

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

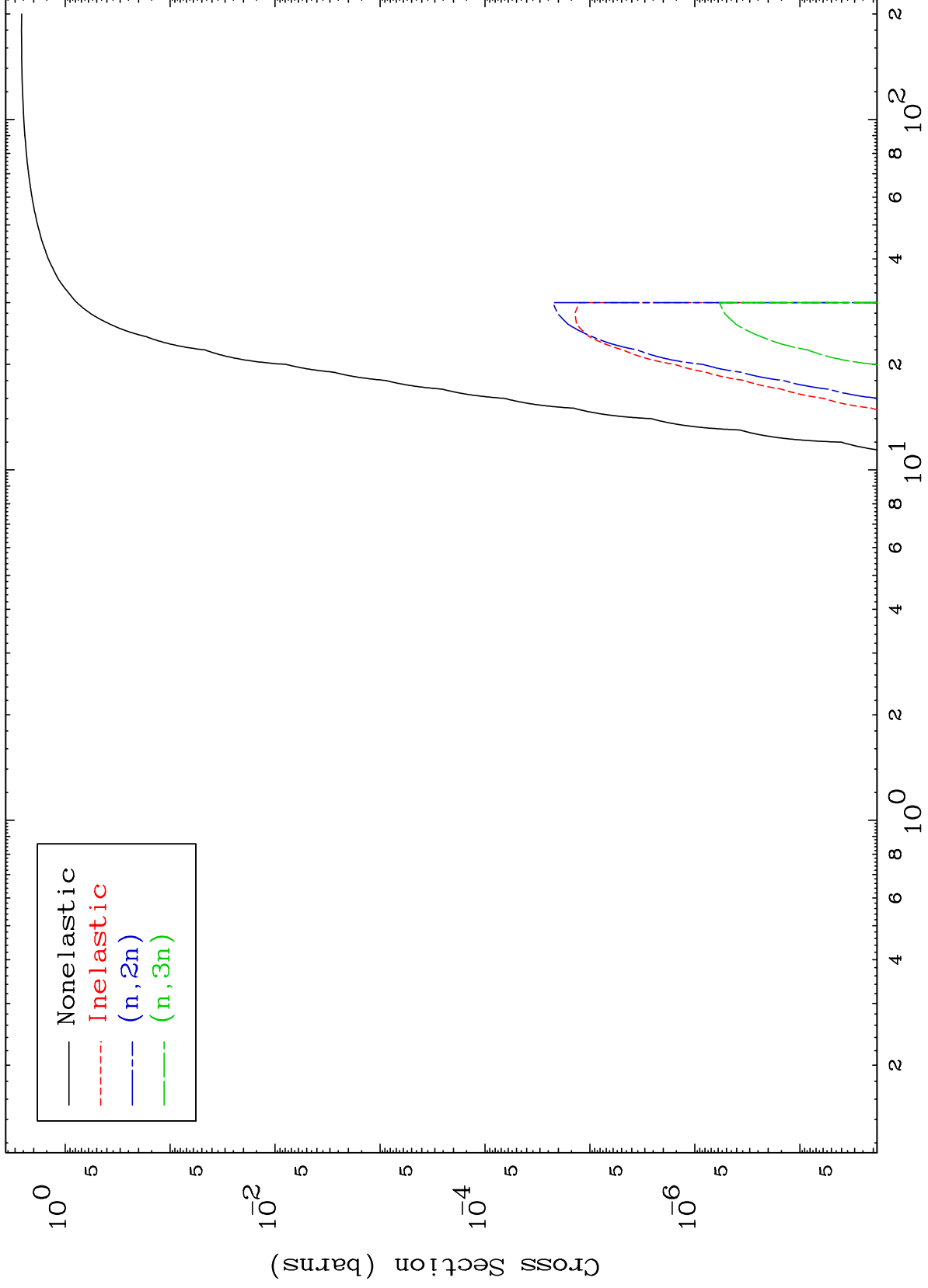
Press Mouse Button to Start

MAT 9231

He-3 Major

0 Kelvin Cross Sections

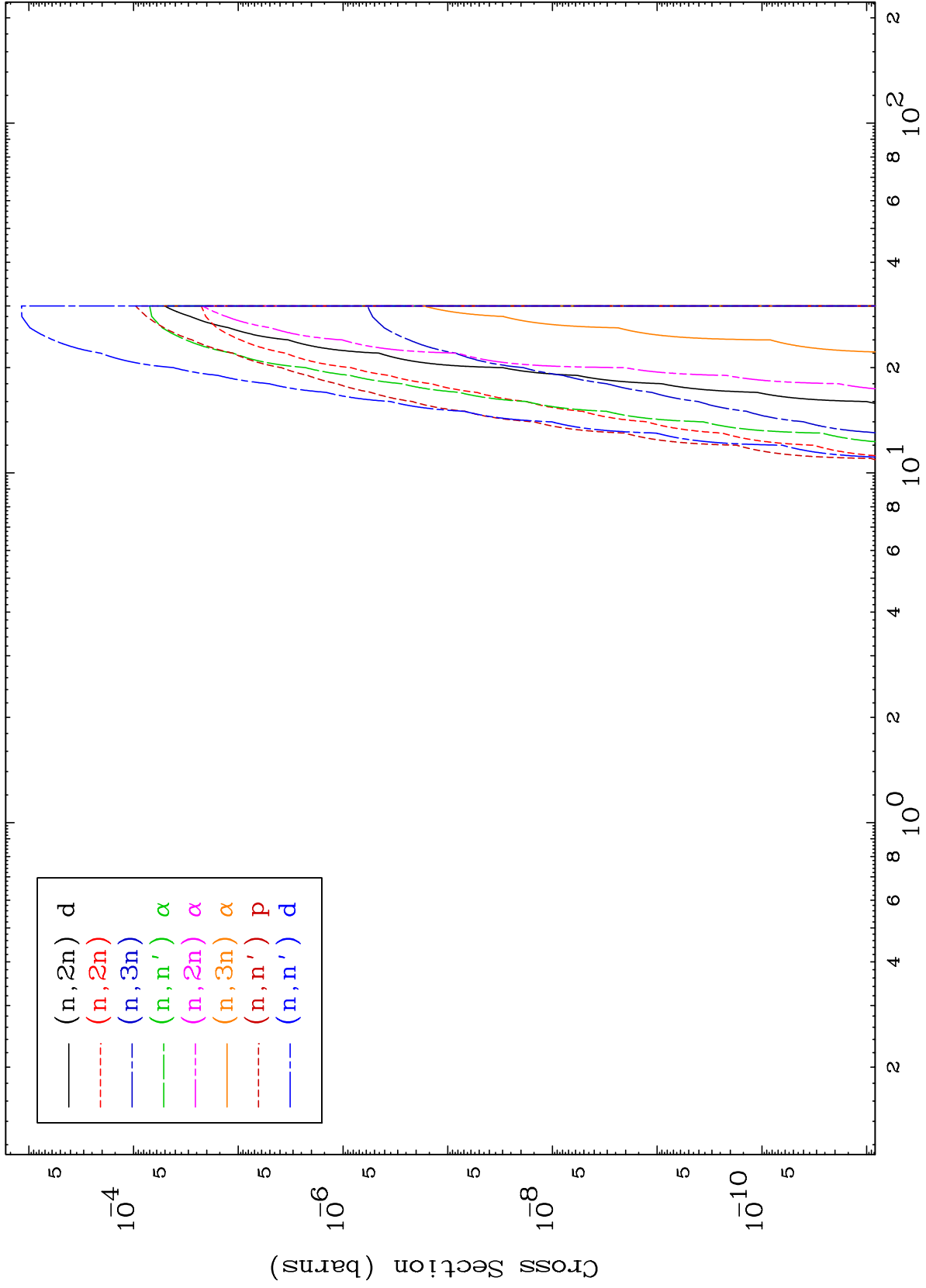
92-U -236



MAT 9231

He-3 Neutron Absorption
0 Kelvin Cross Sections

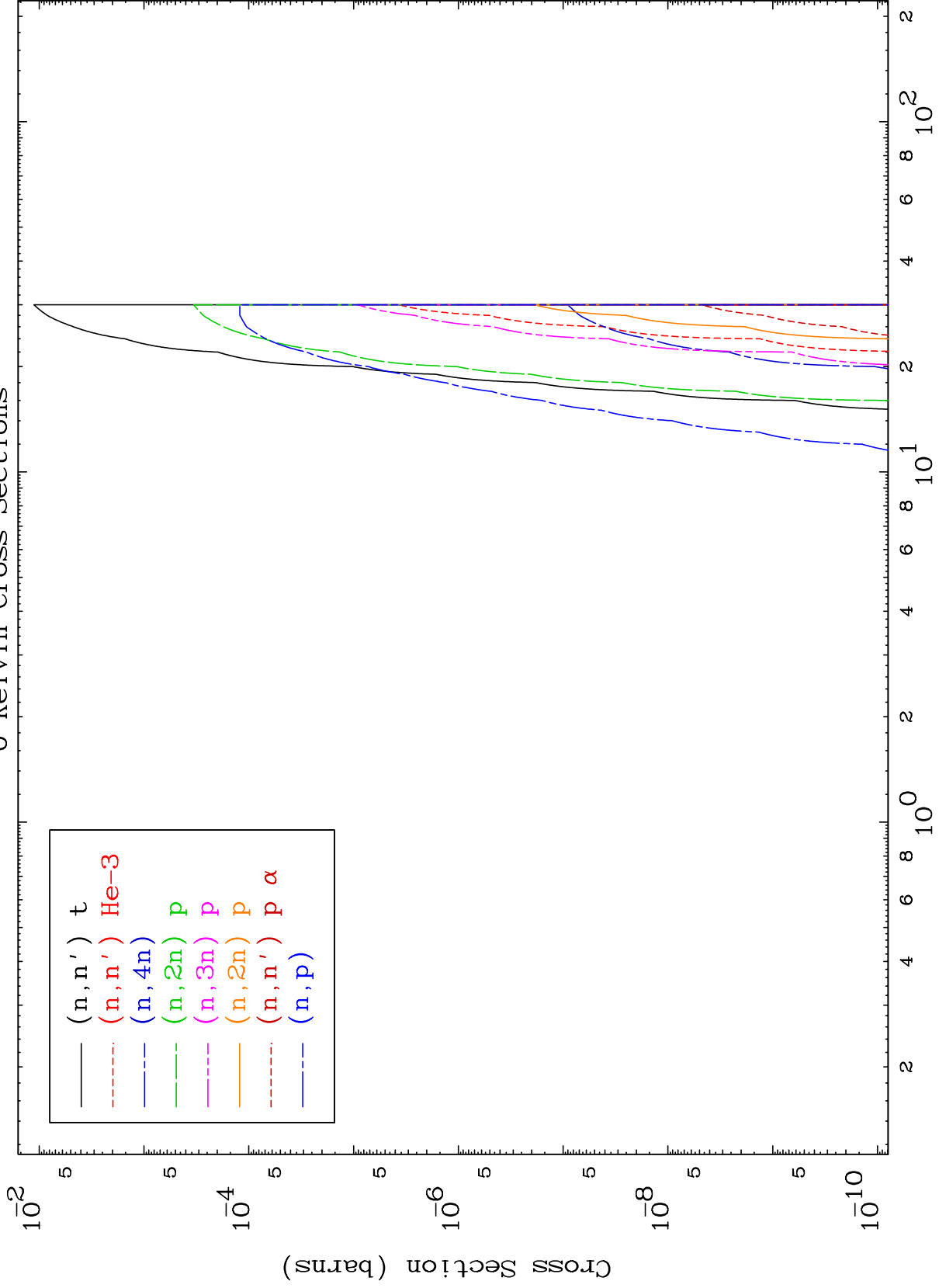
92-U -236



MAT 9231

