

Program EVALPLOT
(Version 2021-1)

by

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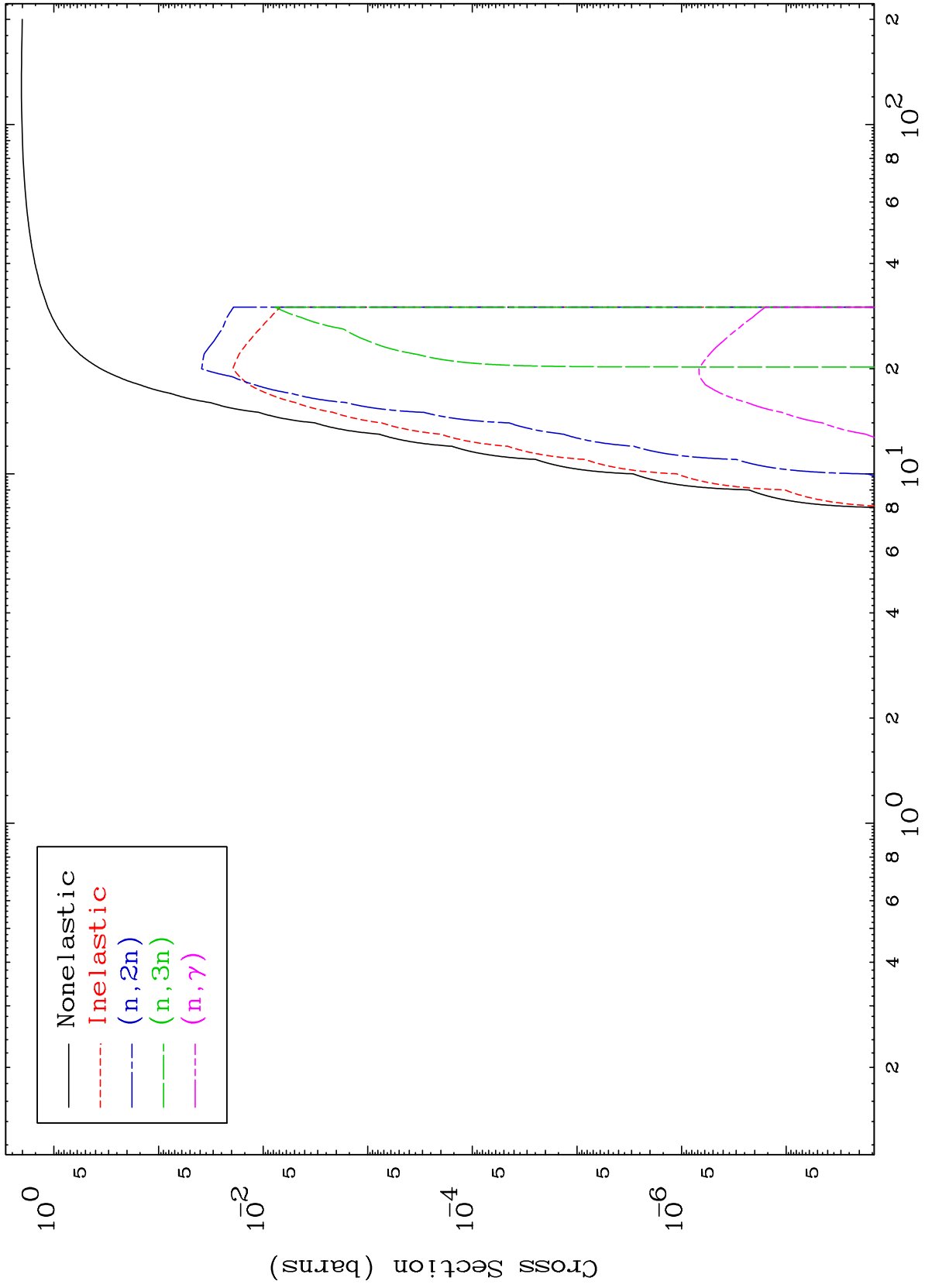
Press Mouse Button to Start

