

Program EVALPLOT  
(Version 2021-1)

by

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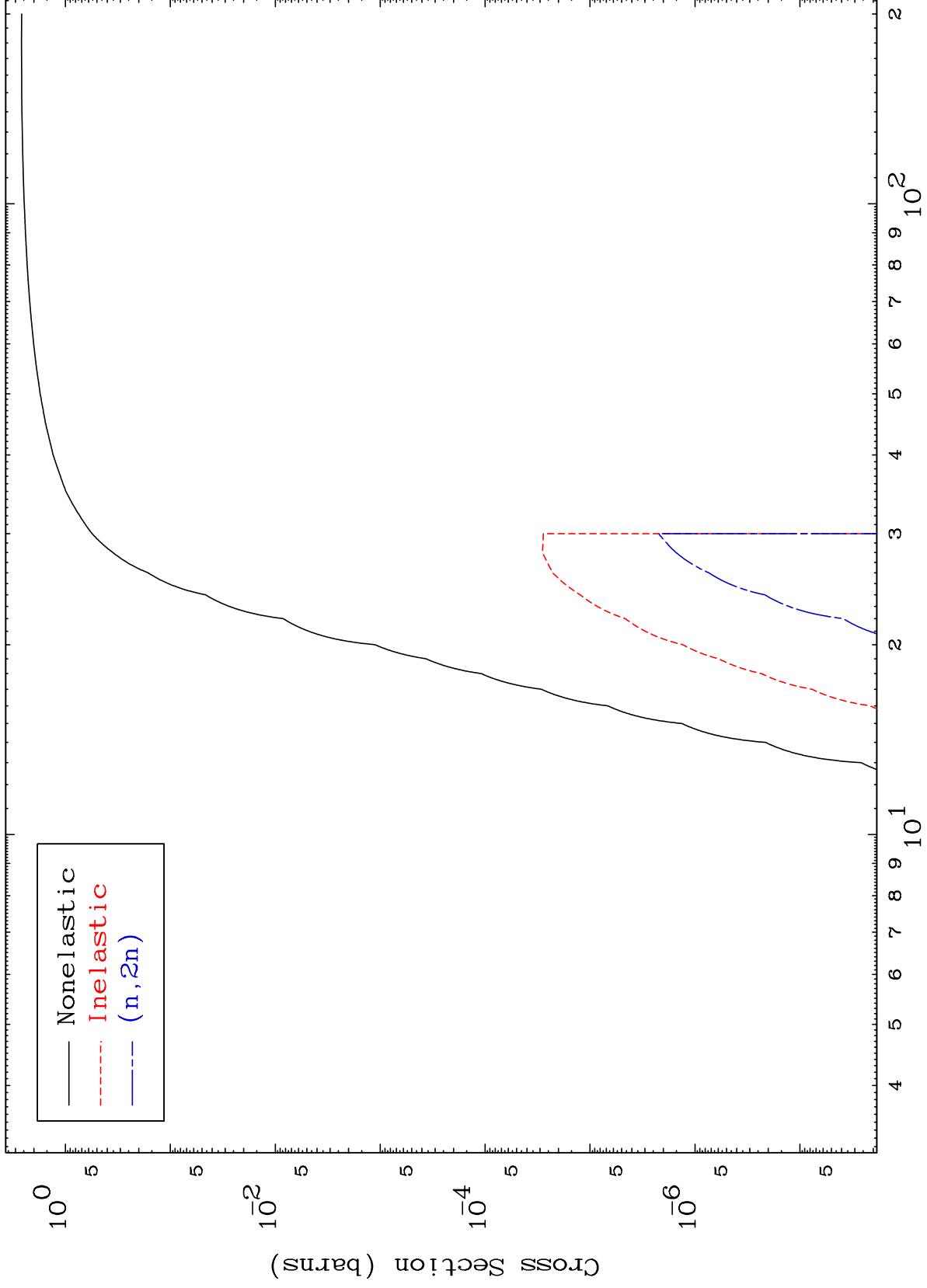
Press Mouse Button to Start

