

Program EVALPLOT  
(Version 2021-1)

by

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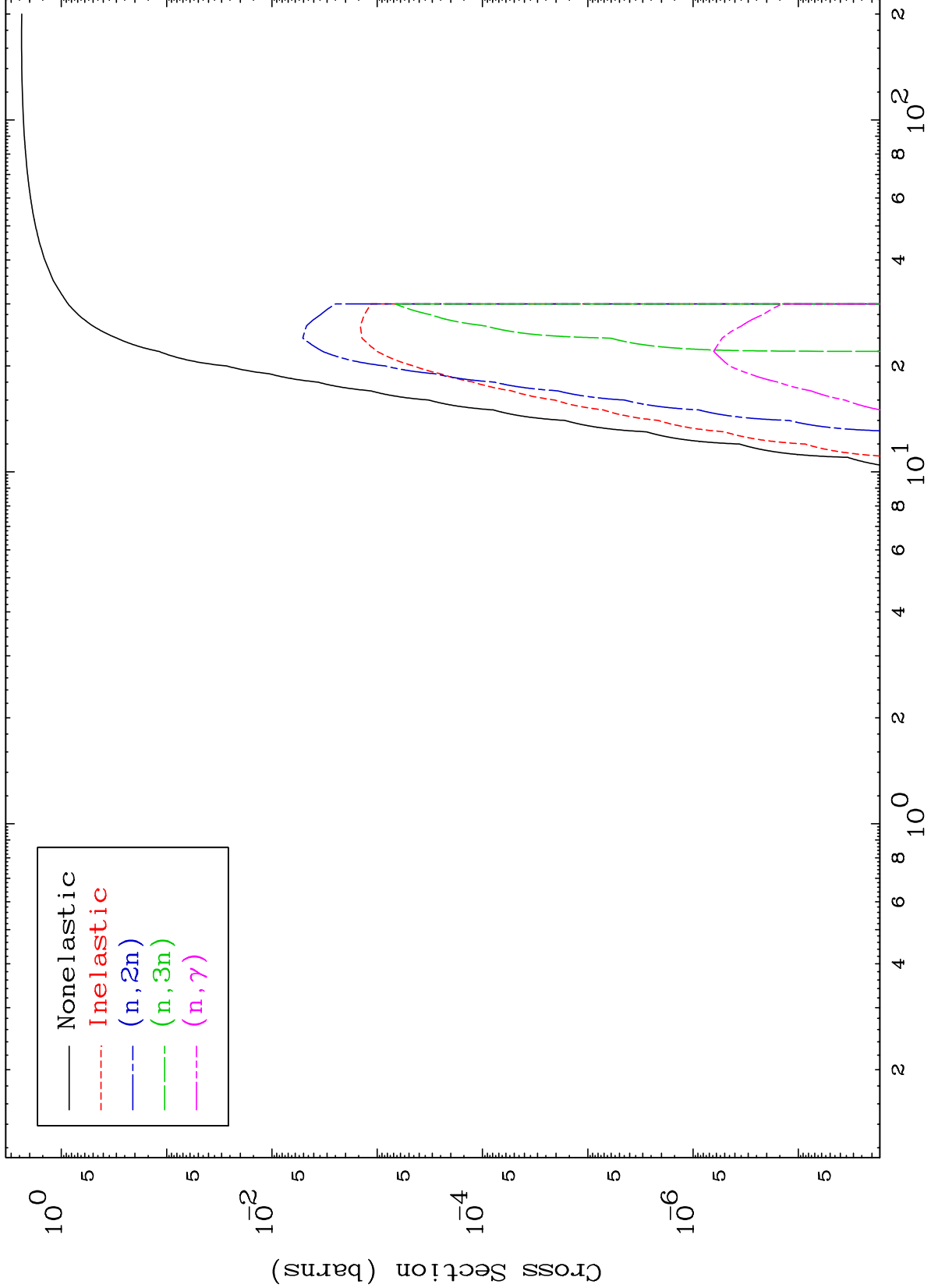
Press Mouse Button to Start

MAT 8293

He-3 Major

83-Bi-198m

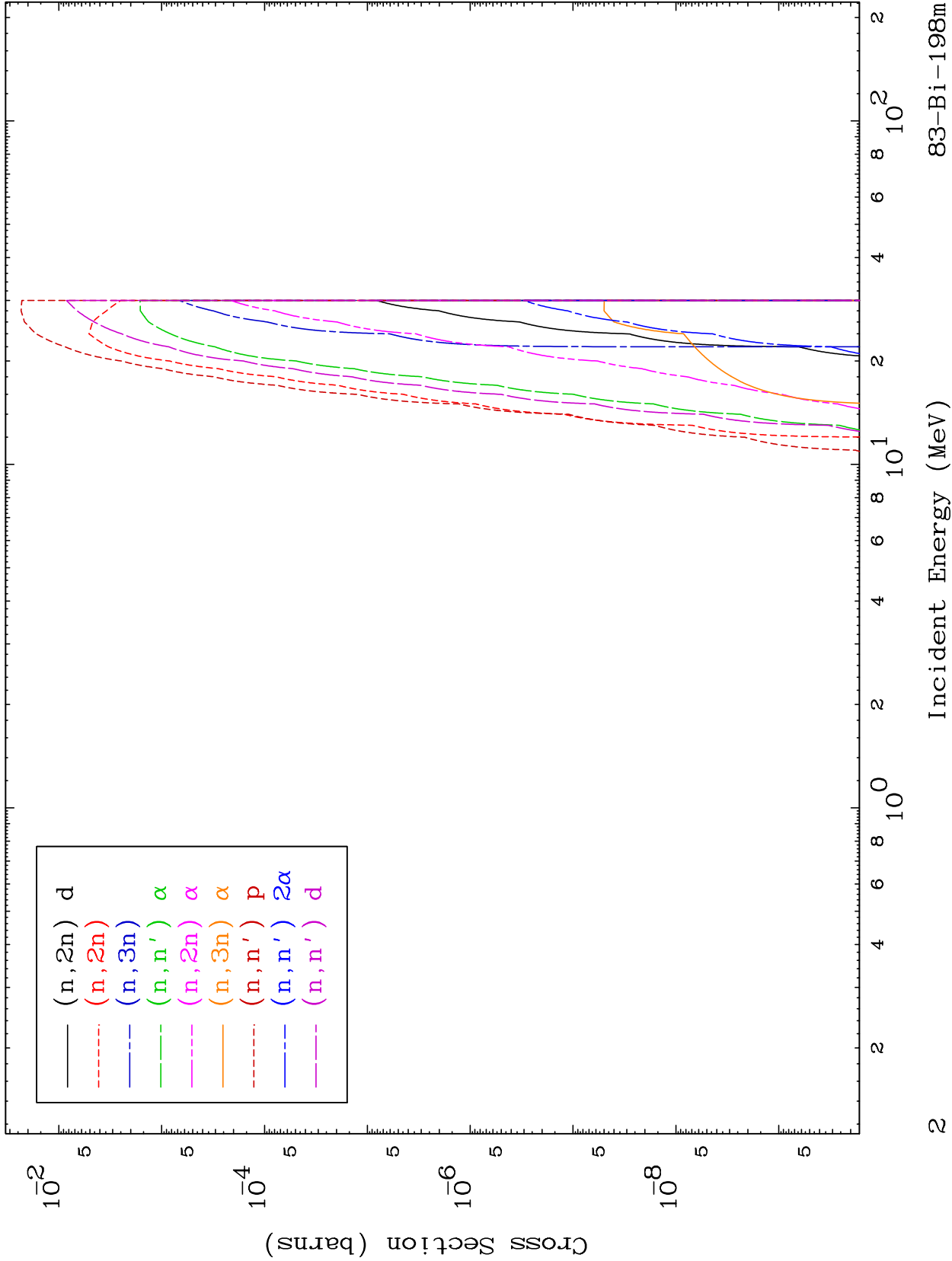
0 Kelvin Cross Sections



MAT 8293

He-3 Neutron Absorption  
0 Kelvin Cross Sections

83-Bi-198m



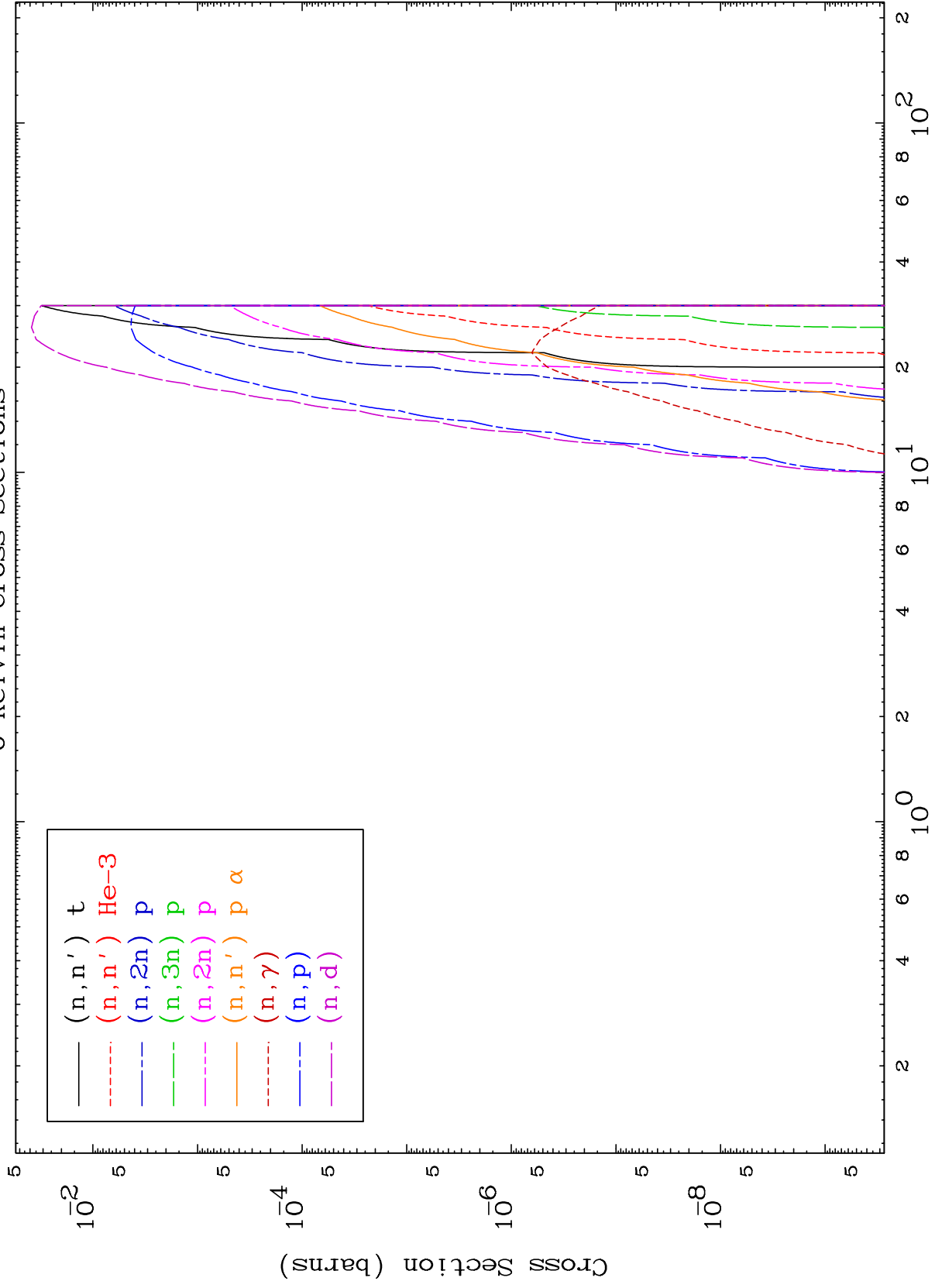
83-Bi-198m

Incident Energy (MeV)

