

Program EVALPLOT  
(Version 2021-1)

by

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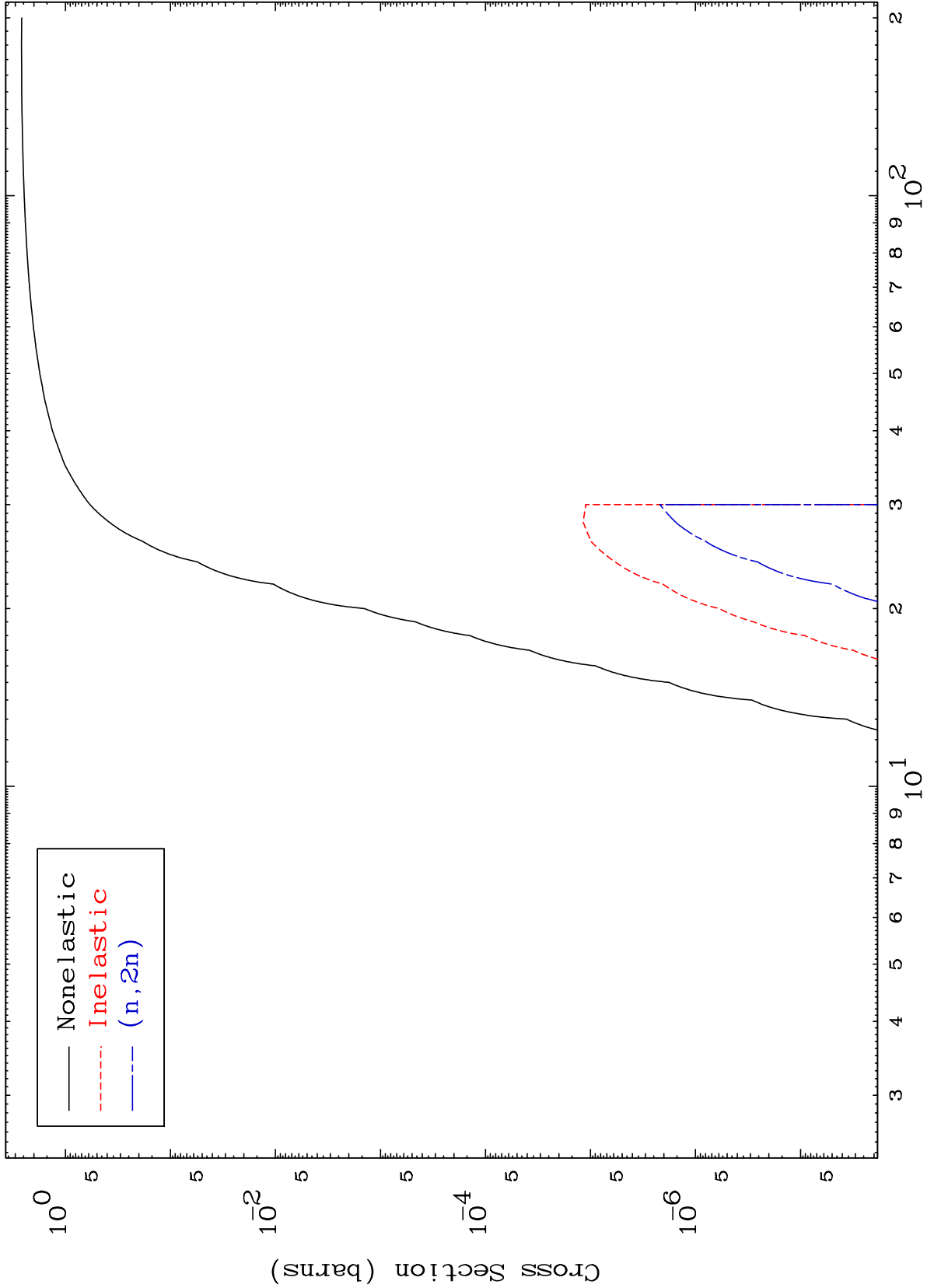
Press Mouse Button to Start

