

Program EVALPLOT  
(Version 2021-1)

by

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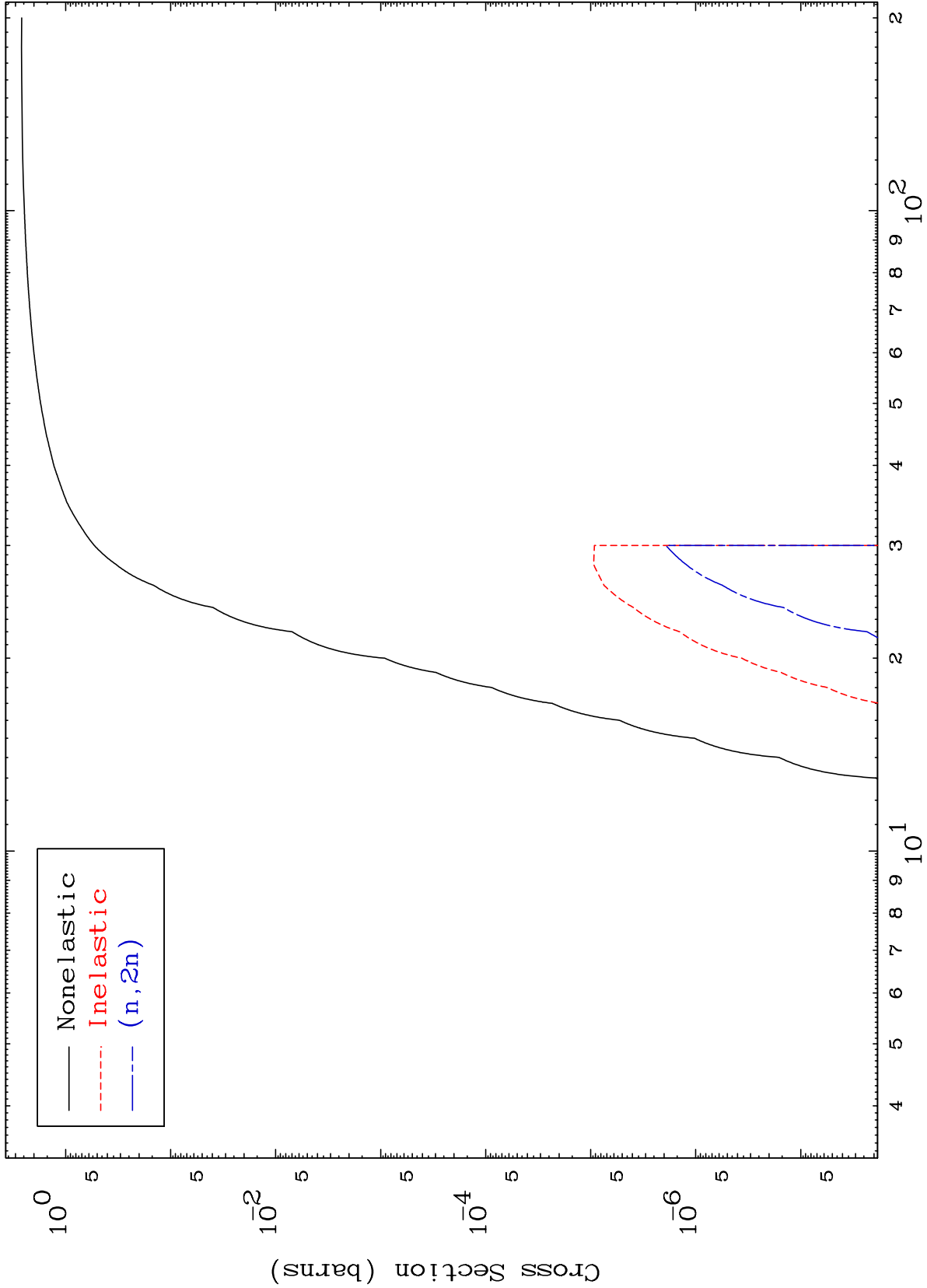
Press Mouse Button to Start

