

Program EVALPLOT
(Version 2021-1)

by

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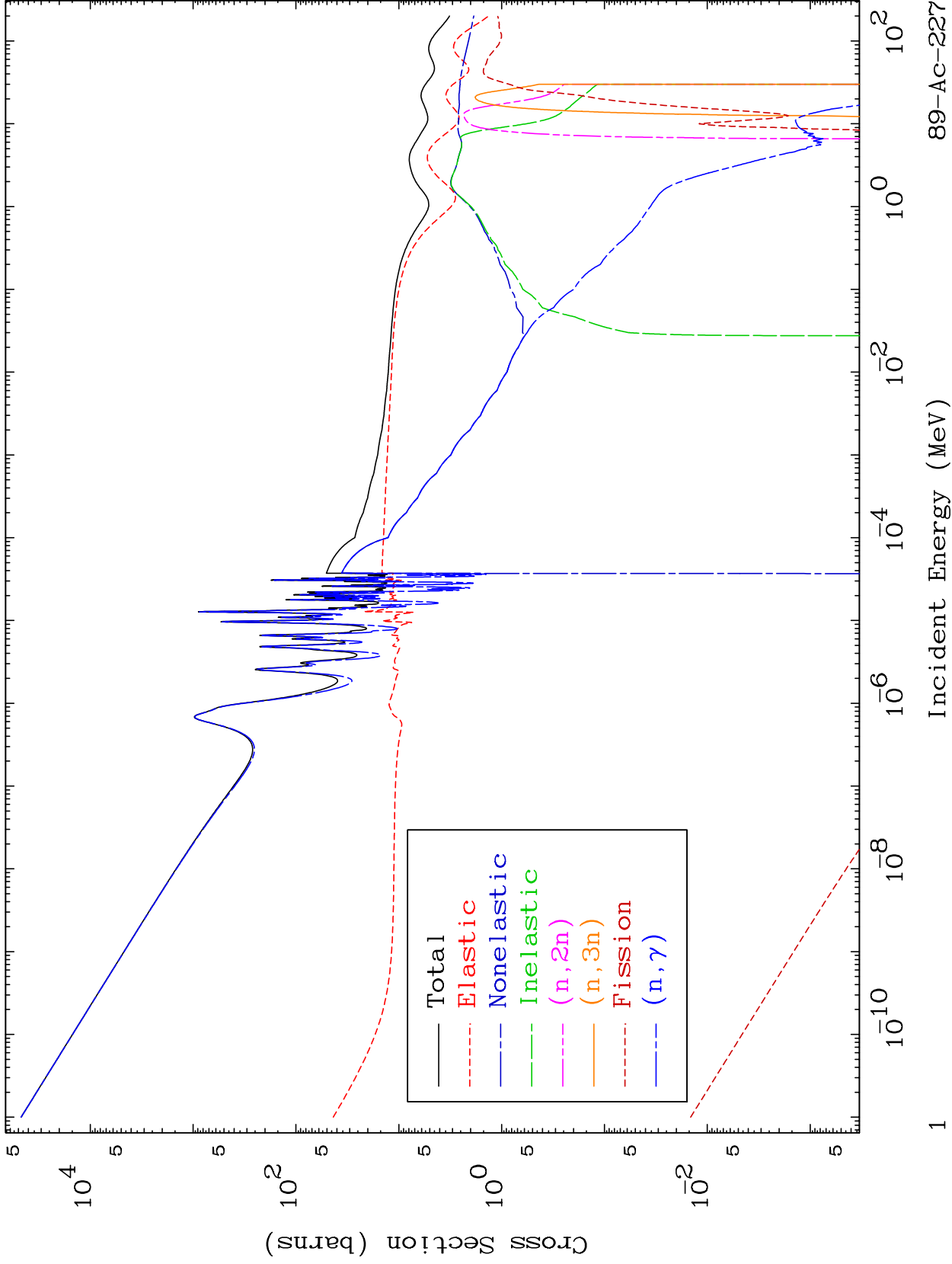
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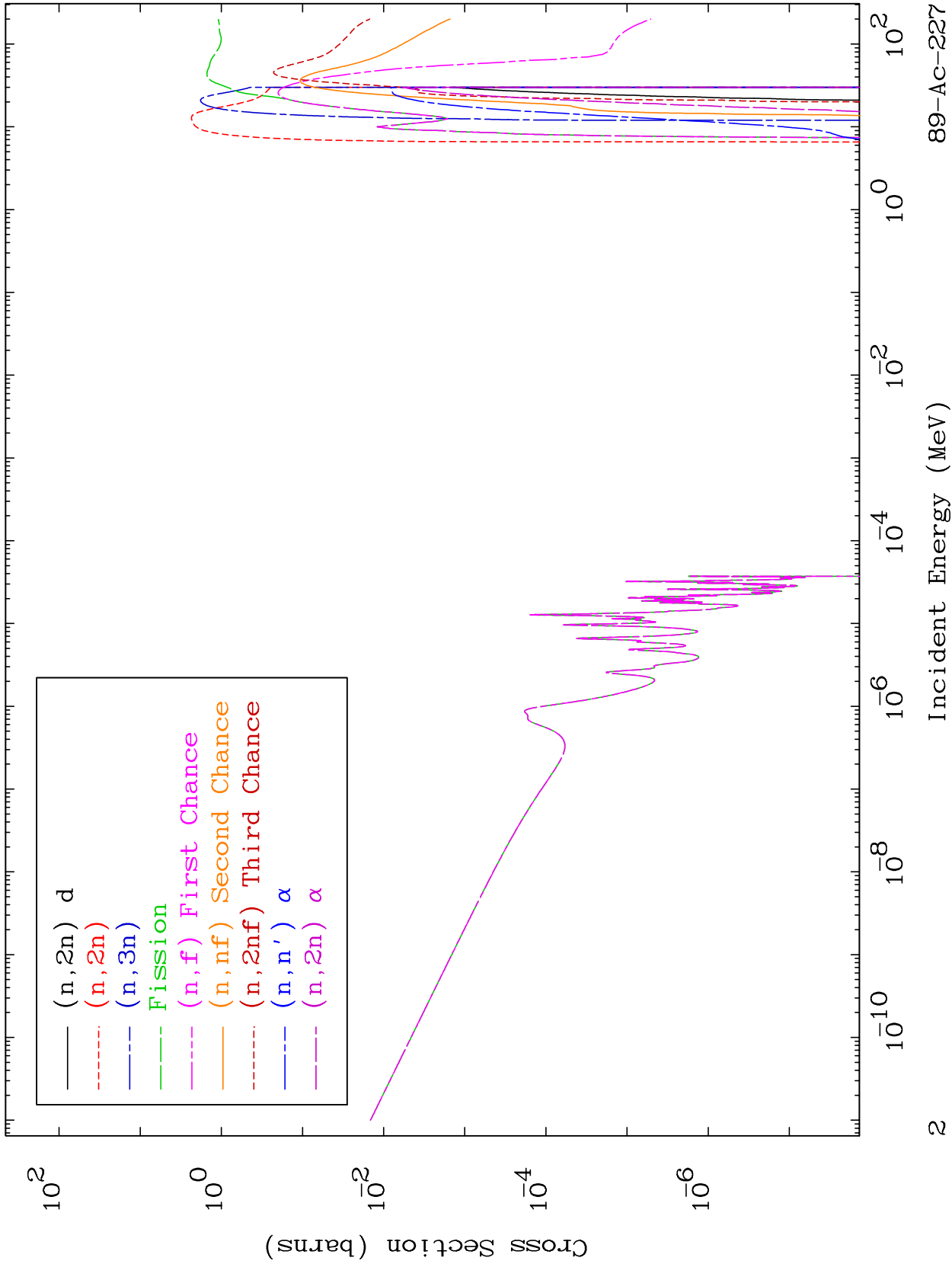
Press Mouse Button to Start

MAT 8931

Neutron Major
293 Kelvin Cross Sections

89-Ac-227

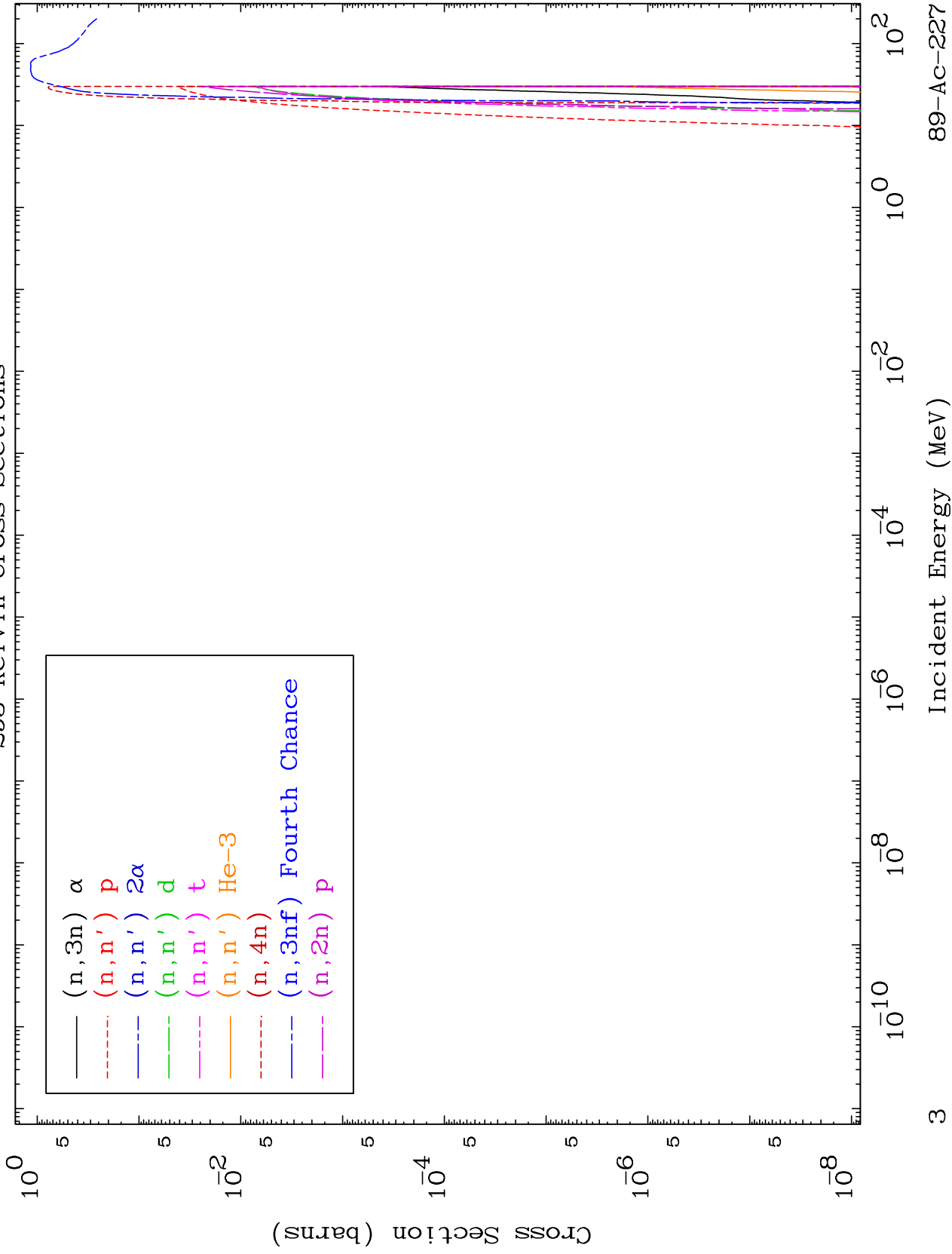




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Neutron Absorption
293 Kelvin Cross Sections

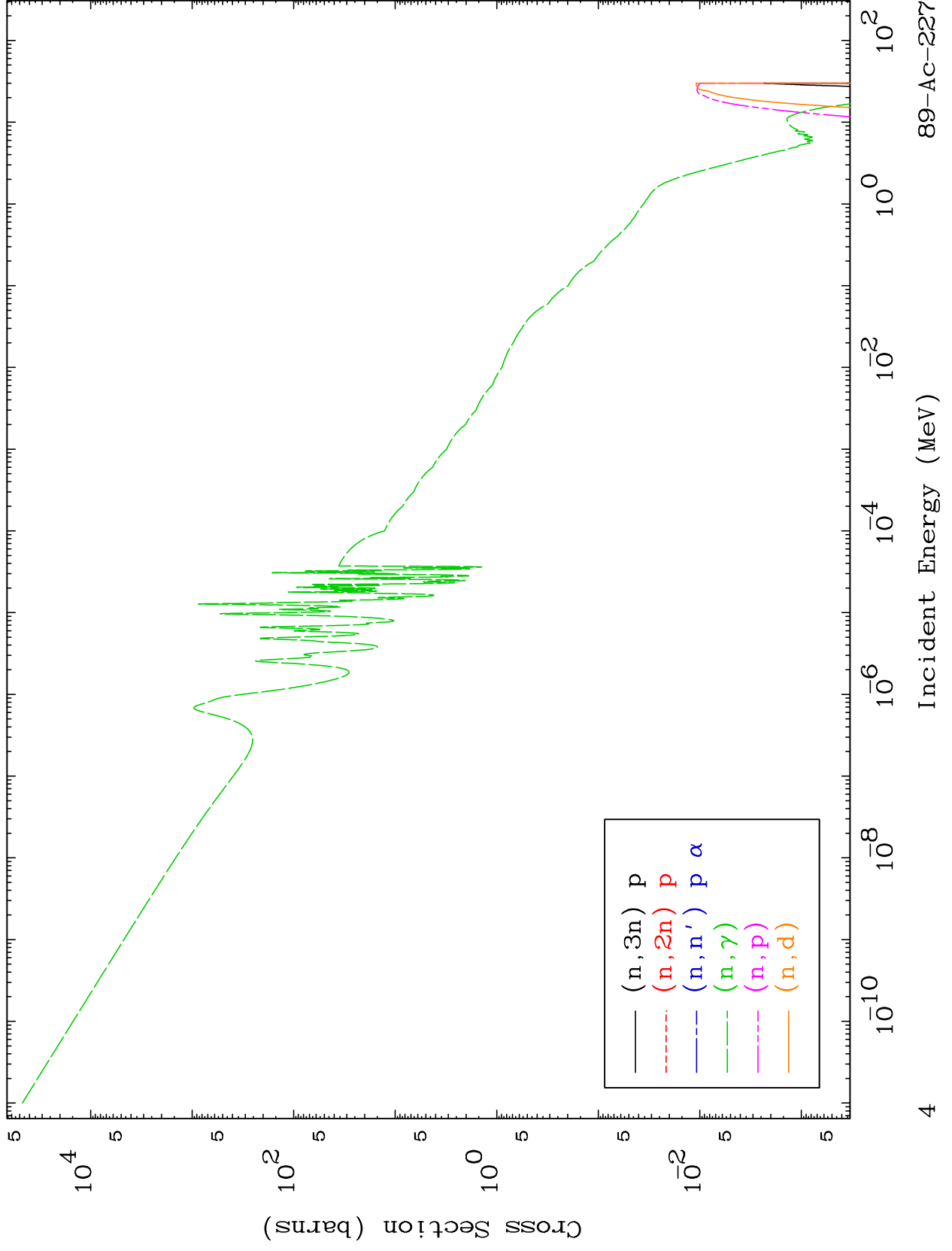
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Neutron Absorption
293 Kelvin Cross Sections