MAT 9930

He-3 Major

100-Fm-249

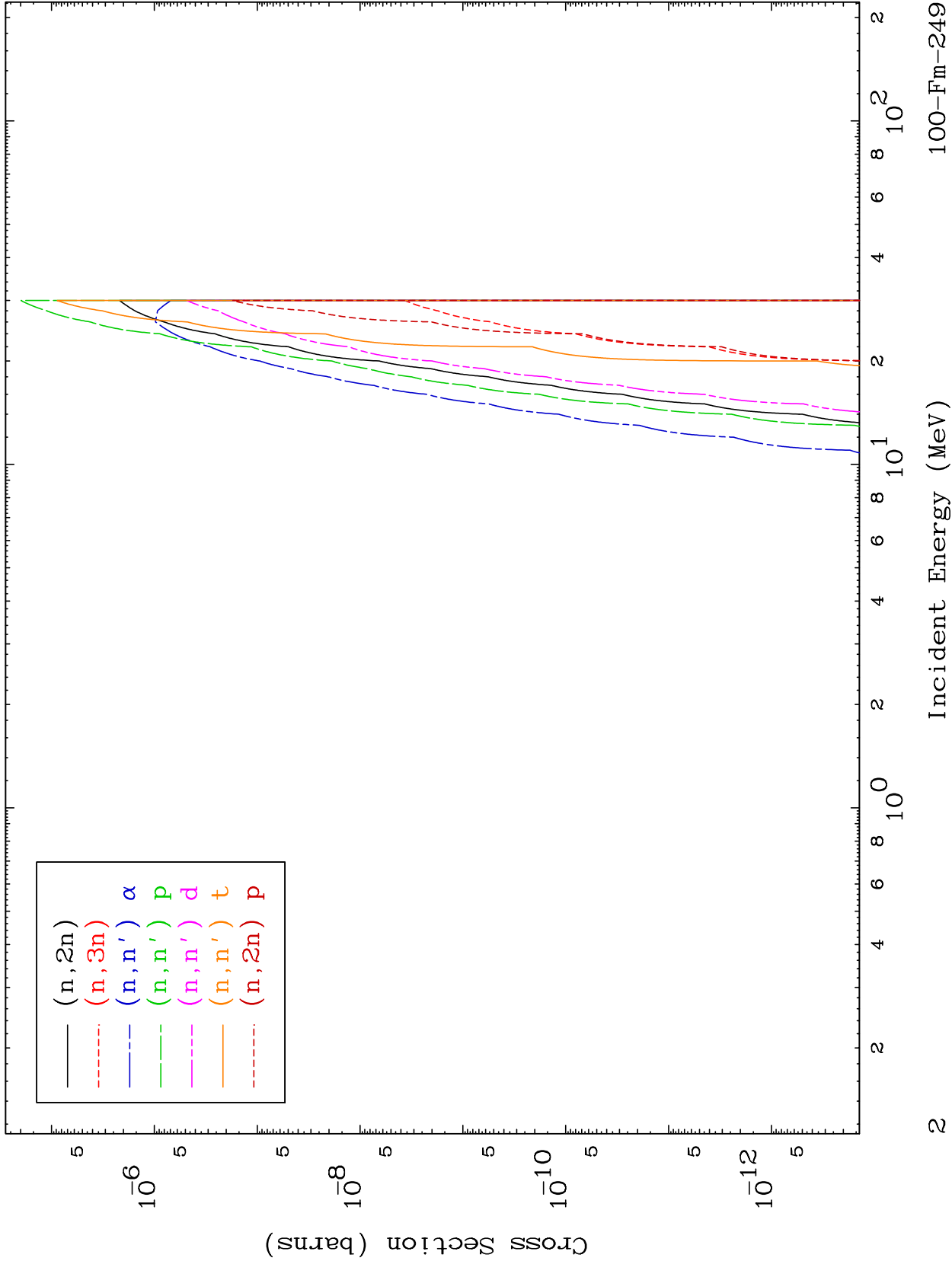
0 Kelvin Cross Sections



MAT 9930

He-3 Neutron Absorption  
0 Kelvin Cross Sections

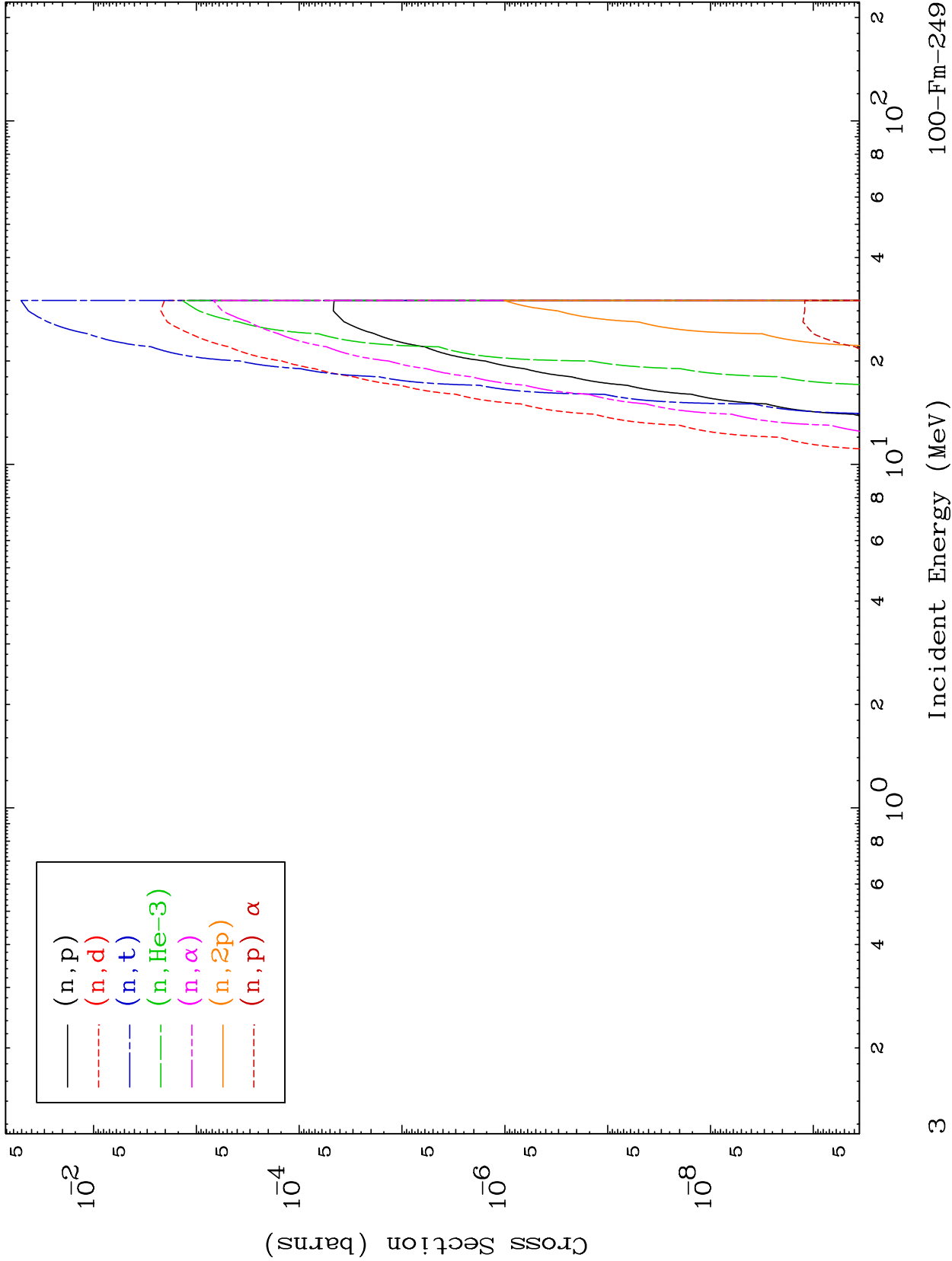
100-Fm-249



MAT 9930

He-3 Neutron Absorption  