He-3 Neutron Absorption
0 Kelvin Cross Sections

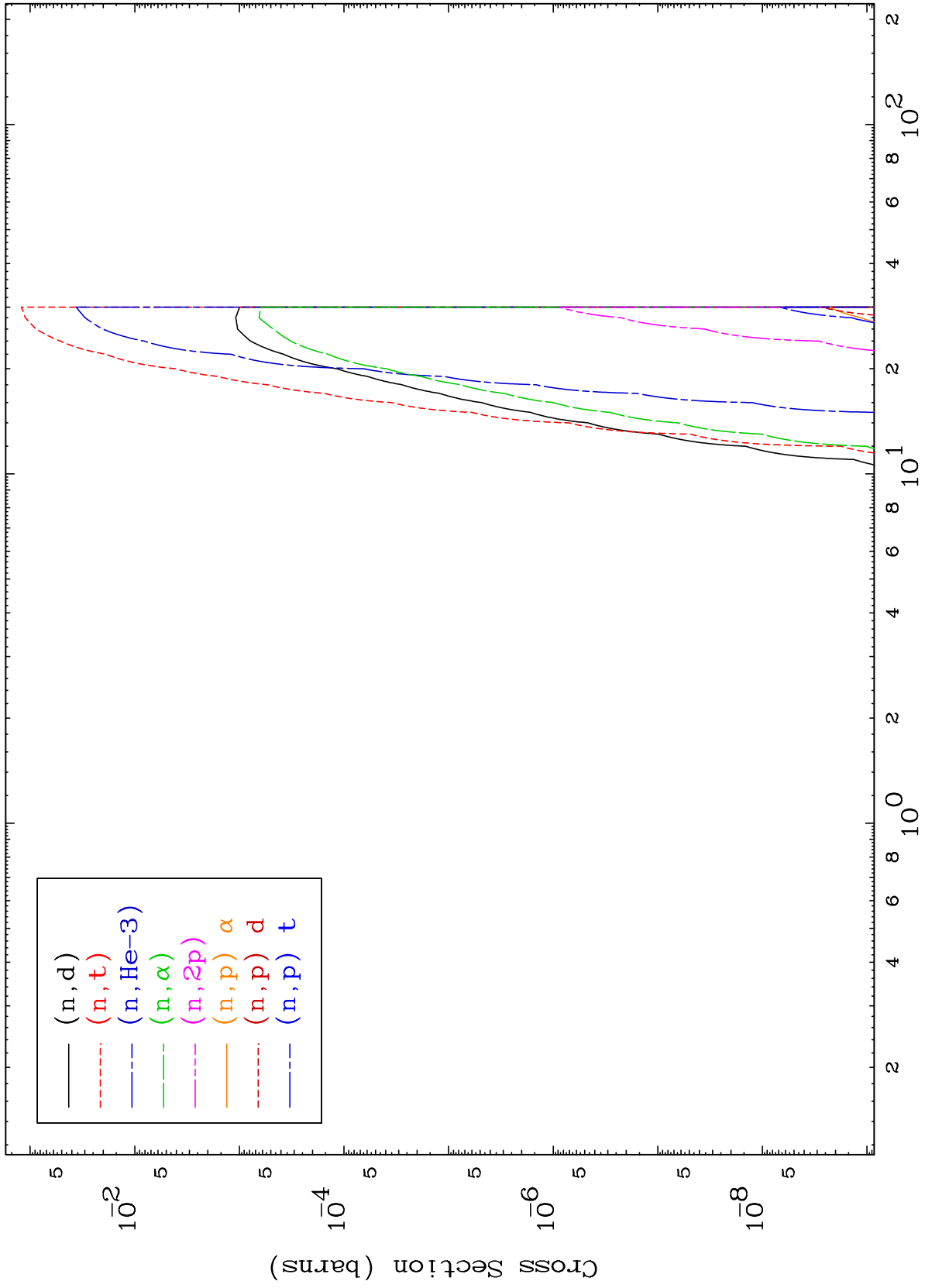
92-U -236



MAT 9231

He-3 Neutron Absorption
0 Kelvin Cross Sections

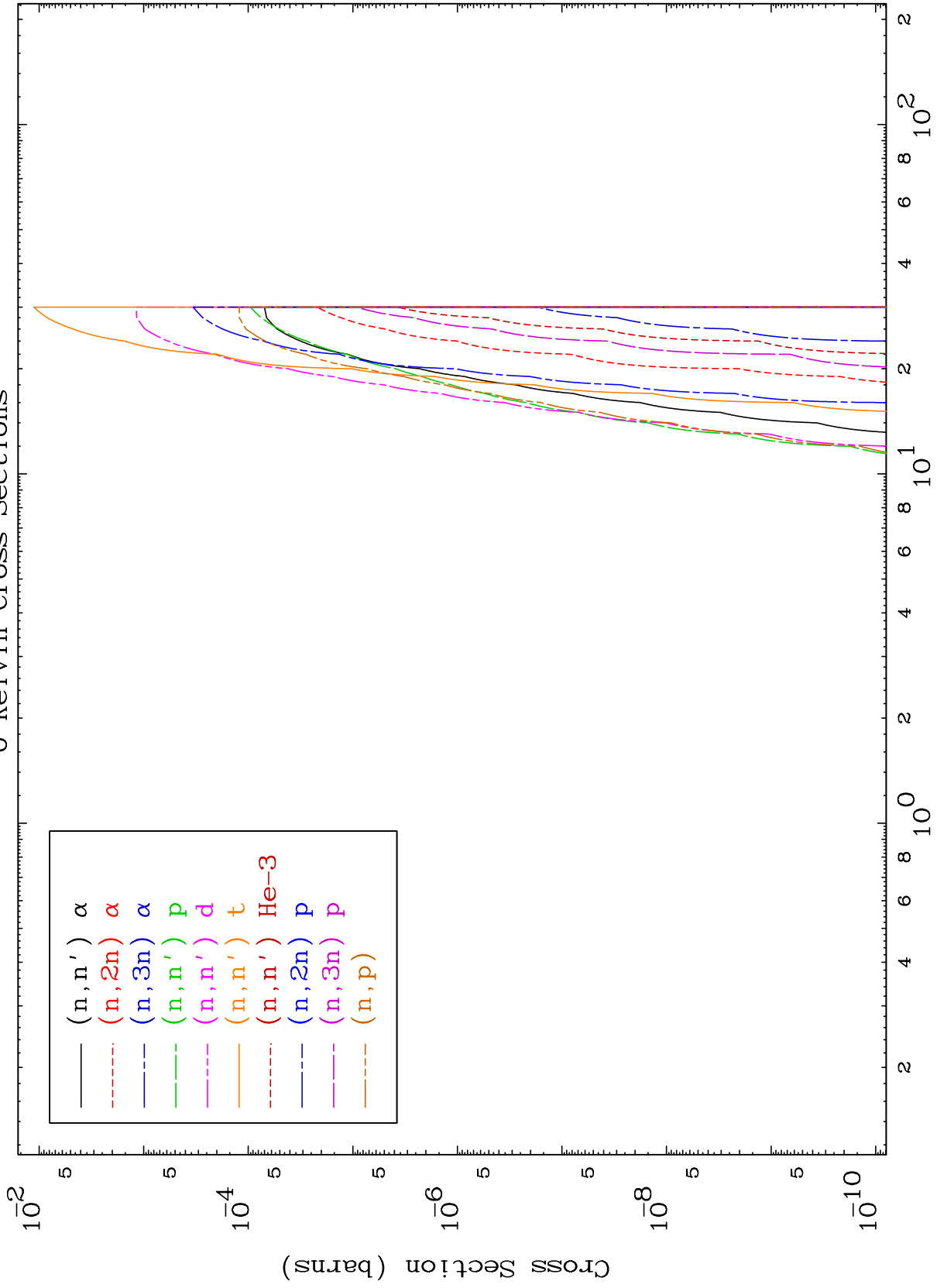
92-U -236



MAT 9231

He-3 Charged Particle
0 Kelvin Cross Sections

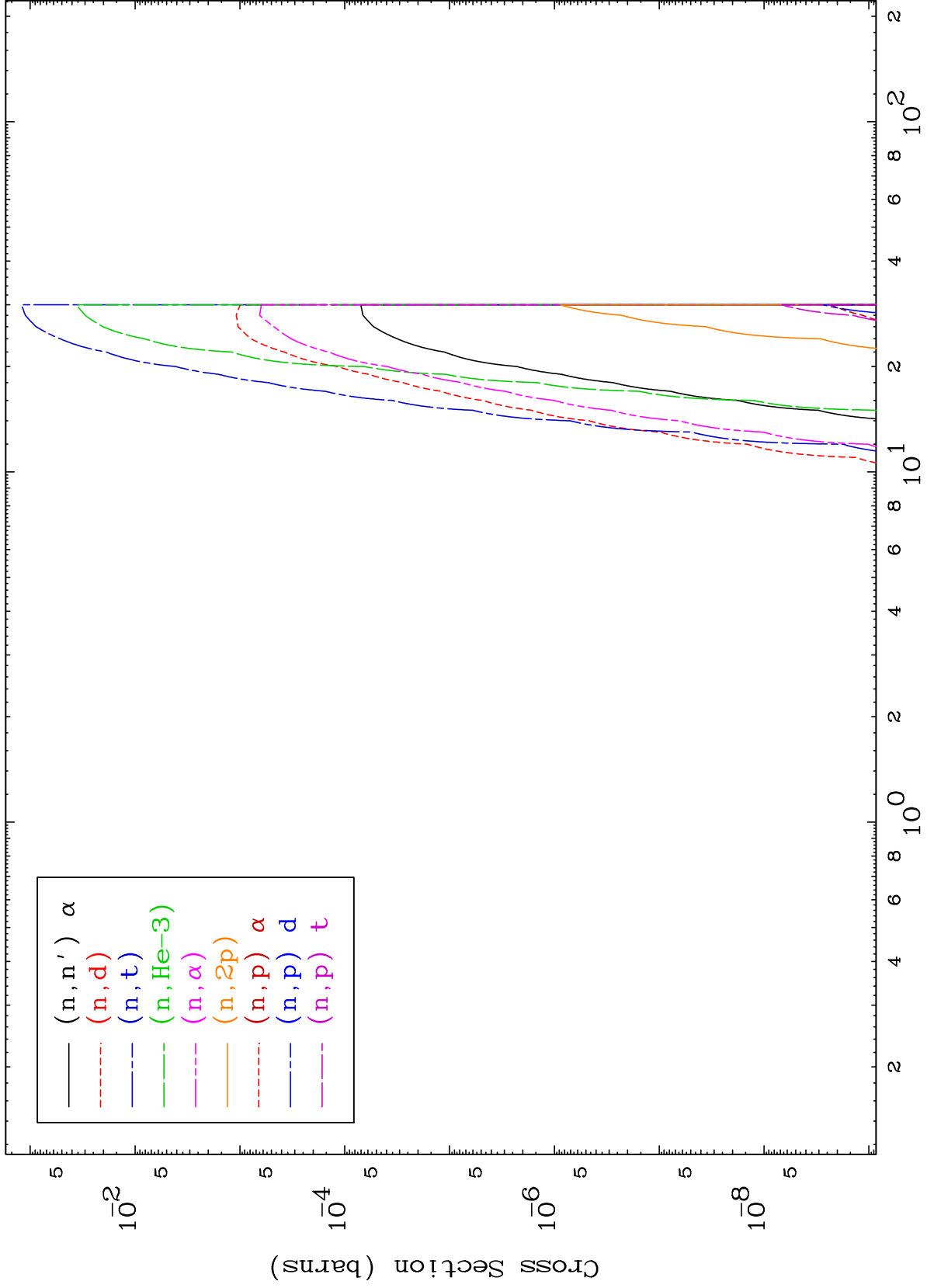
92-U -236



5

Incident Energy (MeV)