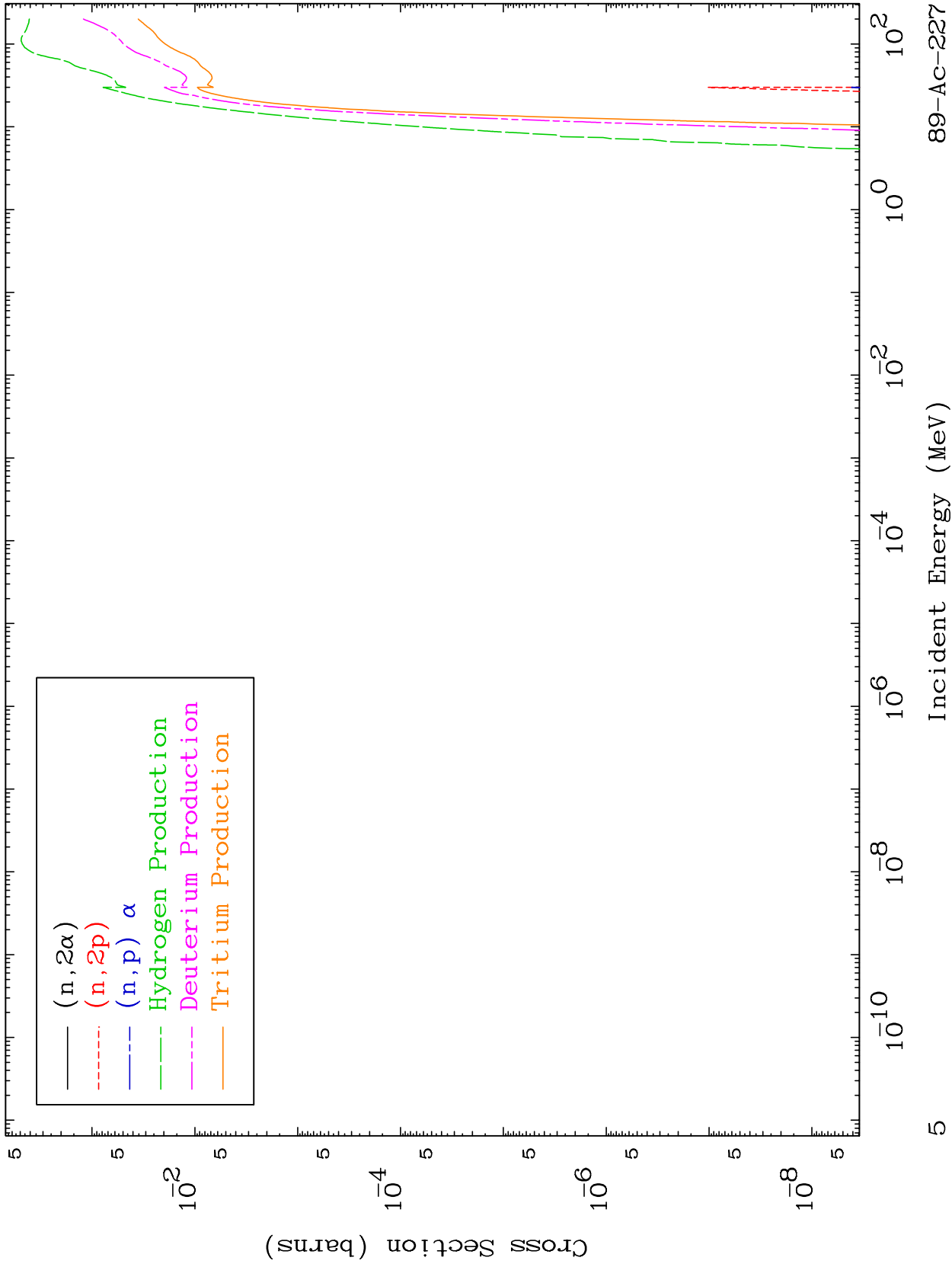
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Neutron Absorption
293 Kelvin Cross Sections

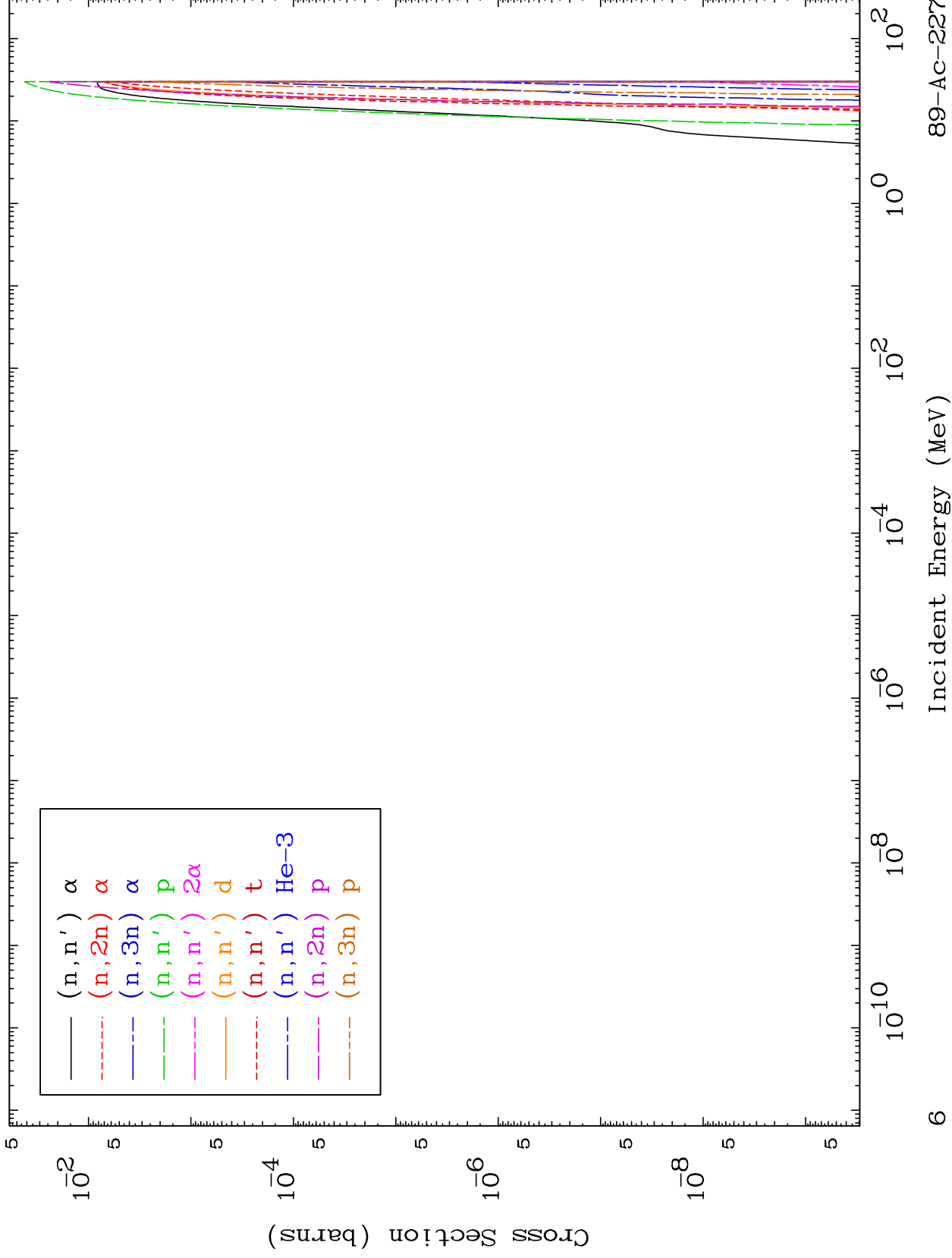
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Charged Particle
293 Kelvin Cross Sections

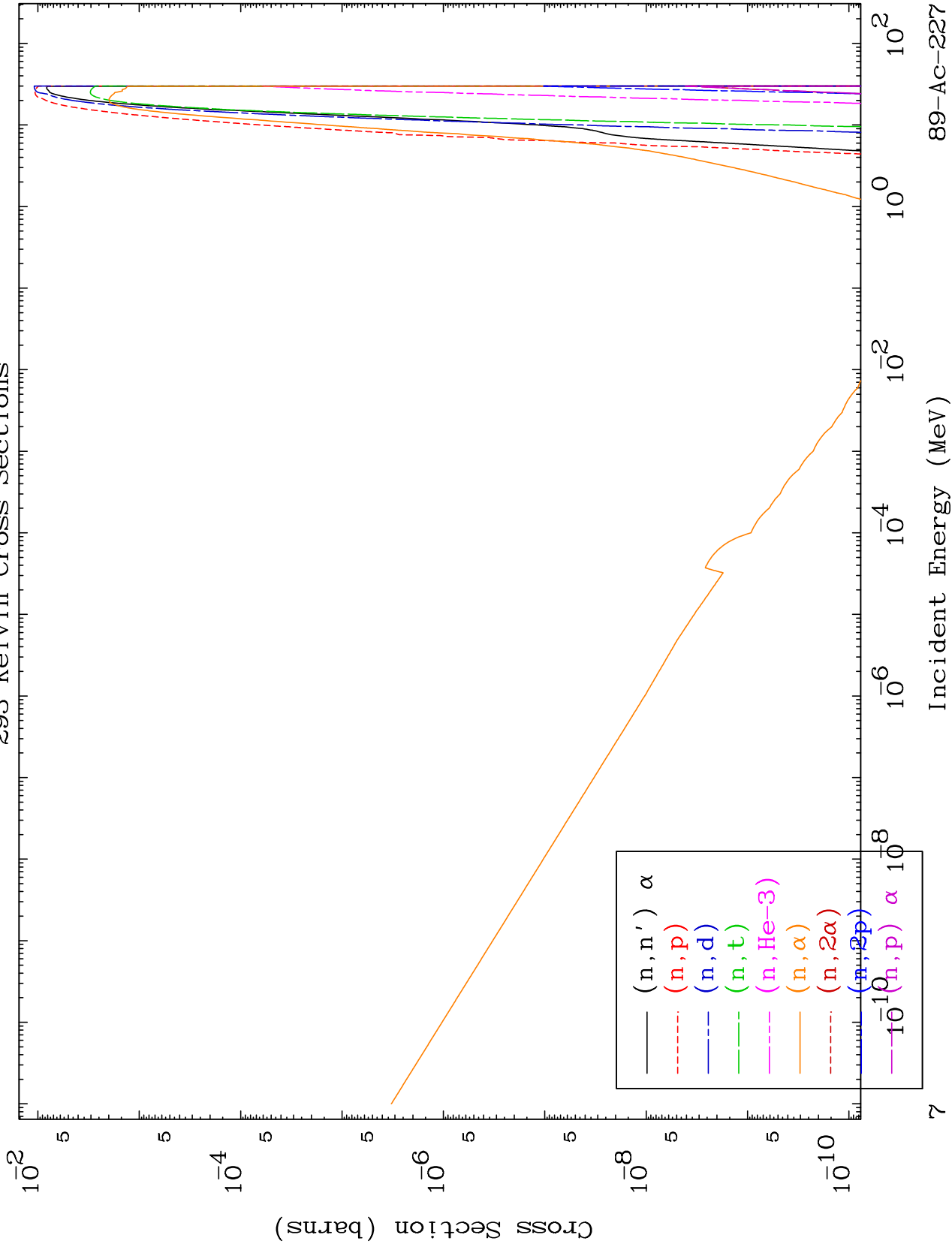
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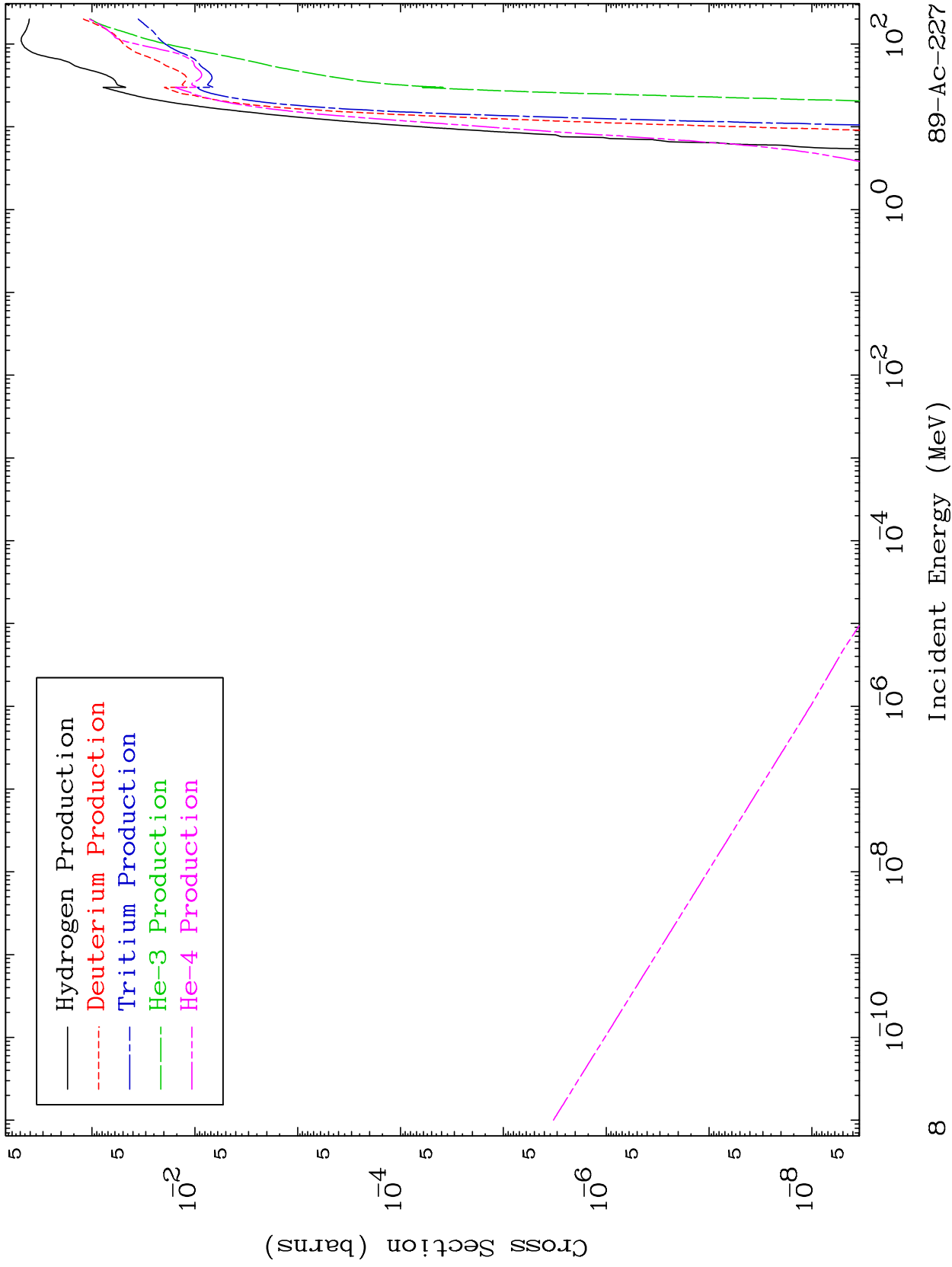


