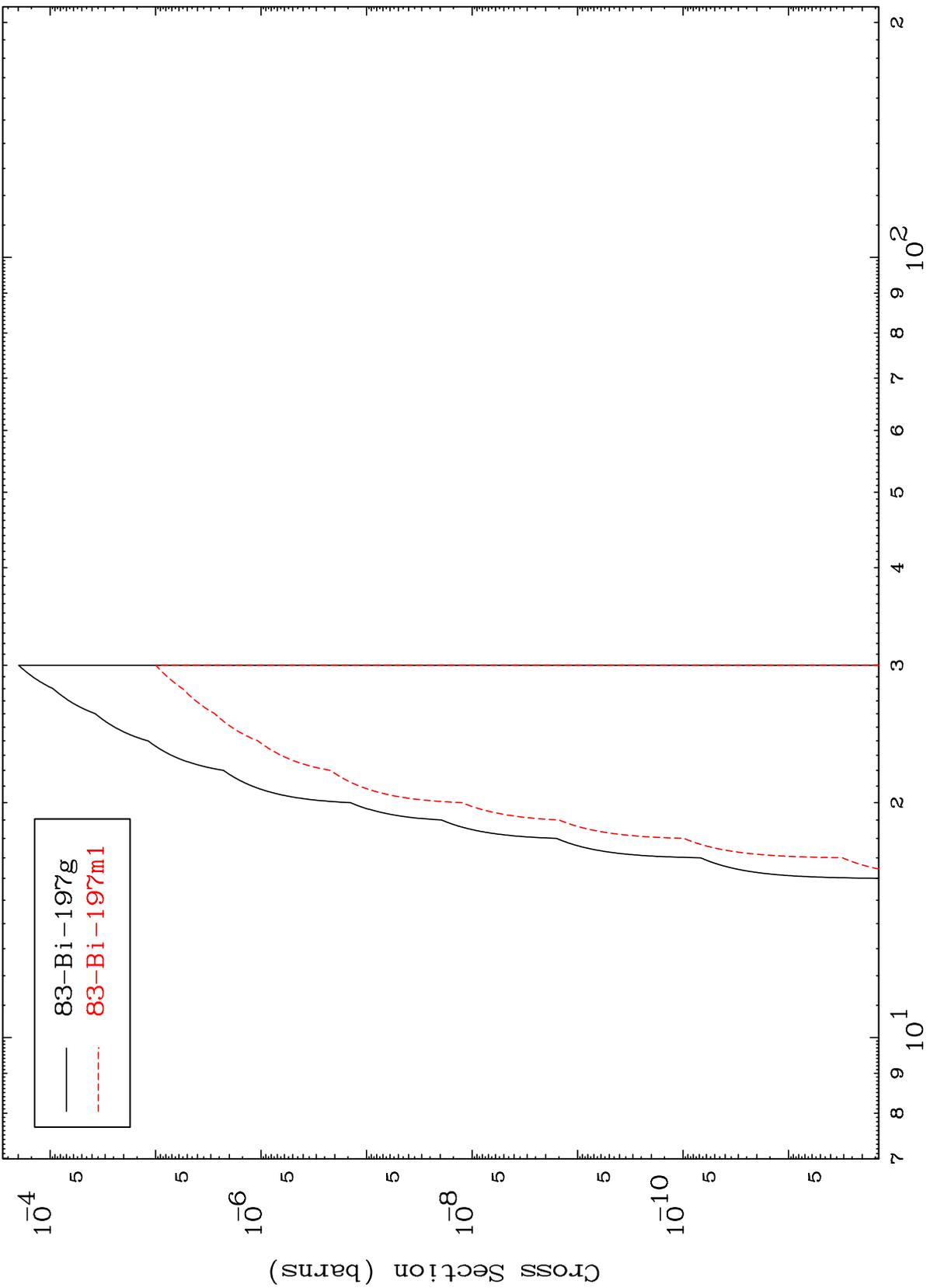
83-Bi-197m

MAT 8290

(n,2n) p

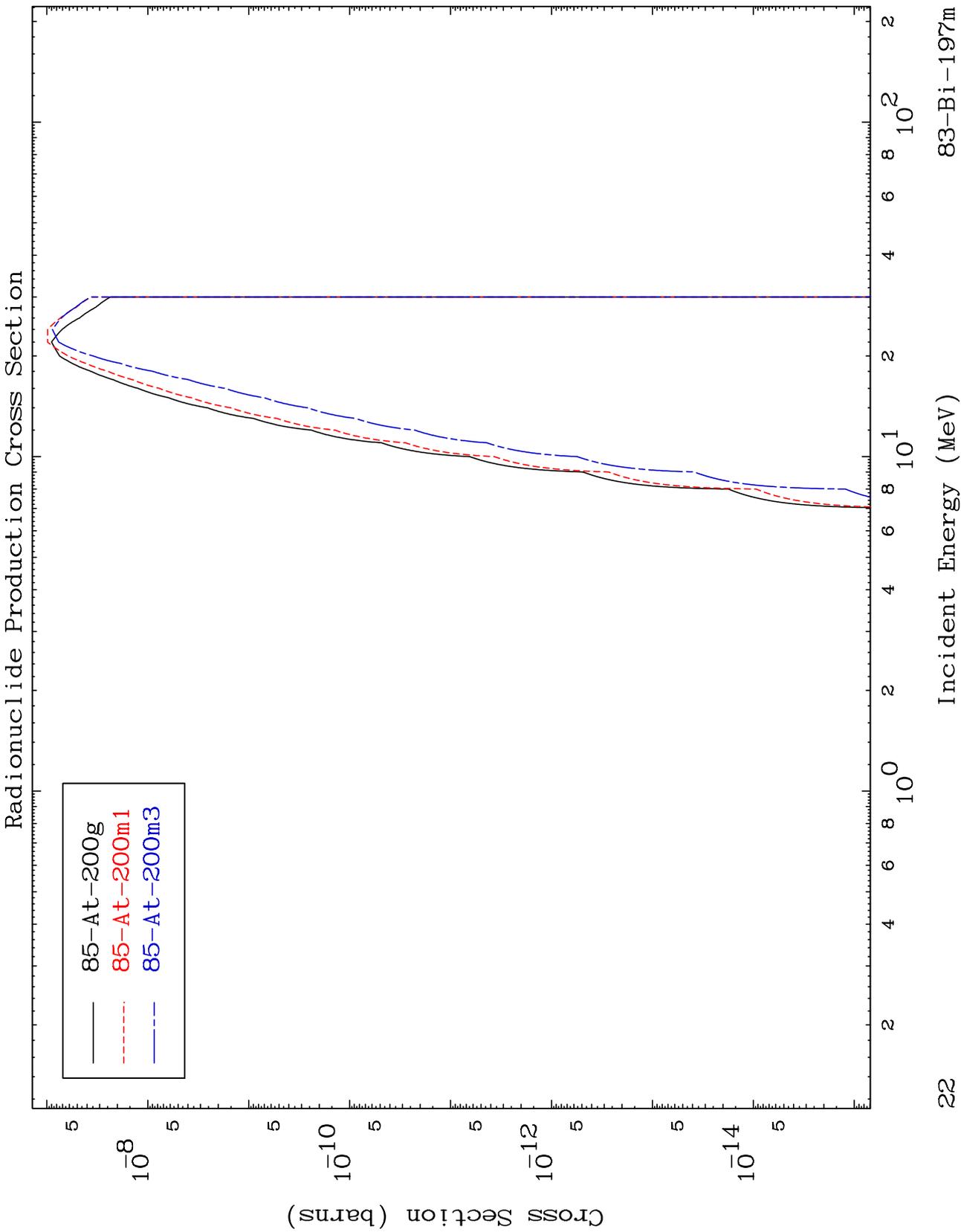
83-Bi-197m