MAT 6223

He-3 Major

0 Kelvin Cross Sections

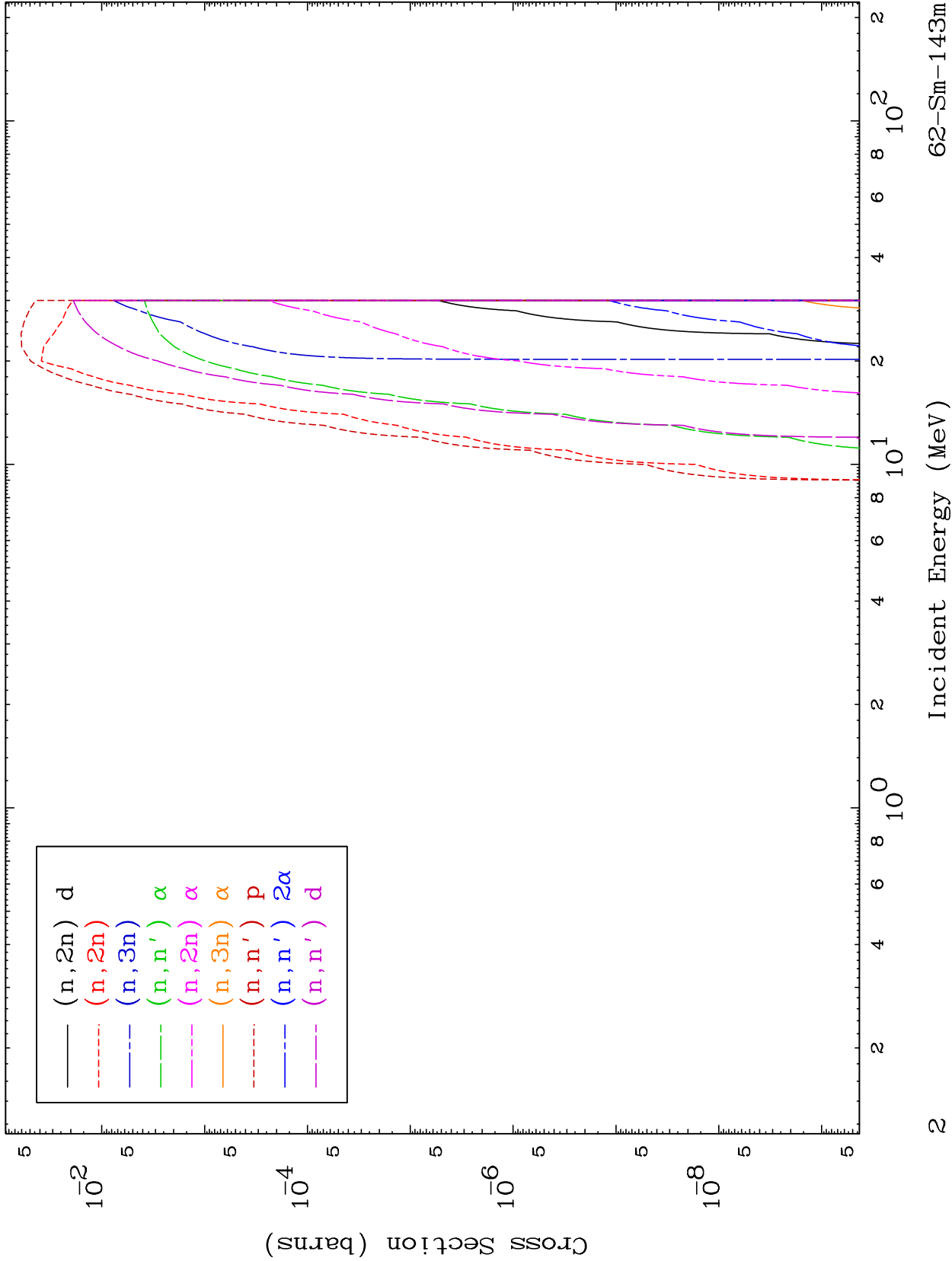
62-Sm-143m



MAT 6223

He-3 Neutron Absorption
0 Kelvin Cross Sections

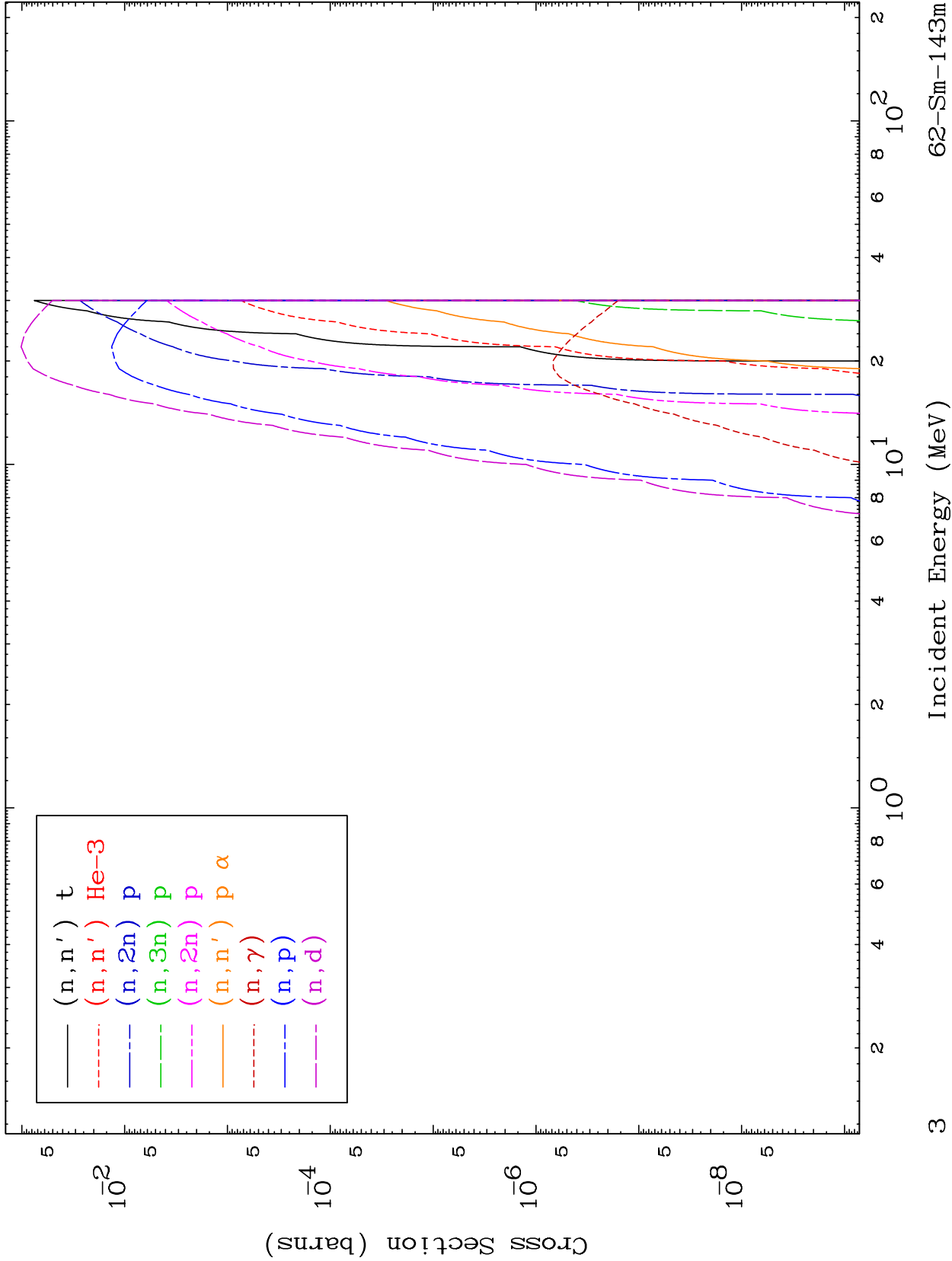
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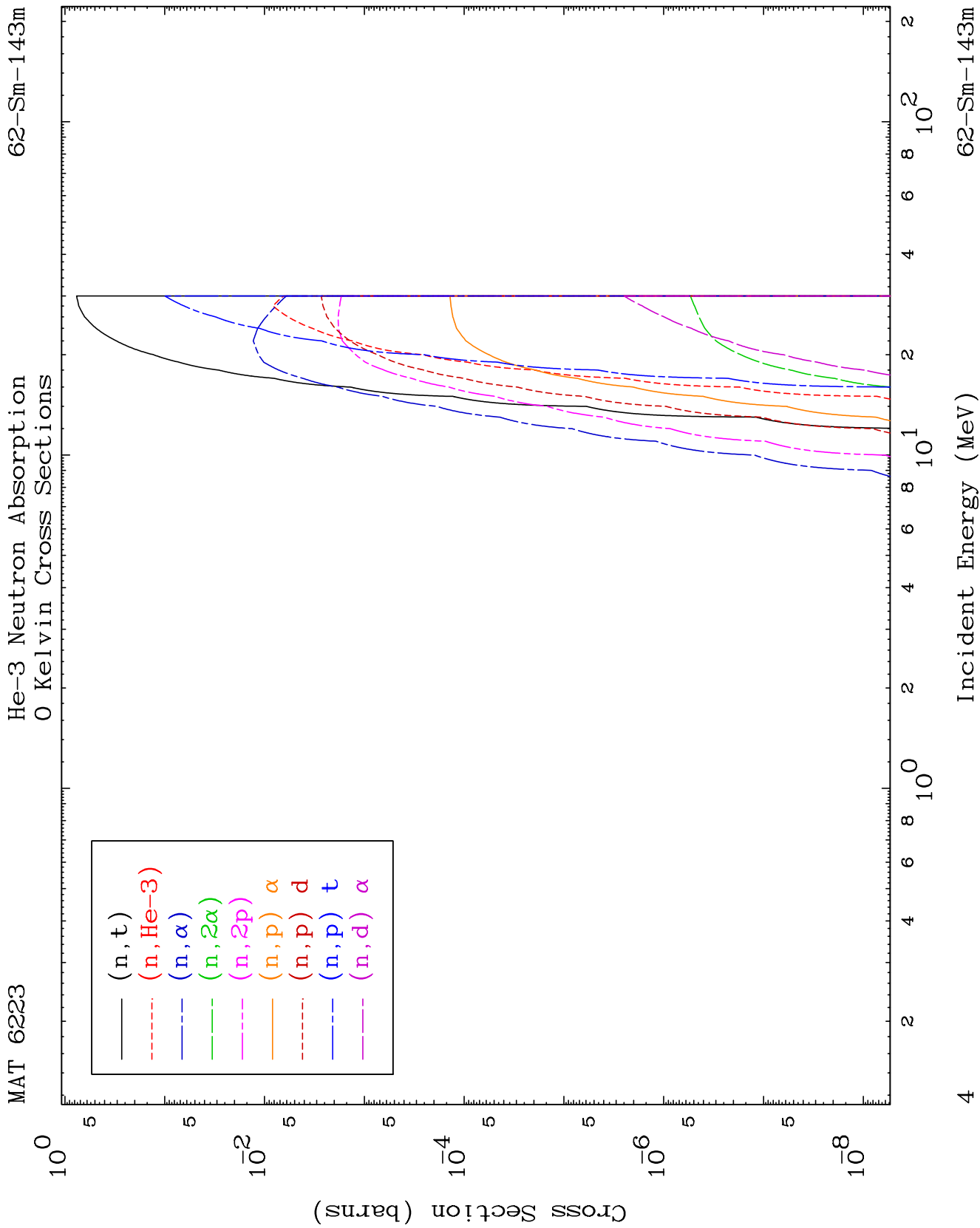


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He-3 Neutron Absorption
0 Kelvin Cross Sections

