

Program EVALPLOT  
(Version 2021-1)

by

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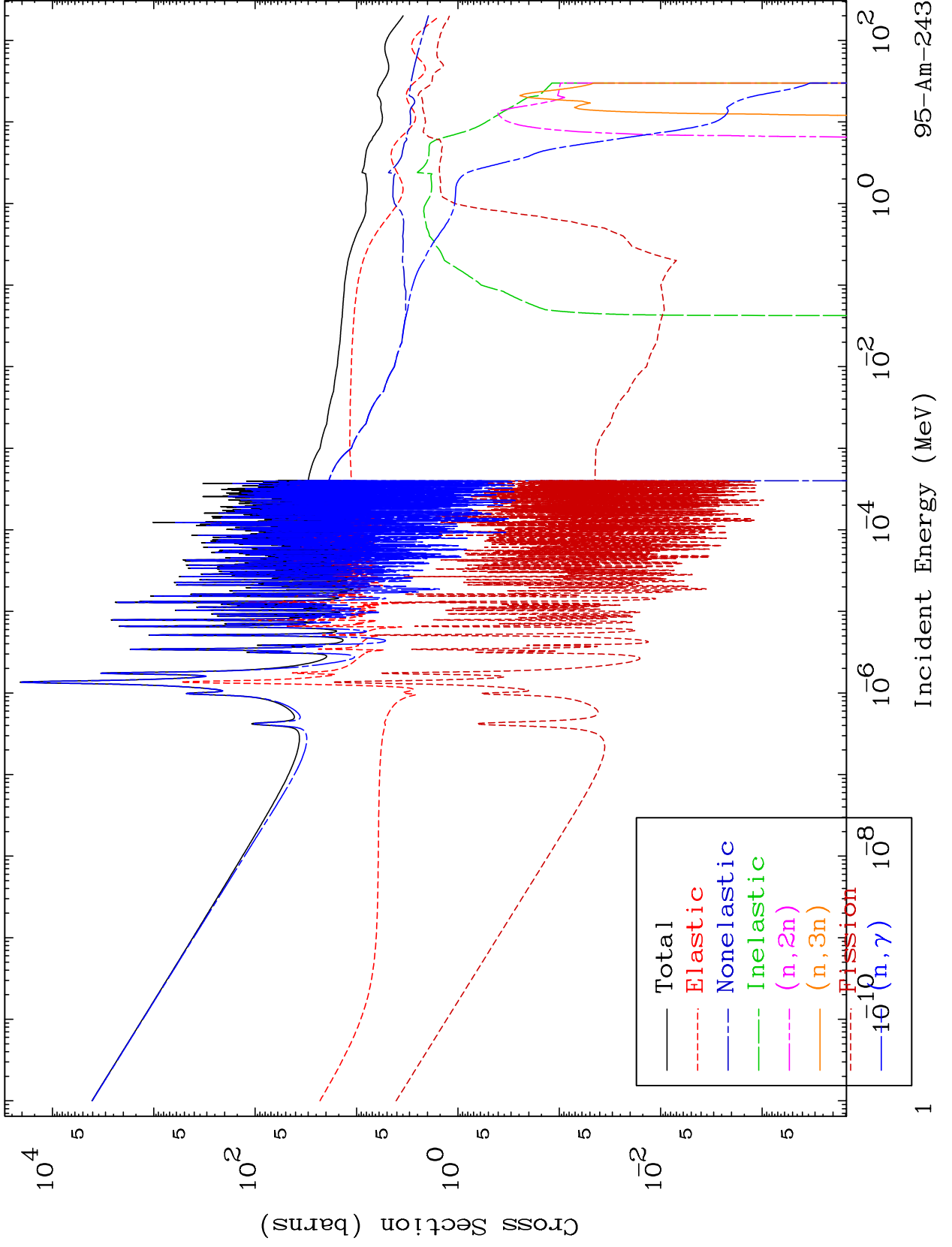
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9549

Neutron Major  
293 Kelvin Cross Sections

95-Am-243



95-Am-243

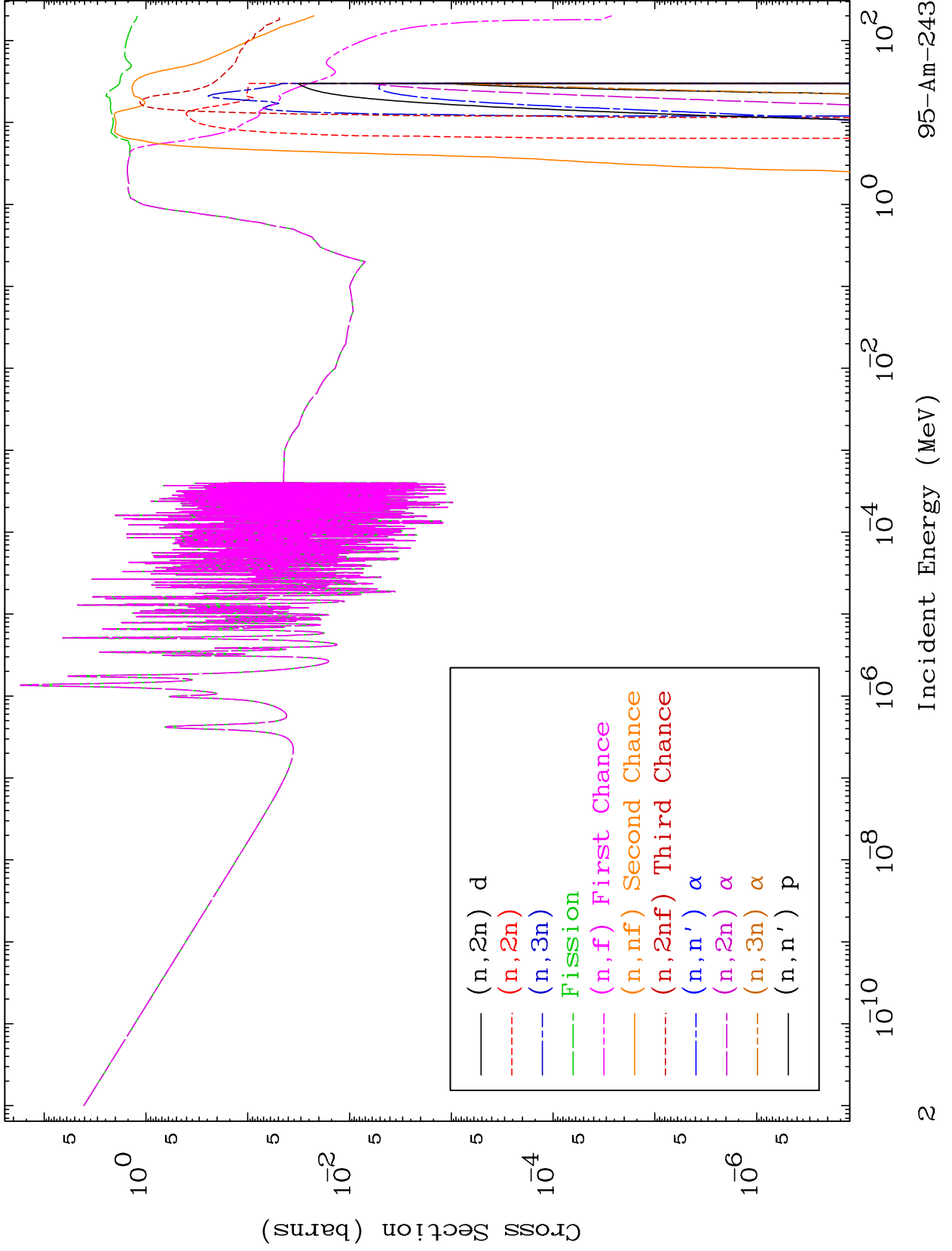
Incident Energy (MeV)

