

Program EVALPLOT  
(Version 2021-1)

by

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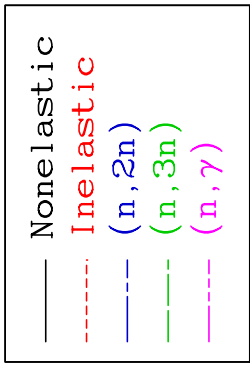
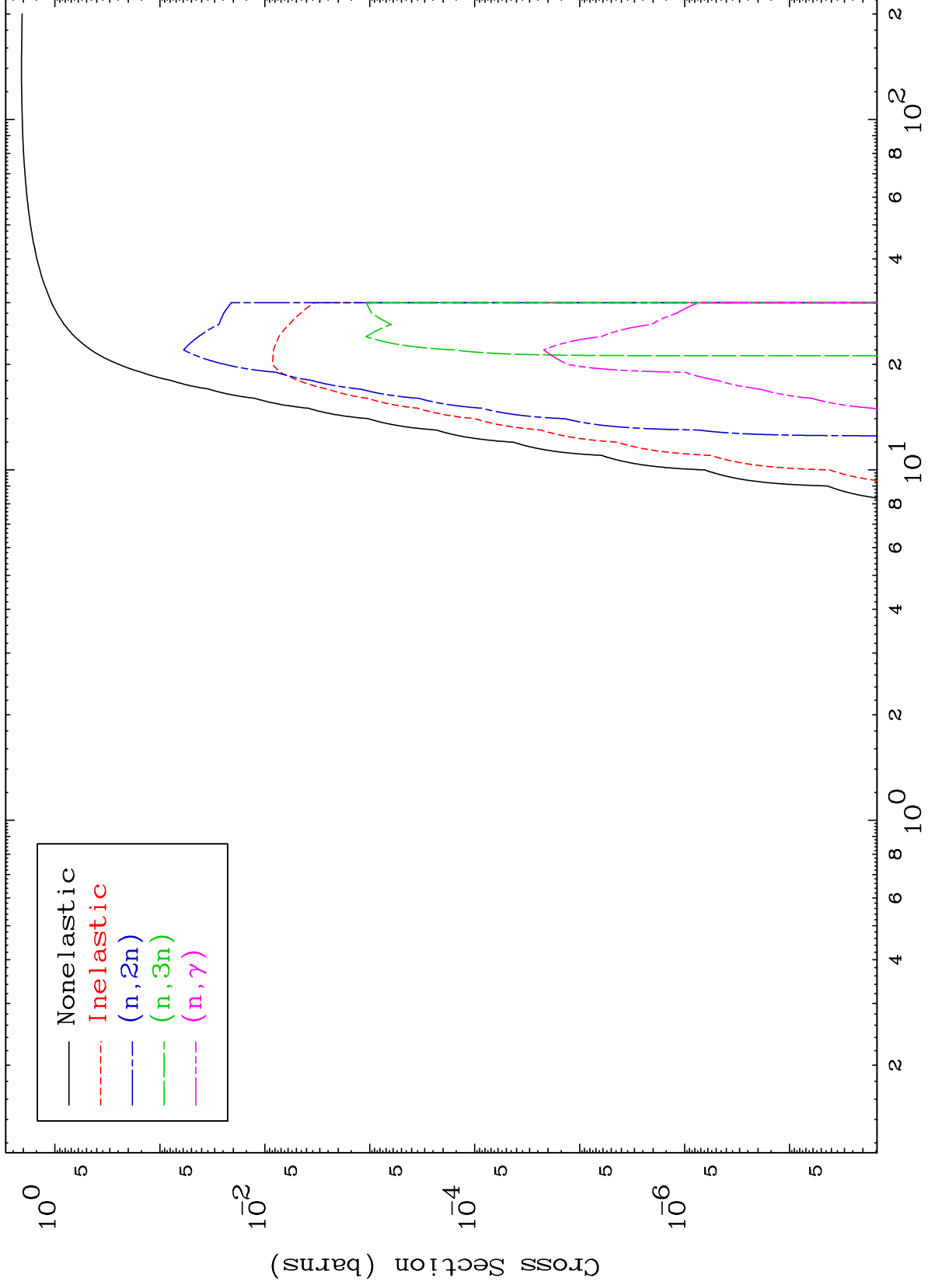
Press Mouse Button to Start

