

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail: [redcullen1@comcast.net](mailto:redcullen1@comcast.net)

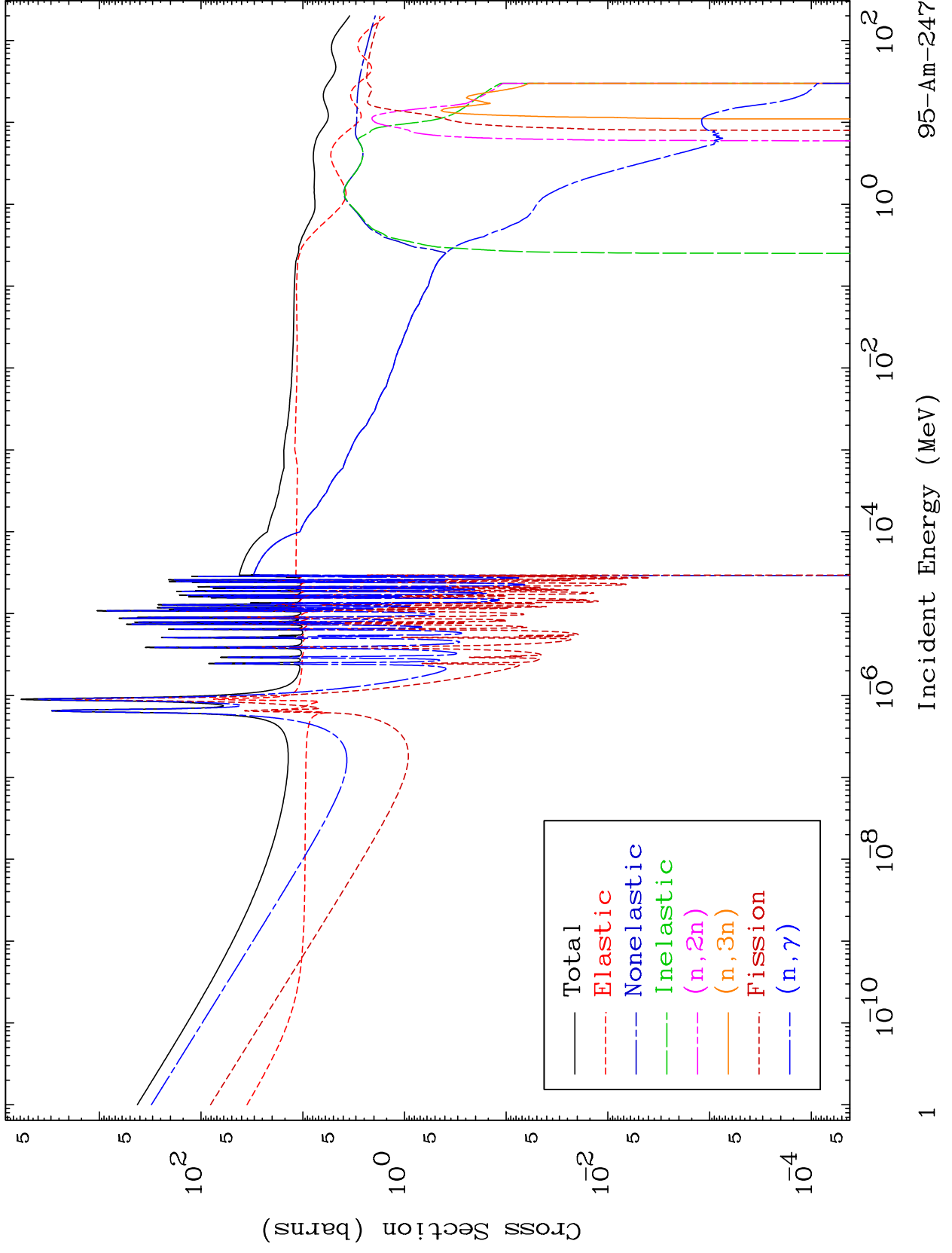
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 9561