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Charged Particle
293 Kelvin Cross Sections

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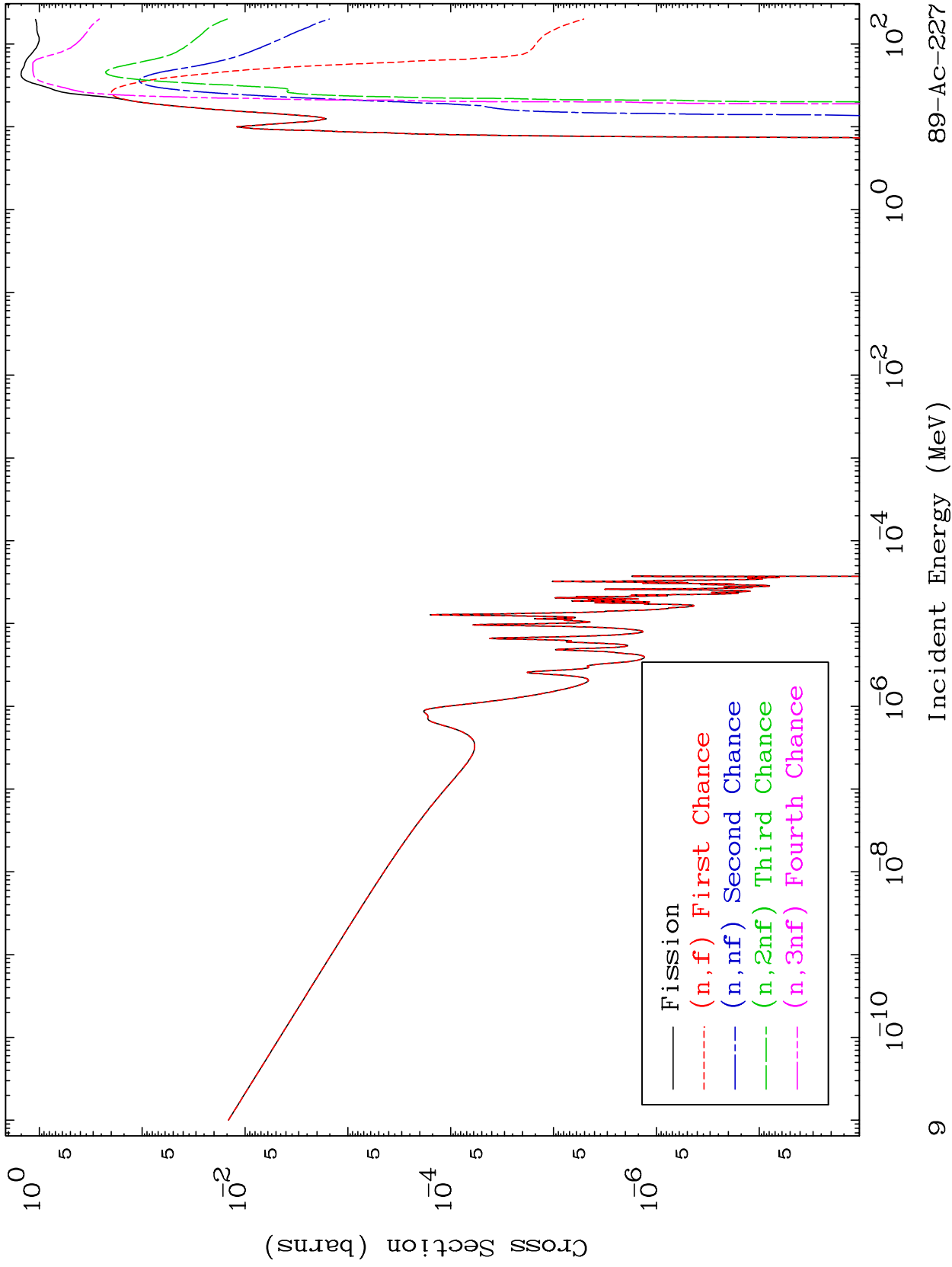