1

MAT 9549

Neutron Absorption  
293 Kelvin Cross Sections

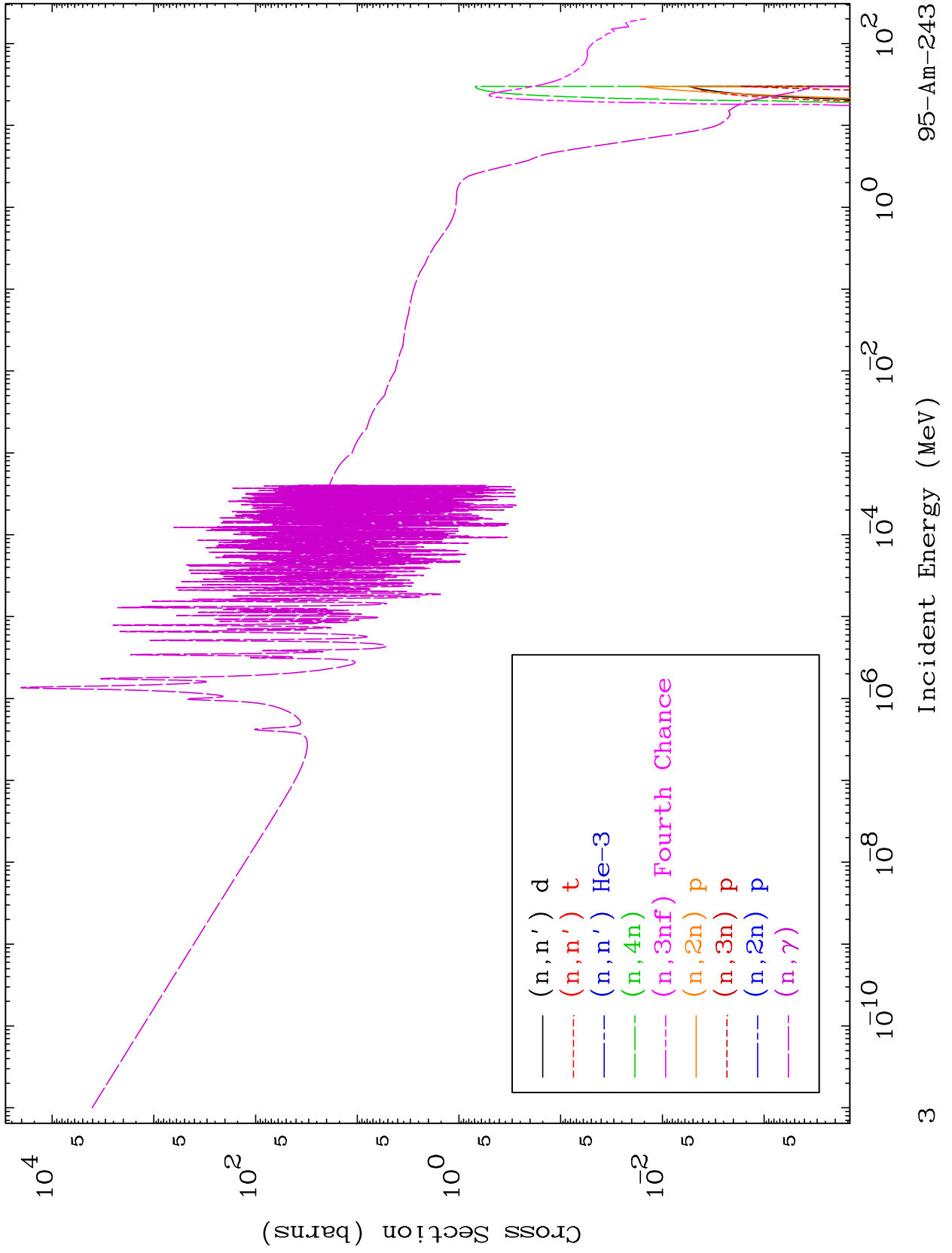
95-Am-243



MAT 9549

Neutron Absorption  
293 Kelvin Cross Sections

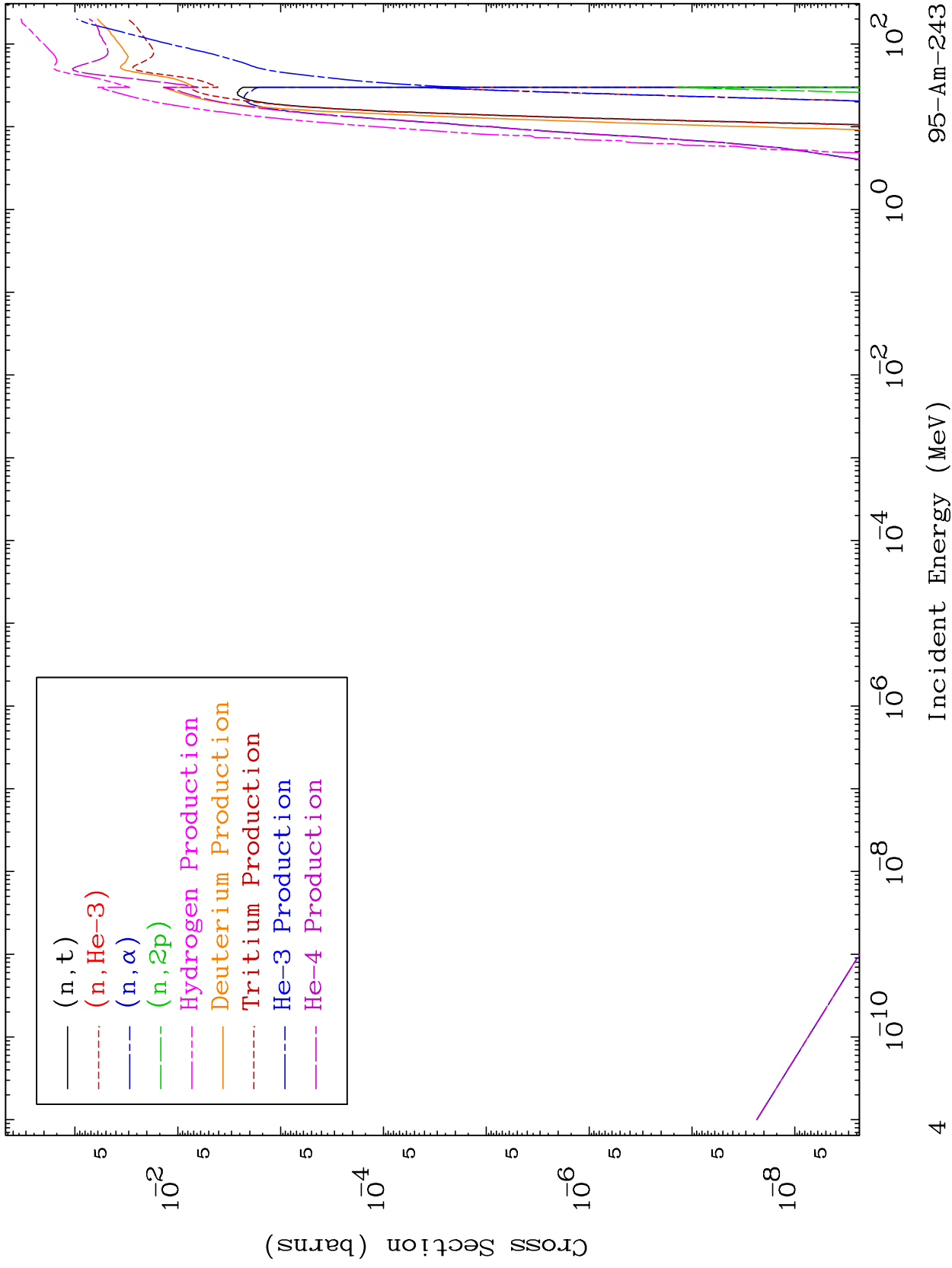
95-Am-243



MAT 9549

Neutron Absorption  
293 Kelvin Cross Sections

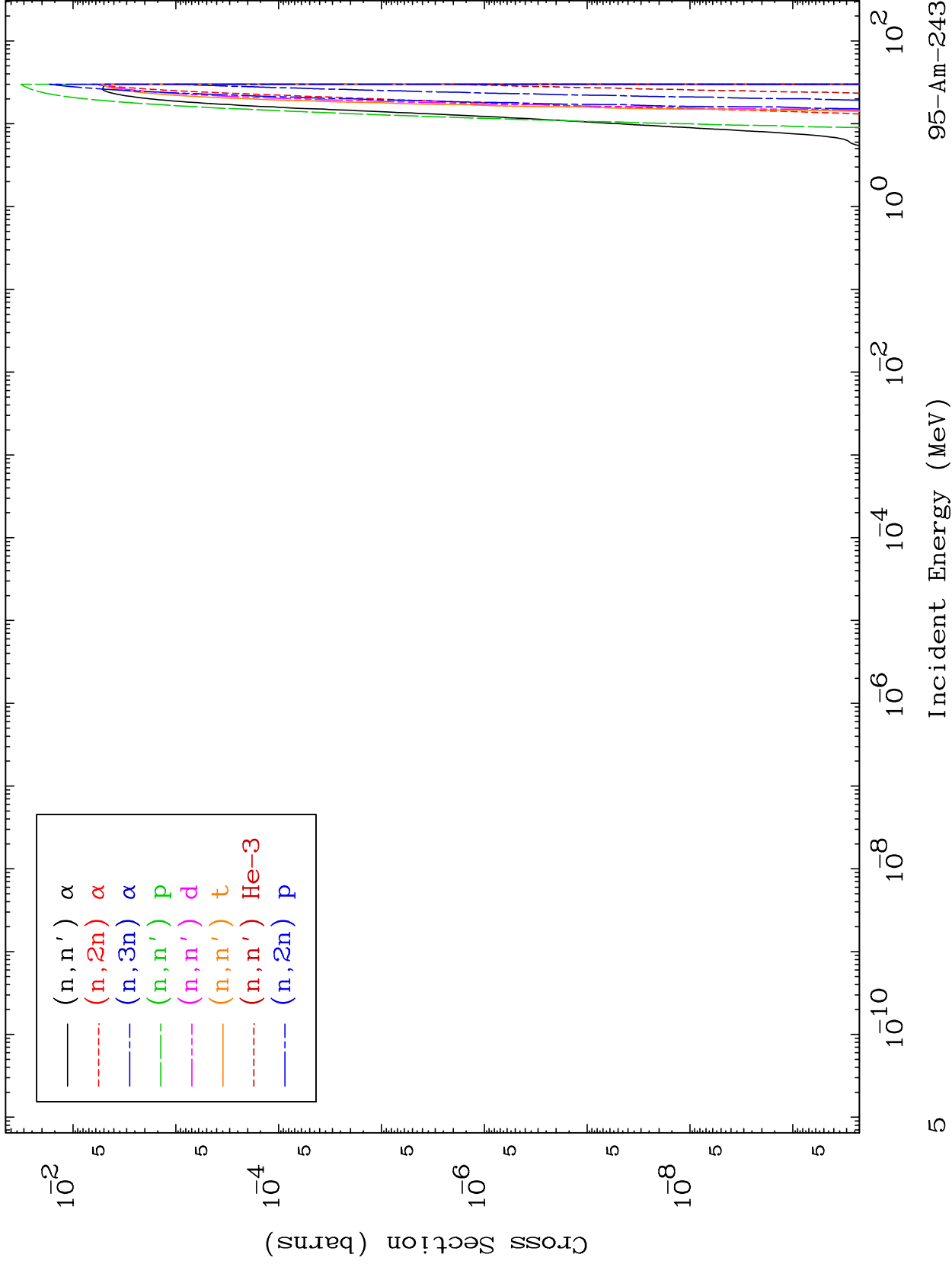
95-Am-243



MAT 9549

Charged Particle  
293 Kelvin Cross Sections

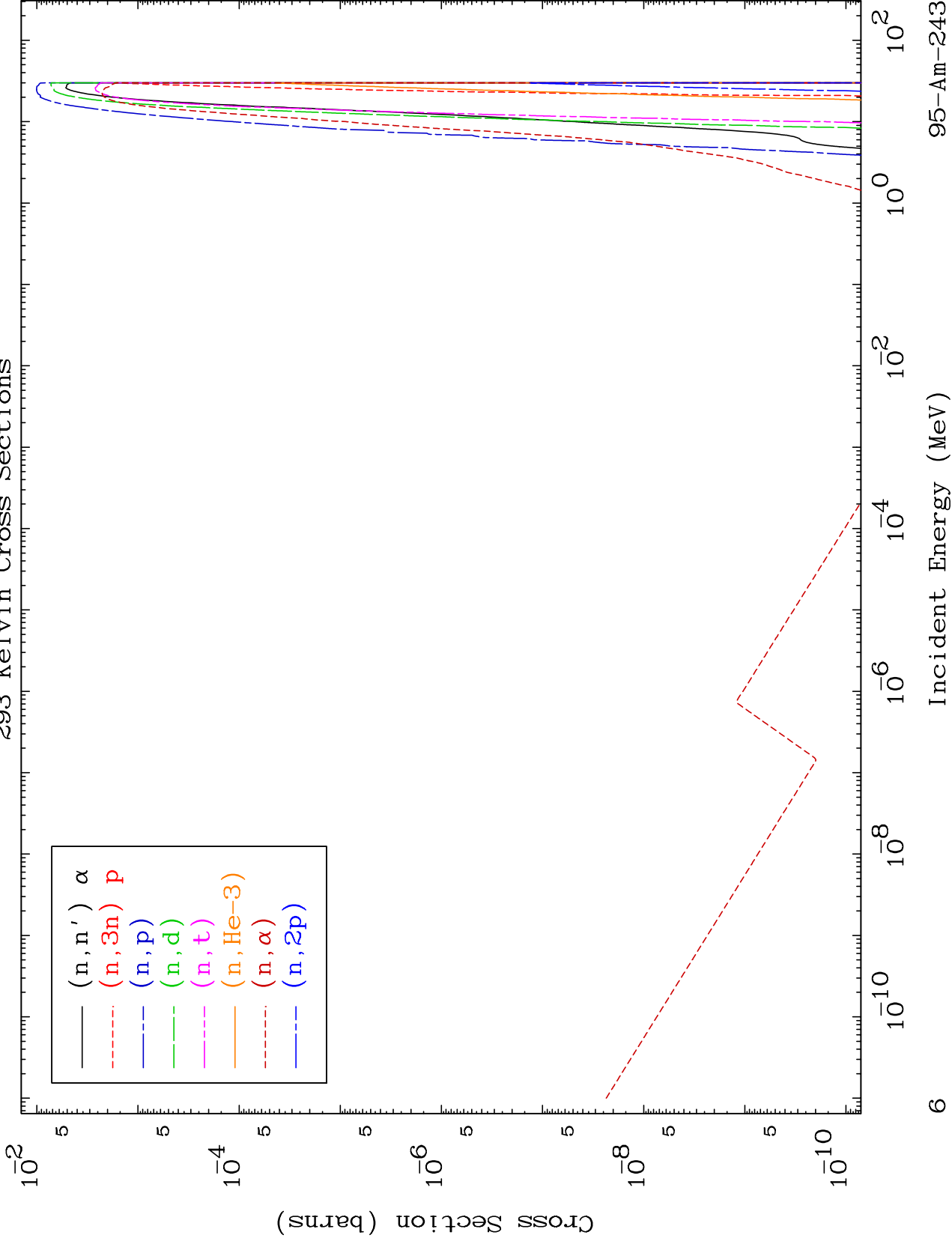
95-Am-243



MAT 9549

Charged Particle  
293 Kelvin Cross Sections