MAT 6607

He-3 Major

0 Kelvin Cross Sections

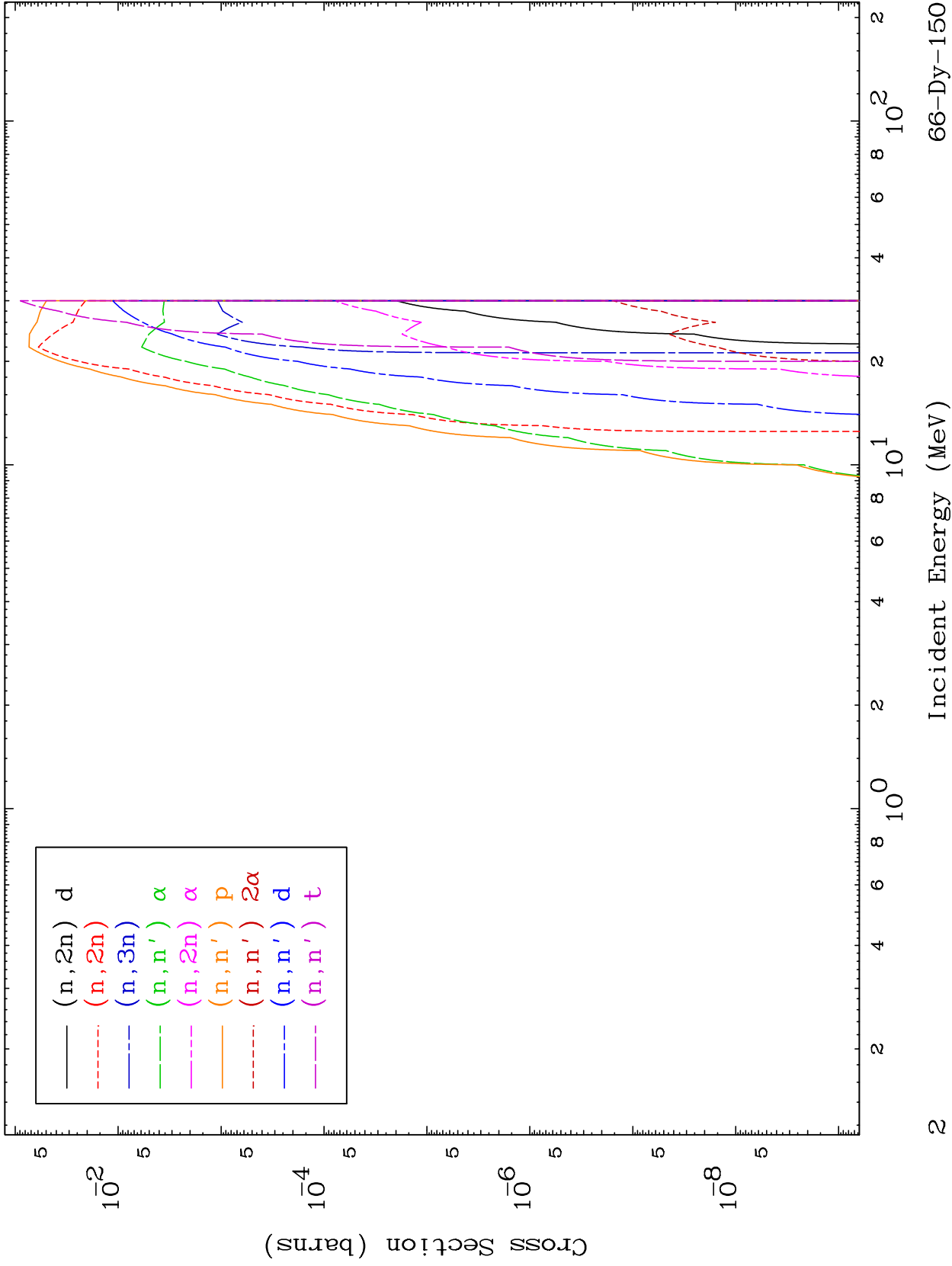
66-Dy-150



MAT 6607

He-3 Neutron Absorption  
0 Kelvin Cross Sections

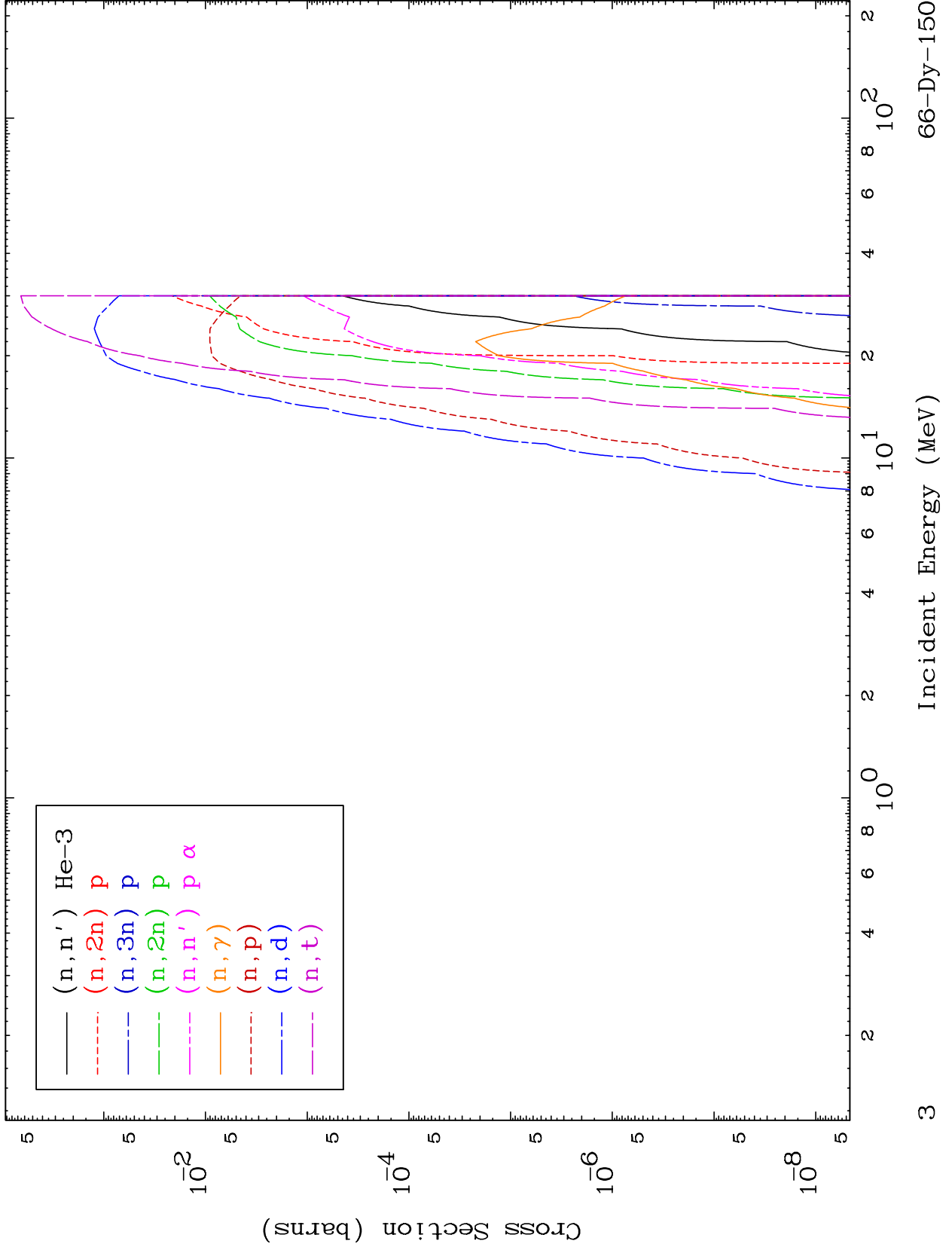
66-Dy-150



MAT 6607

He-3 Neutron Absorption  
0 Kelvin Cross Sections

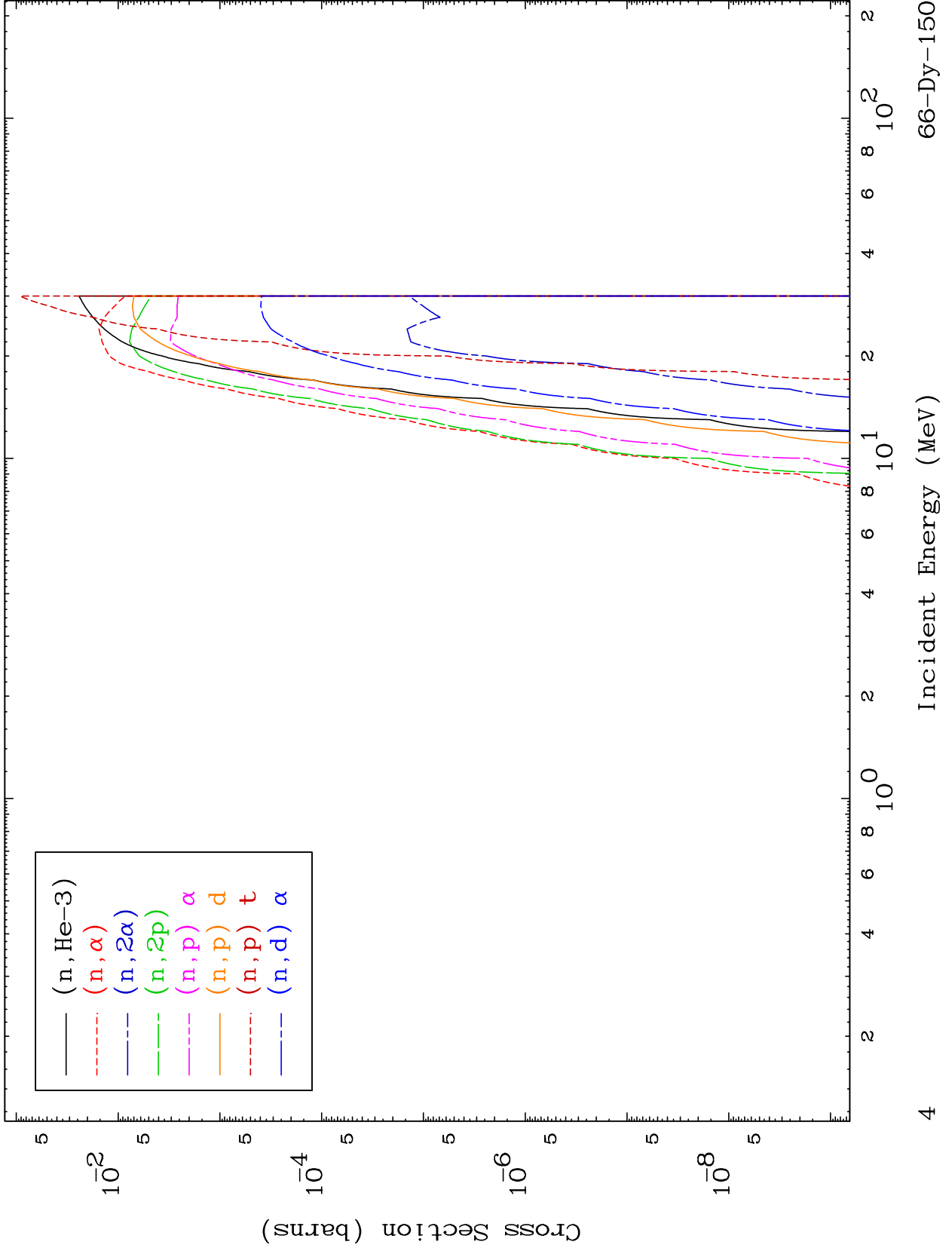
66-Dy-150



MAT 6607

He-3 Neutron Absorption  
0 Kelvin Cross Sections

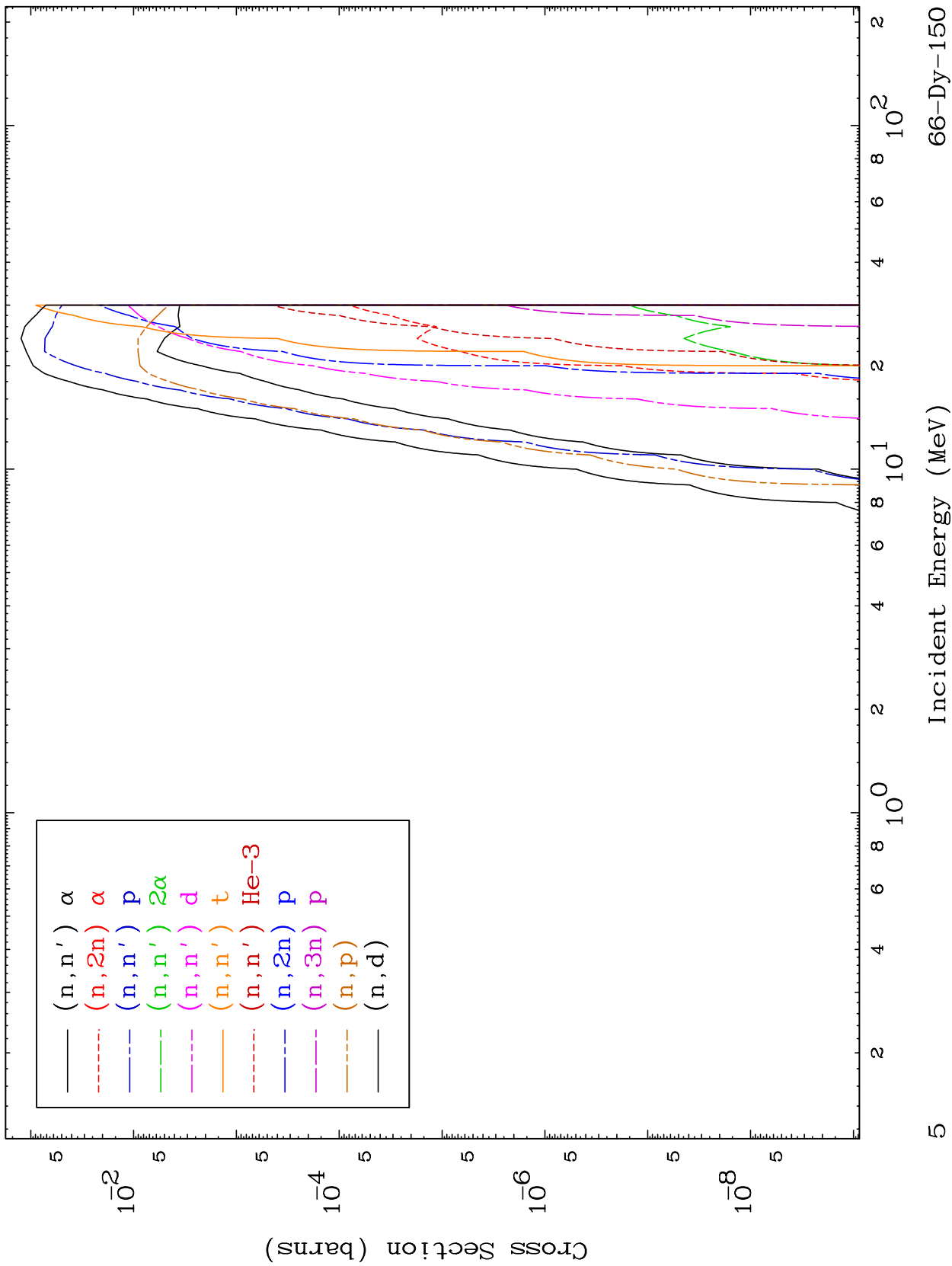
66-Dy-150



MAT 6607

He-3 Charged Particle  
0 Kelvin Cross Sections

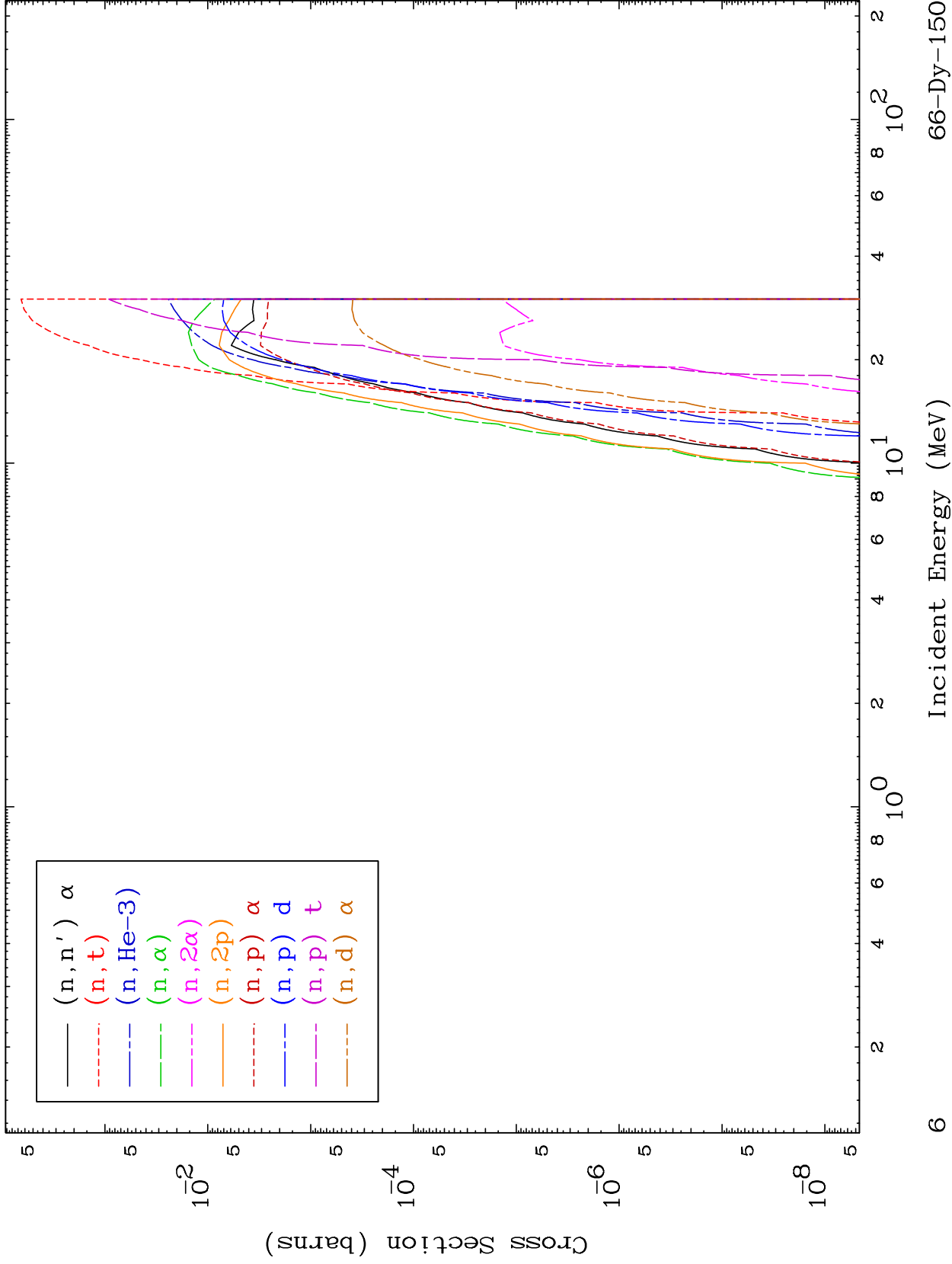