Neutron Major  
293 Kelvin Cross Sections

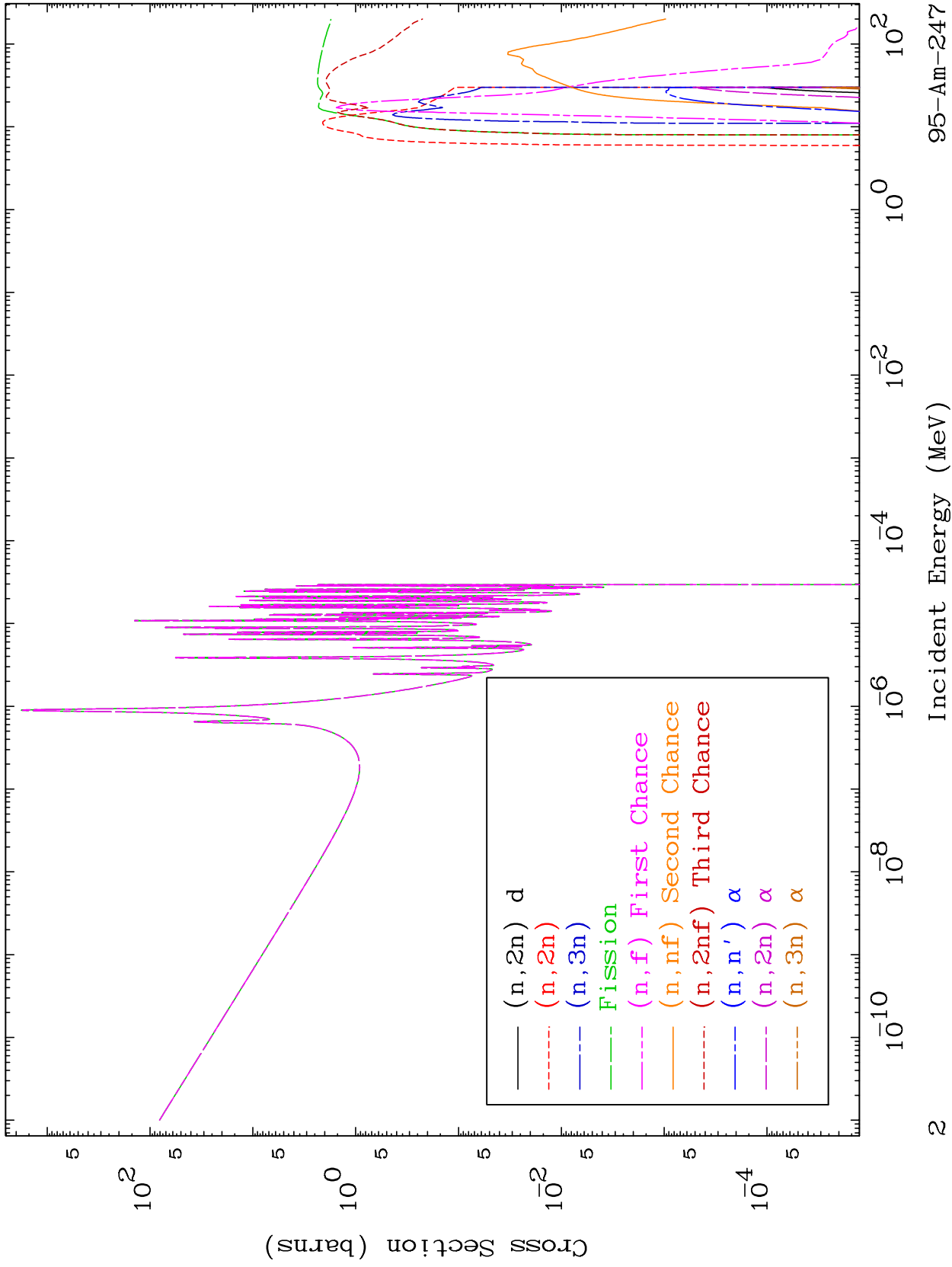
95-Am-247



MAT 9561

Neutron Absorption  
293 Kelvin Cross Sections

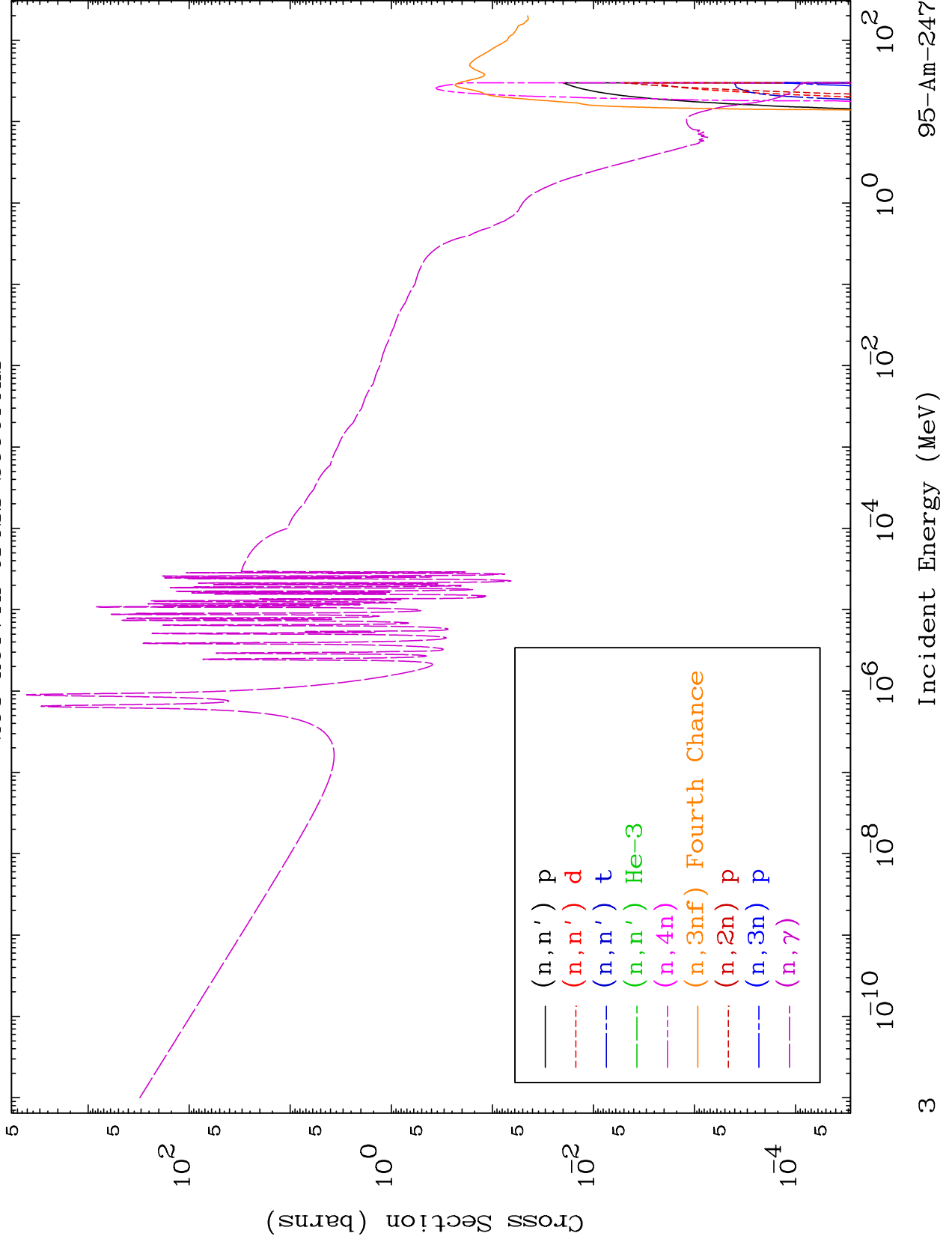
95-Am-247