MAT 8293

He-3 Neutron Absorption  
0 Kelvin Cross Sections

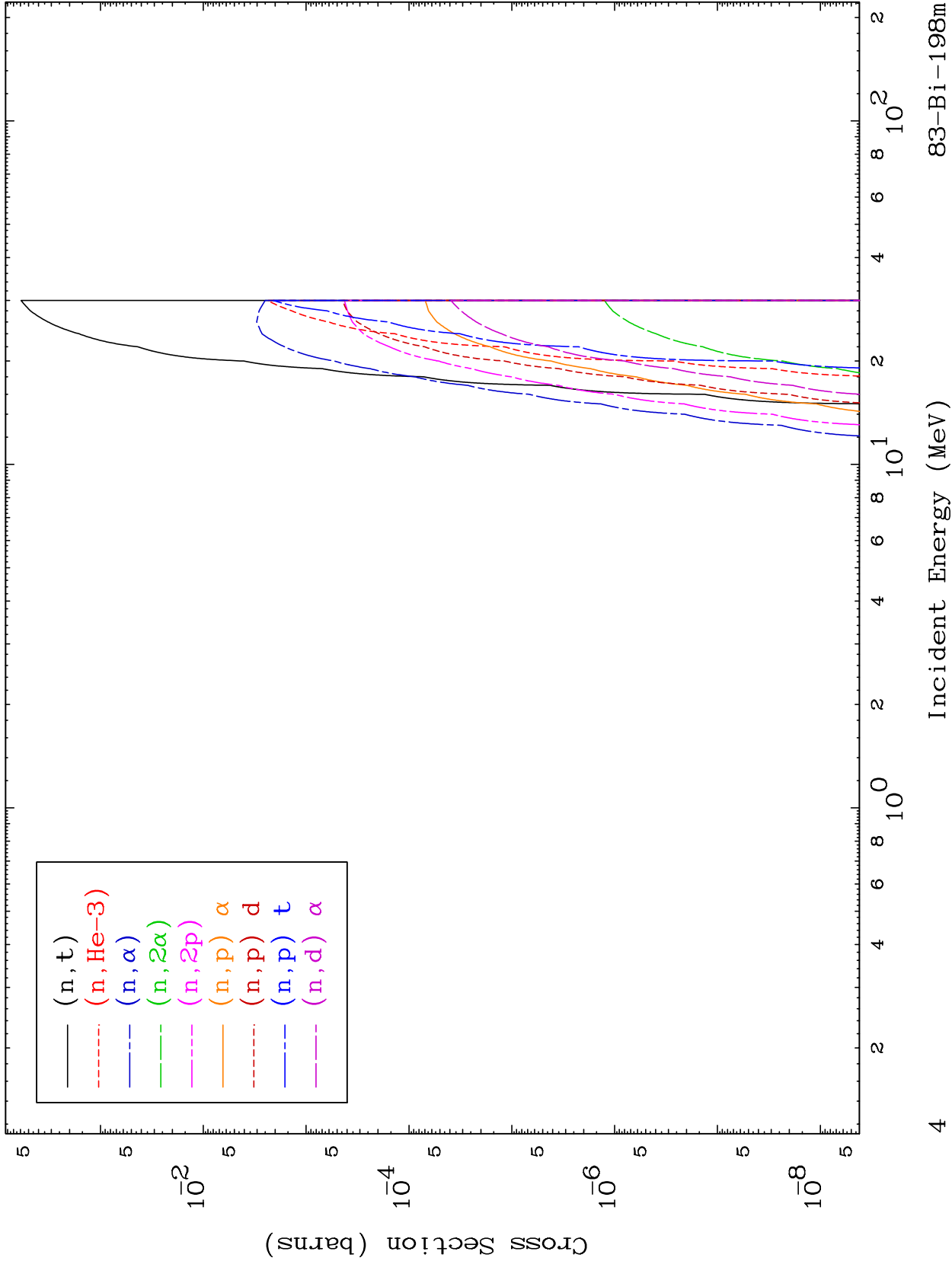
83-Bi-198m



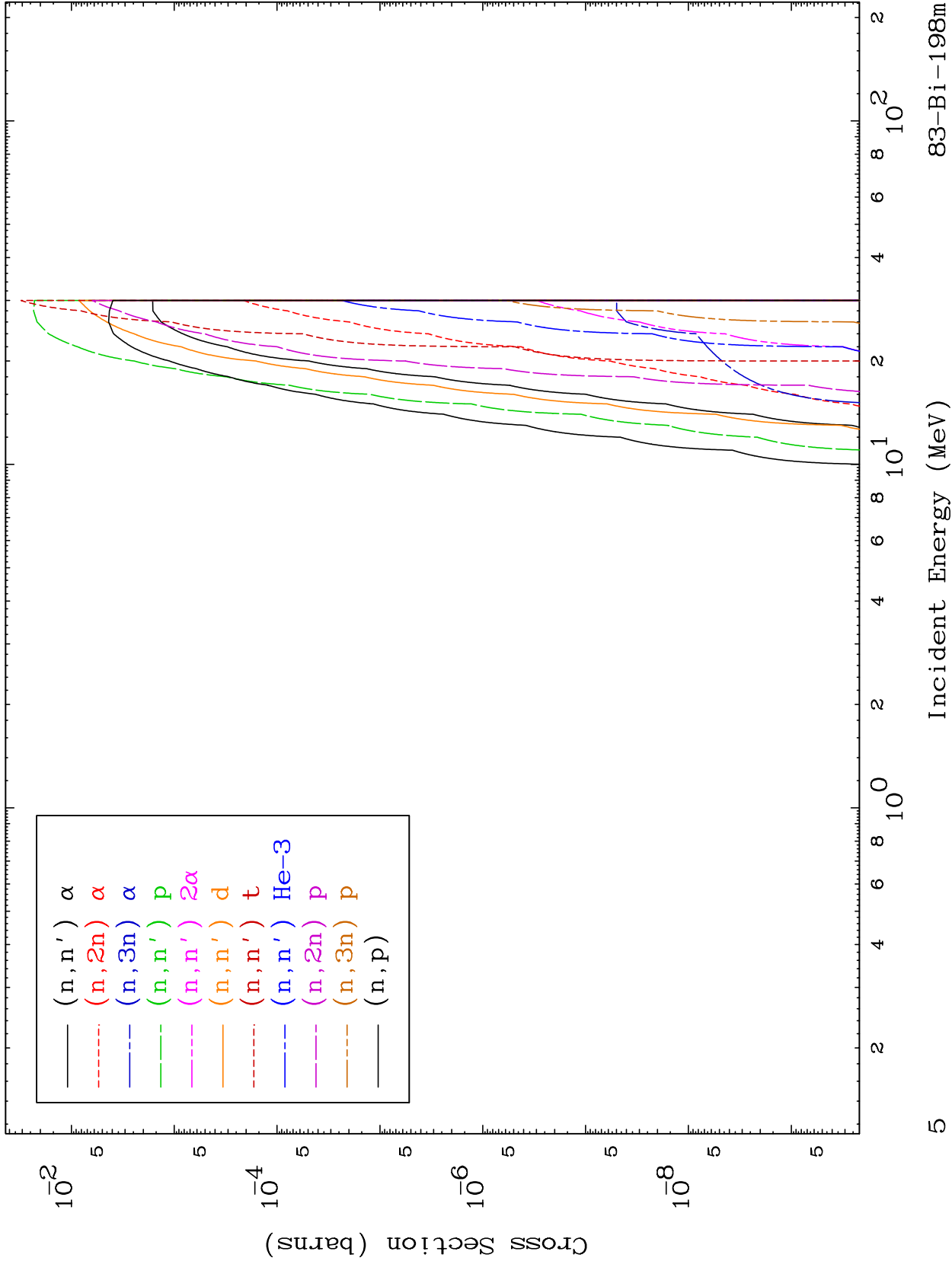
MAT 8293

He-3 Neutron Absorption  
0 Kelvin Cross Sections

83-Bi-198m



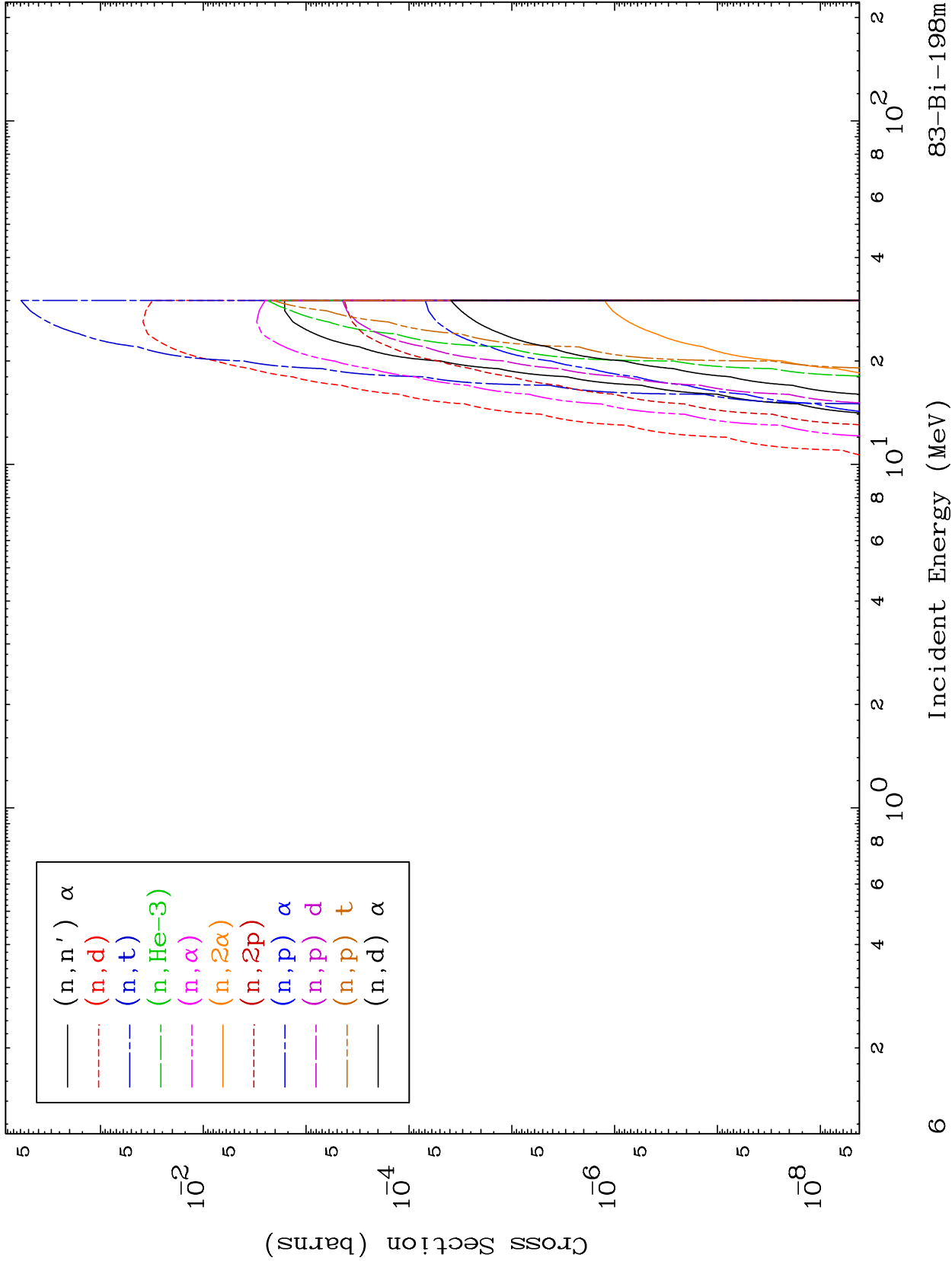
83-Bi-198m



MAT 8293

He-3 Charged Particle  
0 Kelvin Cross Sections

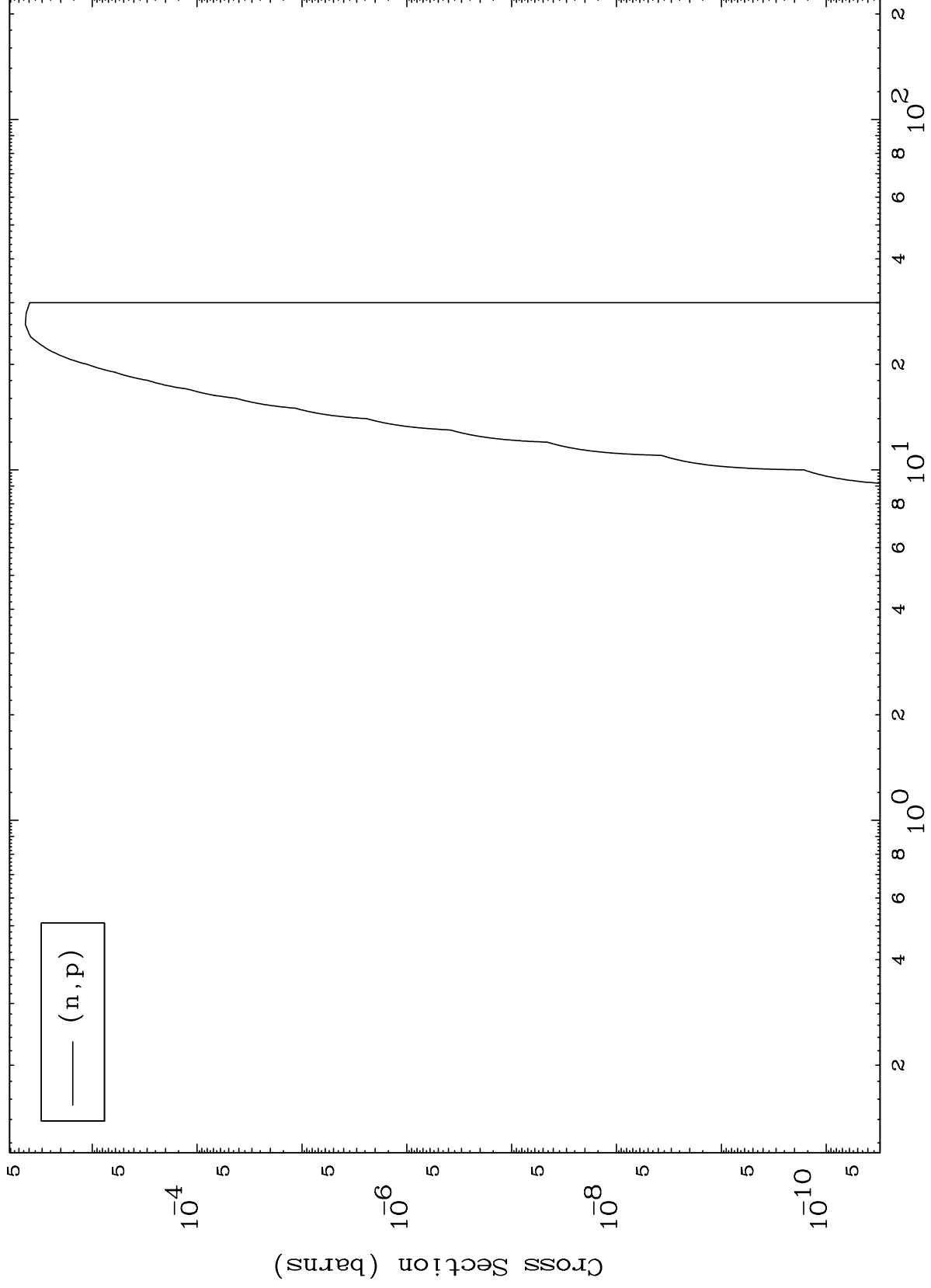
83-Bi-198m



MAT 8293

(He-3,p) Levels  
0 Kelvin Cross Sections

83-Bi-198m



7

Incident Energy (MeV)

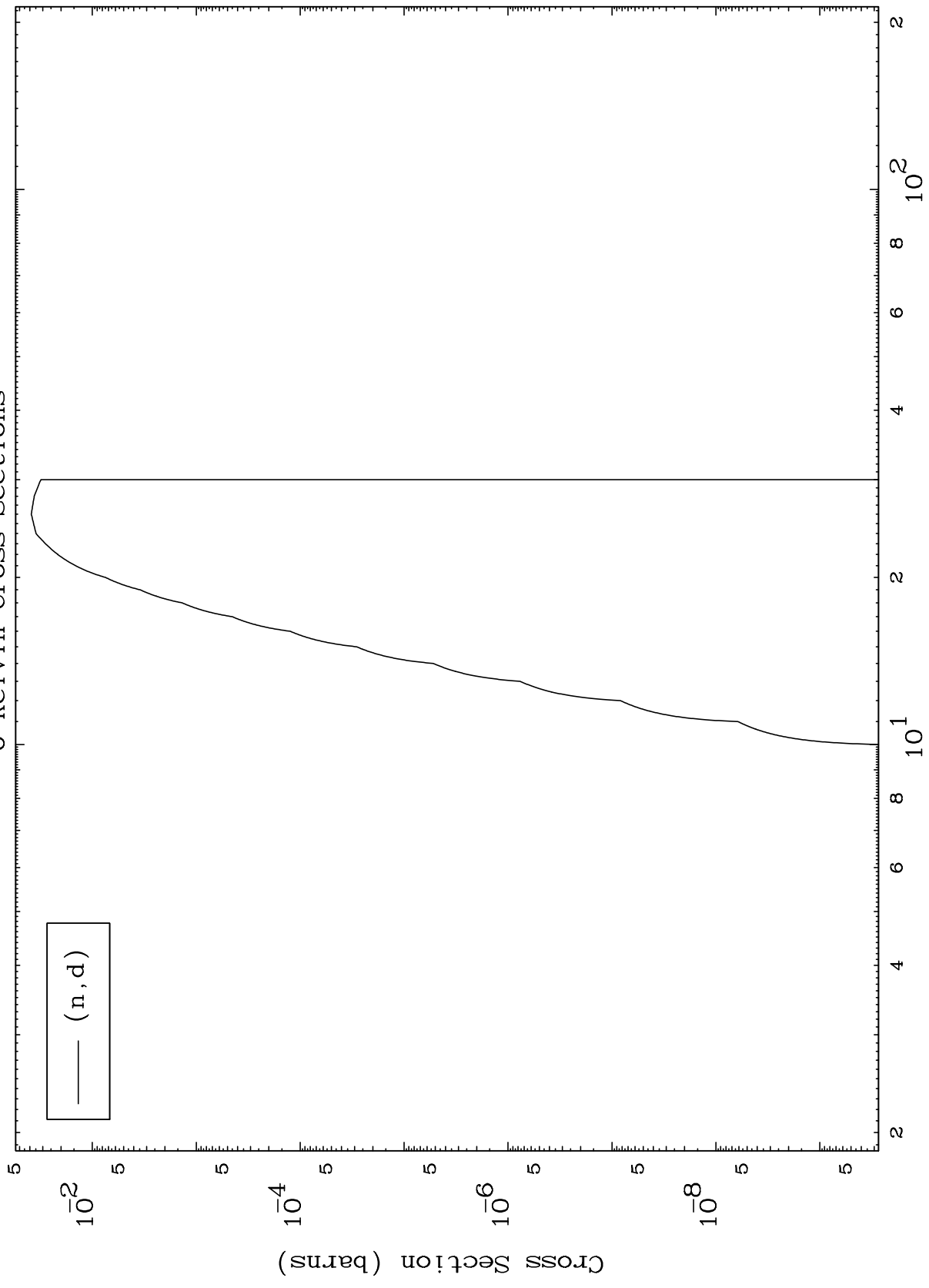
83-Bi-198m

MAT 8293

(He-3,d) Levels

83-Bi-198m

0 Kelvin Cross Sections



8

Incident Energy (MeV)