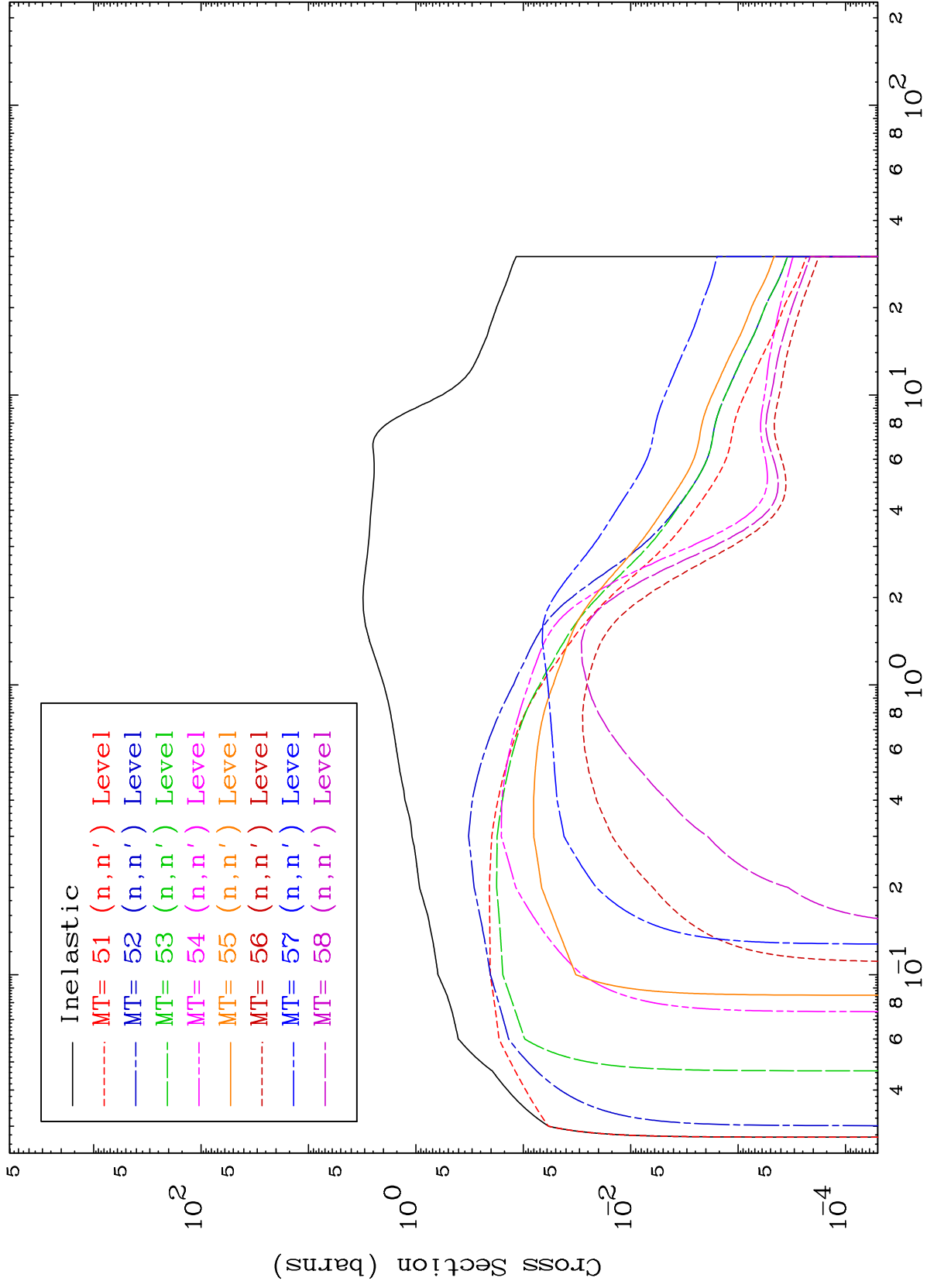


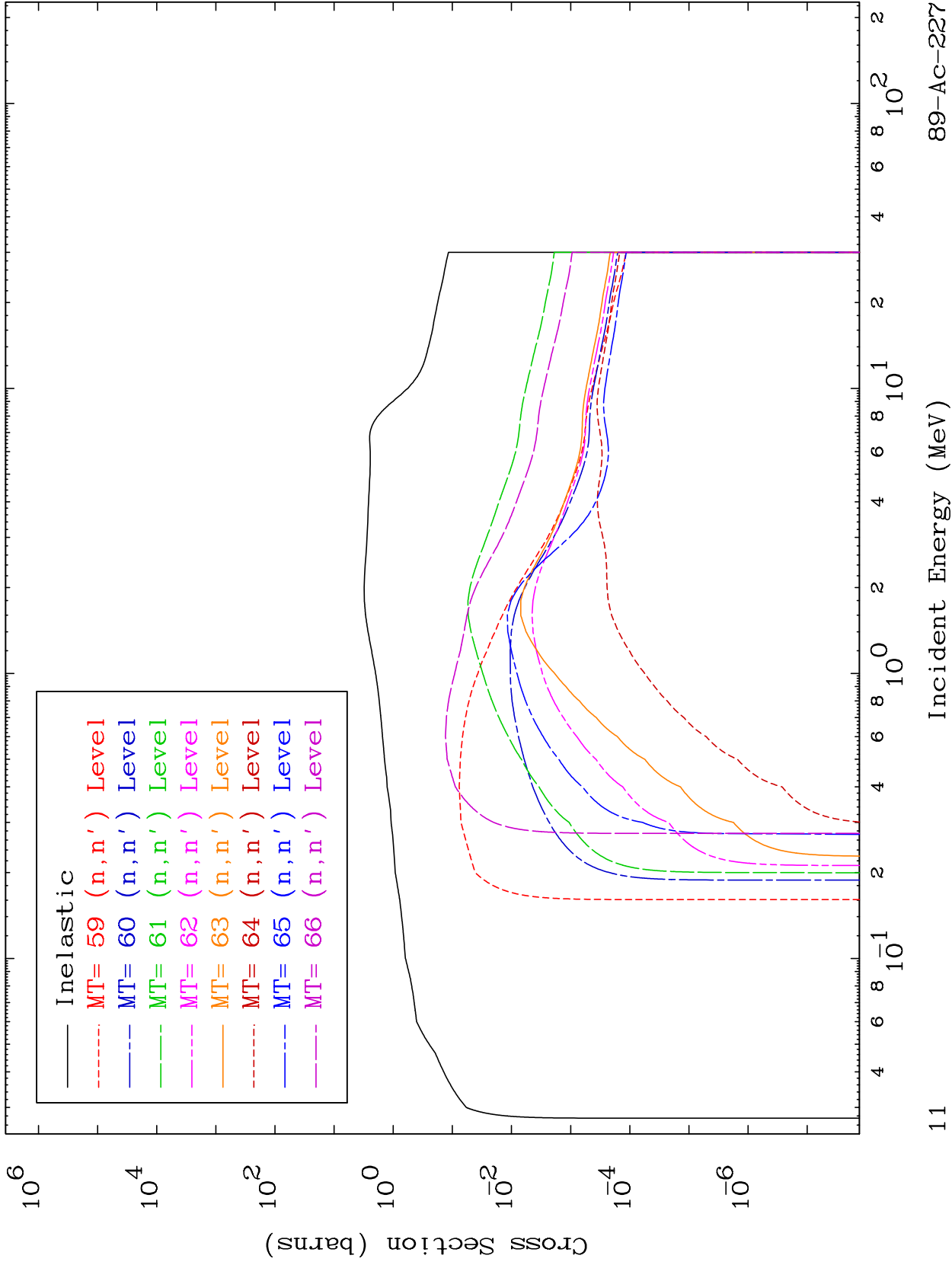
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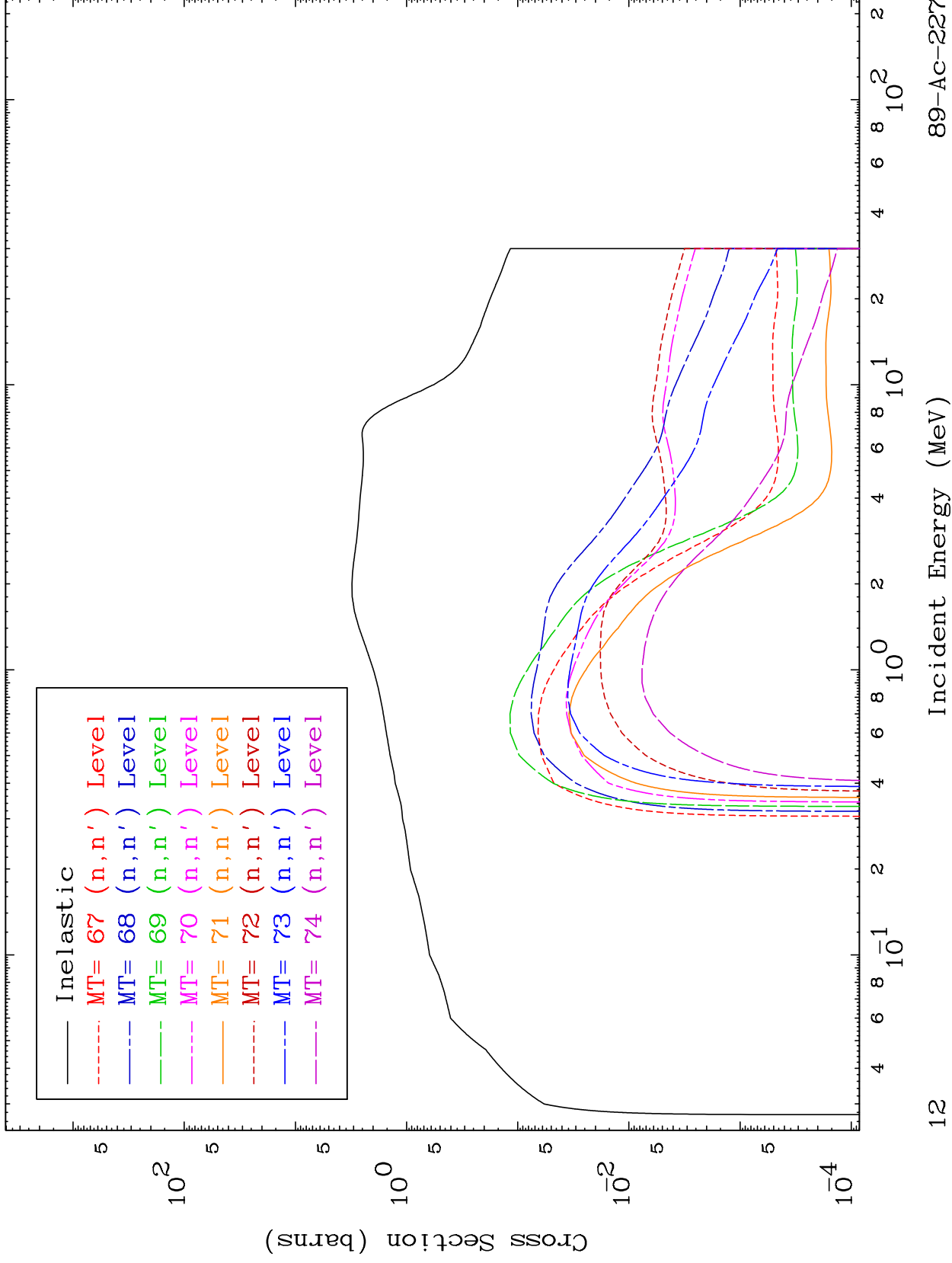
Fission
293 Kelvin Cross Sections

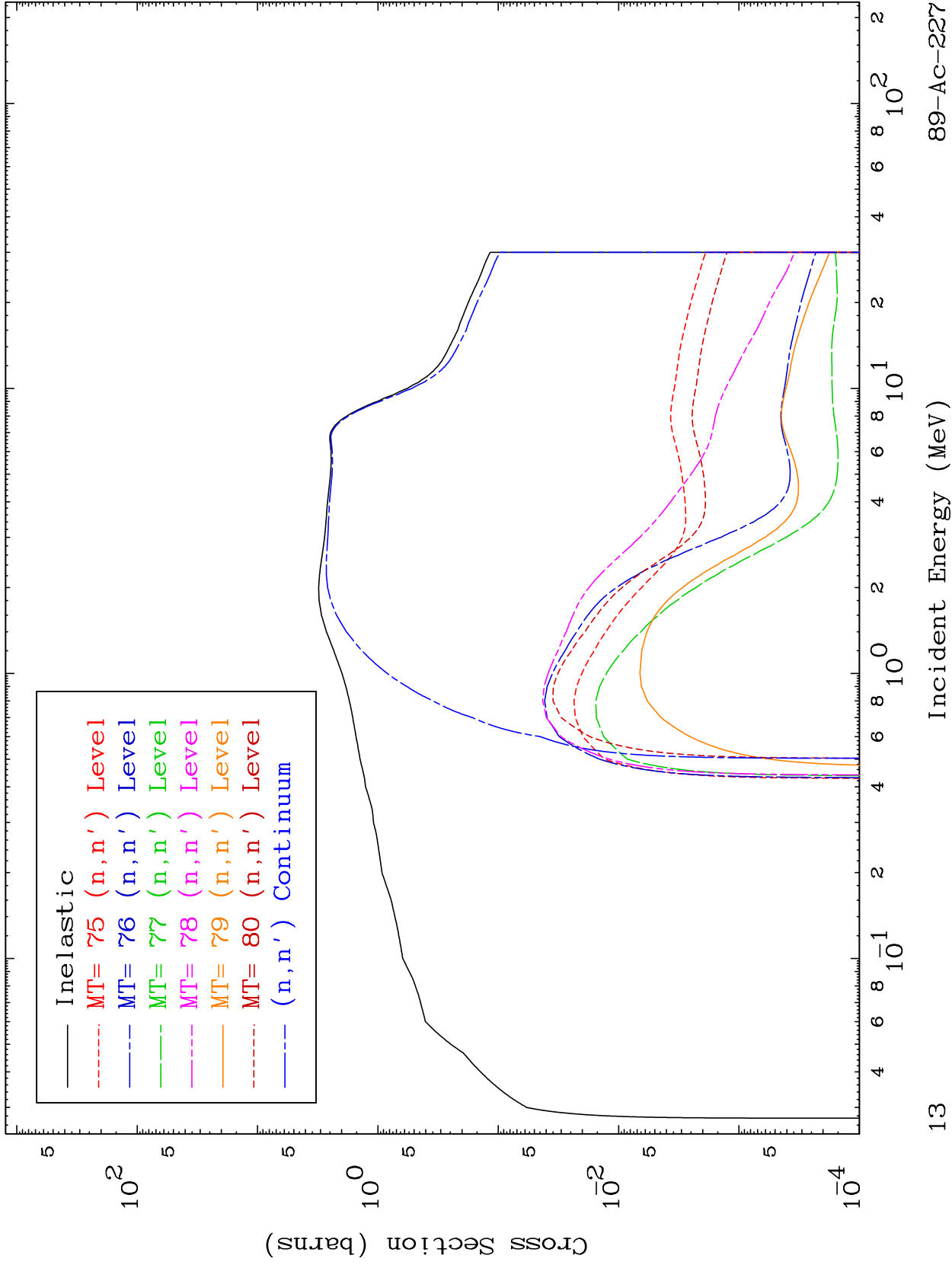
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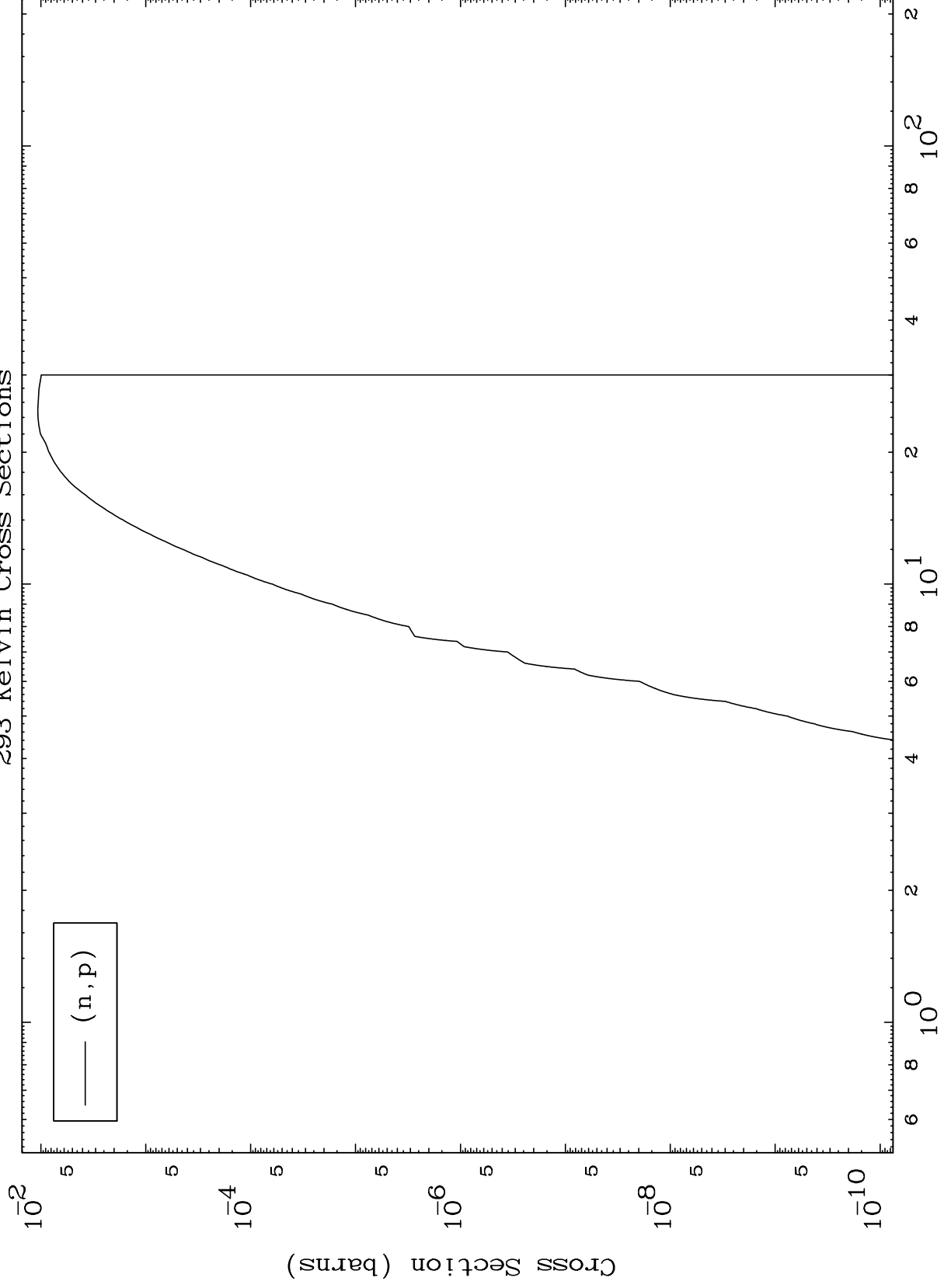




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(n,p) Levels
293 Kelvin Cross Sections

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14

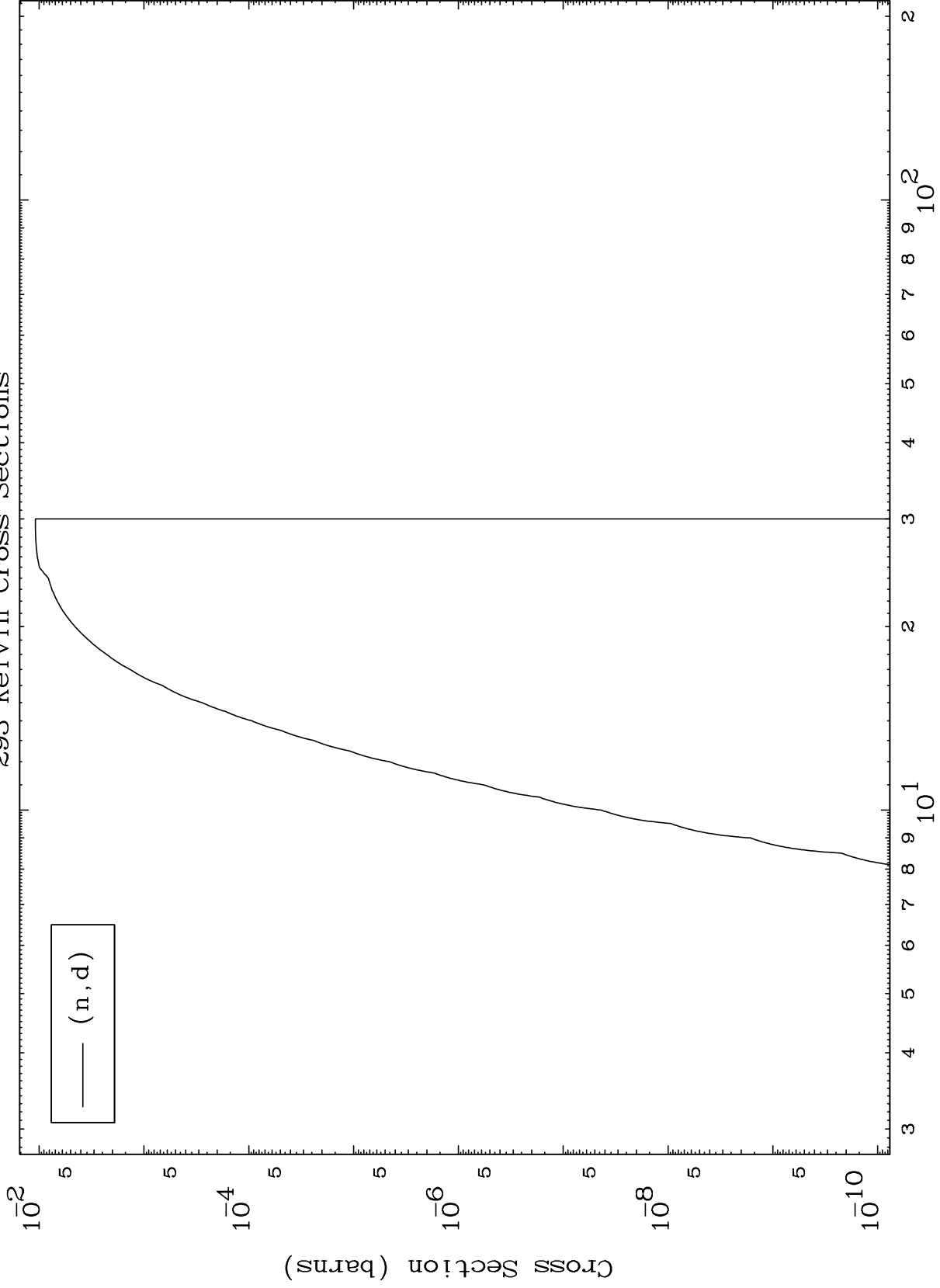
Incident Energy (MeV)