66-Dy-150



MAT 6607

He-3 Charged Particle  
0 Kelvin Cross Sections

66-Dy-150

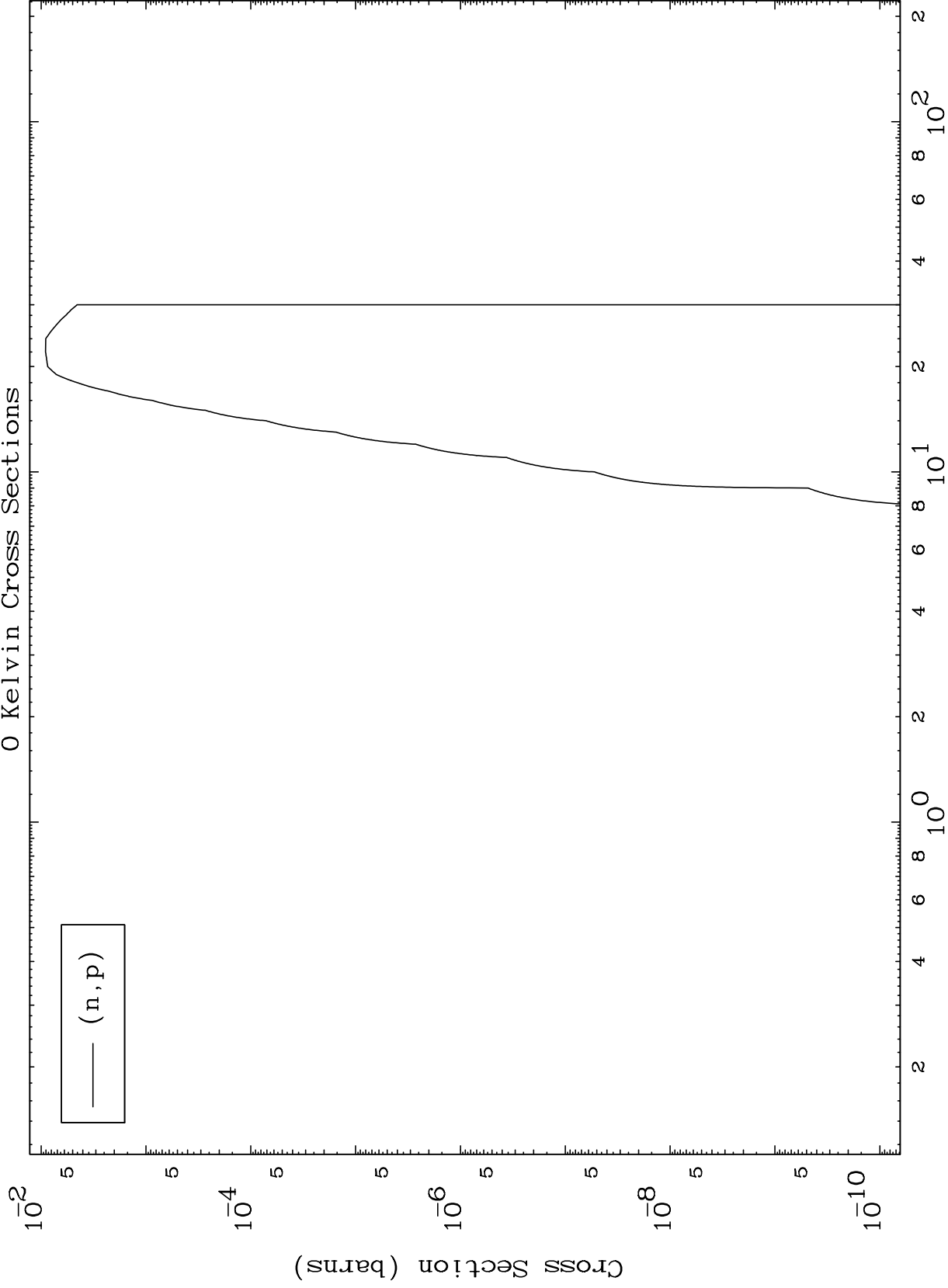


MAT 6607

(He-3,p) Levels

66-Dy-150

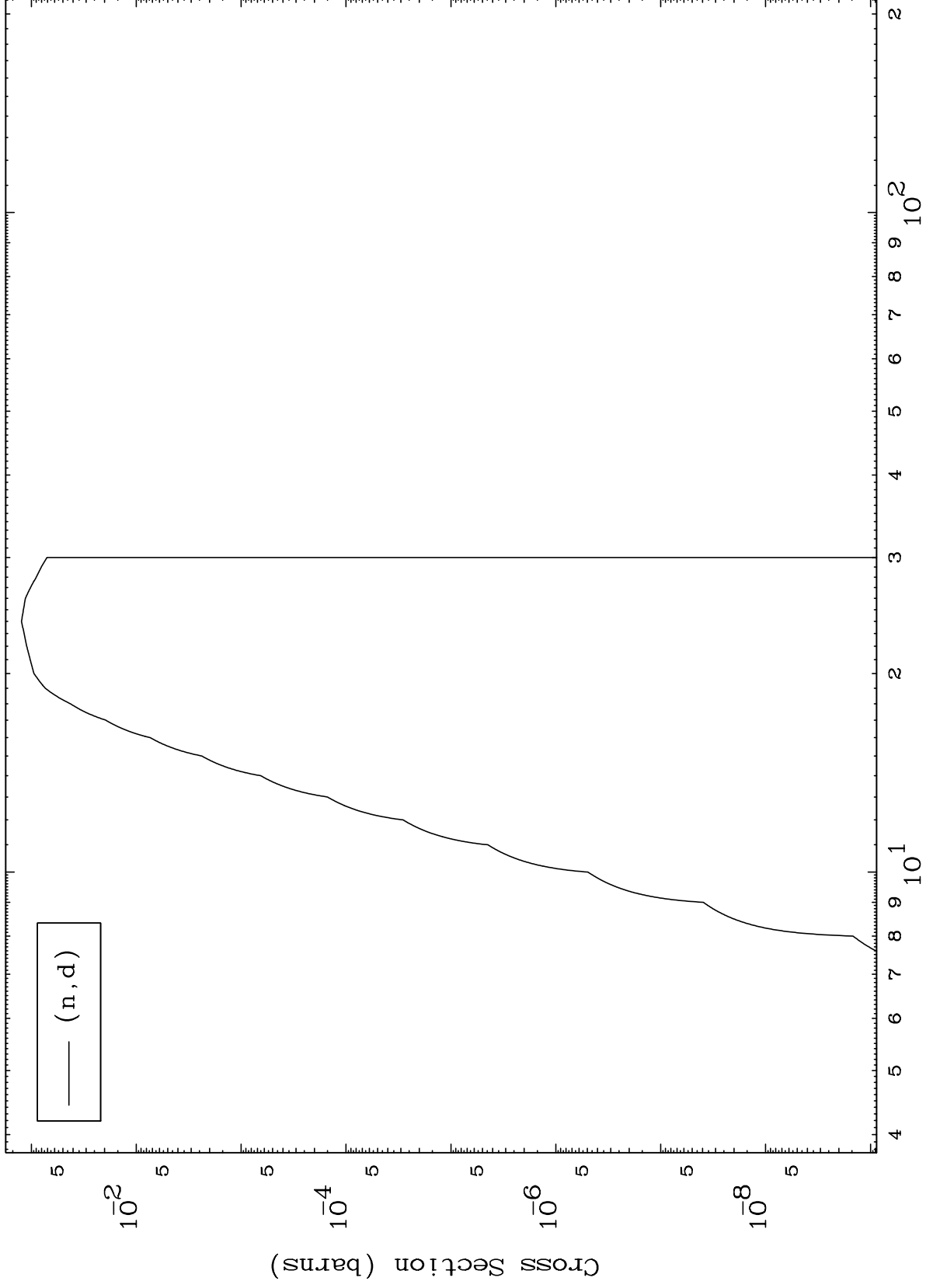
0 Kelvin Cross Sections



MAT 6607

(He-3,d) Levels  
0 Kelvin Cross Sections

66-Dy-150



8

Incident Energy (MeV)

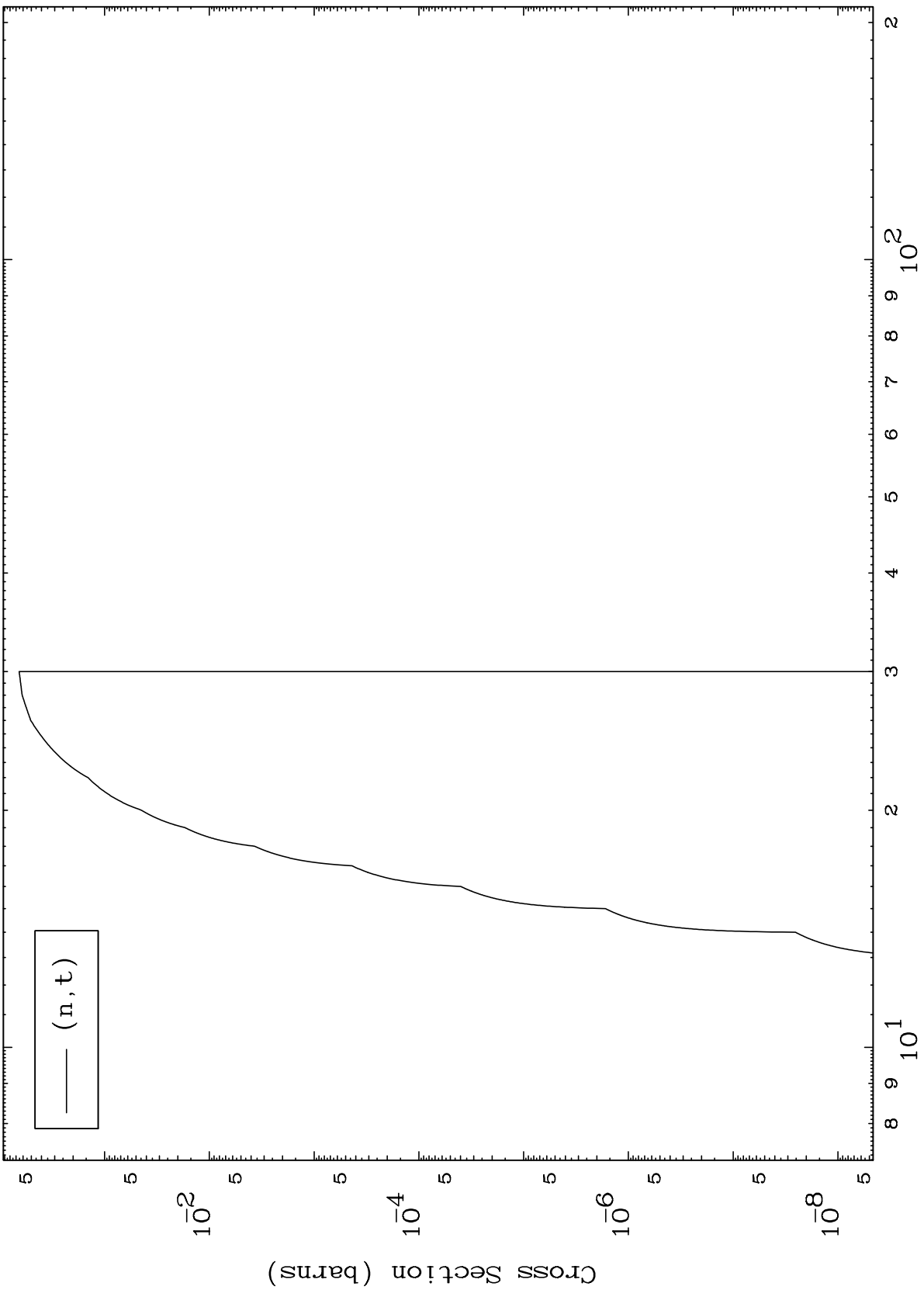
66-Dy-150

MAT 6607

(He-3,t) Levels

66-Dy-150

0 Kelvin Cross Sections



(n, t)

9

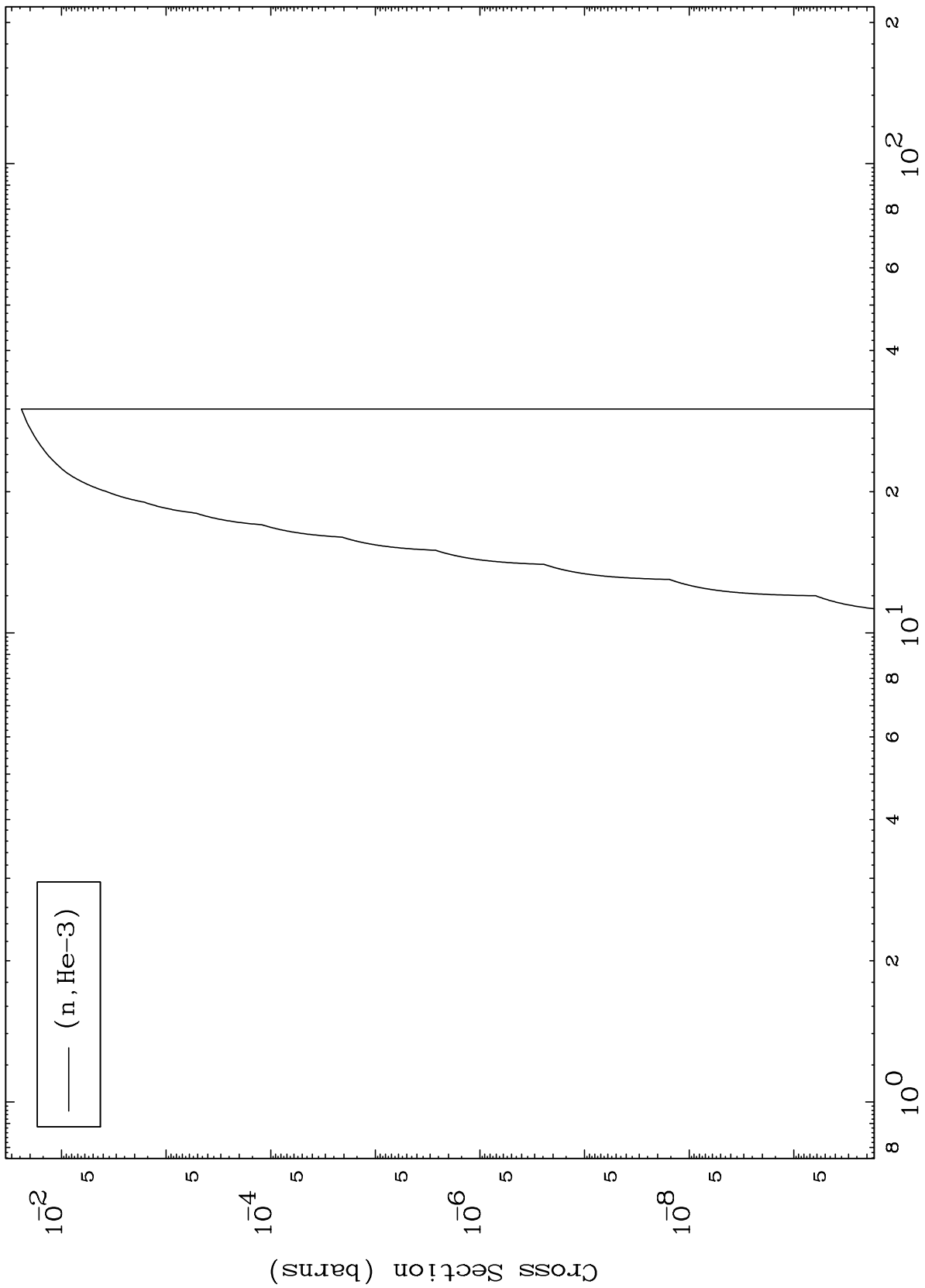
Incident Energy (MeV)