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Neutron Absorption  
293 Kelvin Cross Sections

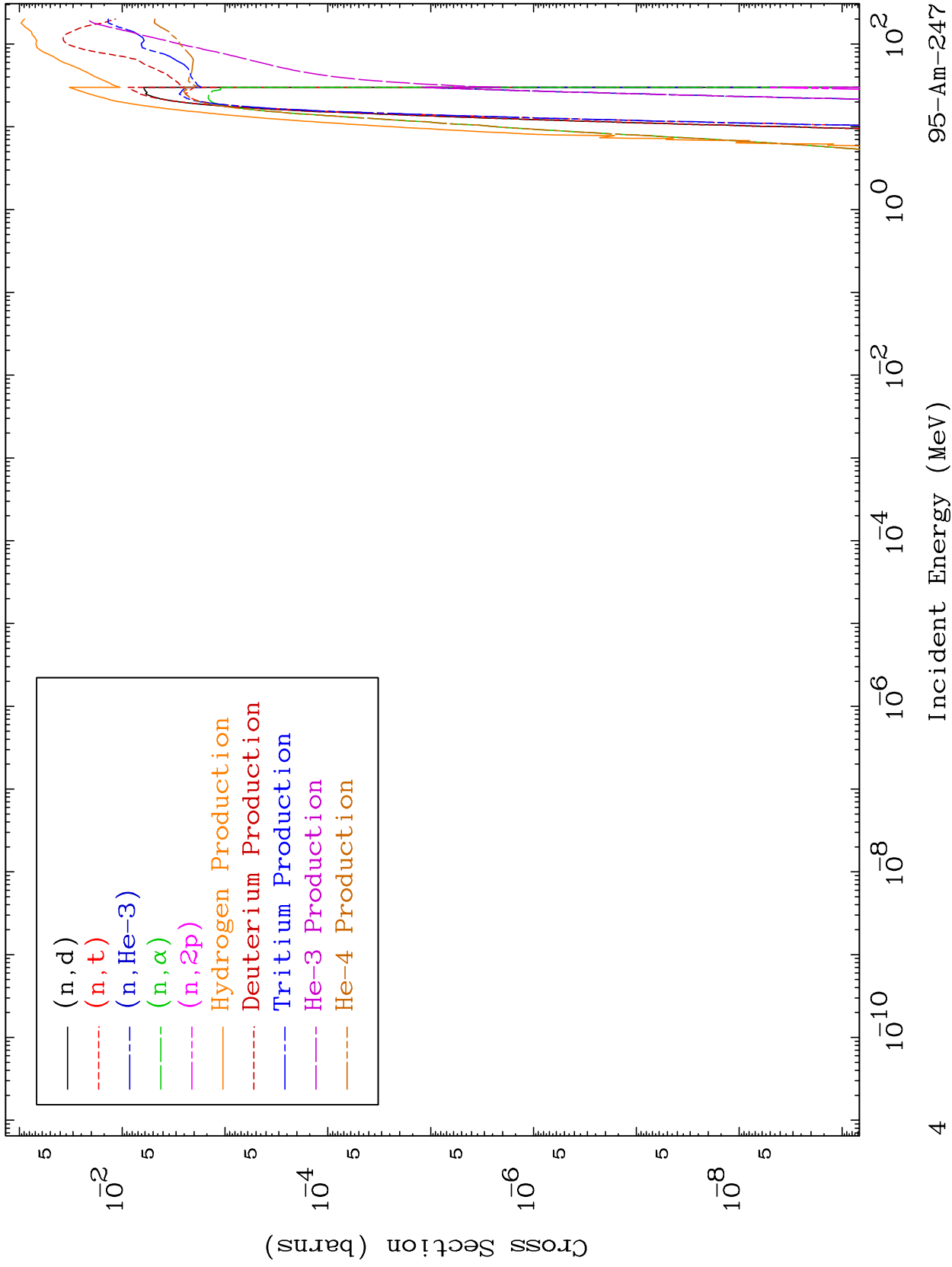
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Neutron Absorption  
293 Kelvin Cross Sections