92-U -236

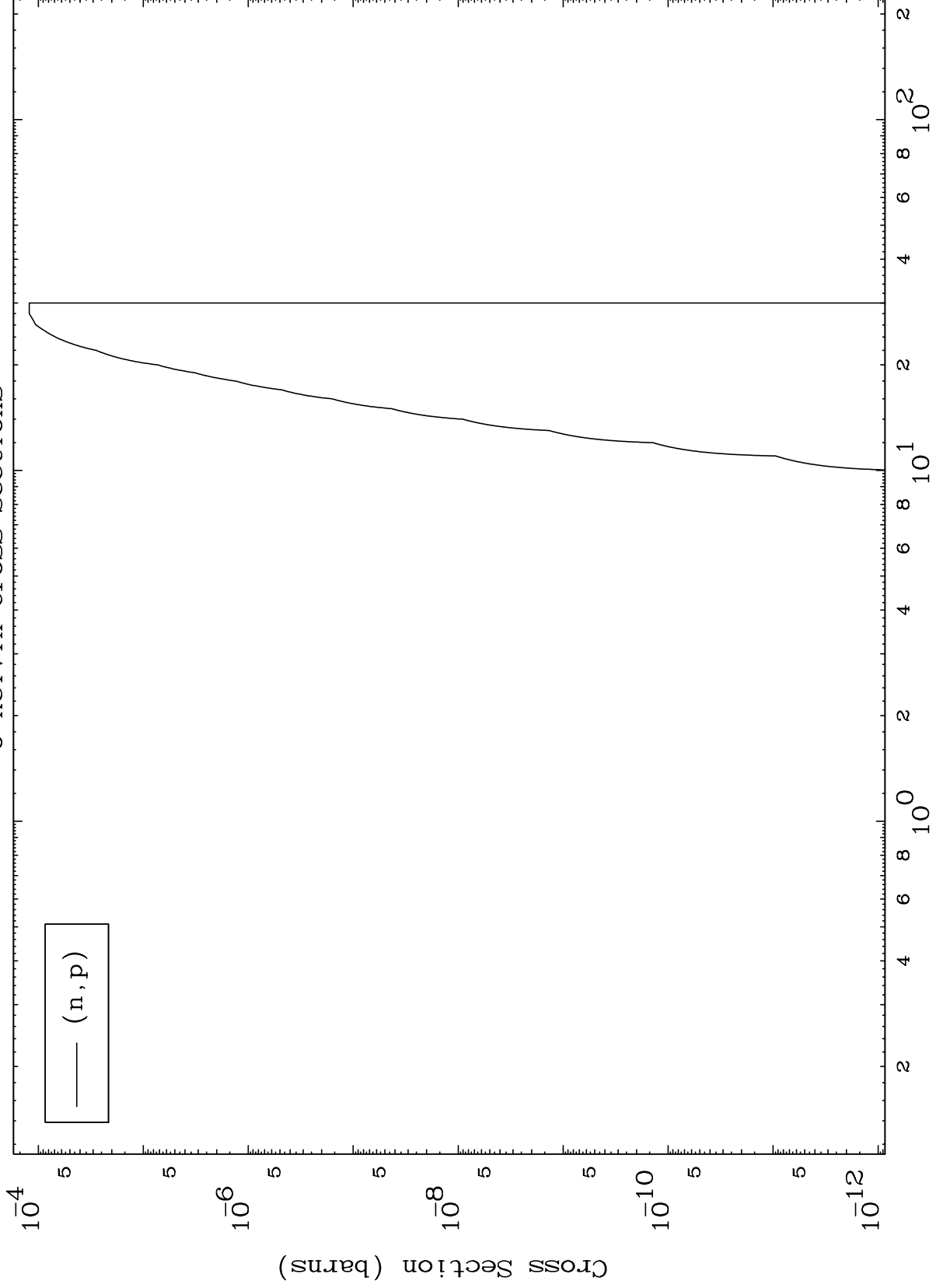


MAT 9231

(He-3,p) Levels

92-U -236

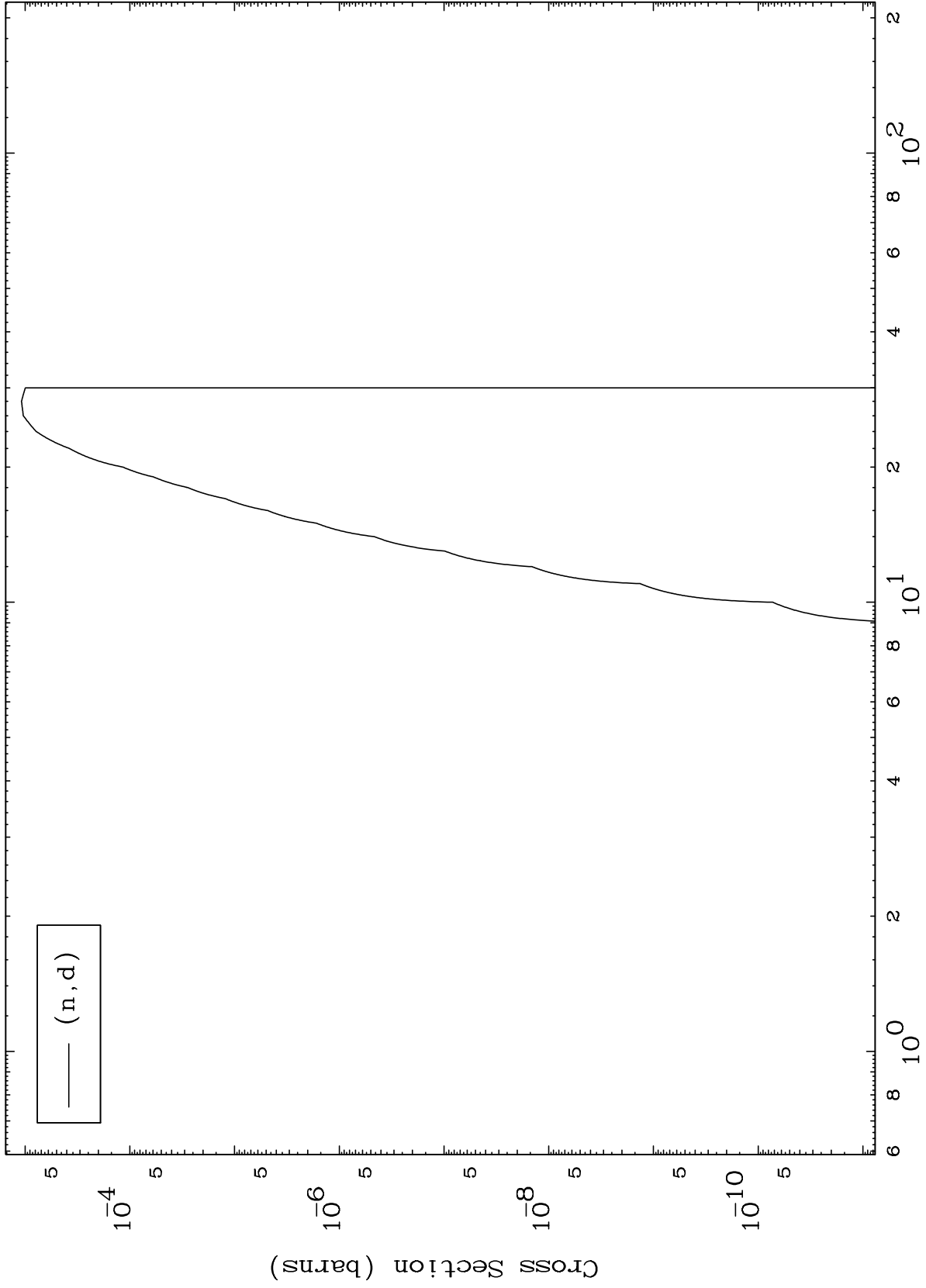
0 Kelvin Cross Sections



MAT 9231

(He-3,d) Levels
0 Kelvin Cross Sections

92-U -236



8

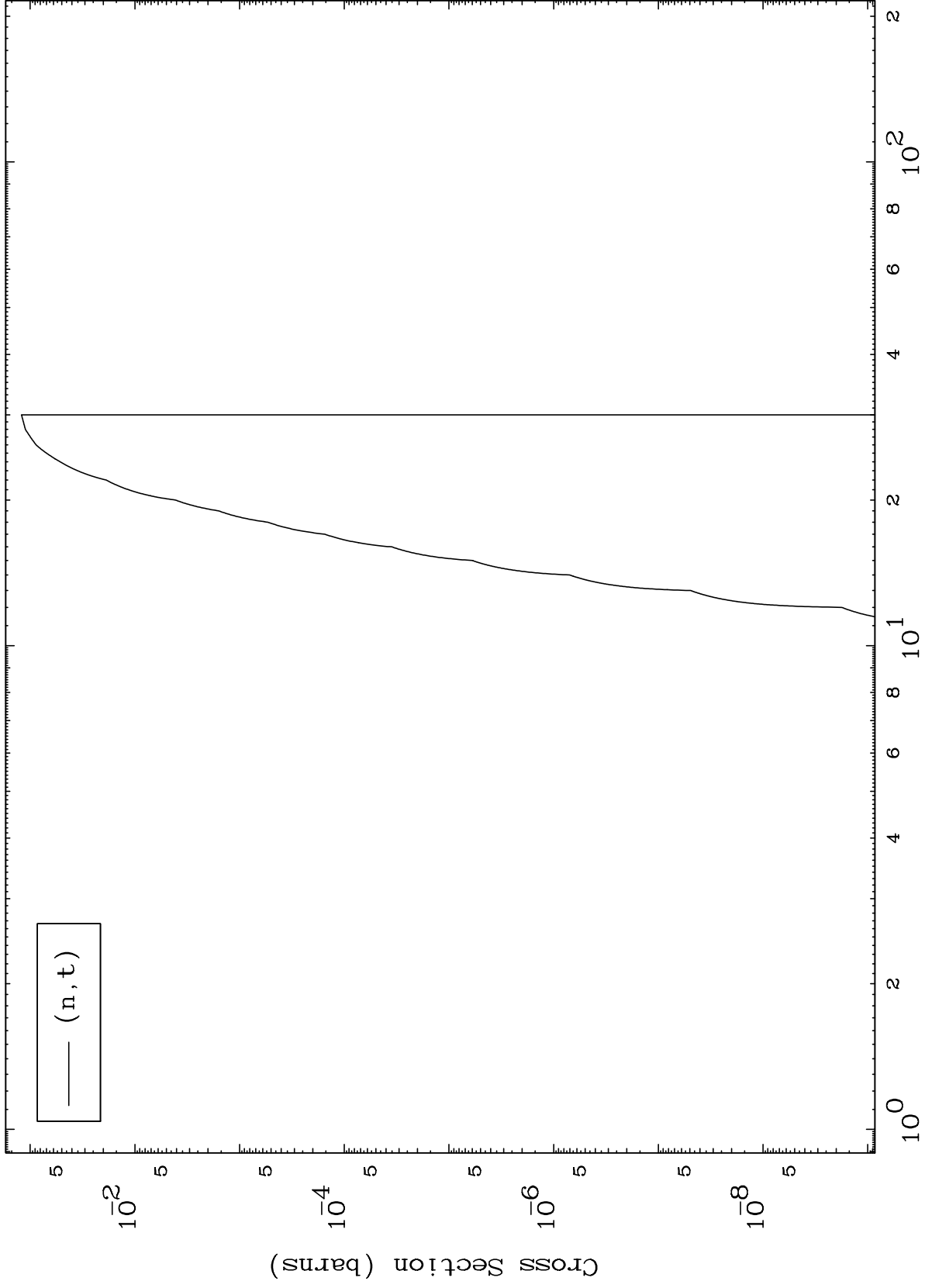