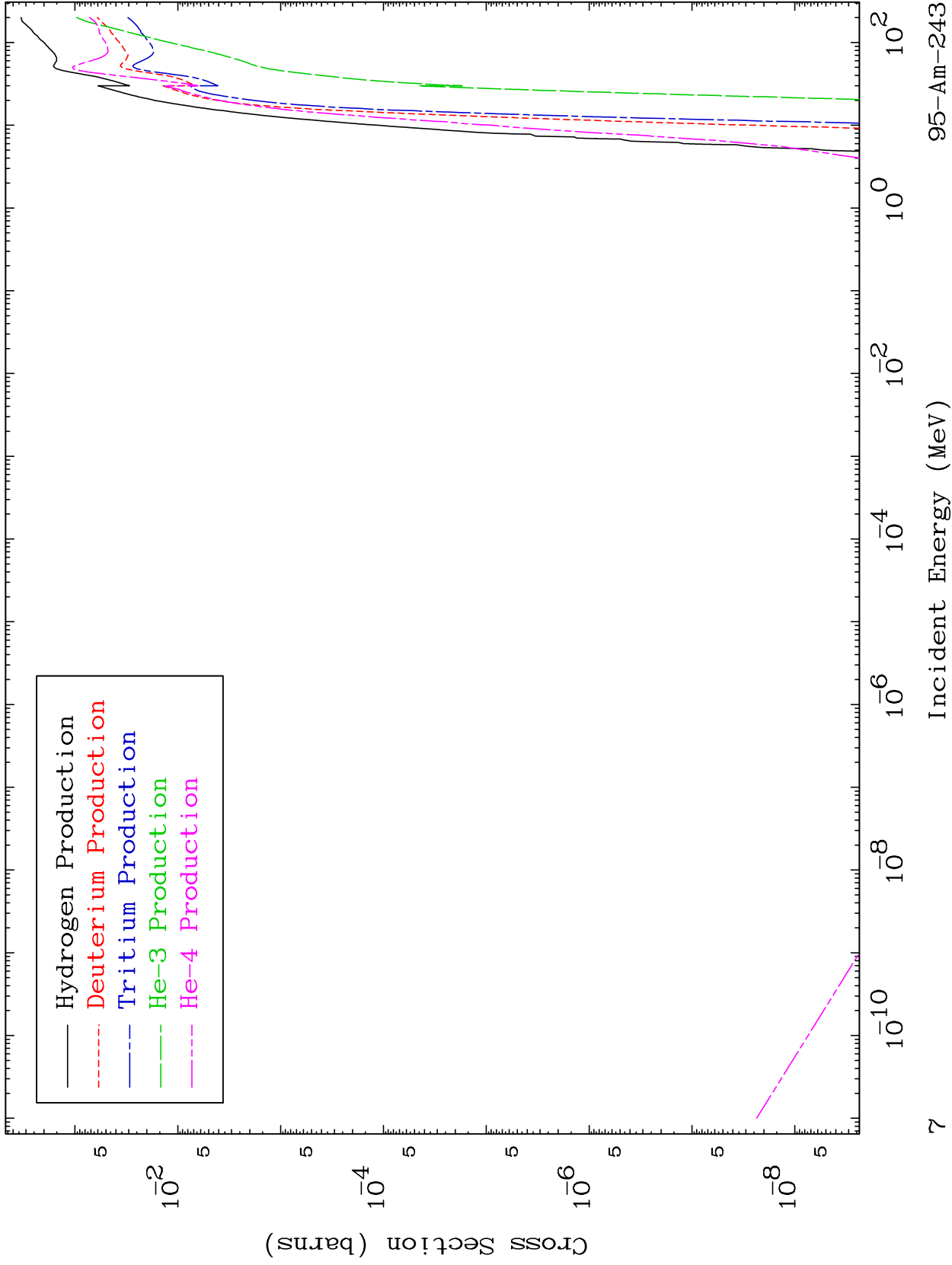
95-Am-243



MAT 9549

Particle Production  
293 Kelvin Cross Sections

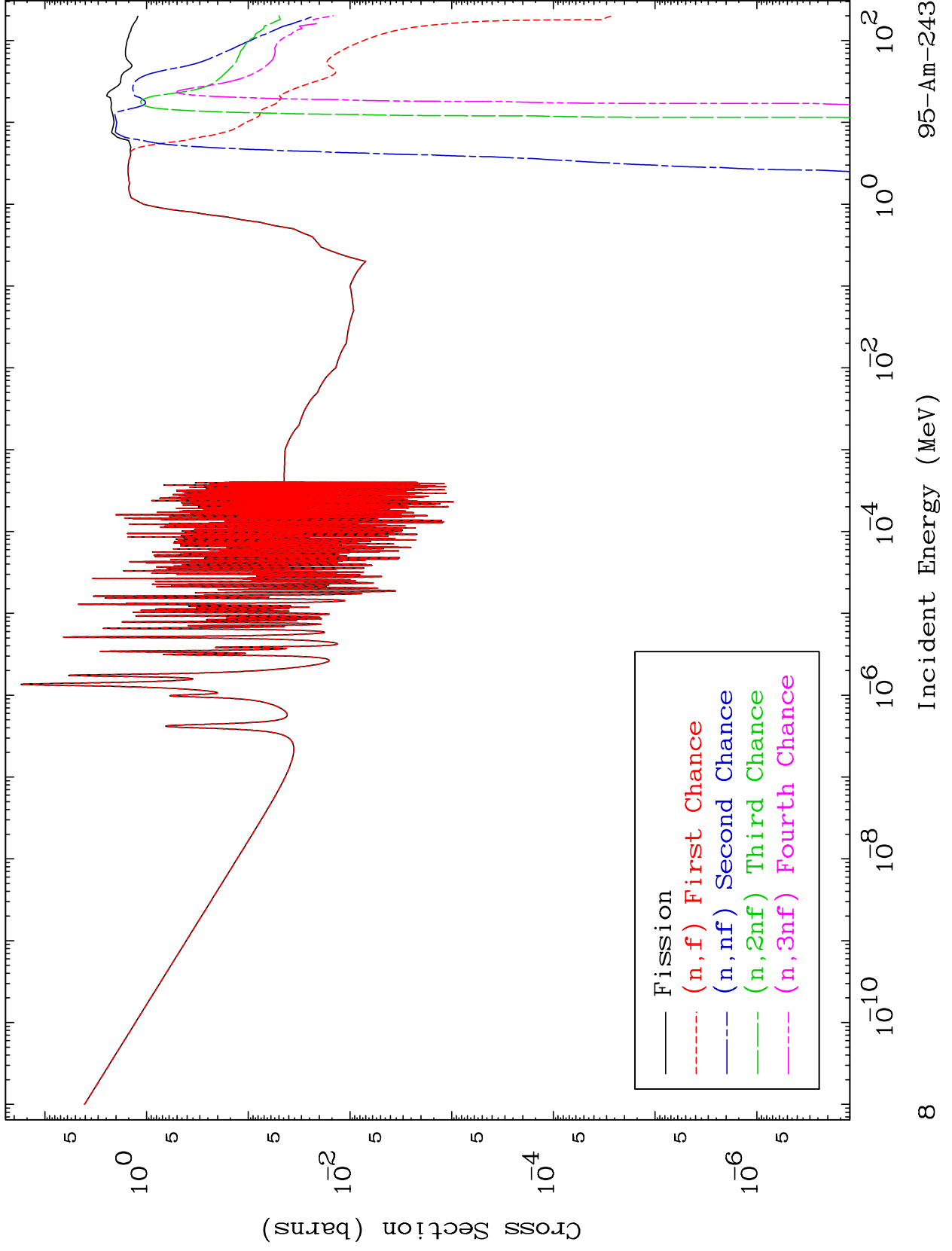
95-Am-243



MAT 9549

Fission  
293 Kelvin Cross Sections

95-Am-243

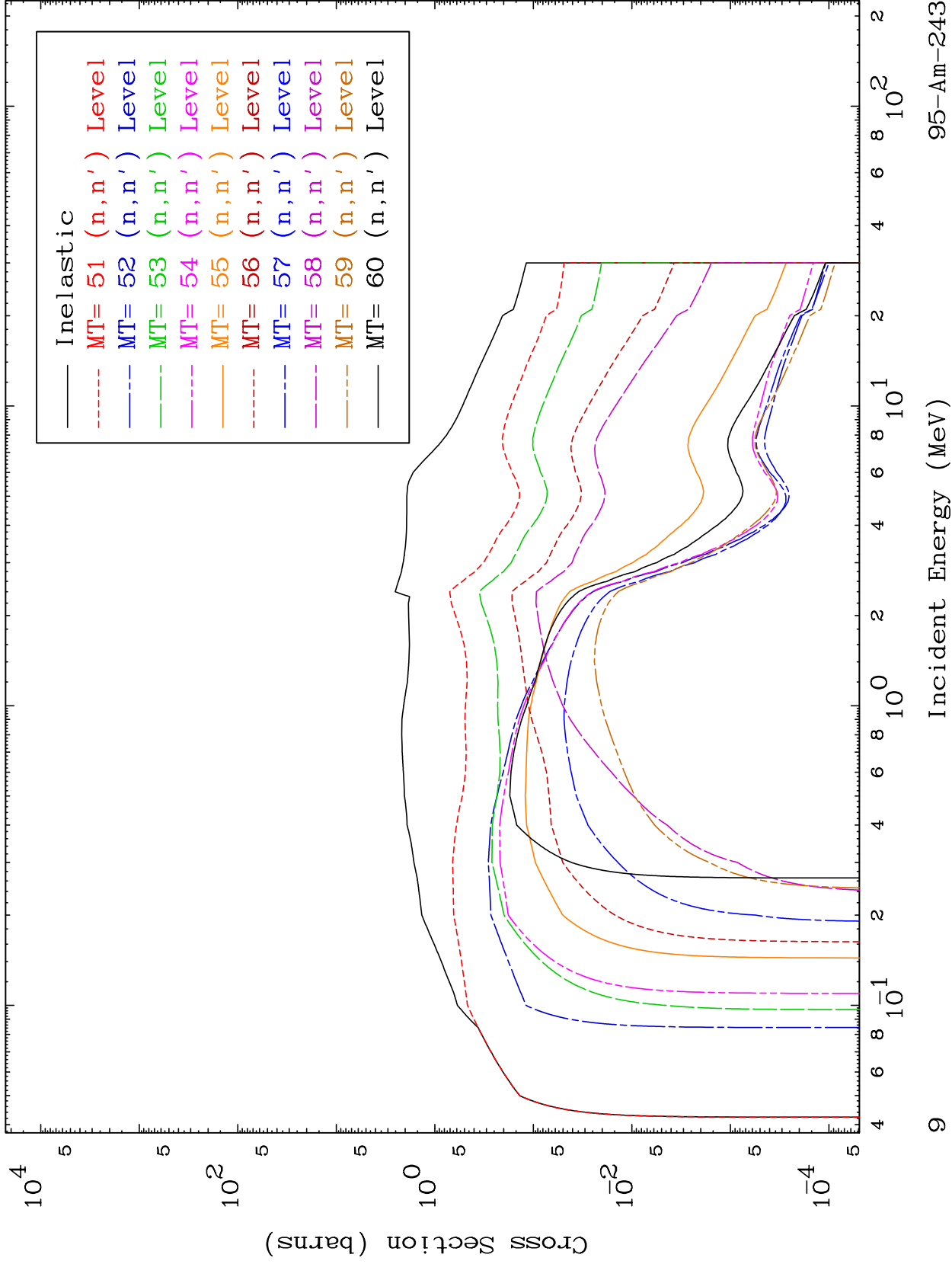


MAT 9549

(n,n') Levels

95-Am-243

293 Kelvin Cross Sections

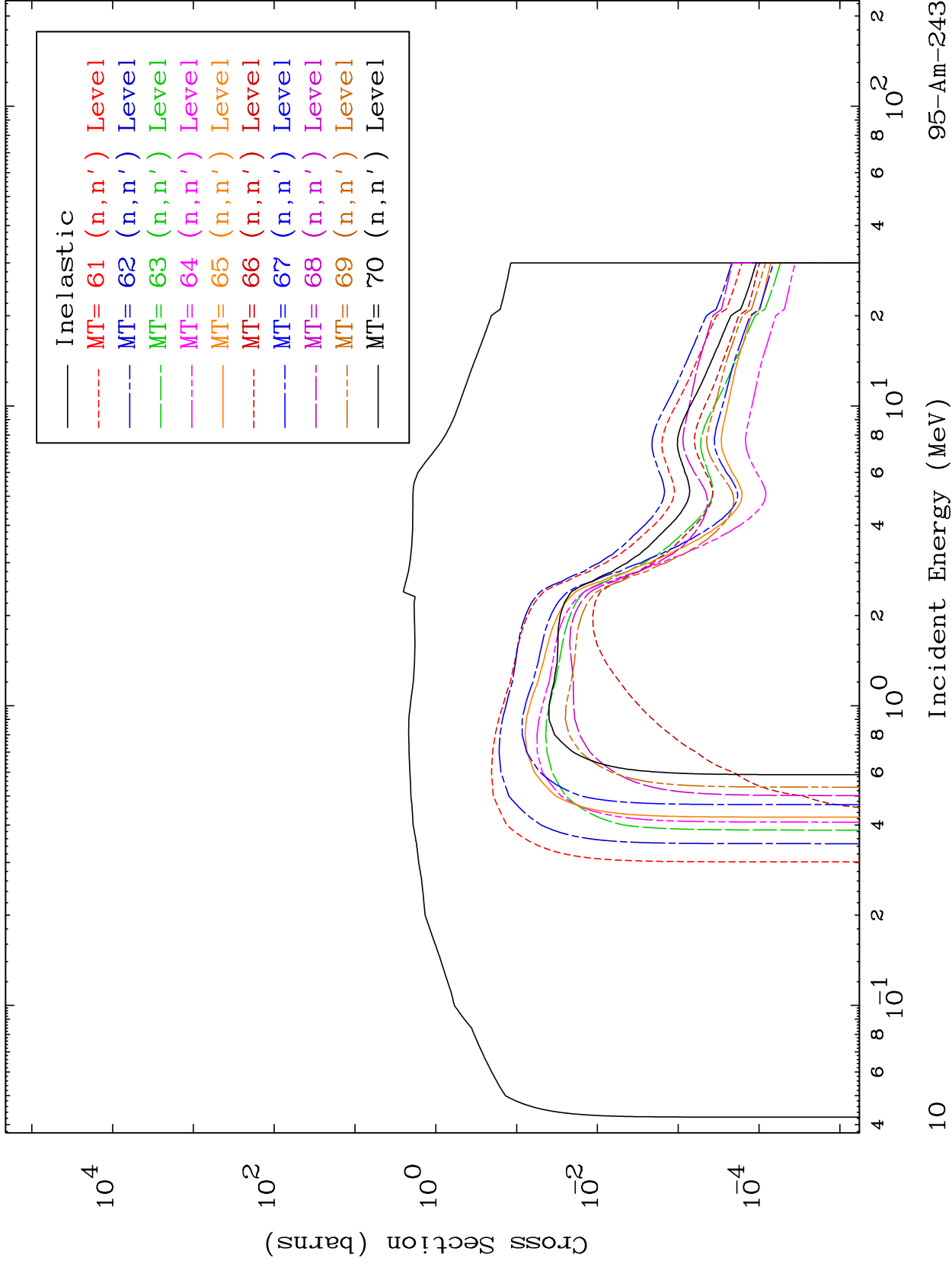


MAT 9549

(n,n') Levels

95-Am-243

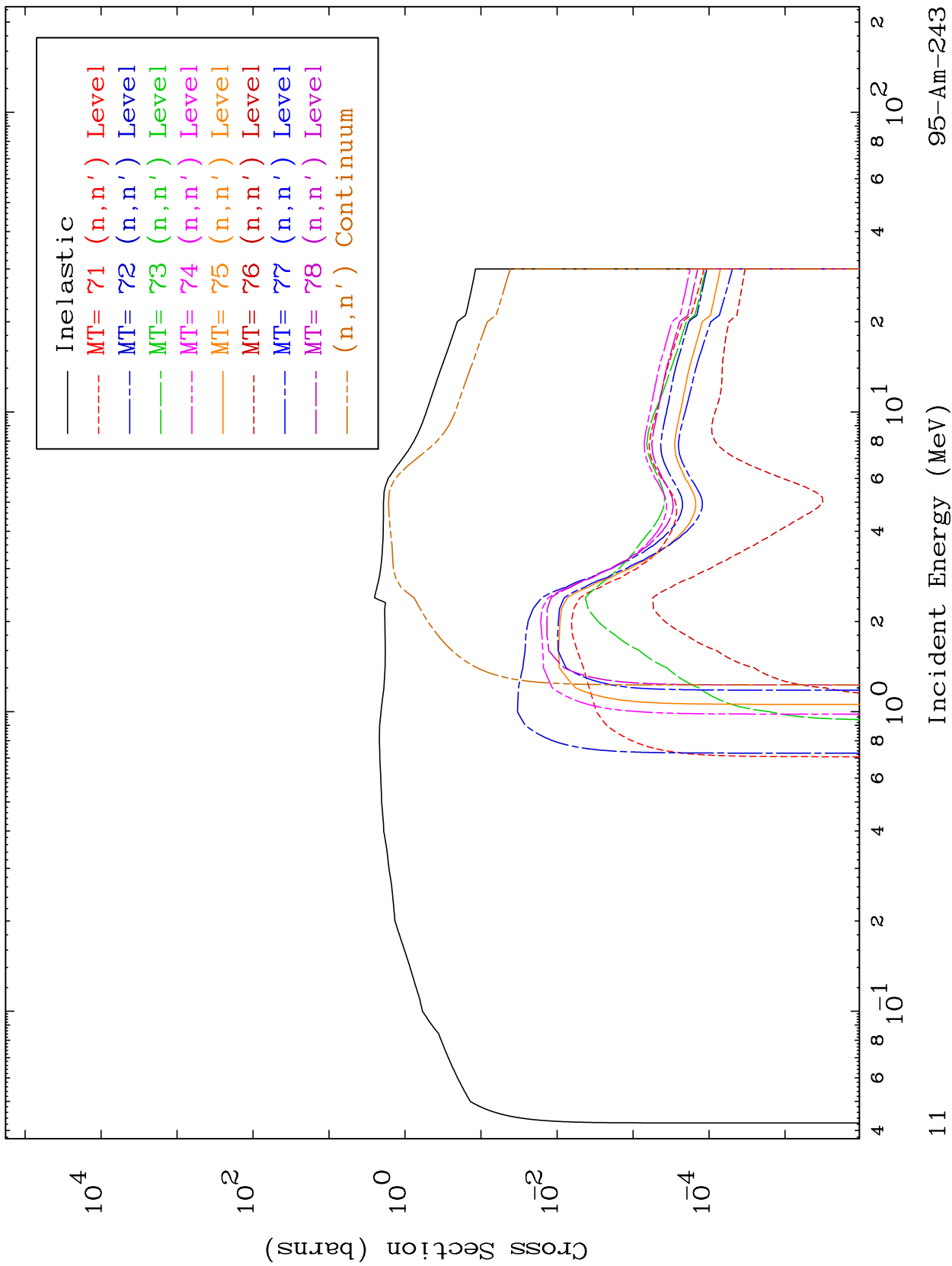
293 Kelvin Cross Sections



10

Incident Energy (MeV)