MAT 9970

He-3 Major

102-No-252

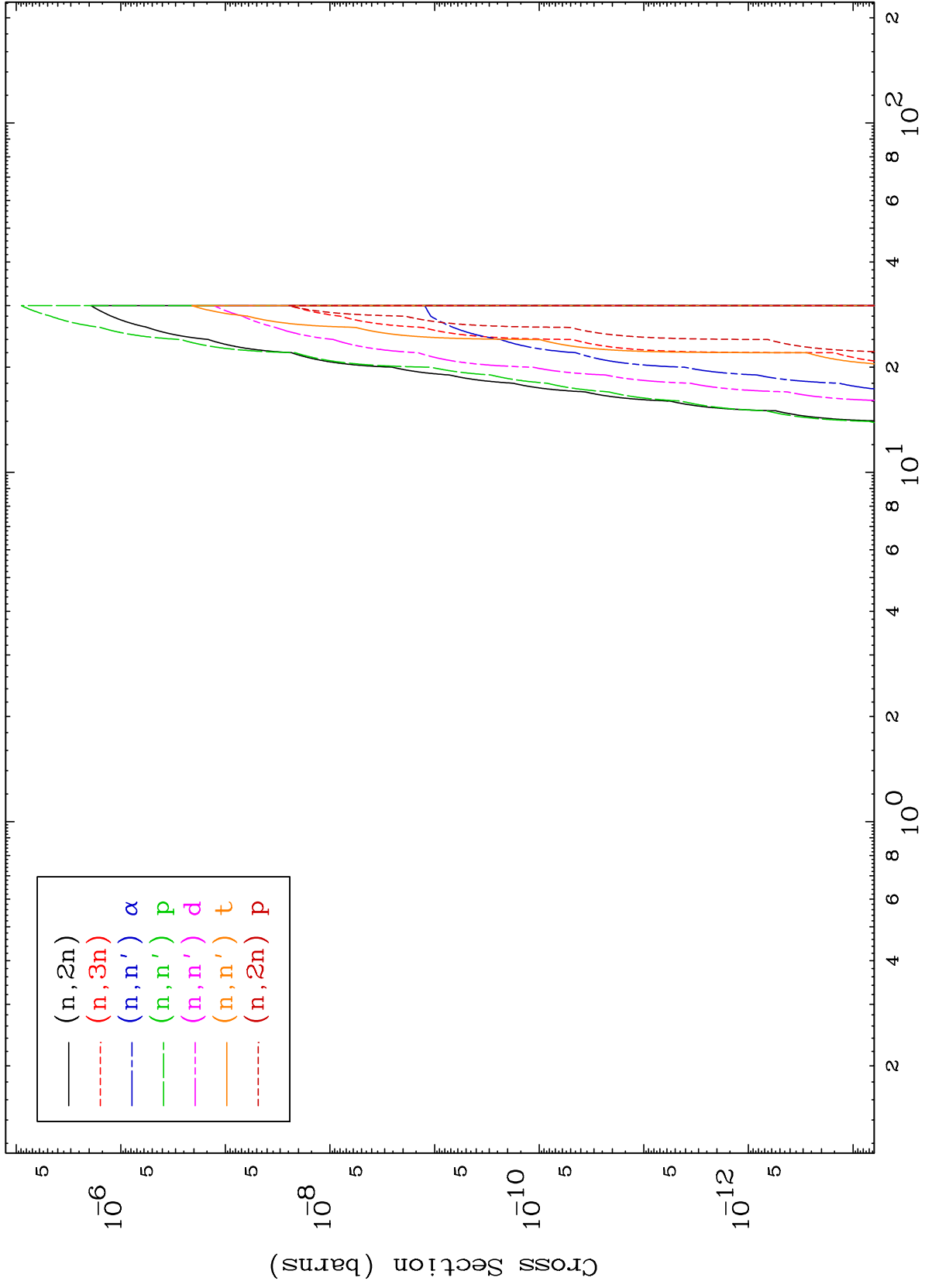
0 Kelvin Cross Sections



MAT 9970

He-3 Neutron Absorption  
0 Kelvin Cross Sections

102-No-252



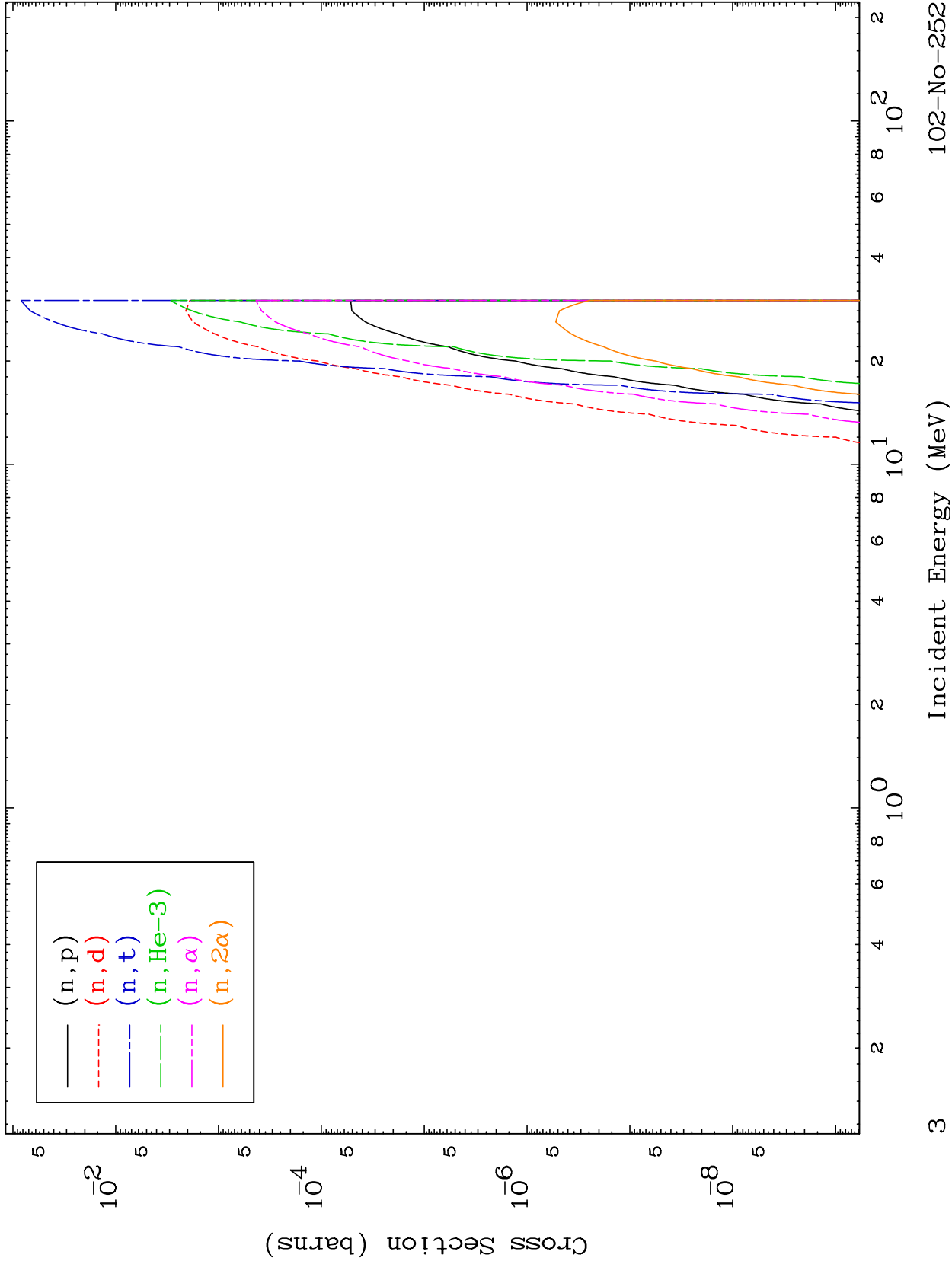
102-No-252

Incident Energy (MeV)