62-Sm-143m

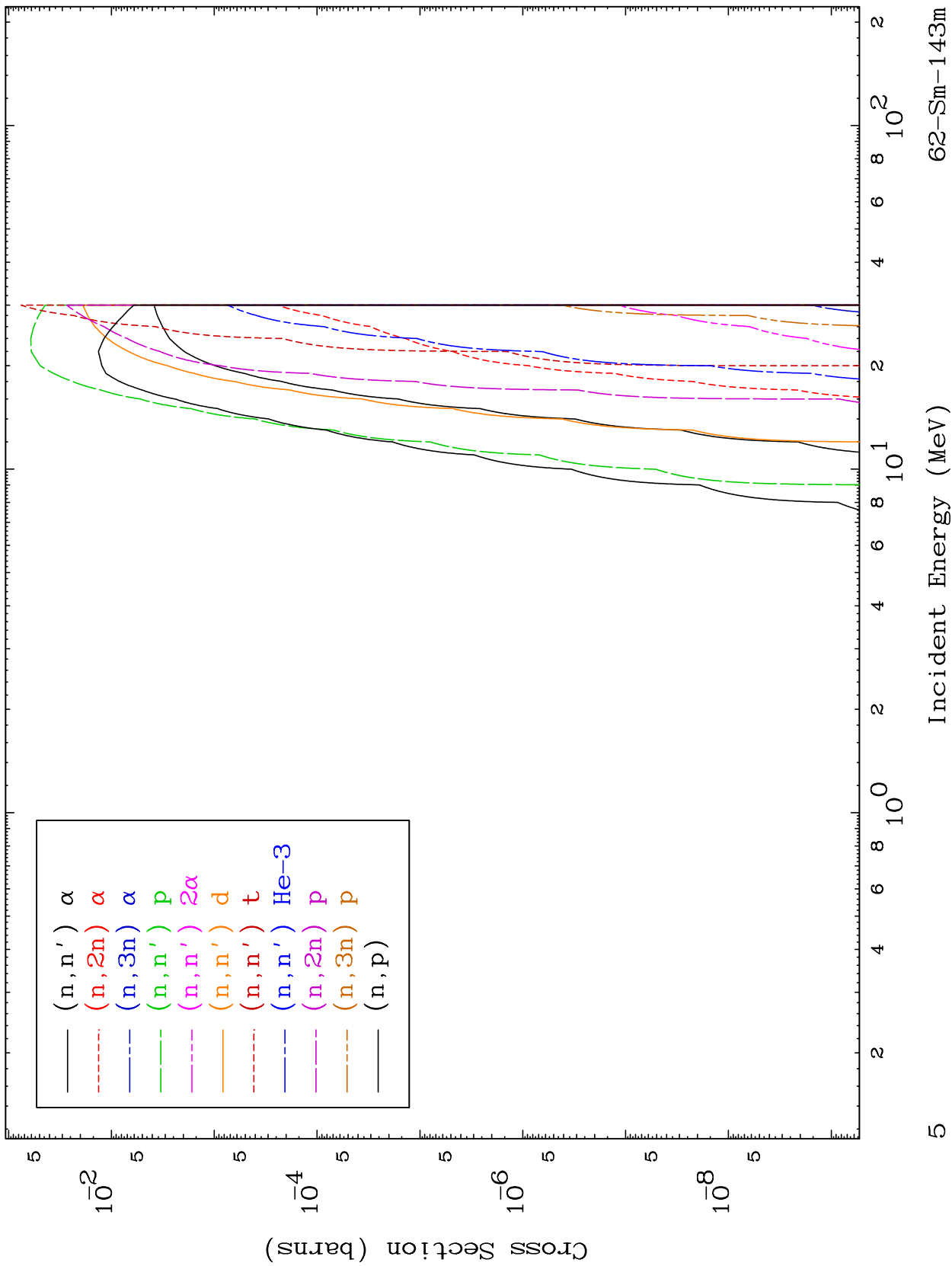




MAT 6223

He-3 Charged Particle
0 Kelvin Cross Sections

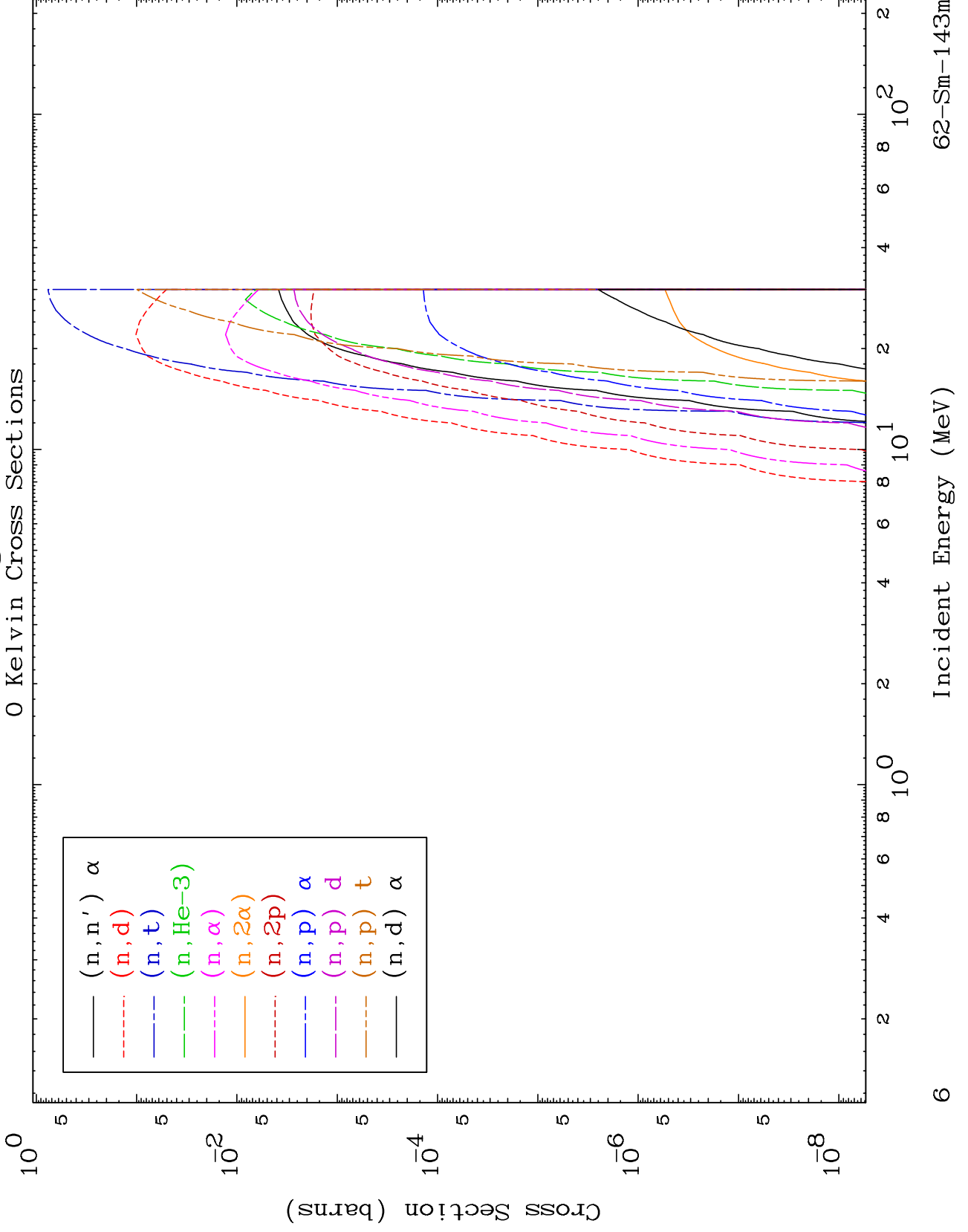
62-Sm-143m



MAT 6223

He-3 Charged Particle
0 Kelvin Cross Sections

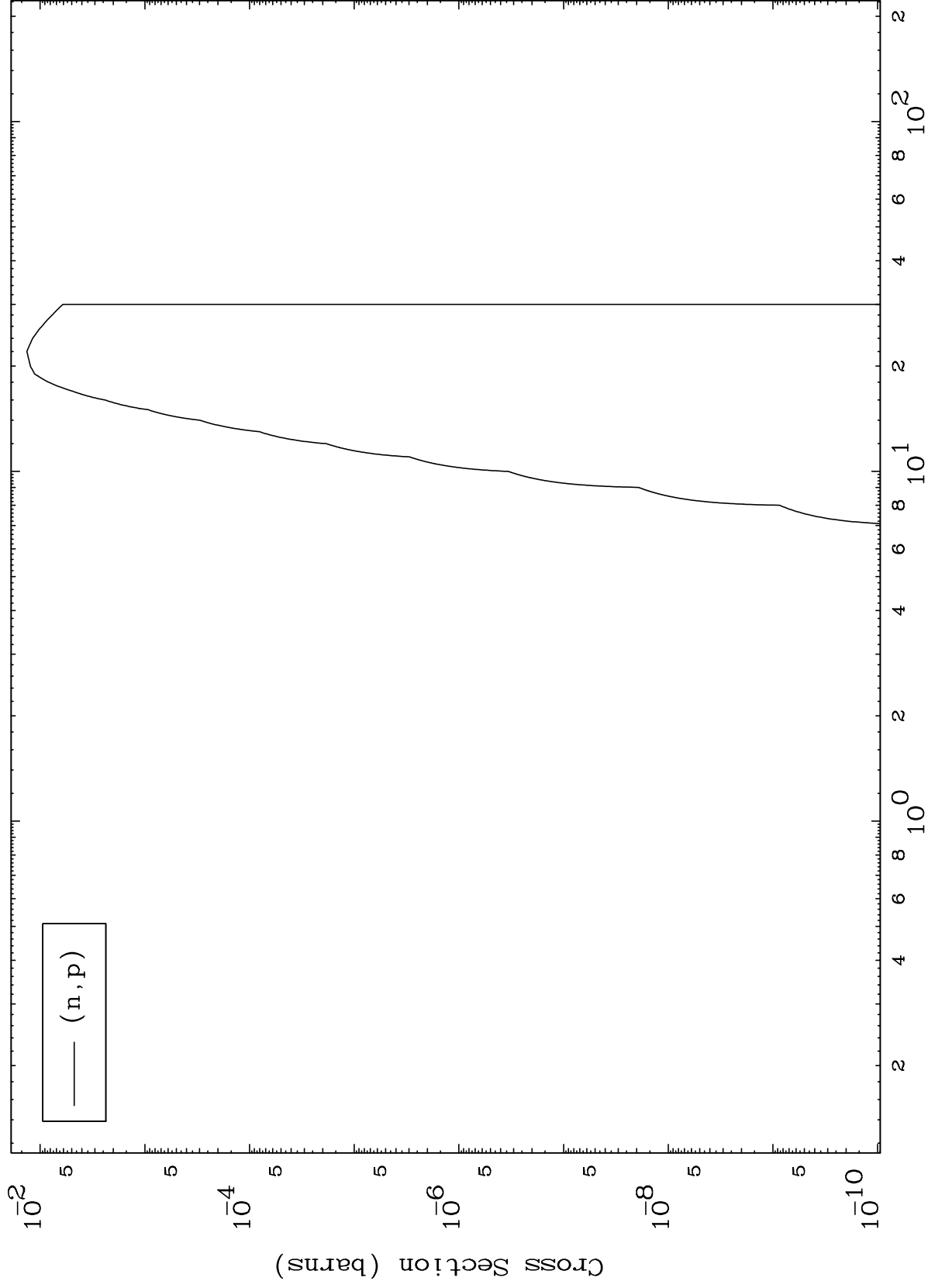
62-Sm-143m



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(He-3,p) Levels
0 Kelvin Cross Sections

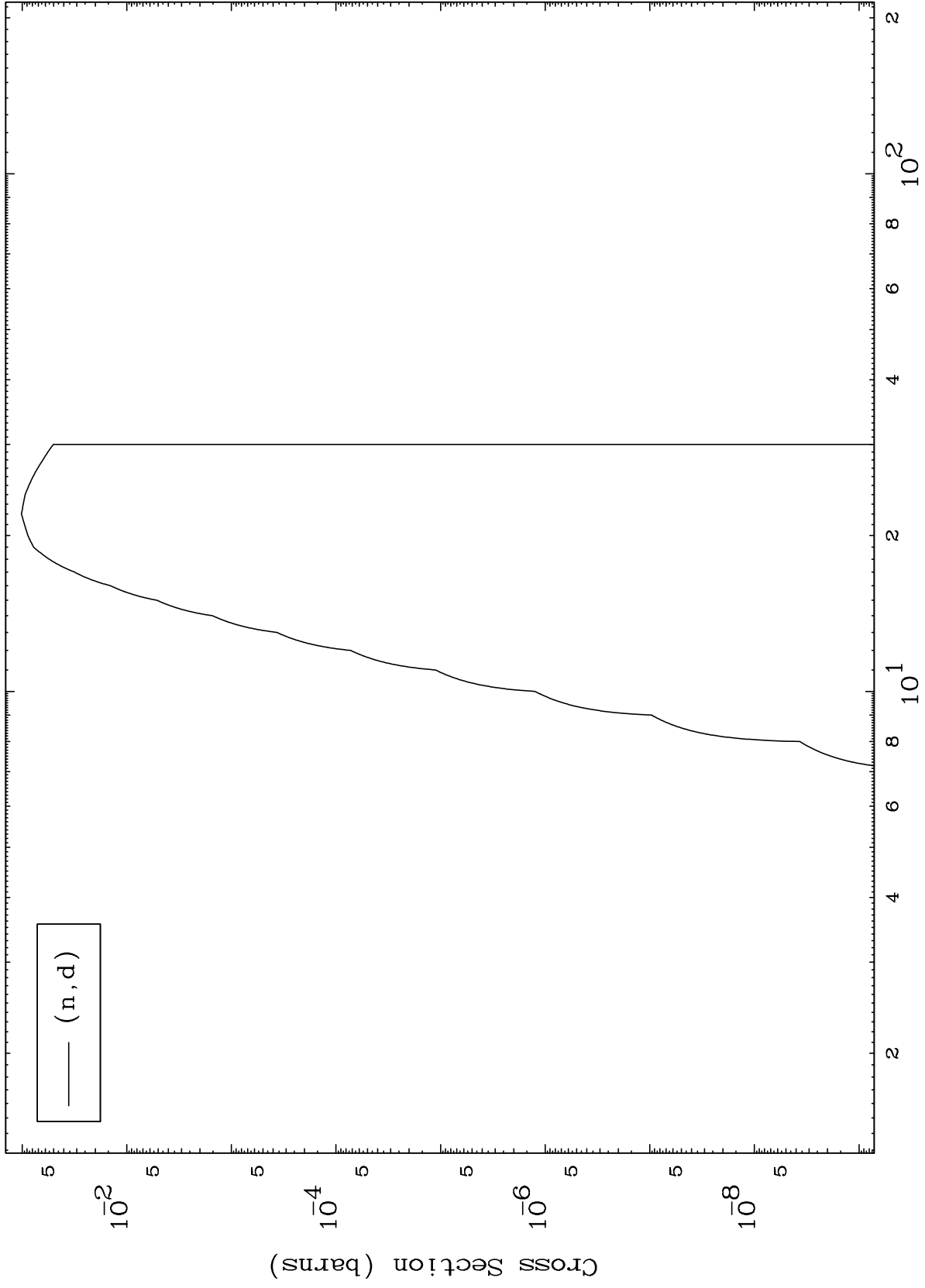
62-Sm-143m



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(He-3,d) Levels
0 Kelvin Cross Sections

62-Sm-143m



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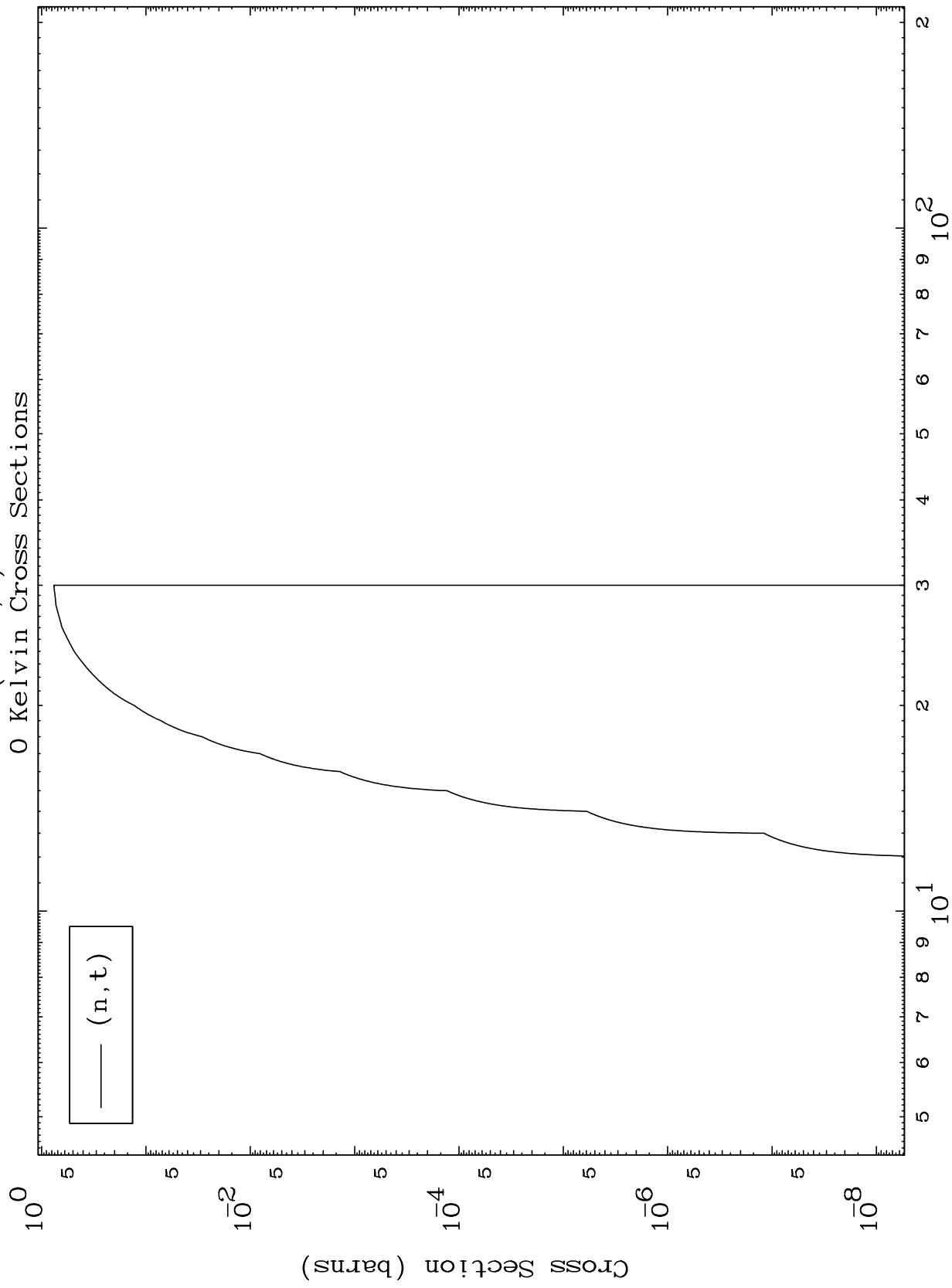
Incident Energy (MeV)