66-Dy-150

MAT 6607

(He-3, He3) Levels  
0 Kelvin Cross Sections

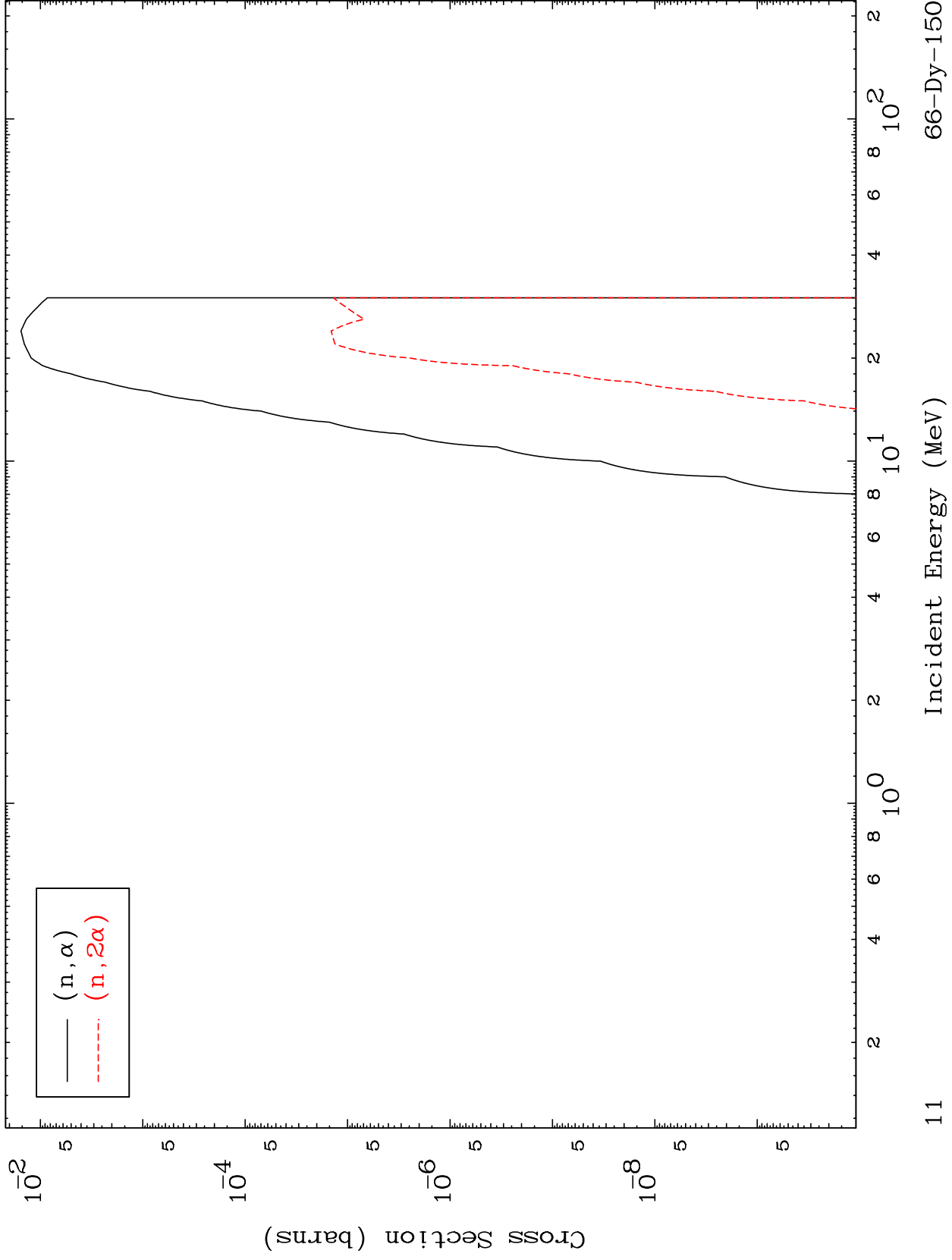
66-Dy-150



MAT 6607

(He-3,  $\alpha$ ) Levels  
0 Kelvin Cross Sections

66-Dy-150

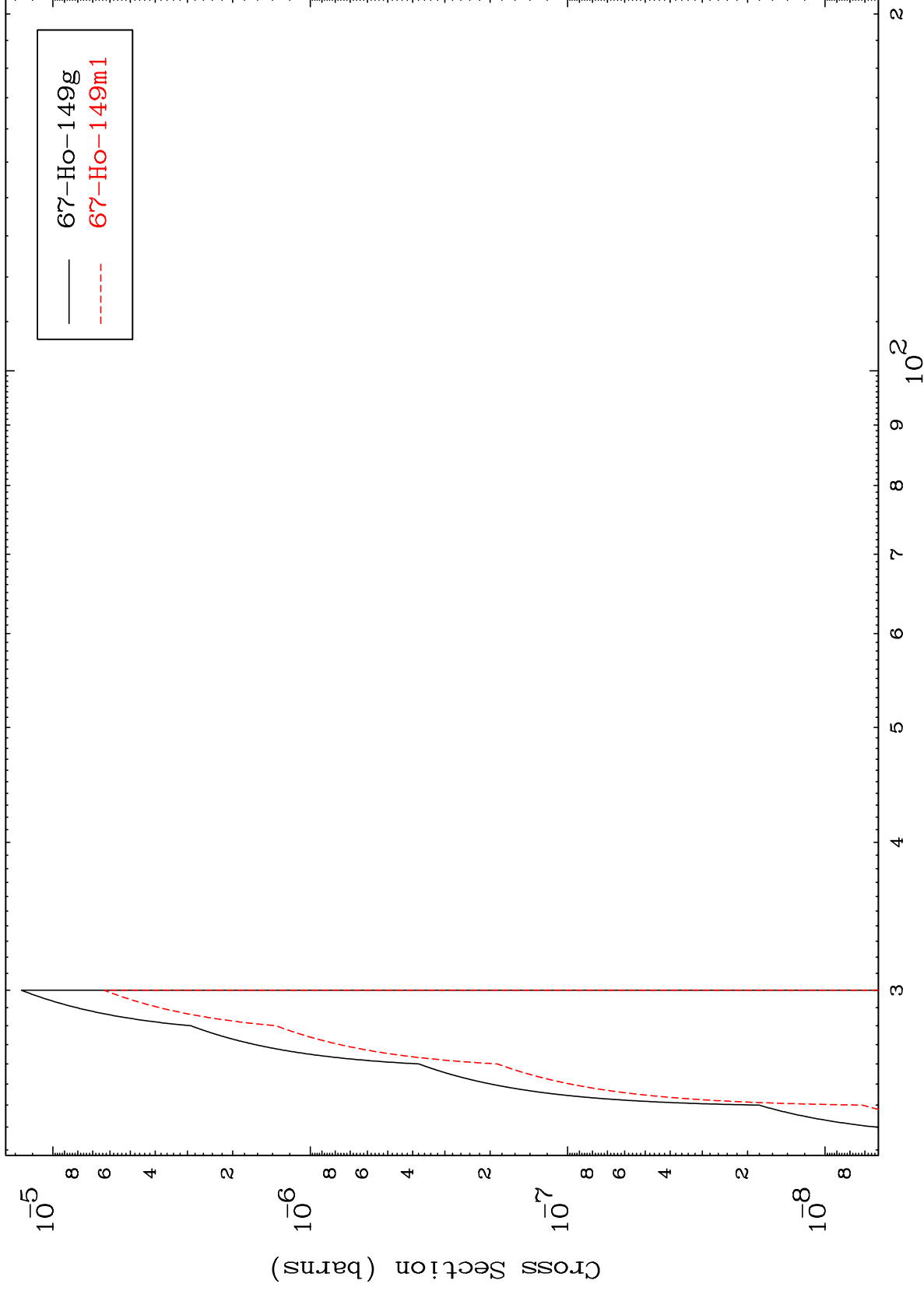


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(n,2n) d

66-Dy-150

Radionuclide Production Cross Section



12

Incident Energy (MeV)

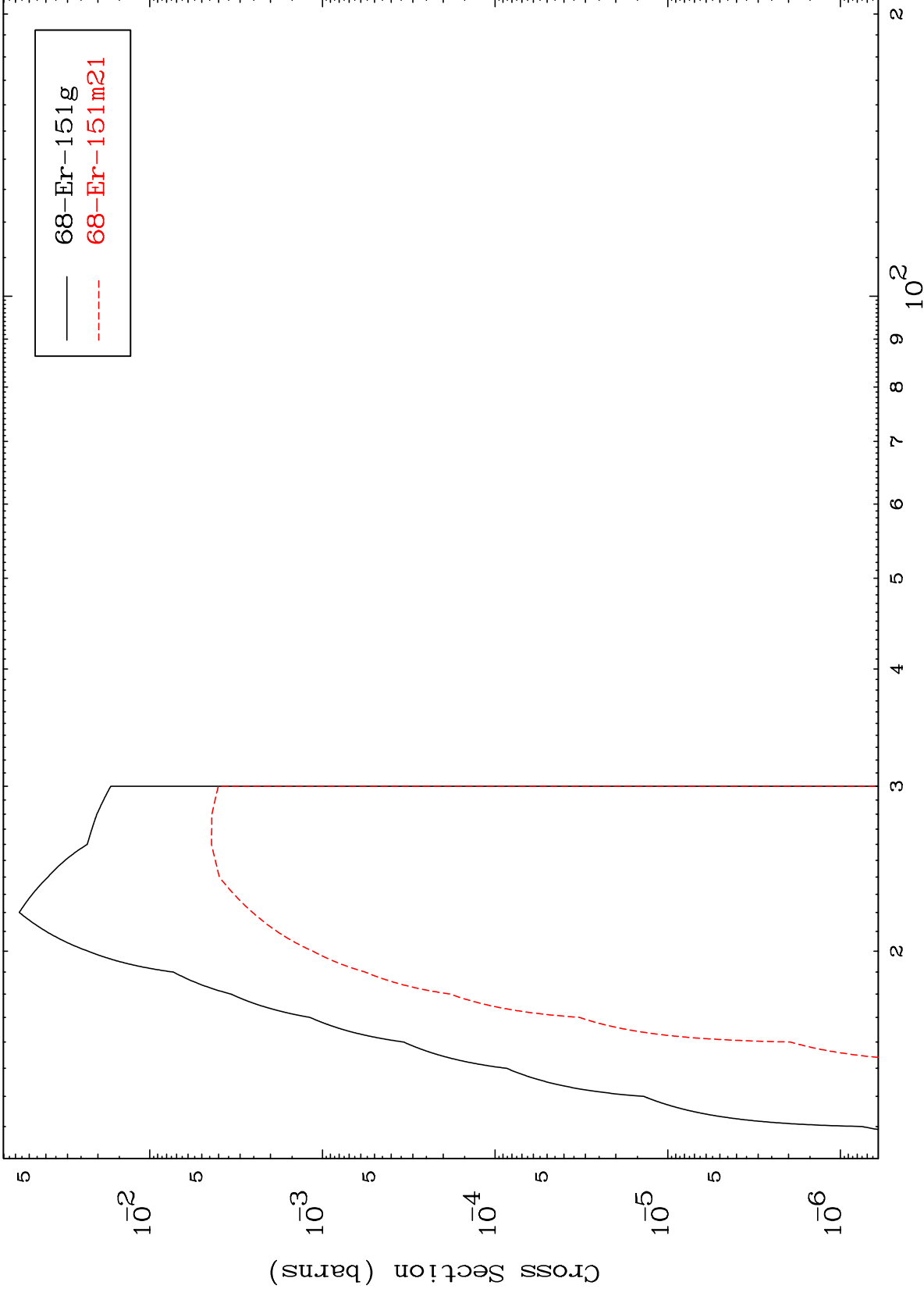
66-Dy-150

MAT 6607

(n,2n)

66-Dy-150

Radionuclide Production Cross Section



13

Incident Energy (MeV)

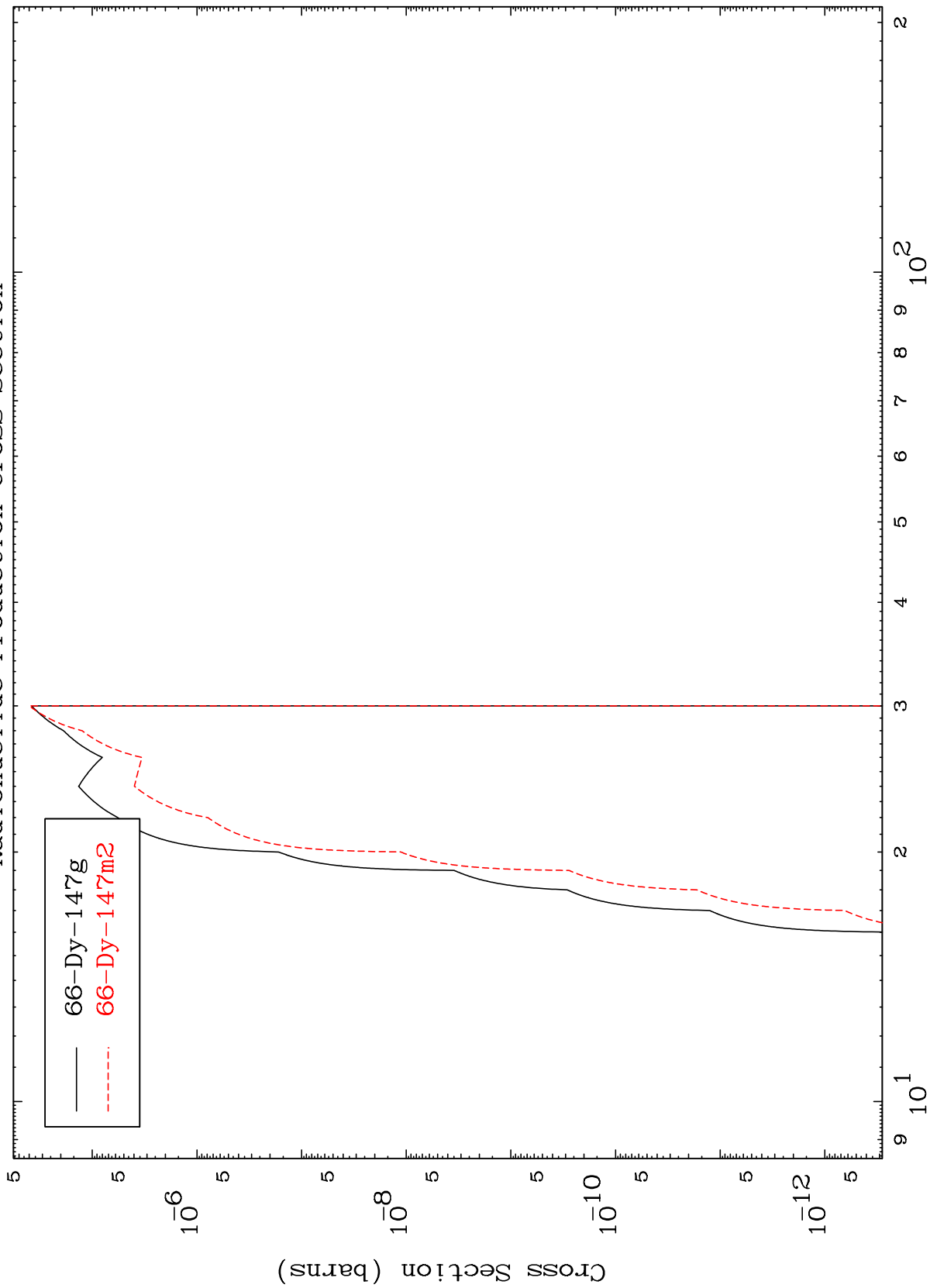
66-Dy-150

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(n,2n)  $\alpha$

66-Dy-150

Radionuclide Production Cross Section



Incident Energy (MeV)