MAT 9970

He-3 Neutron Absorption  
0 Kelvin Cross Sections

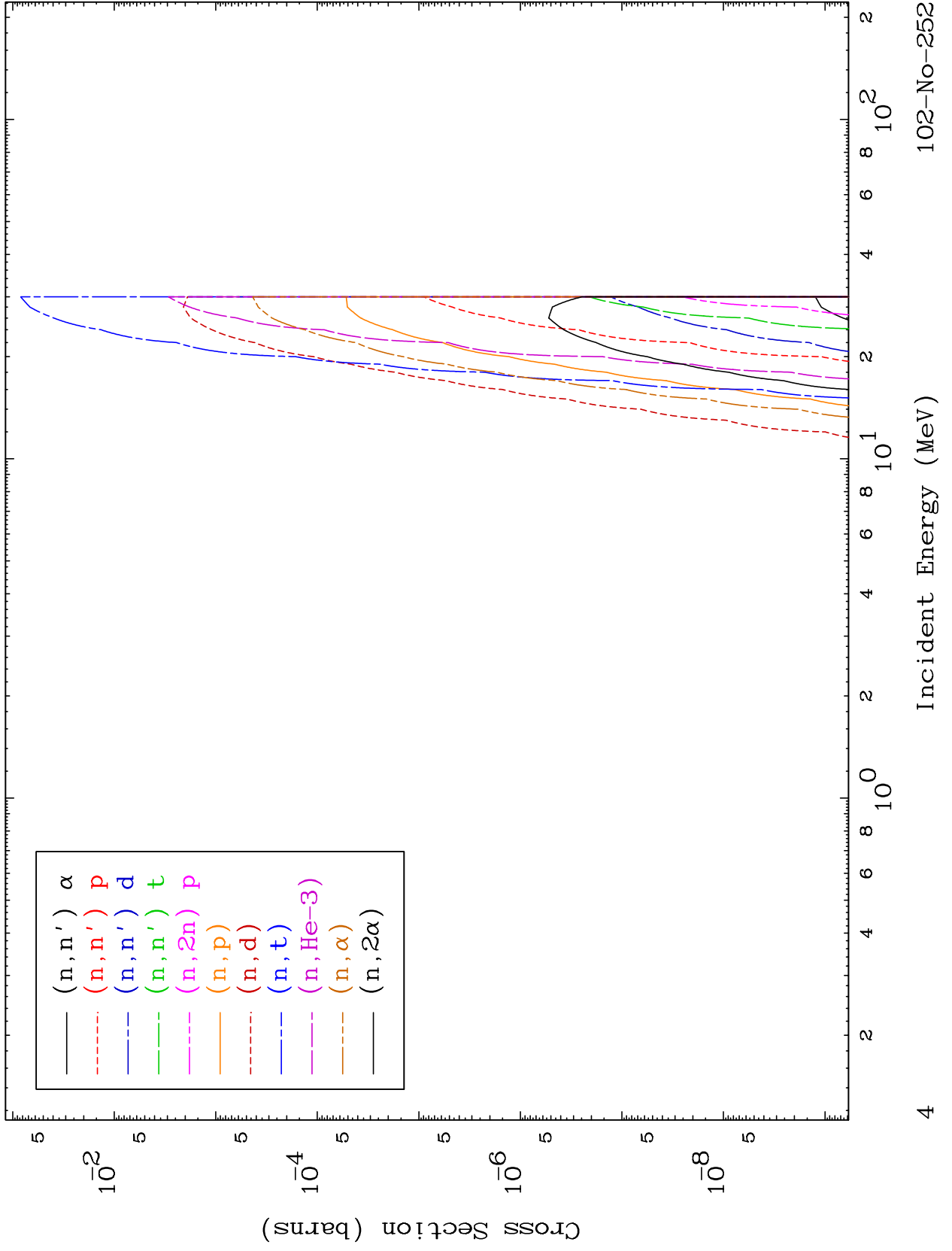
102-No-252



MAT 9970

He-3 Charged Particle  
0 Kelvin Cross Sections

102-No-252

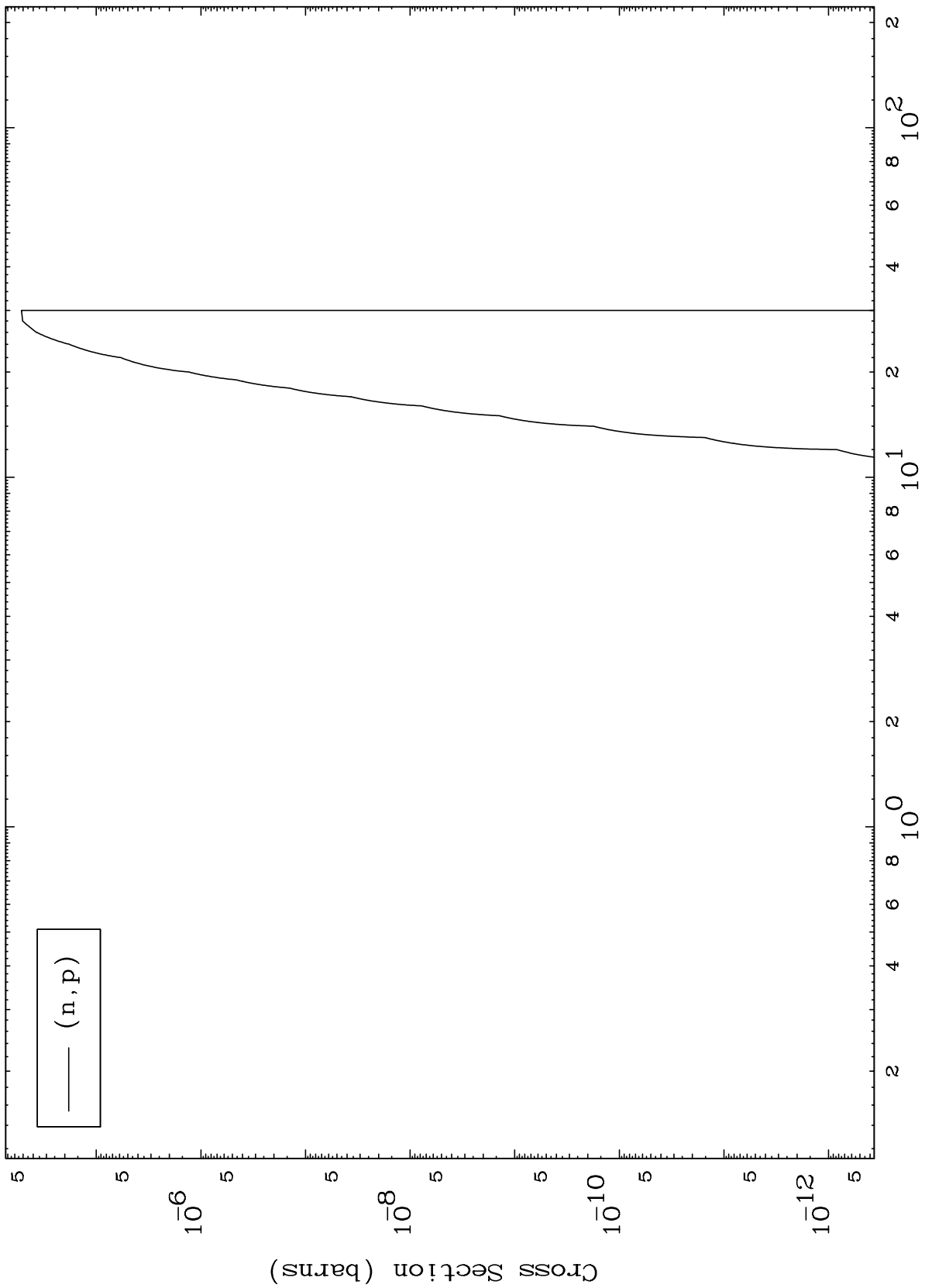


MAT 9970

(He-3,p) Levels

102-No-252

0 Kelvin Cross Sections