MAT 9951

He-3 Major

0 Kelvin Cross Sections

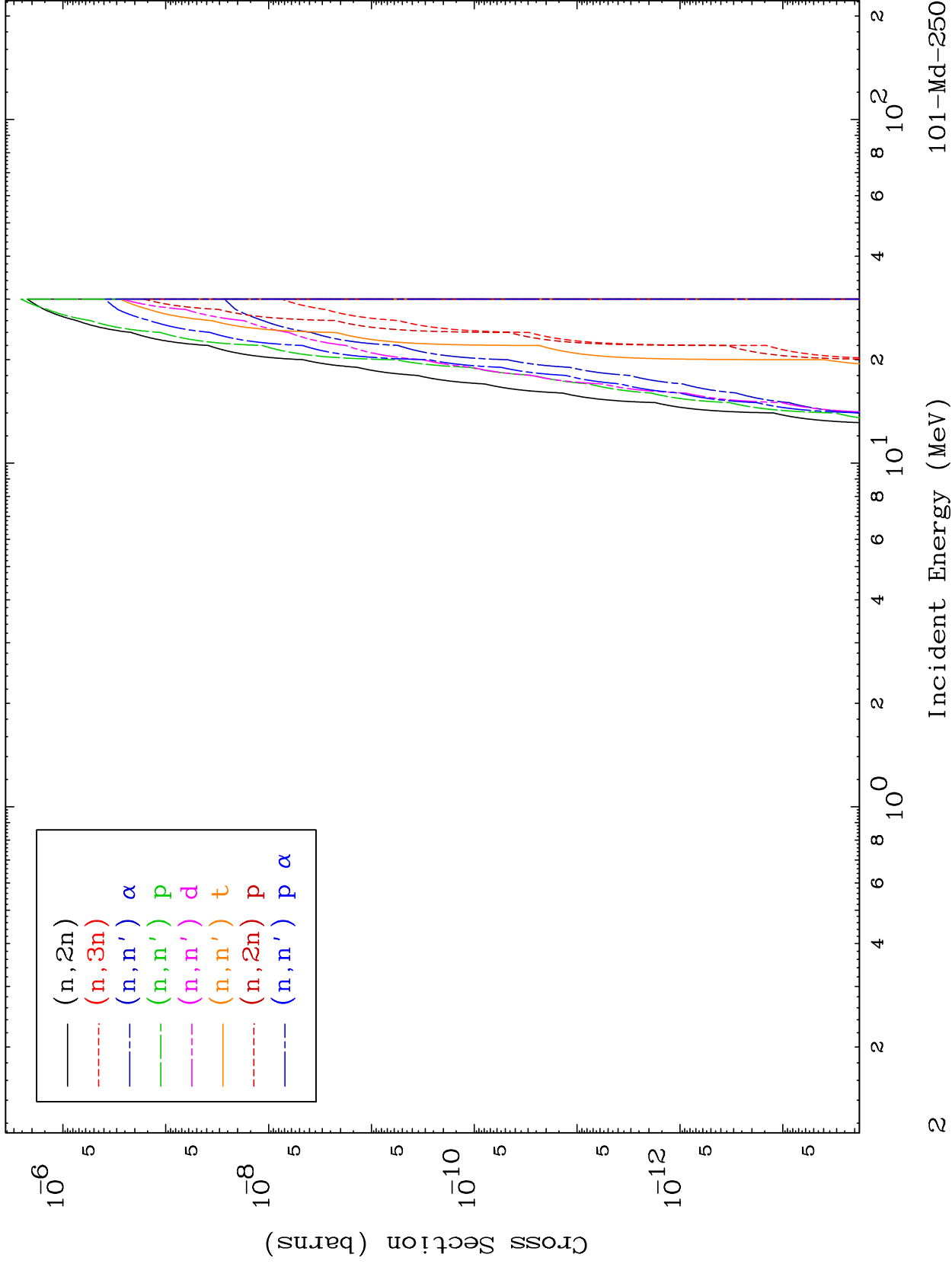
101-Md-250



MAT 9951

He-3 Neutron Absorption  
0 Kelvin Cross Sections

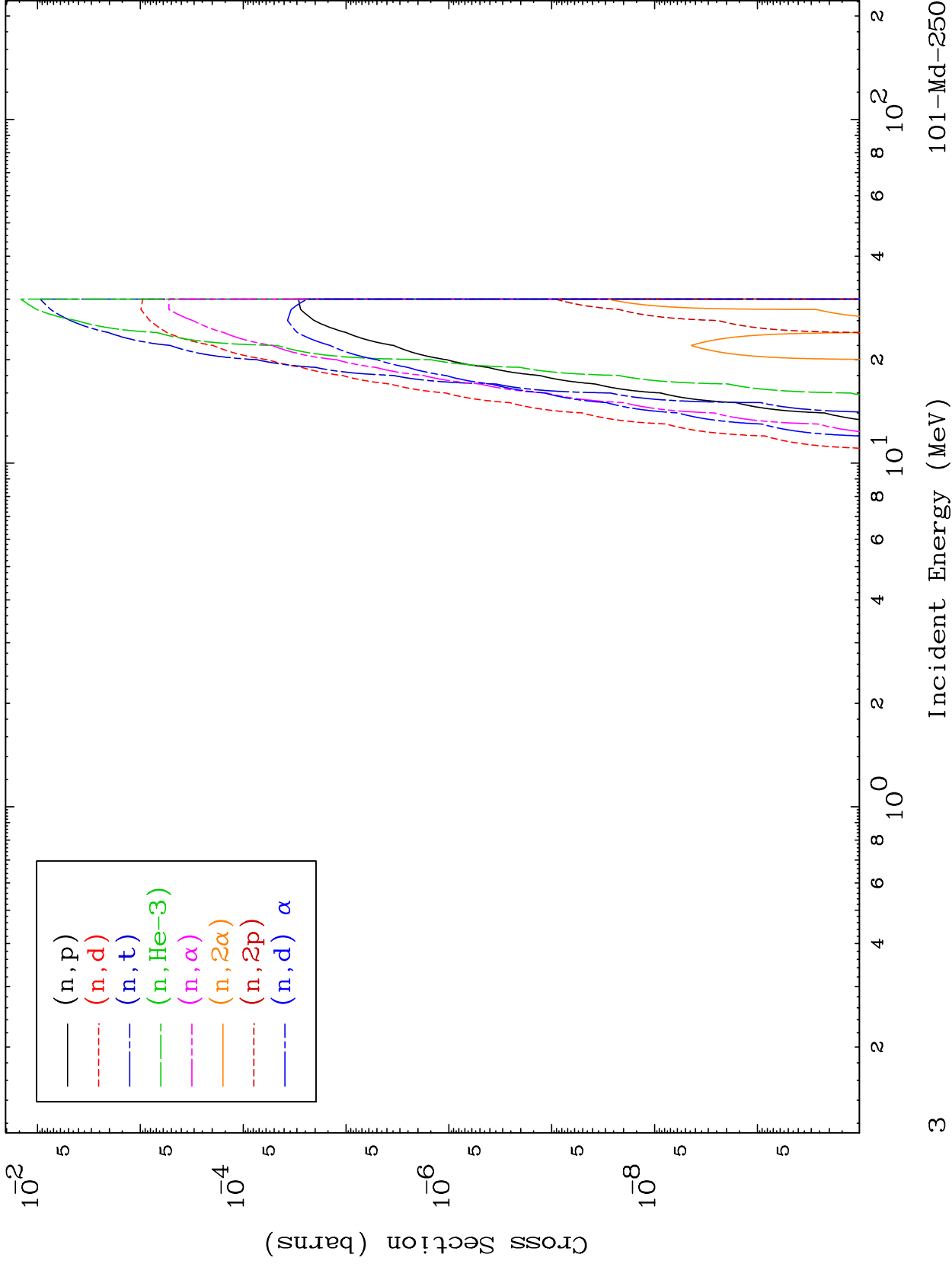
101-Md-250



MAT 9951

He-3 Neutron Absorption  
0 Kelvin Cross Sections