0 Kelvin Cross Sections

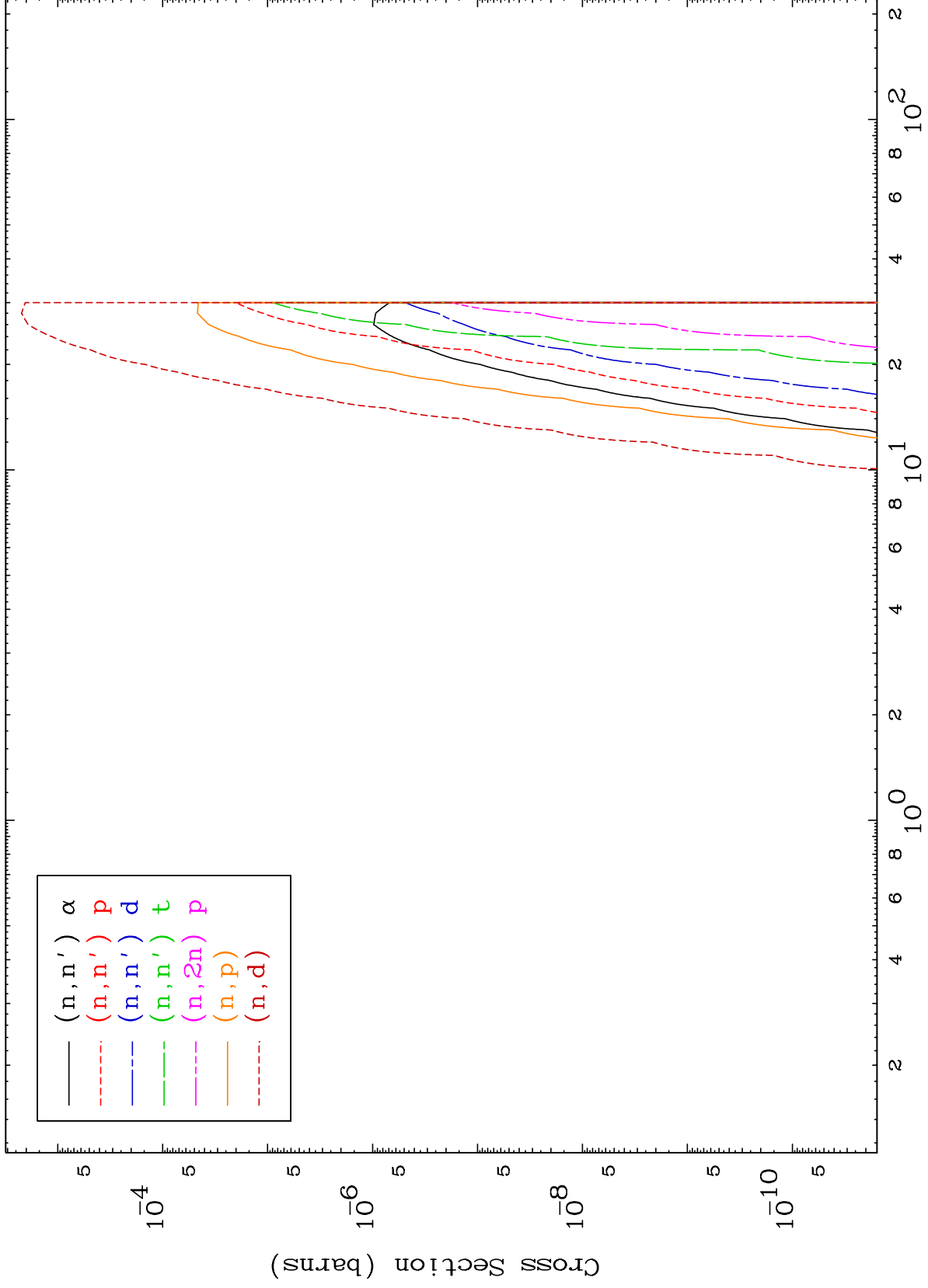
100-Fm-249



MAT 9930

He-3 Charged Particle  
0 Kelvin Cross Sections

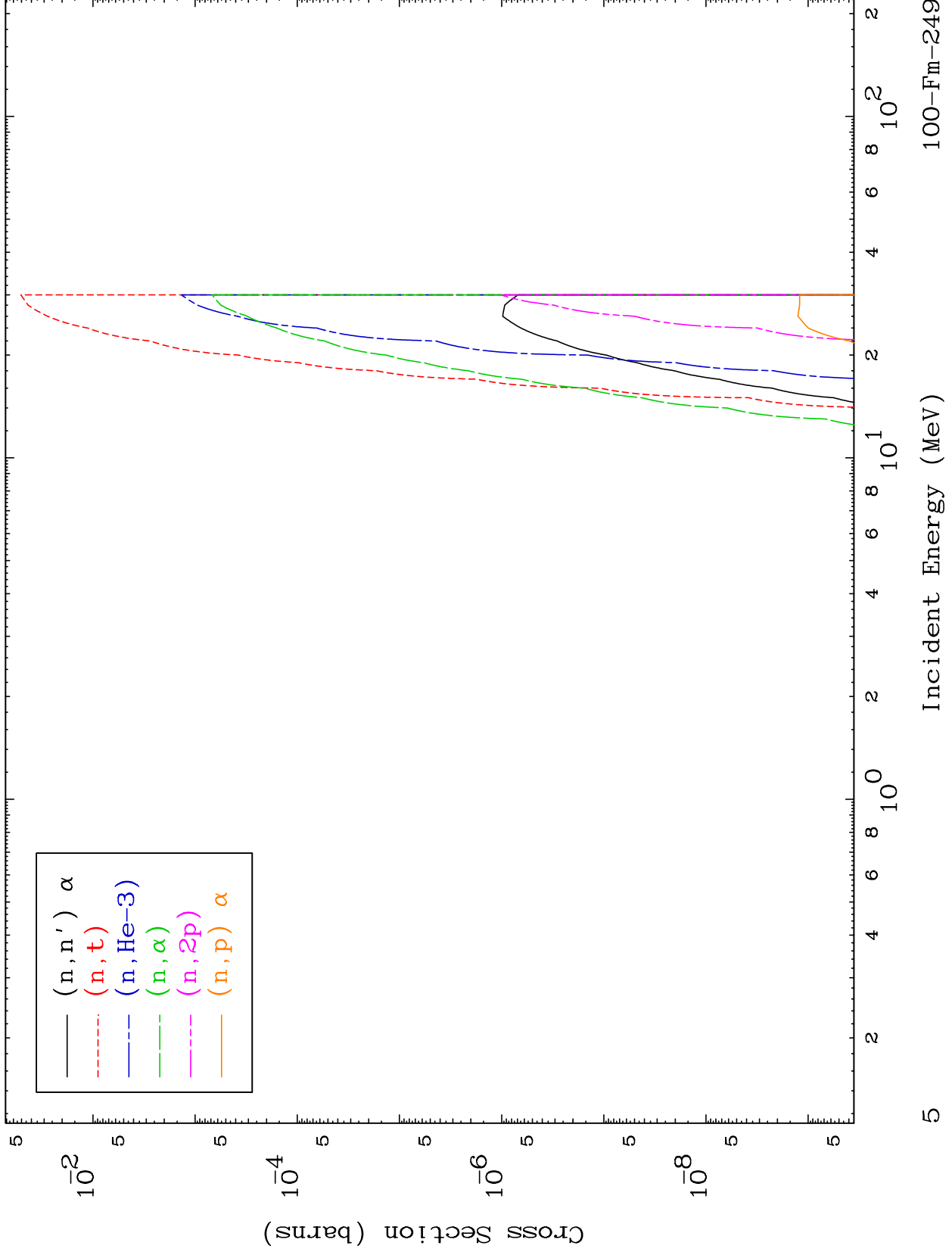
100-Fm-249



MAT 9930

He-3 Charged Particle  
0 Kelvin Cross Sections