Radionuclide Production Cross Section



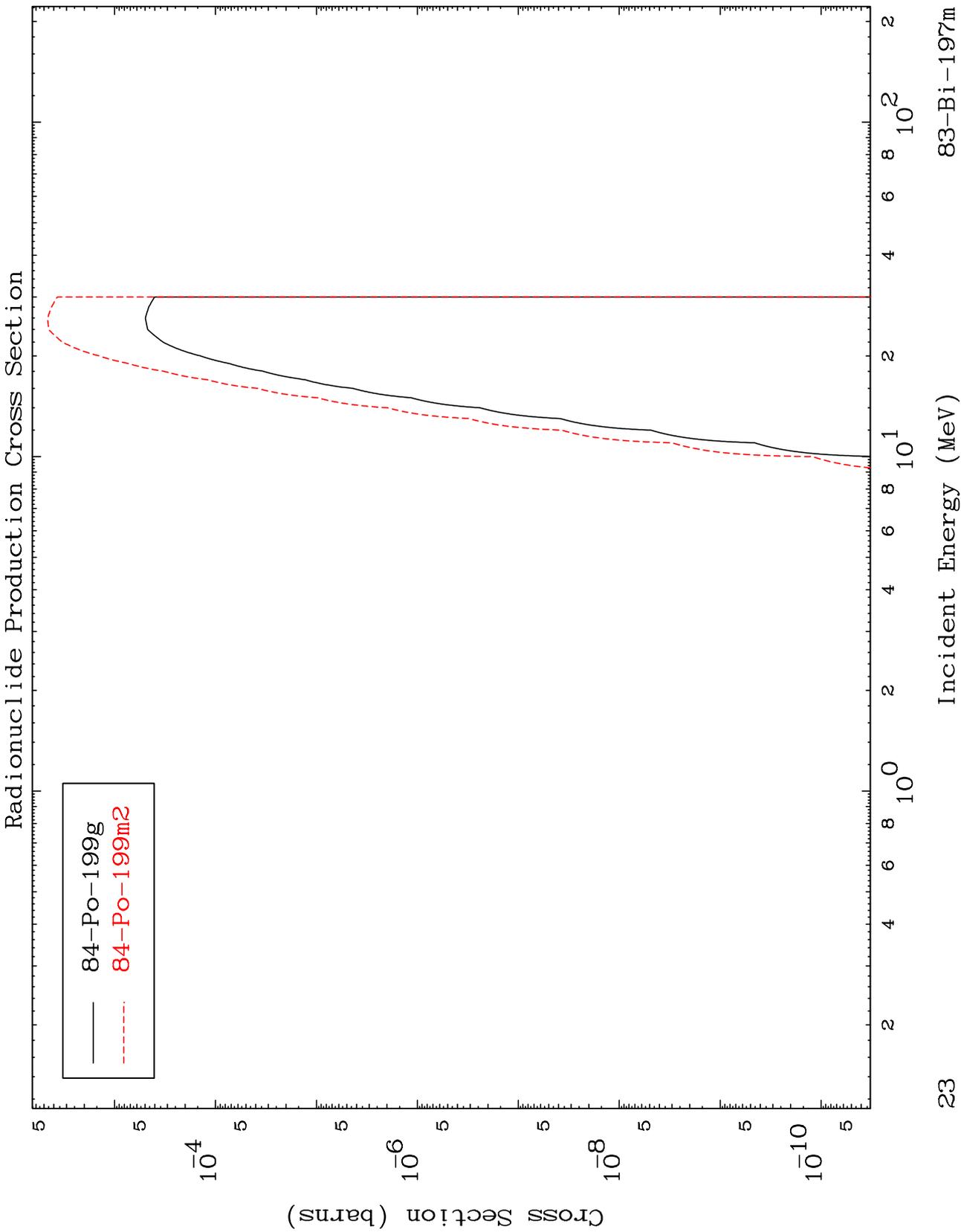
MAT 8290

83-Bi-197m



MAT 8290

83-Bi-197m



83-Bi-197m