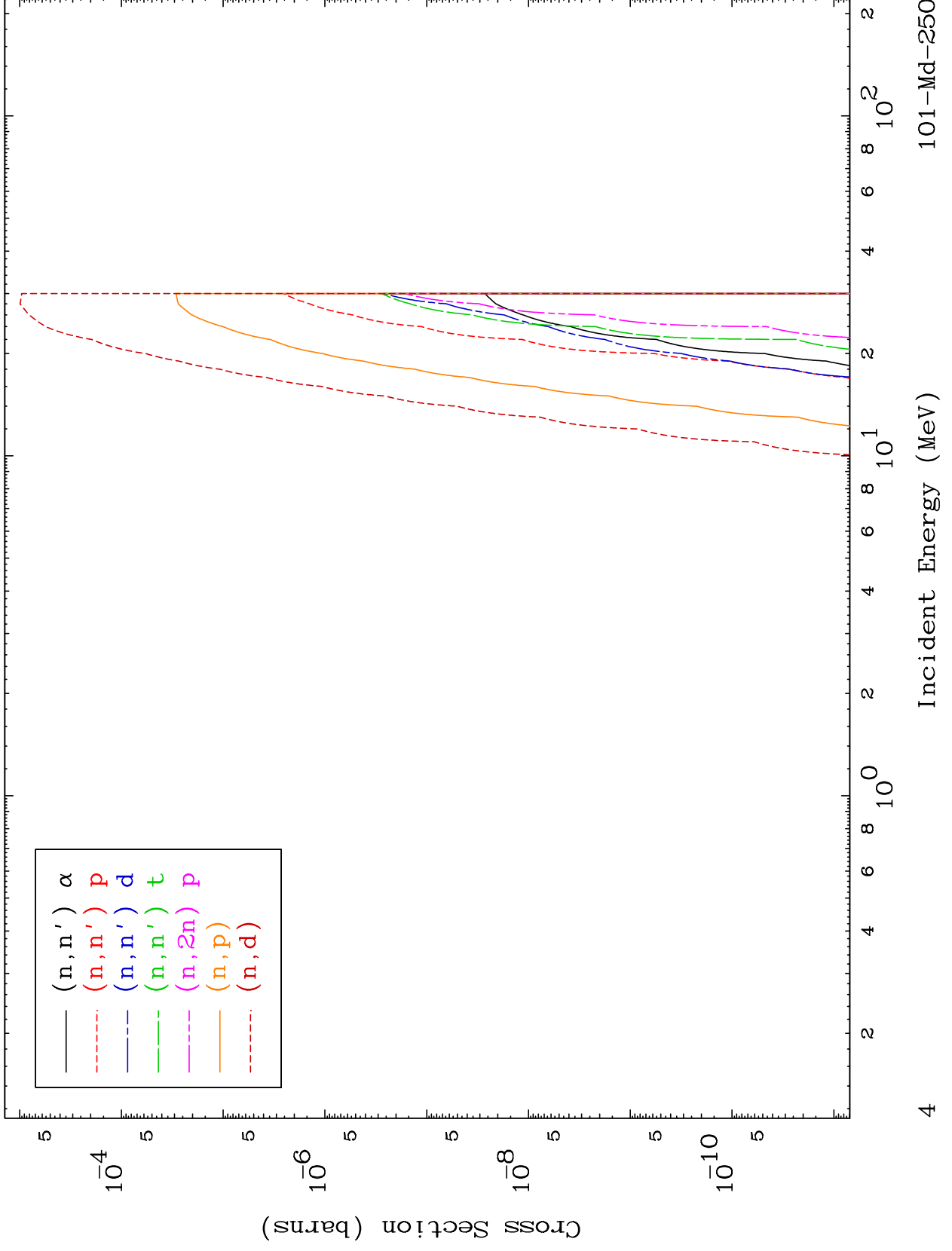
101-Md-250



MAT 9951

He-3 Charged Particle  
0 Kelvin Cross Sections

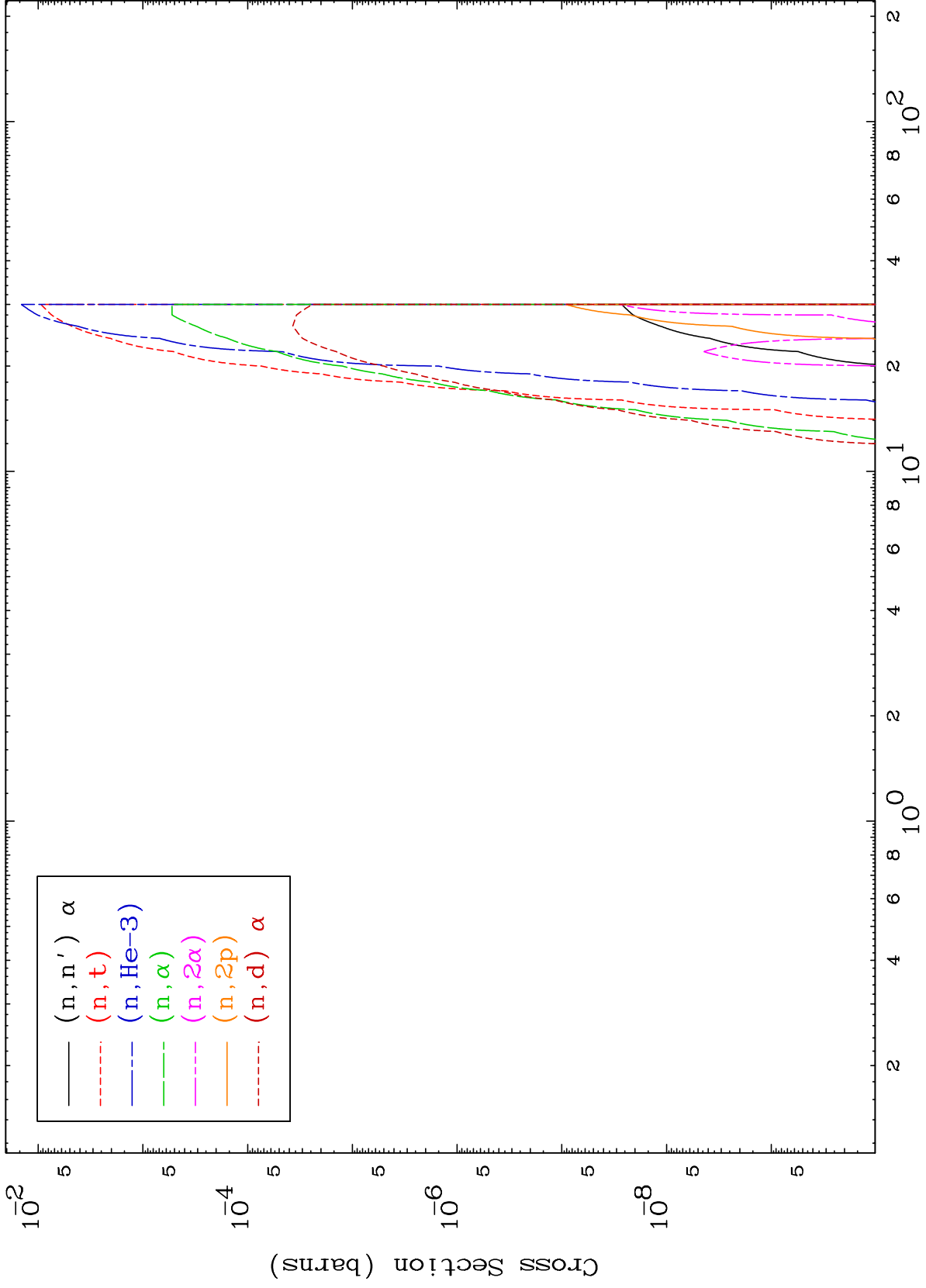
101-Md-250



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He-3 Charged Particle  
0 Kelvin Cross Sections