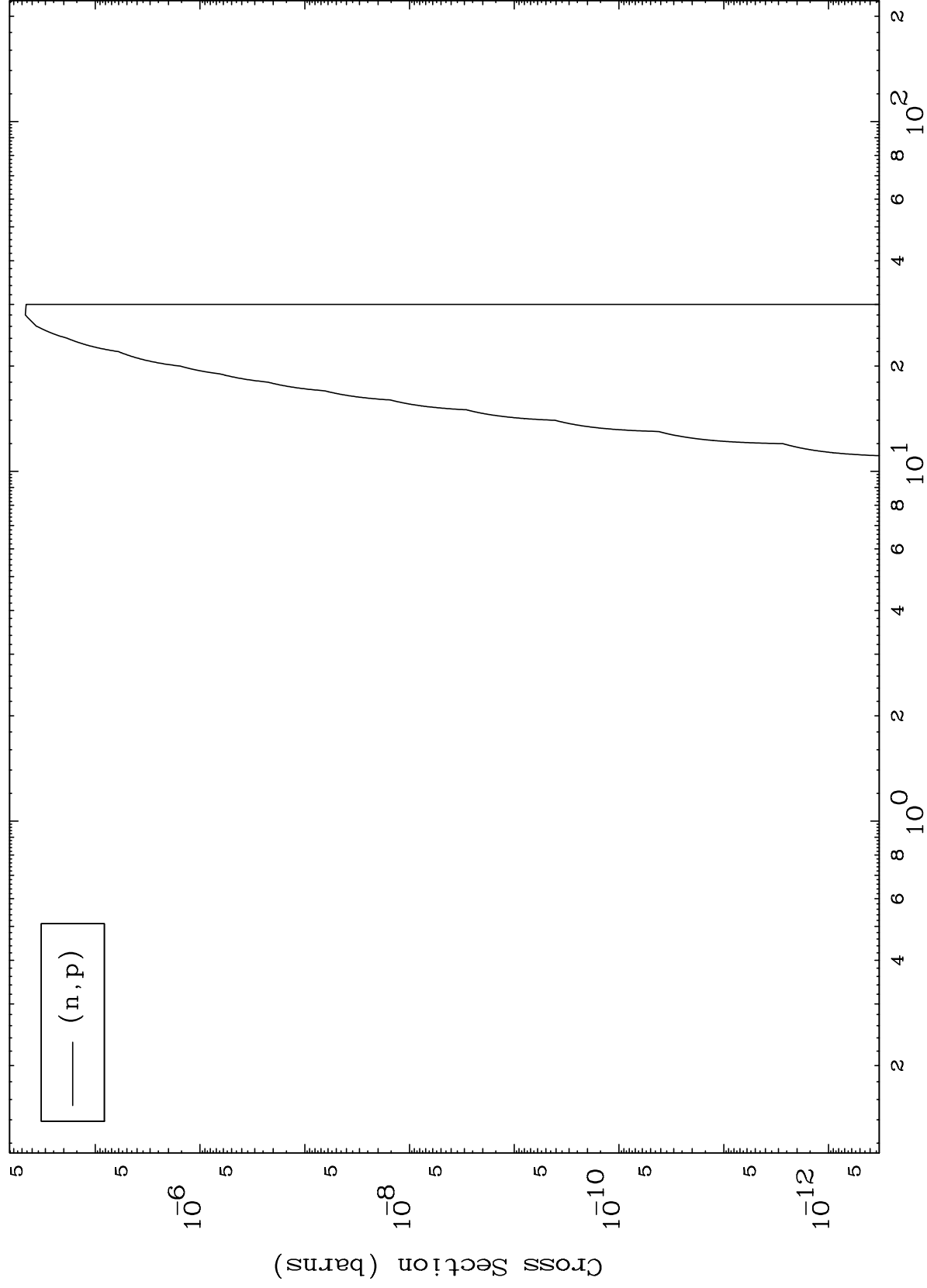
100-Fm-249



MAT 9930

(He-3,p) Levels  
0 Kelvin Cross Sections

100-Fm-249



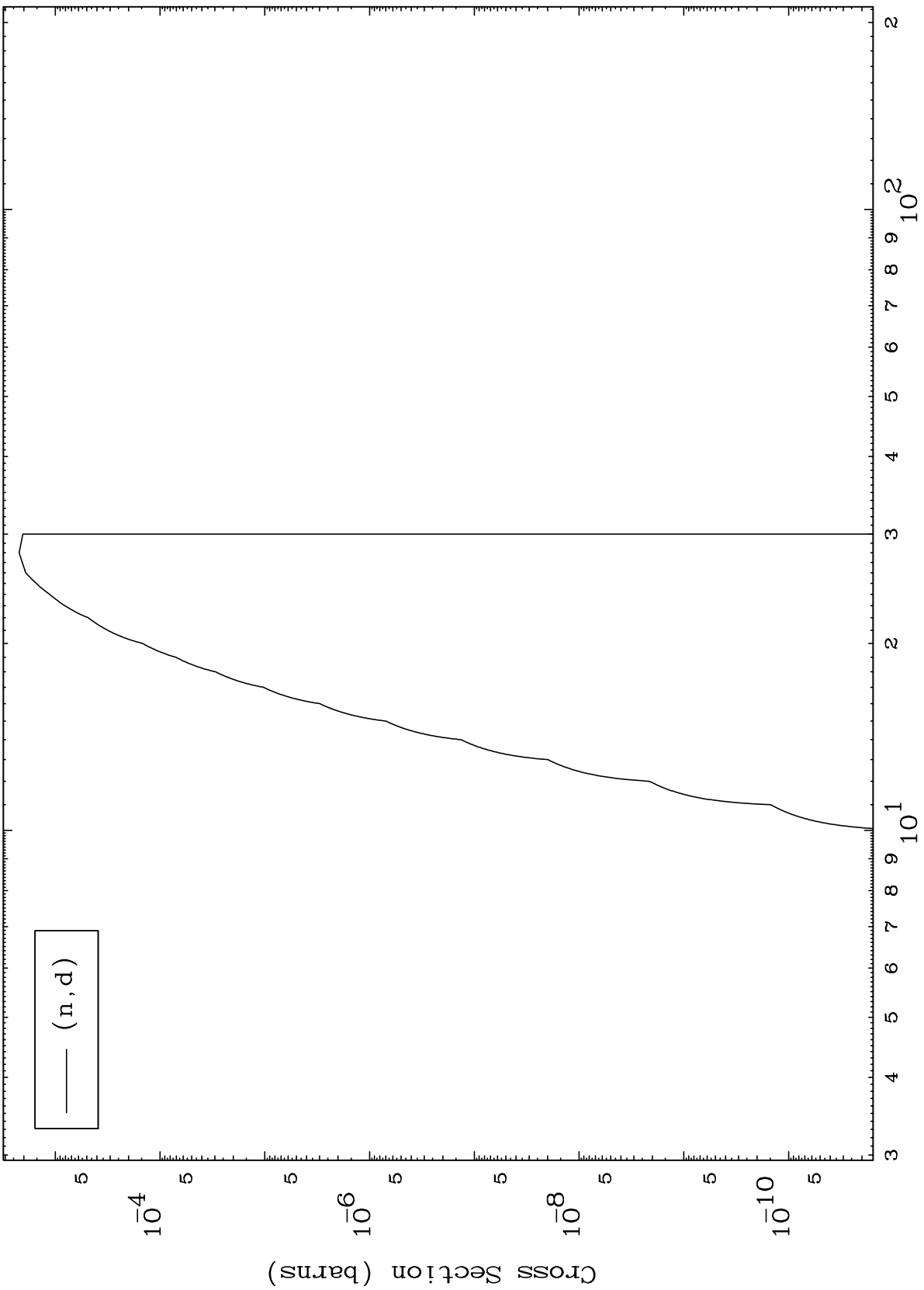
(n,p)

MAT 9930

(He-3,d) Levels

100-Fm-249

0 Kelvin Cross Sections



7

Incident Energy (MeV)