Incident Energy (MeV)

92-U -236

MAT 9231

(He-3,t) Levels
0 Kelvin Cross Sections

92-U -236



9

Incident Energy (MeV)

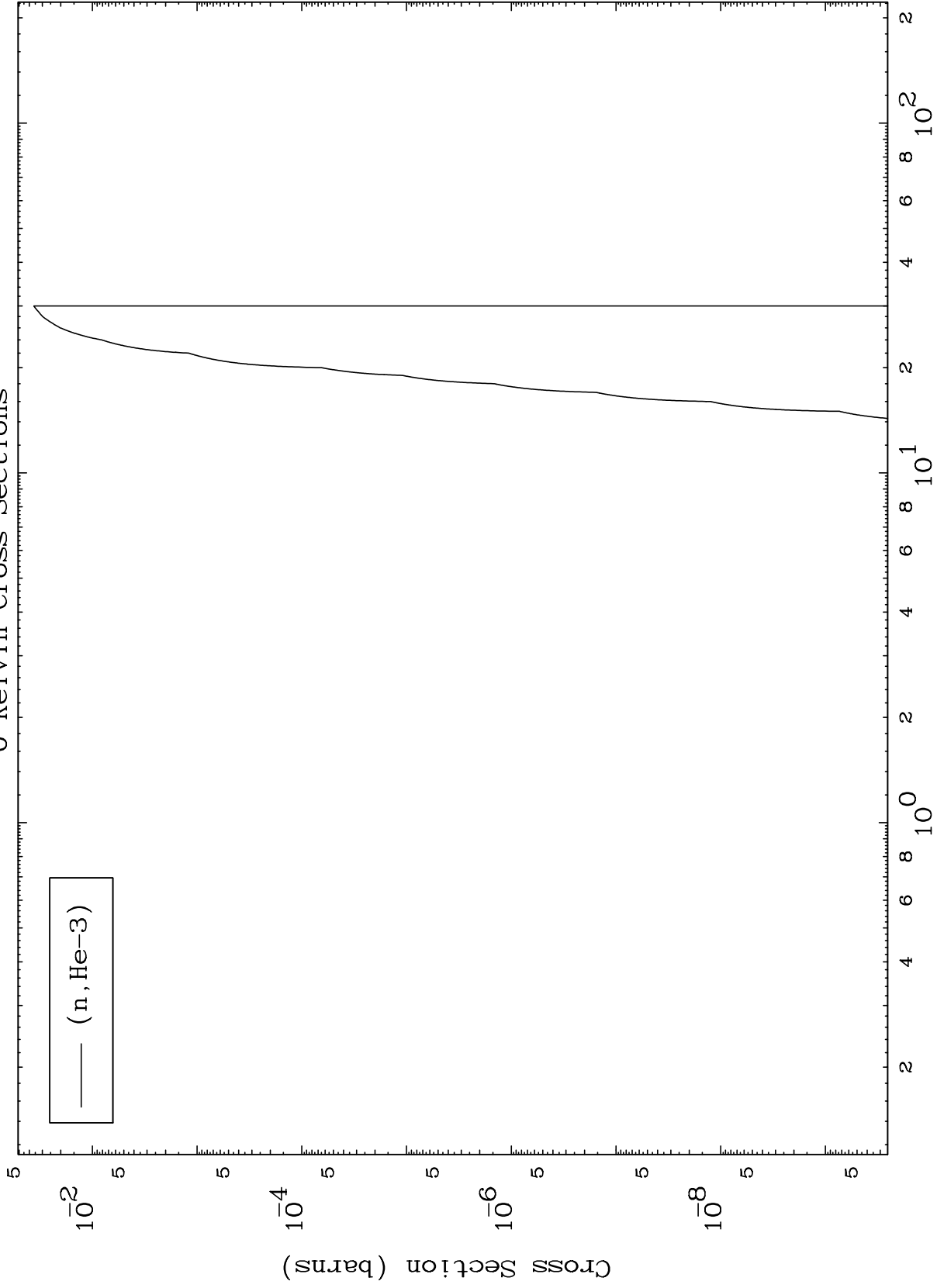
92-U -236

MAT 9231

(He-3, He3) Levels

92-U -236

0 Kelvin Cross Sections



10

Incident Energy (MeV)