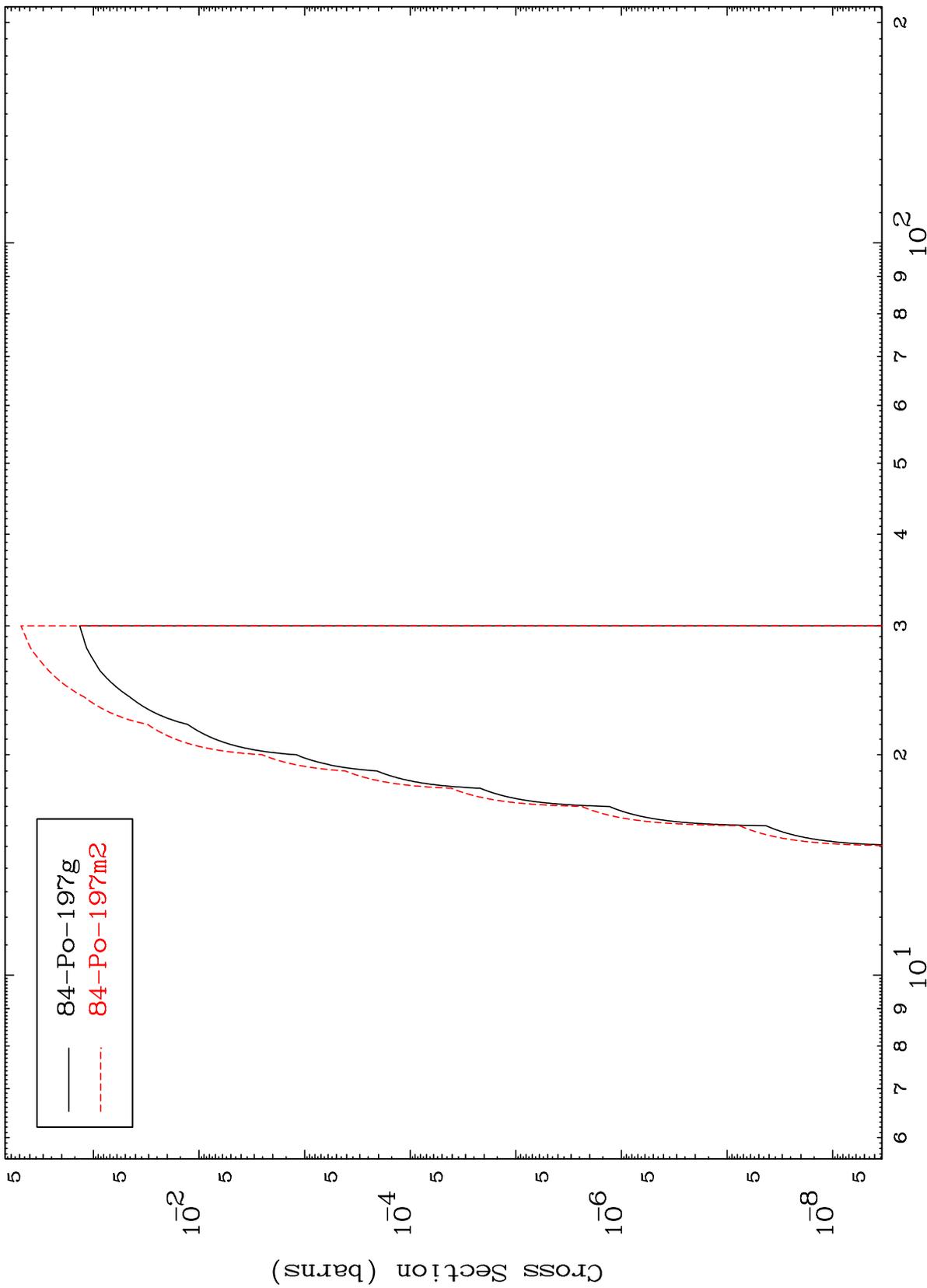
Incident Energy (MeV)

23

MAT 8290

83-Bi-197m

(n,t)
Radionuclide Production Cross Section



24

Incident Energy (MeV)