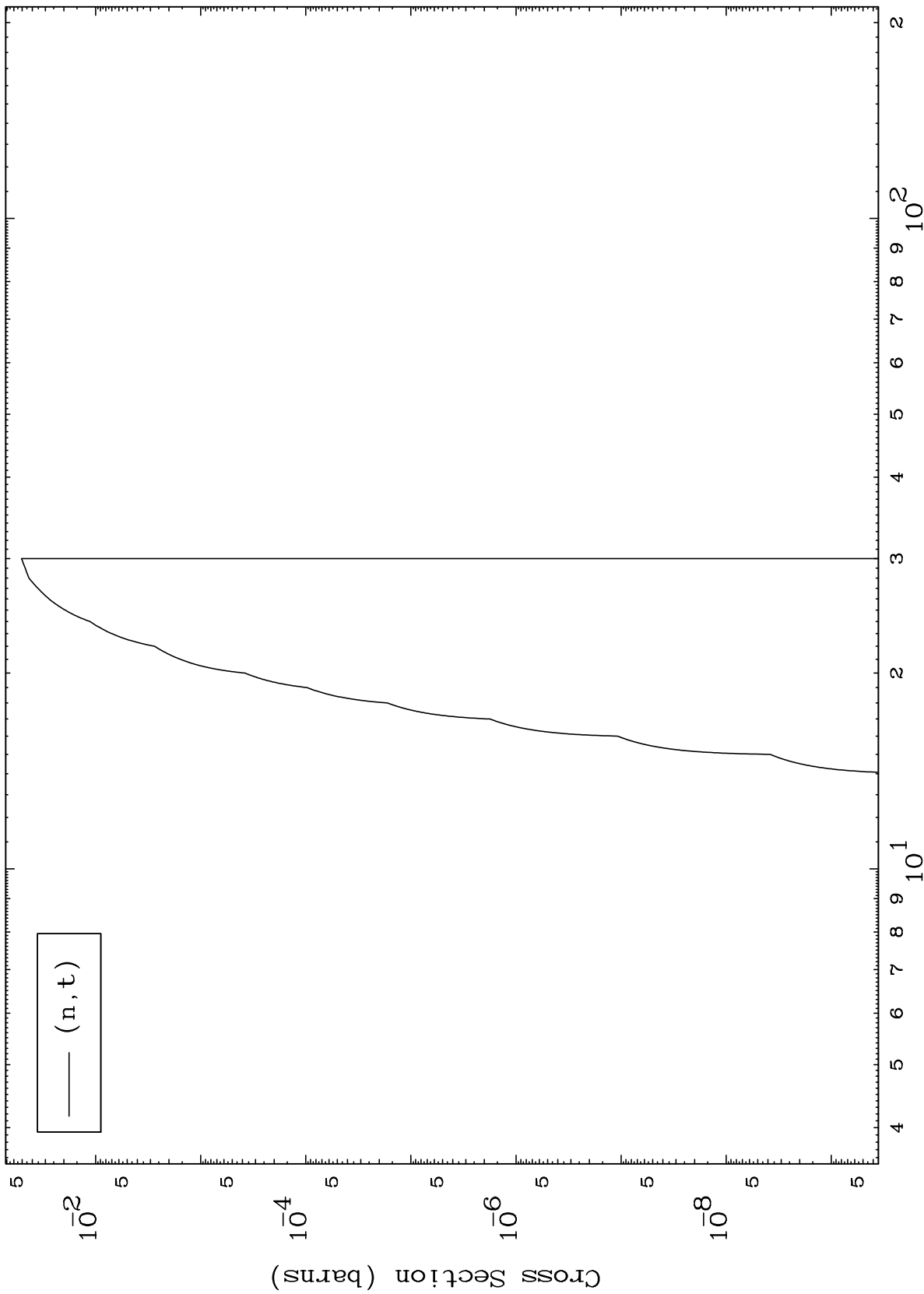
100-Fm-249

MAT 9930

(He-3,t) Levels

100-Fm-249

0 Kelvin Cross Sections



8

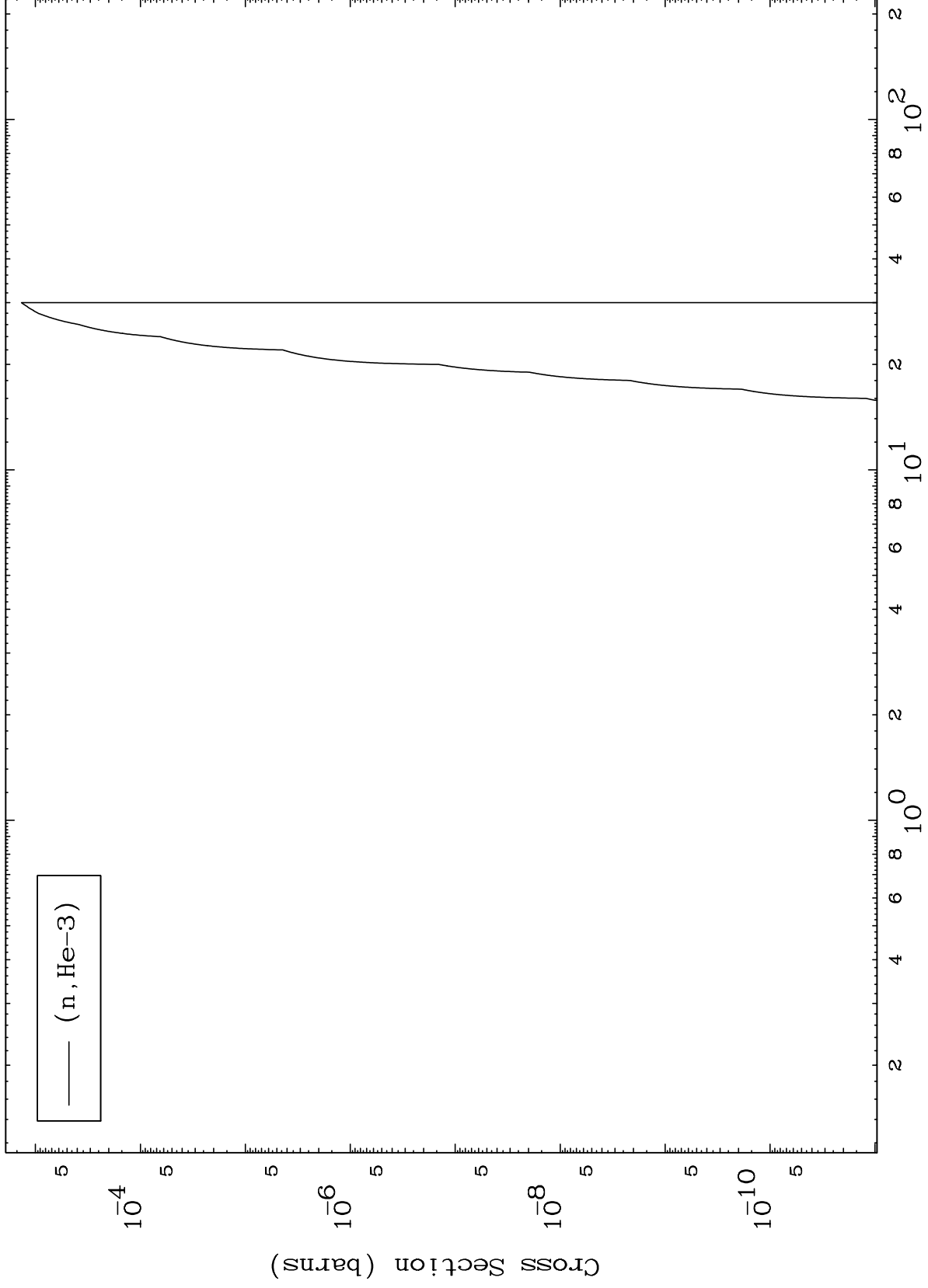
Incident Energy (MeV)