66-Dy-150

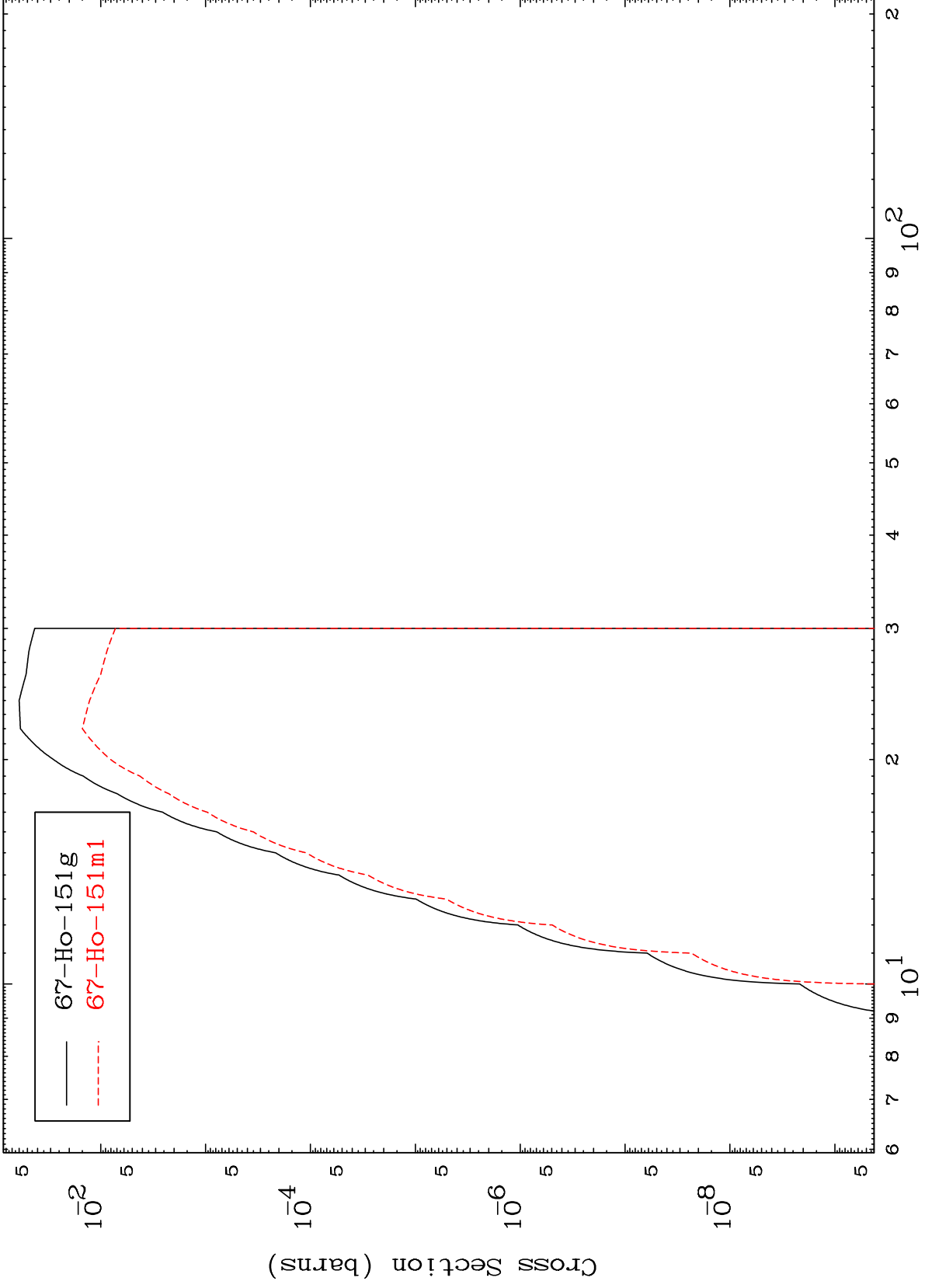
14

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(n,n') p

66-Dy-150

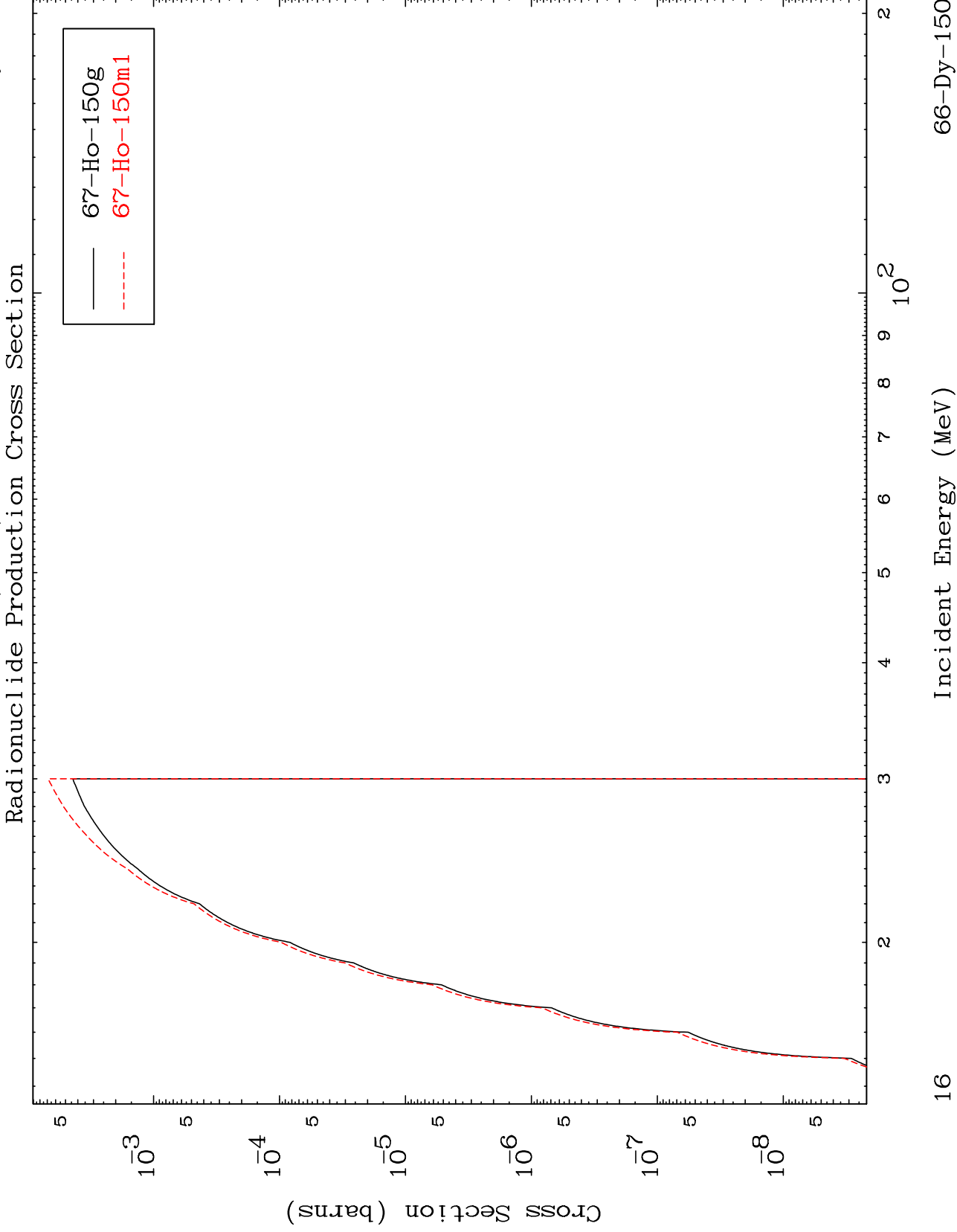
Radionuclide Production Cross Section



15

Incident Energy (MeV)

66-Dy-150

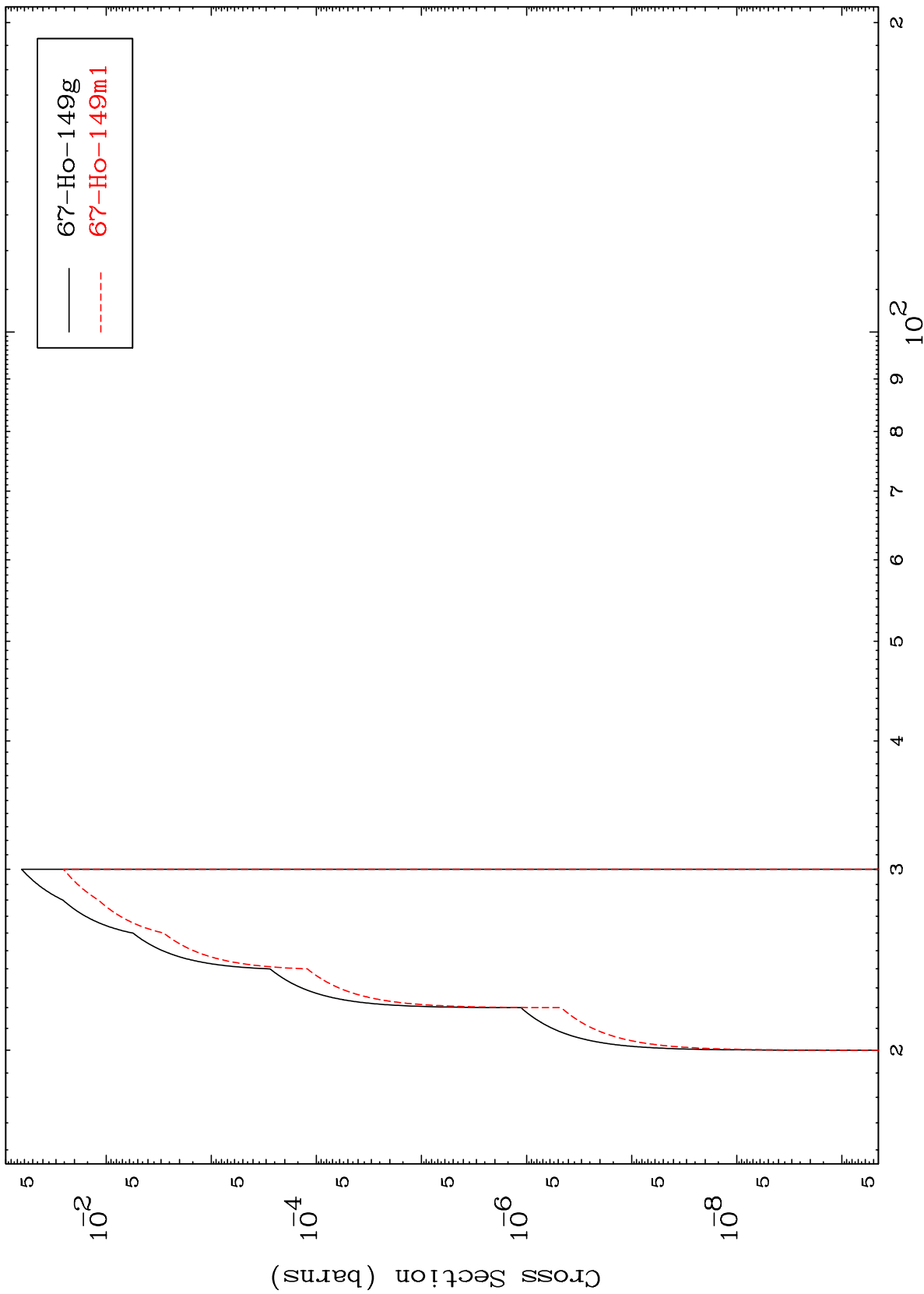


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(n,n') t

66-Dy-150

Radionuclide Production Cross Section



17

Incident Energy (MeV)