62-Sm-143m

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62-Sm-143m

(He-3,t) Levels
0 Kelvin Cross Sections



9

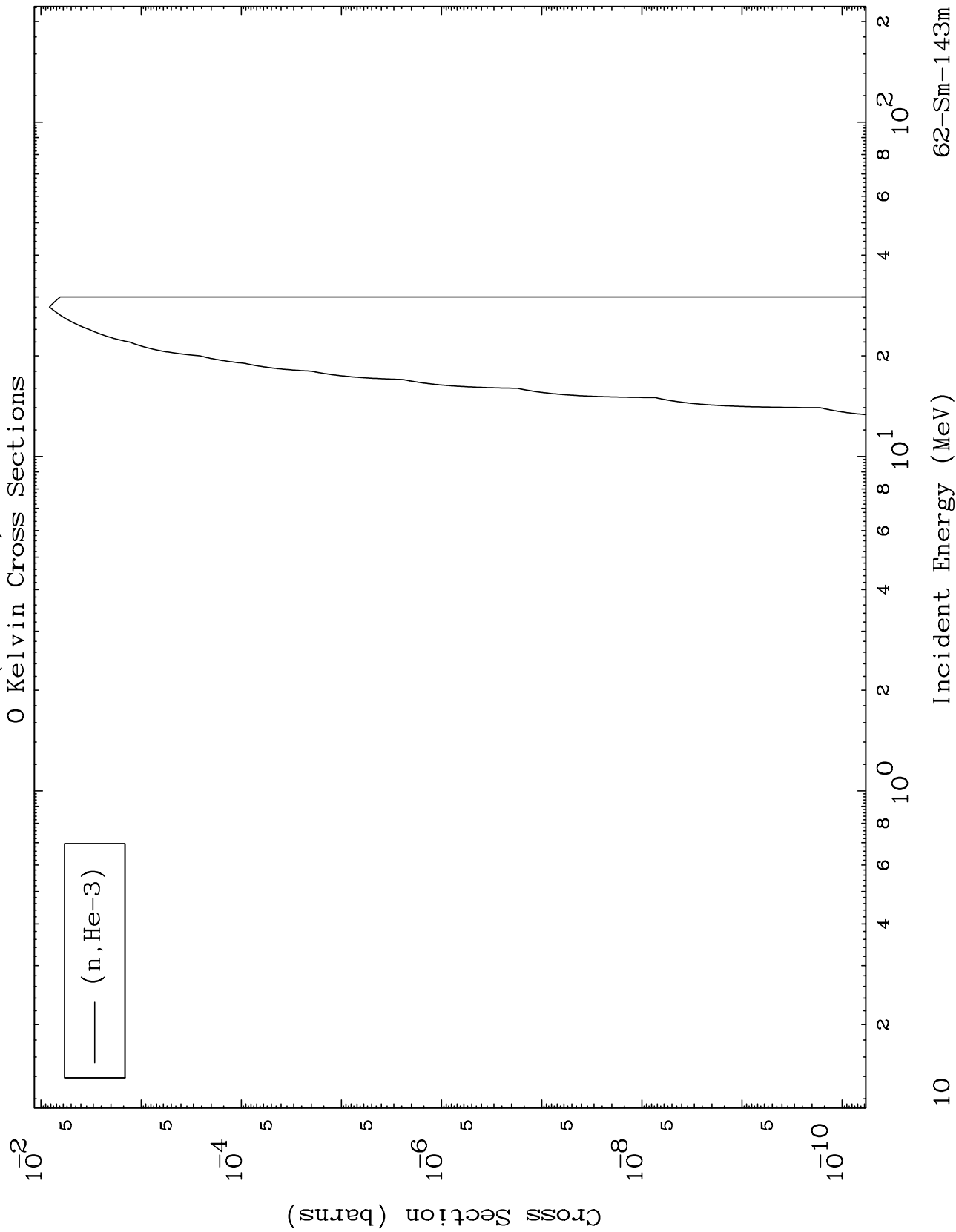
Incident Energy (MeV)