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(n,d) Levels
293 Kelvin Cross Sections

89-Ac-227



15

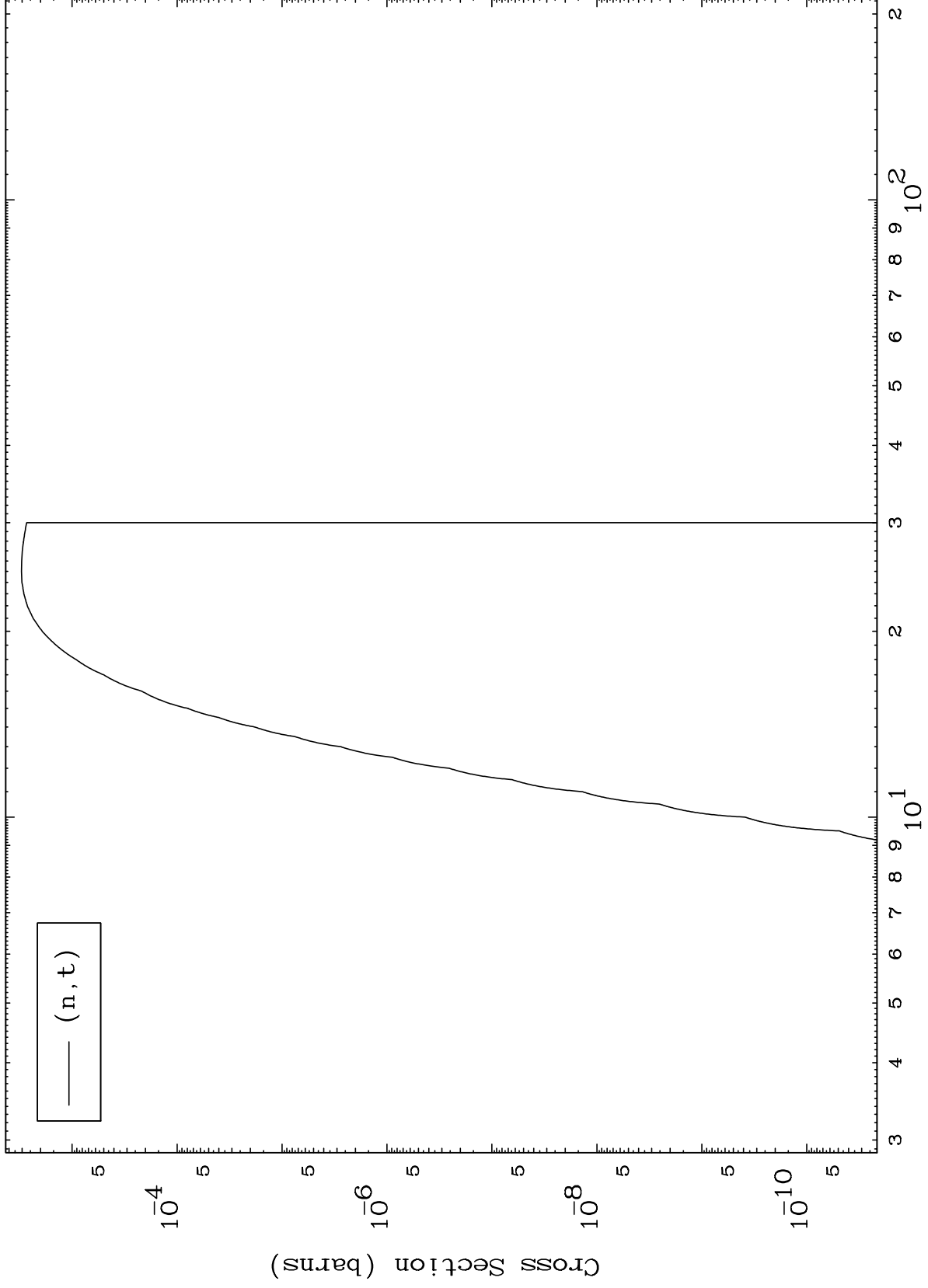
Incident Energy (MeV)

89-Ac-227

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(n,t) Levels
293 Kelvin Cross Sections

89-Ac-227



16

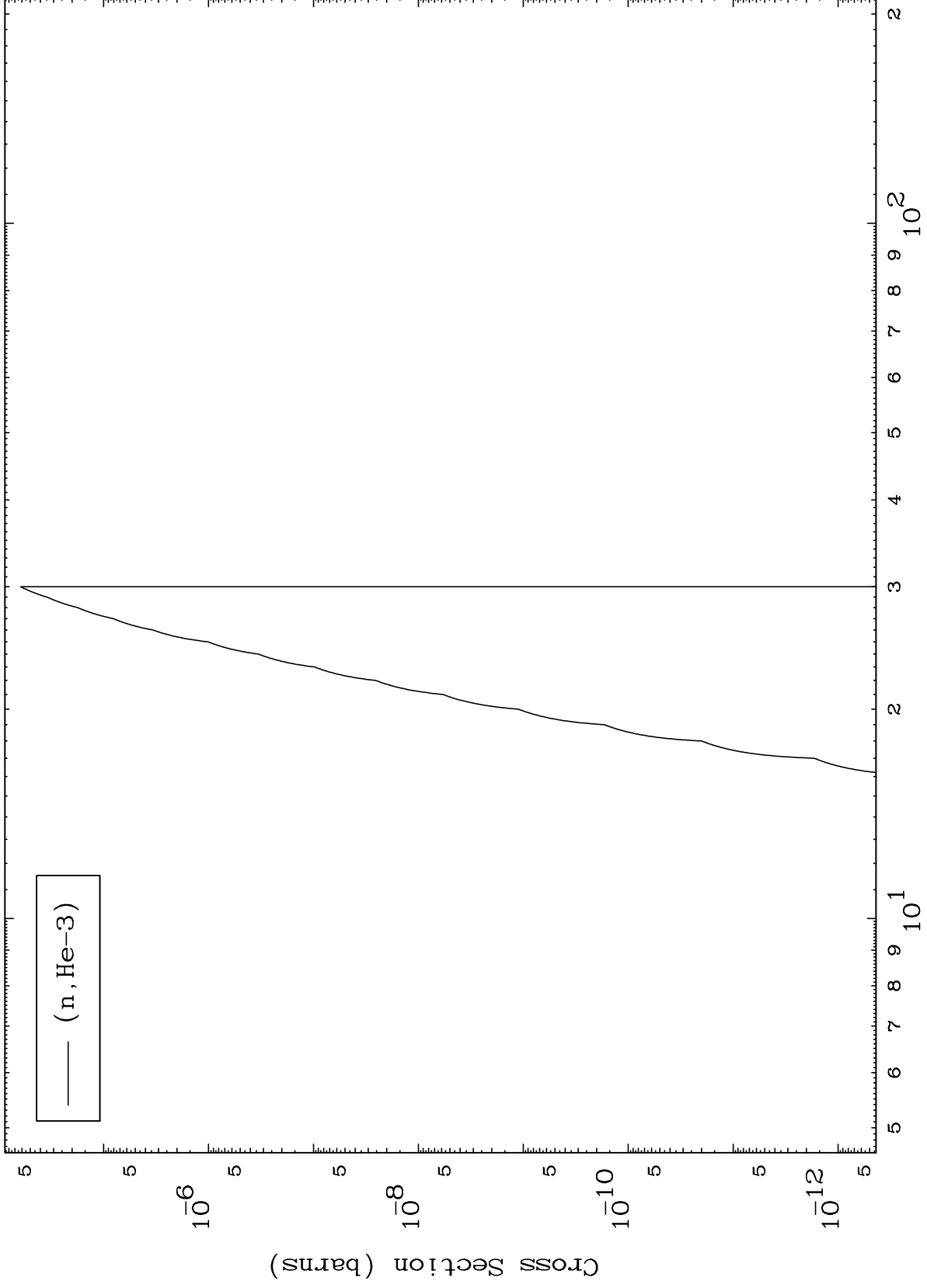
Incident Energy (MeV)

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(n,He3) Levels
293 Kelvin Cross Sections

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17

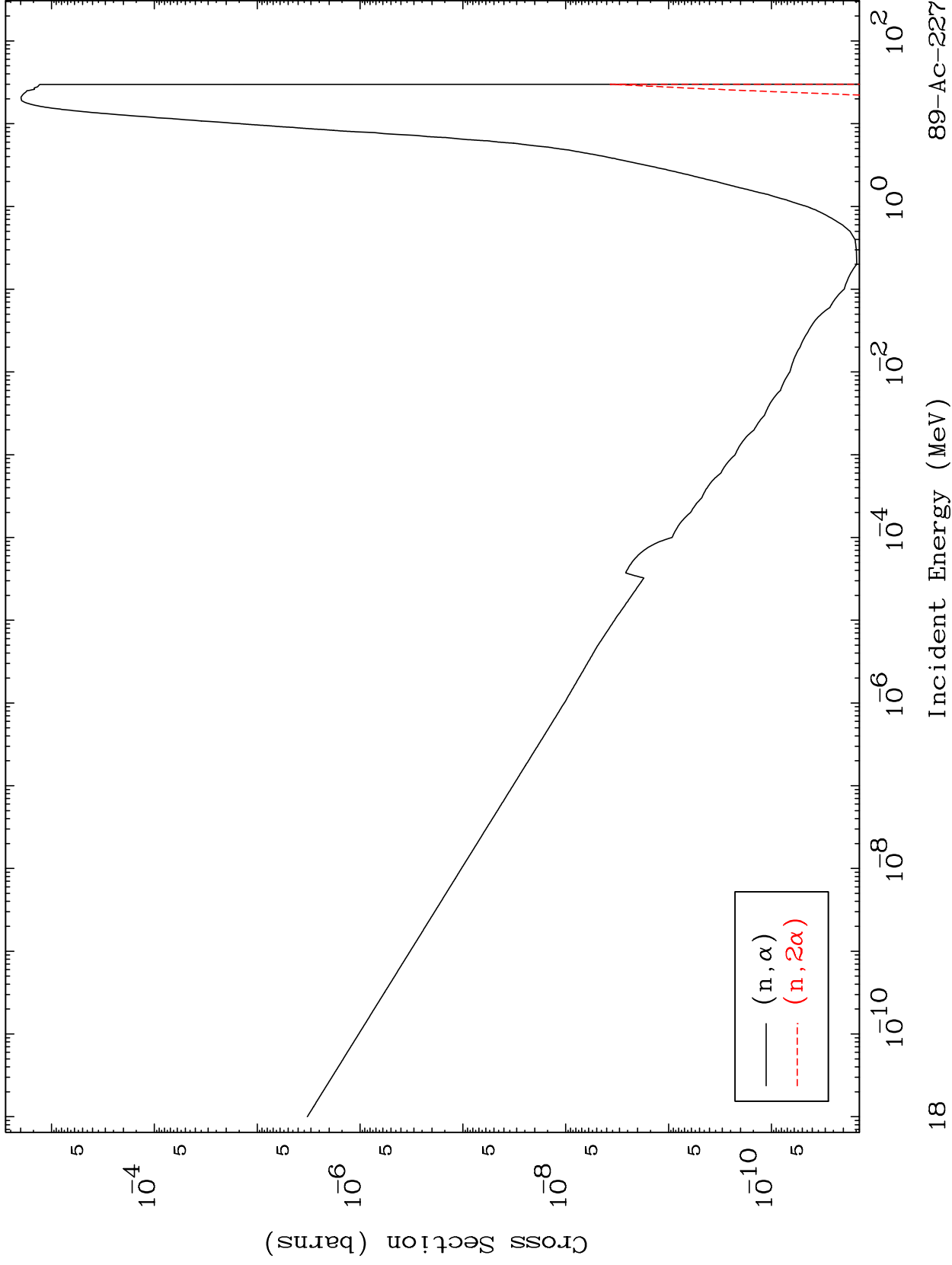
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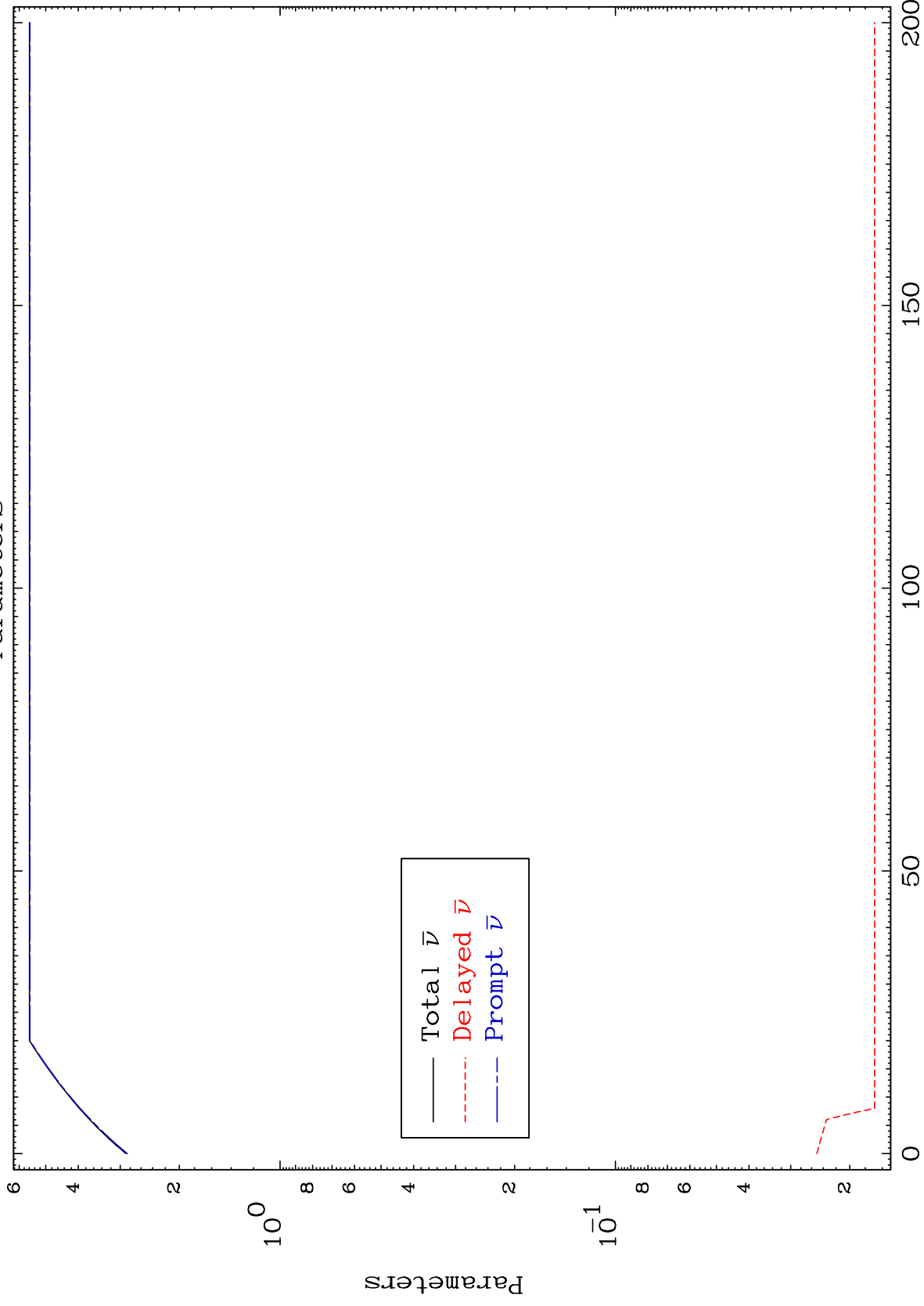
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(n,α) Levels
293 Kelvin Cross Sections