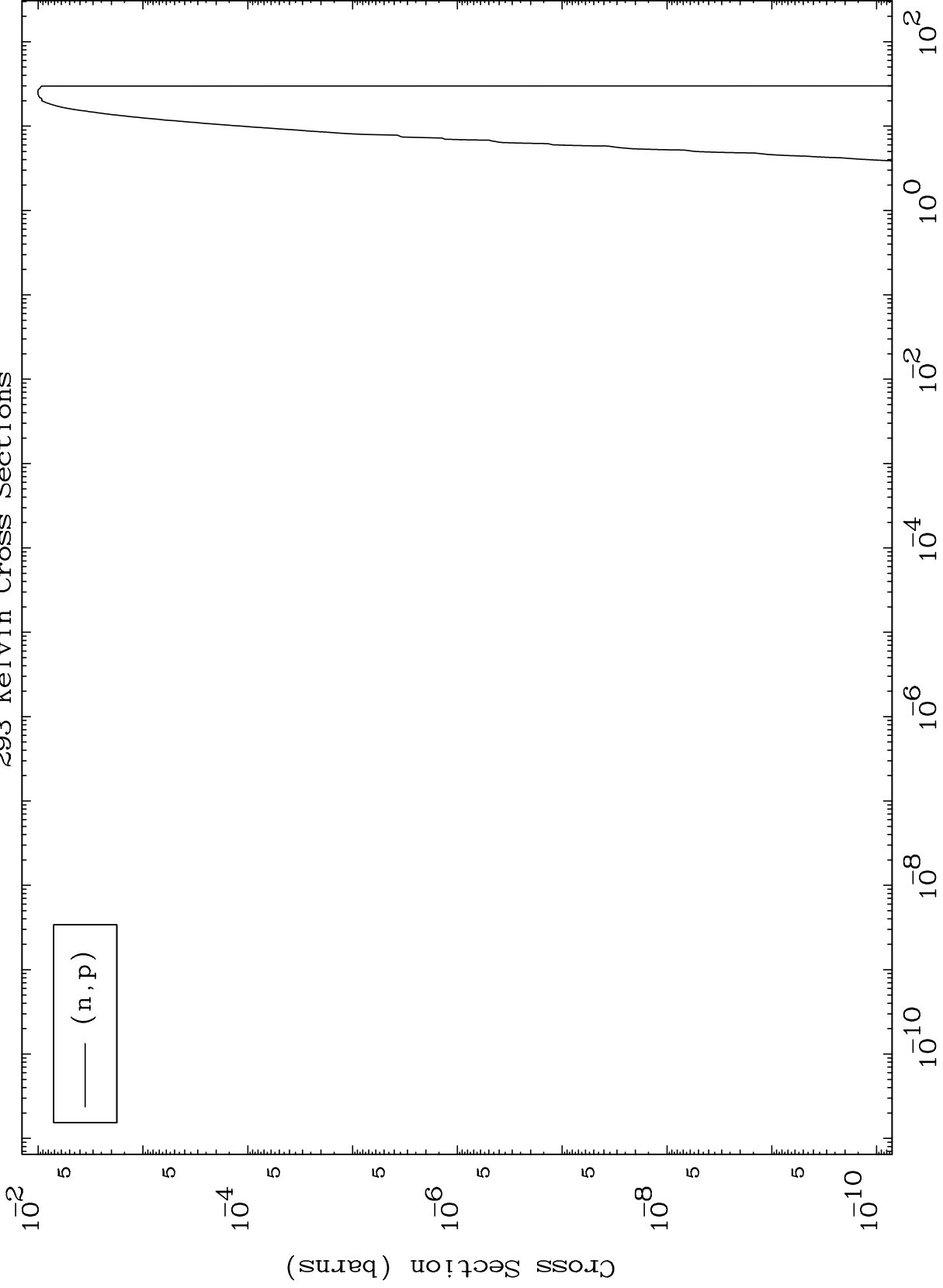
95-Am-243



MAT 9549

(n,p) Levels  
293 Kelvin Cross Sections

95-Am-243



12

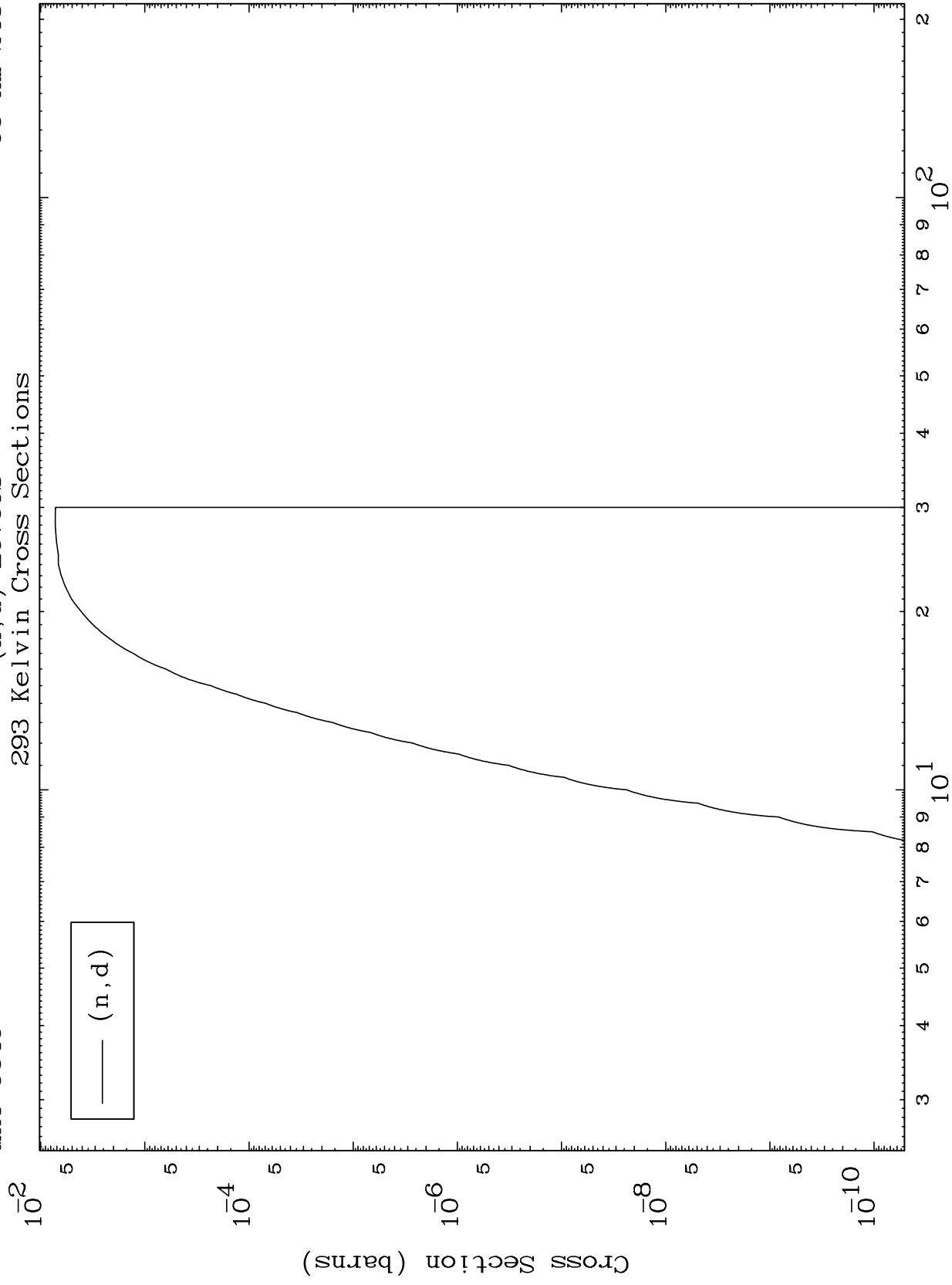
Incident Energy (MeV)

95-Am-243

MAT 9549

(n,d) Levels  
293 Kelvin Cross Sections

95-Am-243



13

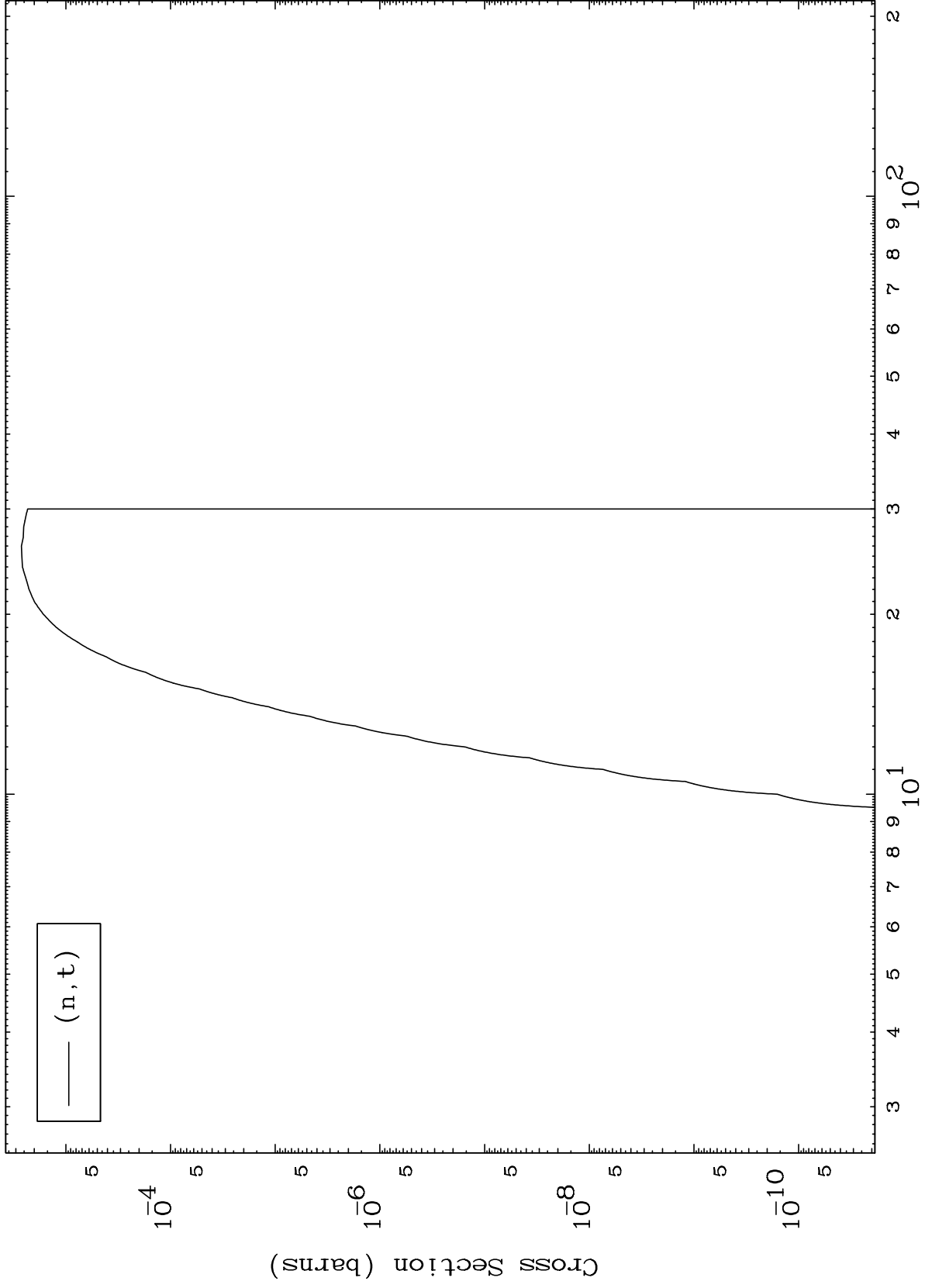
Incident Energy (MeV)

95-Am-243

MAT 9549

(n,t) Levels  
293 Kelvin Cross Sections

95-Am-243



14

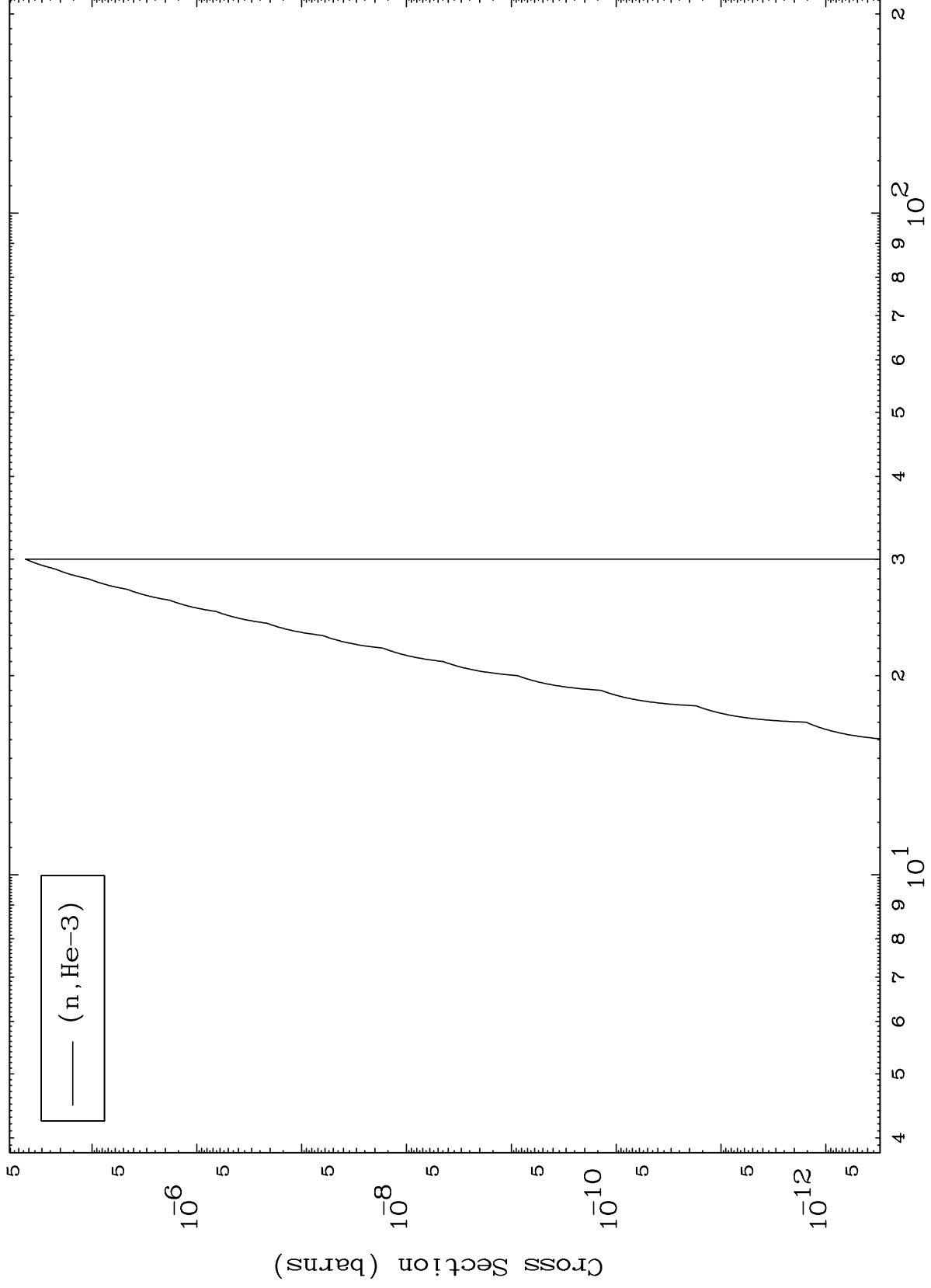
Incident Energy (MeV)