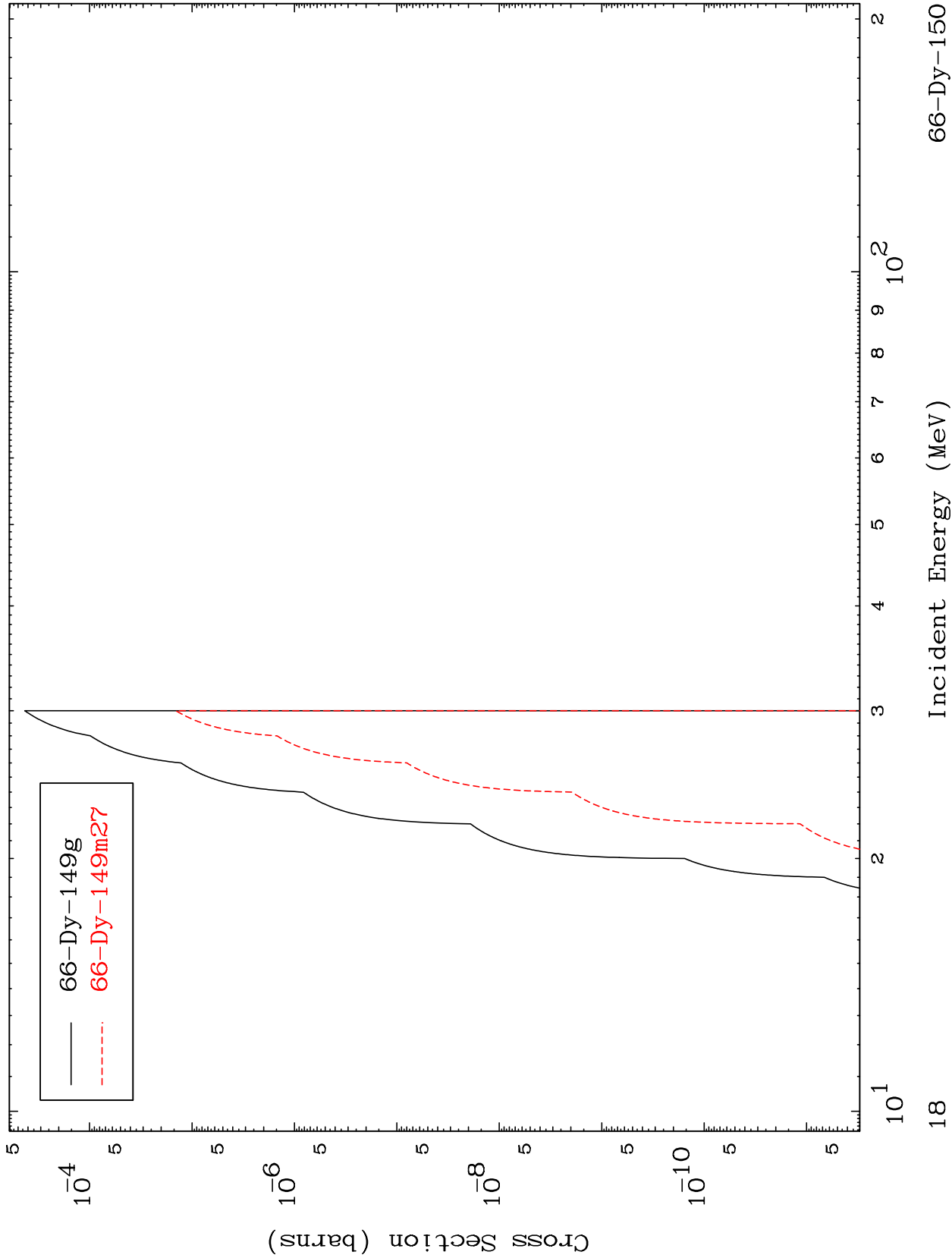
66-Dy-150

MAT 6607

(n,n') He-3

66-Dy-150

Radionuclide Production Cross Section

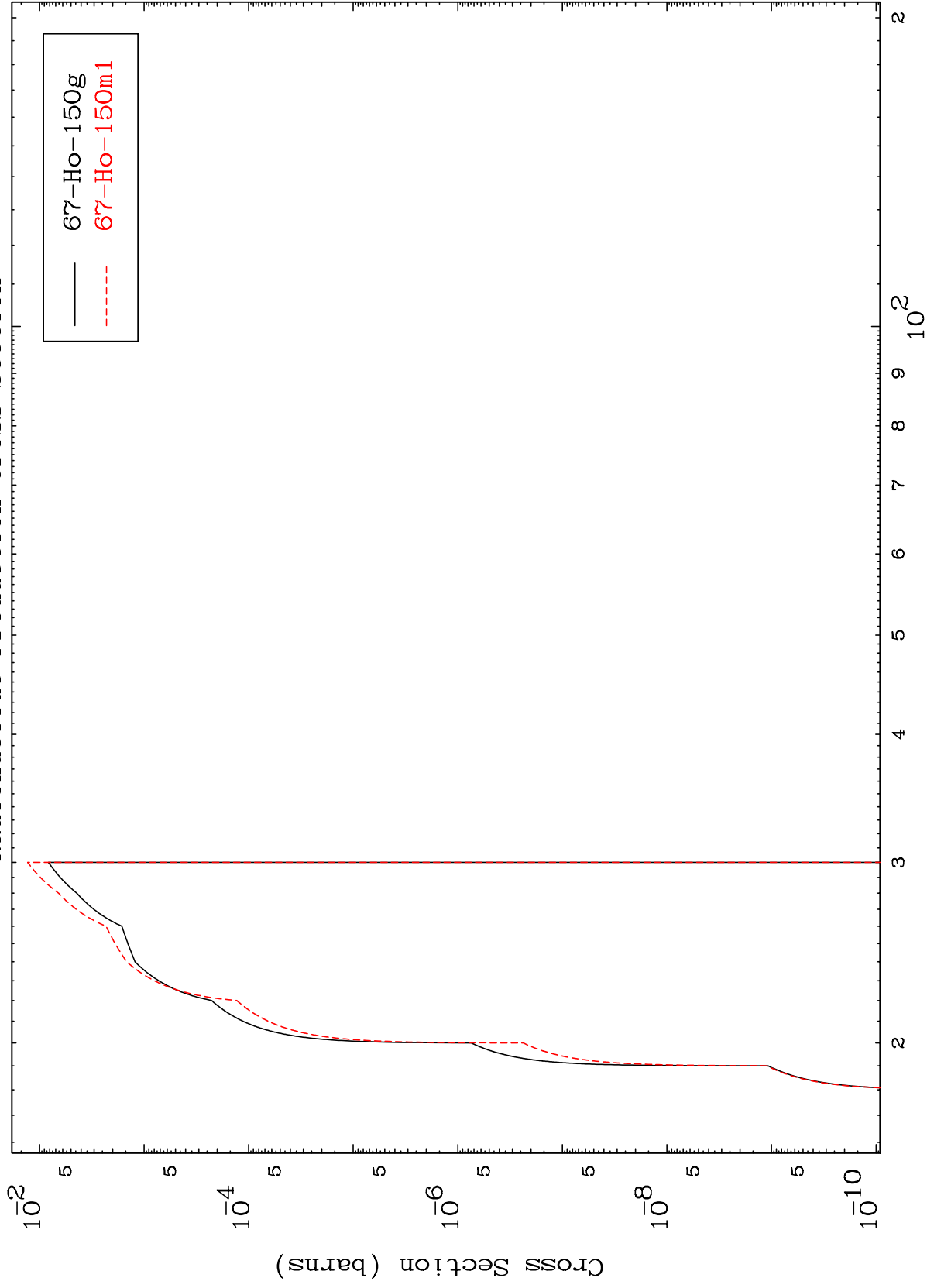


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(n,2n) p

66-Dy-150

Radionuclide Production Cross Section



19

Incident Energy (MeV)

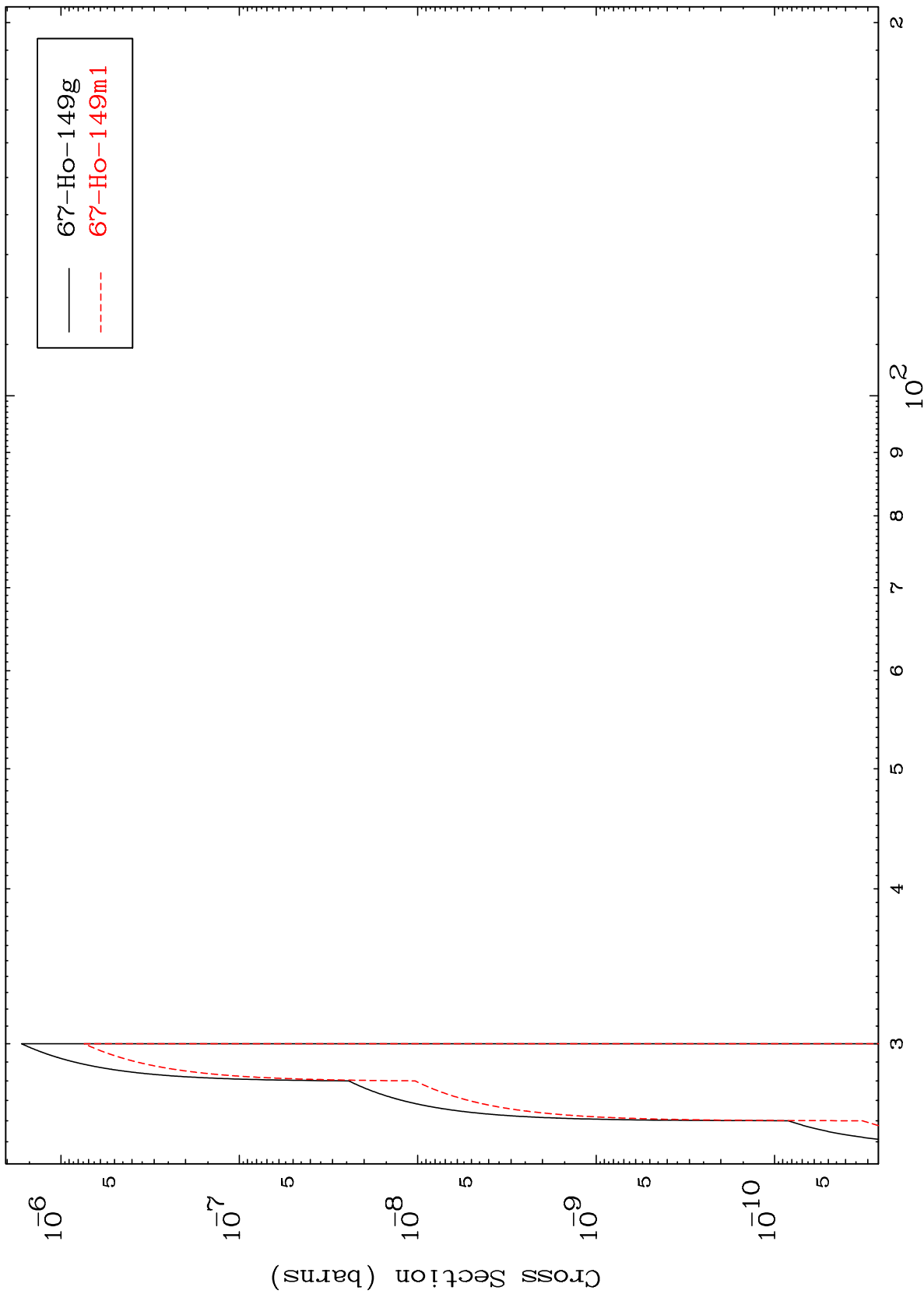
66-Dy-150

MAT 6607

(n,3n) p

66-Dy-150

Radionuclide Production Cross Section



20

Incident Energy (MeV)