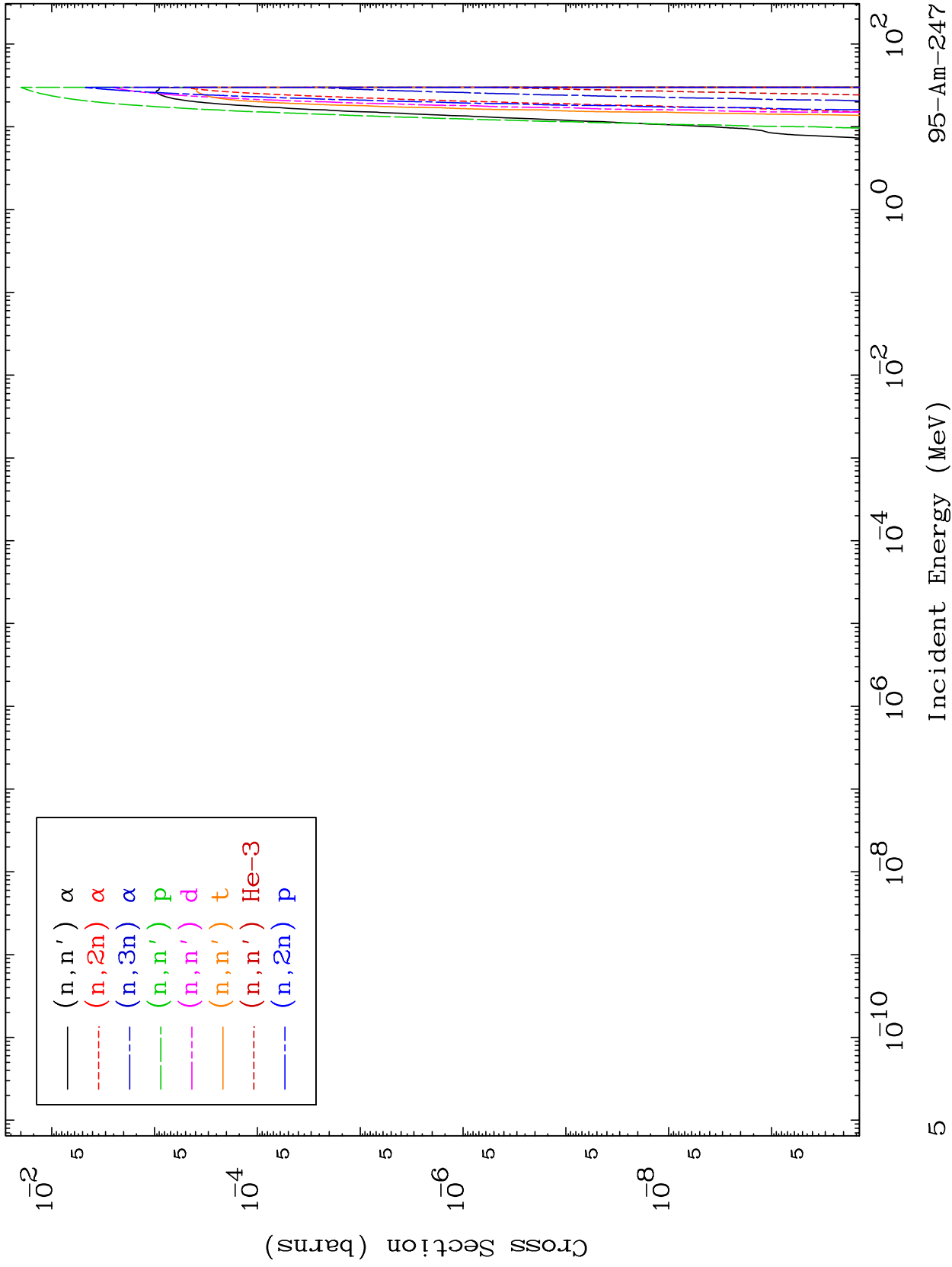
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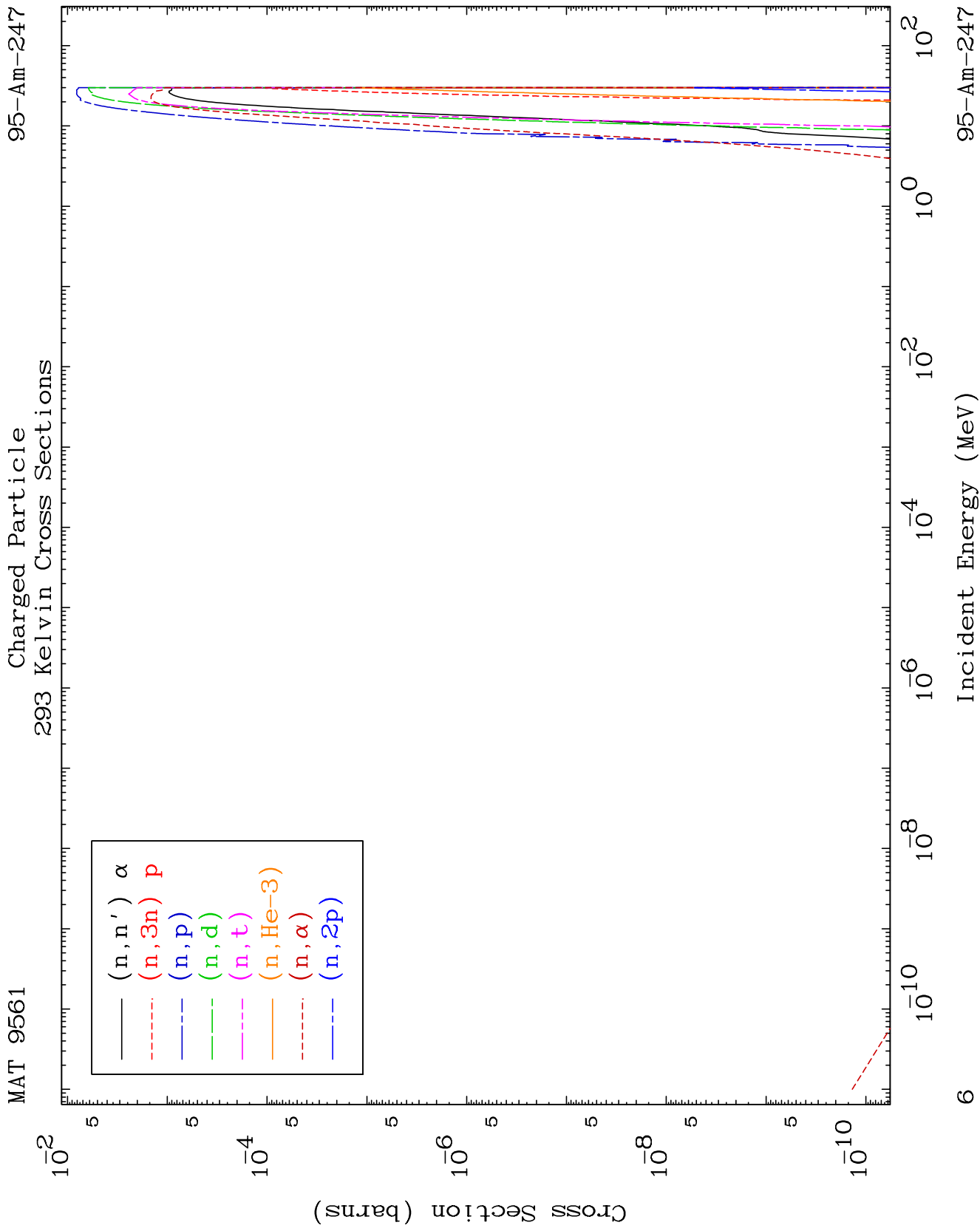