101-Md-250

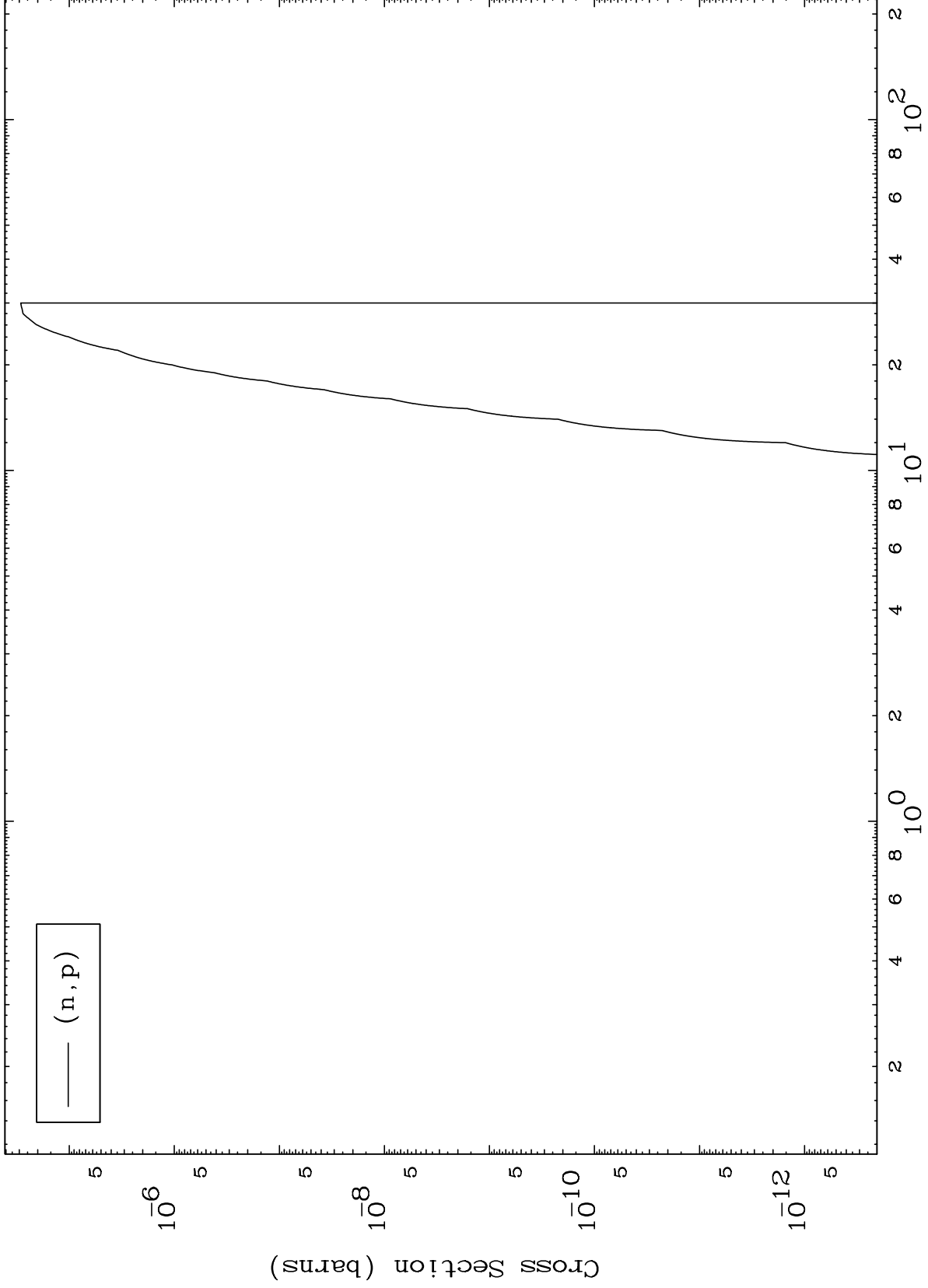


MAT 9951

(He-3,p) Levels

101-Md-250

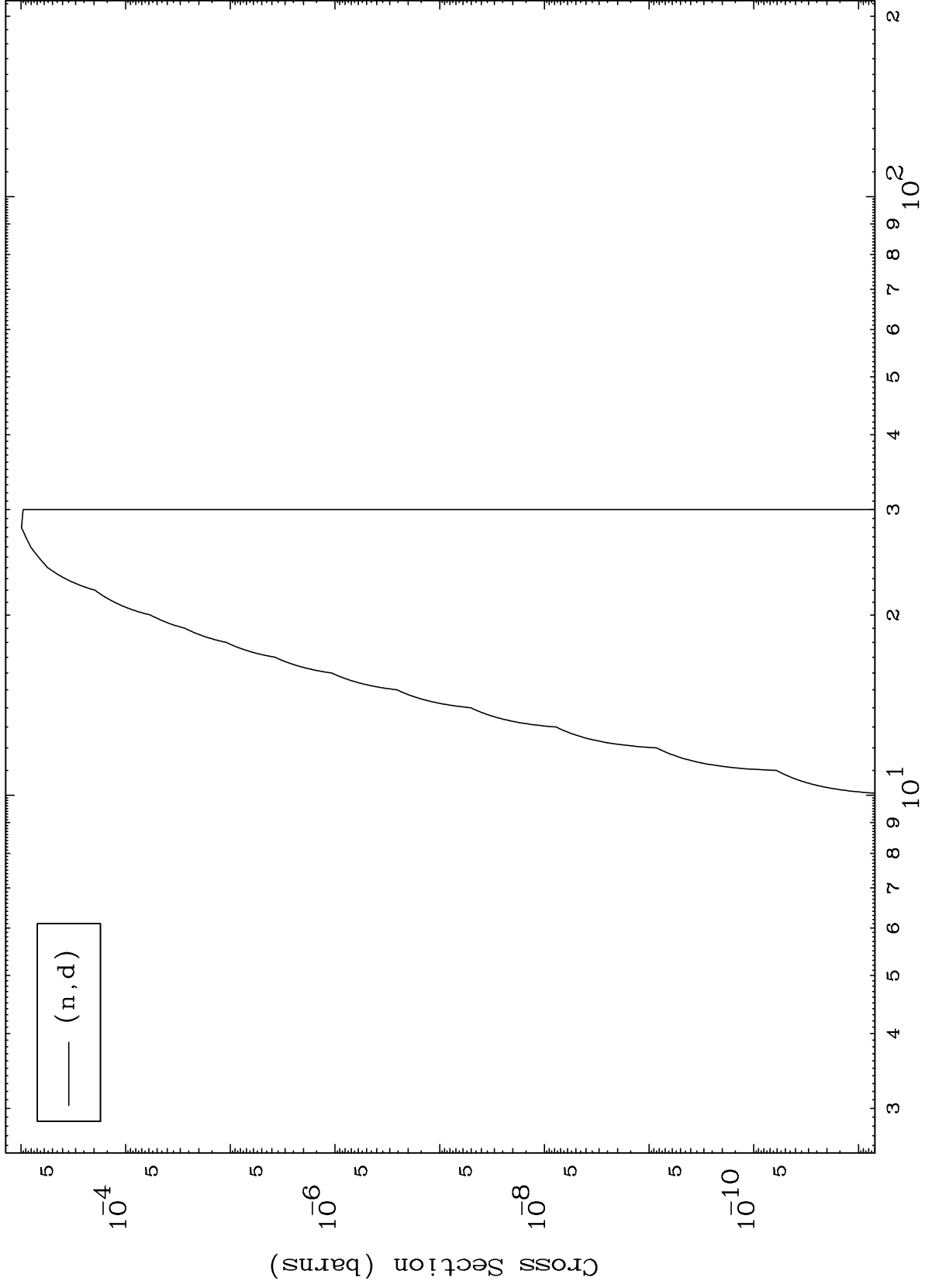
0 Kelvin Cross Sections



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(He-3,d) Levels  
0 Kelvin Cross Sections

101-Md-250



7

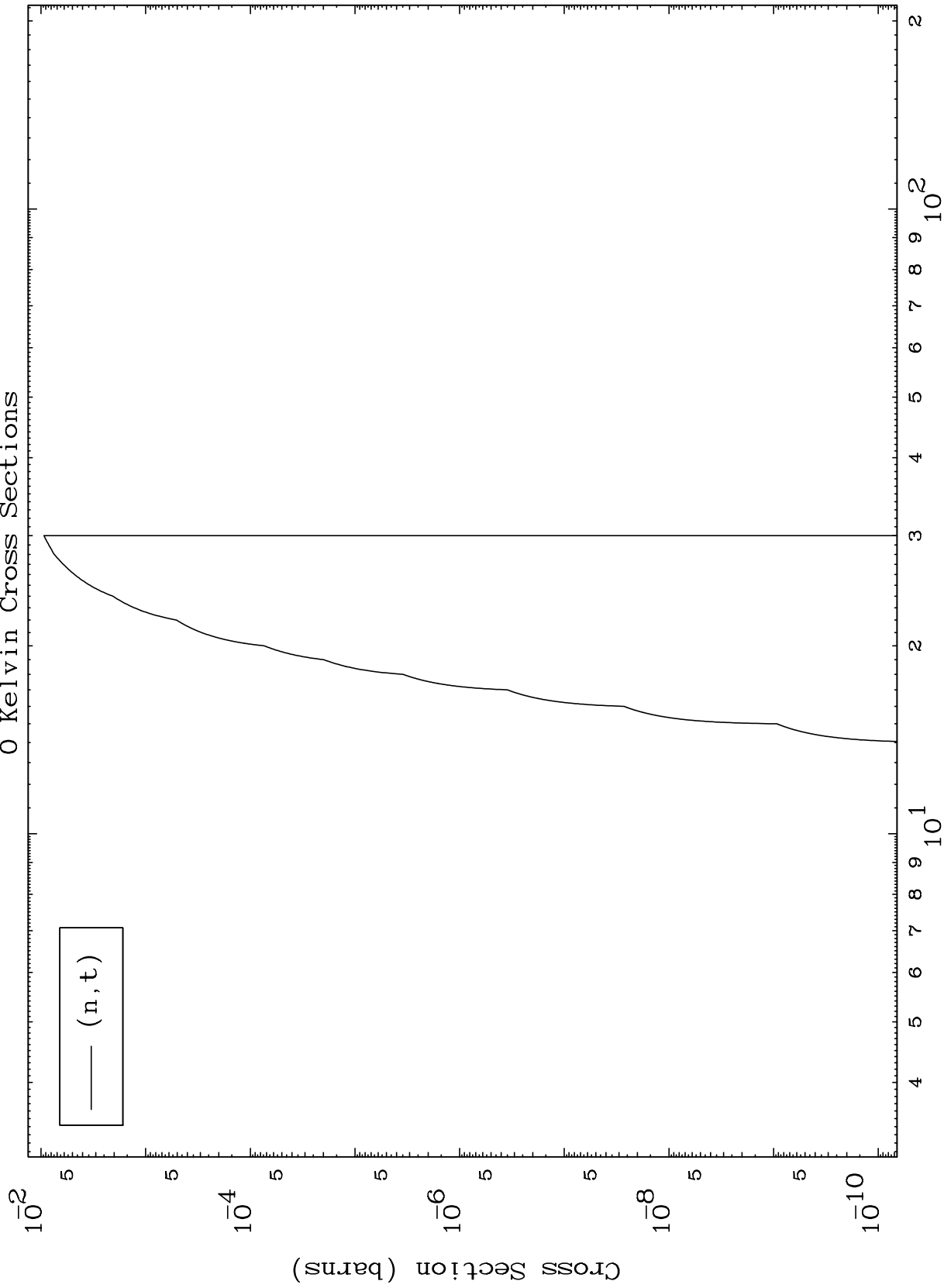
Incident Energy (MeV)