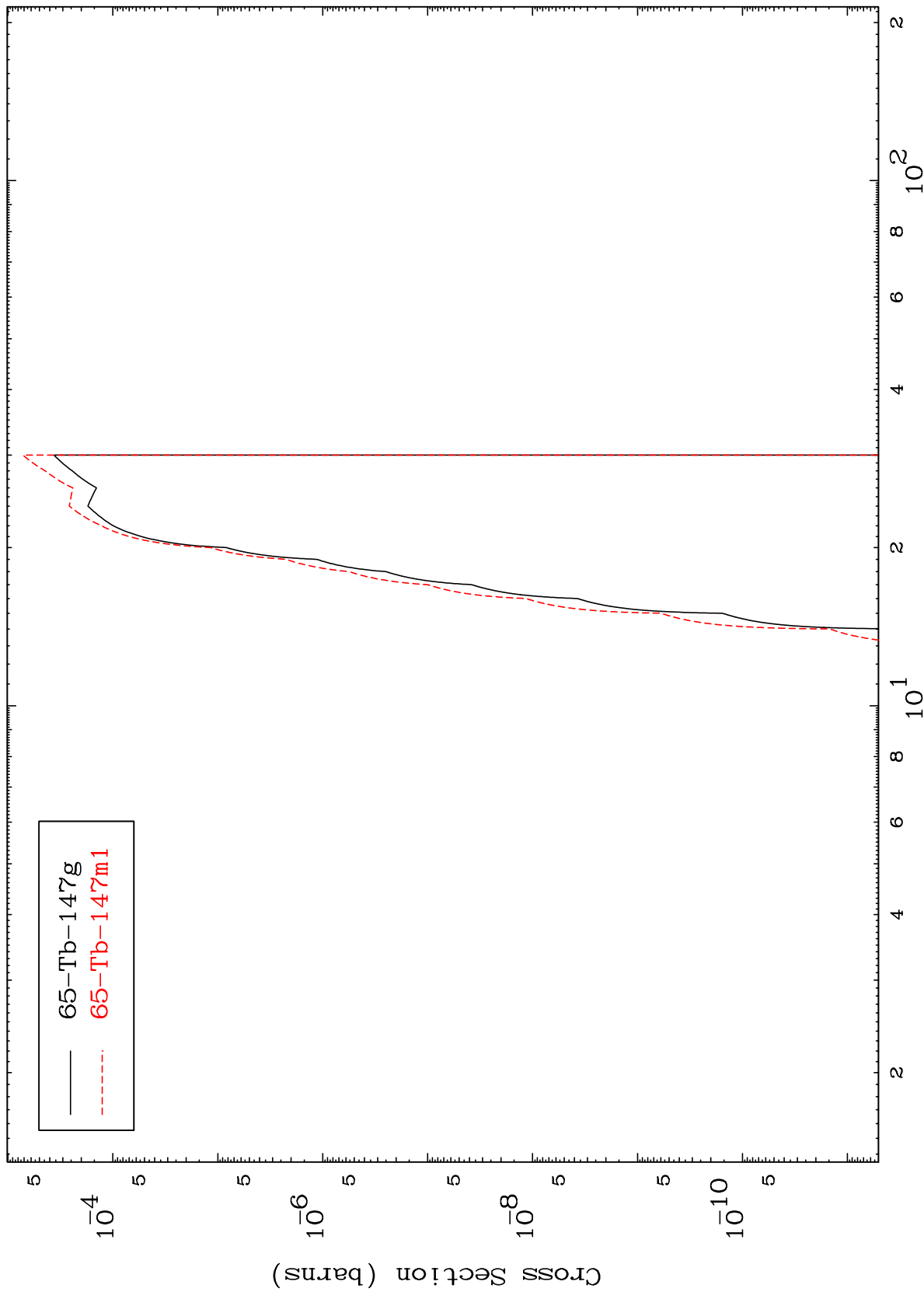
66-Dy-150

MAT 6607

(n,n') p  $\alpha$

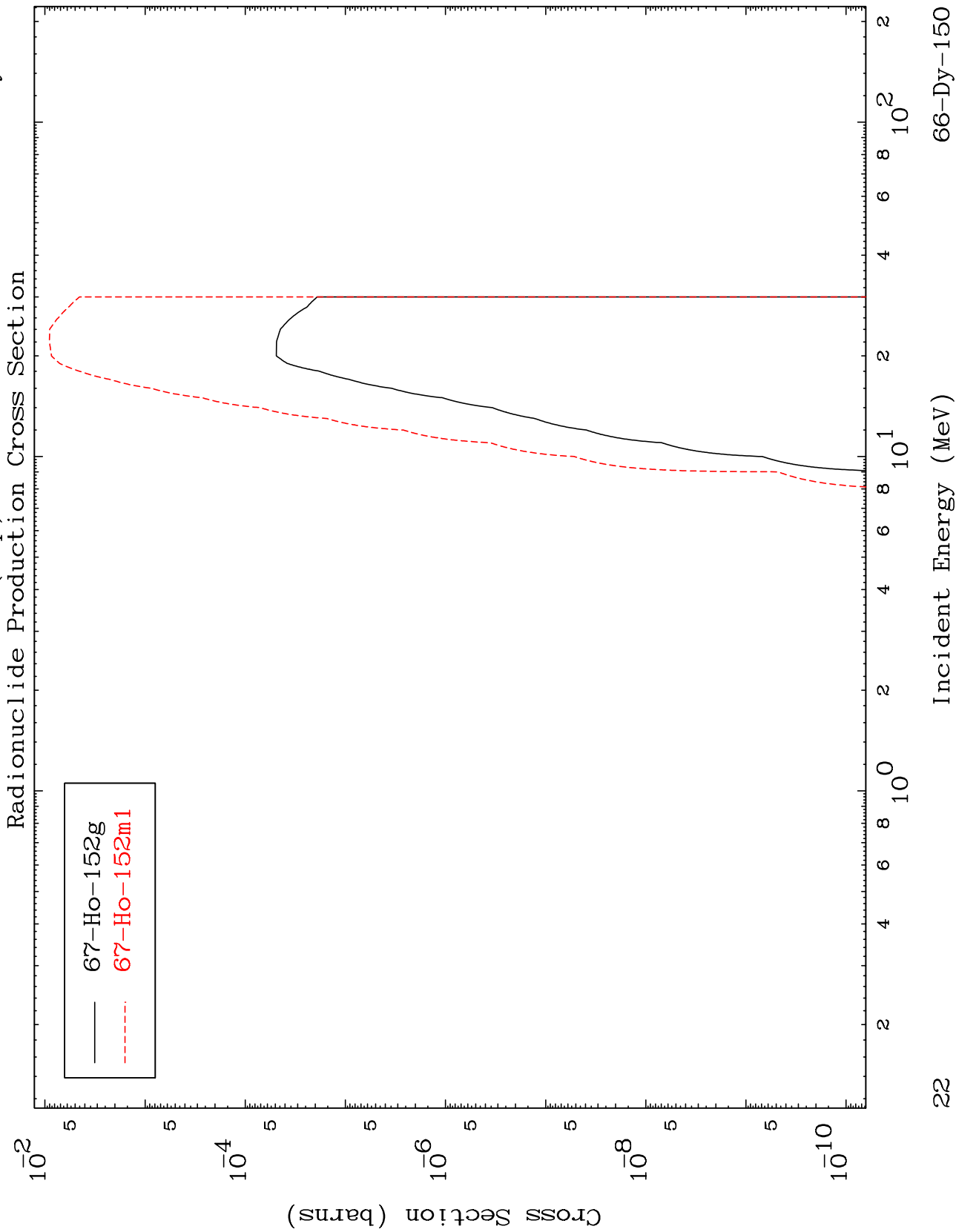
66-Dy-150

Radionuclide Production Cross Section



MAT 6607

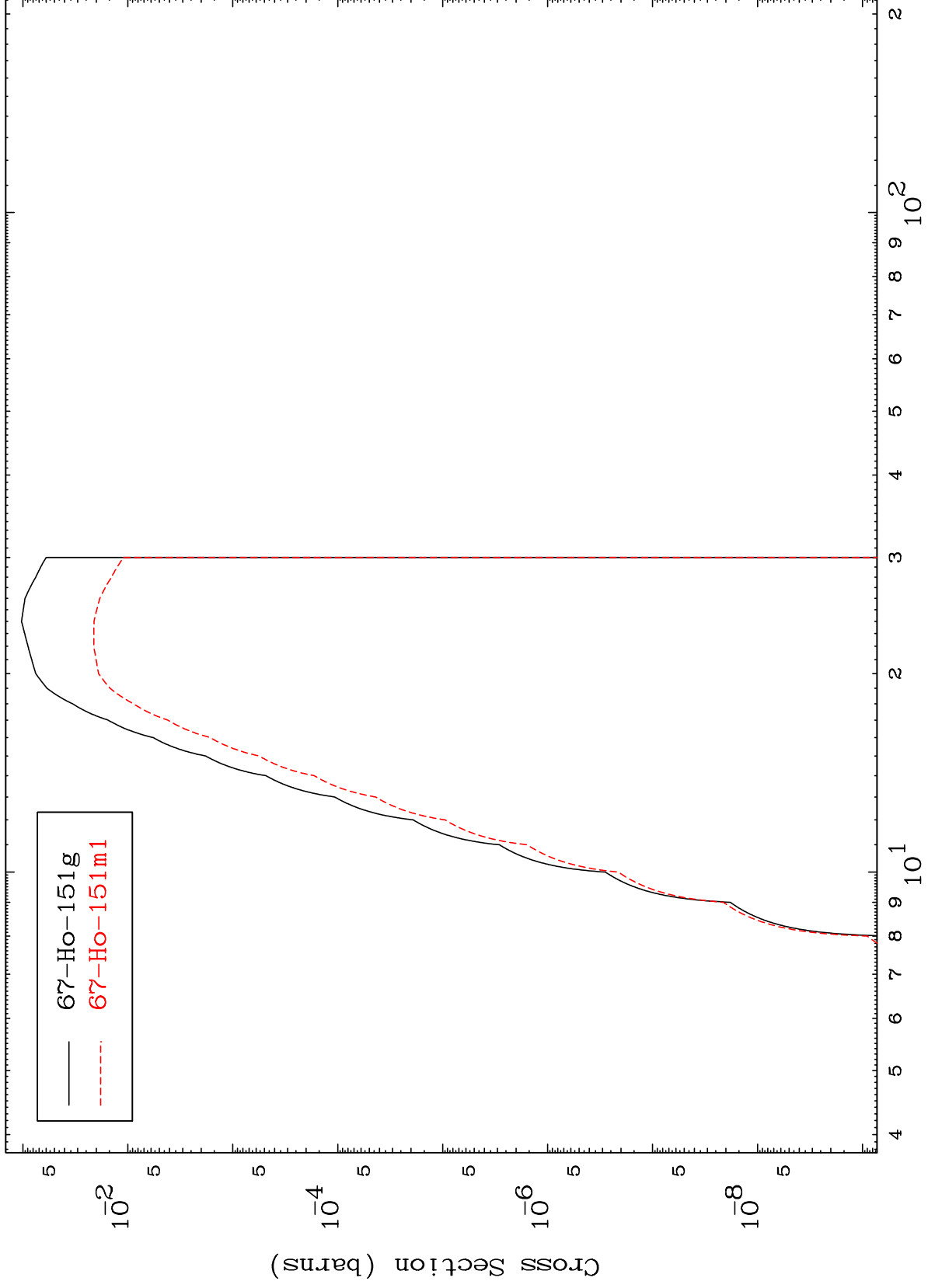
66-Dy-150



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66-Dy-150

(n,d)  
Radionuclide Production Cross Section



23

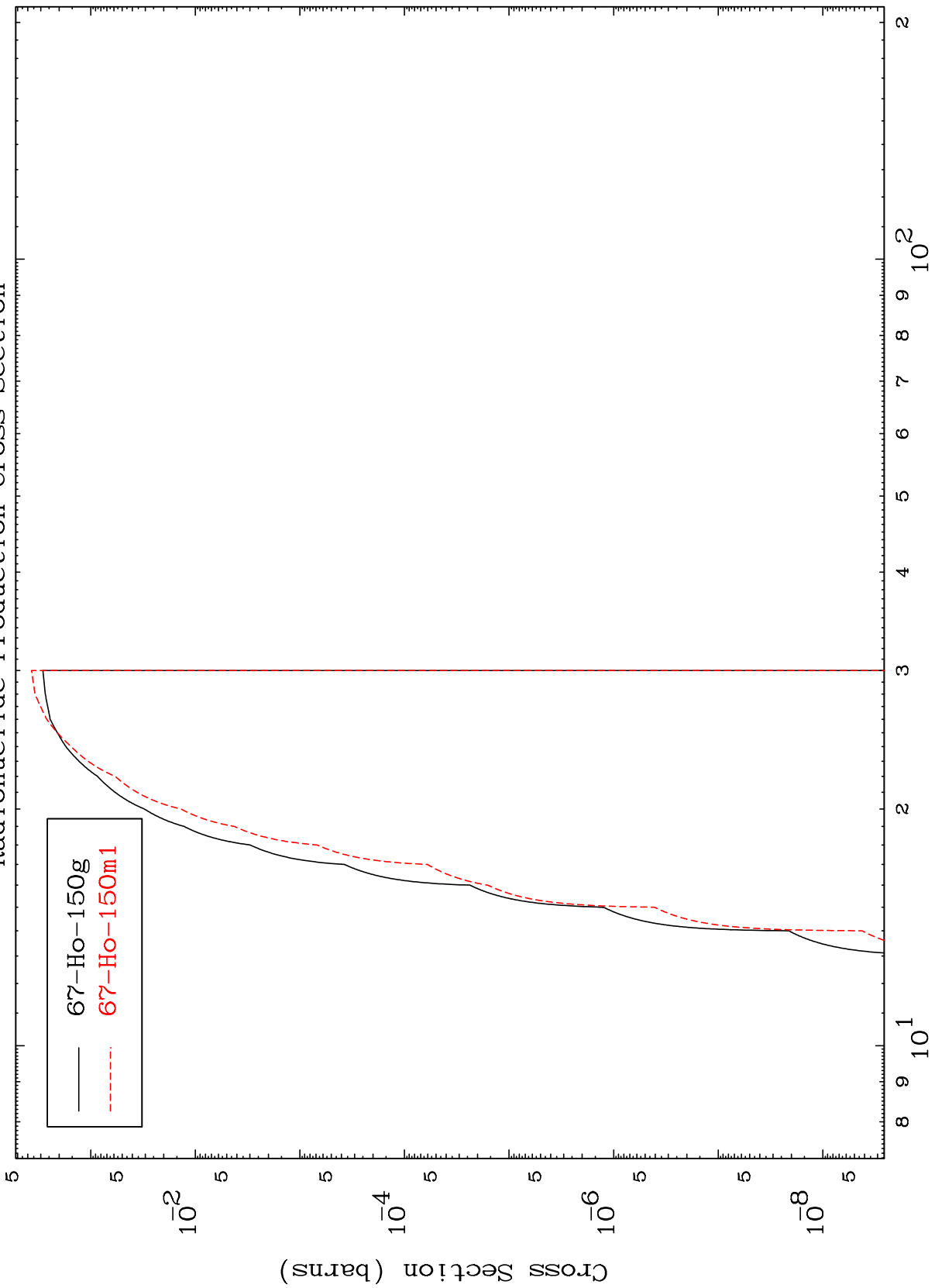
Incident Energy (MeV)

66-Dy-150

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66-Dy-150

(n, t)  
Radionuclide Production Cross Section



66-Dy-150

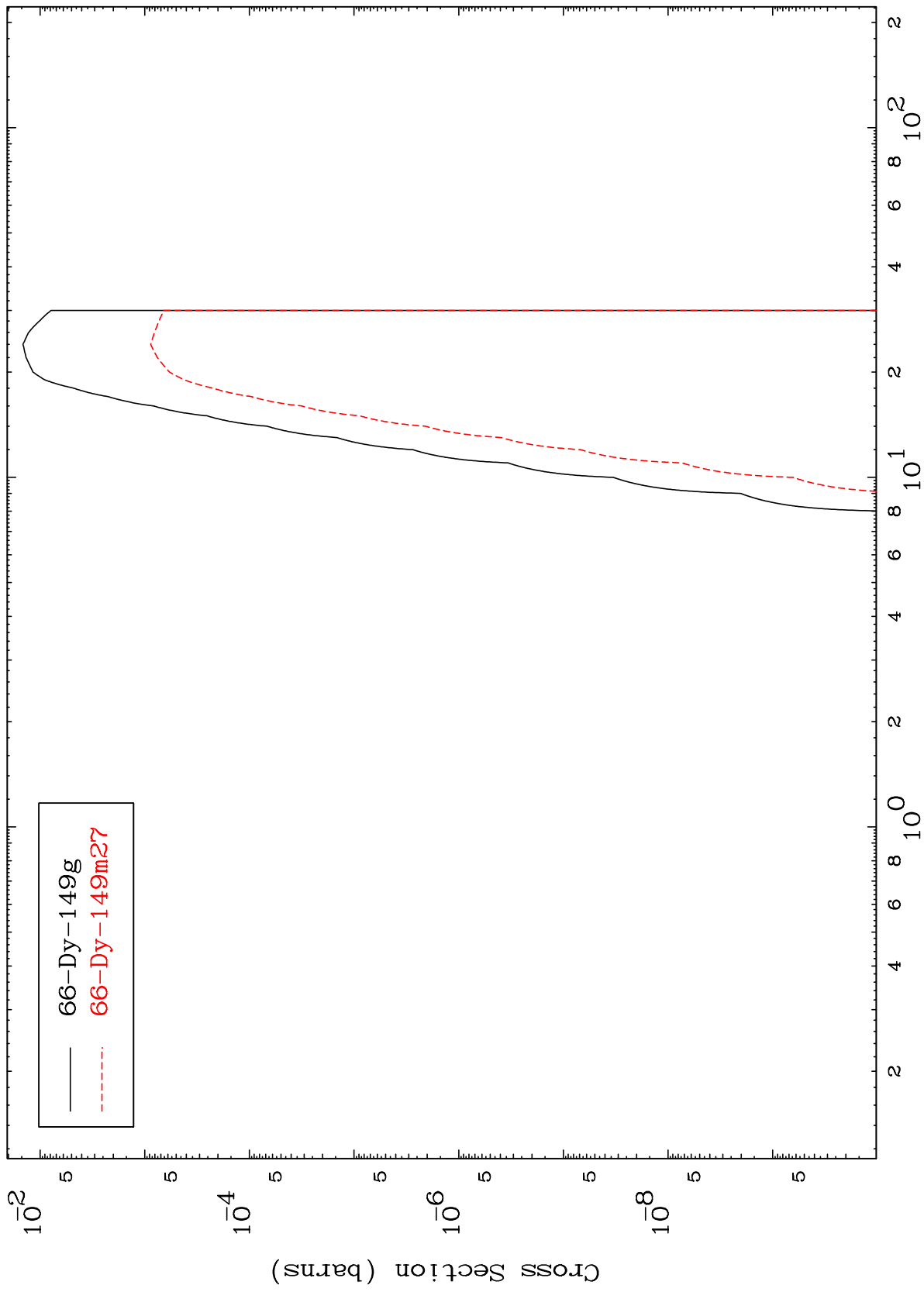
Incident Energy (MeV)