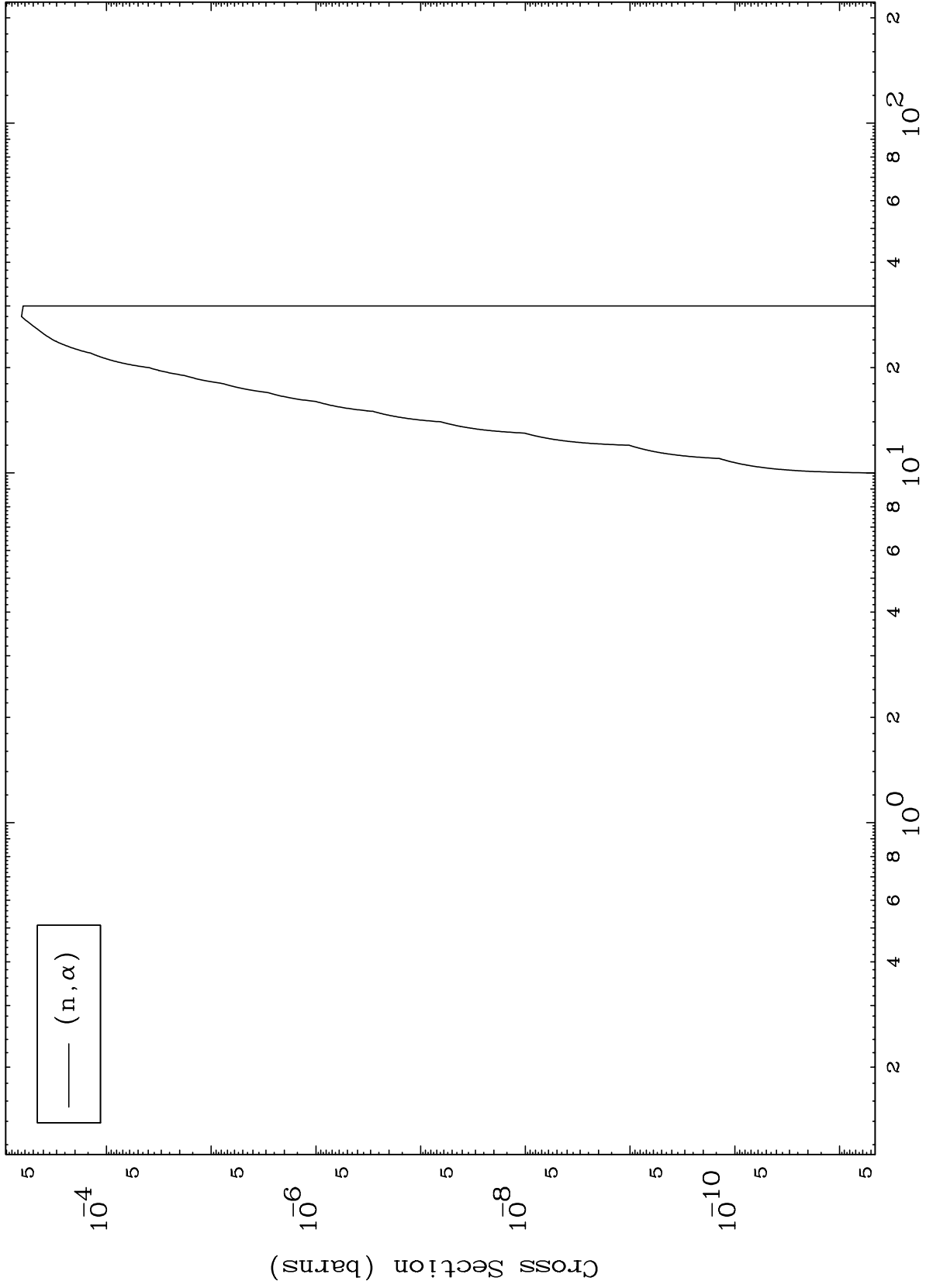
92-U -236

MAT 9231

(He-3, α) Levels

92-U -236

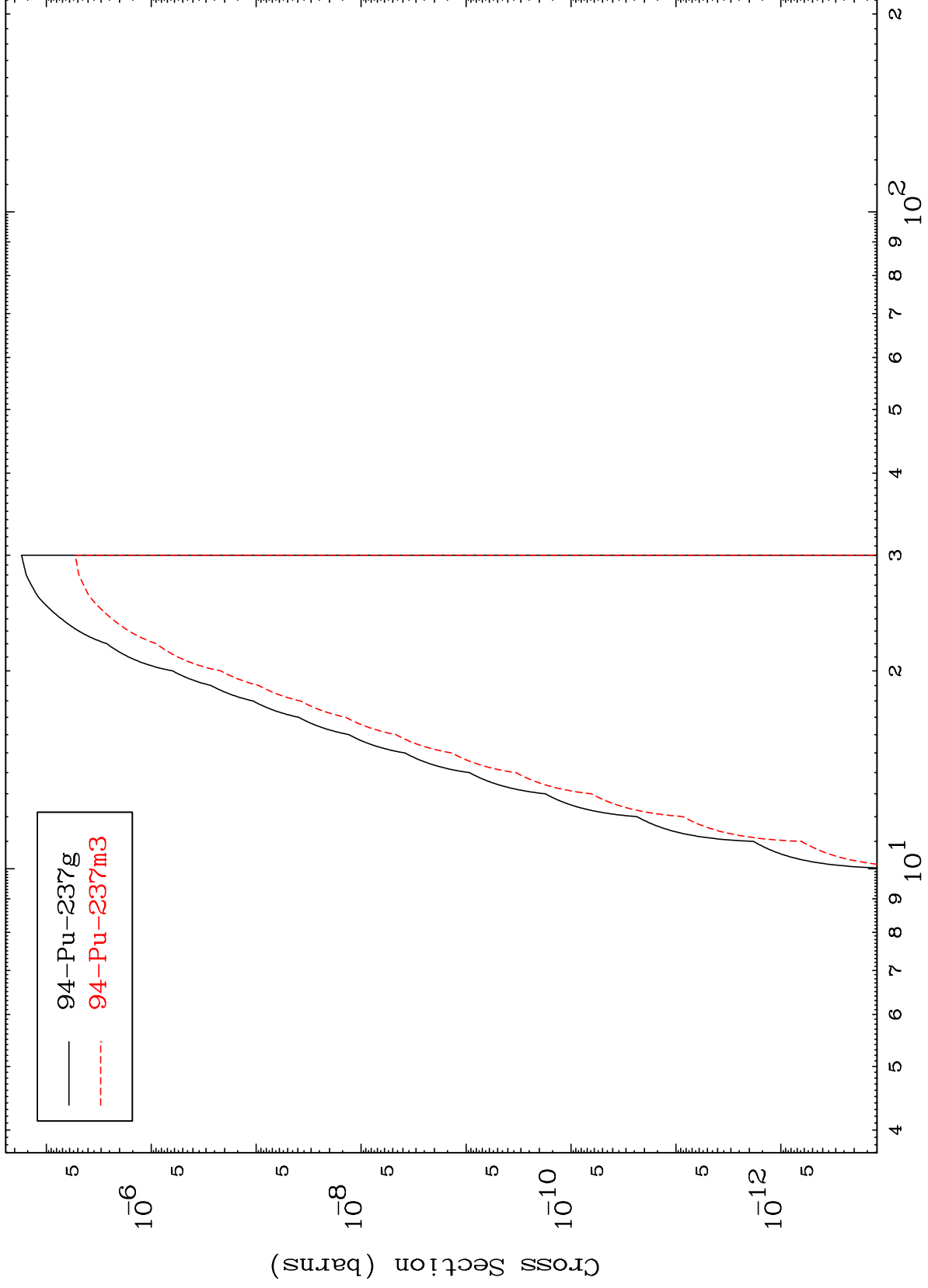
0 Kelvin Cross Sections



MAT 9231

92-U -236

Radionuclide Production Cross Section (n,2n)



12

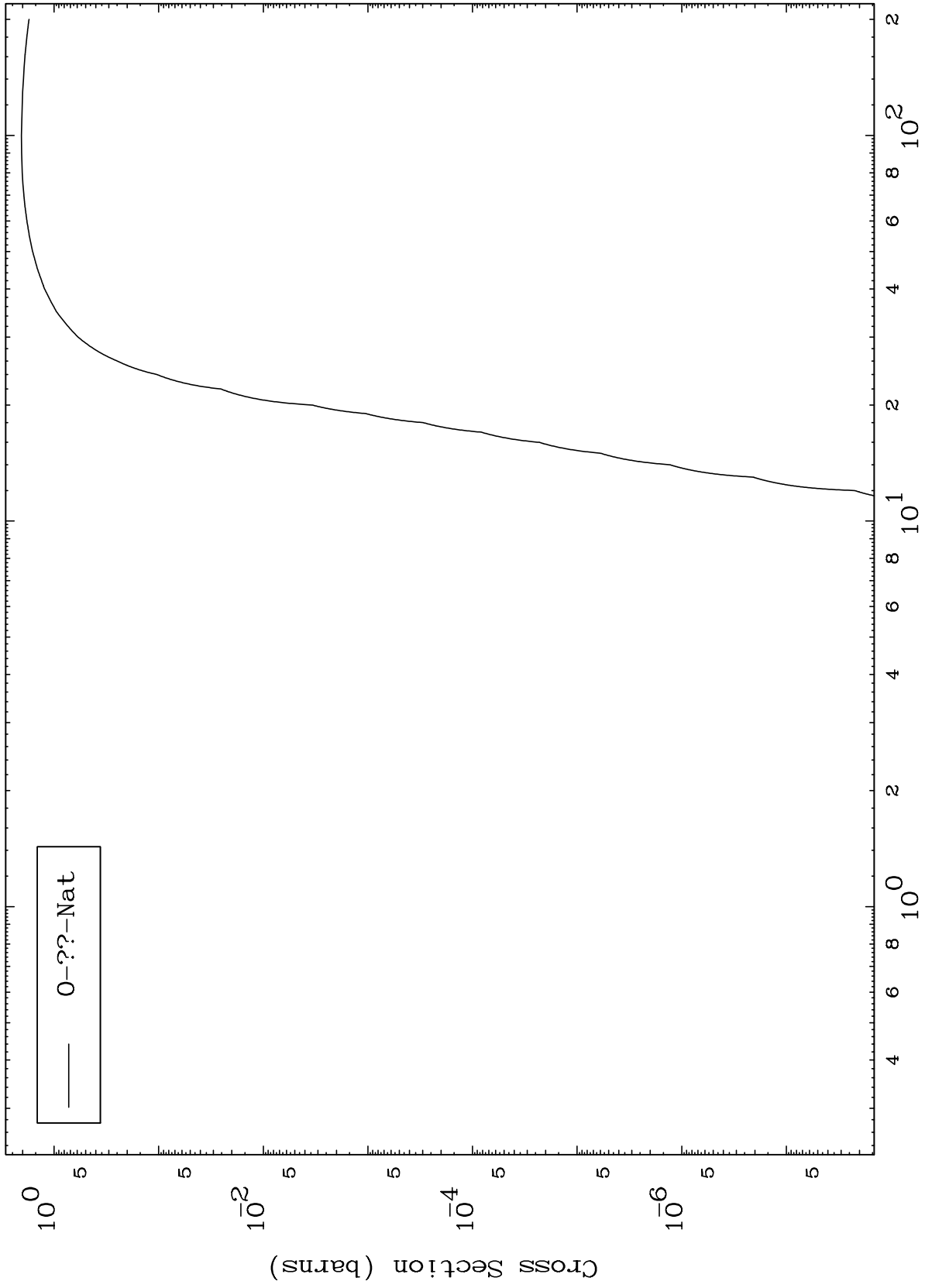
Incident Energy (MeV)