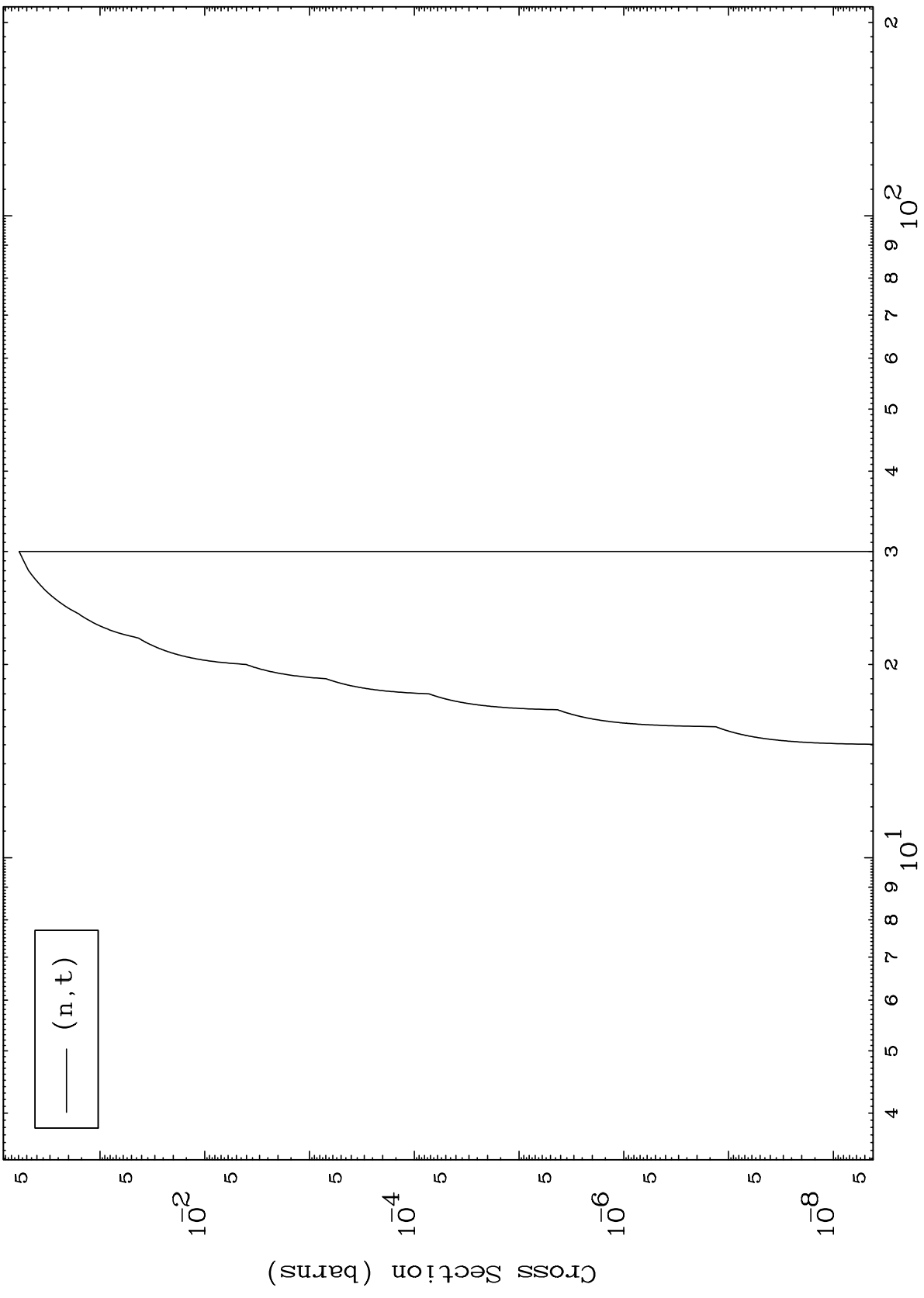
83-Bi-198m

MAT 8293

(He-3,t) Levels

83-Bi-198m

0 Kelvin Cross Sections

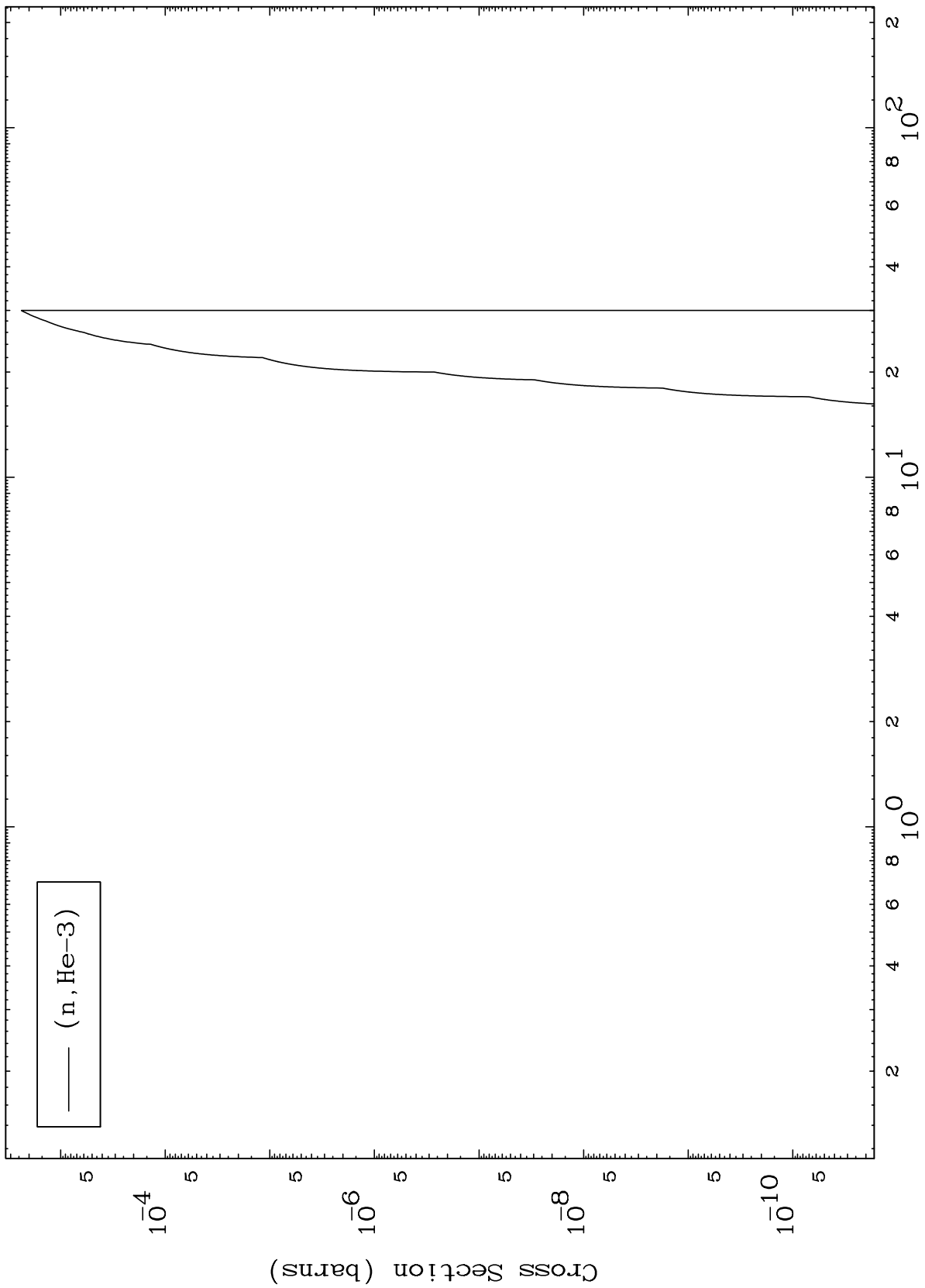


MAT 8293

(He-3, He3) Levels

83-Bi-198m

0 Kelvin Cross Sections



10

Incident Energy (MeV)