24

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66-Dy-150

Radionuclide Production Cross Section  
(n,  $\alpha$ )



25

66-Dy-150

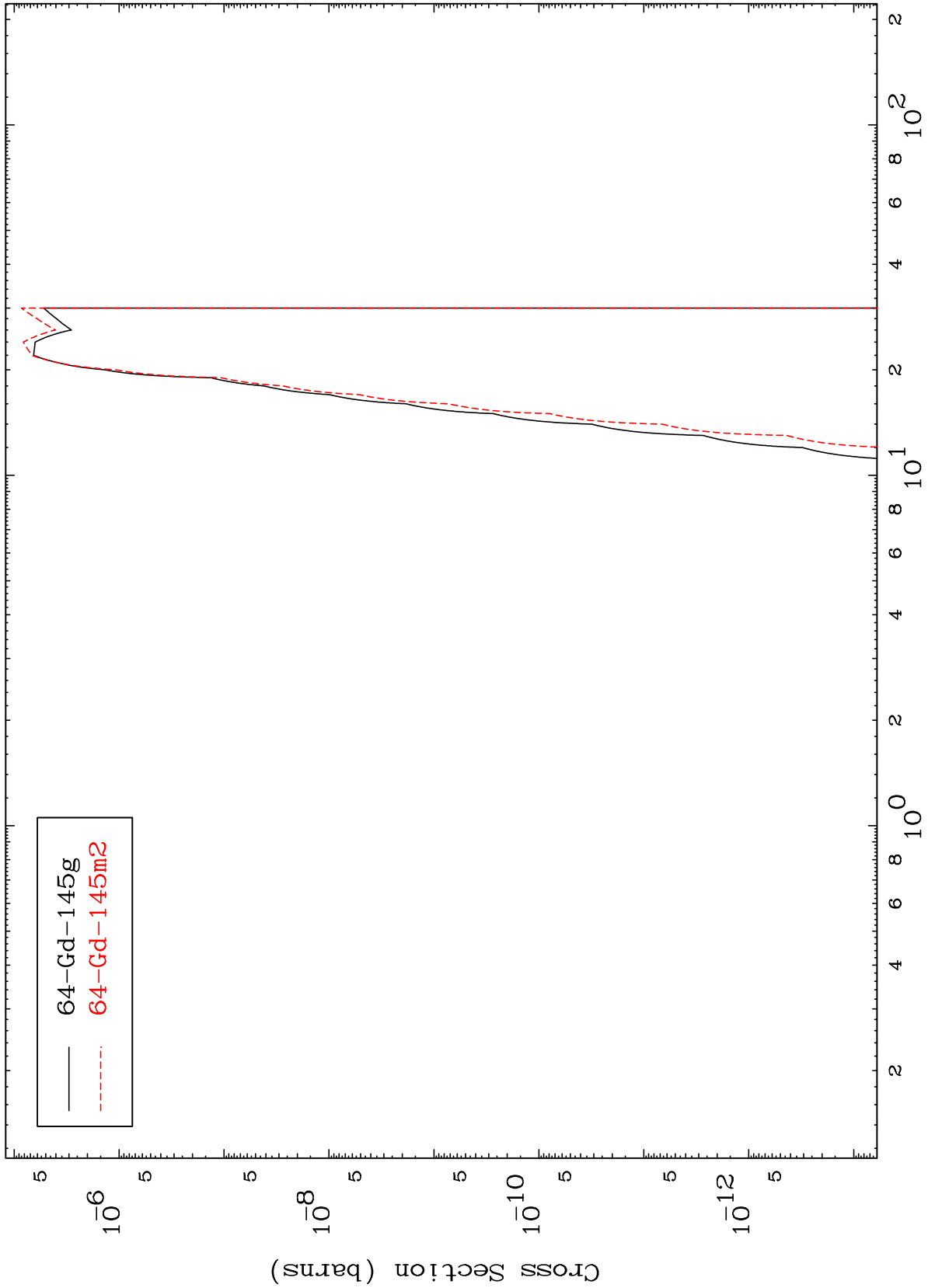
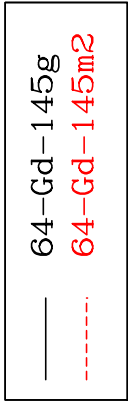
Incident Energy (MeV)

MAT 6607

(n,2α)

66-Dy-150

Radionuclide Production Cross Section



26

Incident Energy (MeV)

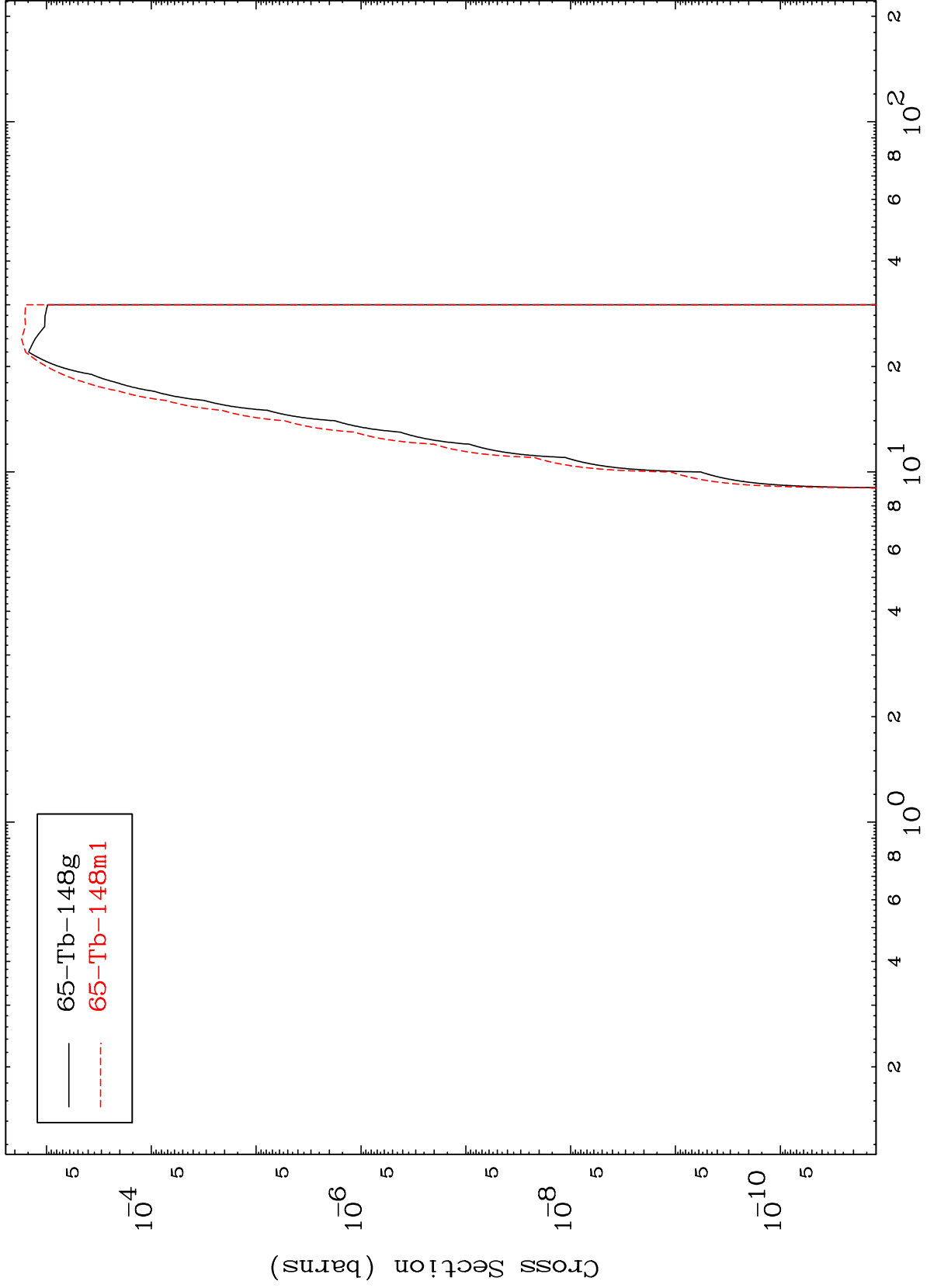
66-Dy-150

MAT 6607

(n,p)  $\alpha$

66-Dy-150

Radionuclide Production Cross Section

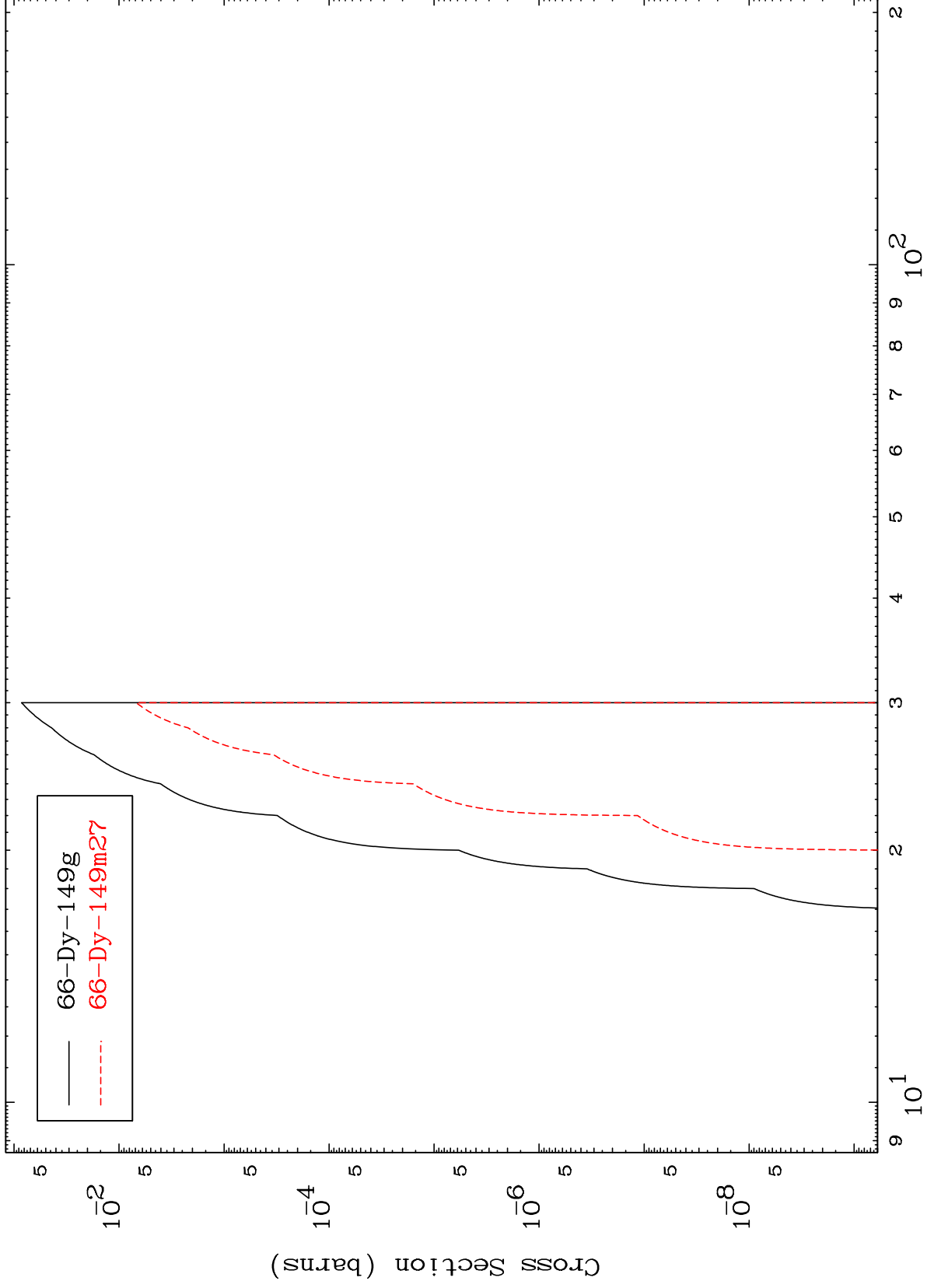


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(n,p) t

66-Dy-150

Radionuclide Production Cross Section



28

Incident Energy (MeV)

66-Dy-150

MAT 6607

(n,d)  $\alpha$

66-Dy-150

Radionuclide Production Cross Section