101-Md-250

MAT 9951

101-Md-250

(He-3,t) Levels  
0 Kelvin Cross Sections

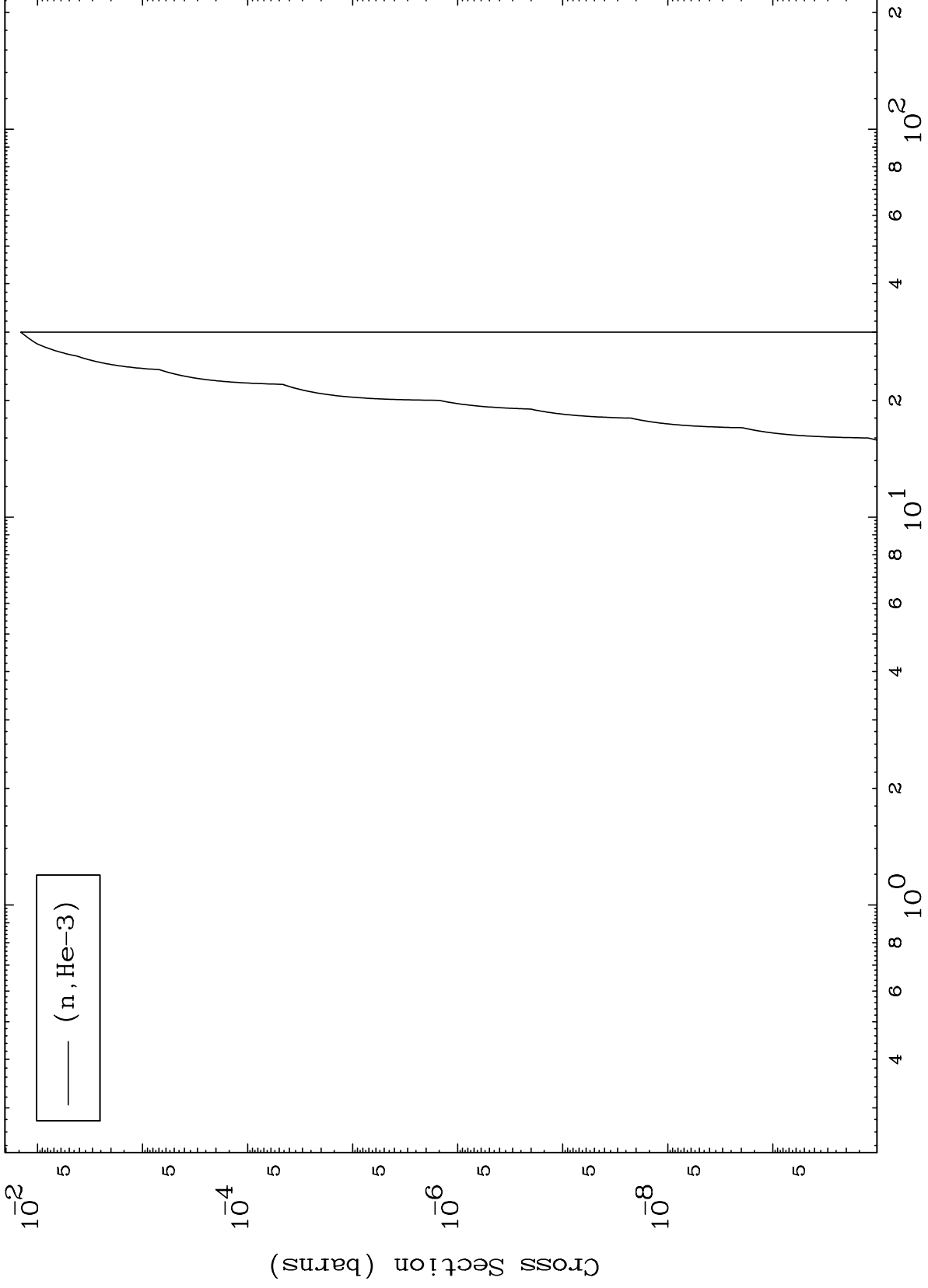


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(He-3, He3) Levels

101-Md-250

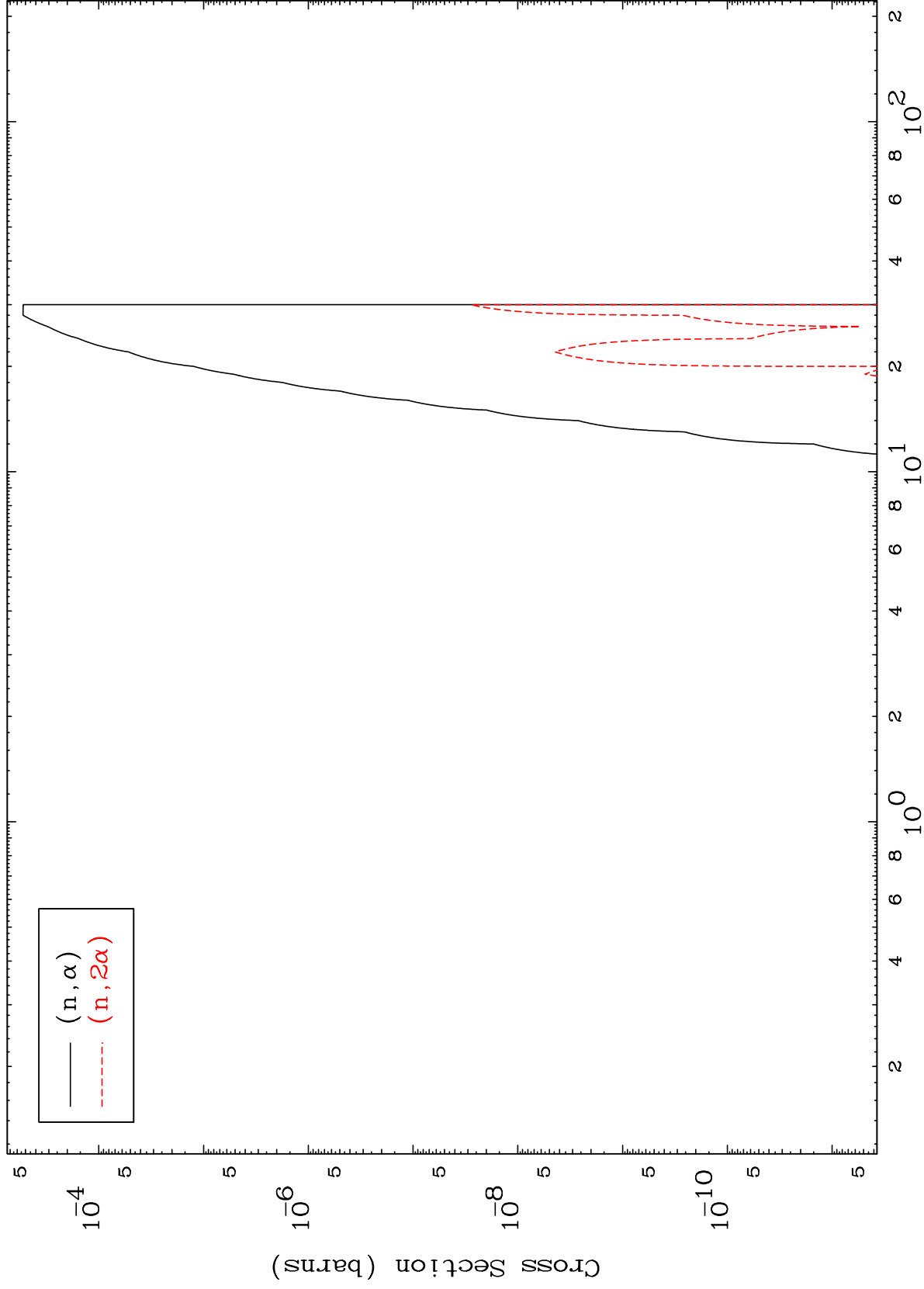
0 Kelvin Cross Sections



MAT 9951

101-Md-250

(He-3,  $\alpha$ ) Levels  
0 Kelvin Cross Sections



10

Incident Energy (MeV)