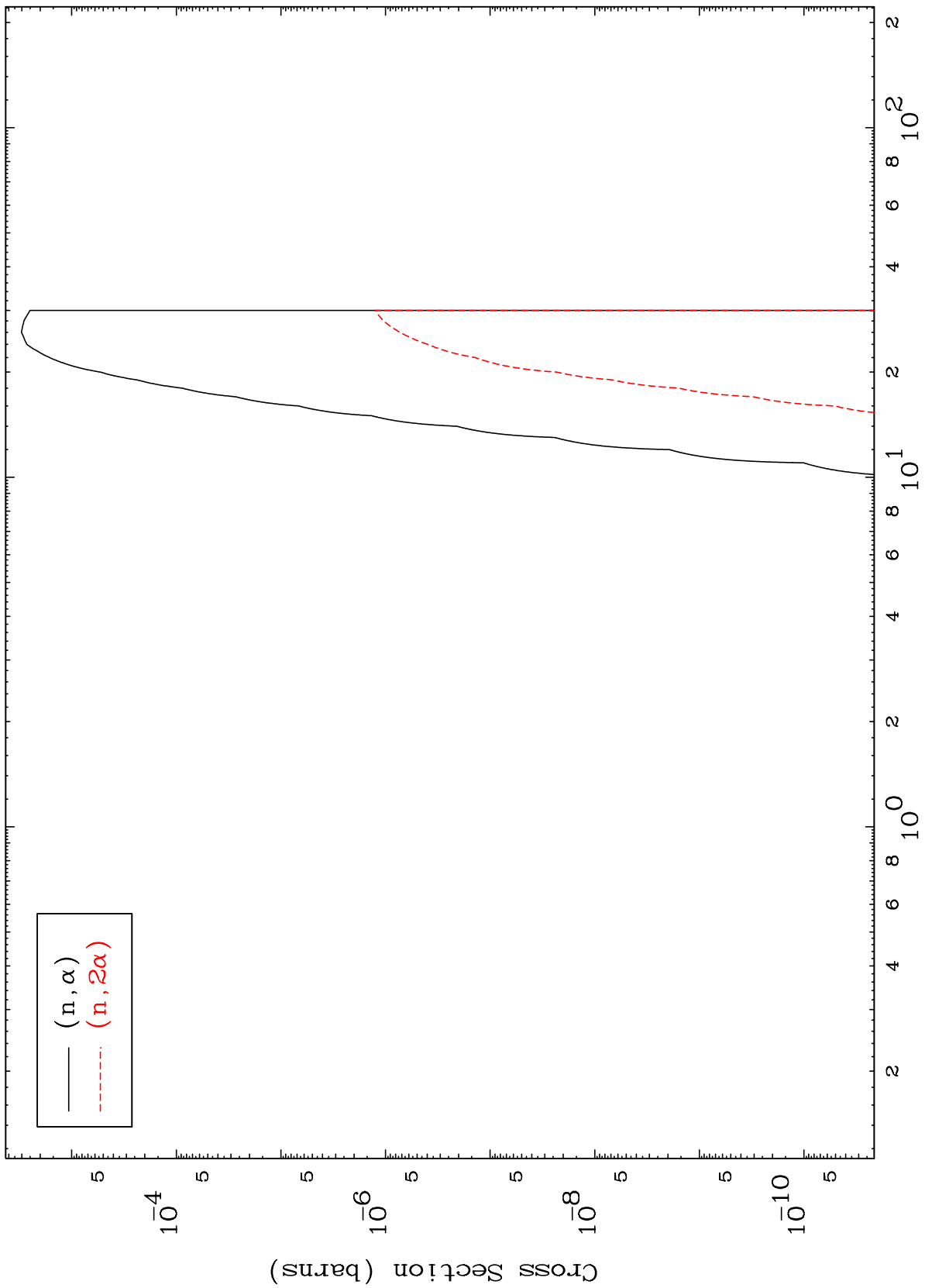
83-Bi-198m

MAT 8293

(He-3,  $\alpha$ ) Levels

83-Bi-198m

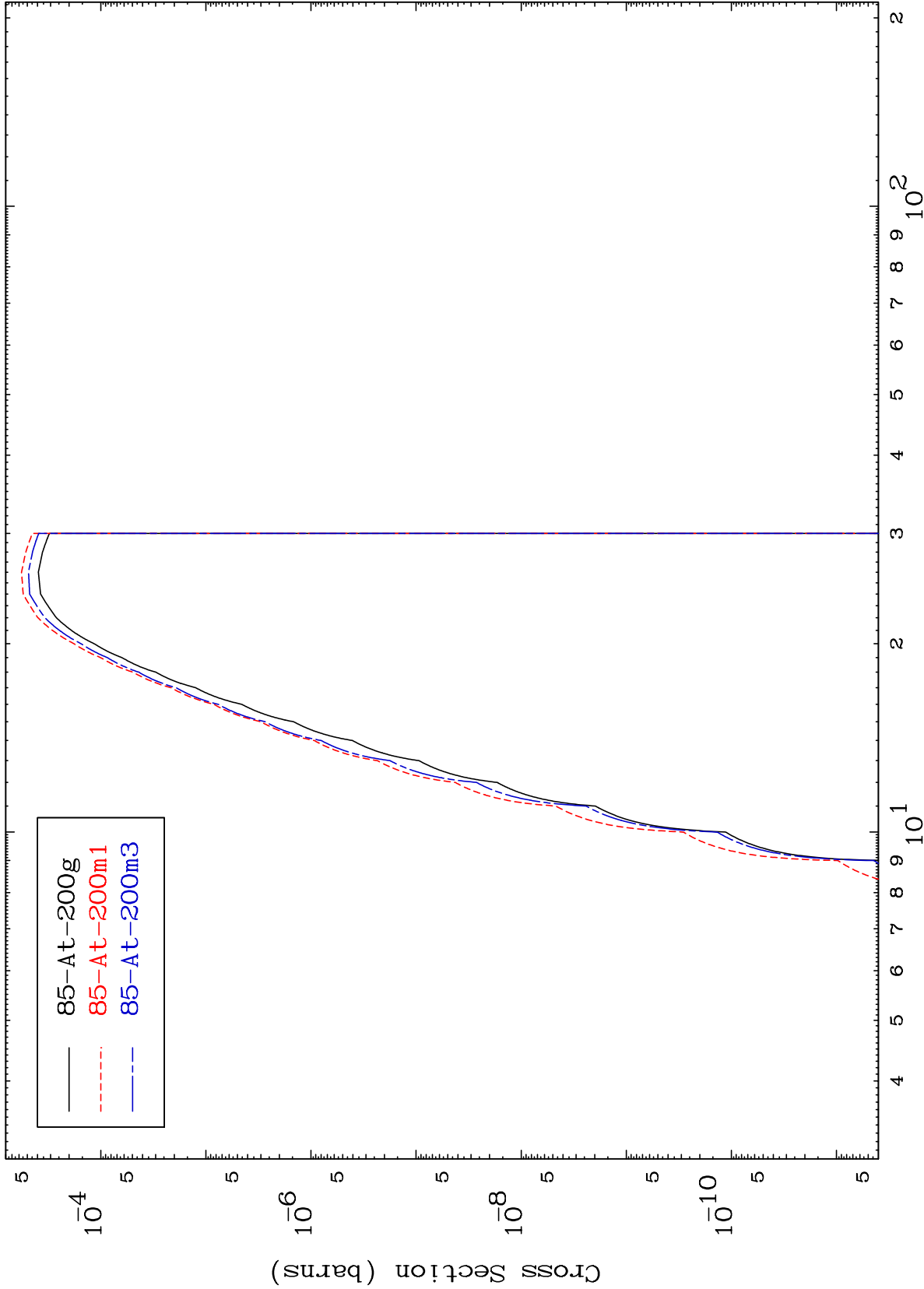
0 Kelvin Cross Sections



MAT 8293

83-Bi-198m

Radionuclide Production Cross Section



12

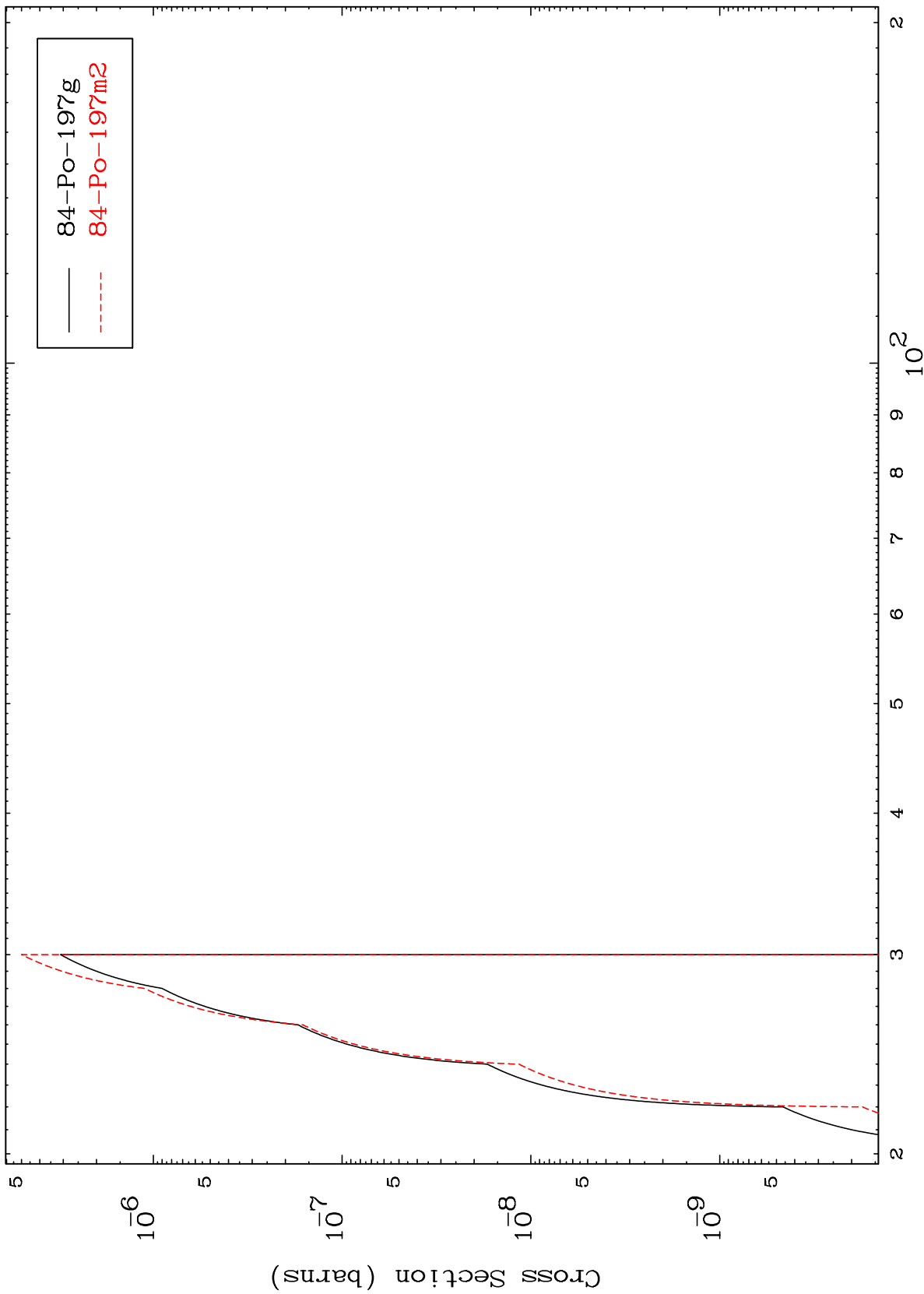
83-Bi-198m

MAT 8293

(n,2n) d

83-Bi-198m

Radionuclide Production Cross Section



13

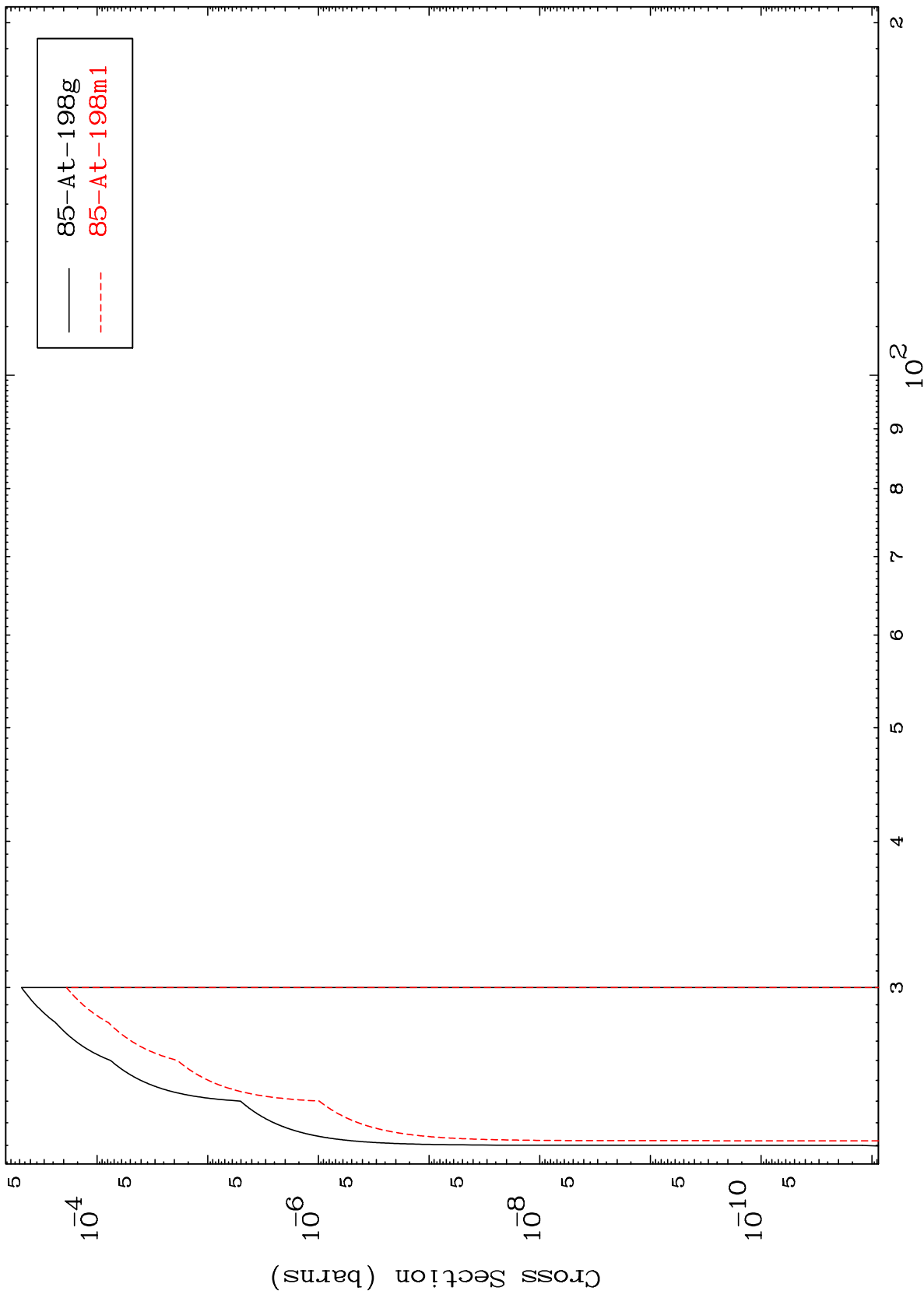
Incident Energy (MeV)

83-Bi-198m

MAT 8293

83-Bi-198m

(n,3n)  
Radionuclide Production Cross Section



83-Bi-198m

Incident Energy (MeV)

14

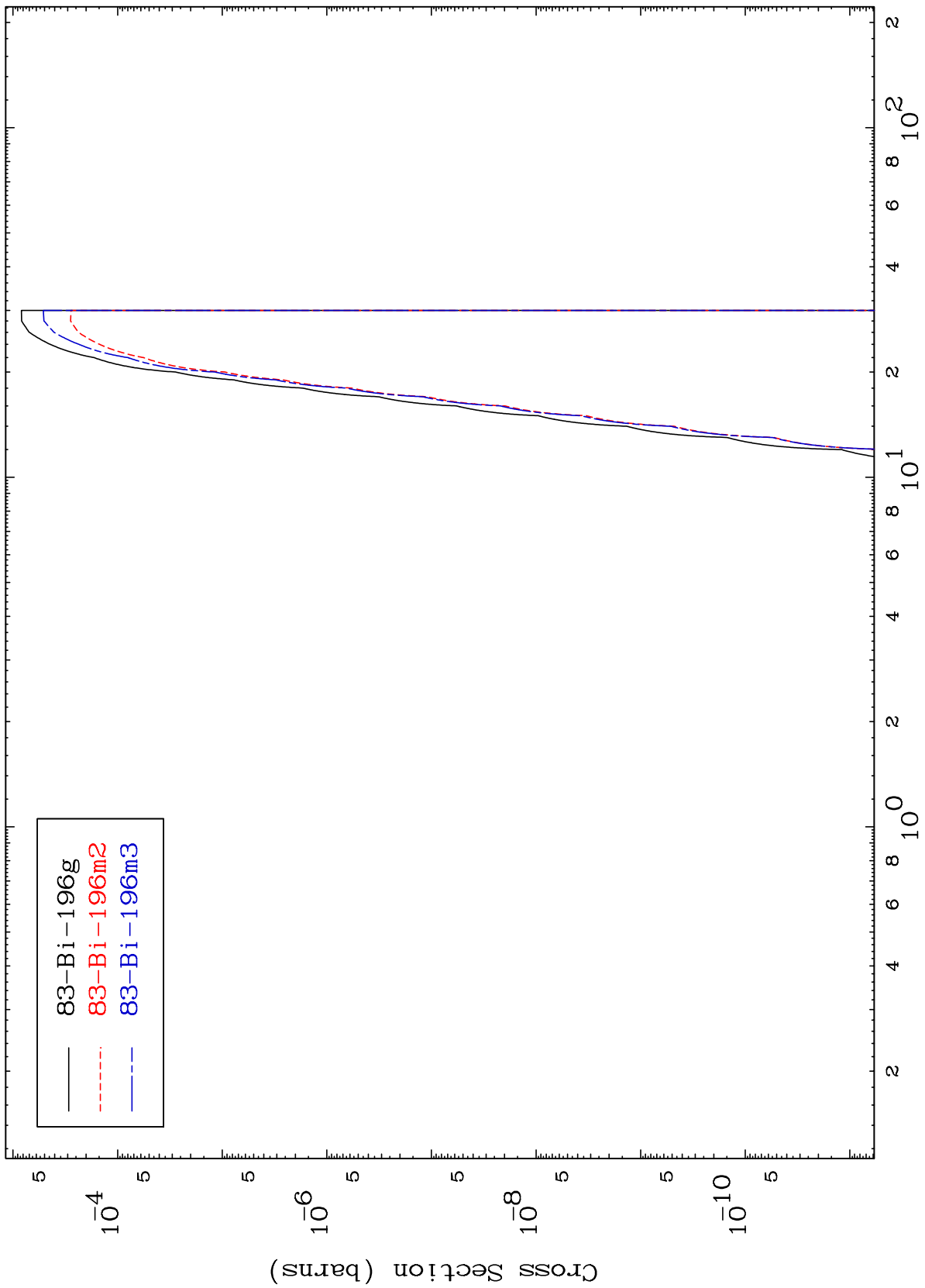


MAT 8293

$(n, n') \alpha$

$^{83}\text{Bi}-198\text{m}$

Radionuclide Production Cross Section



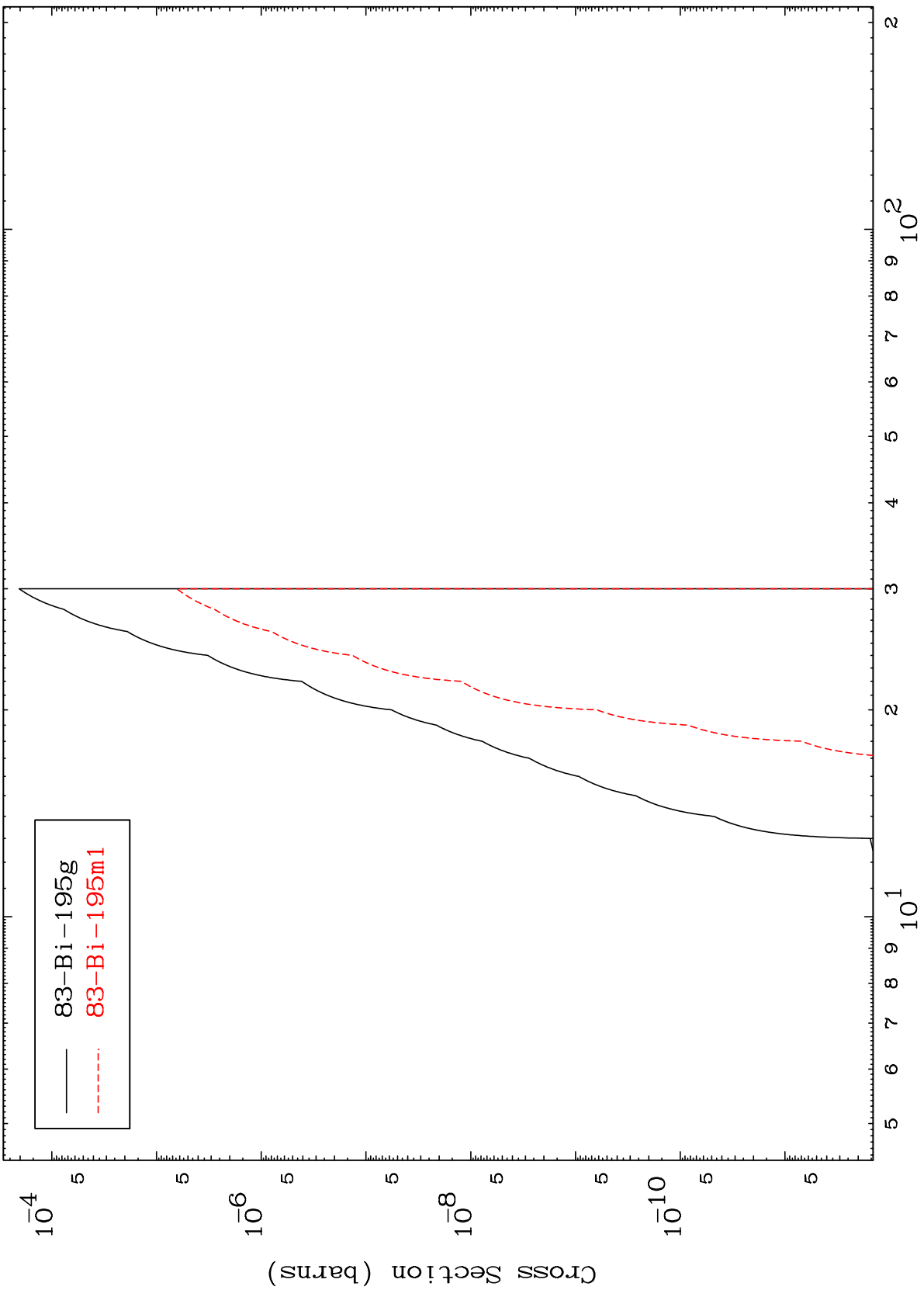
83-Bi-196g  
83-Bi-196m2  
83-Bi-196m3

MAT 8293

$(n, 2n) \alpha$

$^{83}\text{Bi}-198\text{m}$

Radionuclide Production Cross Section



17

Incident Energy (MeV)