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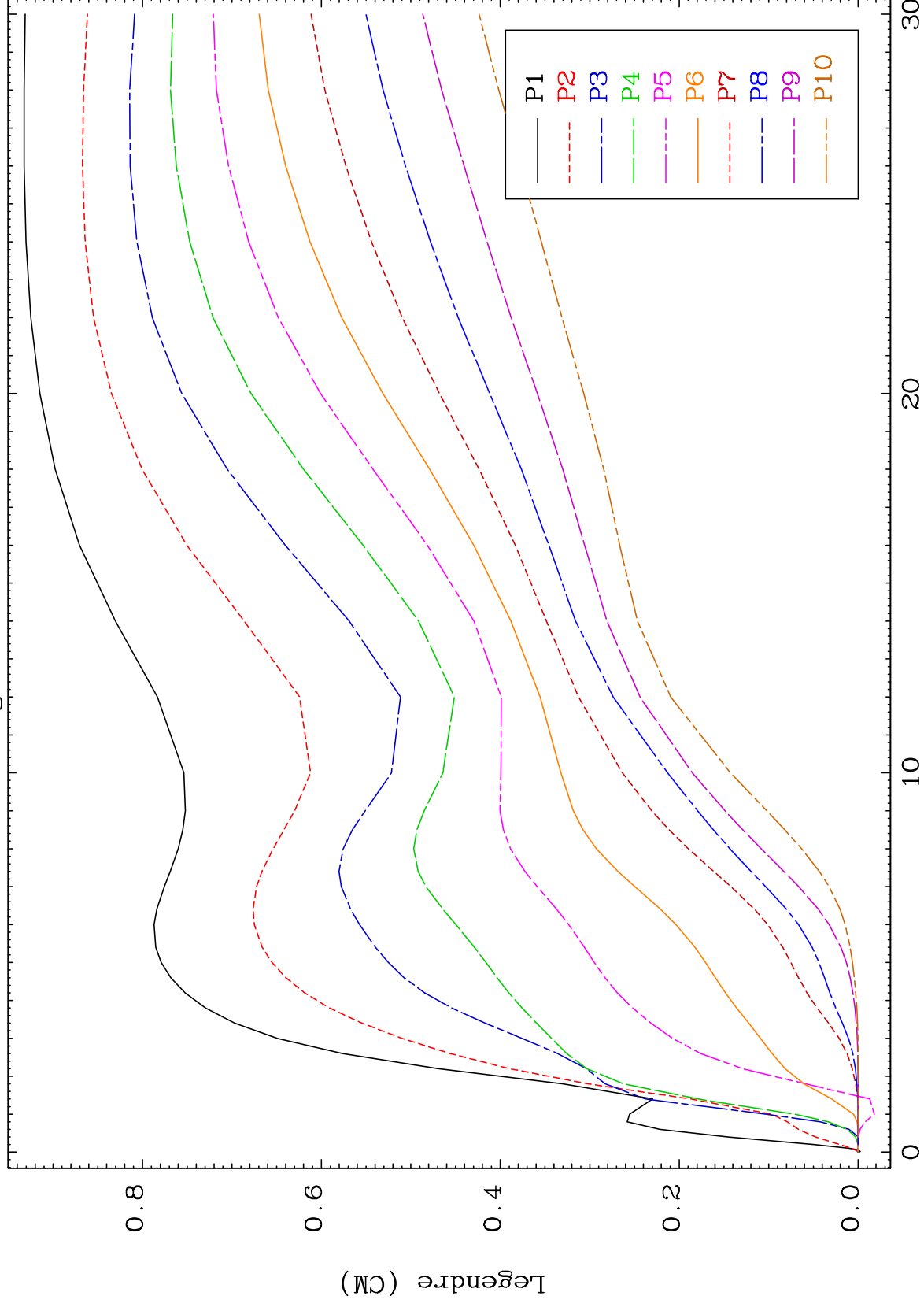




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Elastic Legendre Coefficients

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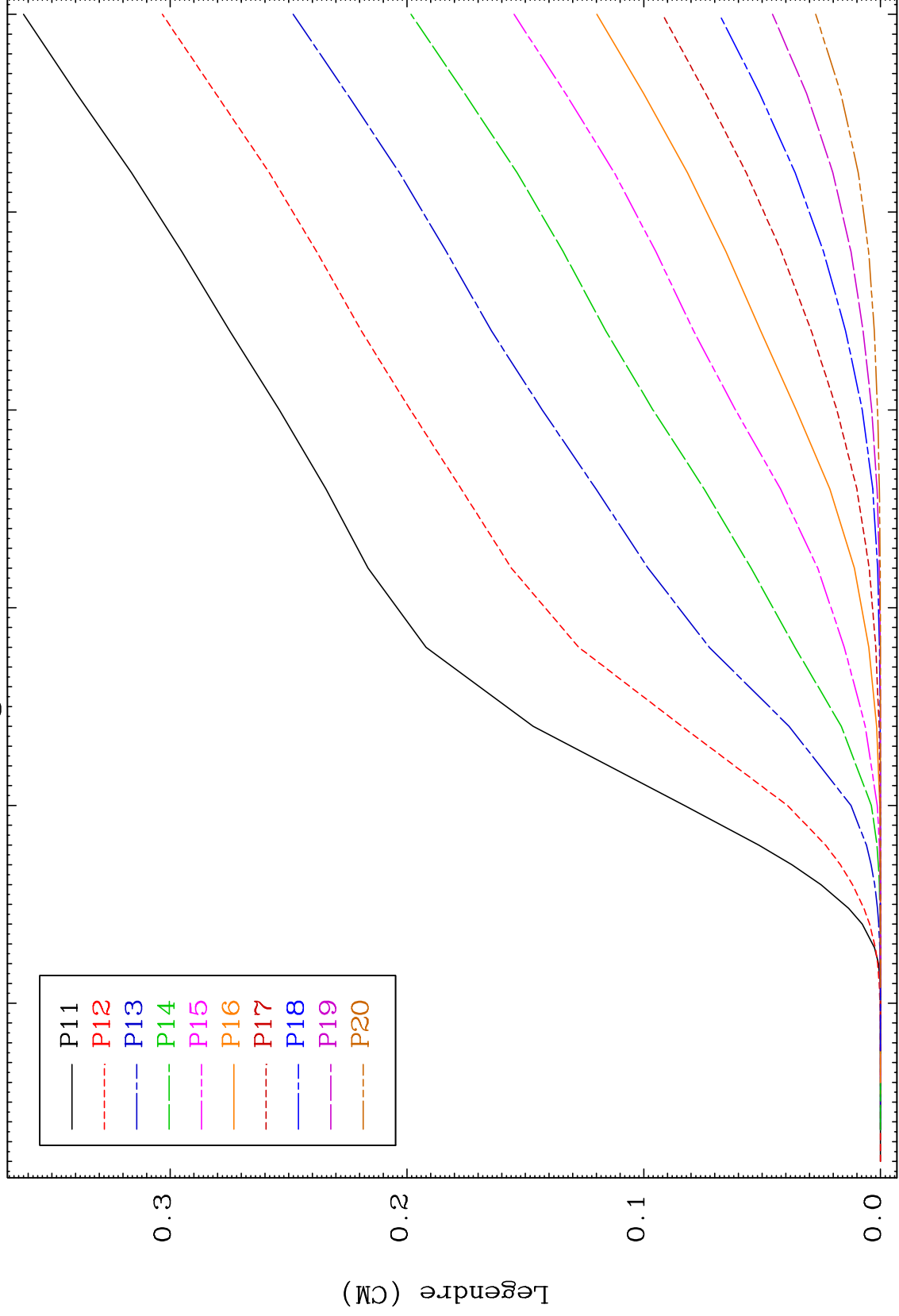
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Elastic Legendre Coefficients

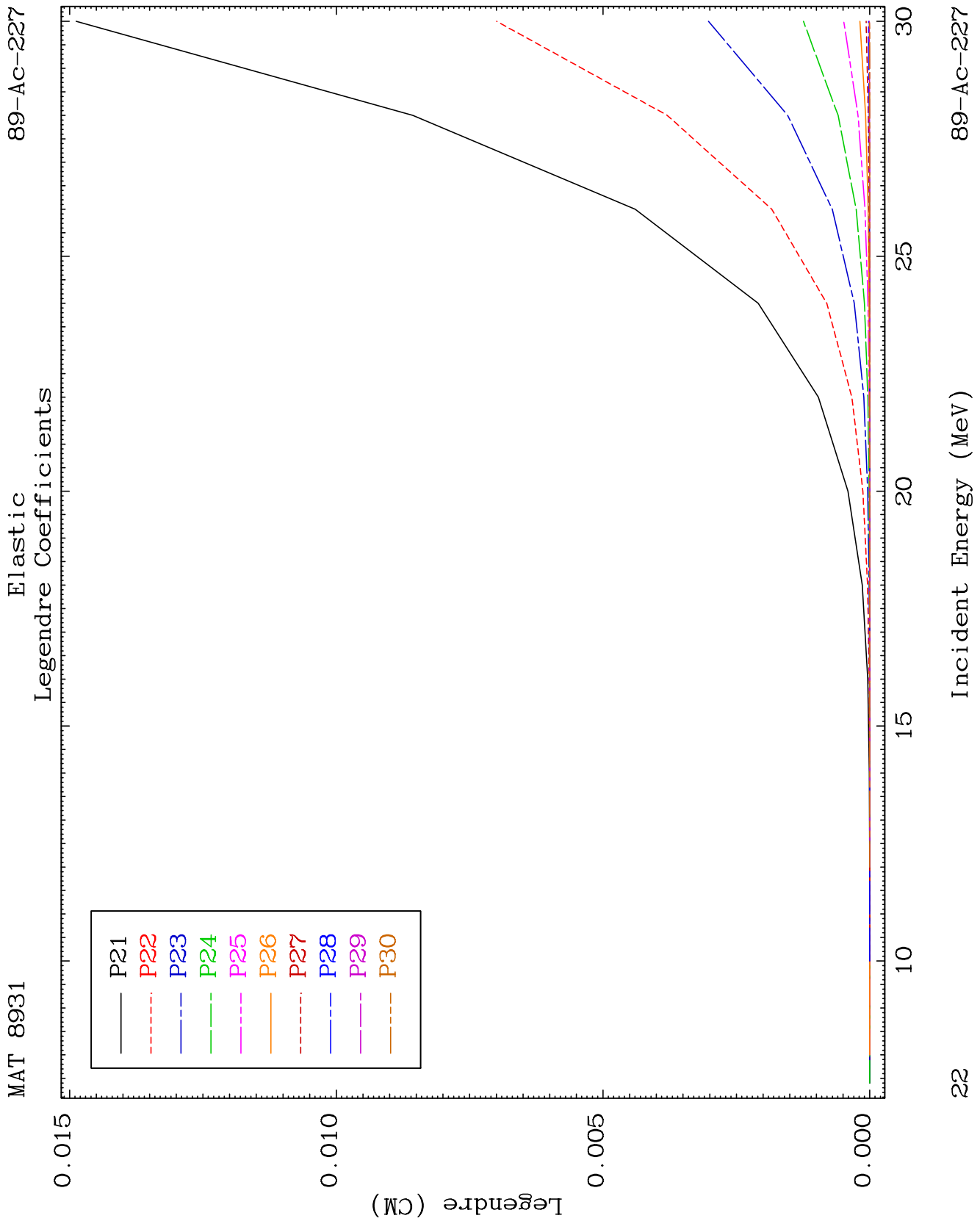
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21