100-Fm-249

MAT 9930

(He-3, He3) Levels  
0 Kelvin Cross Sections

100-Fm-249

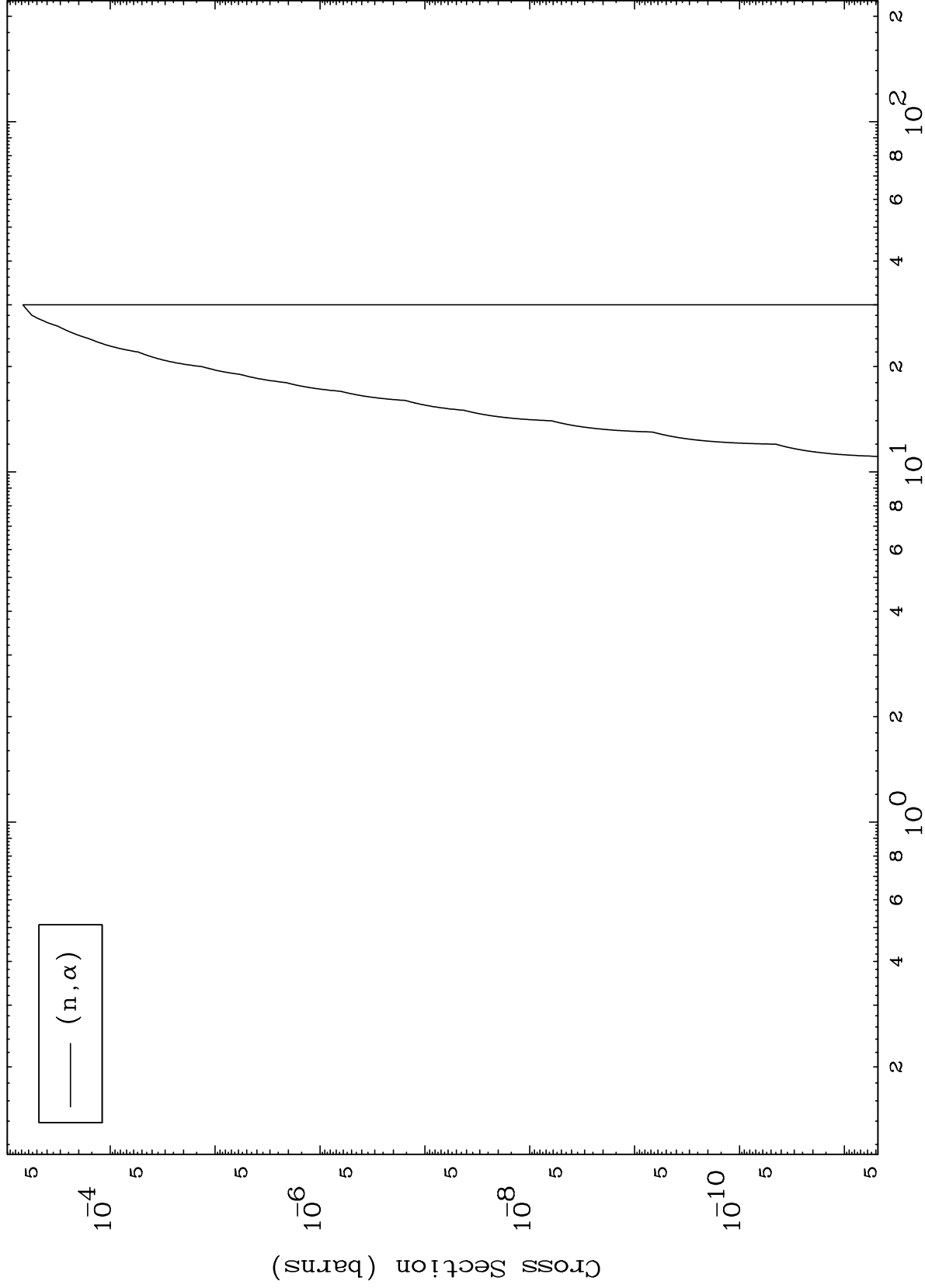


MAT 9930

(He-3,  $\alpha$ ) Levels

100-Fm-249

0 Kelvin Cross Sections



10

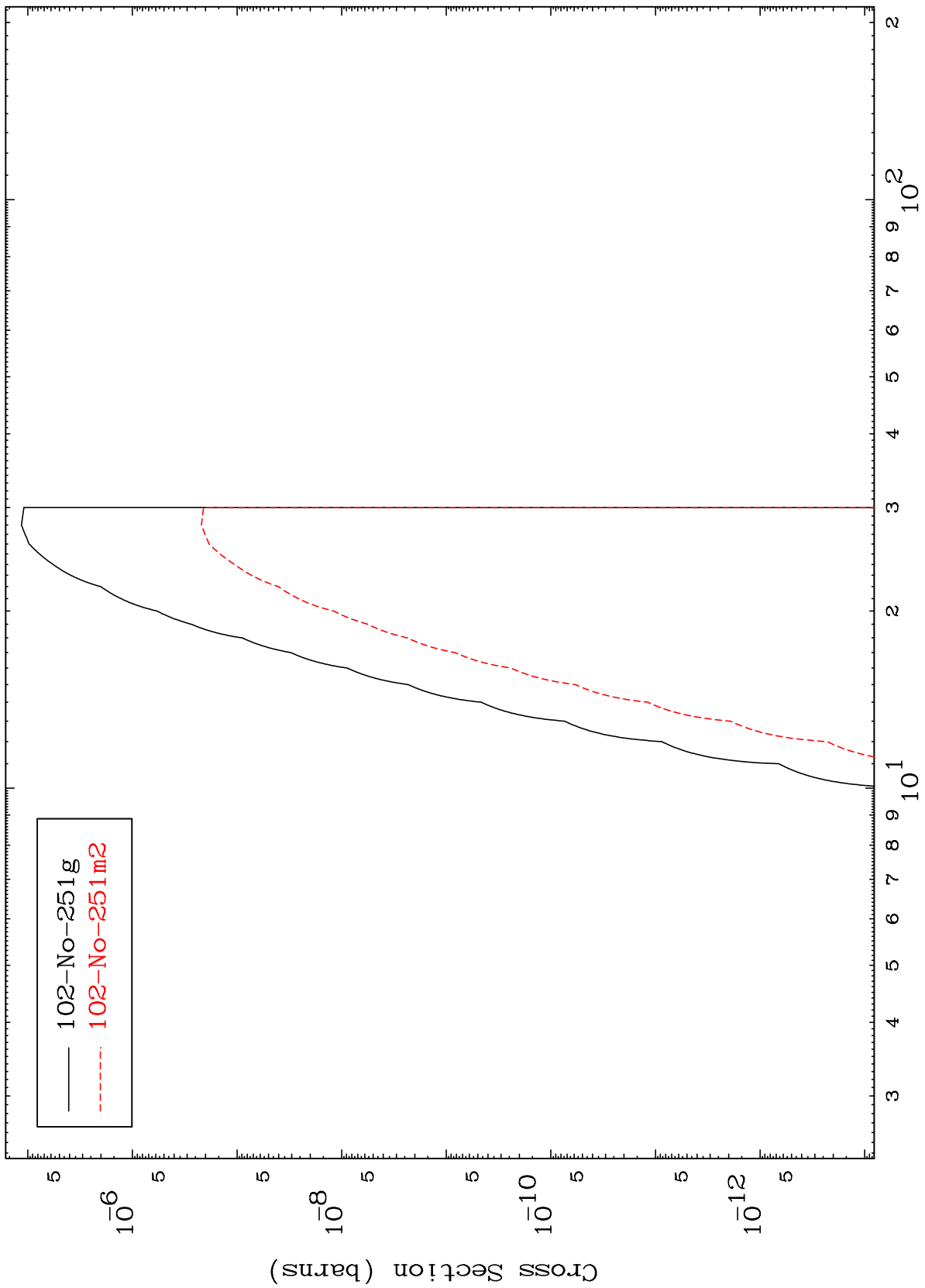
Incident Energy (MeV)