92-U -236

MAT 9231

92-U -236

Fission
Radionuclide Production Cross Section

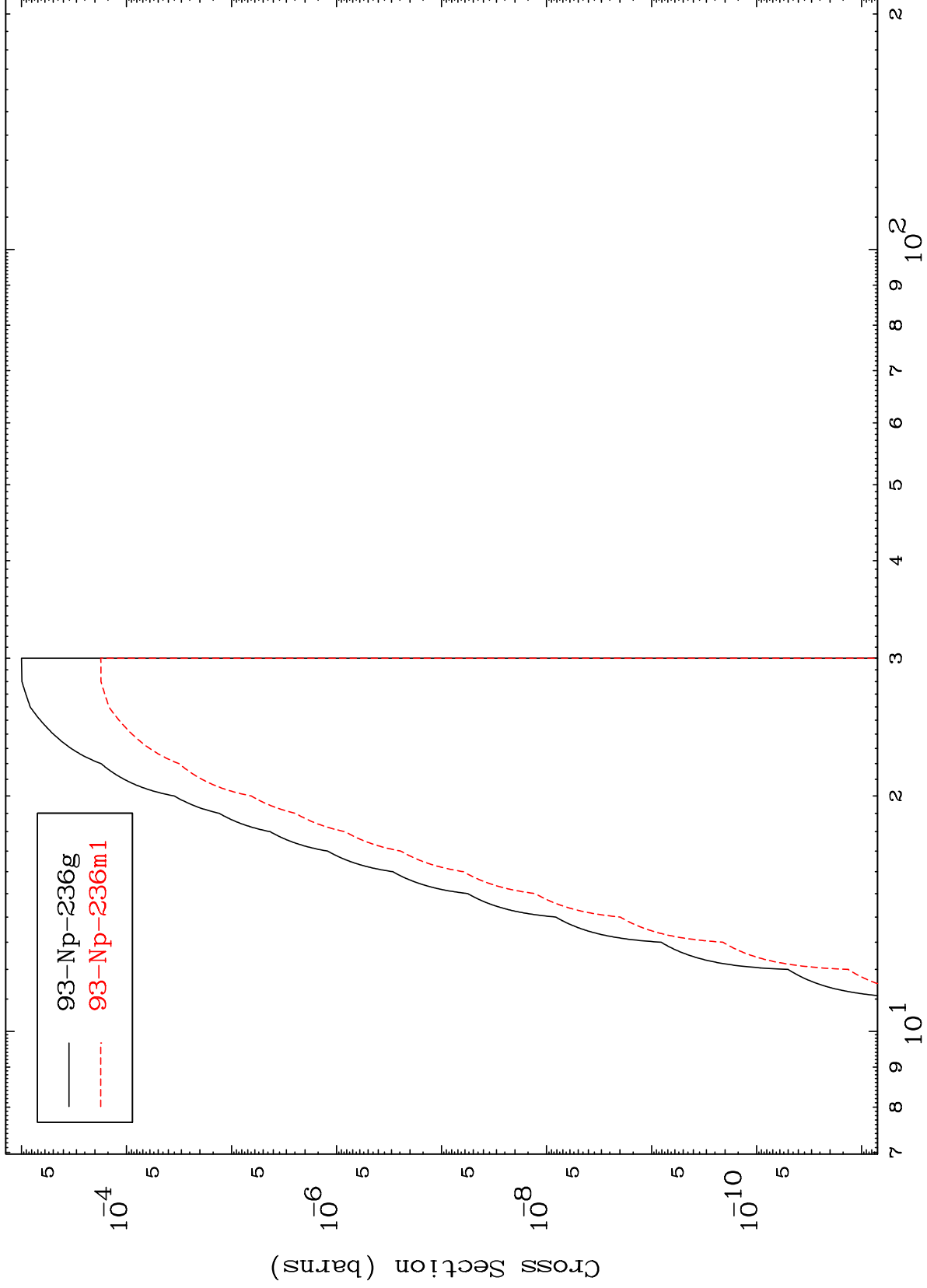


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(n,n') d

92-U -236

Radionuclide Production Cross Section



14

Incident Energy (MeV)