Incident Energy (MeV)

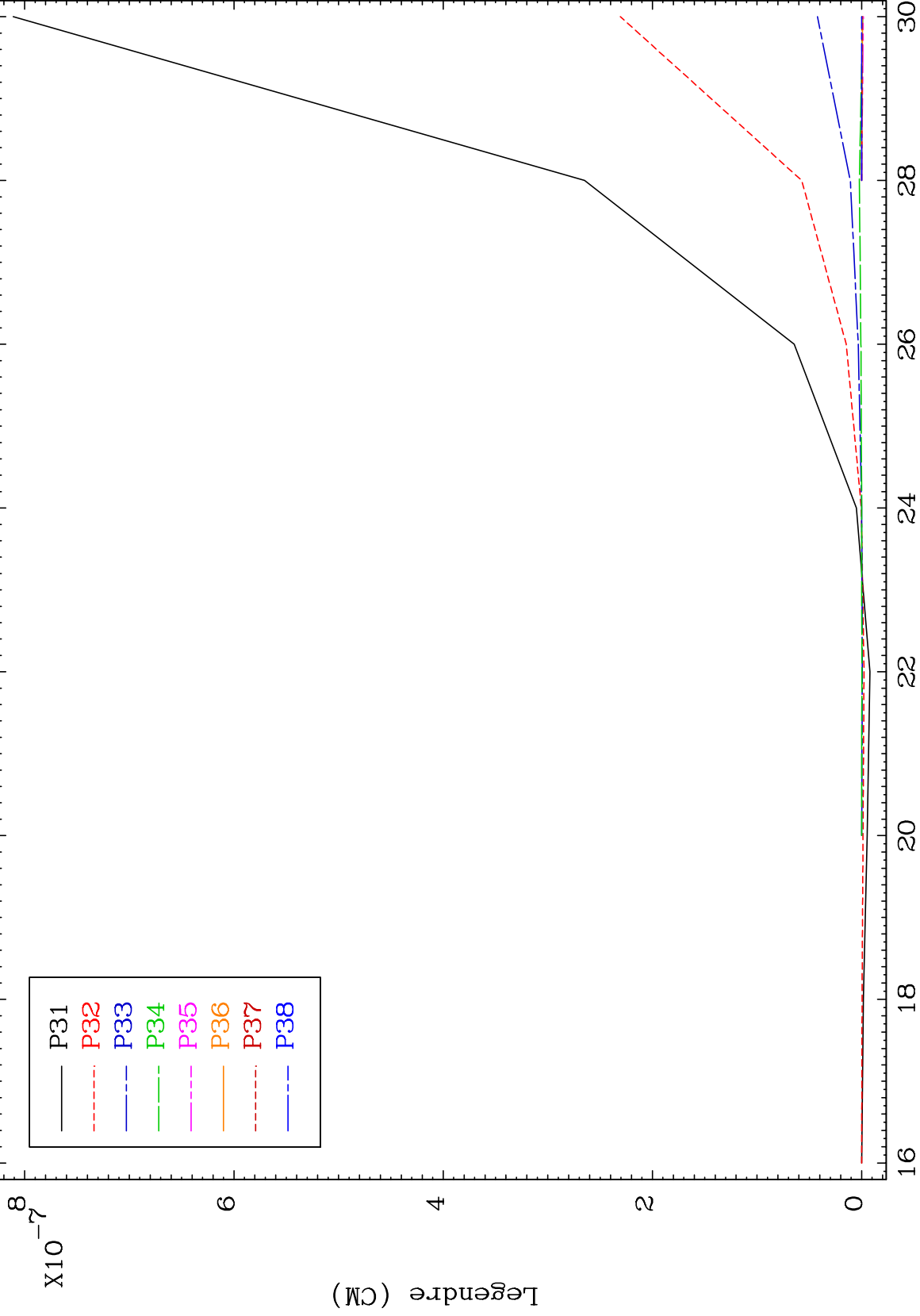
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Elastic Legendre Coefficients

89-Ac-227

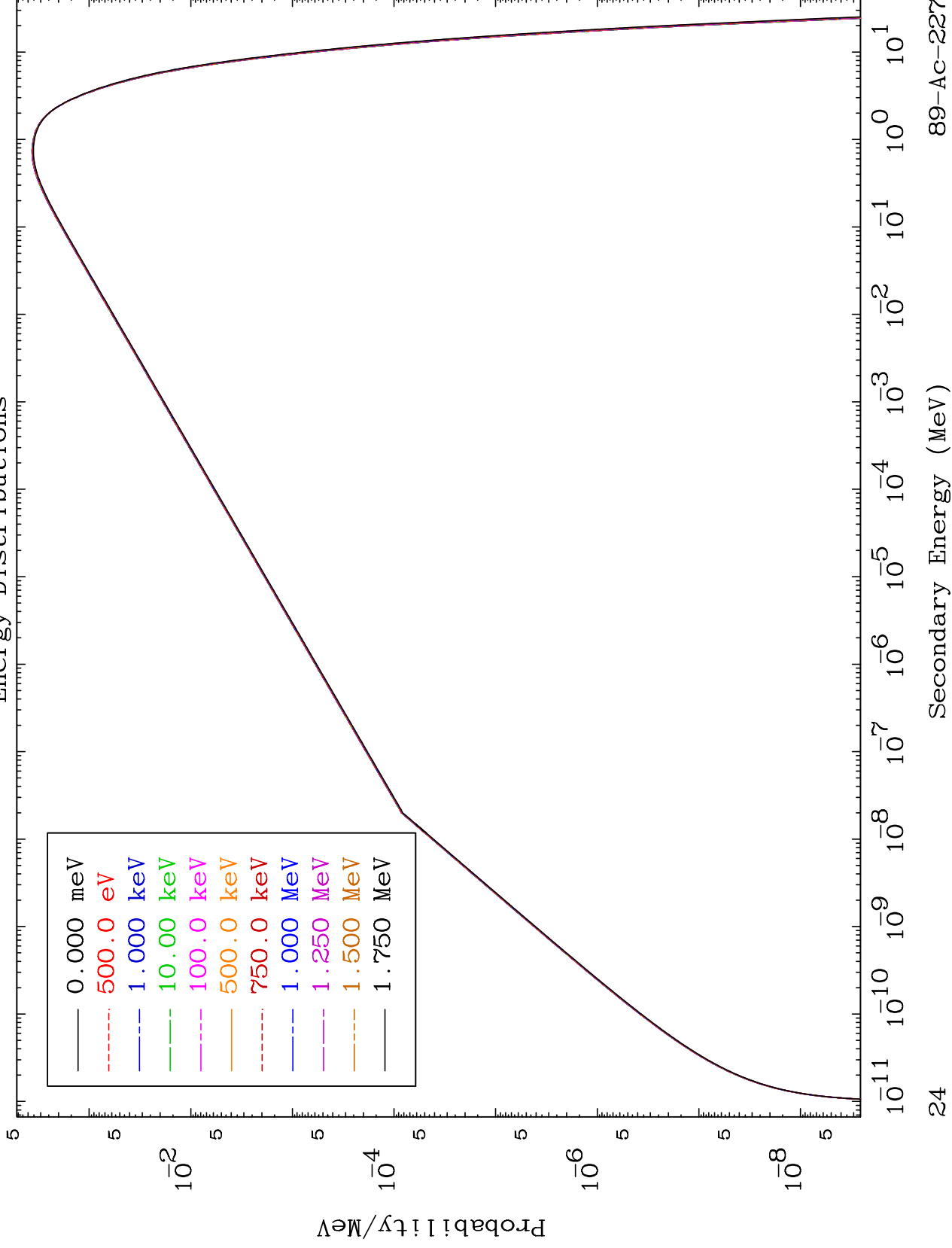


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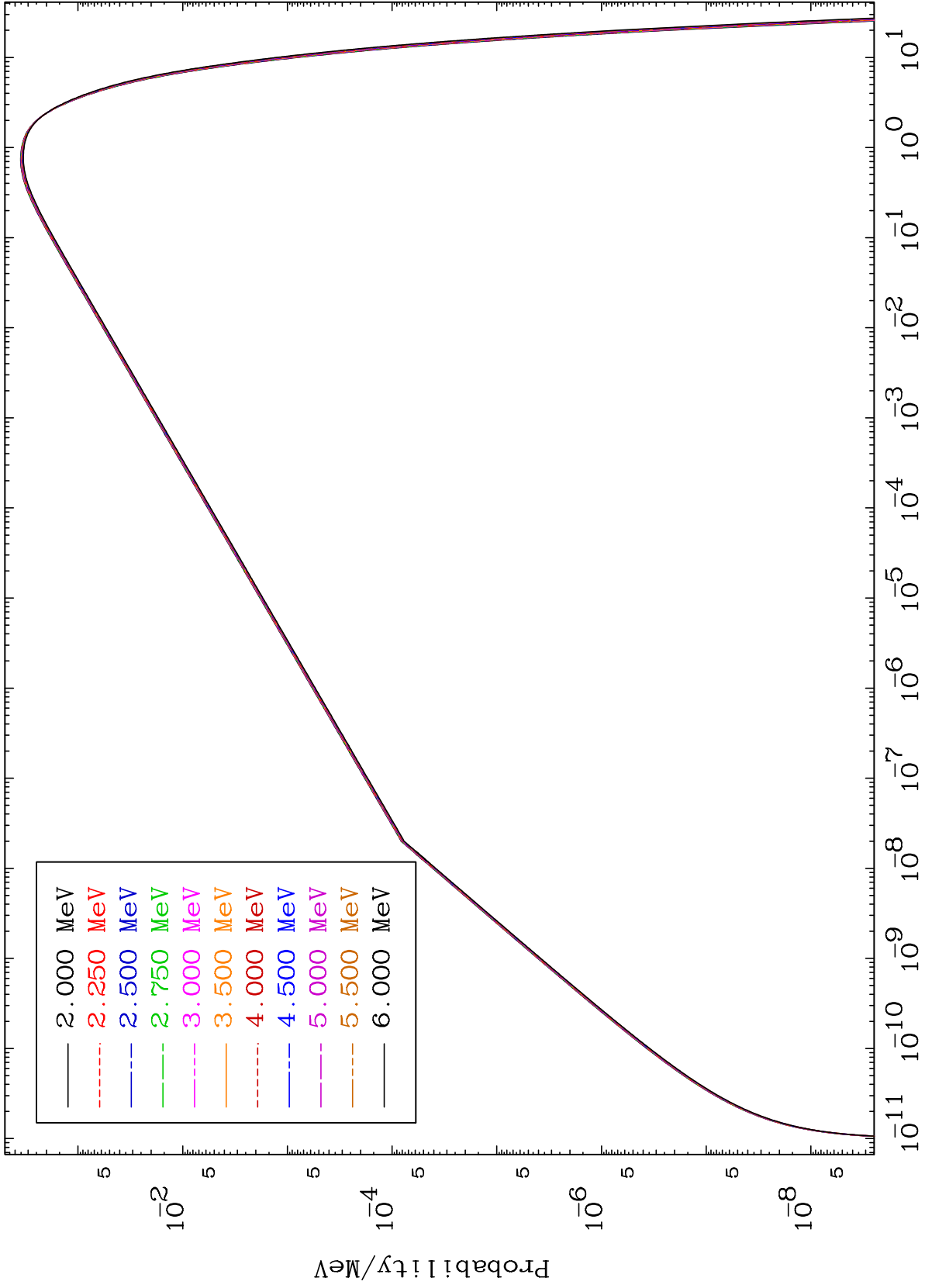
Incident Energy (MeV)