62-Sm-143m

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(He-3, He3) Levels

62-Sm-143m



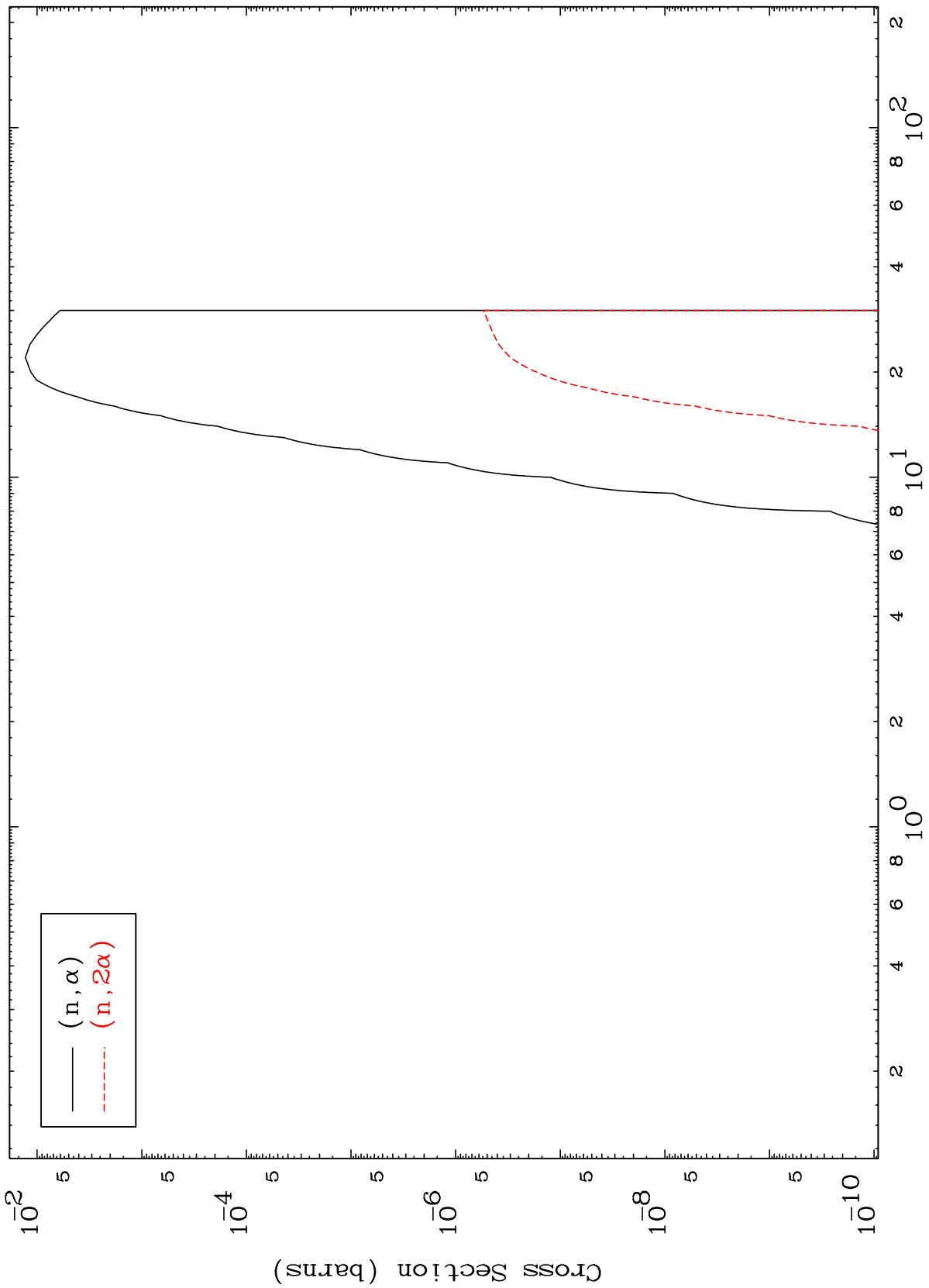
10

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(He-3, α) Levels

62-Sm-143m

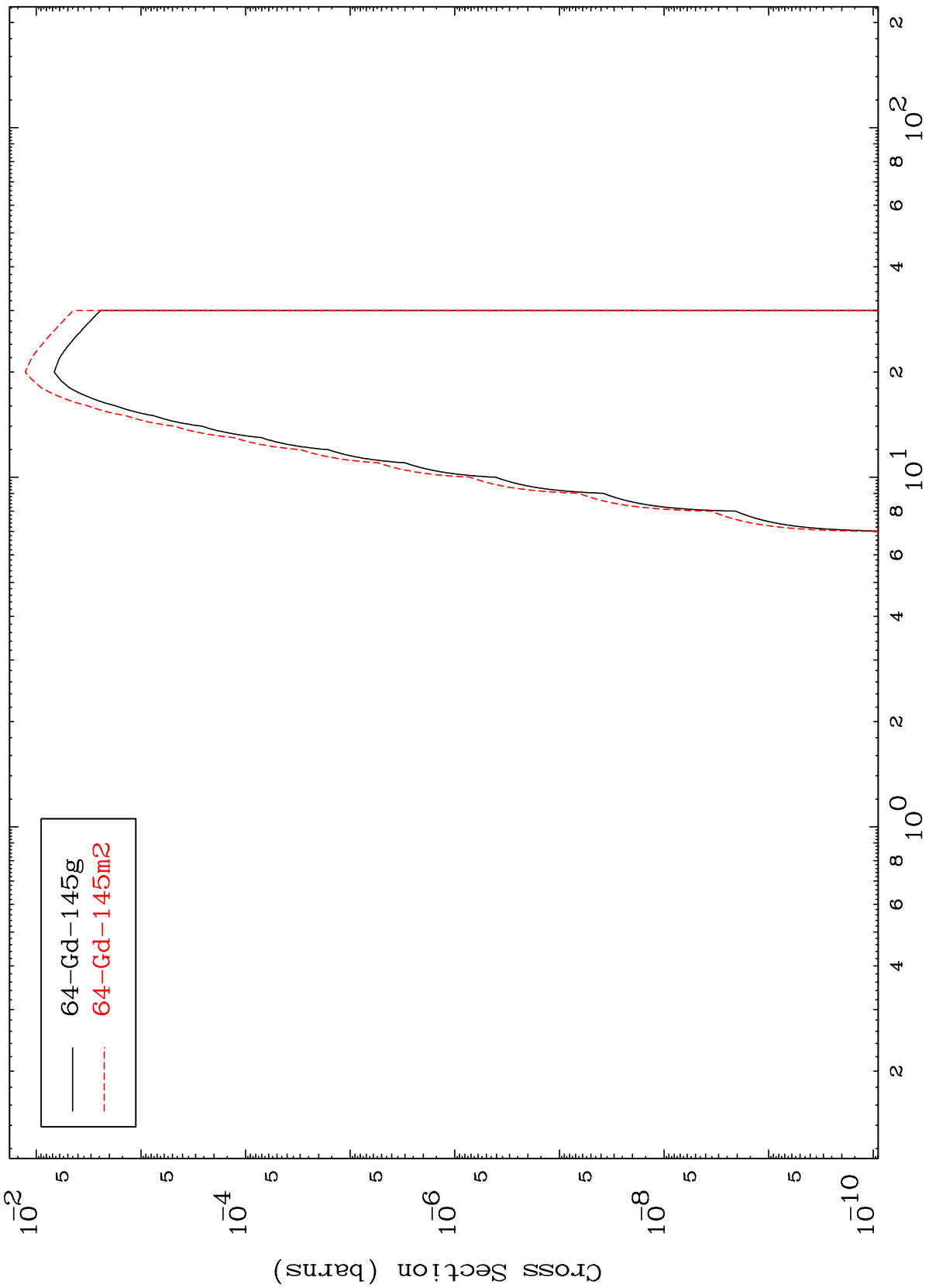
0 Kelvin Cross Sections



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62-Sm-143m

Inelastic
Radionuclide Production Cross Section



64-Gd-145g
64-Gd-145m2

62-Sm-143m

Incident Energy (MeV)

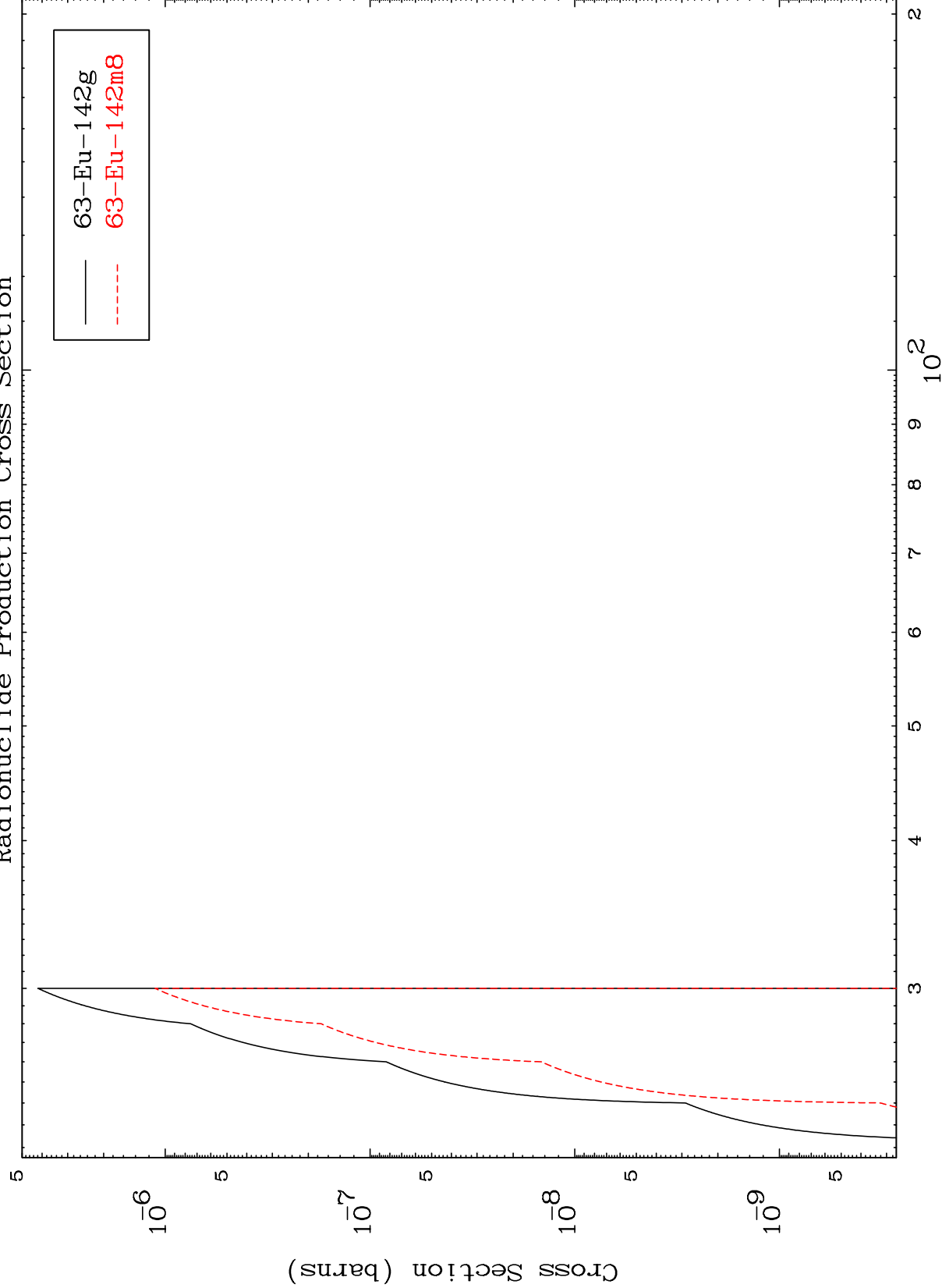
12

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(n,2n) d

62-Sm-143m

Radionuclide Production Cross Section



63-Eu-142g
63-Eu-142m8

13

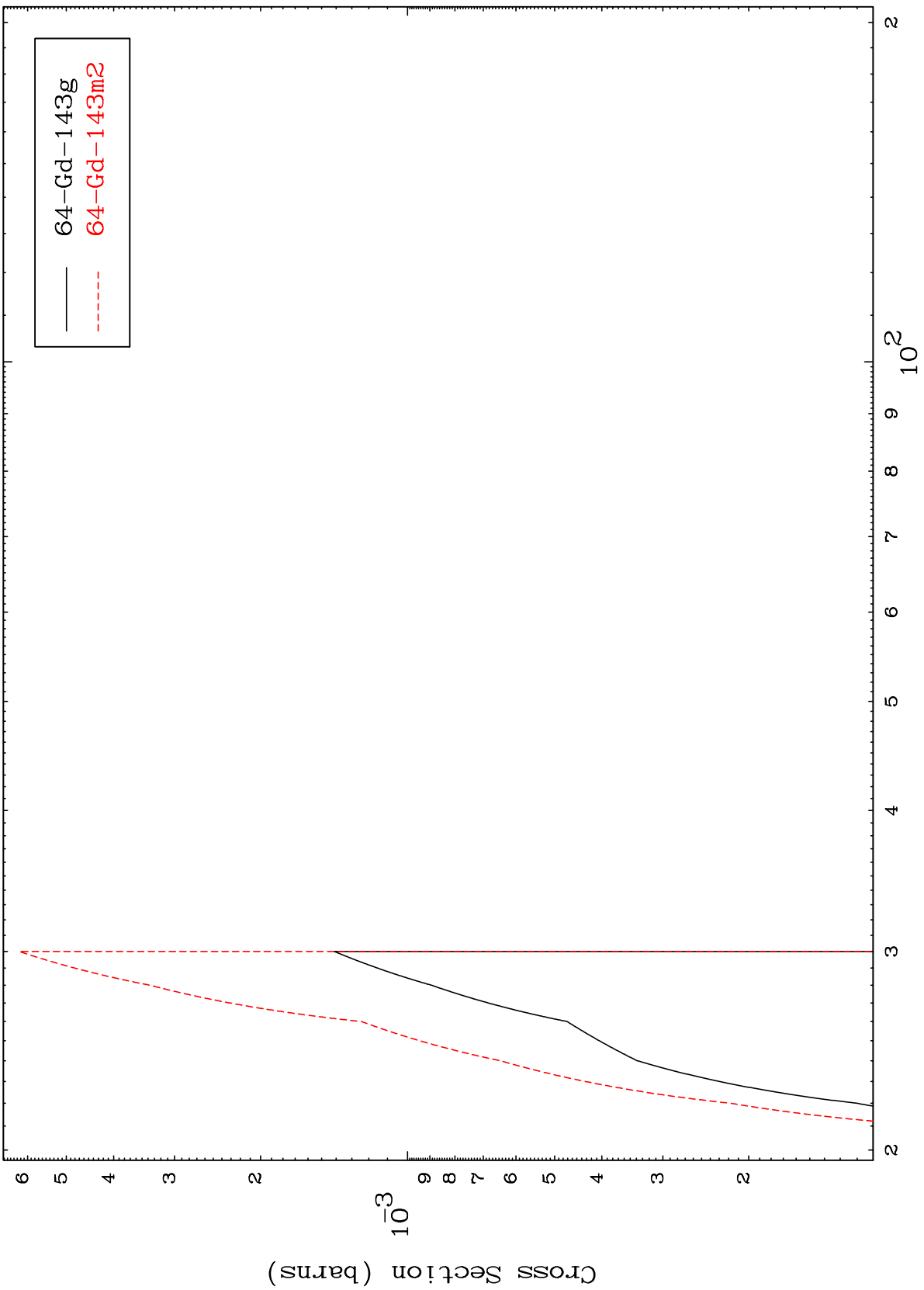
Incident Energy (MeV)

62-Sm-143m

MAT 6223

62-Sm-143m

(n,3n)
Radionuclide Production Cross Section



64-Gd-143g
64-Gd-143m2

62-Sm-143m

Incident Energy (MeV)