95-Am-243

MAT 9549

(n,He3) Levels  
293 Kelvin Cross Sections

95-Am-243



15

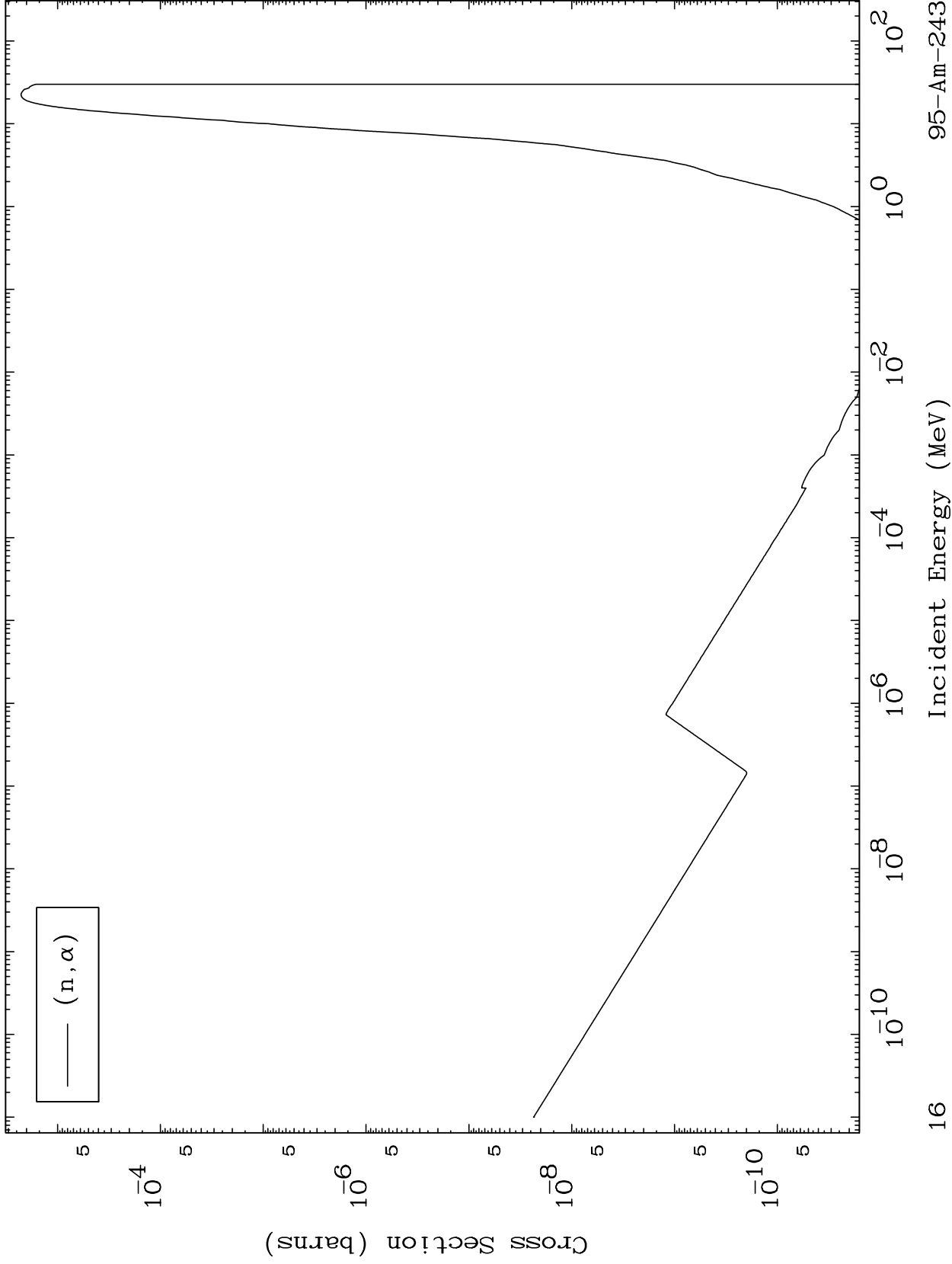
Incident Energy (MeV)

95-Am-243

MAT 9549

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

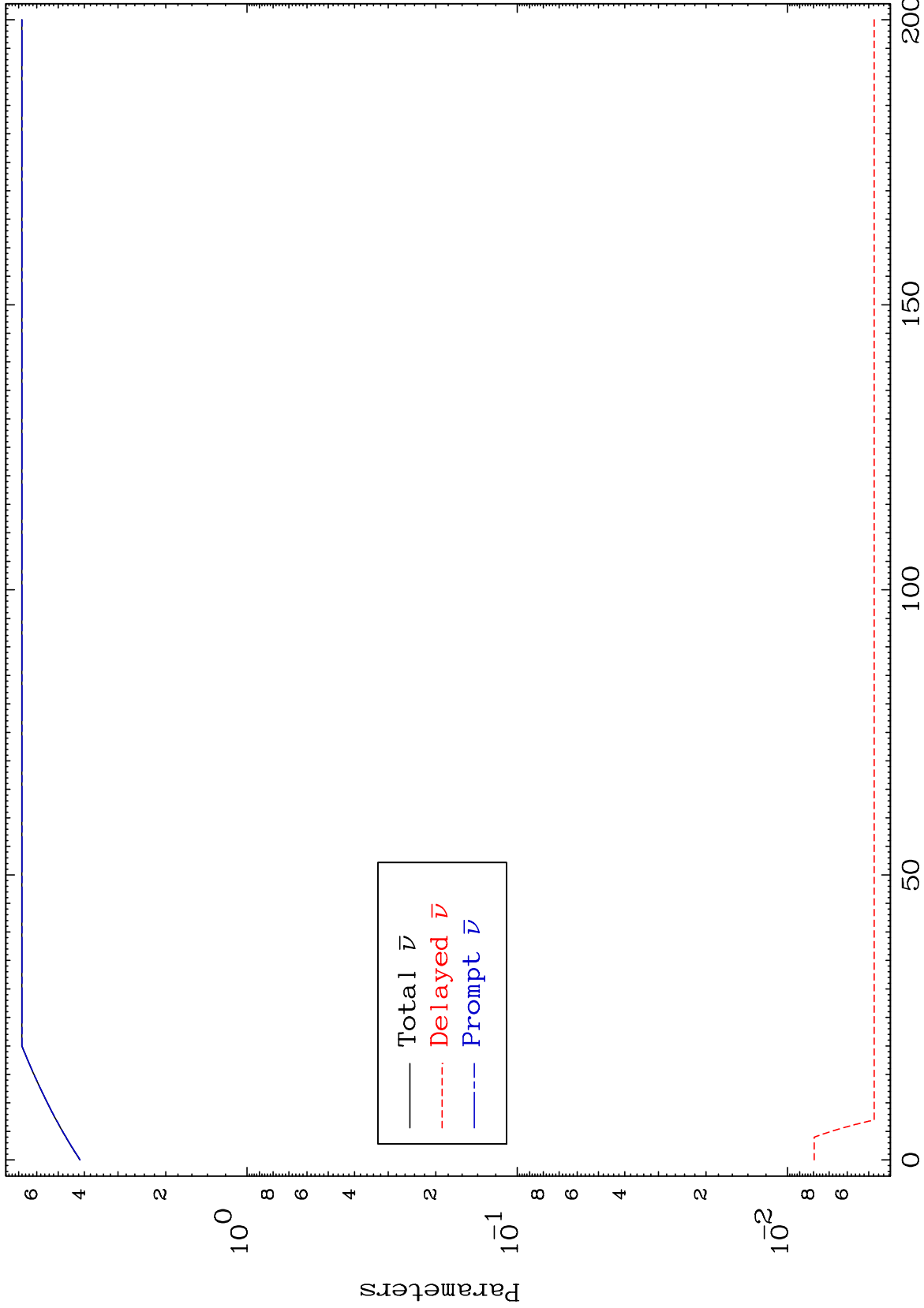
95-Am-243



MAT 9549

Energy Release  
Parameters

95-Am-243



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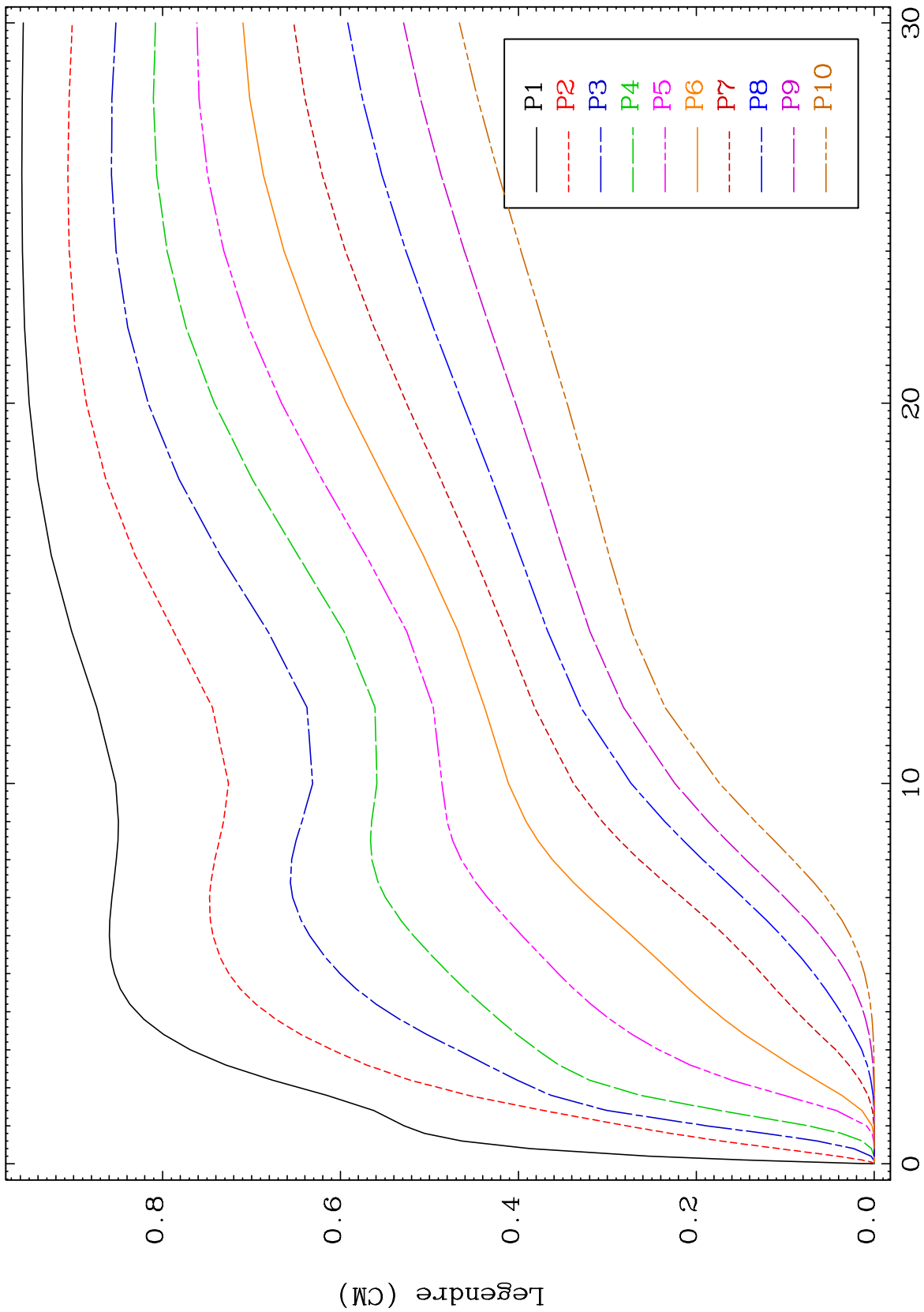
Incident Energy (MeV)

95-Am-243

MAT 9549

95-Am-243

Elastic Legendre Coefficients



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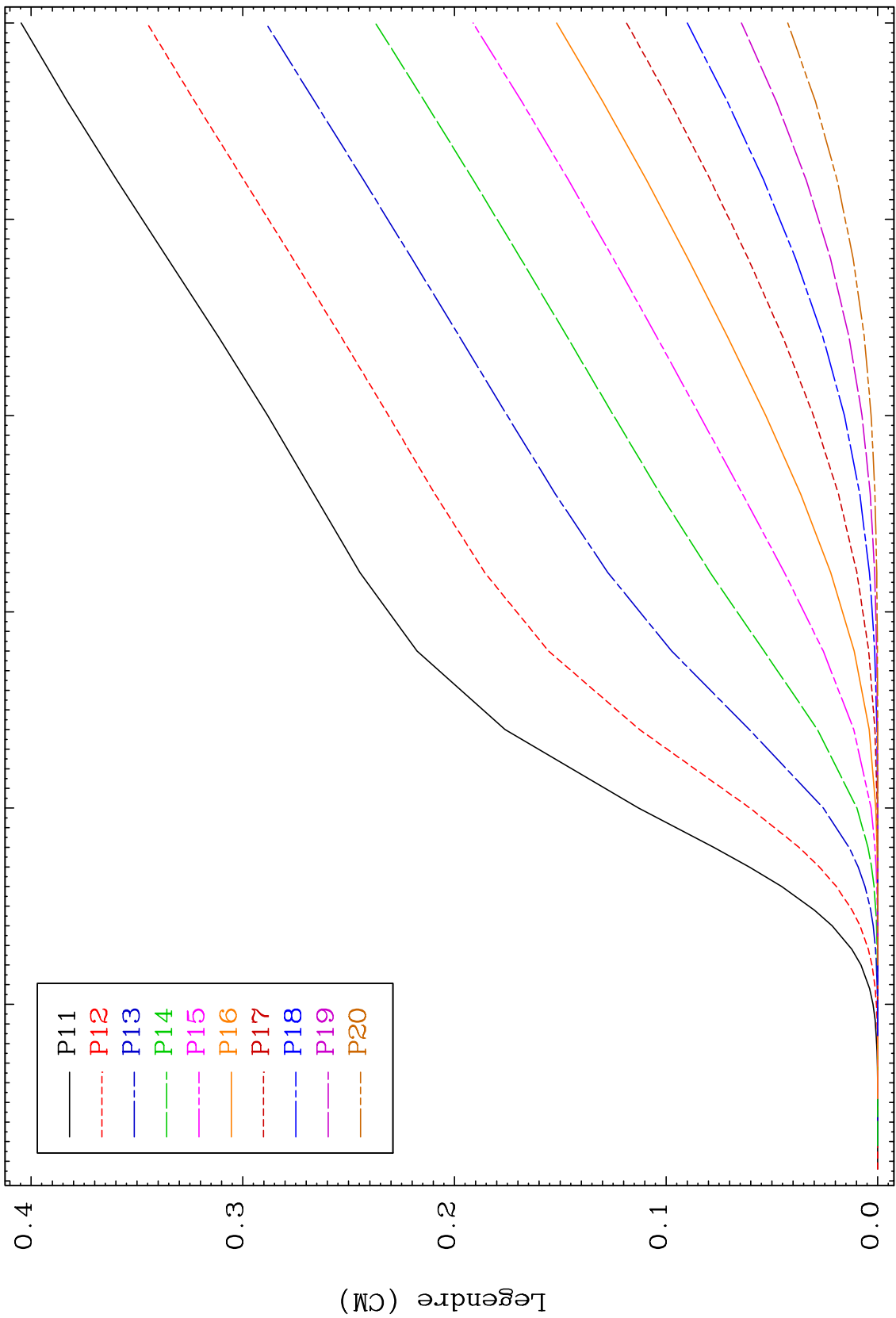
Incident Energy (MeV)