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Charged Particle  
293 Kelvin Cross Sections

95-Am-247

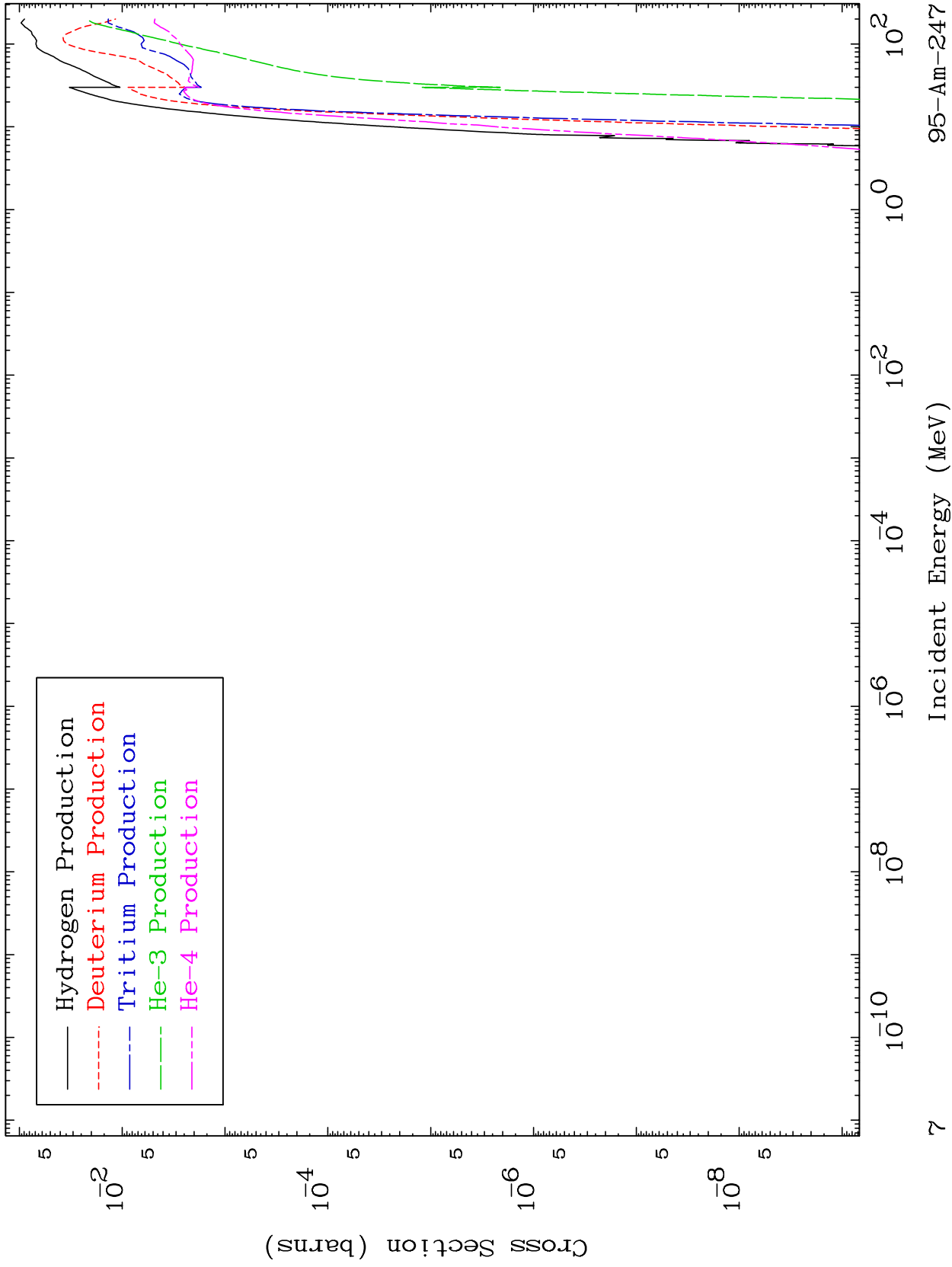




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Particle Production  
293 Kelvin Cross Sections

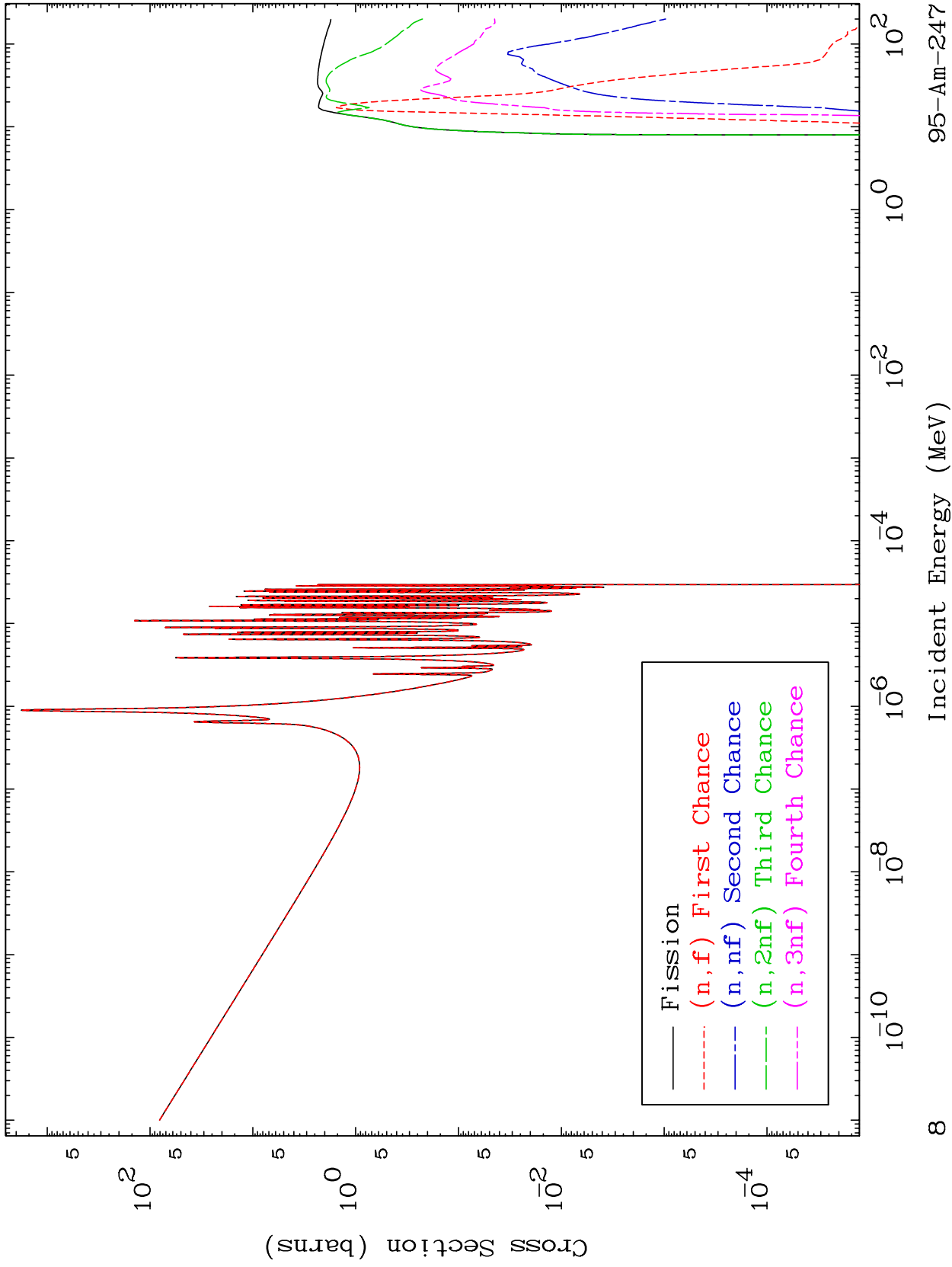
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Fission  
293 Kelvin Cross Sections