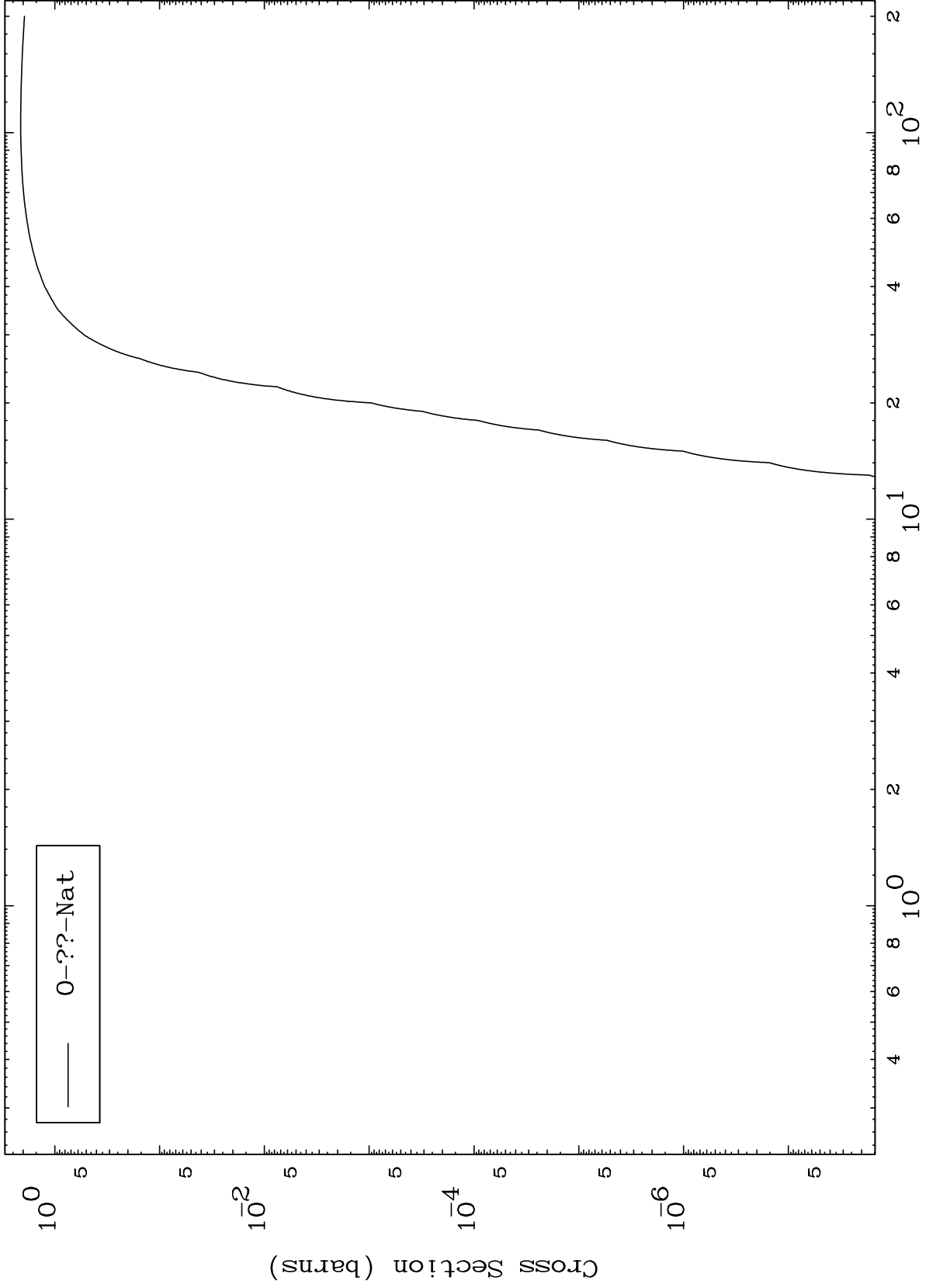
101-Md-250

MAT 9951

Fission

101-Md-250

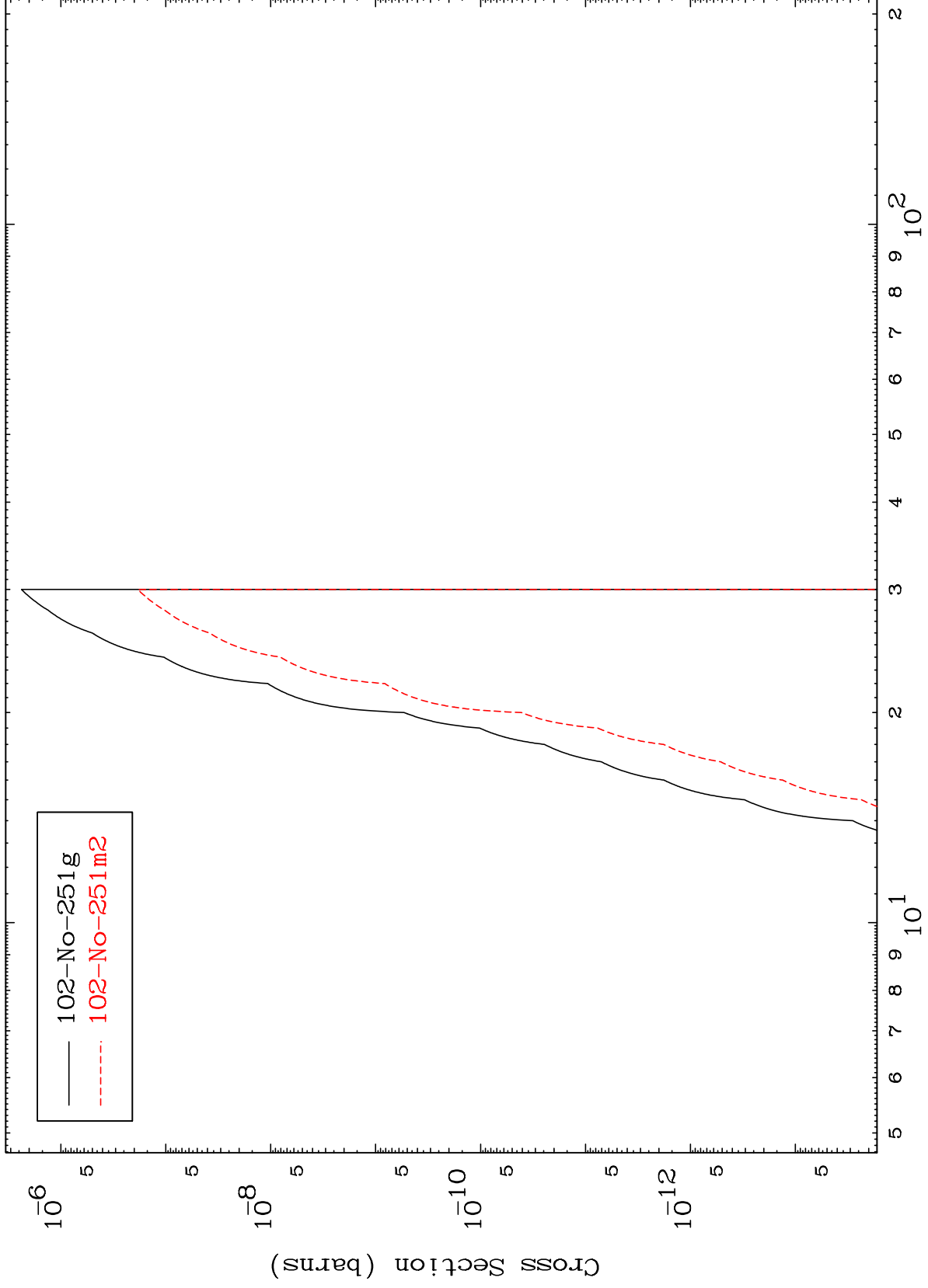
Radionuclide Production Cross Section



MAT 9951

101-Md-250

(n,n') p  
Radionuclide Production Cross Section



12

Incident Energy (MeV)