95-Am-243

MAT 9549

Elastic Legendre Coefficients

<sup>95</sup>Am-243



19

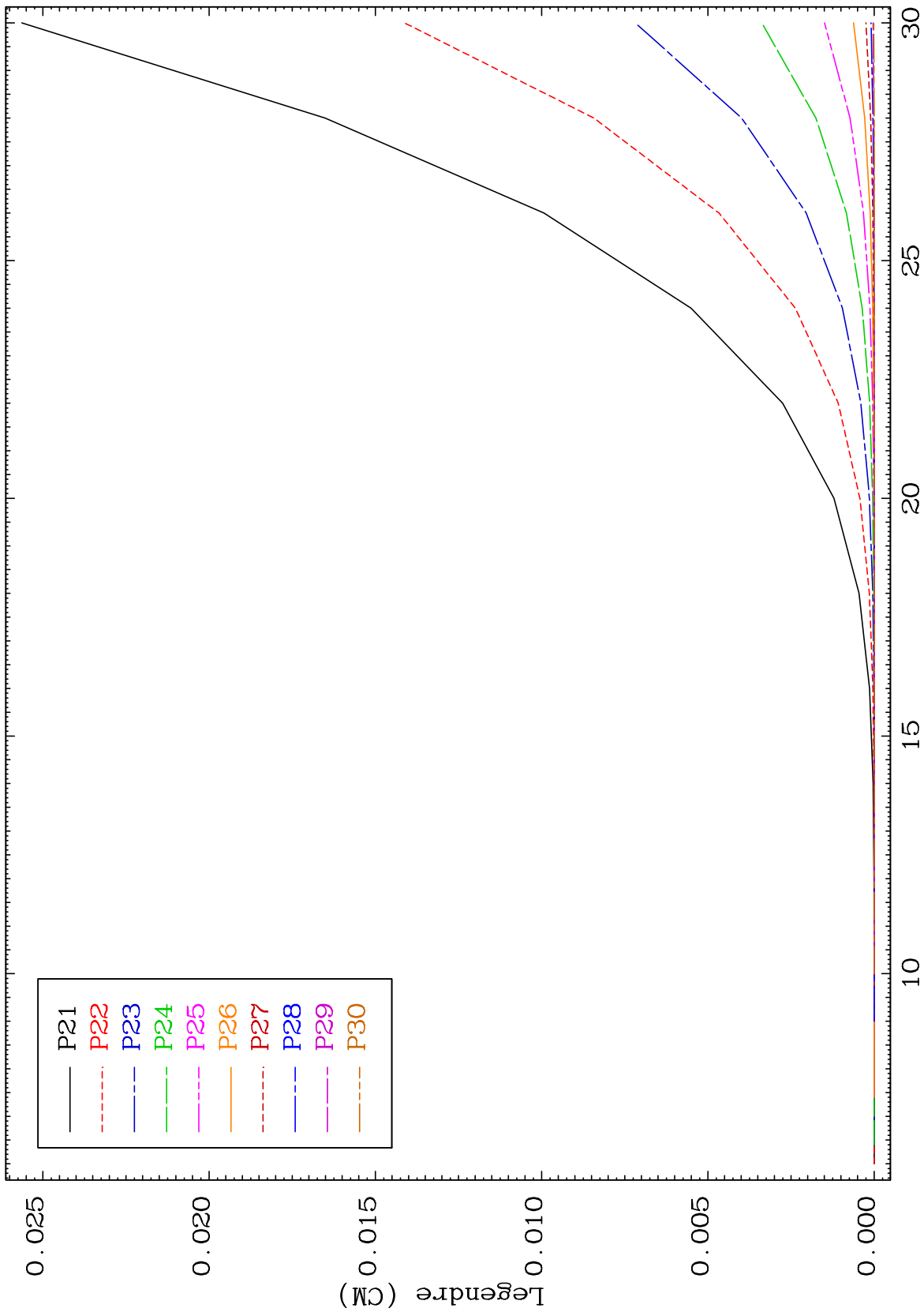
Incident Energy (MeV)

<sup>95</sup>Am-243

MAT 9549

Elastic Legendre Coefficients

95-Am-243



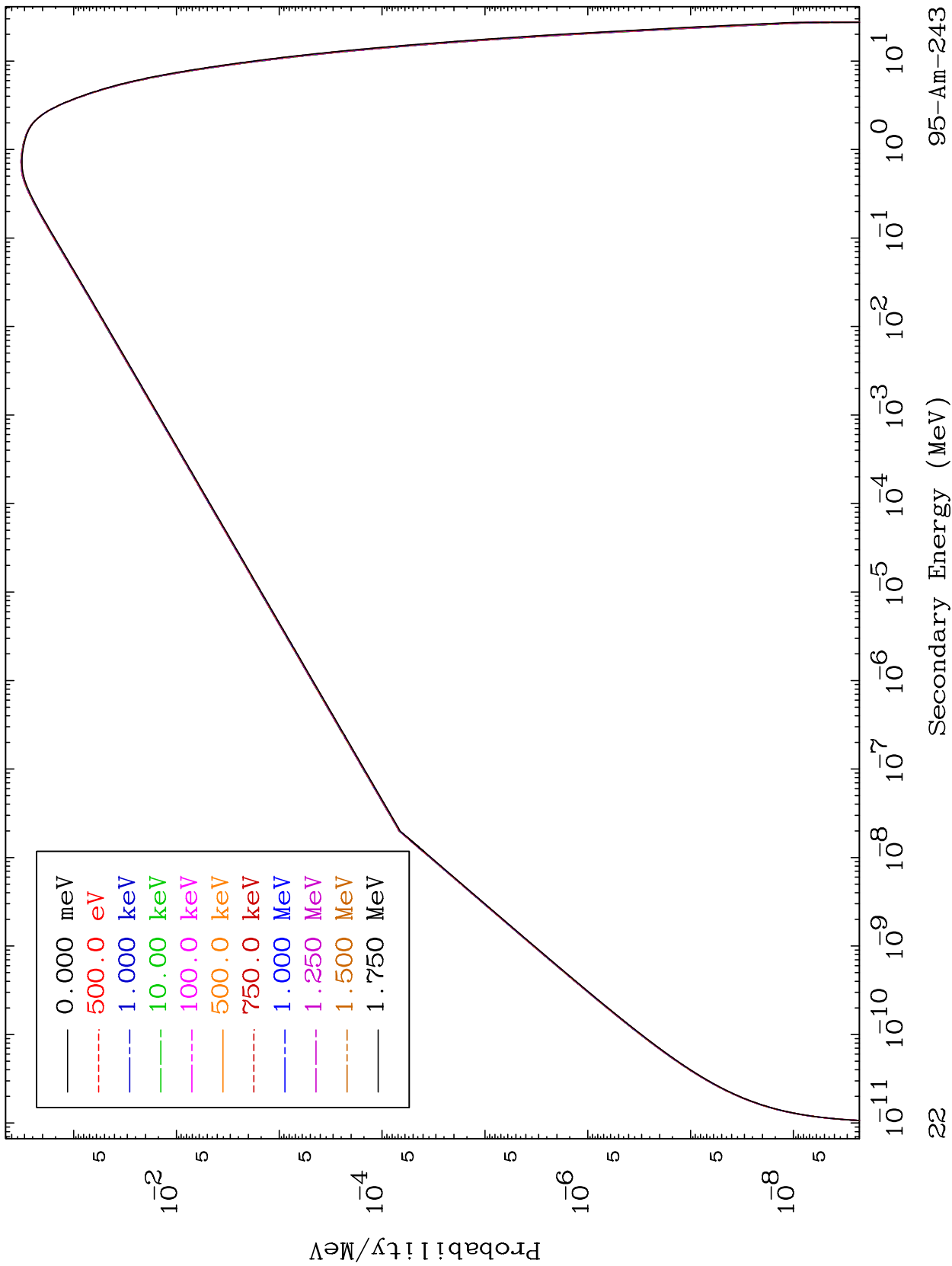
20

Incident Energy (MeV)

95-Am-243



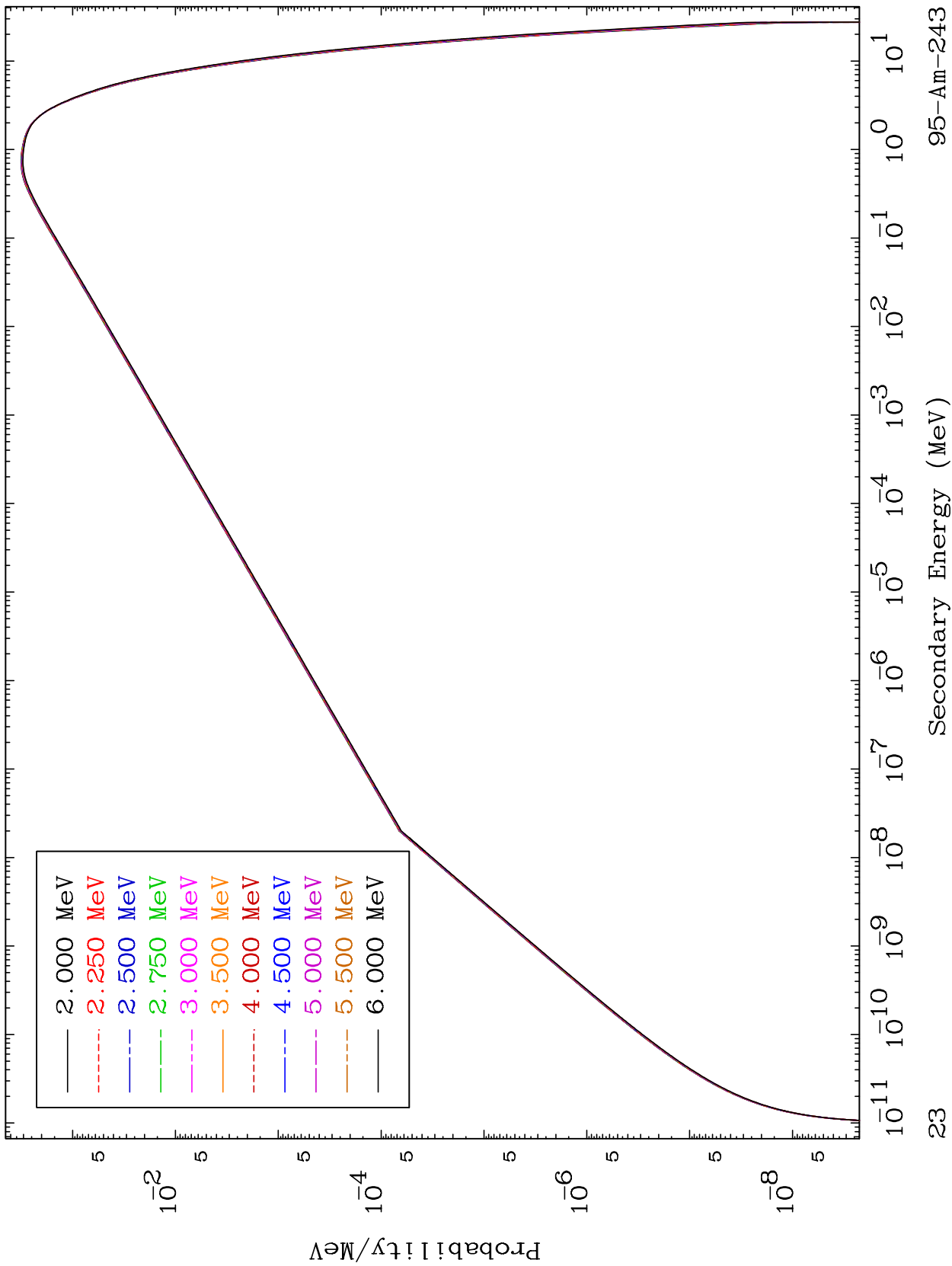
Fission Energy Distributions



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### Fission Energy Distributions

95-Am-243



95-Am-243

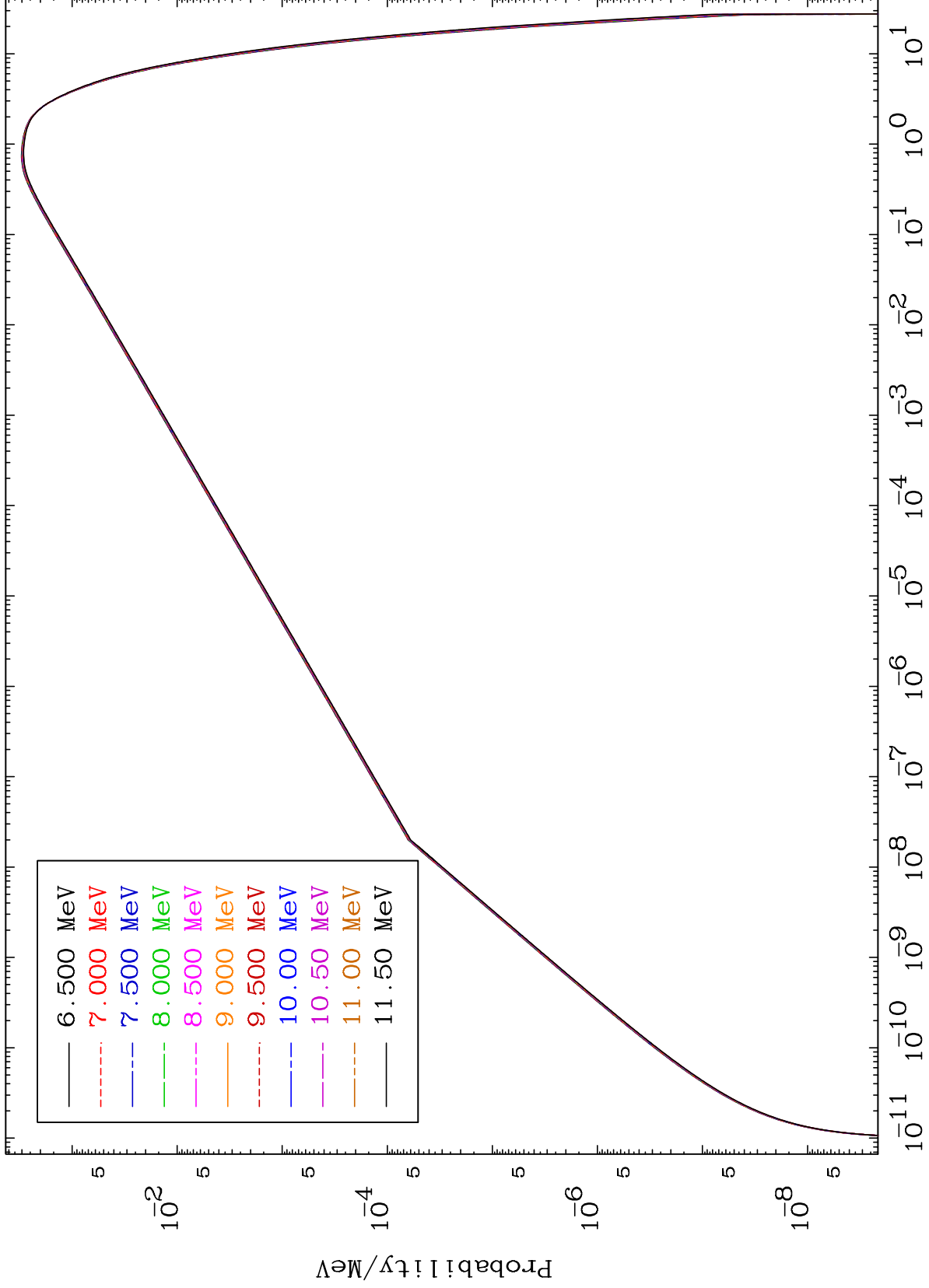
Secondary Energy (MeV)