95-Am-247

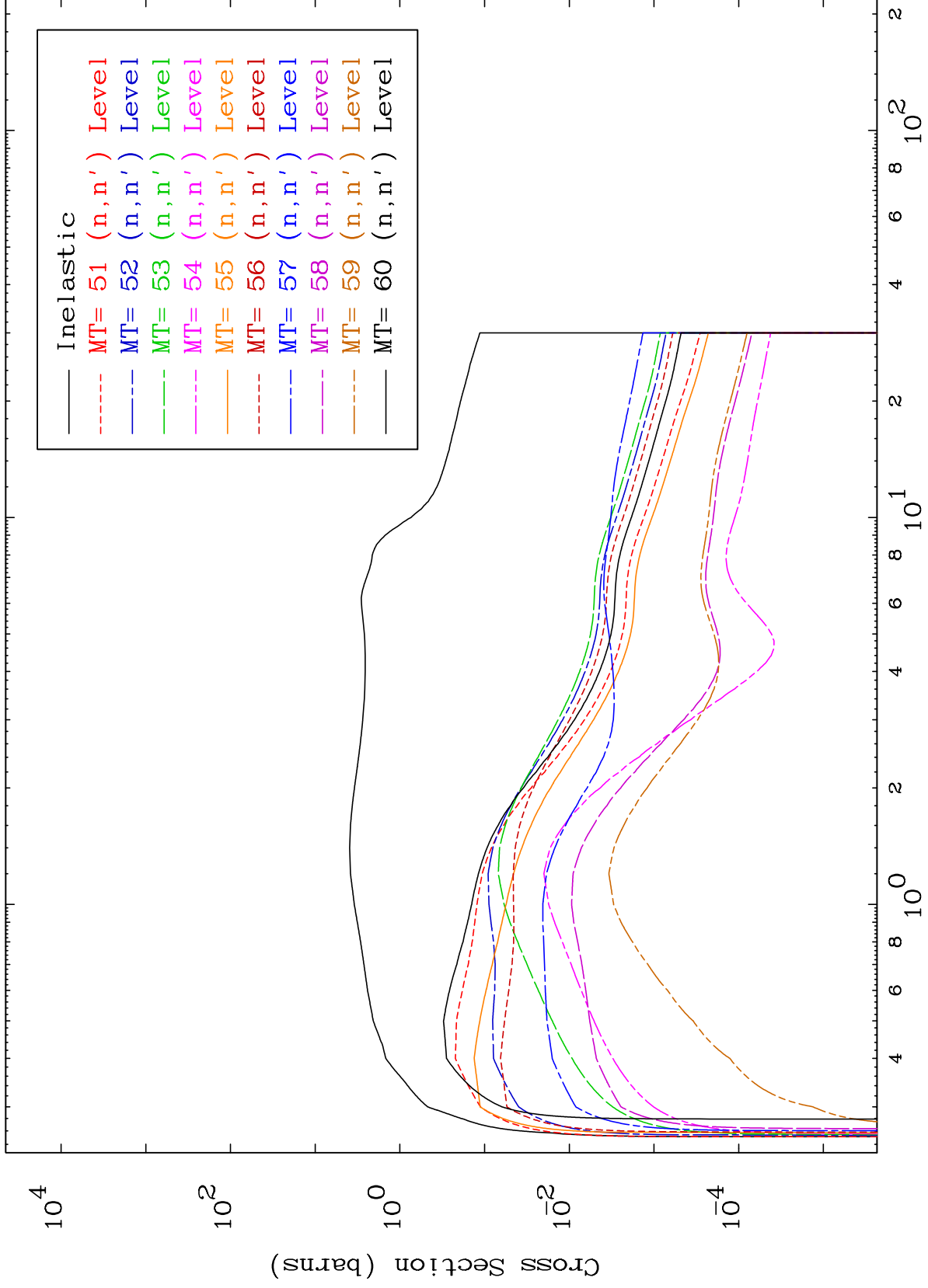


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(n,n') Levels

95-Am-247

293 Kelvin Cross Sections

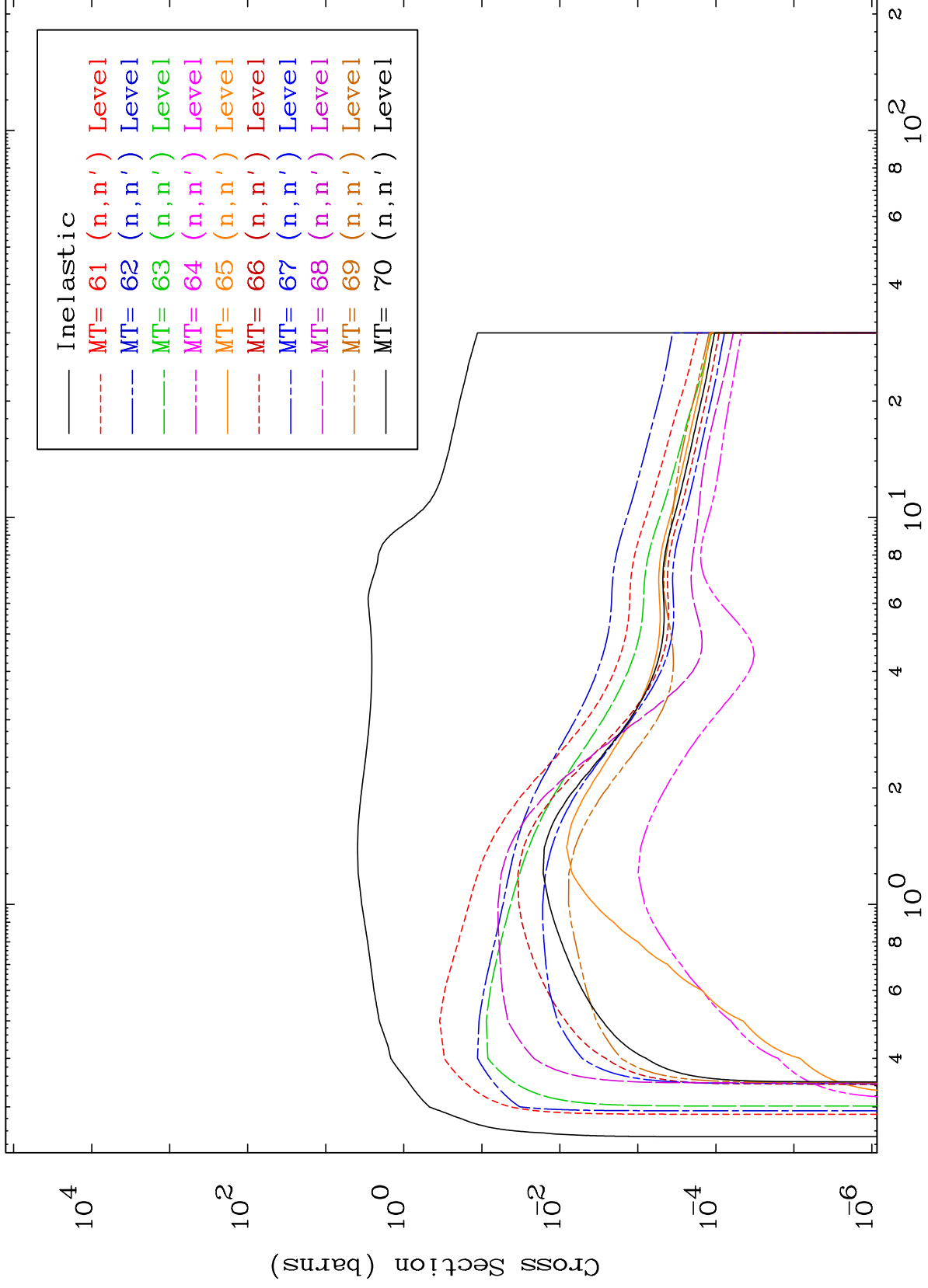


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(n,n') Levels

95-Am-247

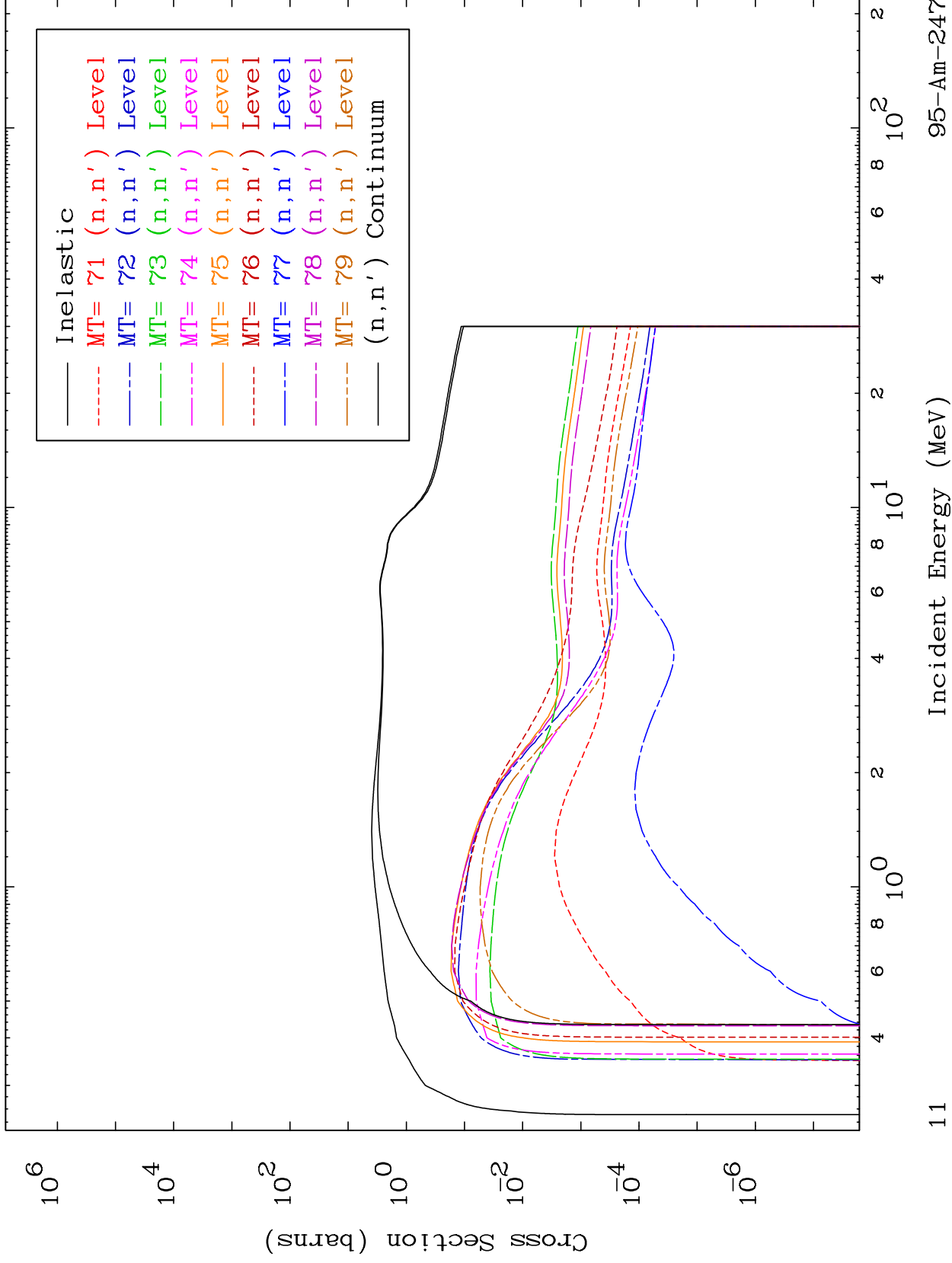
293 Kelvin Cross Sections



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Incident Energy (MeV)