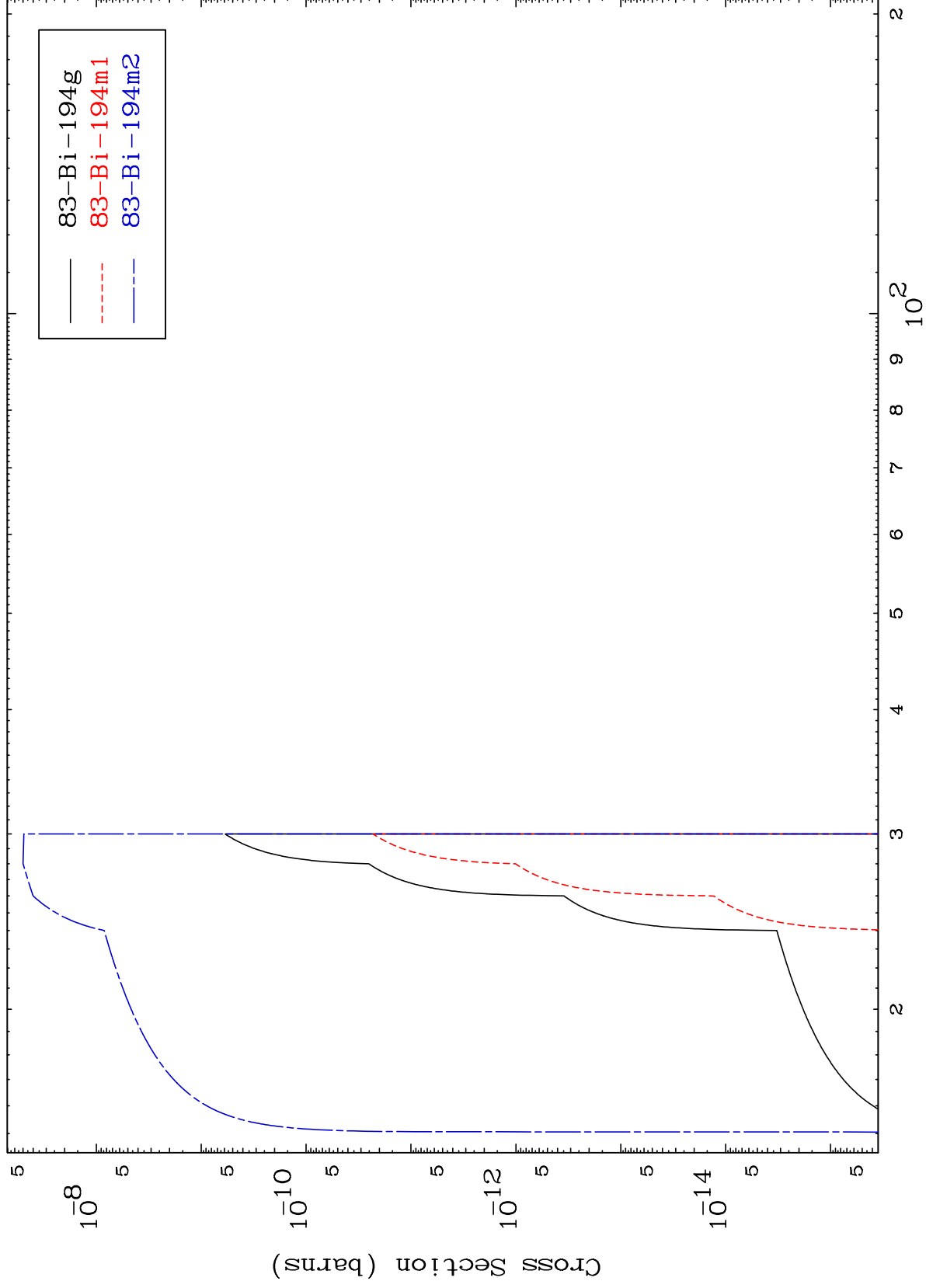
$^{83}\text{Bi}-198\text{m}$

MAT 8293

$(n,3n) \alpha$

83-Bi-198m

Radionuclide Production Cross Section



18

Incident Energy (MeV)

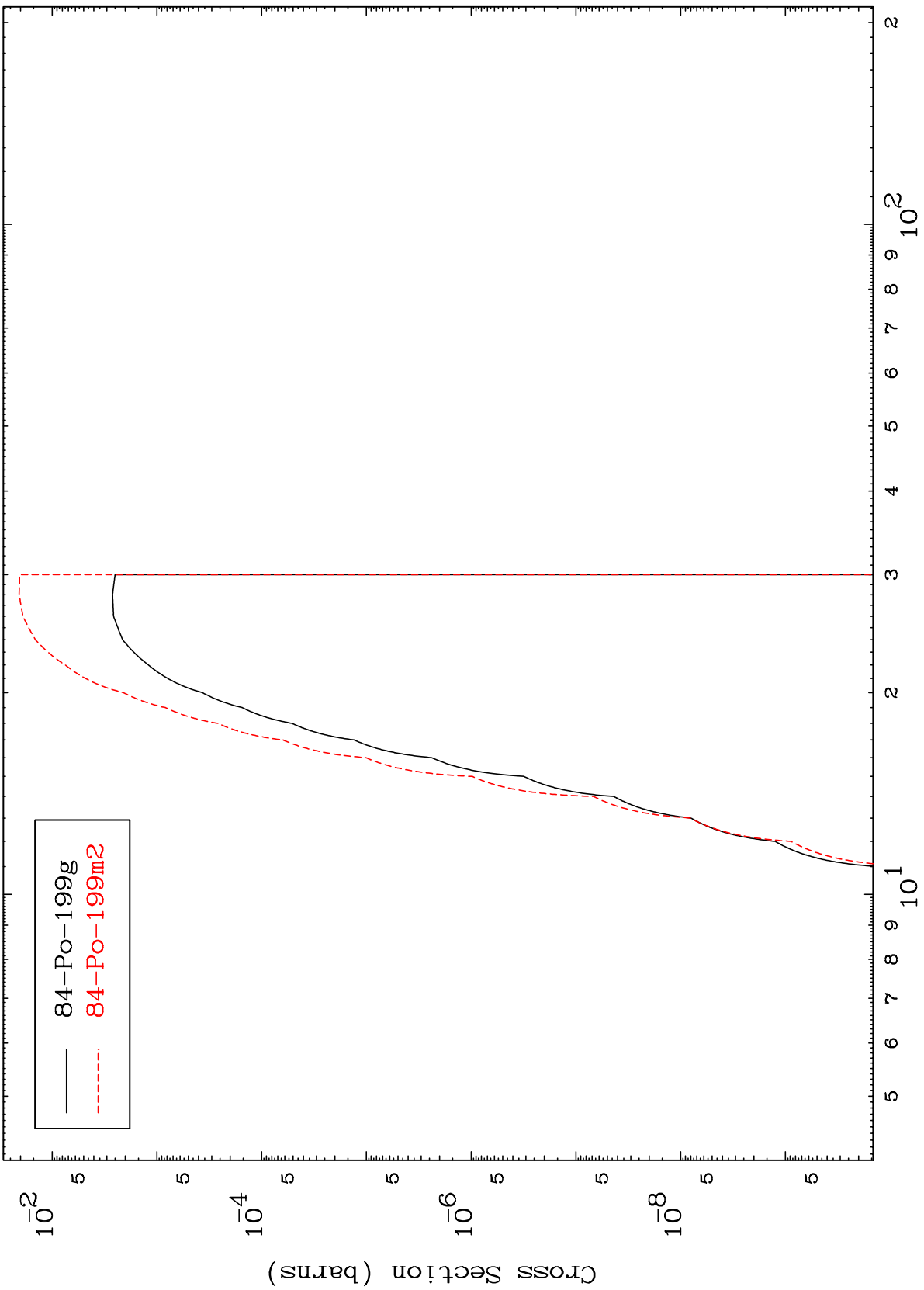
83-Bi-198m

MAT 8293

(n,n') p

83-Bi-198m

Radionuclide Production Cross Section



84-Po-199g  
84-Po-199m2

19

Incident Energy (MeV)