100-Fm-249

MAT 9930

100-Fm-249

Radionuclide Production Cross Section



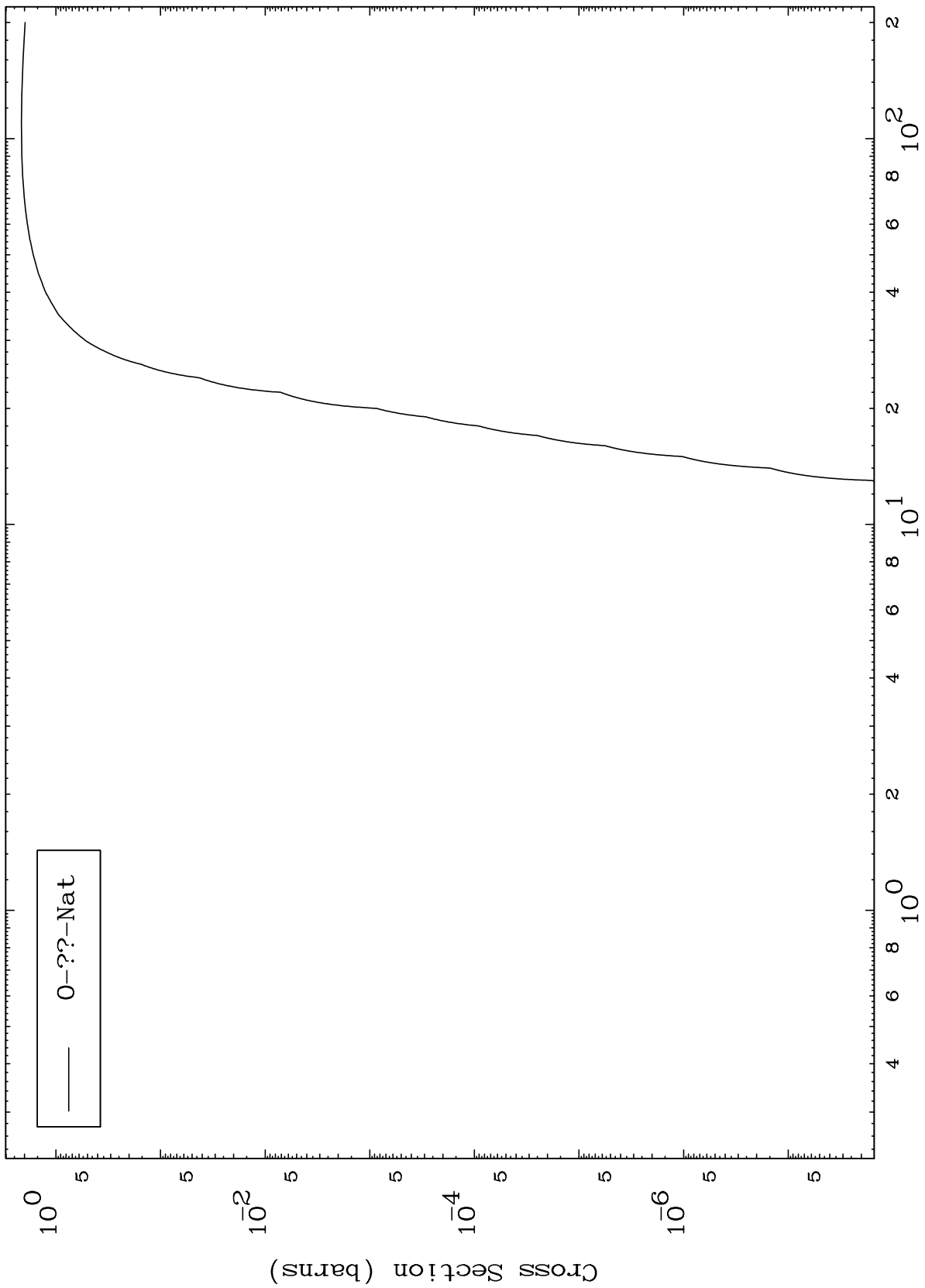
— 102-No-251g  
- - - 102-No-251m2

MAT 9930

100-Fm-249

Fission

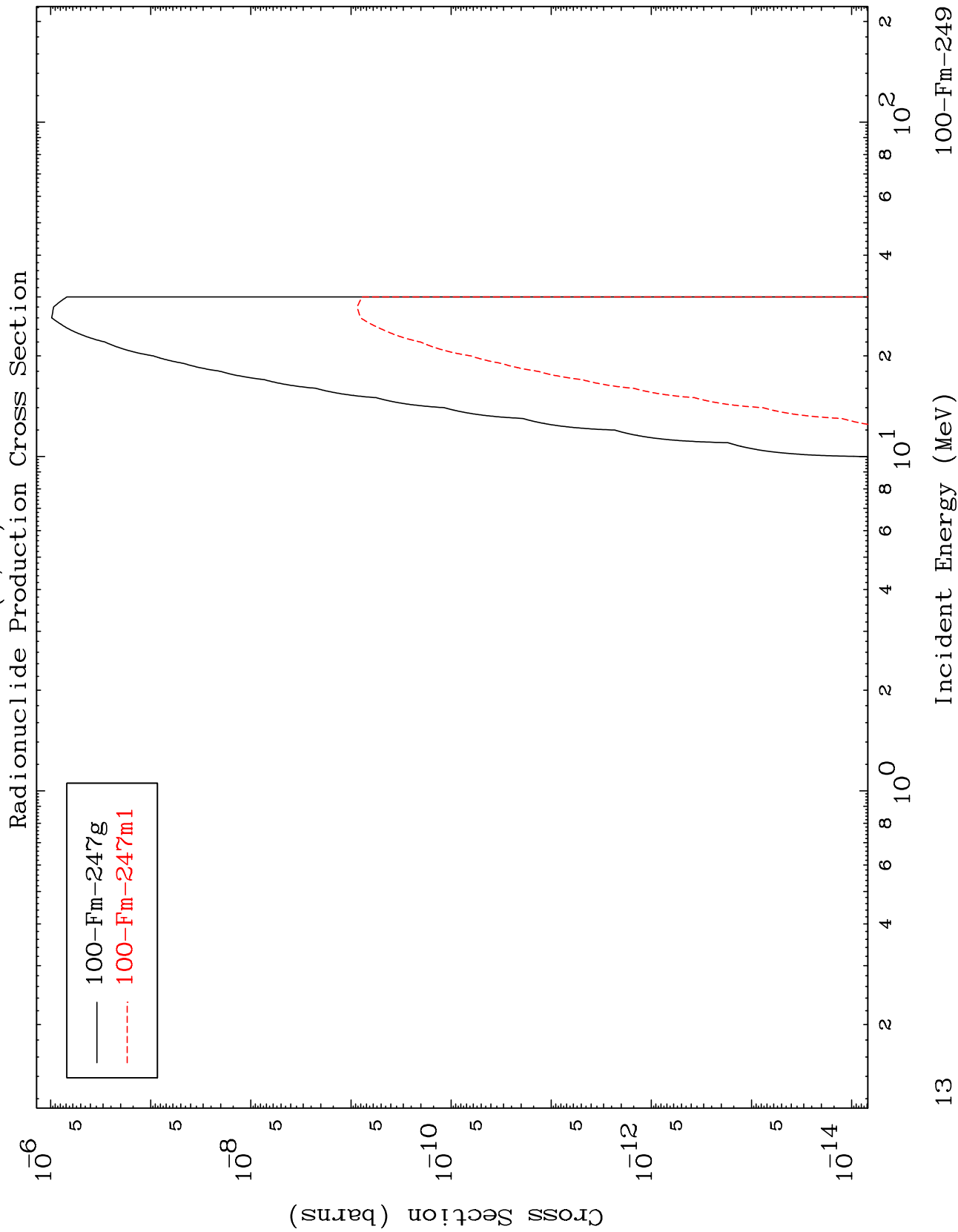
Radionuclide Production Cross Section



MAT 9930

(n,n')  $\alpha$

100-Fm-249

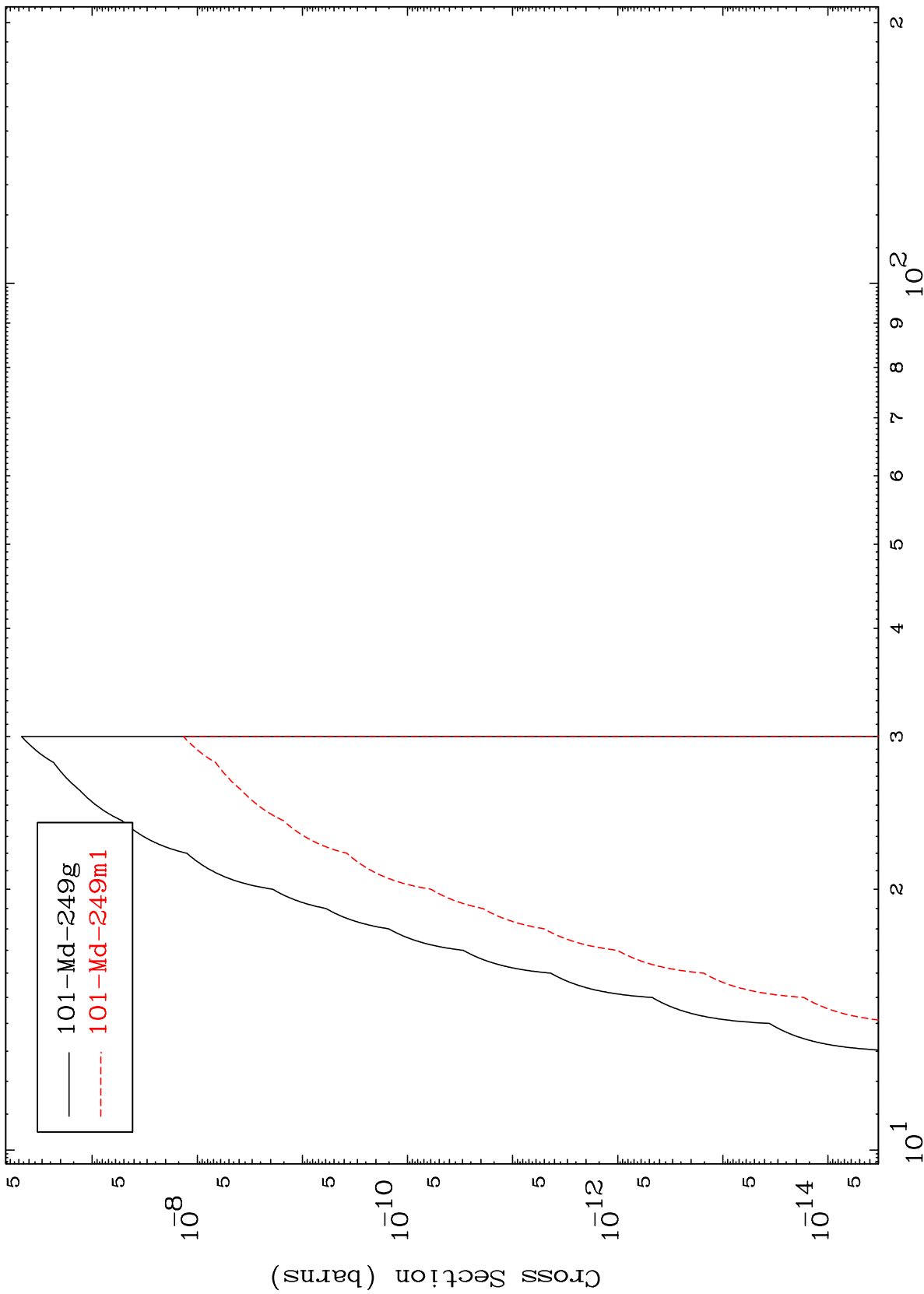


MAT 9930

(n,n') d

100-Fm-249

Radionuclide Production Cross Section



101-Md-249g  
101-Md-249m1

Incident Energy (MeV)