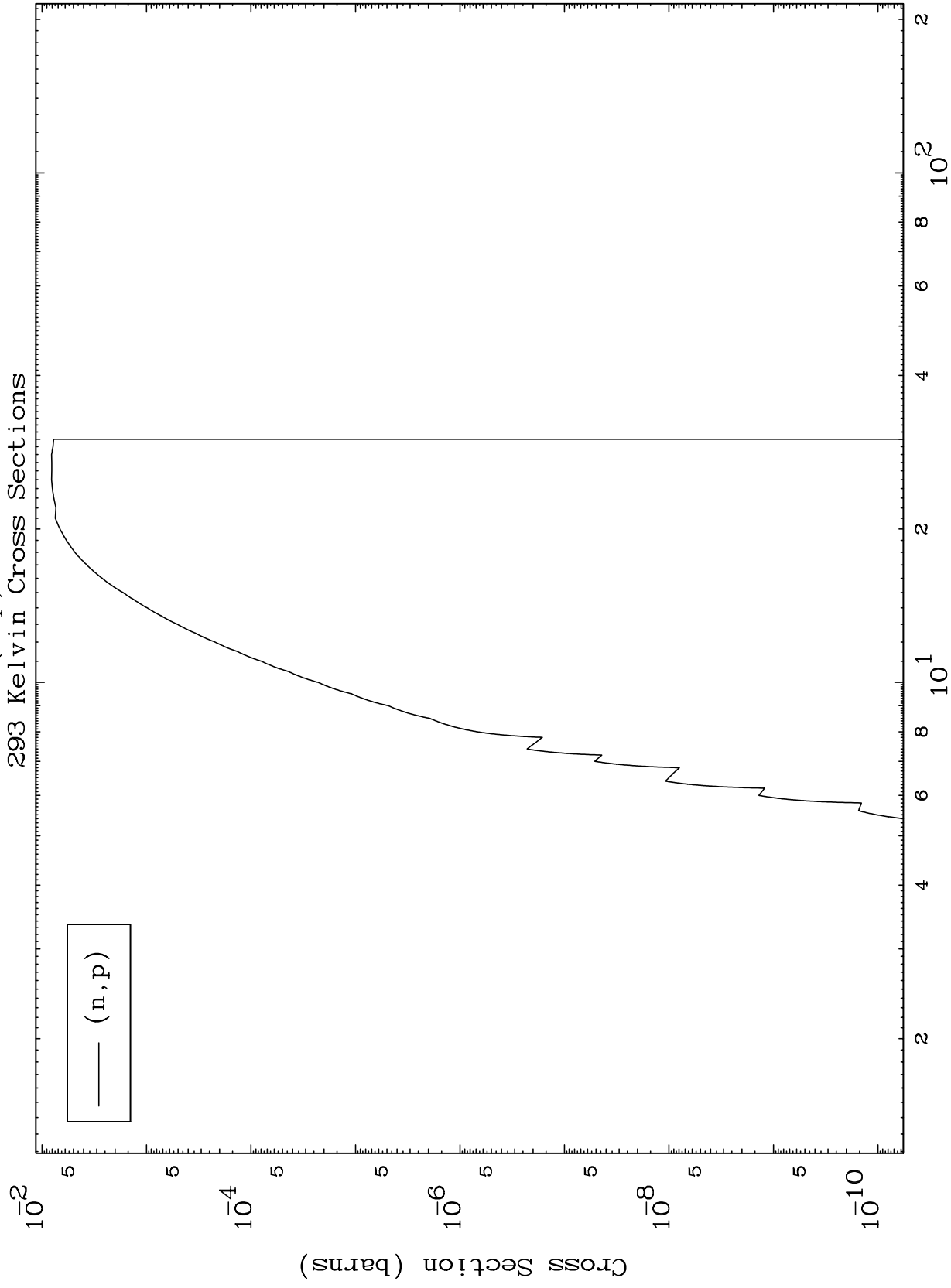
95-Am-247



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95-Am-247

(n,p) Levels  
293 Kelvin Cross Sections



95-Am-247

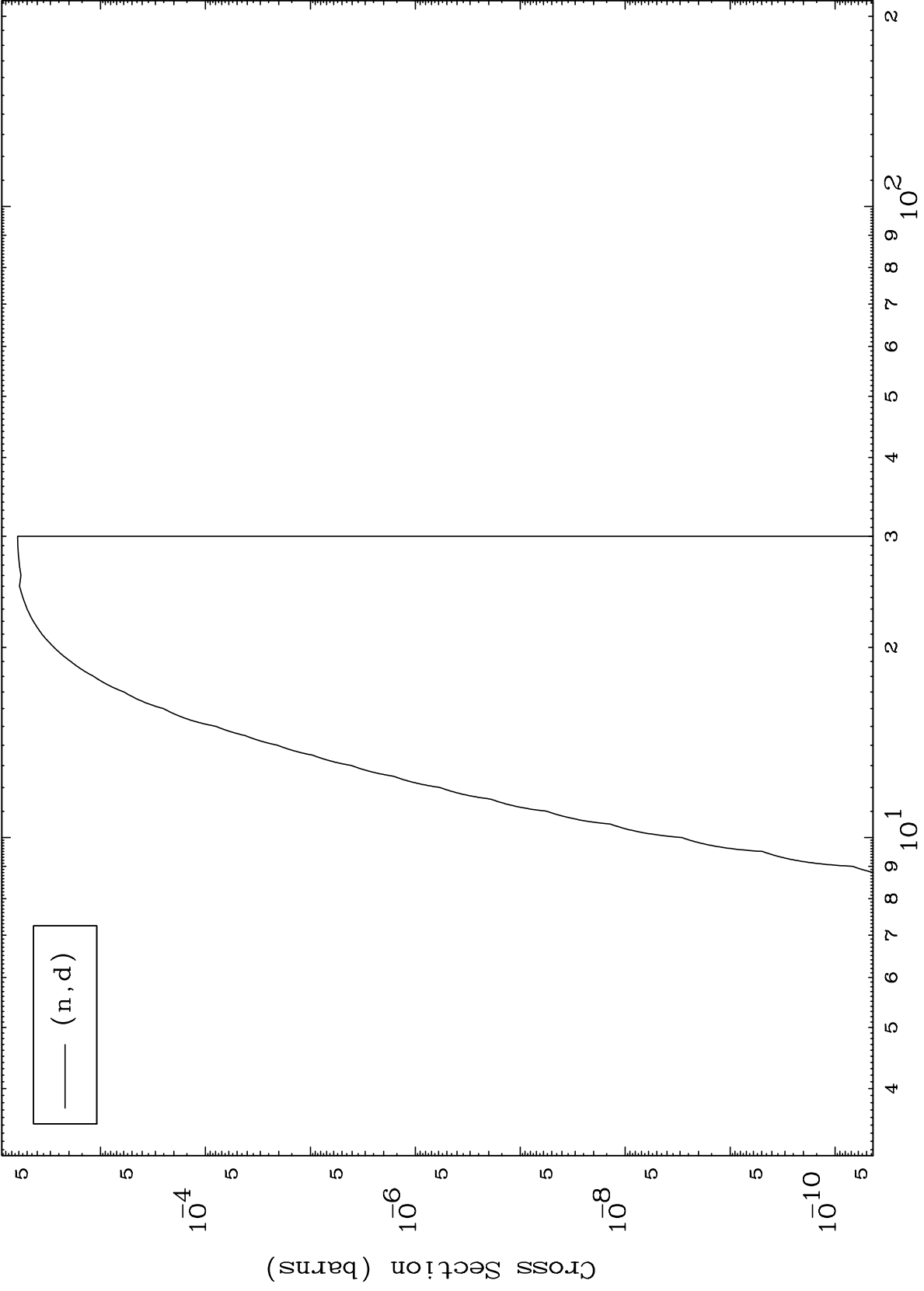
Incident Energy (MeV)

12

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(n,d) Levels  
293 Kelvin Cross Sections