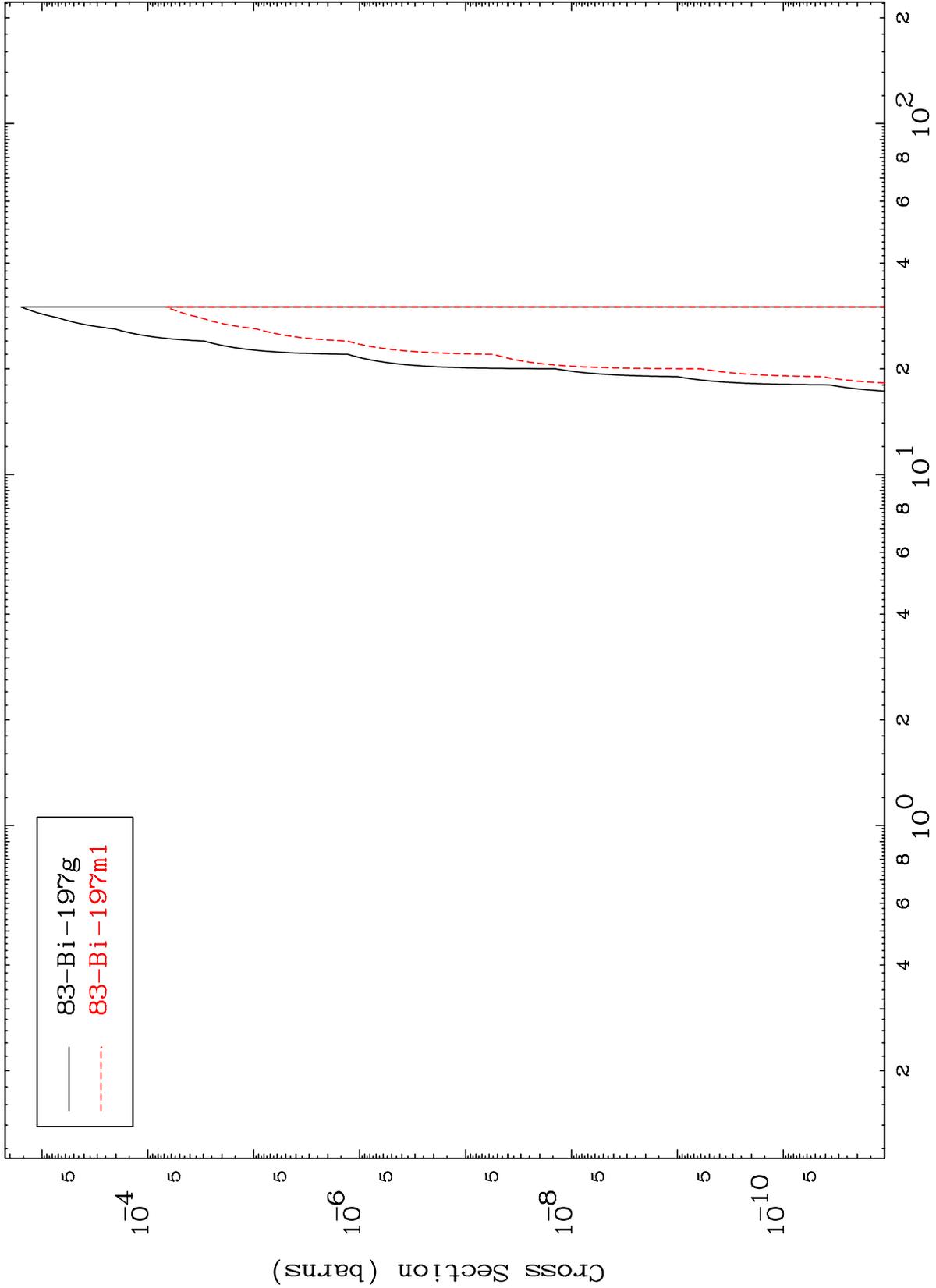
83-Bi-197m

MAT 8290

(n,He-3)

83-Bi-197m

Radionuclide Production Cross Section

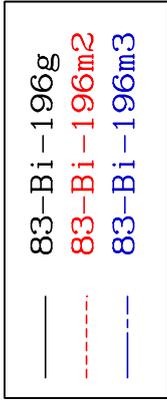
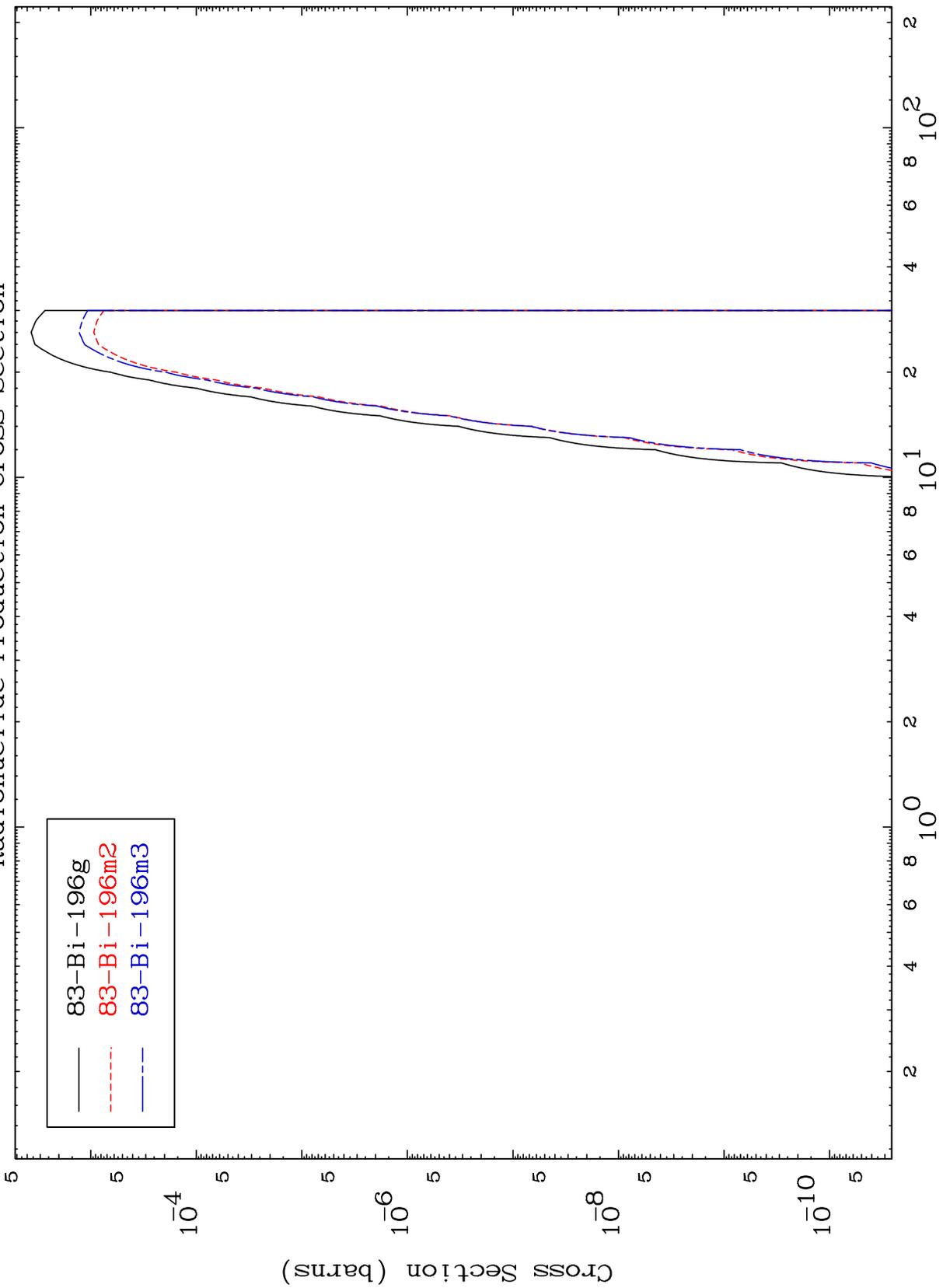