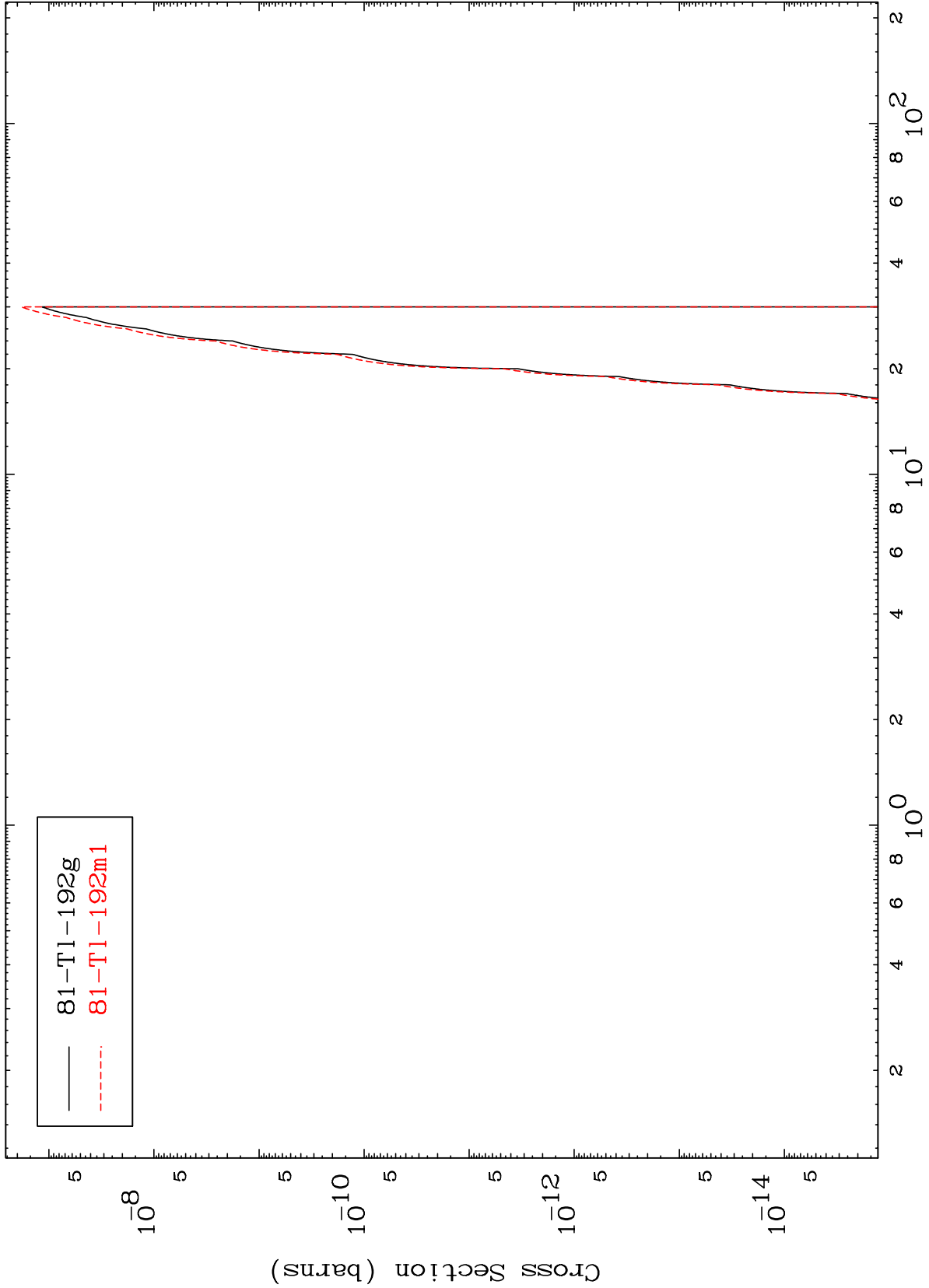
83-Bi-198m

MAT 8293

(n,n') 2 $\alpha$

83-Bi-198m

Radionuclide Production Cross Section

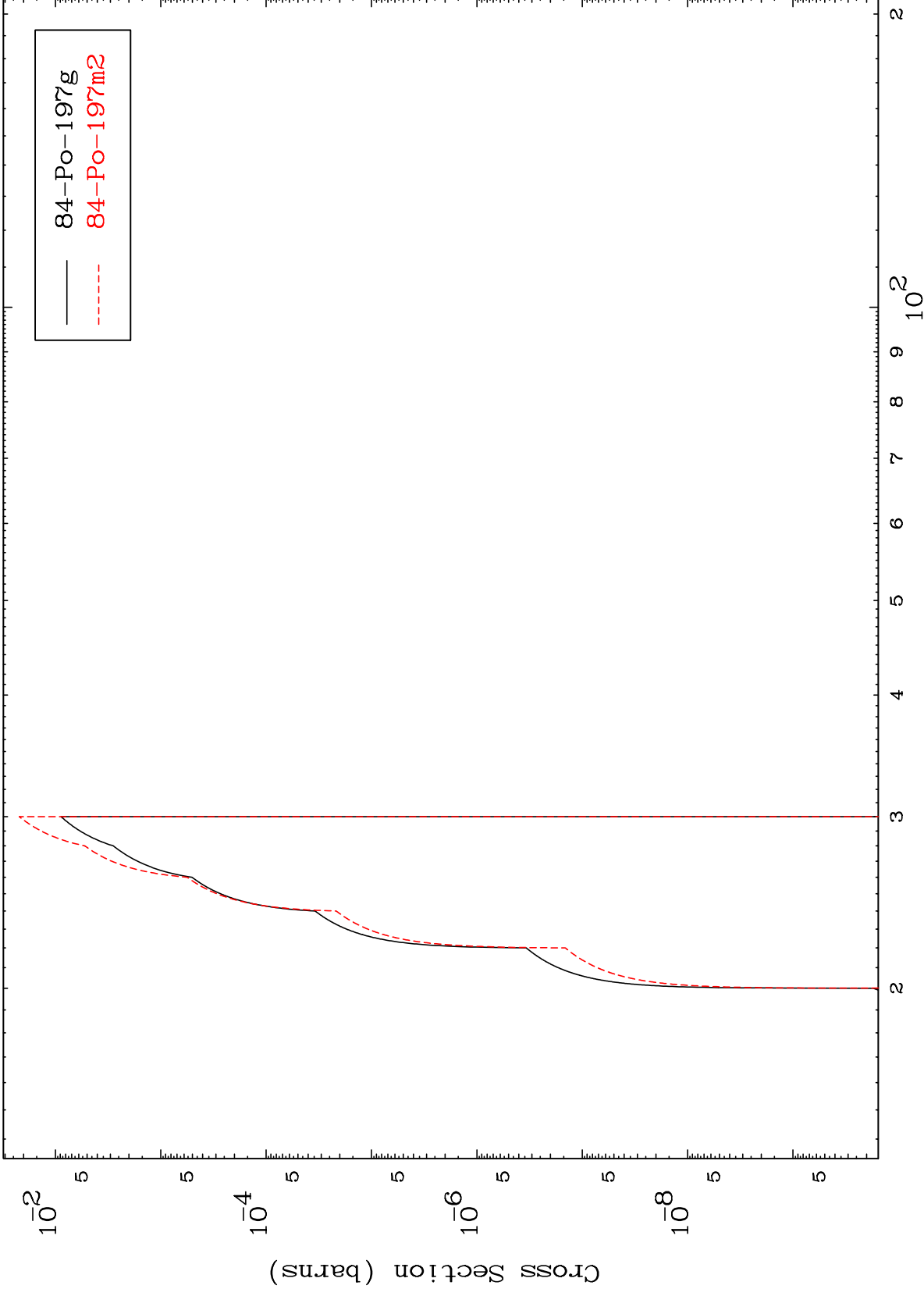


MAT 8293

(n,n') t

83-Bi-198m

Radionuclide Production Cross Section

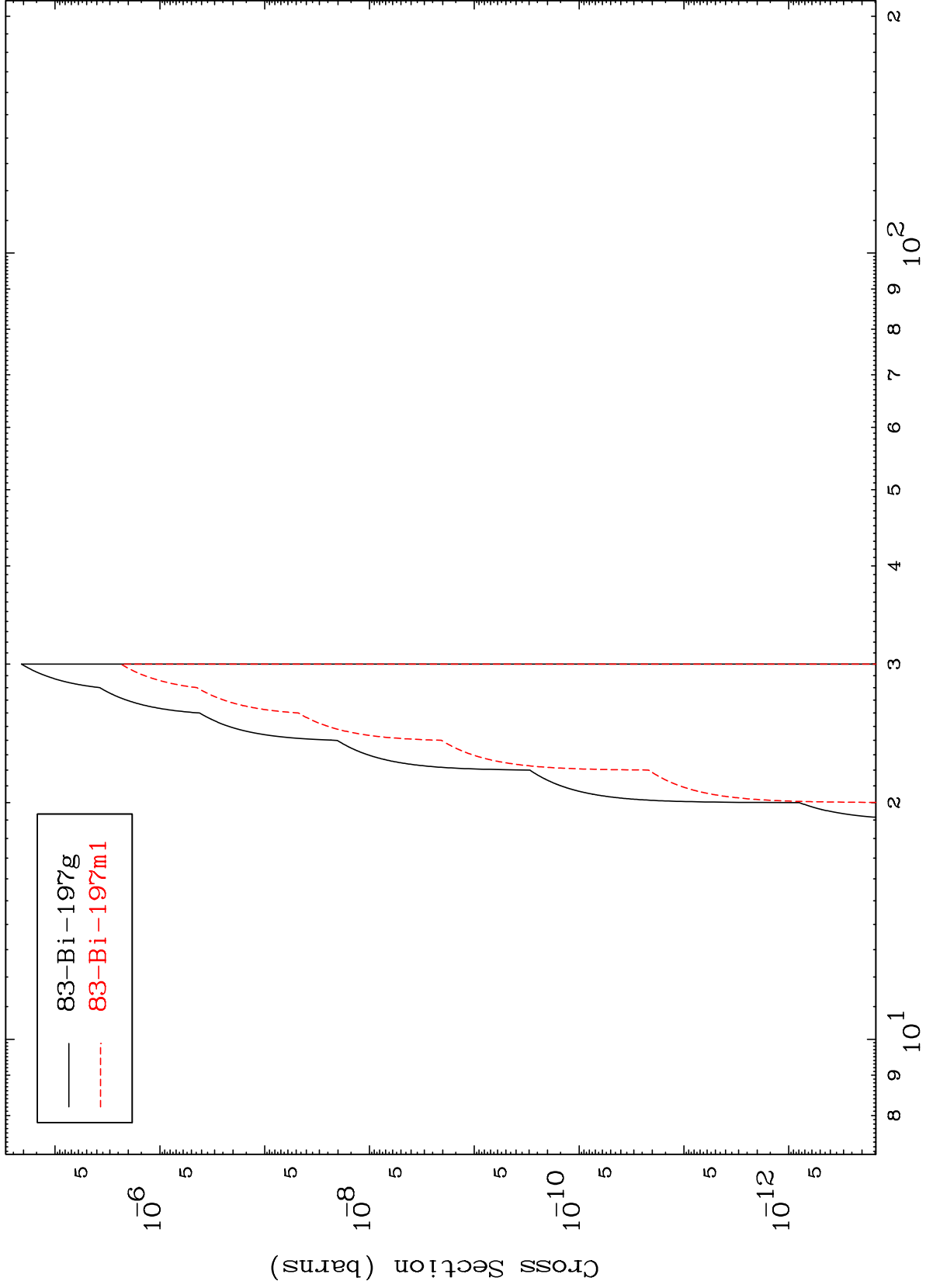


MAT 8293

(n,n') He-3

83-Bi-198m

Radionuclide Production Cross Section

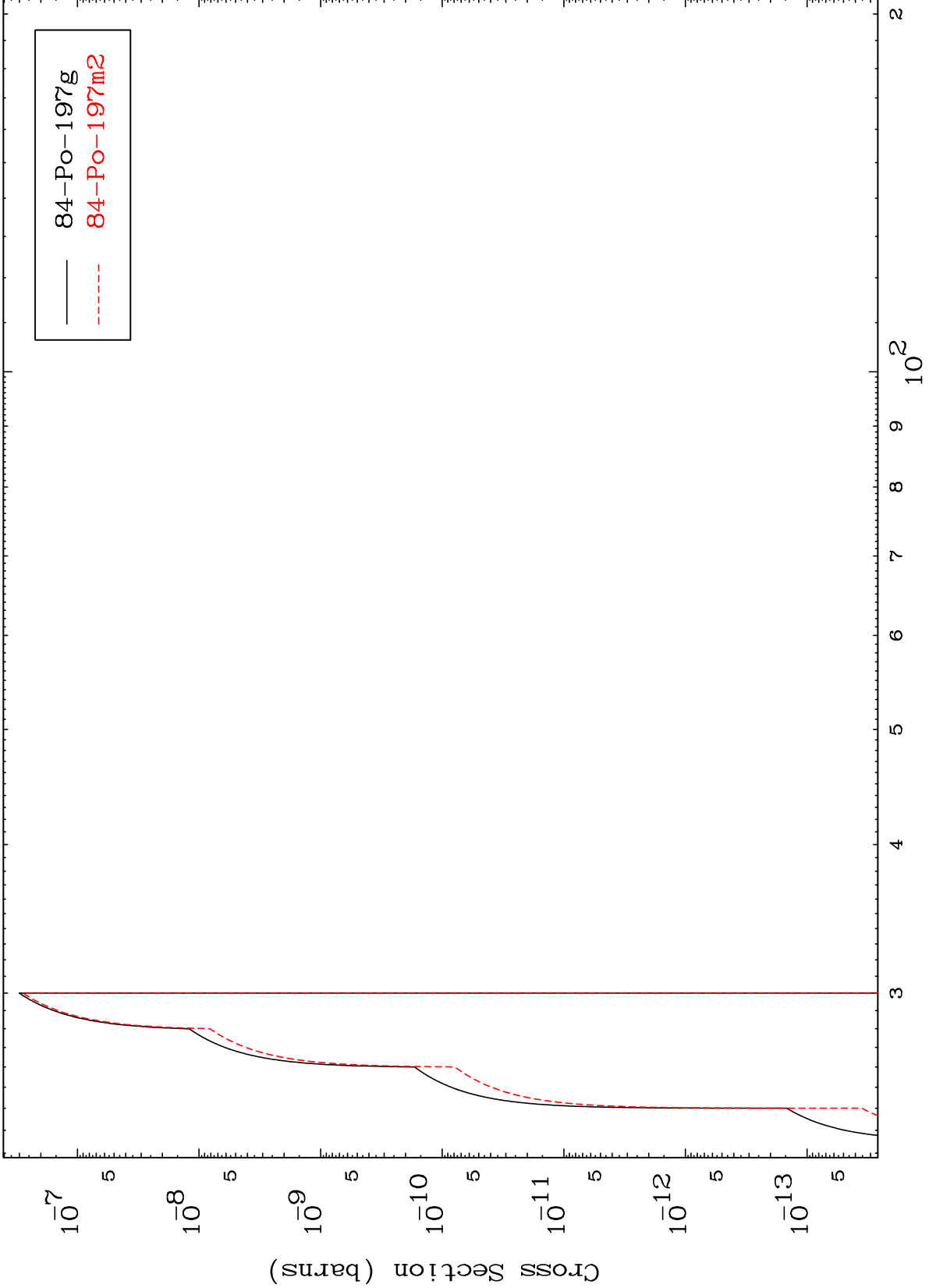


MAT 8293

(n,3n) p

83-Bi-198m

Radionuclide Production Cross Section



23

Incident Energy (MeV)

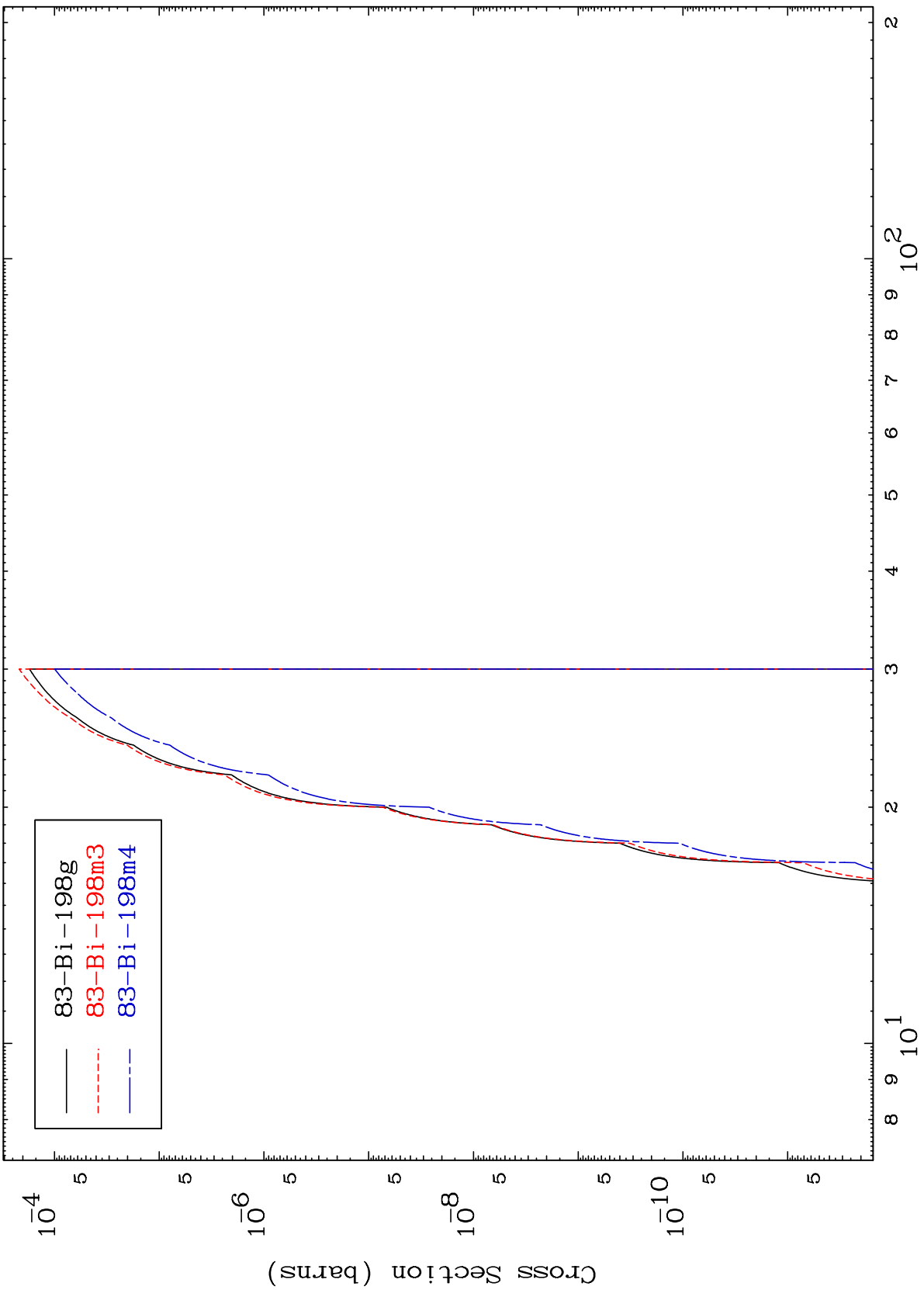
83-Bi-198m

MAT 8293

(n,2n) p

83-Bi-198m

Radionuclide Production Cross Section



24

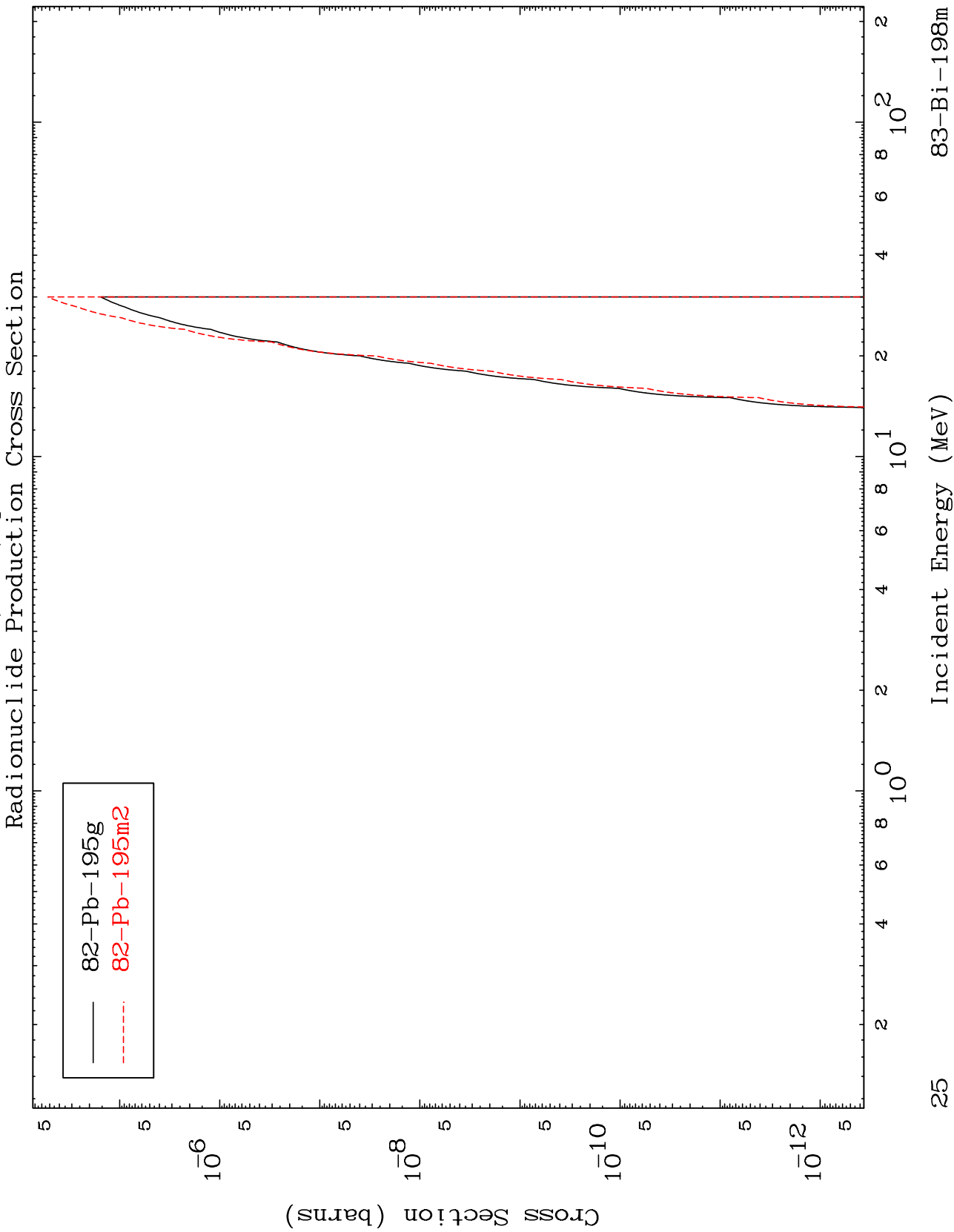
Incident Energy (MeV)