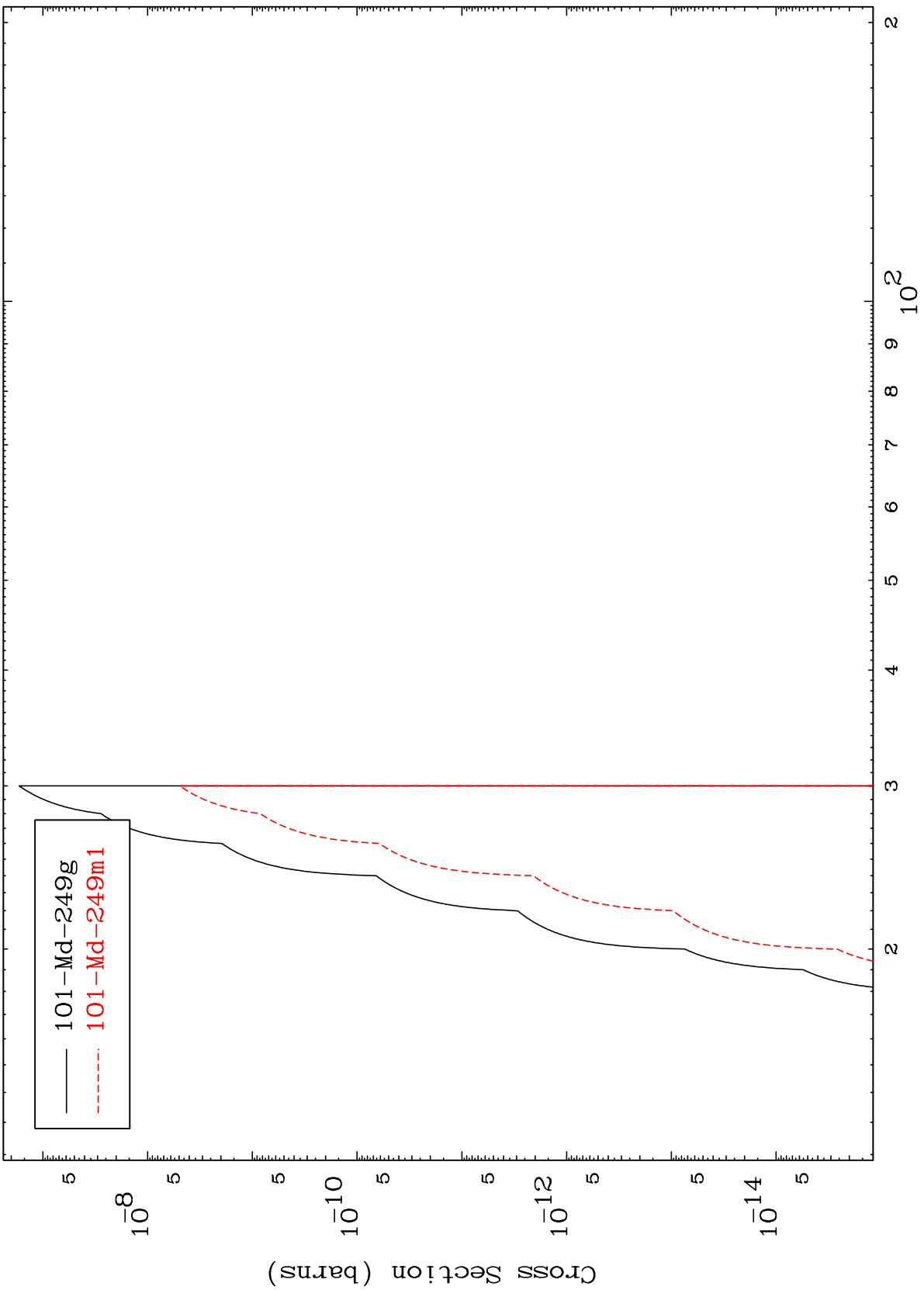
100-Fm-249

MAT 9930

(n,2n) p

100-Fm-249

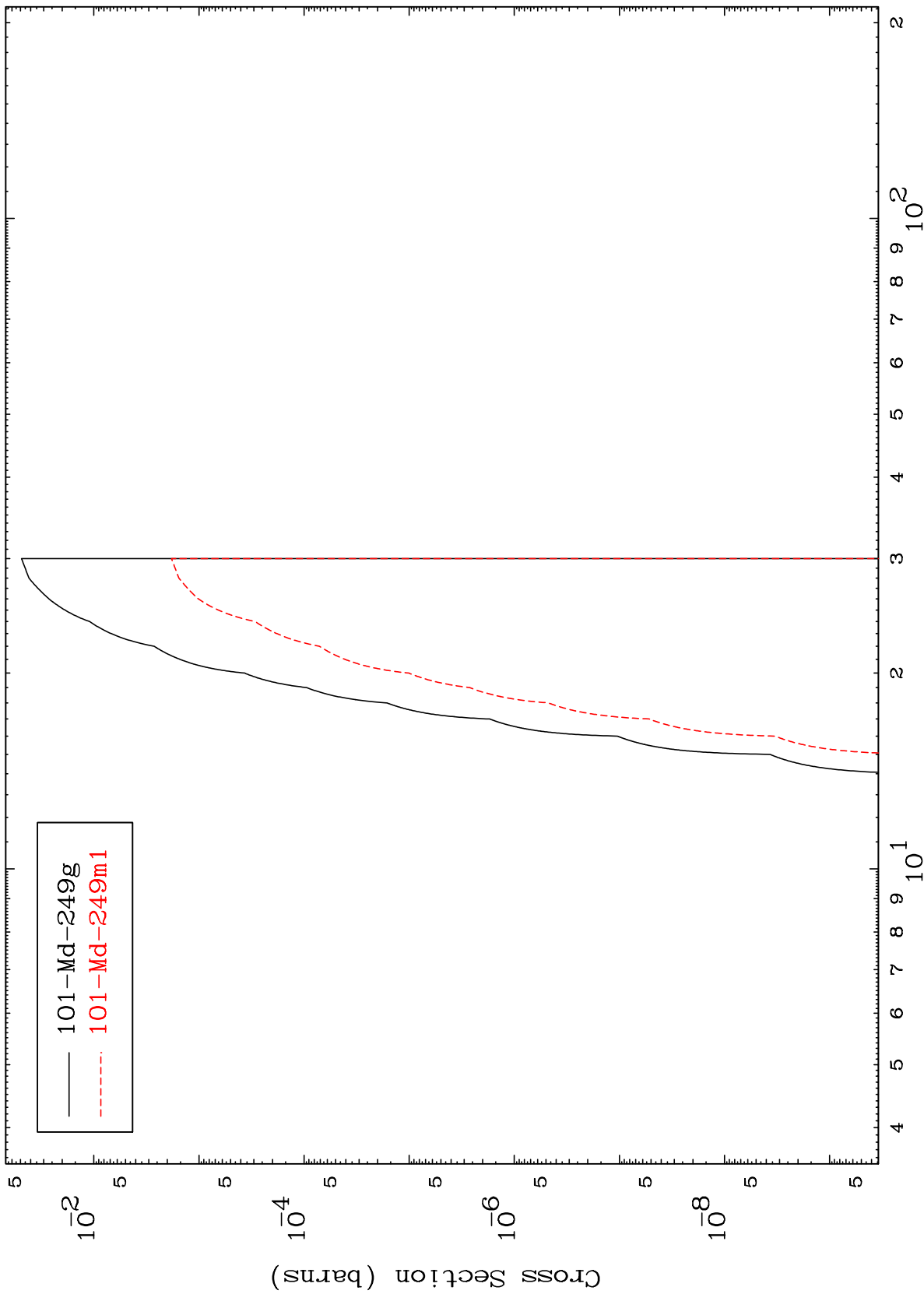
Radionuclide Production Cross Section



MAT 9930

100-Fm-249

(n, t)  
Radionuclide Production Cross Section



16

Incident Energy (MeV)

100-Fm-249

MAT 9930

(n,p)  $\alpha$

100-Fm-249

Radionuclide Production Cross Section