83-Bi-197g
83-Bi-197m1

MAT 8290

$^{83}\text{Bi}-197\text{m}$

Radionuclide Production Cross Section

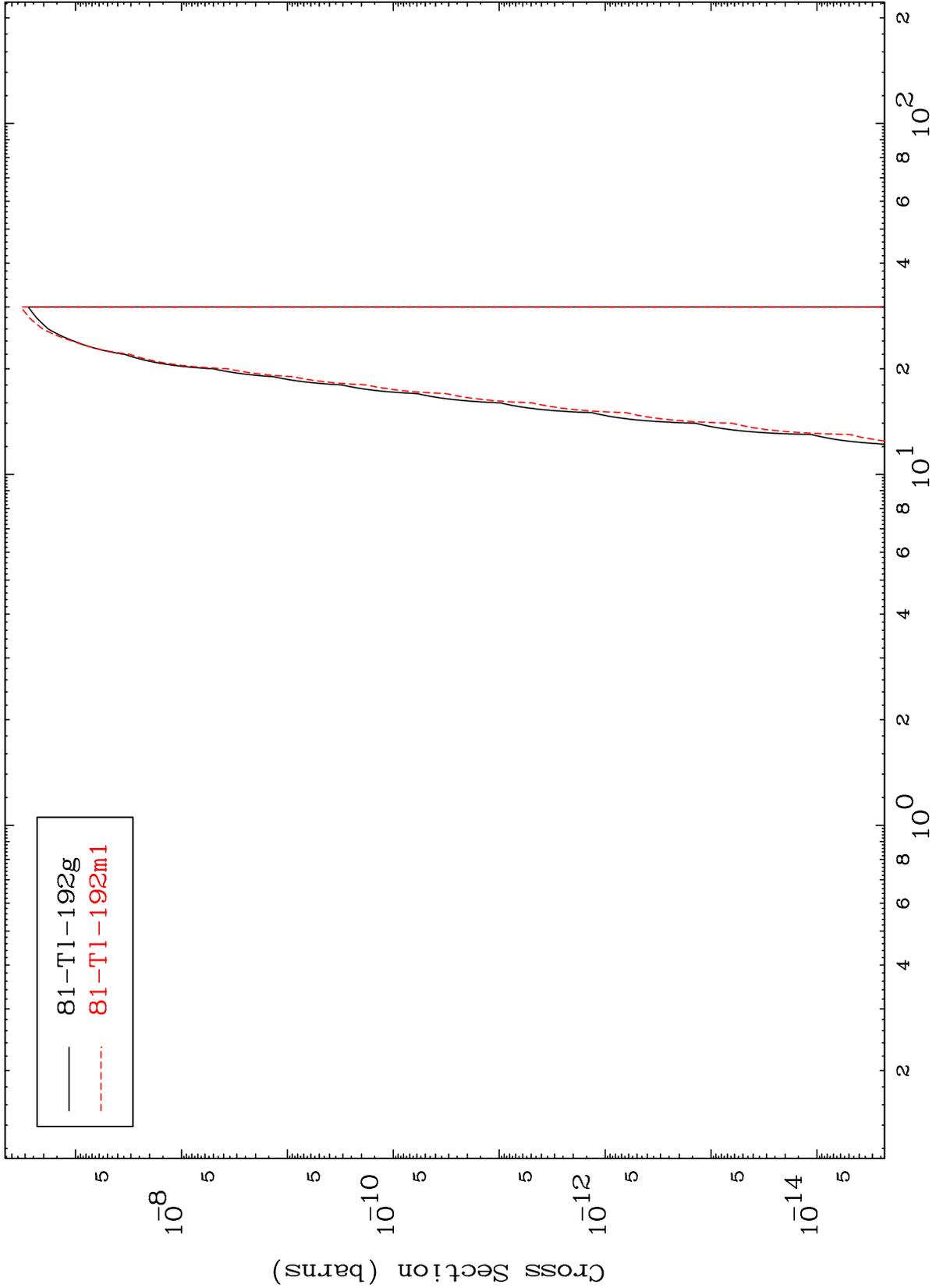


MAT 8290

(n,2α)