83-Bi-198m

MAT 8293

(n,n') p  $\alpha$

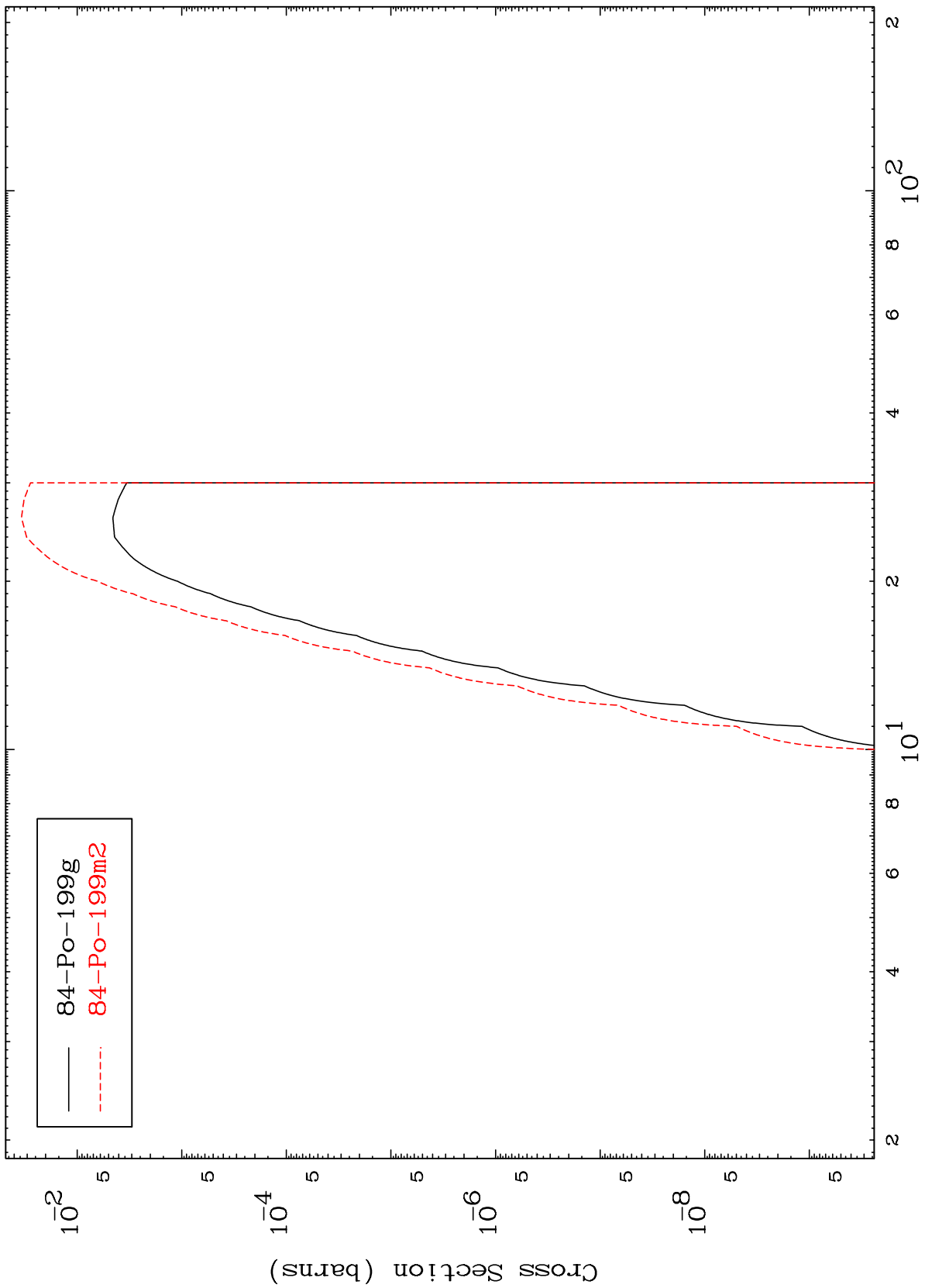
83-Bi-198m



MAT 8293

83-Bi-198m

(n,d)  
Radionuclide Production Cross Section



83-Bi-198m

Incident Energy (MeV)

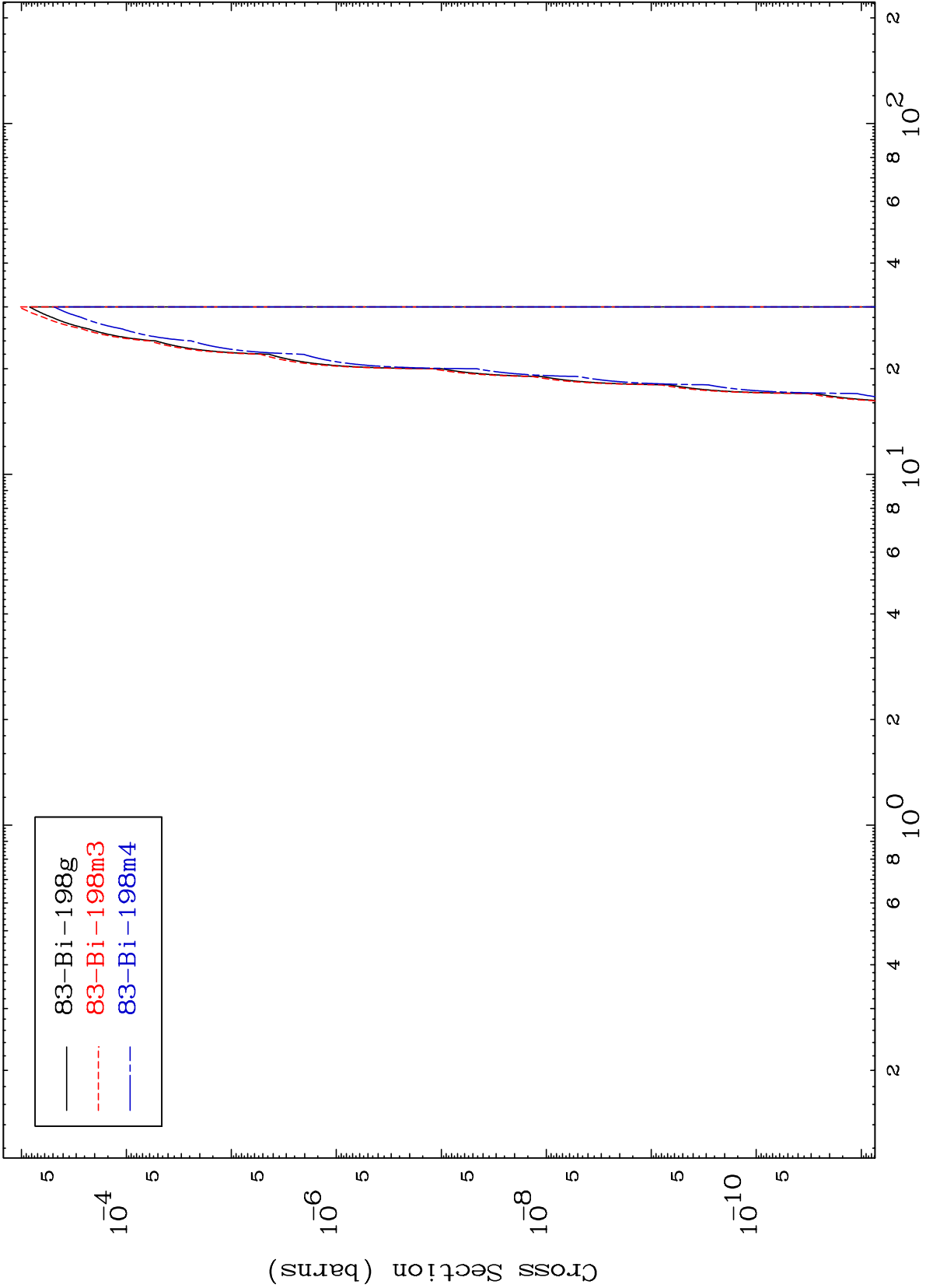
26

MAT 8293

(n,He-3)

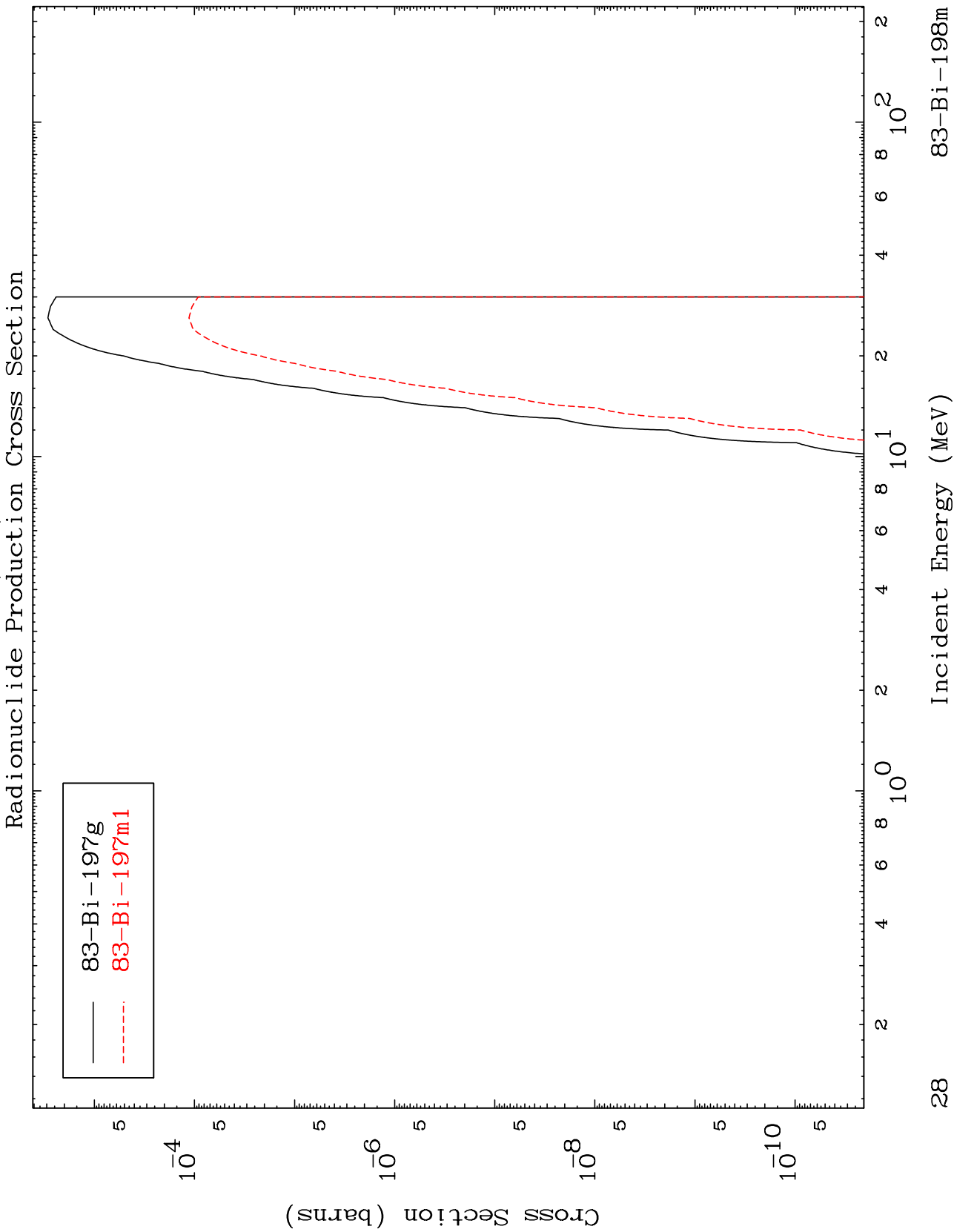
83-Bi-198m

Radionuclide Production Cross Section



MAT 8293

<sup>83</sup>Bi-198m



—	<sup>83</sup> Bi-197g
- - -	<sup>83</sup> Bi-197m1

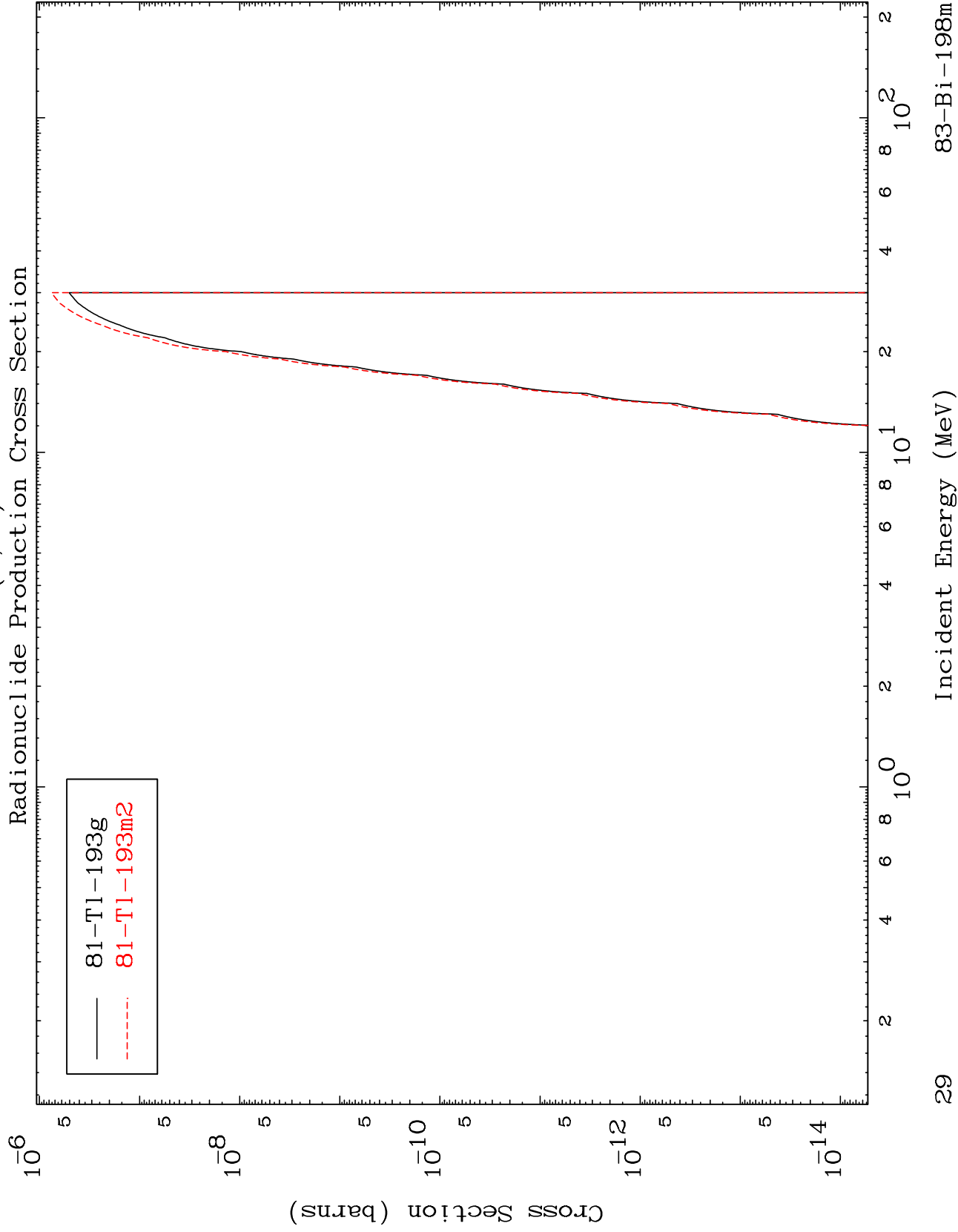
Cross Section (barns)

Incident Energy (MeV)