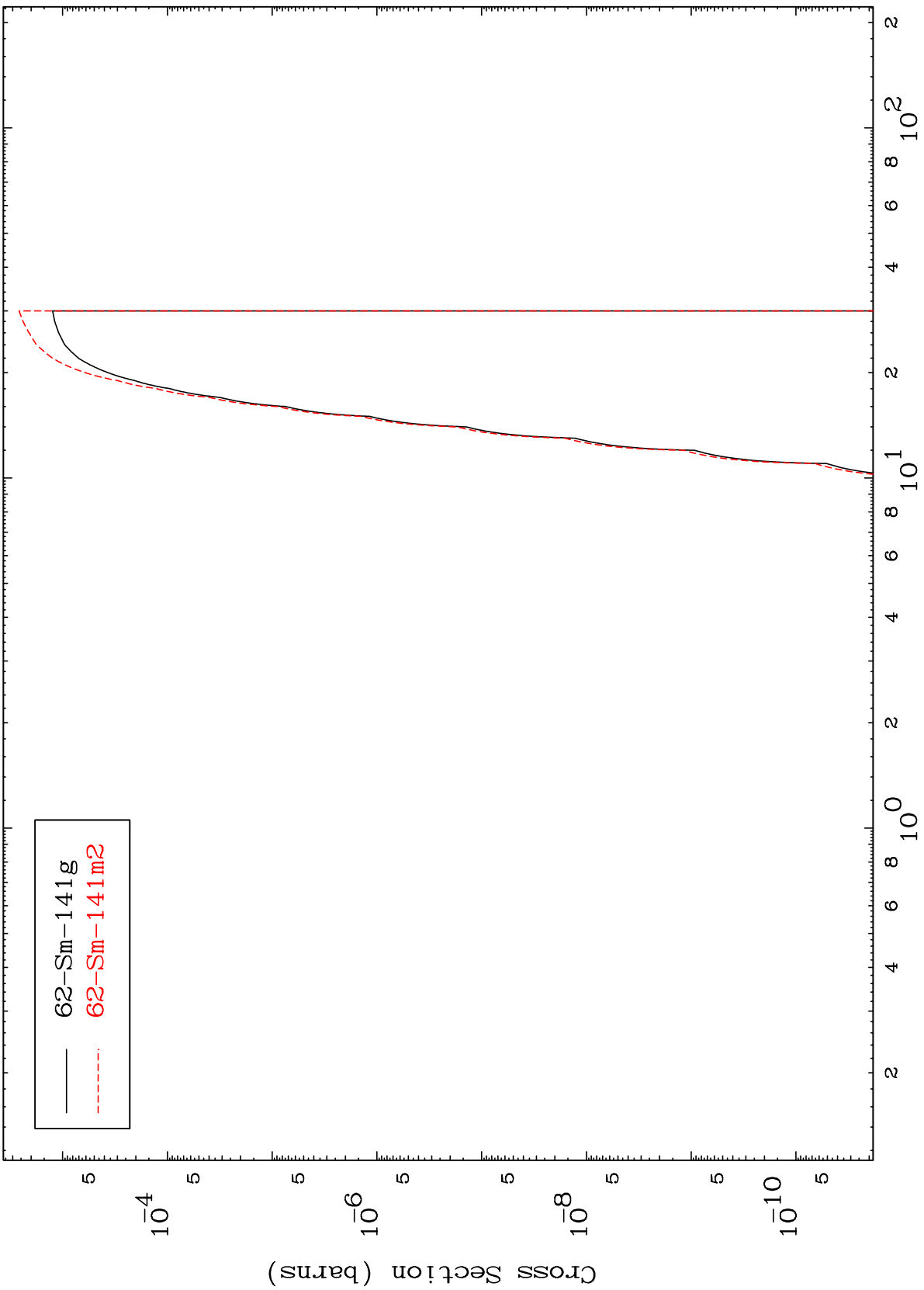
14

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$(n, n') \alpha$

62-Sm-143m

Radionuclide Production Cross Section

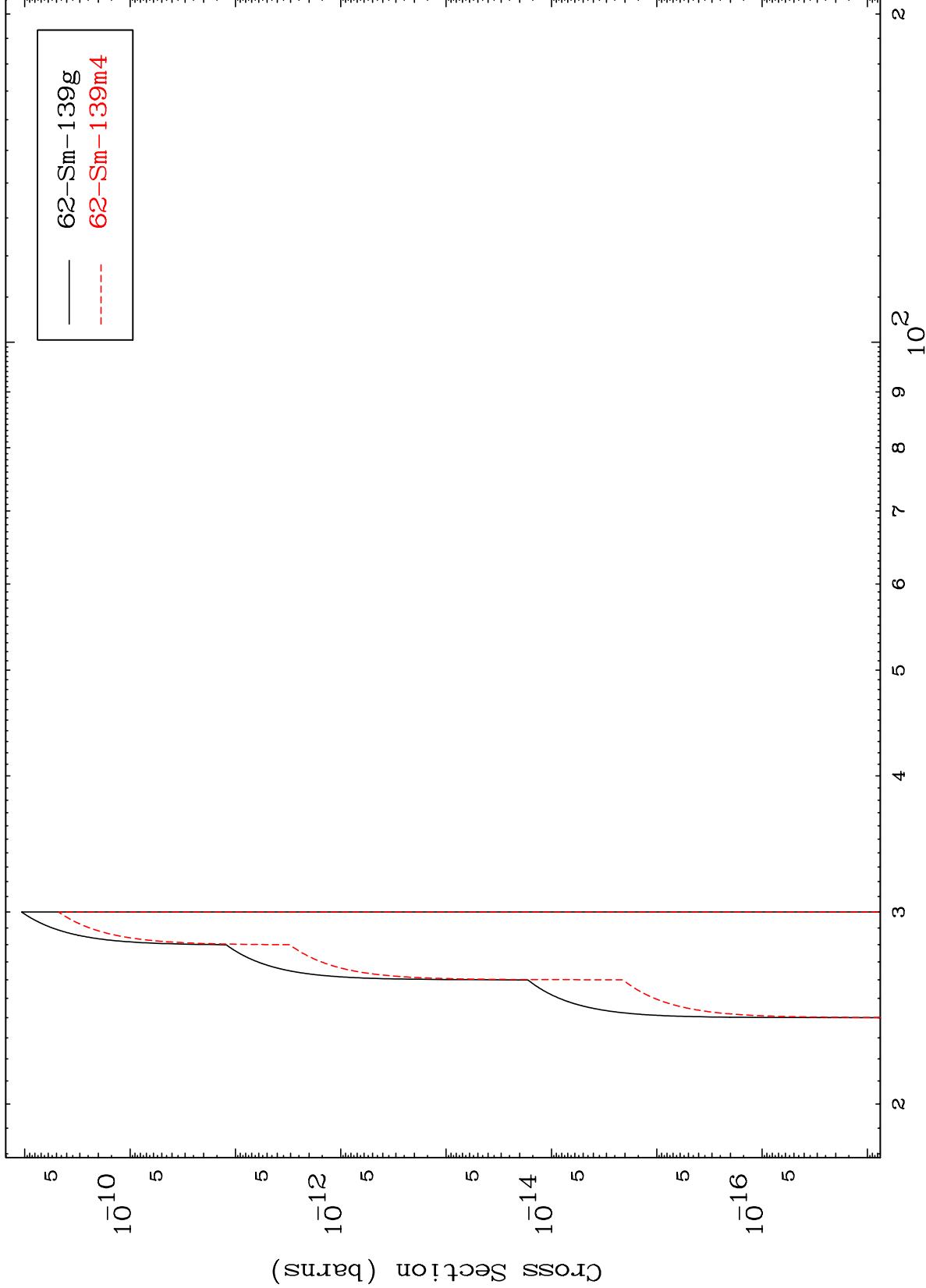


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$(n,3n) \alpha$

62-Sm-143m

Radionuclide Production Cross Section



16

Incident Energy (MeV)