83-Bi-197m

Radionuclide Production Cross Section



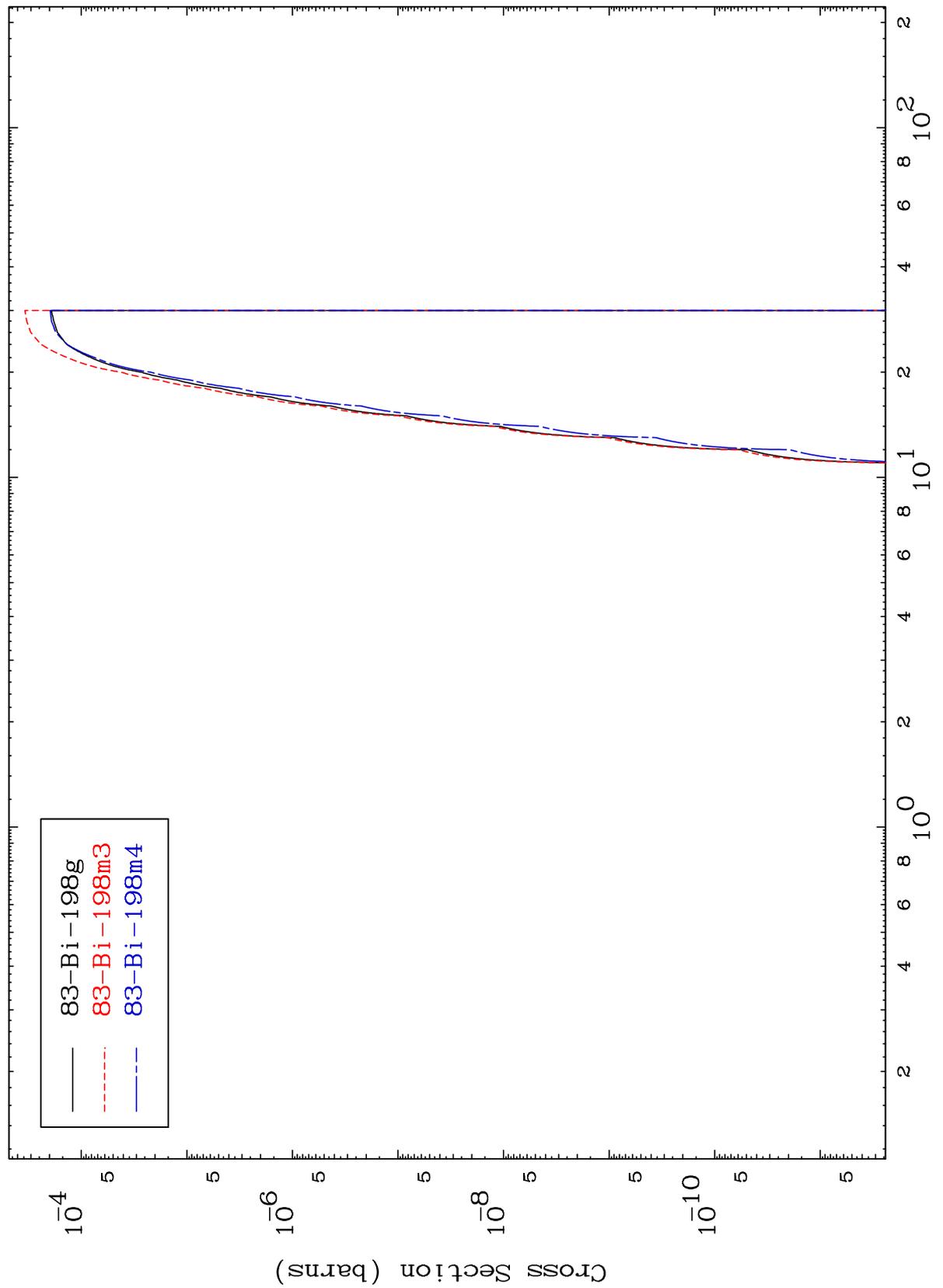
81-Tl-192g
81-Tl-192m1

MAT 8290

⁸³Bi-197m

(n,2p)