23

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### Fission Energy Distributions

95-Am-243



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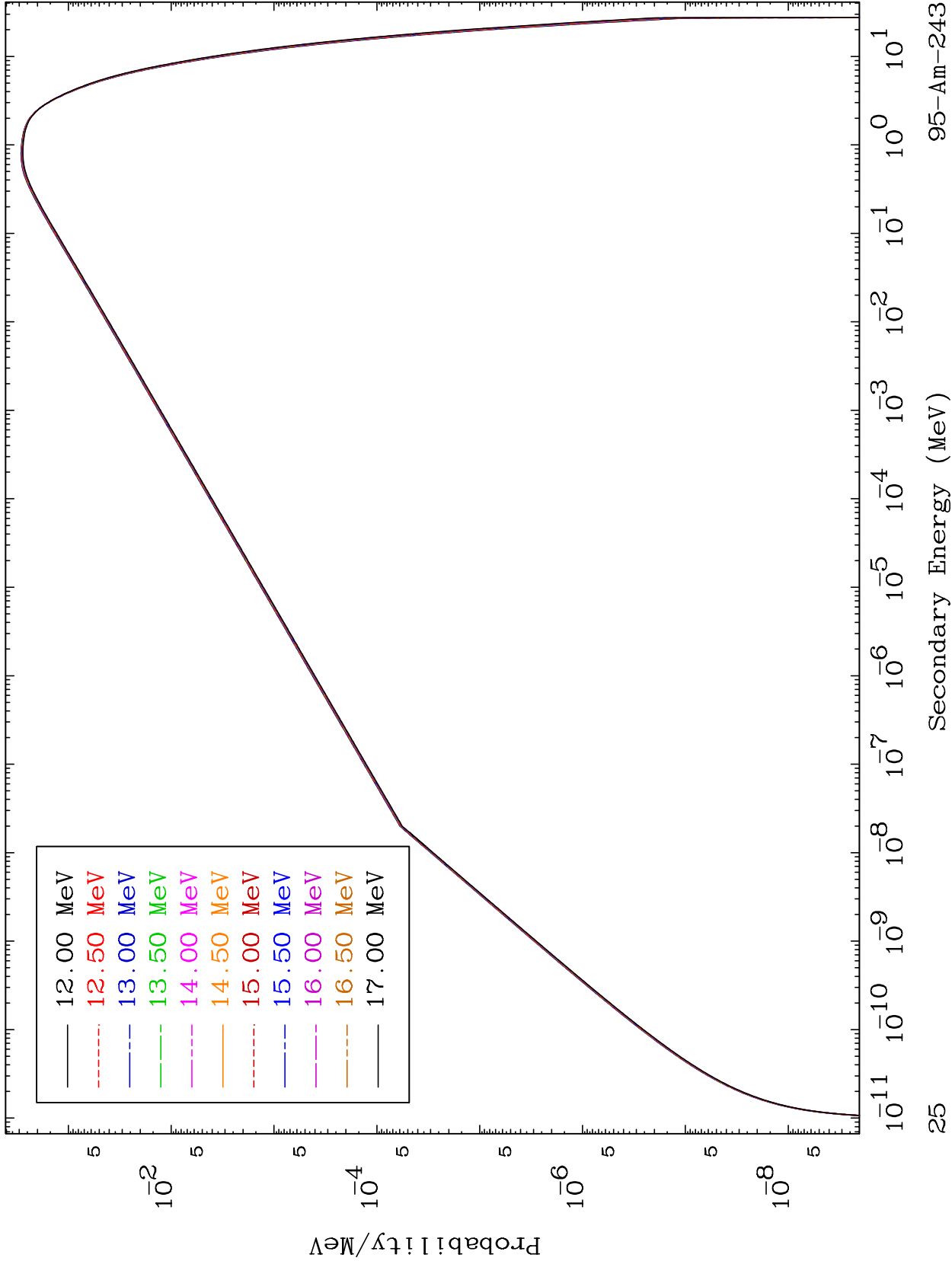
Secondary Energy (MeV)

95-Am-243

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### Fission Energy Distributions

95-Am-243



95-Am-243

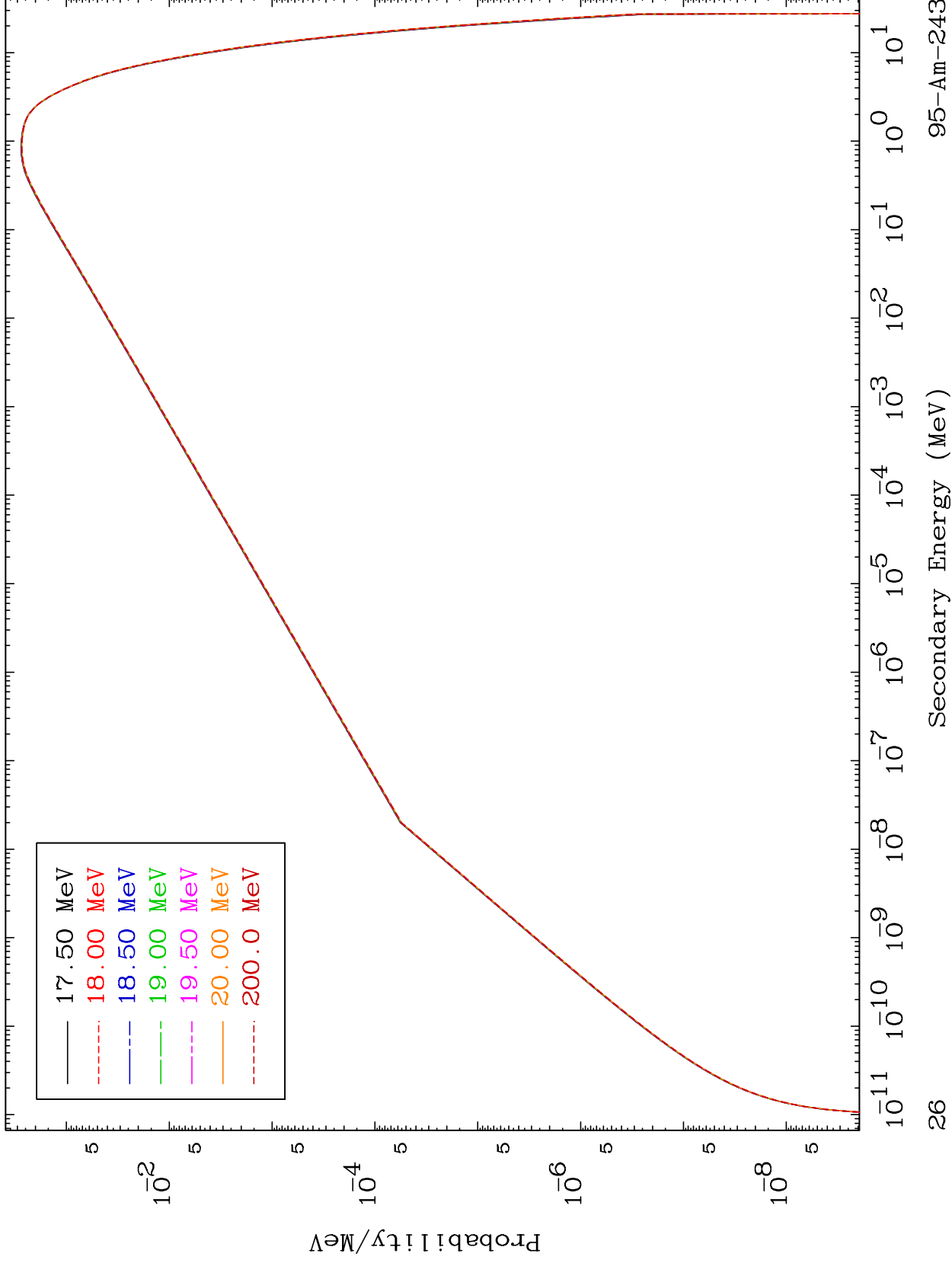
Secondary Energy (MeV)

25

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### Fission Energy Distributions

95-Am-243



95-Am-243

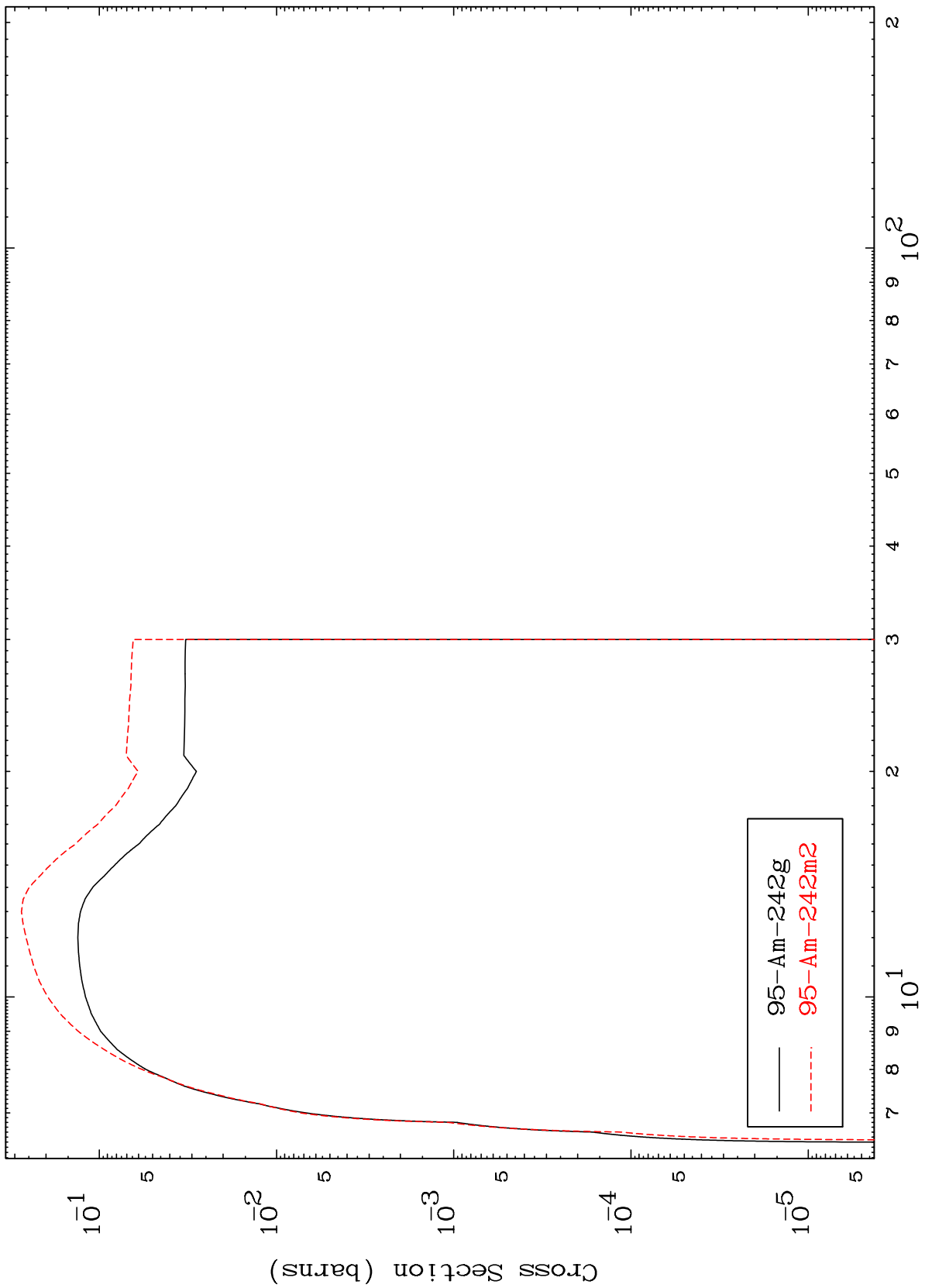
Secondary Energy (MeV)

26

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95-Am-243

(n,2n)  
Radionuclide Production Cross Section



95-Am-243

Incident Energy (MeV)