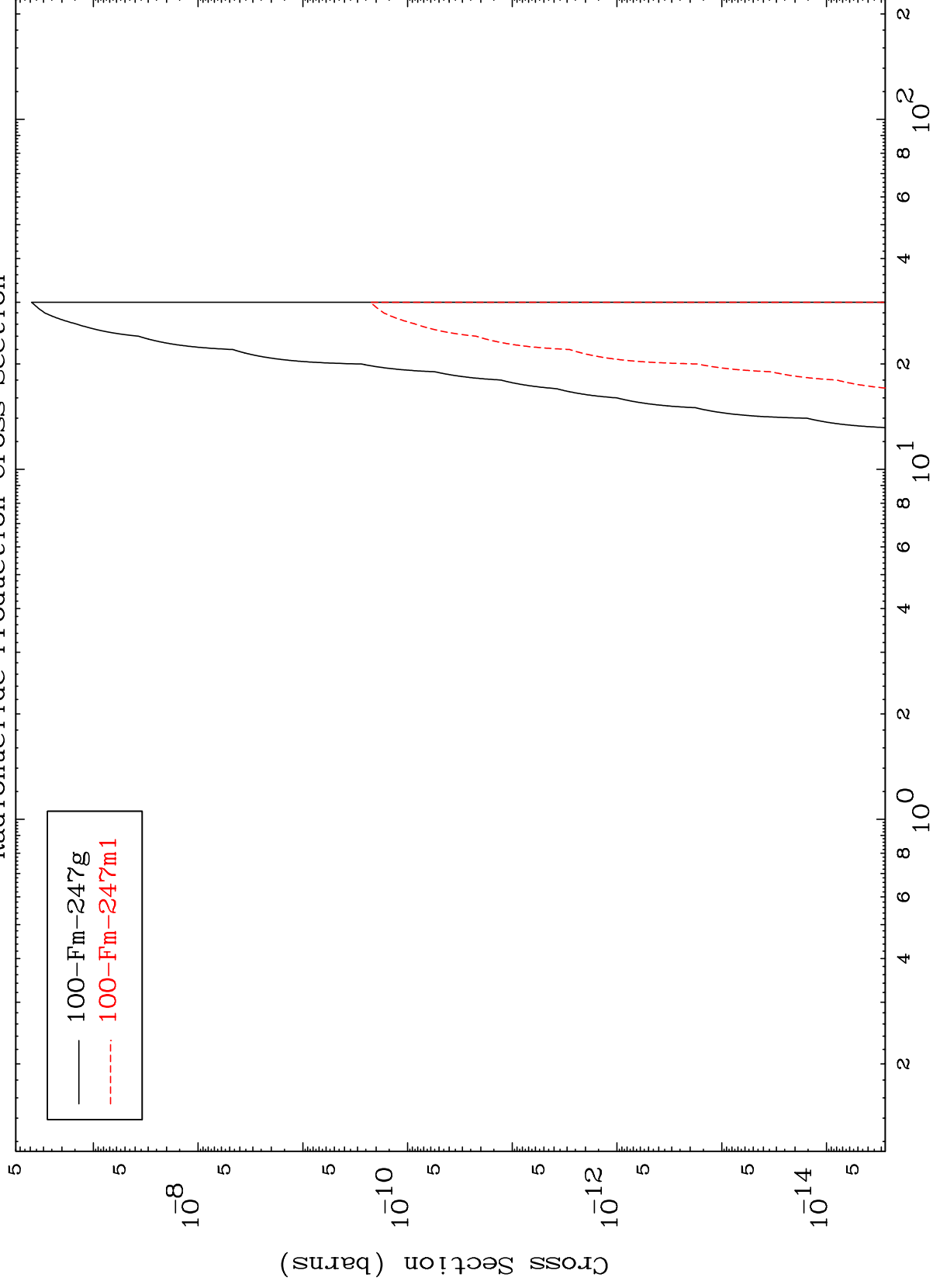
101-Md-250

MAT 9951

(n,n') p  $\alpha$

101-Md-250

Radionuclide Production Cross Section



13

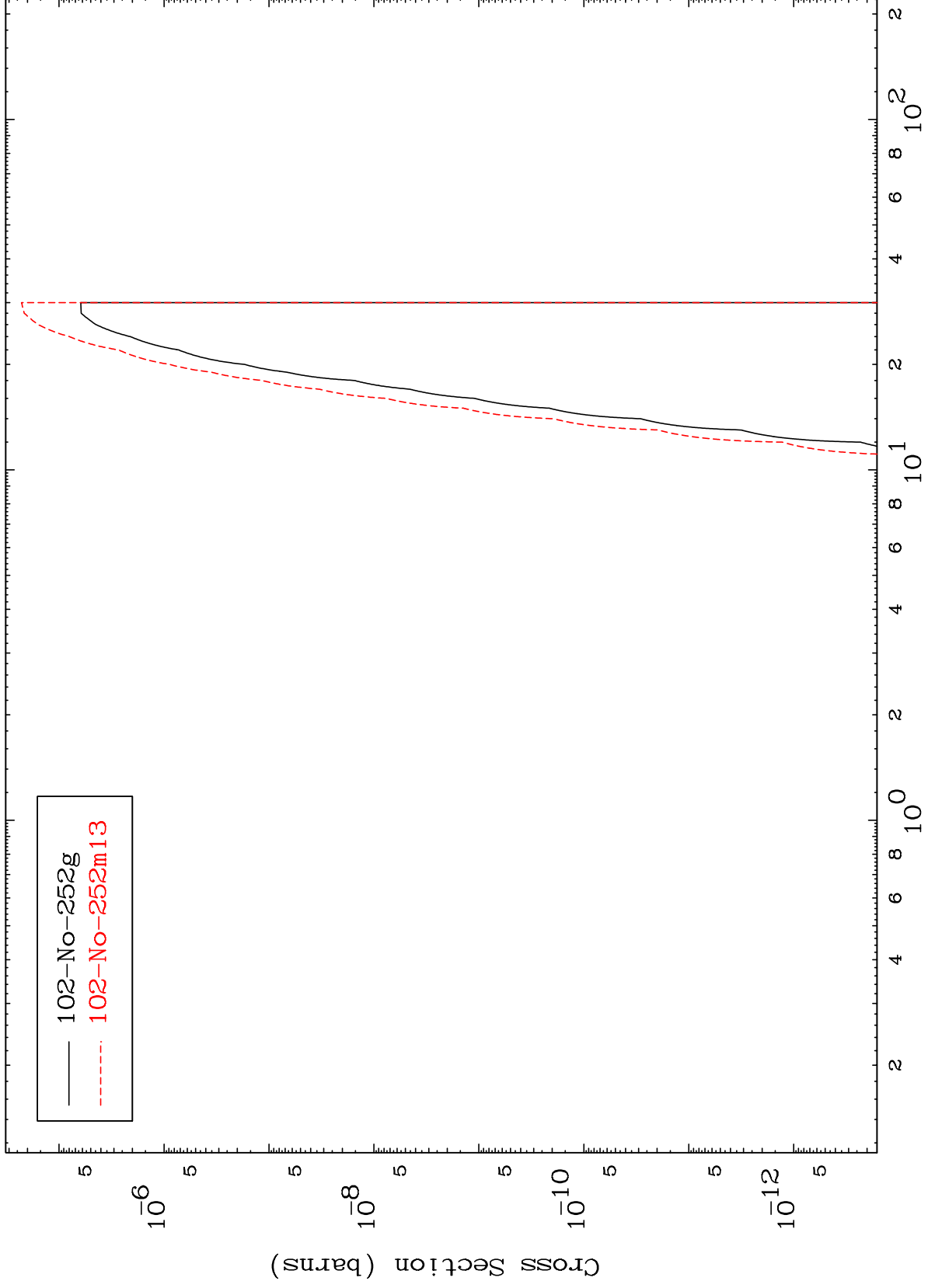
Incident Energy (MeV)