Radionuclide Production Cross Section

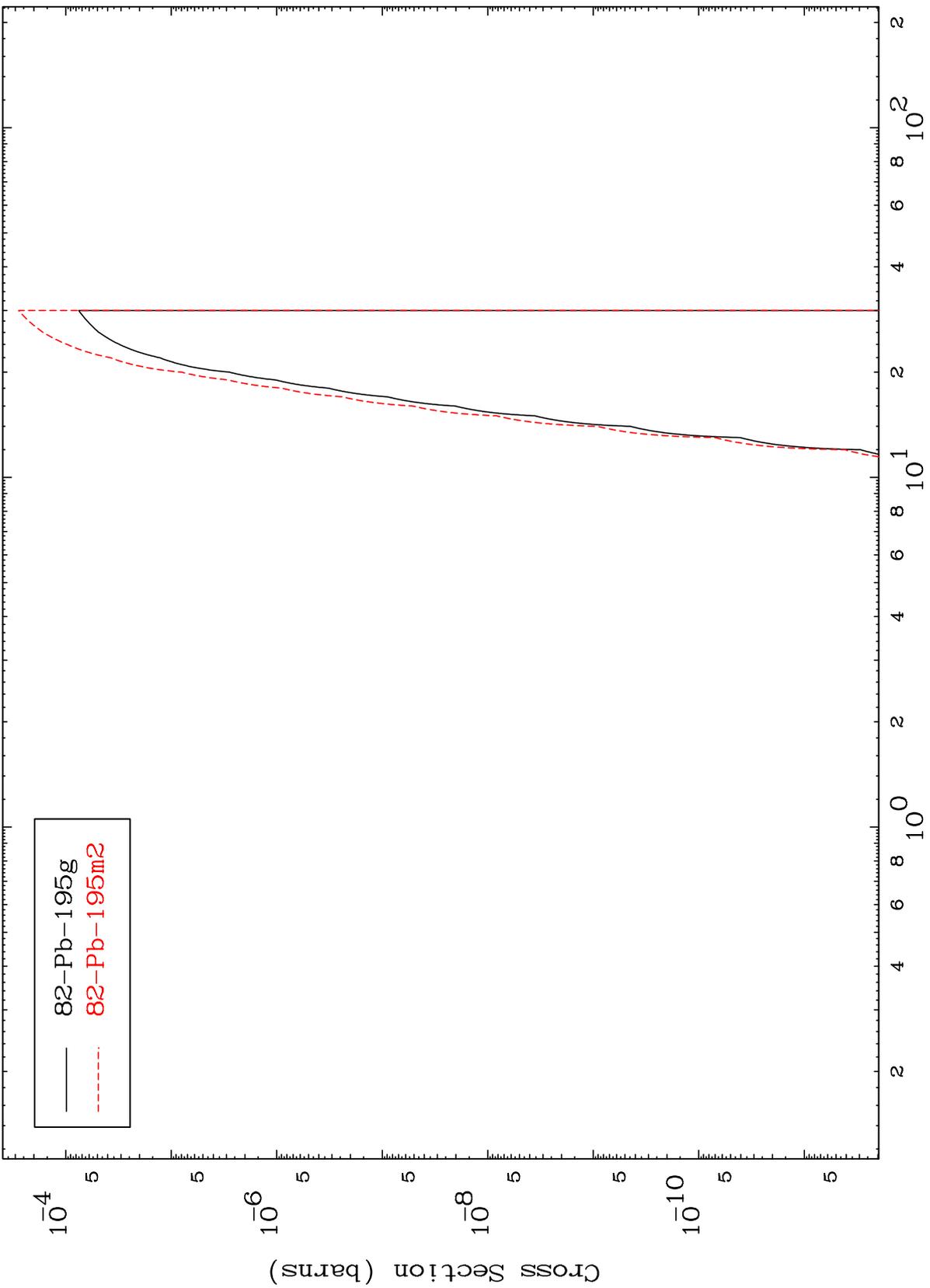


MAT 8290

(n,p) α

83-Bi-197m

Radionuclide Production Cross Section

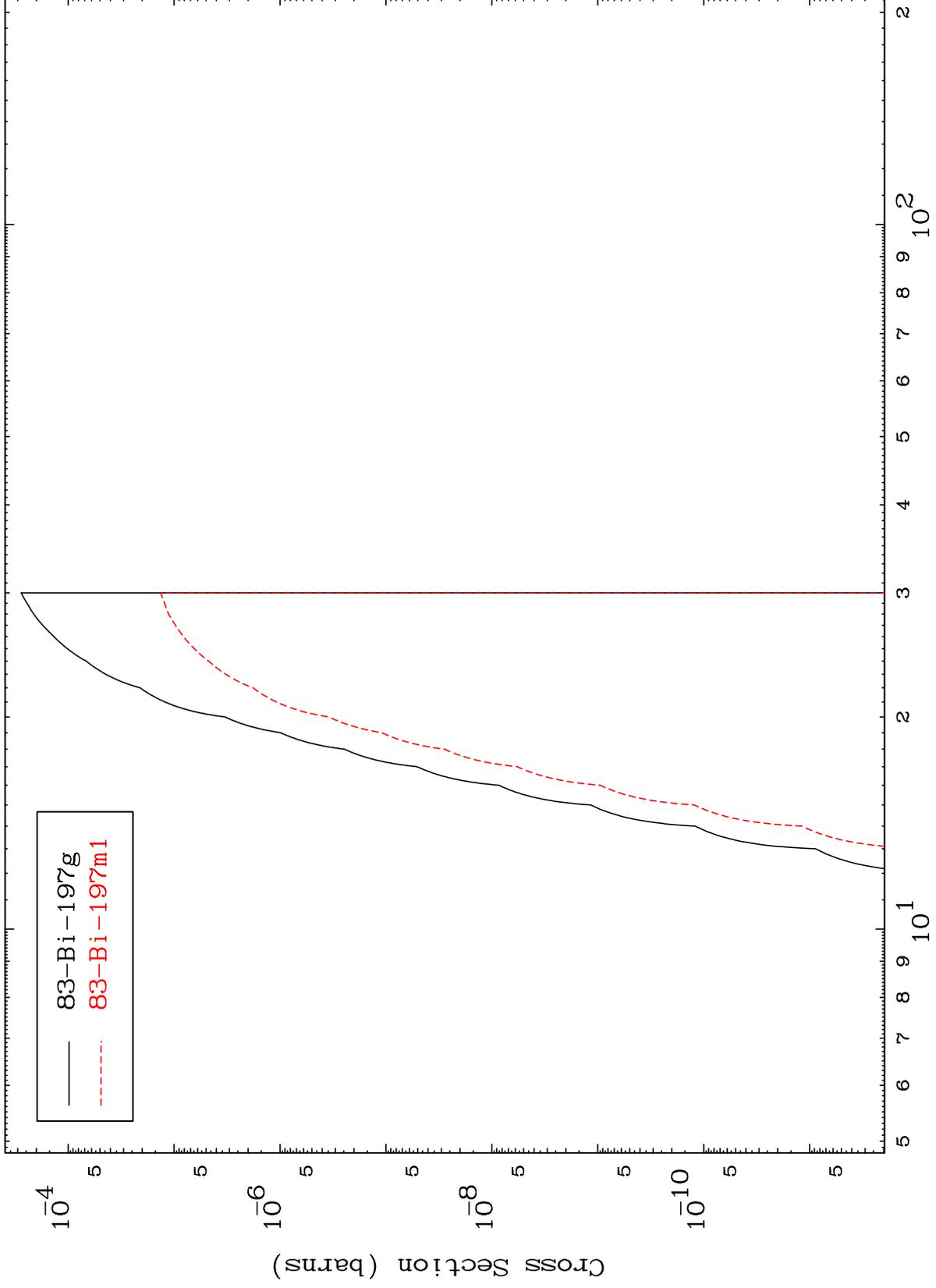


MAT 8290

(n,p) d

83-Bi-197m