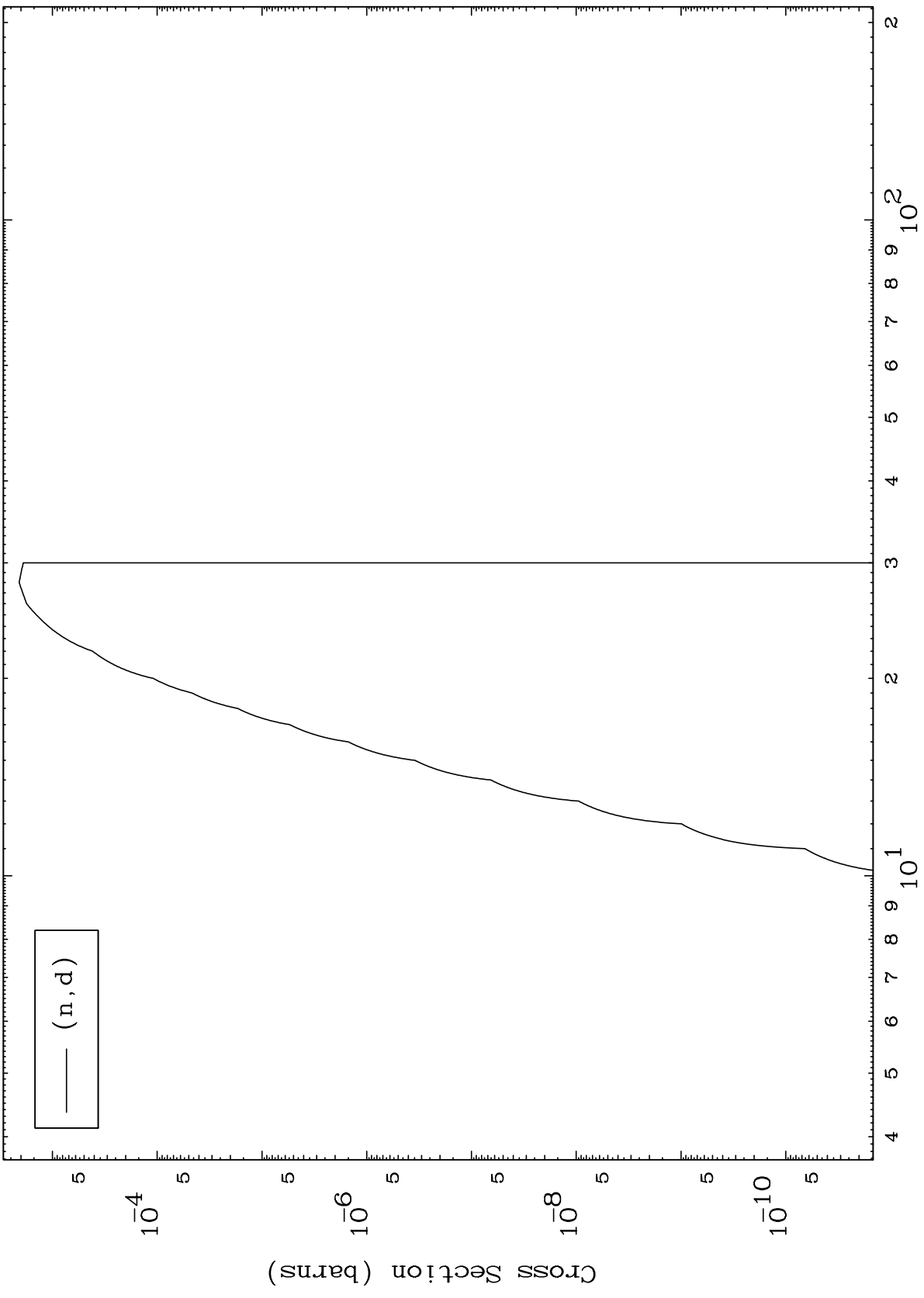


MAT 9970

(He-3,d) Levels

102-No-252

0 Kelvin Cross Sections



Incident Energy (MeV)

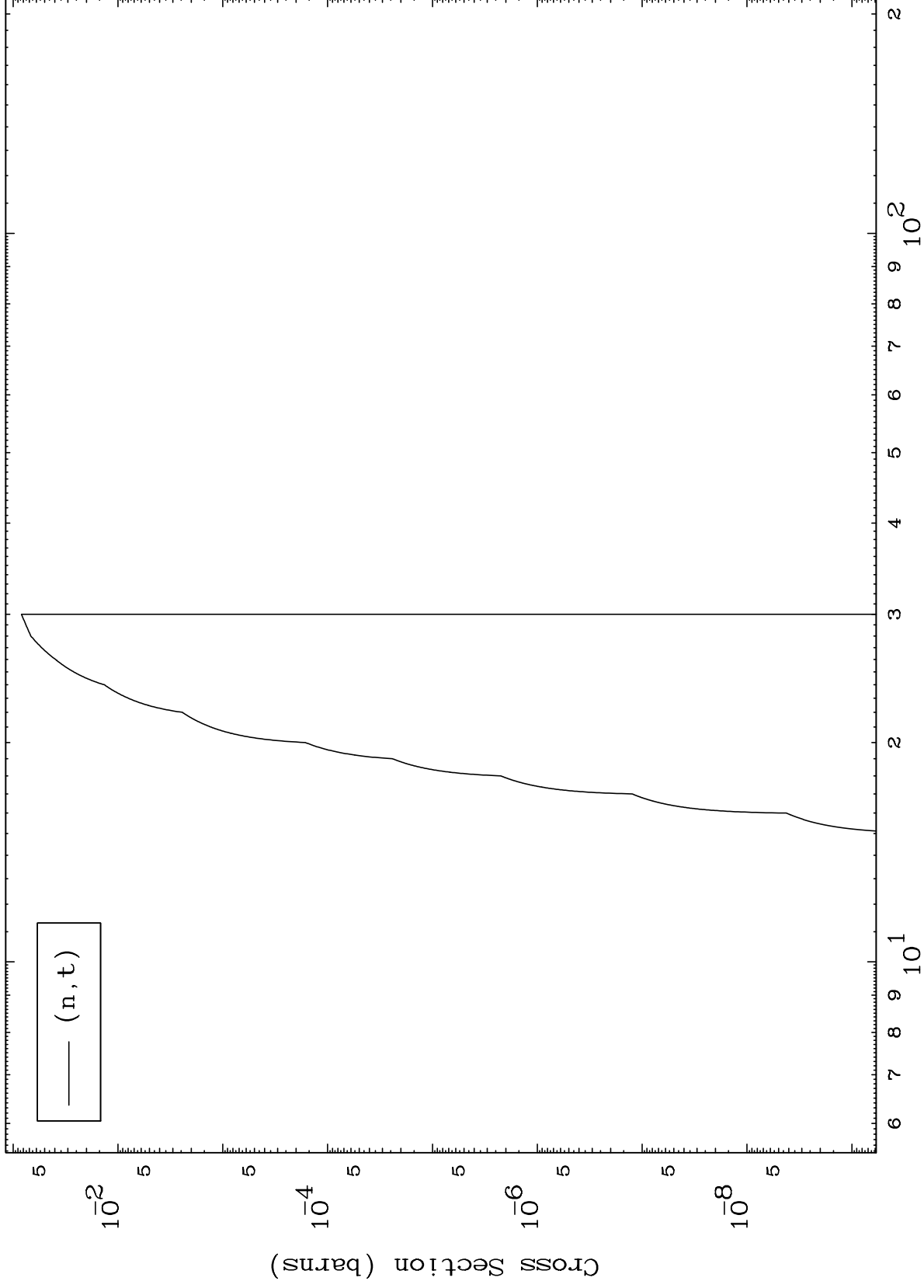
102-No-252

6

MAT 9970

(He-3,t) Levels  
0 Kelvin Cross Sections

102-No-252



7

Incident Energy (MeV)