101-Md-250

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101-Md-250

(n,p)  
Radionuclide Production Cross Section



101-Md-250

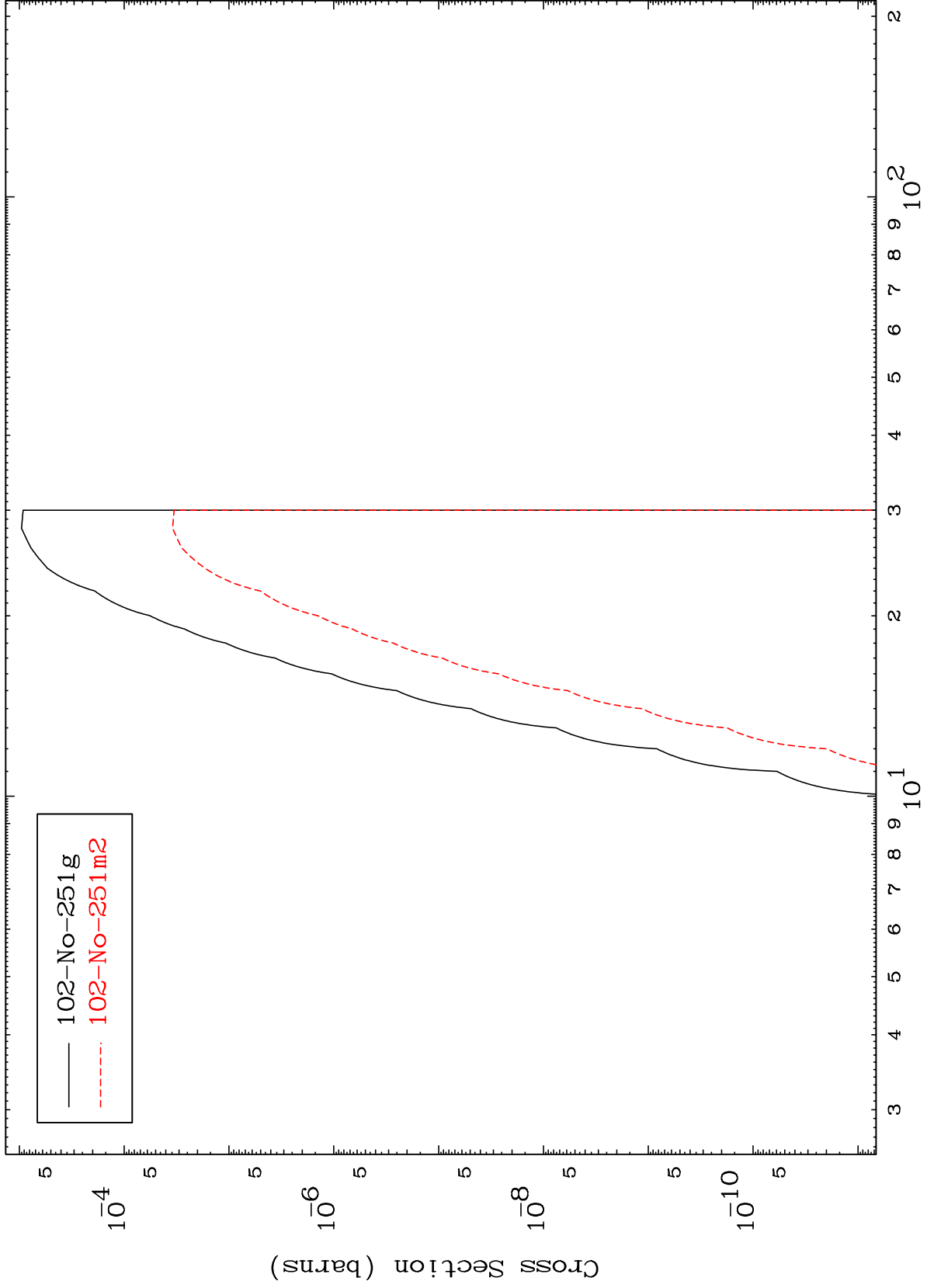
Incident Energy (MeV)

14

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101-Md-250

(n,d)  
Radionuclide Production Cross Section



15

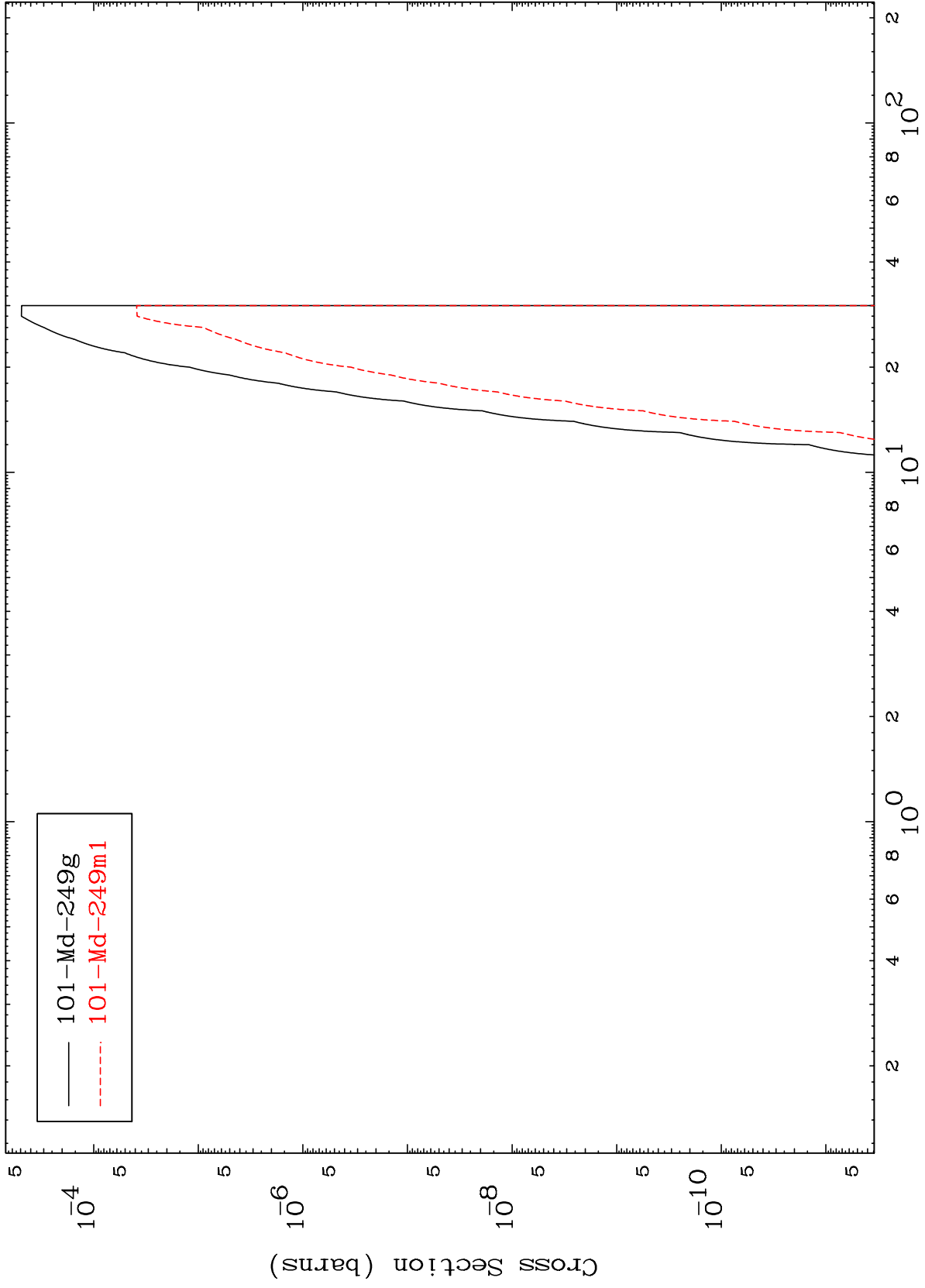
Incident Energy (MeV)

101-Md-250

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101-Md-250

(n,  $\alpha$ )  
Radionuclide Production Cross Section



101-Md-249g  
101-Md-249m1

101-Md-250

Incident Energy (MeV)

16

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(n,d)  $\alpha$

101-Md-250