MAT 8293

(n,2α)

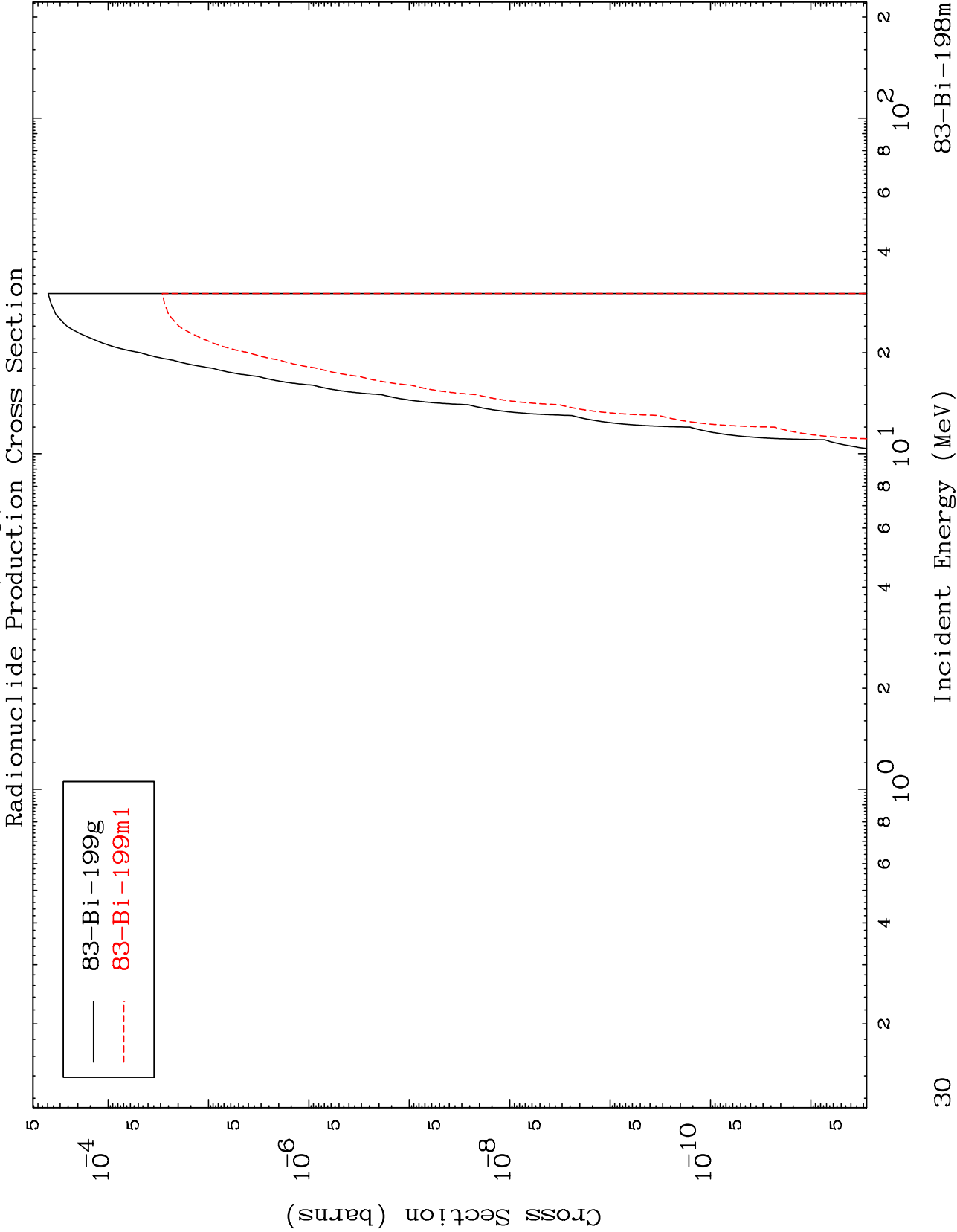
83-Bi-198m



MAT 8293

(n,2p)

<sup>83</sup>Bi-198m



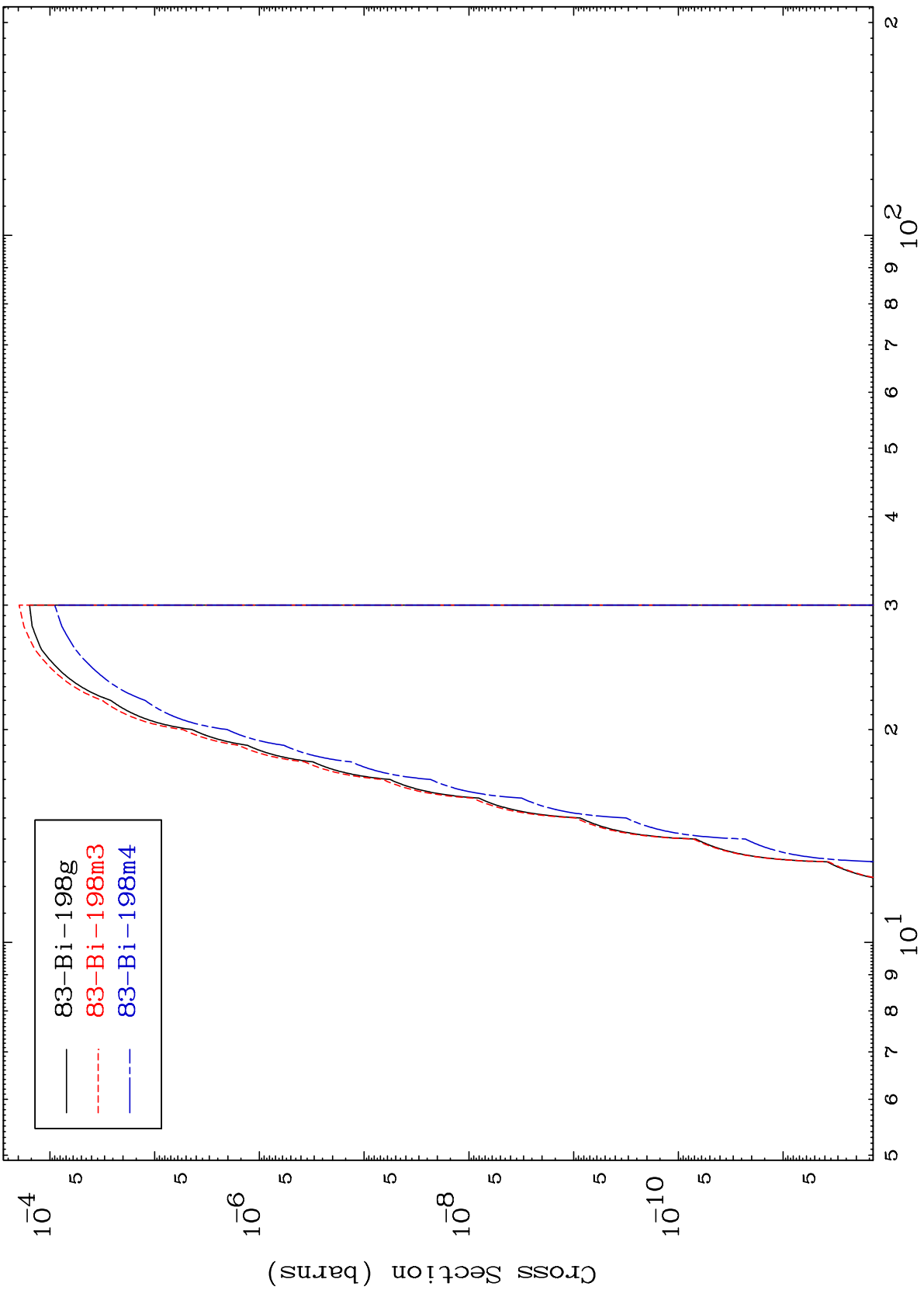
— 83-Bi-199g  
- - - 83-Bi-199m1

MAT 8293

(n,p) d

83-Bi-198m

Radionuclide Production Cross Section

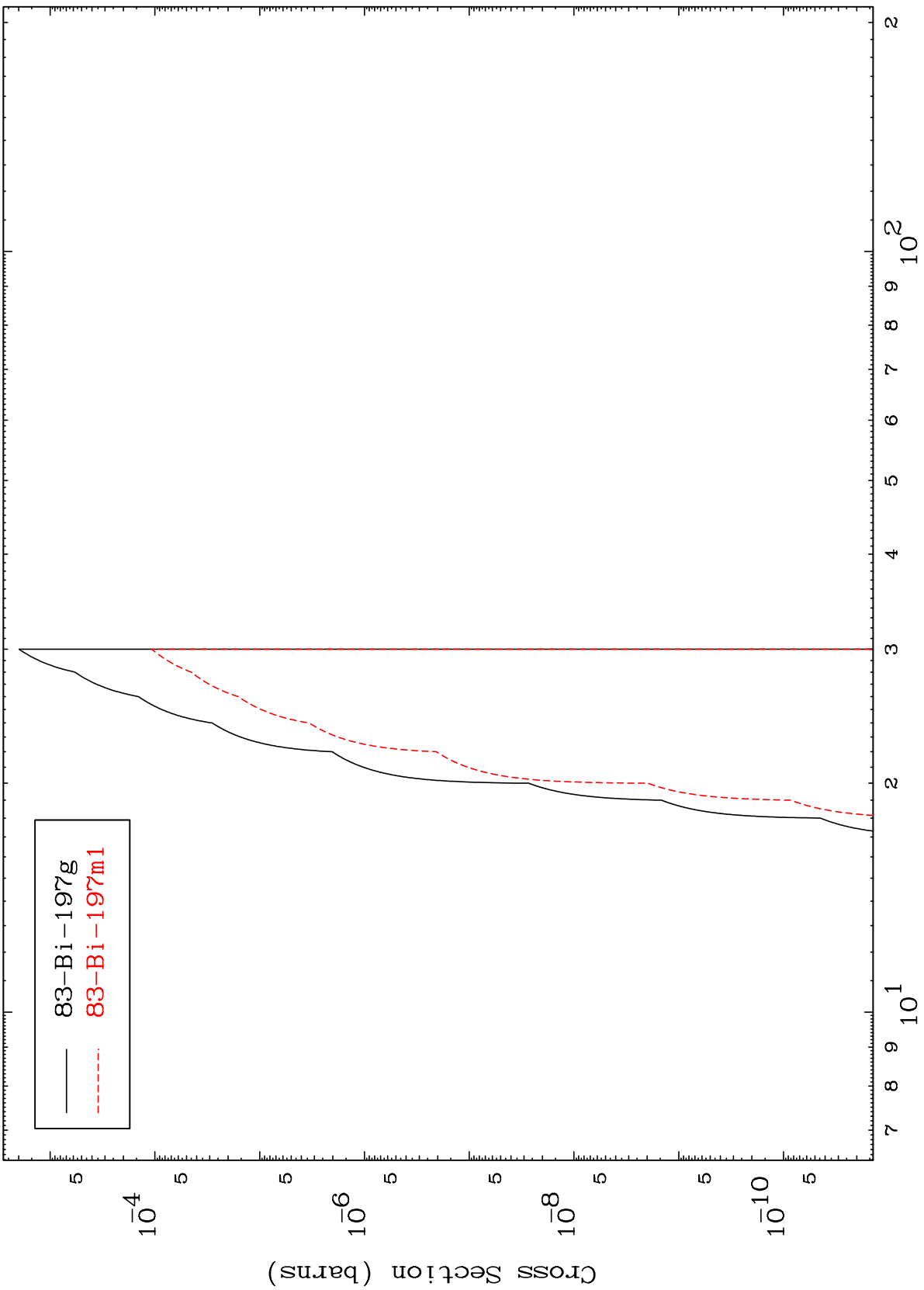


MAT 8293

(n,p) t

83-Bi-198m

Radionuclide Production Cross Section



32

Incident Energy (MeV)

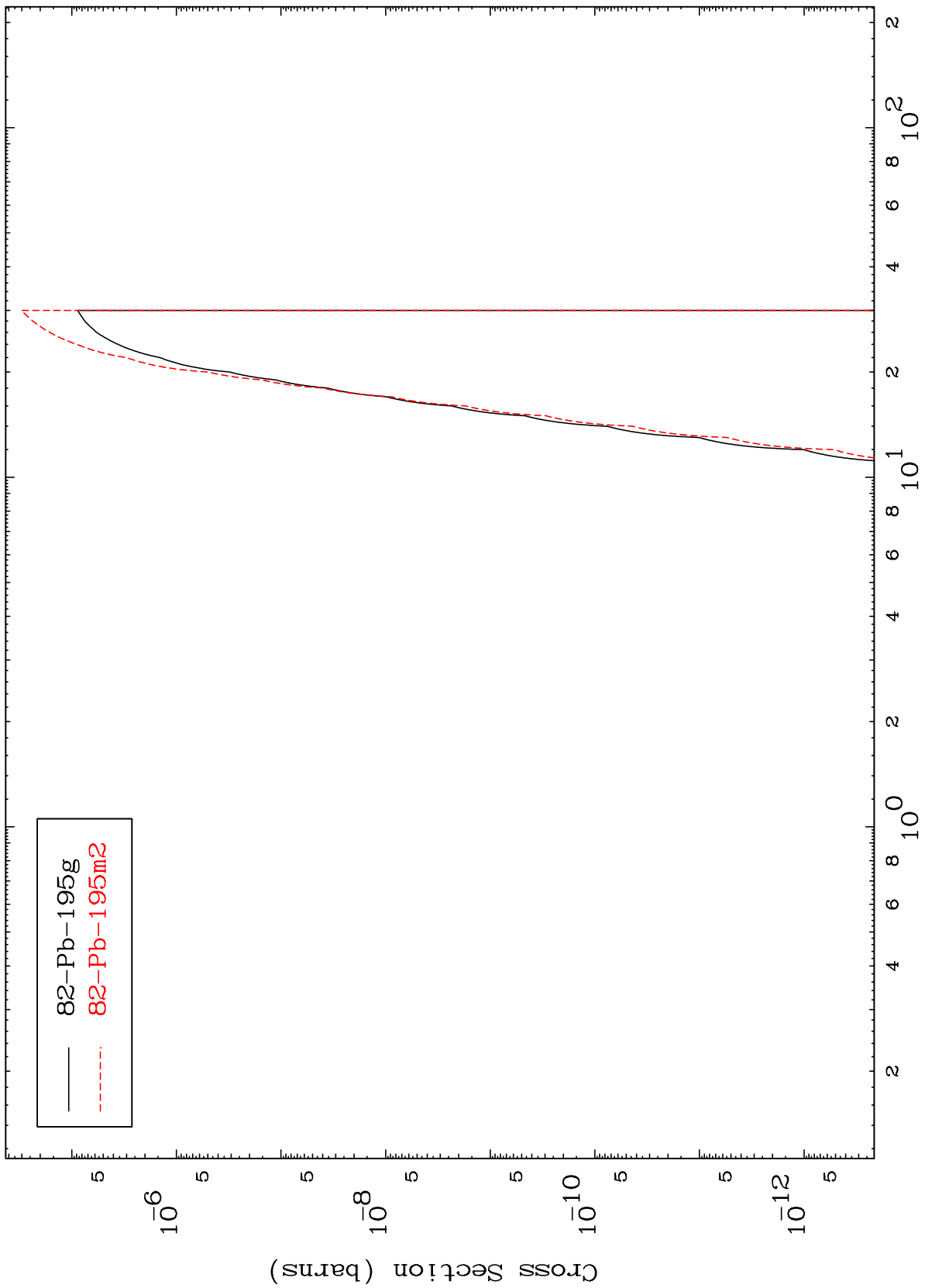
83-Bi-198m

MAT 8293

(n,d)  $\alpha$

83-Bi-198m

Radionuclide Production Cross Section



82-Pb-195g  
82-Pb-195m2