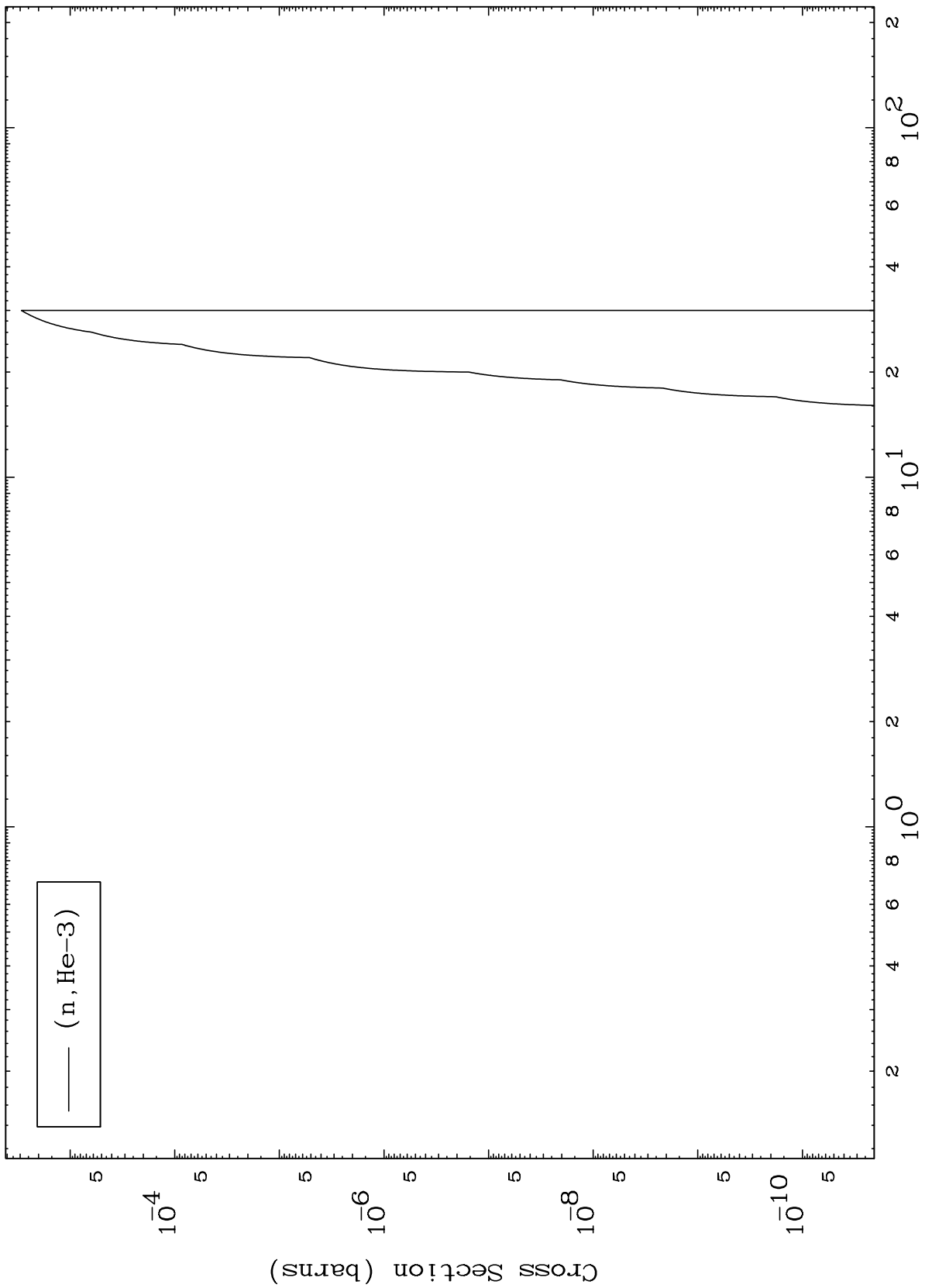
102-No-252

MAT 9970

(He-3, He3) Levels

102-No-252

0 Kelvin Cross Sections

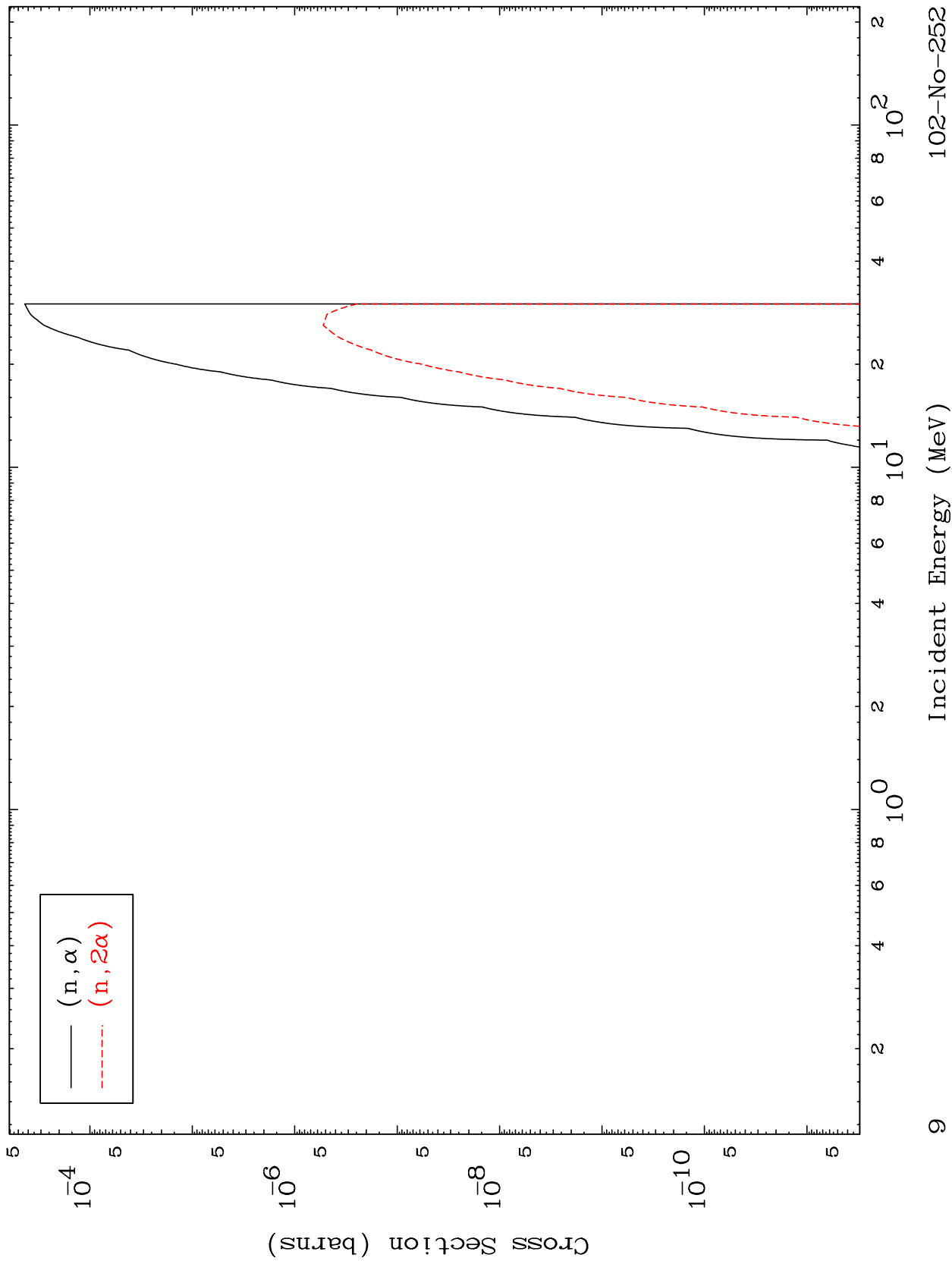


MAT 9970

(He-3,  $\alpha$ ) Levels