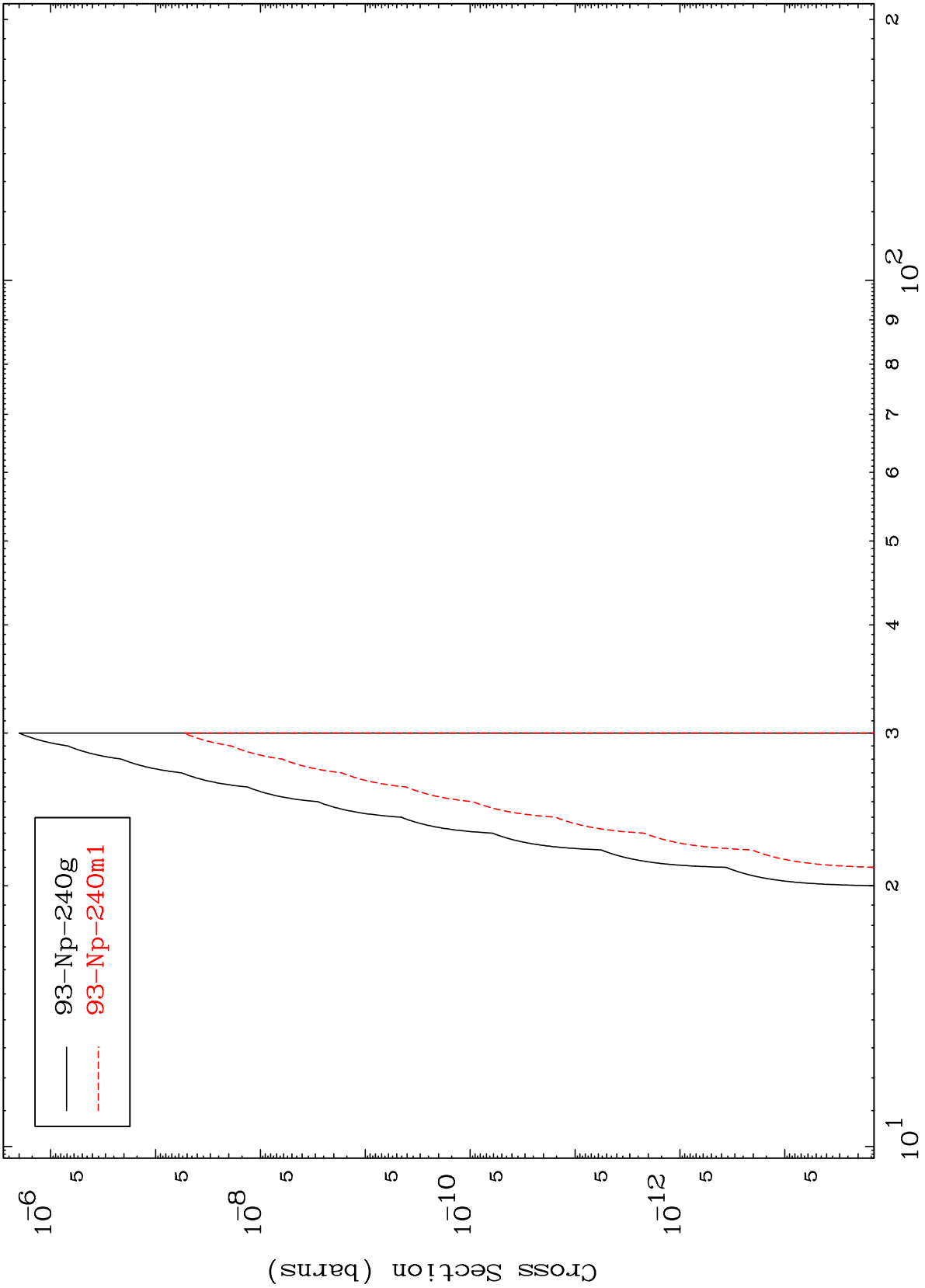
27

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(n,n') He-3

95-Am-243

Radionuclide Production Cross Section



Incident Energy (MeV)

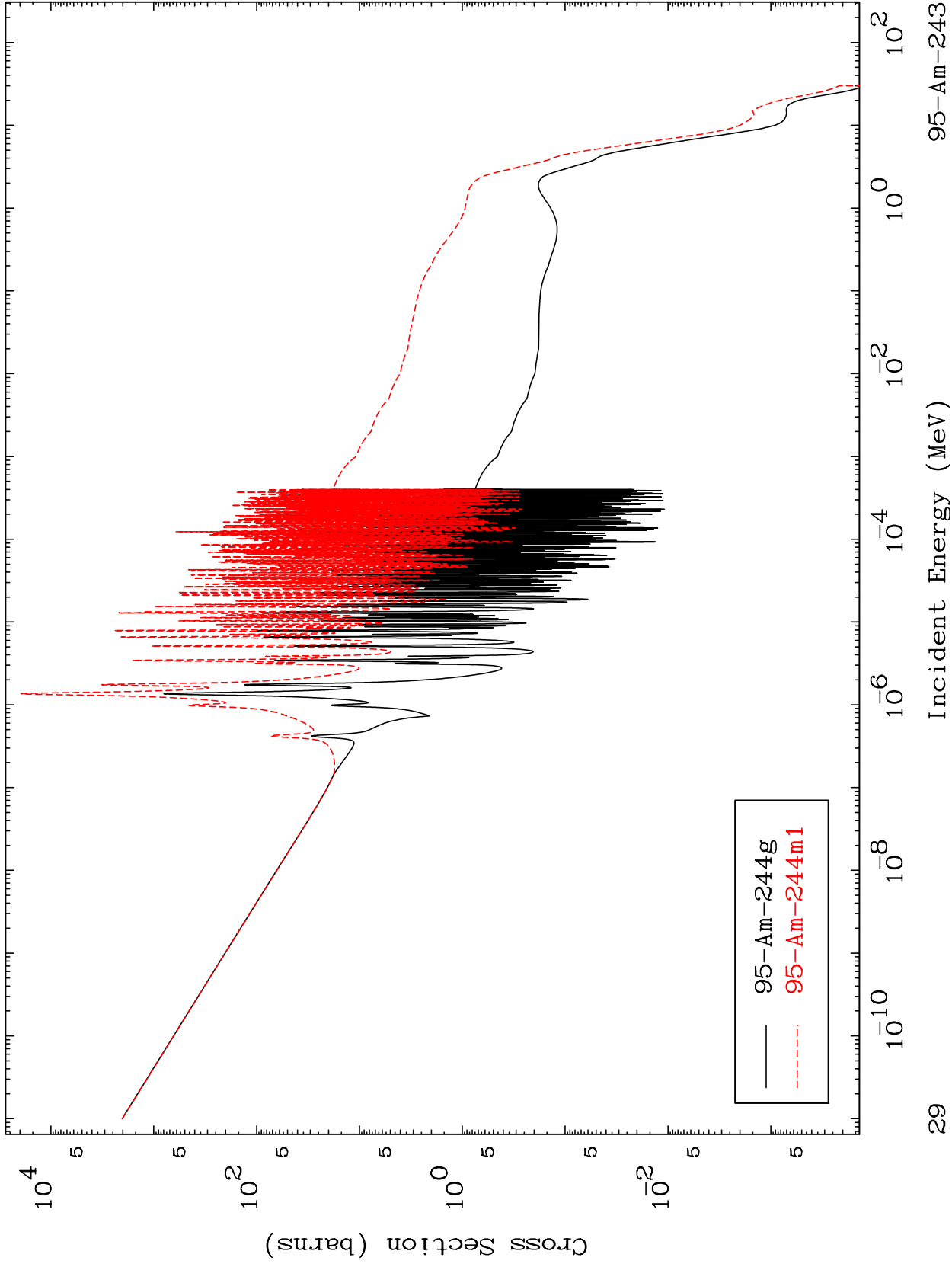
95-Am-243

28

MAT 9549

95-Am-243

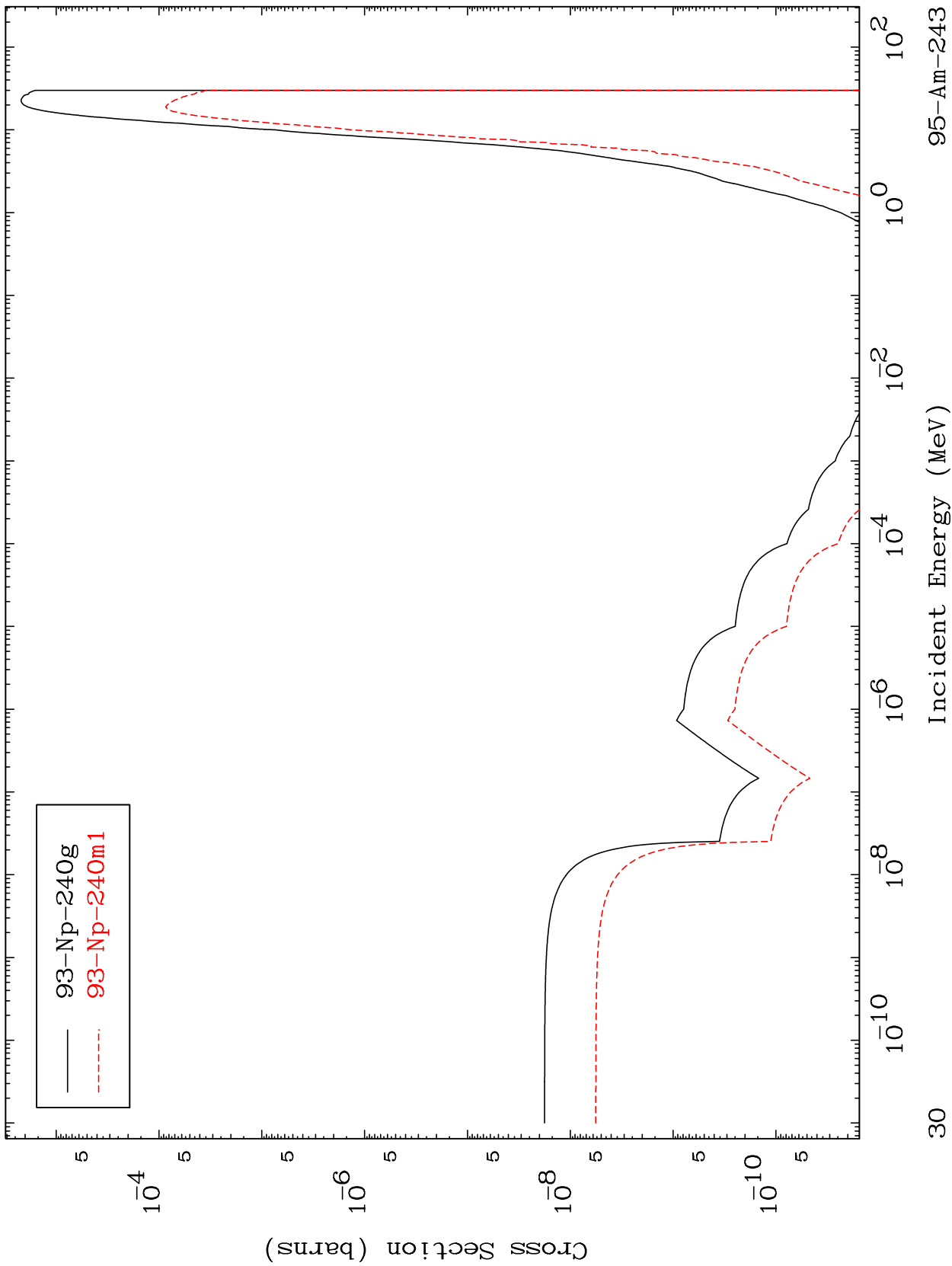
(n,  $\gamma$ )  
Radionuclide Production Cross Section



MAT 9549

95-Am-243

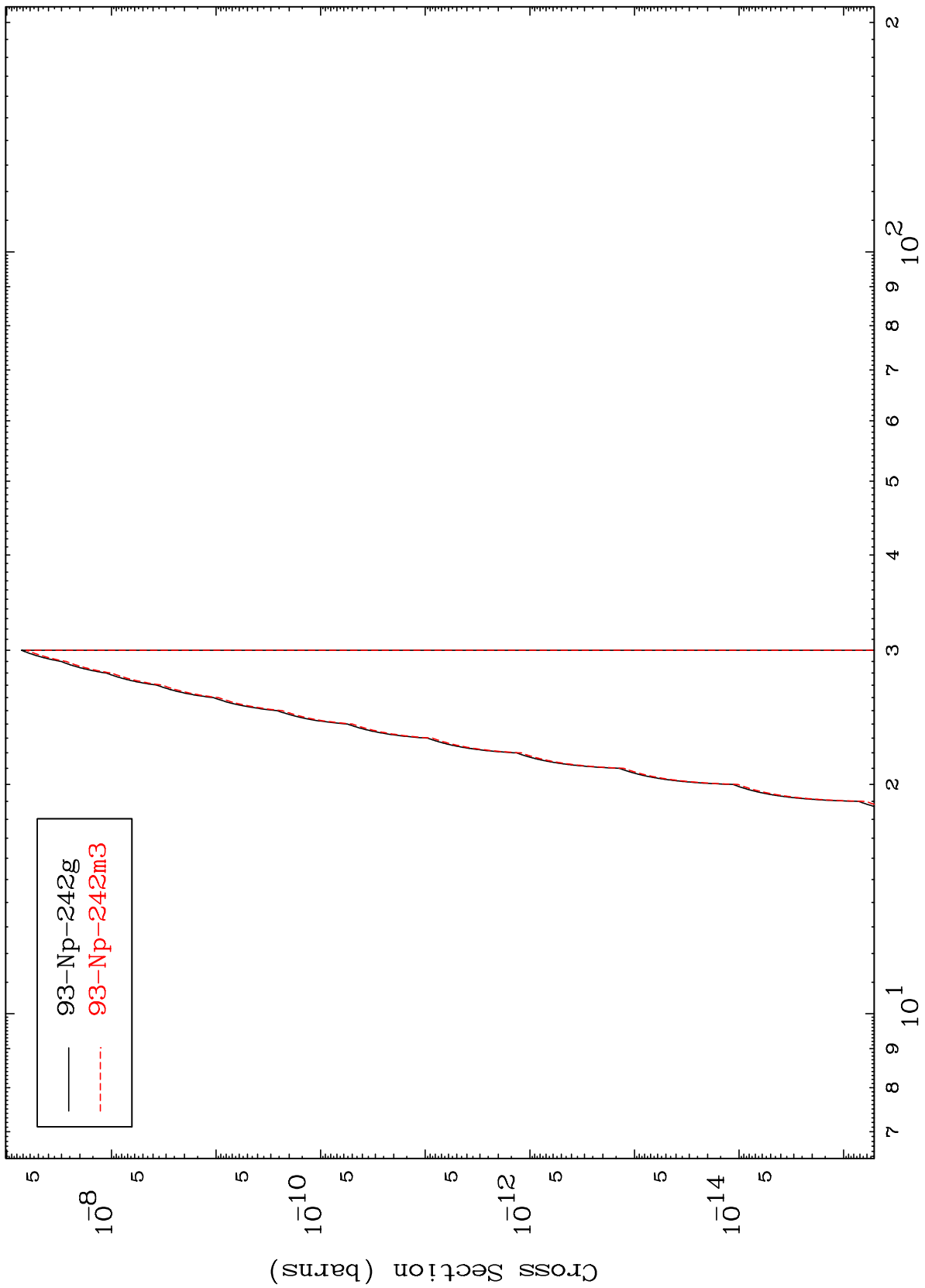
(n,  $\alpha$ )  
Radionuclide Production Cross Section



MAT 9549

95-Am-243

(n,2p)  
Radionuclide Production Cross Section



— 93-Np-242g  
- - - 93-Np-242m3

31

Incident Energy (MeV)

95-Am-243