Radionuclide Production Cross Section



30

Incident Energy (MeV)

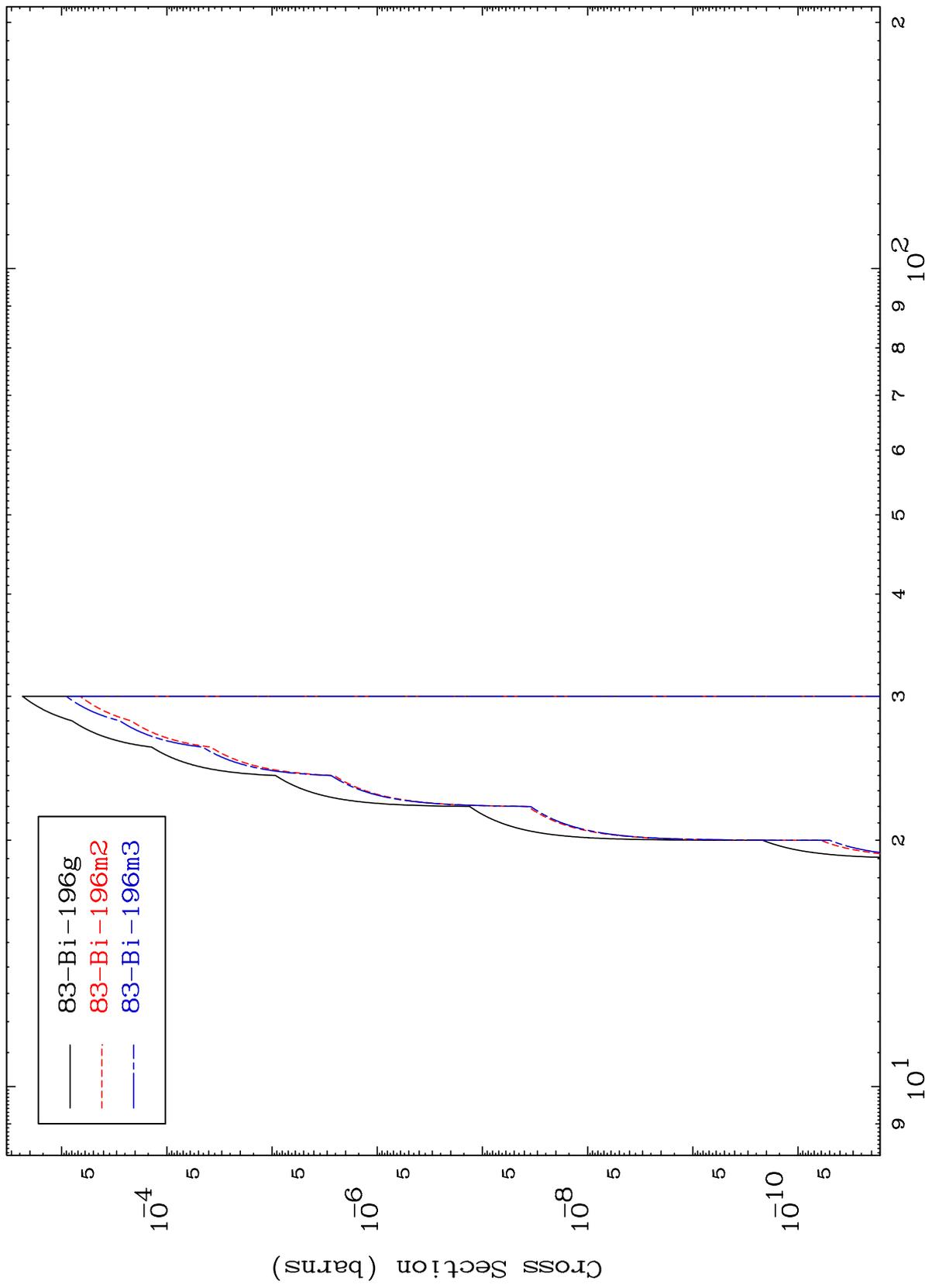
83-Bi-197m

MAT 8290

83-Bi-197m

(n,p) t

Radionuclide Production Cross Section



31

Incident Energy (MeV)

83-Bi-197m