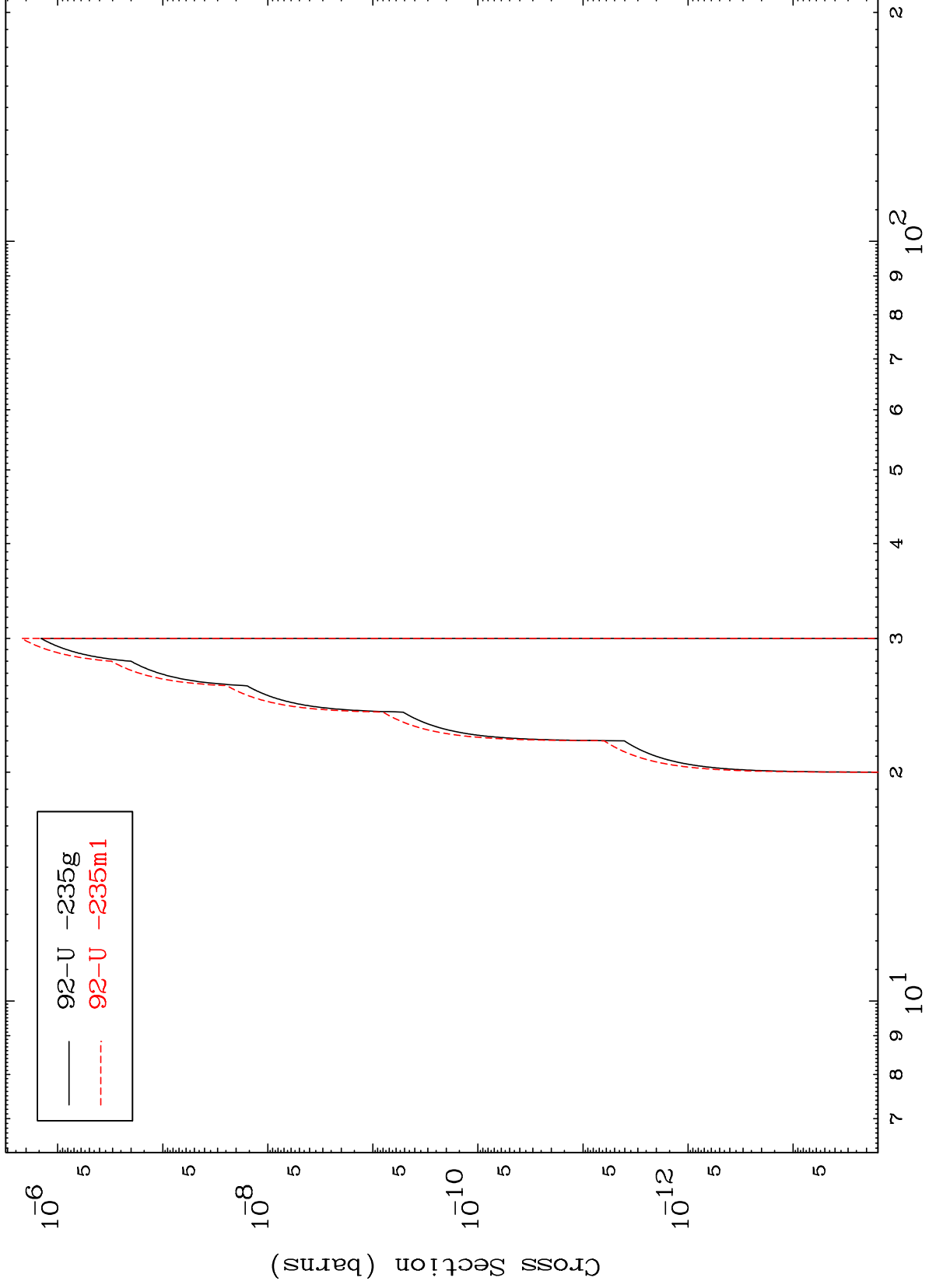
92-U -236

MAT 9231

(n,n') He-3

92-U -236

Radionuclide Production Cross Section



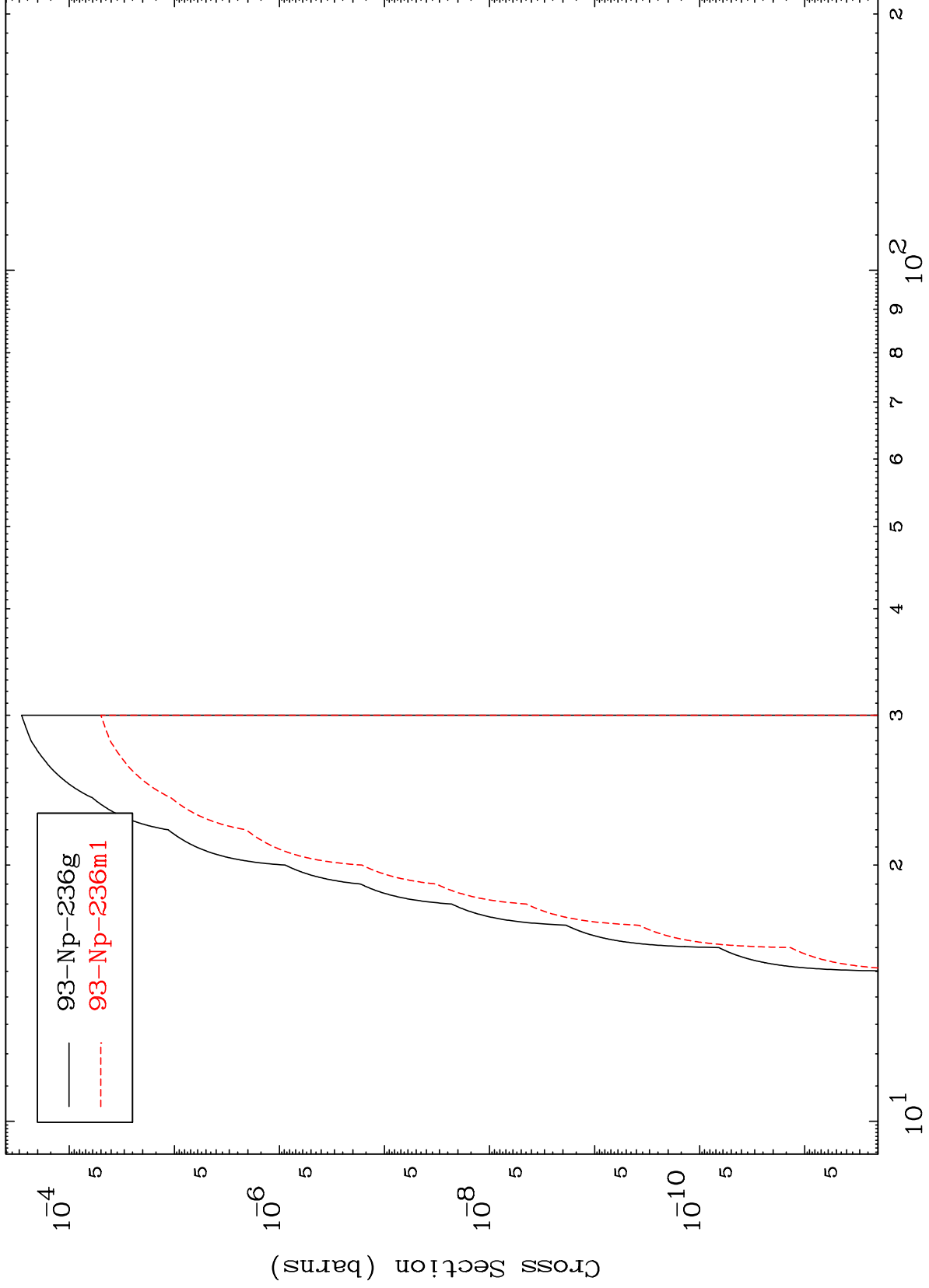
Legend:
— 92-U -235g
- - - 92-U -235m1

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(n,2n) p

92-U -236

Radionuclide Production Cross Section



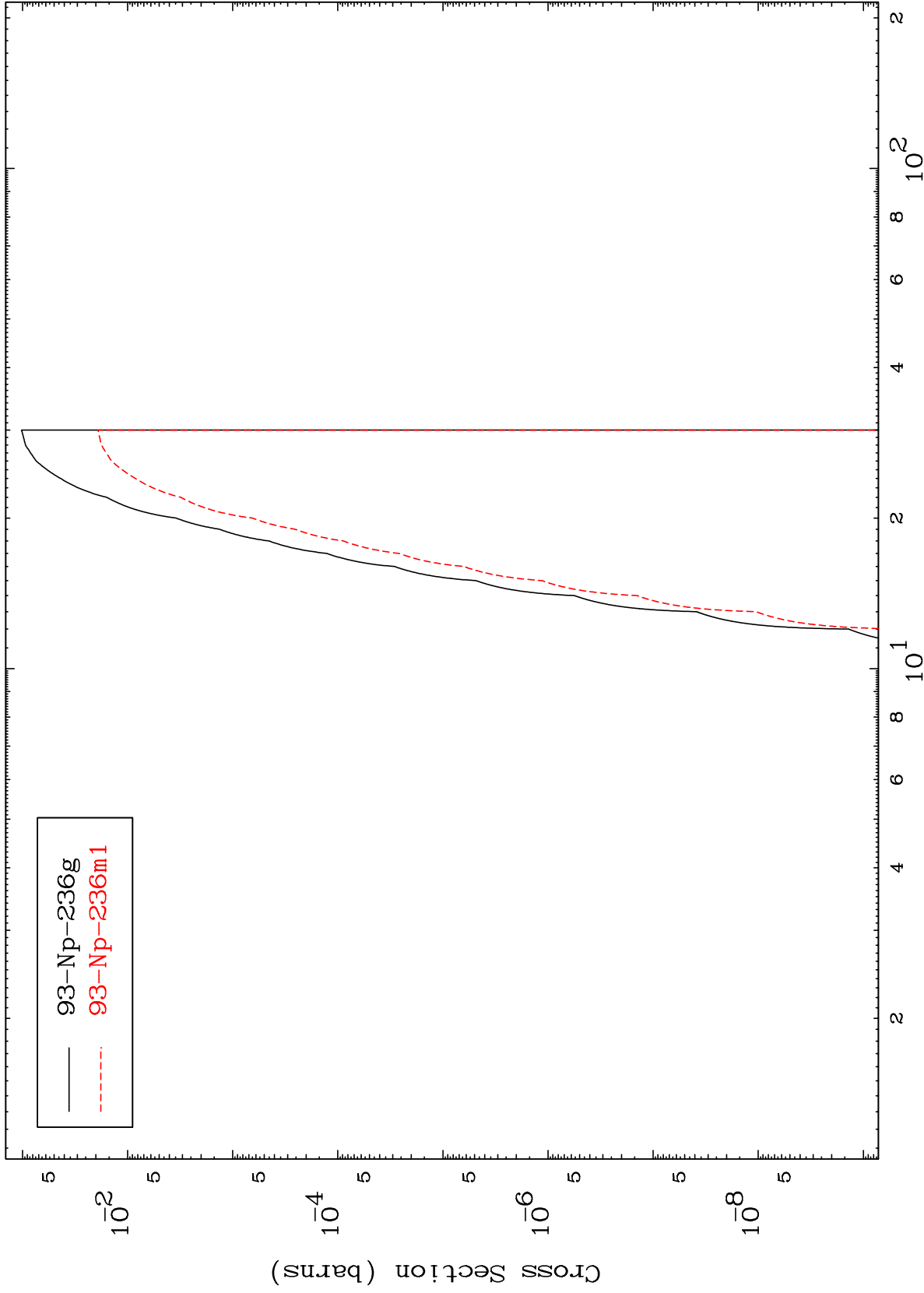
Incident Energy (MeV)

92-U -236

MAT 9231

92-U -236

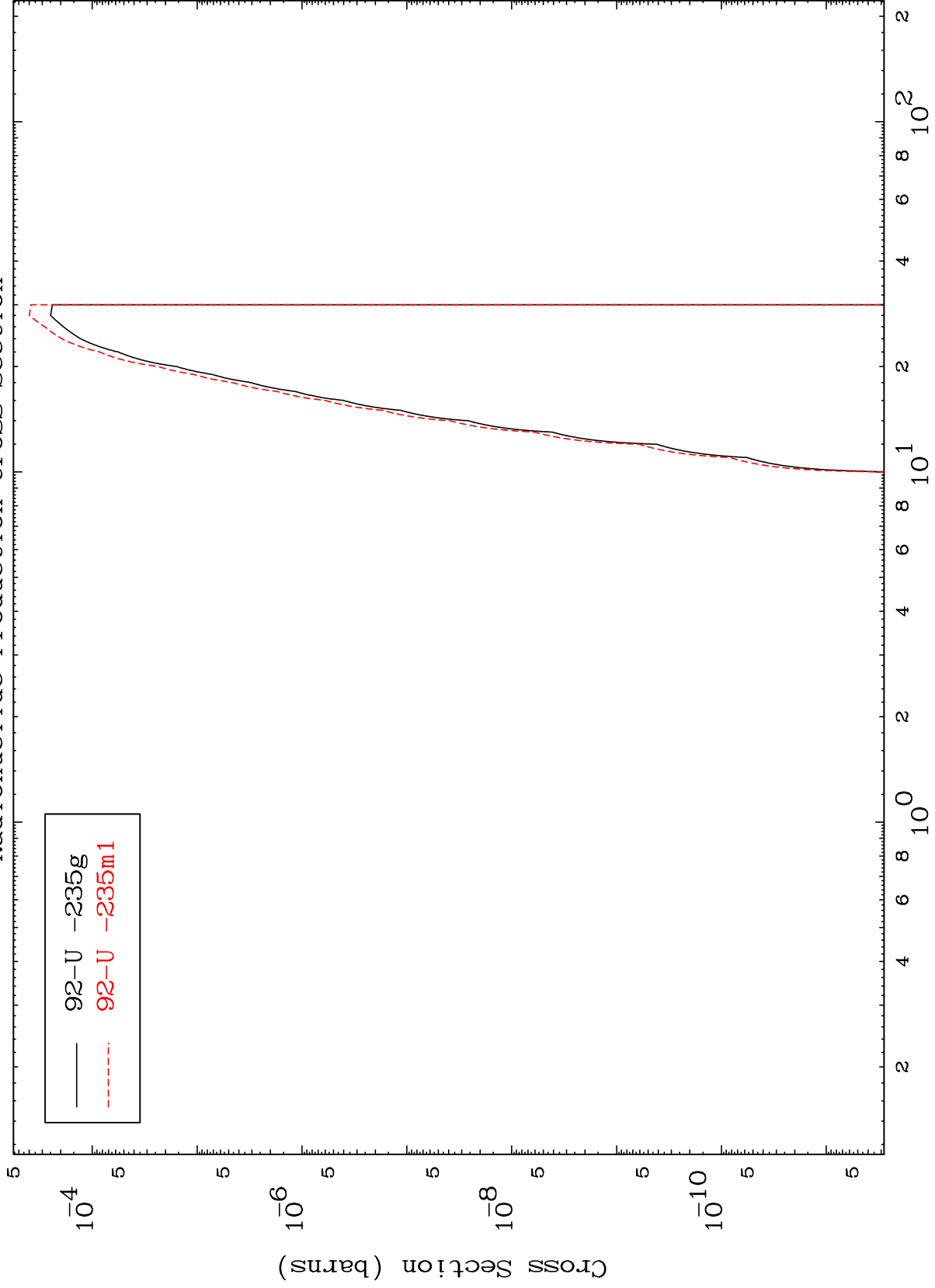
(n,t)
Radionuclide Production Cross Section