102-No-252

0 Kelvin Cross Sections

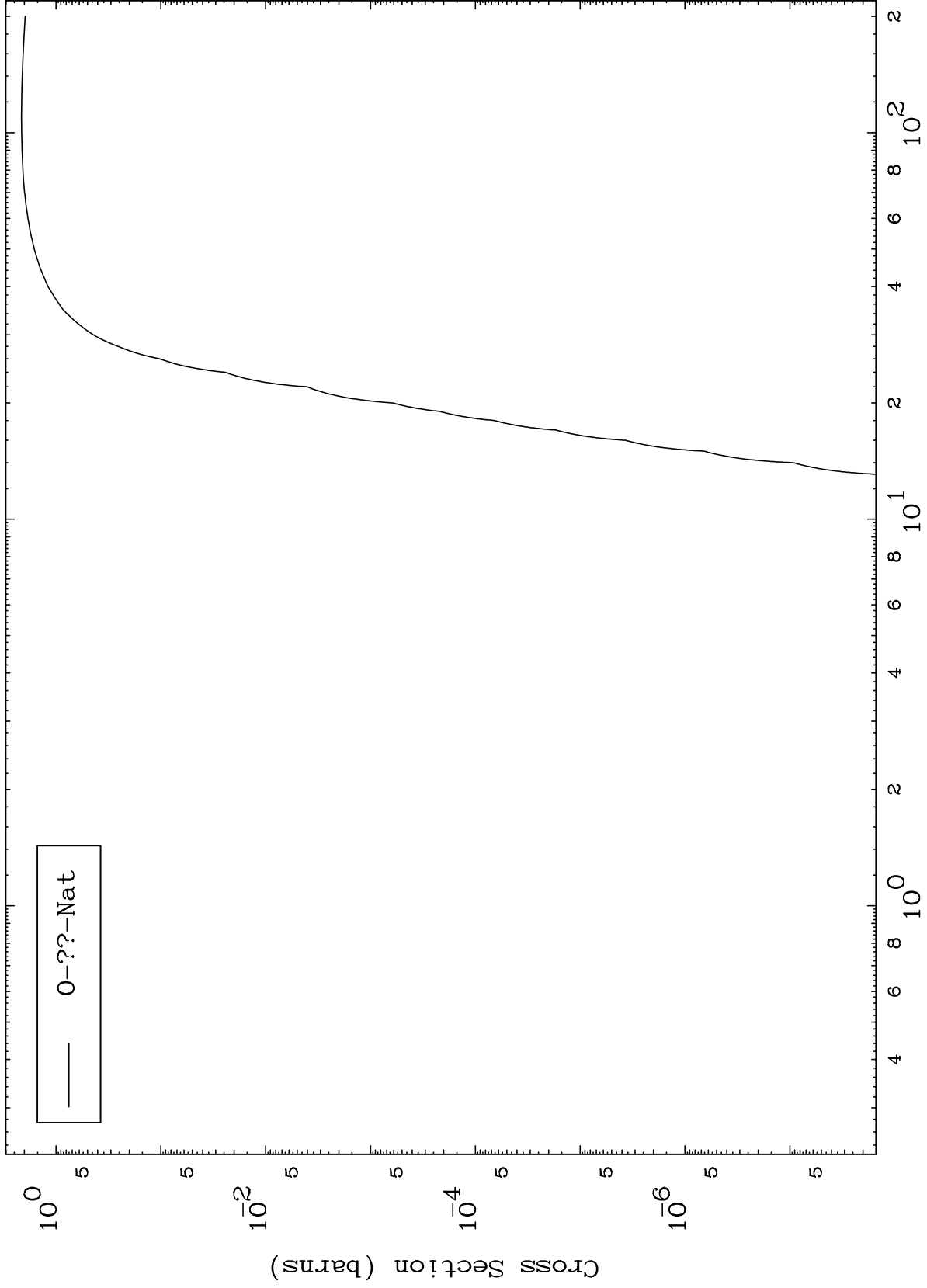


MAT 9970

Fission

102-No-252

Radionuclide Production Cross Section



10

Incident Energy (MeV)