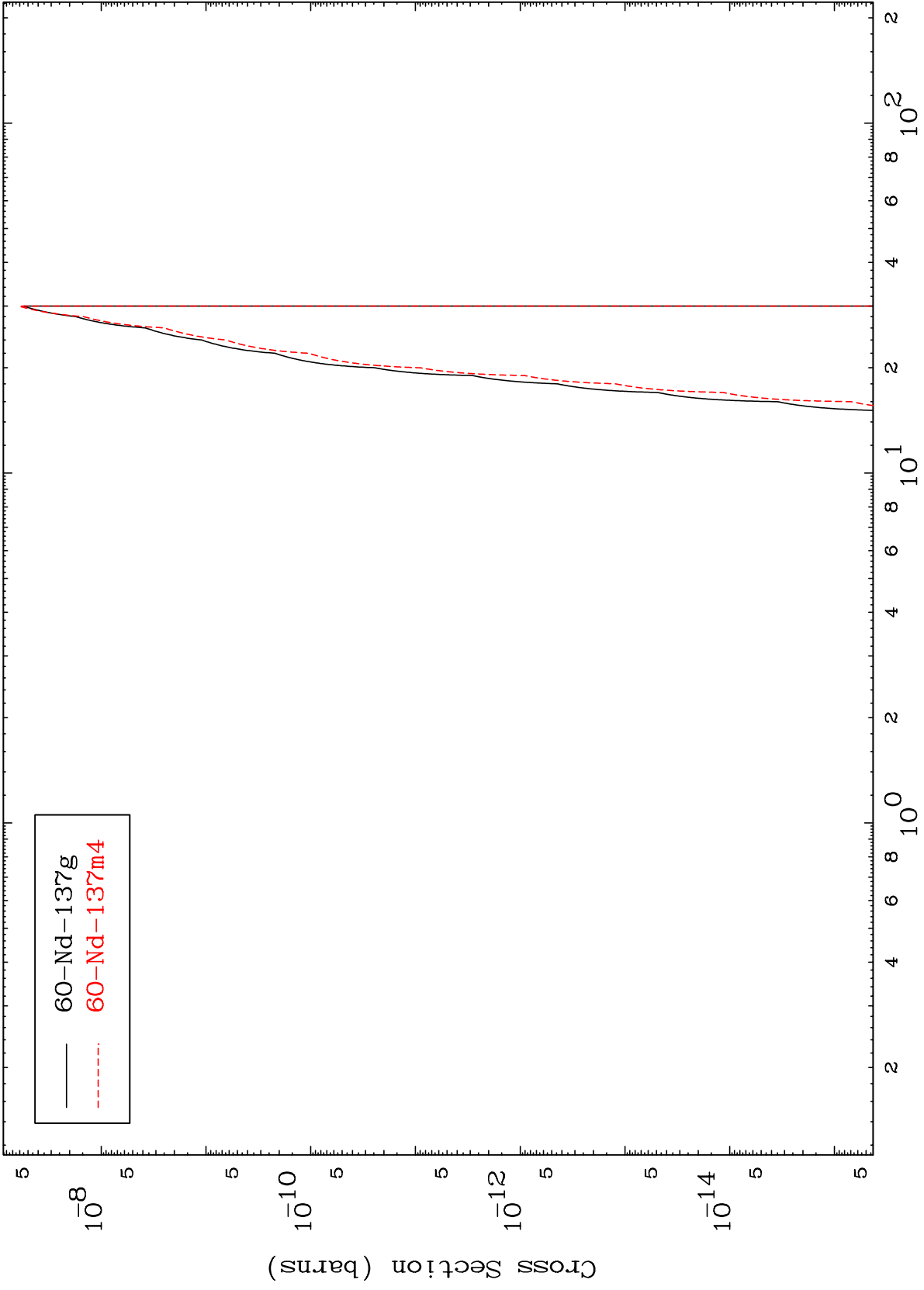
62-Sm-143m

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(n,n') 2 α

62-Sm-143m

Radionuclide Production Cross Section

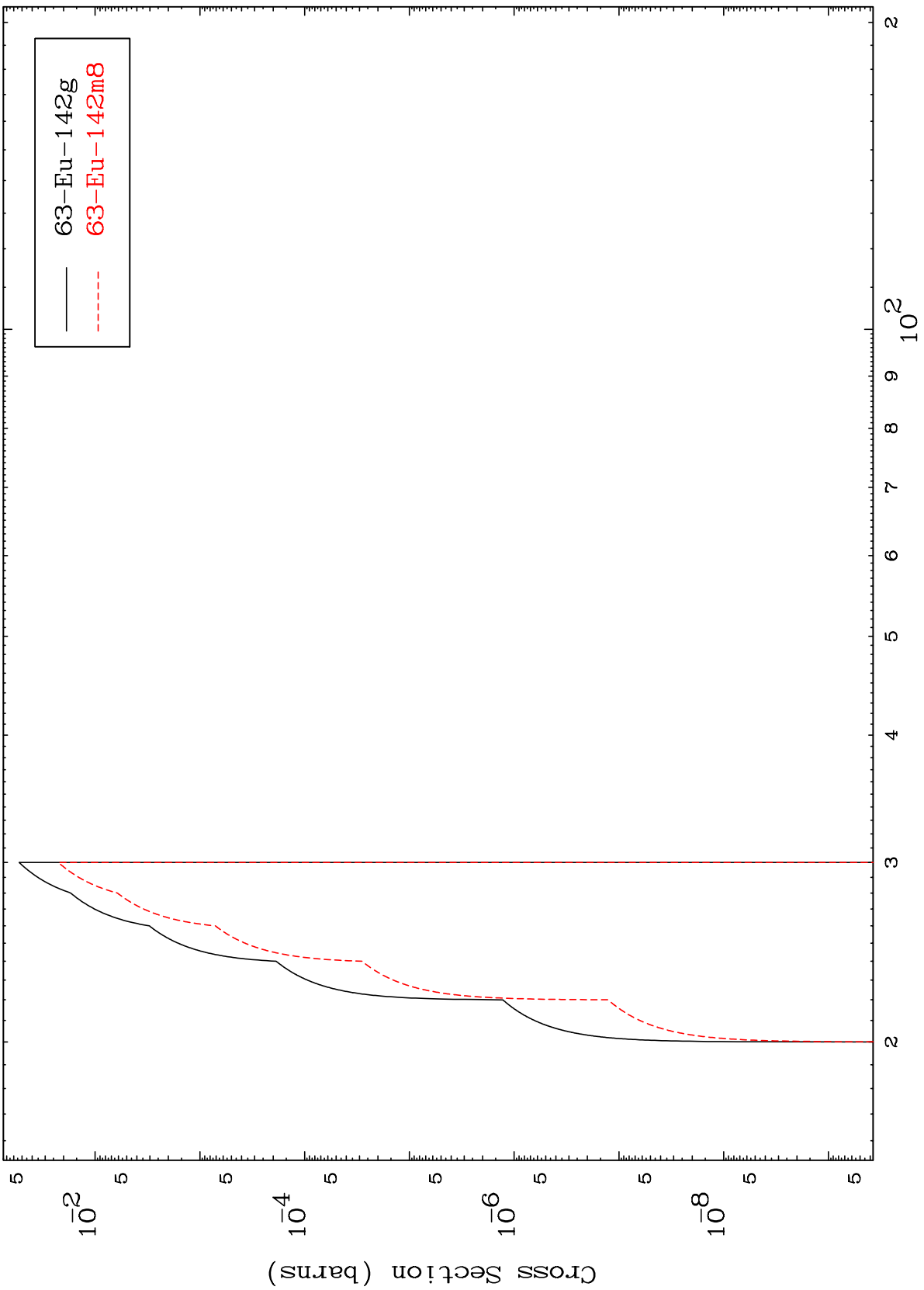


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(n,n') t

62-Sm-143m

Radionuclide Production Cross Section



18

Incident Energy (MeV)

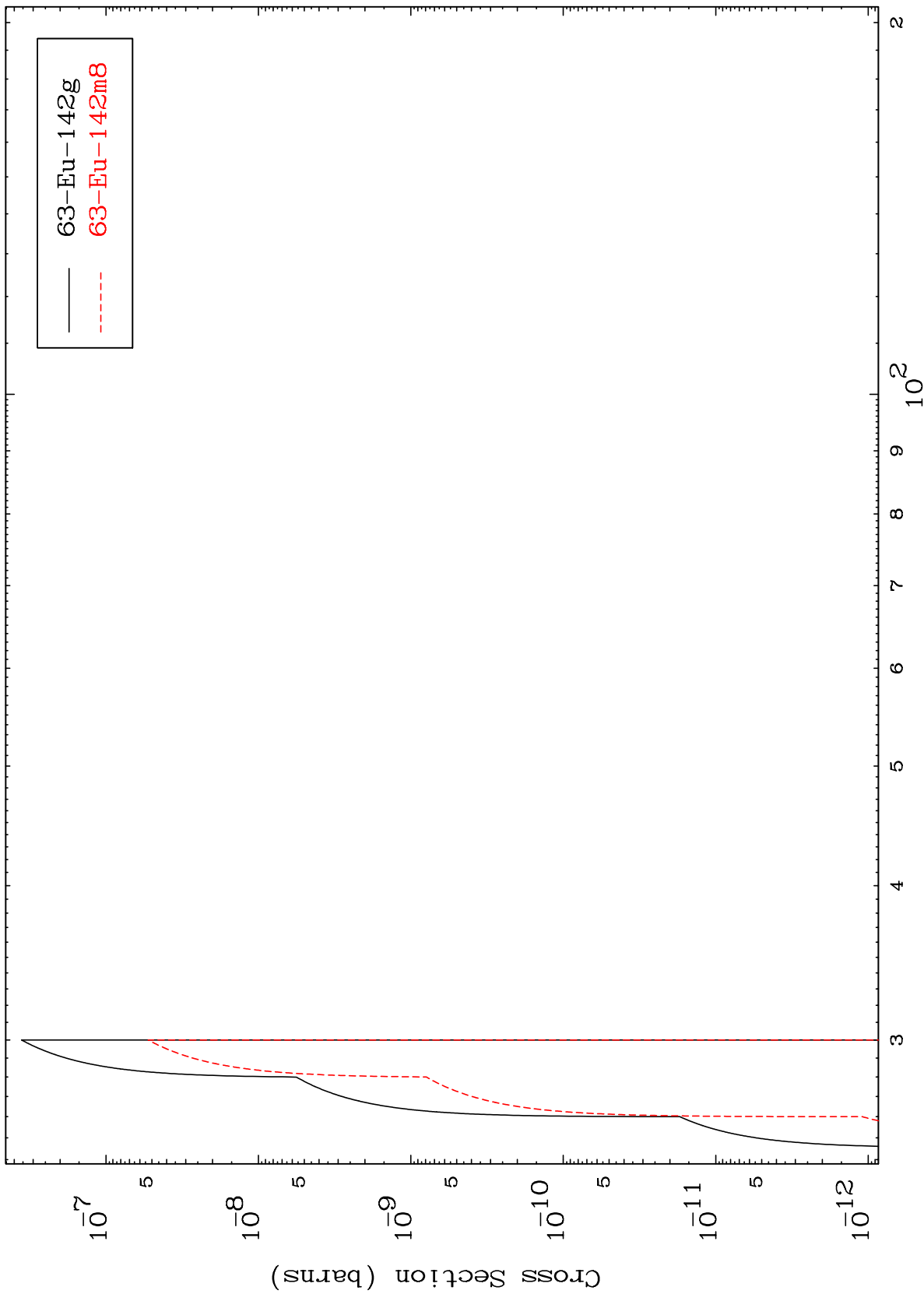
62-Sm-143m

MAT 6223

(n,3n) p

62-Sm-143m

Radionuclide Production Cross Section

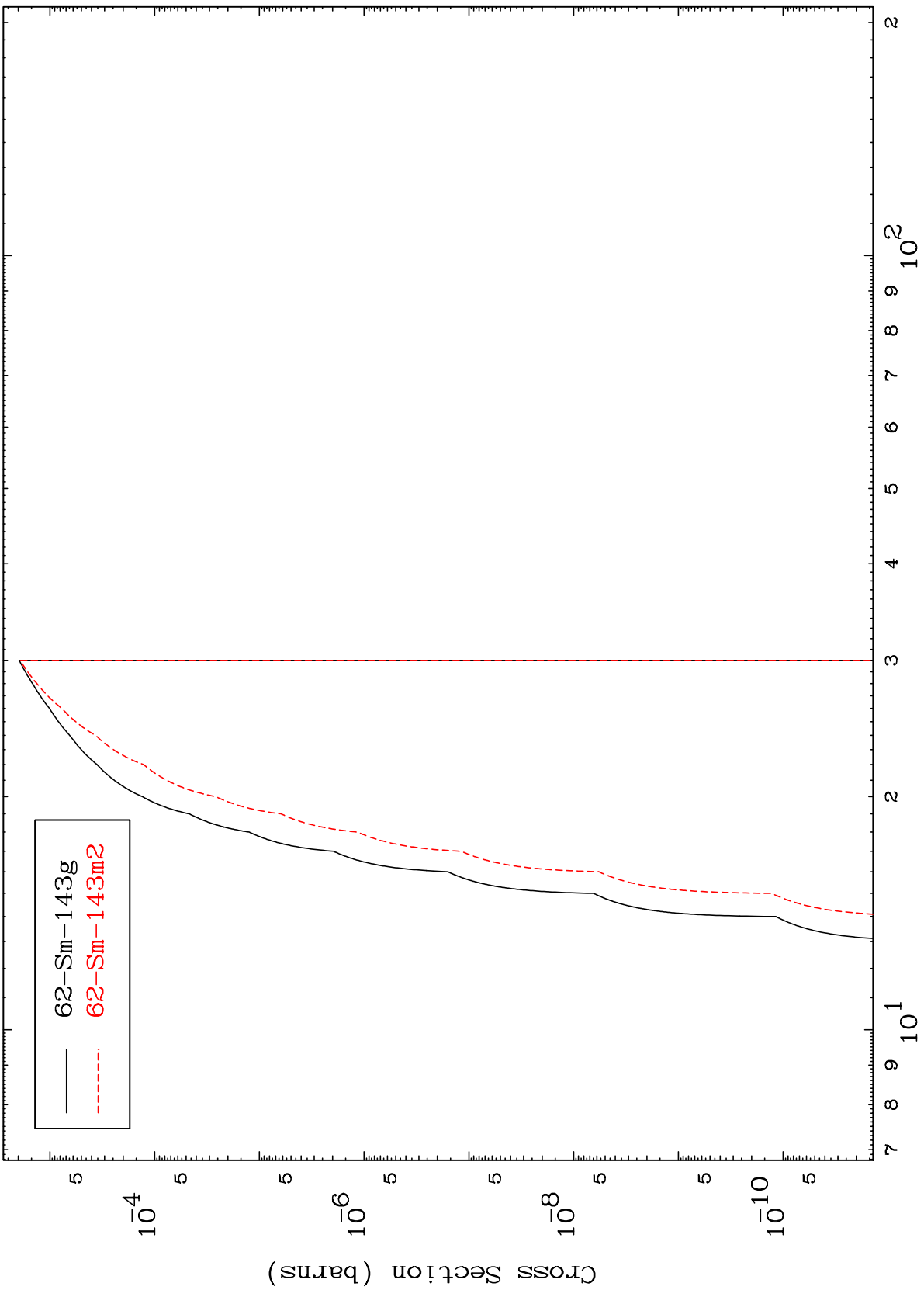


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62-Sm-143m

(n,2n) p

Radionuclide Production Cross Section



20

Incident Energy (MeV)