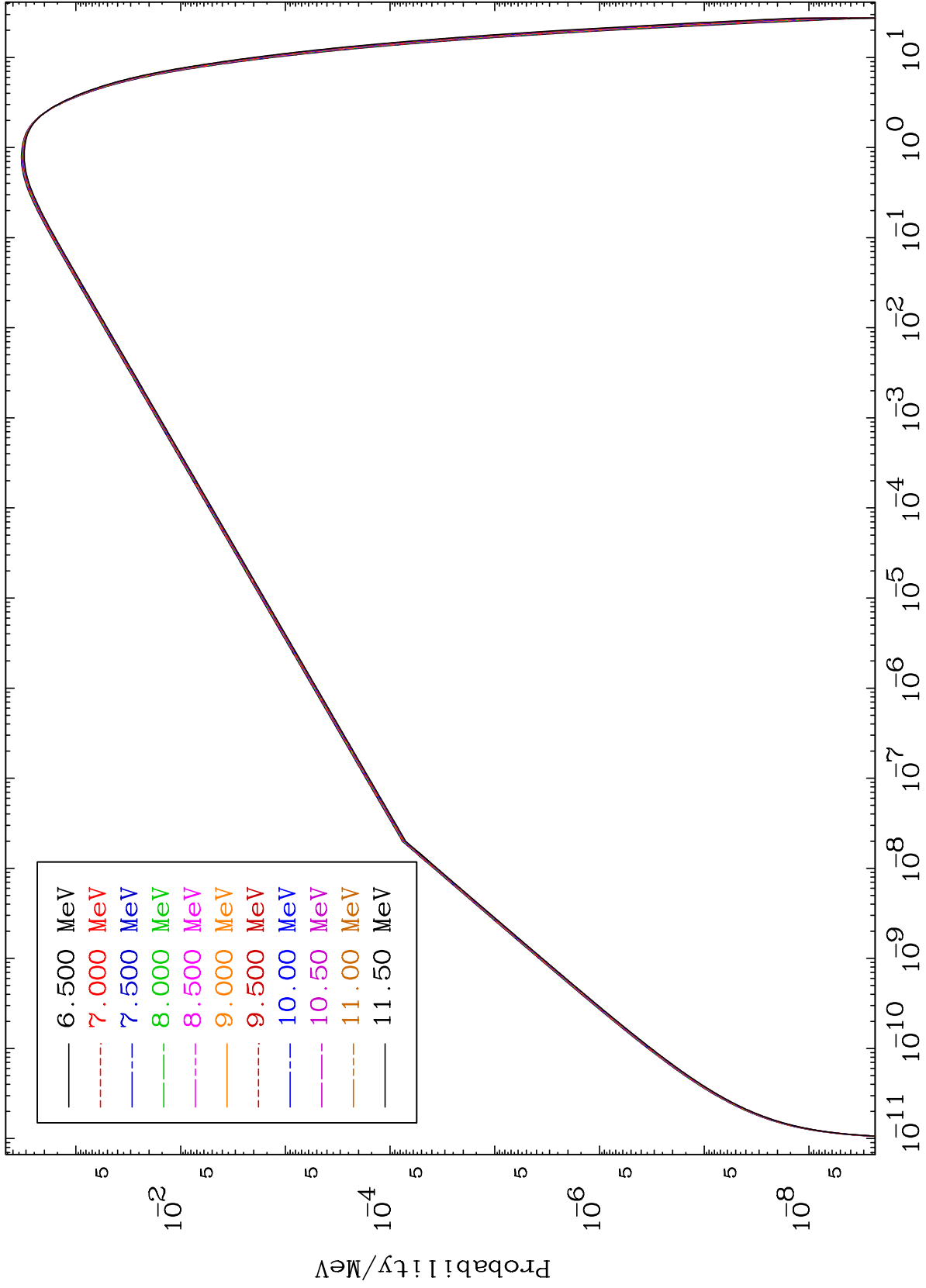
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Fission Energy Distributions

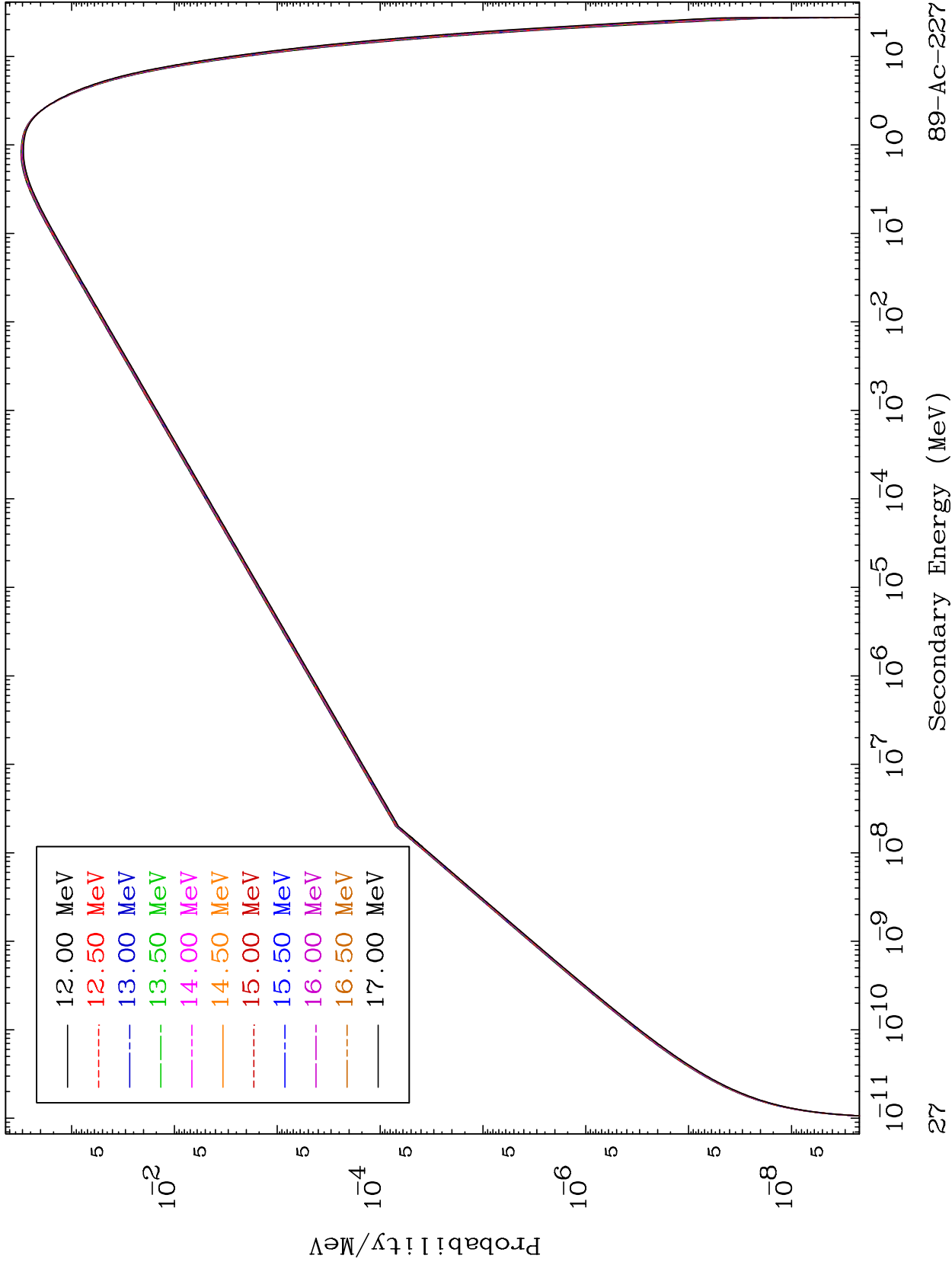


Fission Energy Distributions

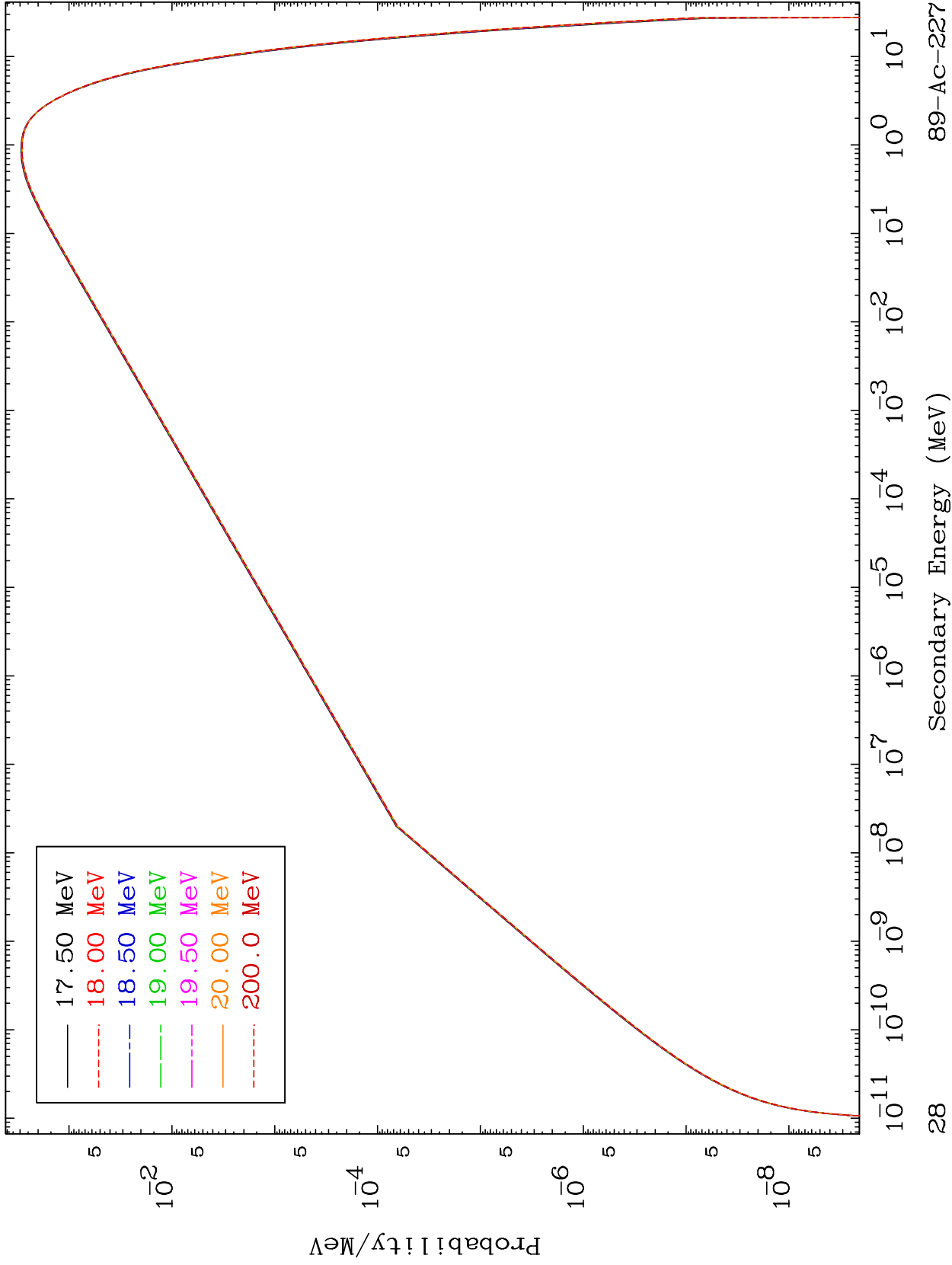




Fission Energy Distributions



Fission Energy Distributions

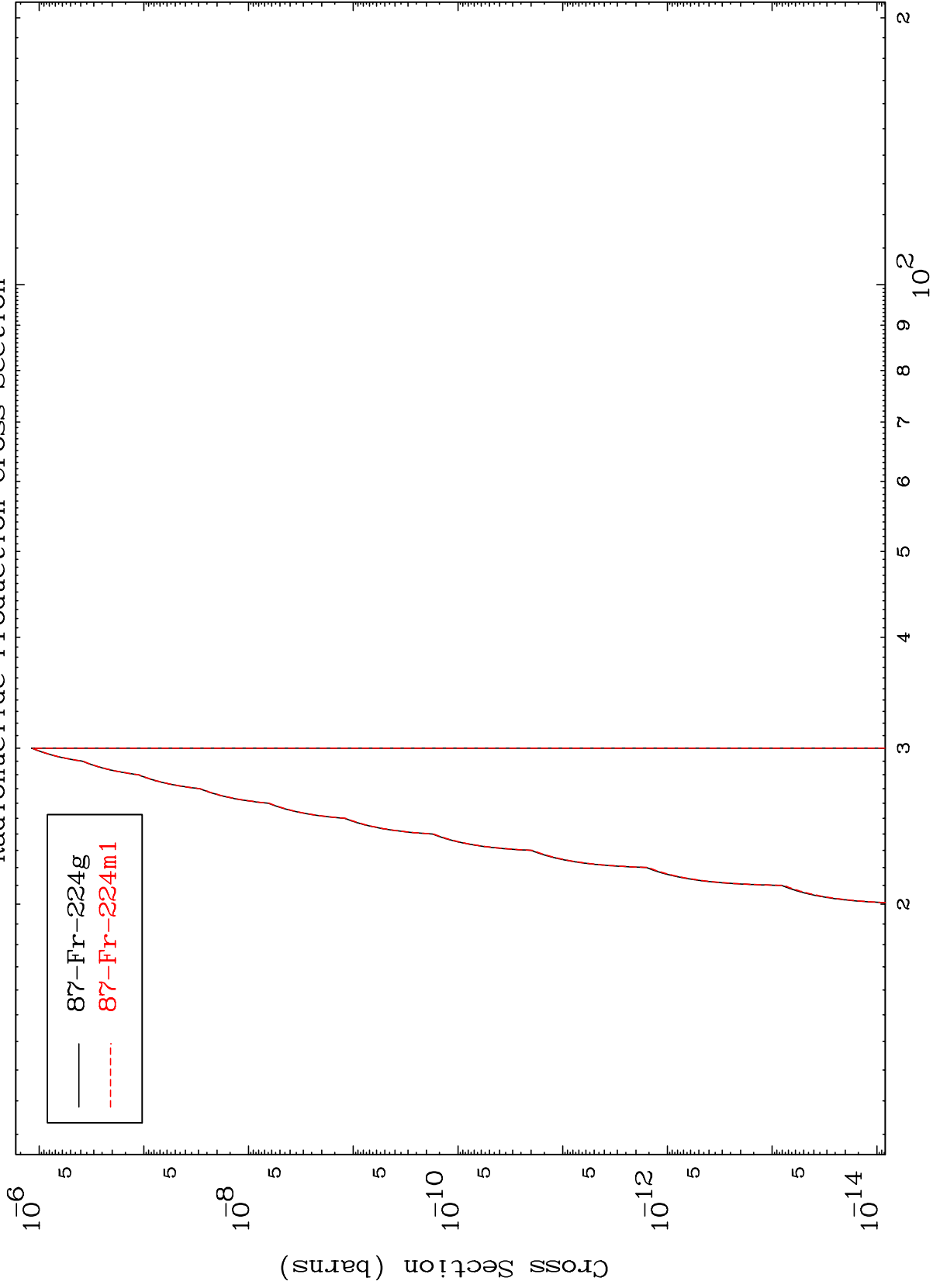


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(n,n') He-3

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Radionuclide Production Cross Section



Incident Energy (MeV)

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29

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89-Ac-227

(n, α)
Radionuclide Production Cross Section