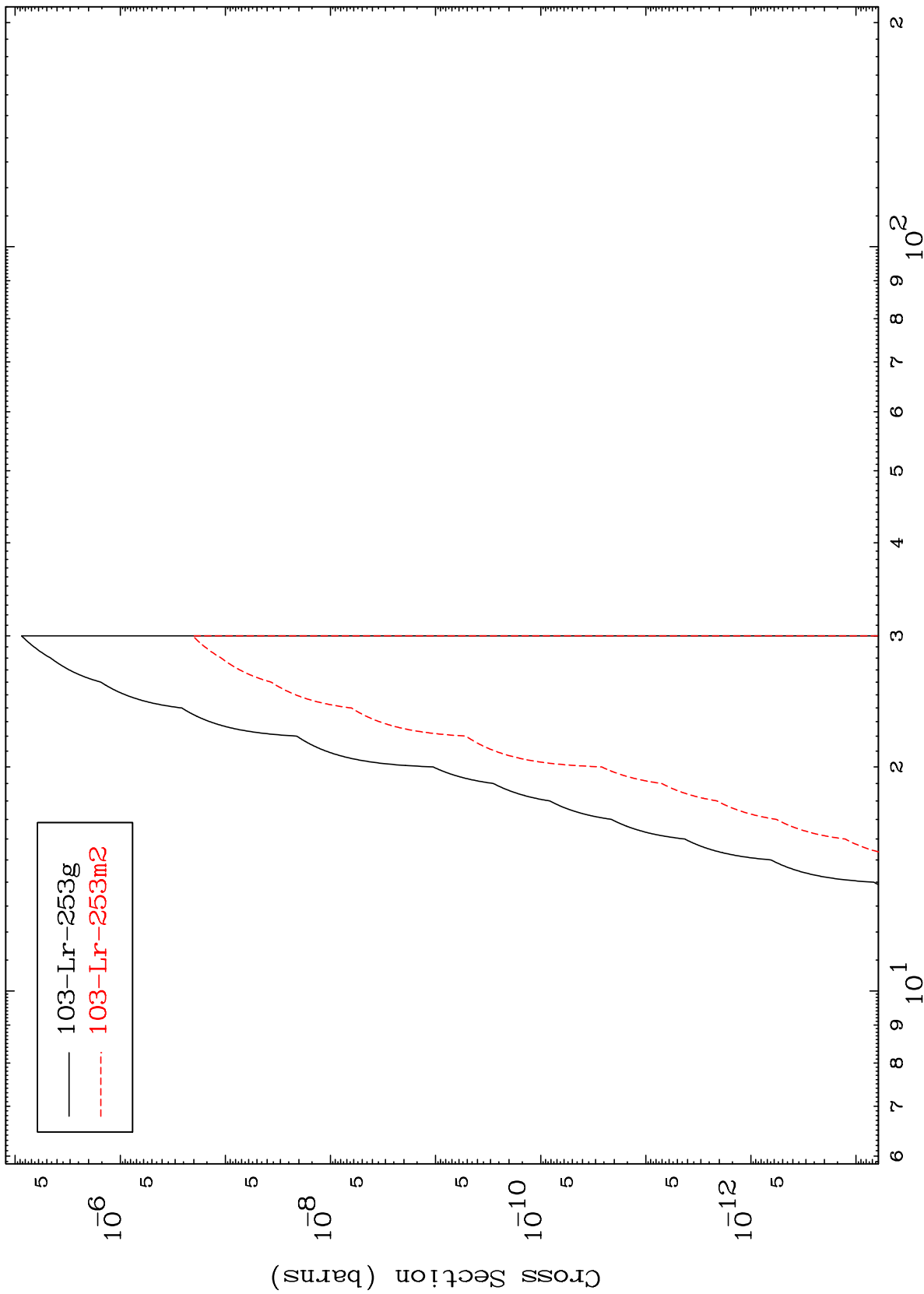
102-No-252

MAT 9970

(n,n') p

102-No-252

Radionuclide Production Cross Section



11

Incident Energy (MeV)

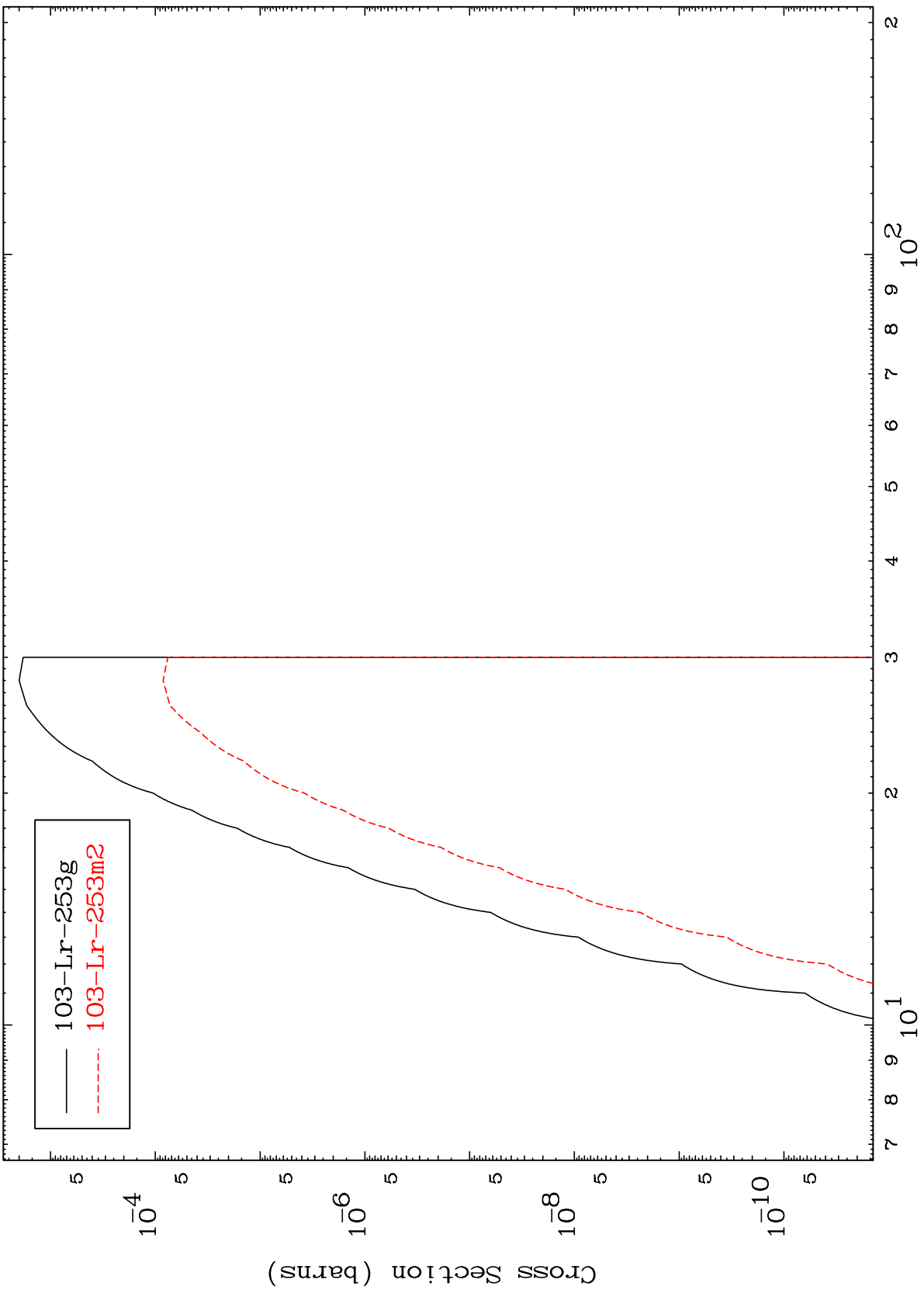
102-No-252

MAT 9970

(n,d)

102-No-252

Radionuclide Production Cross Section



12

Incident Energy (MeV)