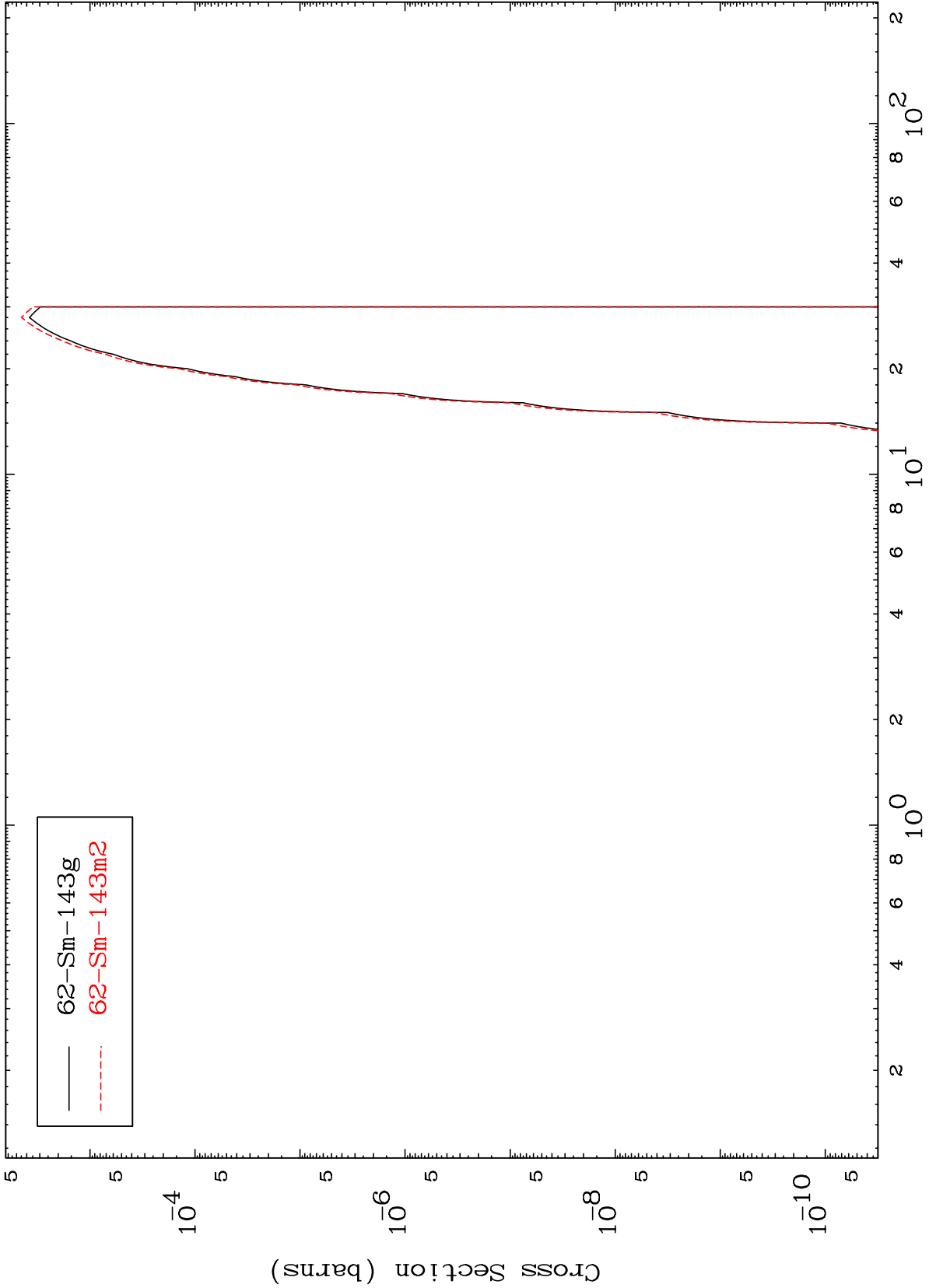
62-Sm-143m

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(n,He-3)

62-Sm-143m

Radionuclide Production Cross Section

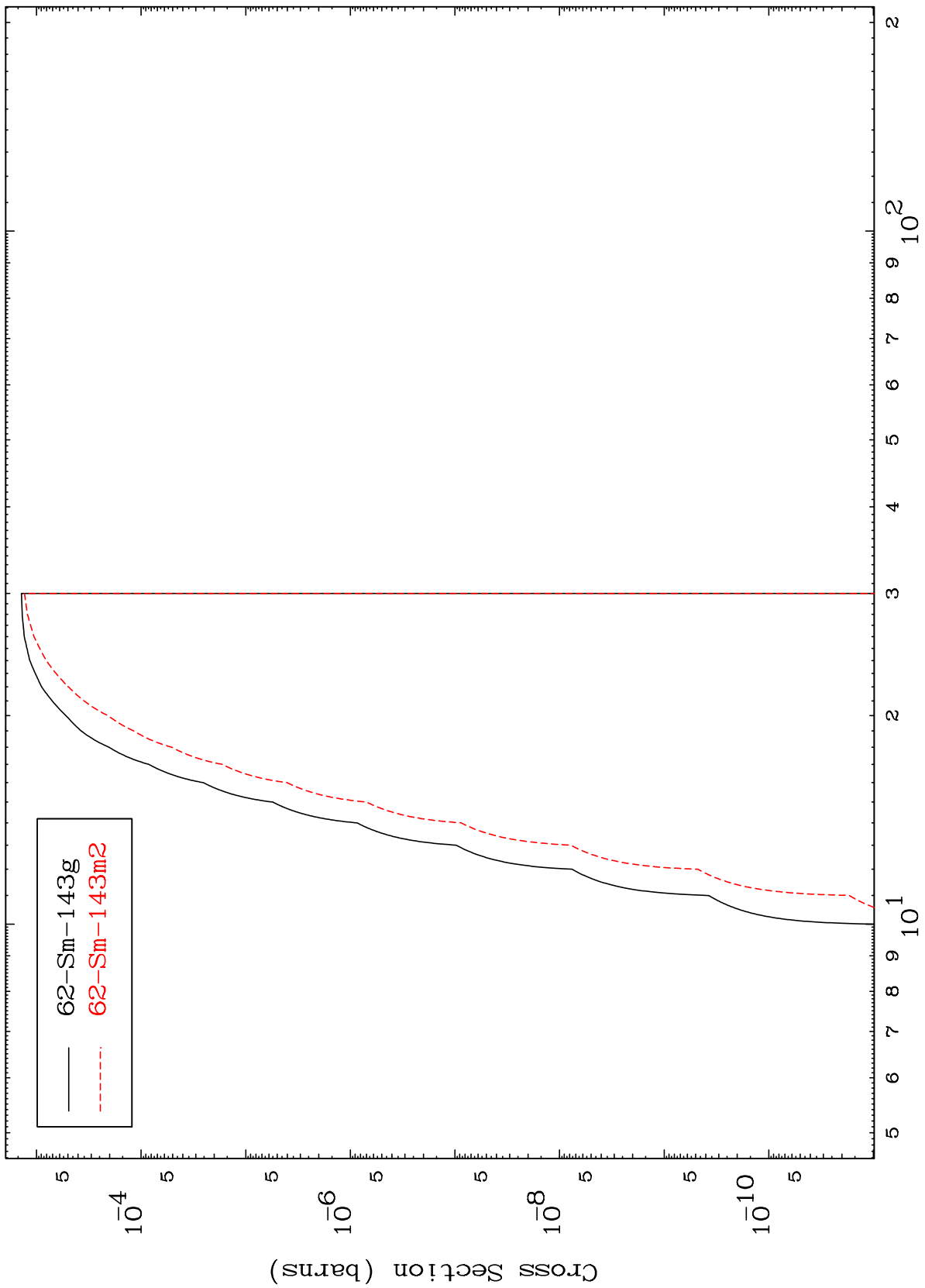


MAT 6223

(n,p) d

62-Sm-143m

Radionuclide Production Cross Section



23

Incident Energy (MeV)

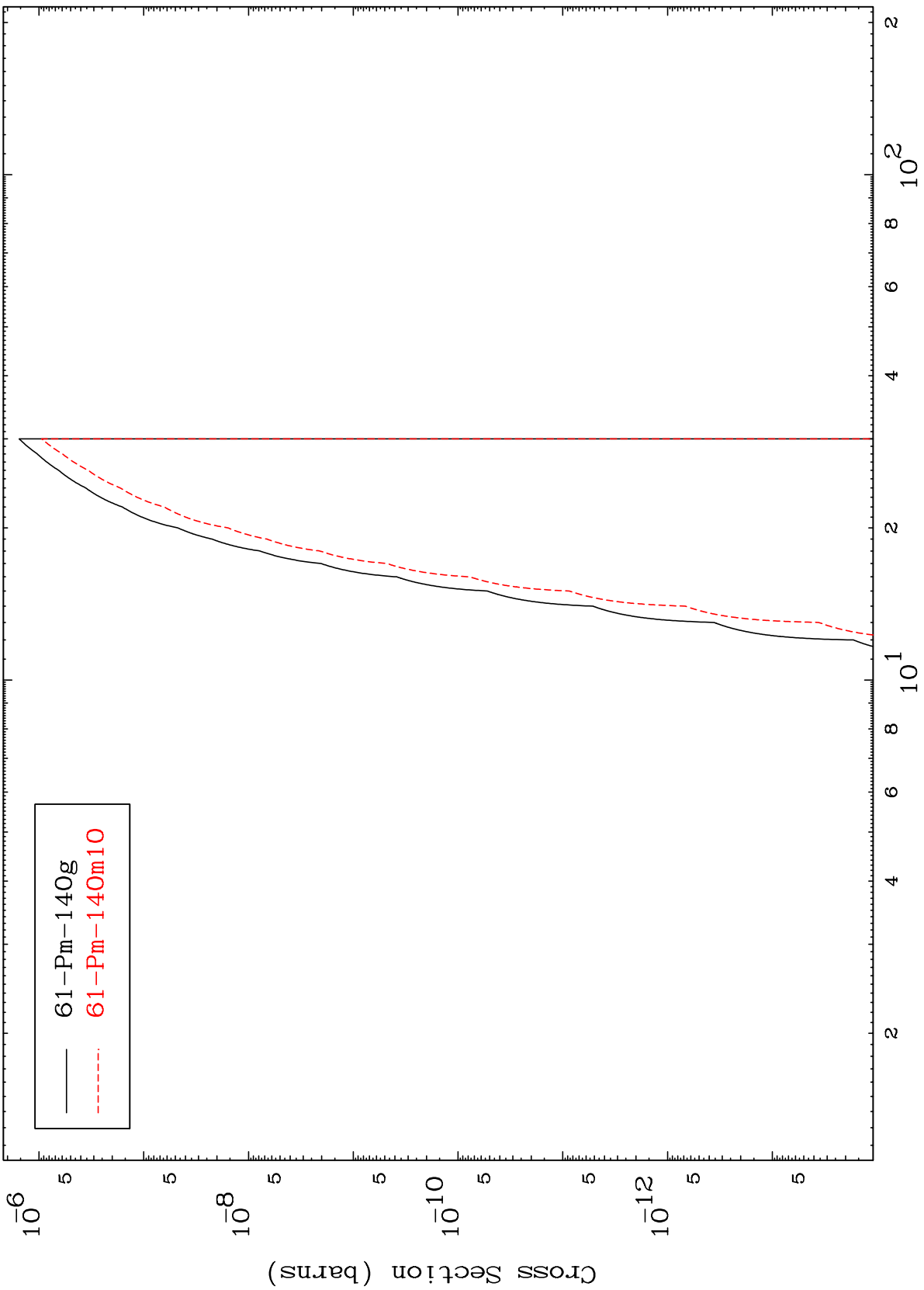
62-Sm-143m

MAT 6223

(n,d) α

62-Sm-143m

Radionuclide Production Cross Section



24

Incident Energy (MeV)

62-Sm-143m