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92-U -236

(n, α)
Radionuclide Production Cross Section



18

92-U -236

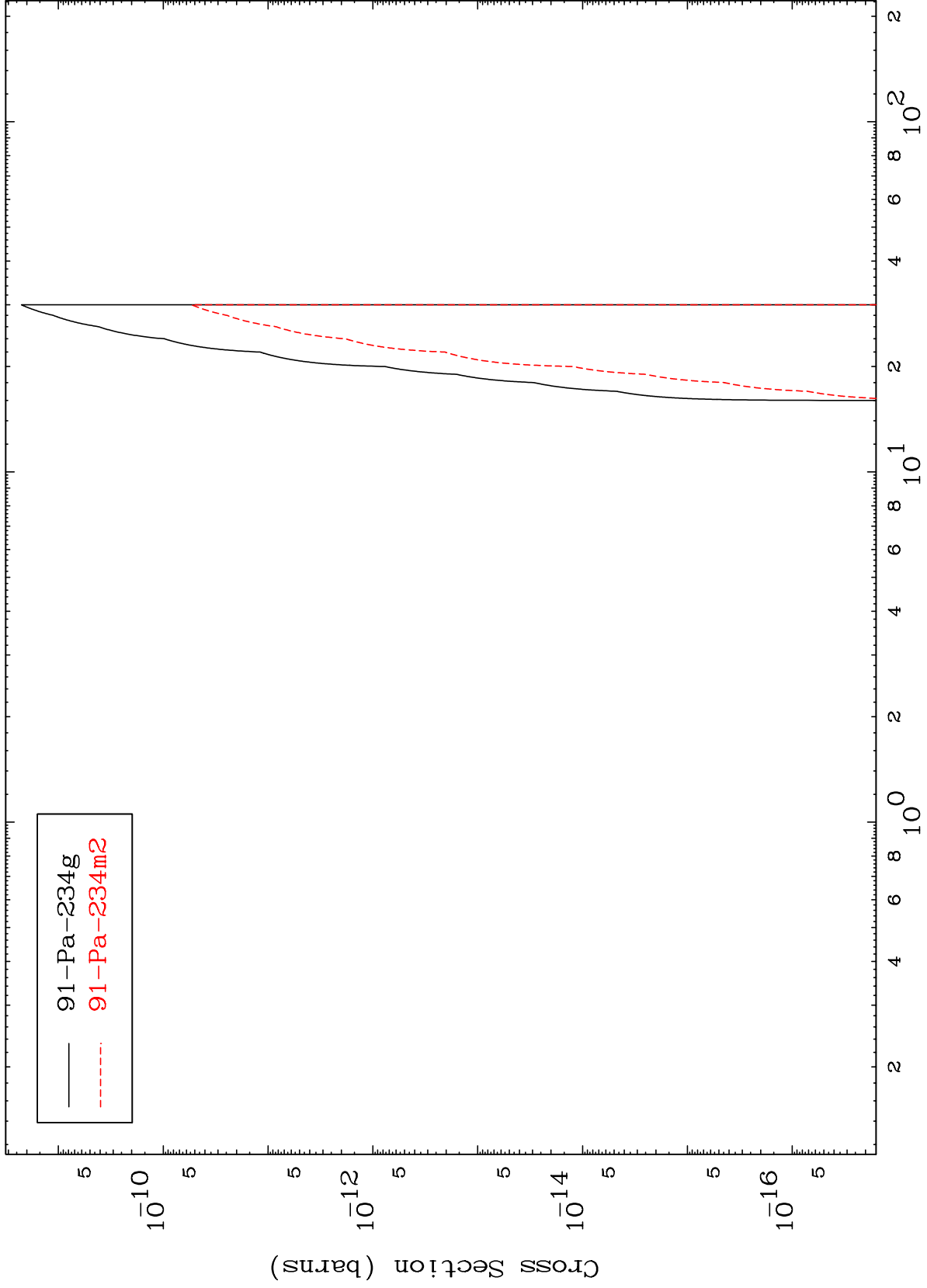
Incident Energy (MeV)

MAT 9231

(n,p) α

92-U -236

Radionuclide Production Cross Section



19

Incident Energy (MeV)

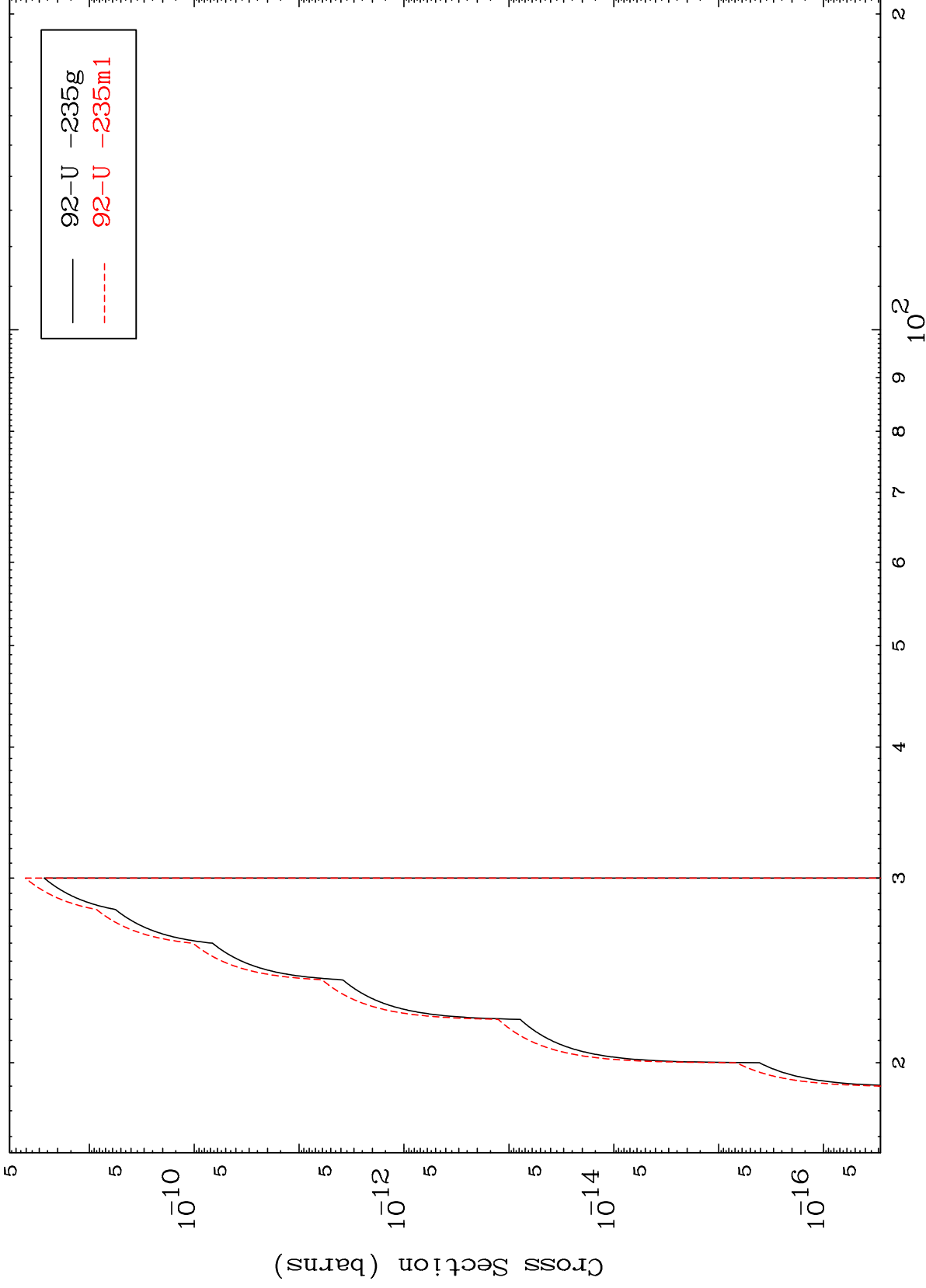
92-U -236

MAT 9231

(n,p) t

92-U -236

Radionuclide Production Cross Section



20

Incident Energy (MeV)

92-U -236