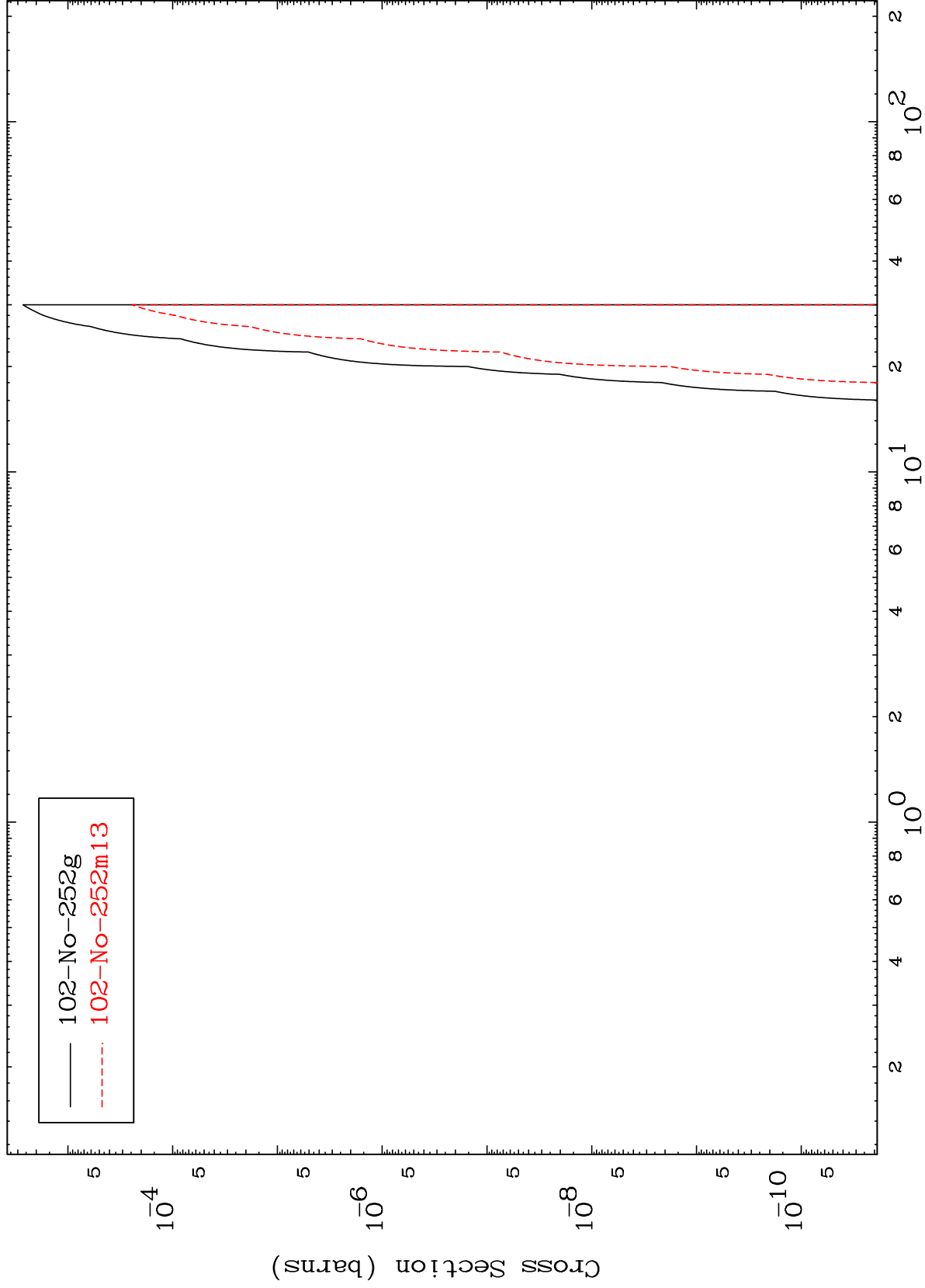
102-No-252

MAT 9970

(n,He-3)

102-No-252

Radionuclide Production Cross Section



13

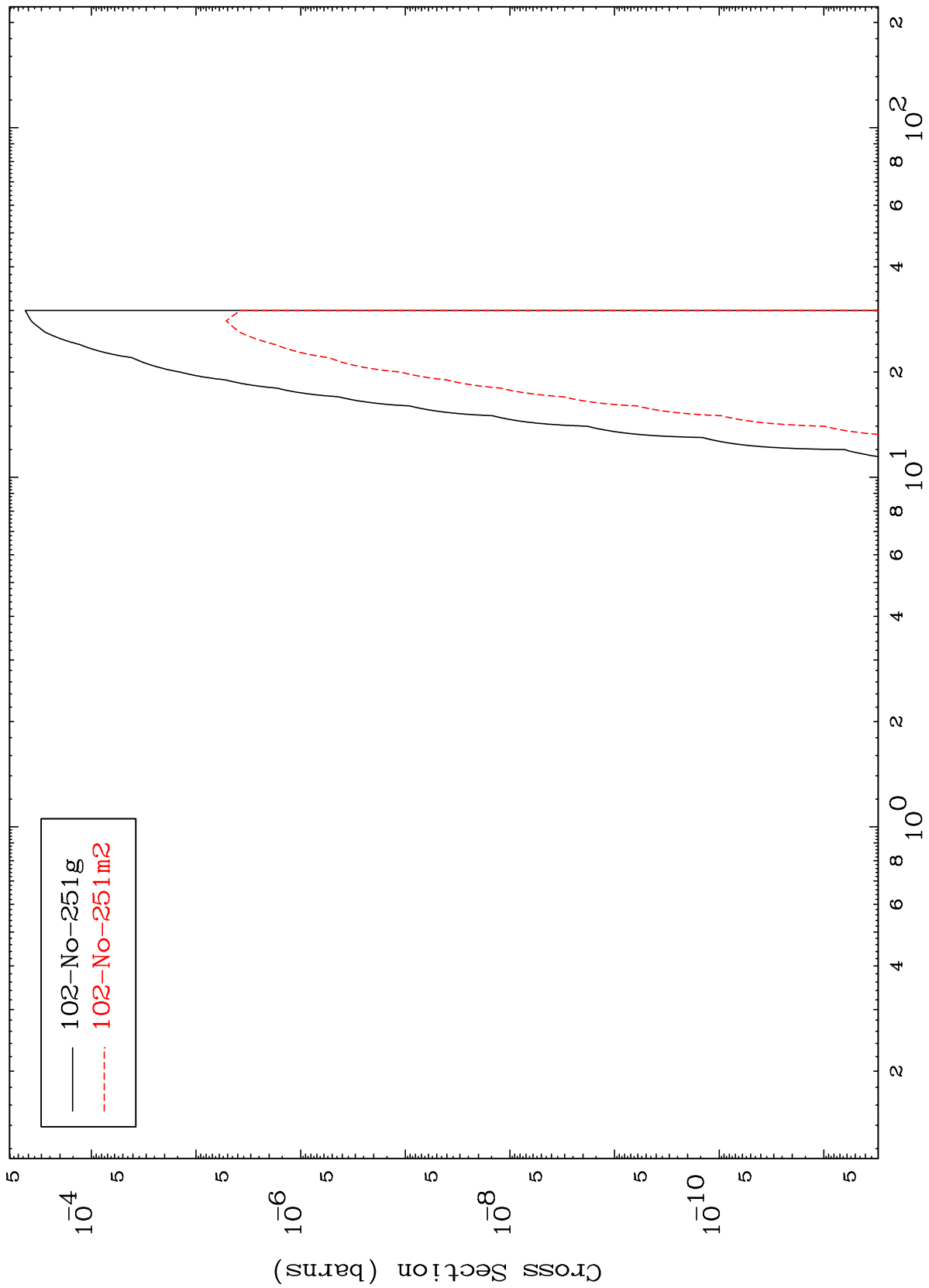
Incident Energy (MeV)

102-No-252

MAT 9970

102-No-252

Radionuclide Production Cross Section  
(n,  $\alpha$ )



102-No-252

Incident Energy (MeV)

MAT 9970

(n,2α)

102-No-252

Radionuclide Production Cross Section