95-Am-247



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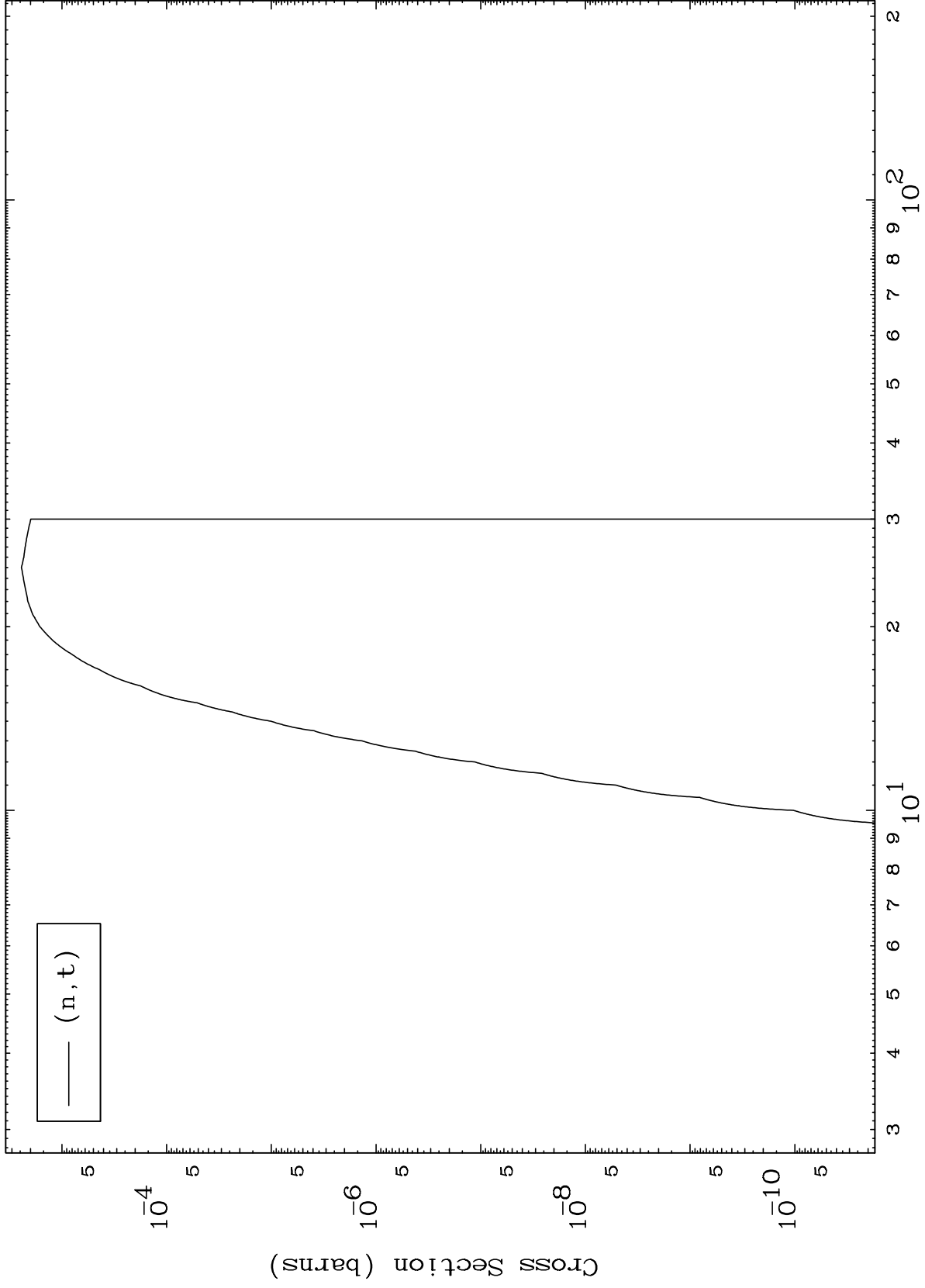
Incident Energy (MeV)

95-Am-247

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(n,t) Levels  
293 Kelvin Cross Sections

95-Am-247



14

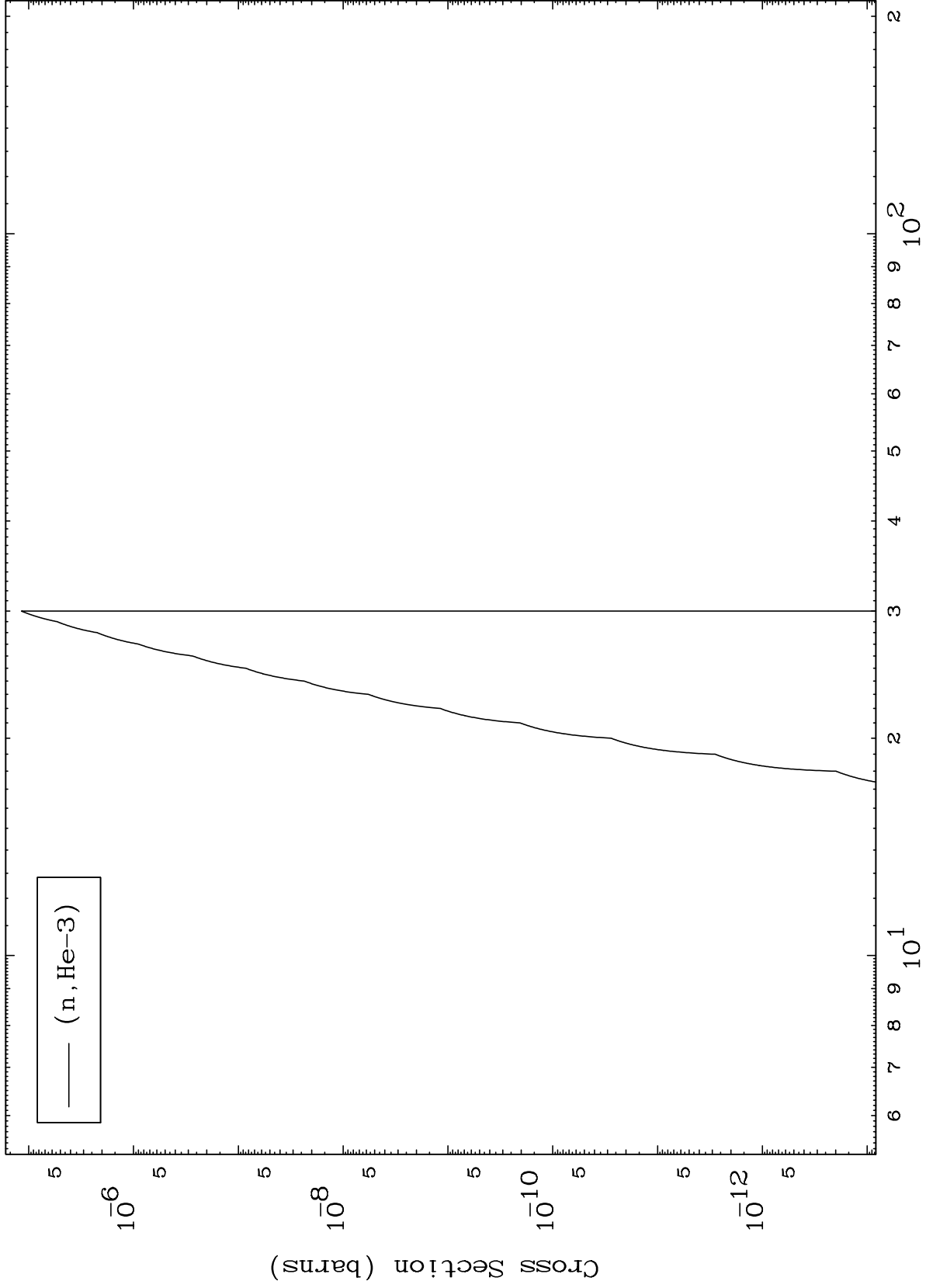
Incident Energy (MeV)

95-Am-247

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(n,He3) Levels  
293 Kelvin Cross Sections

95-Am-247



15

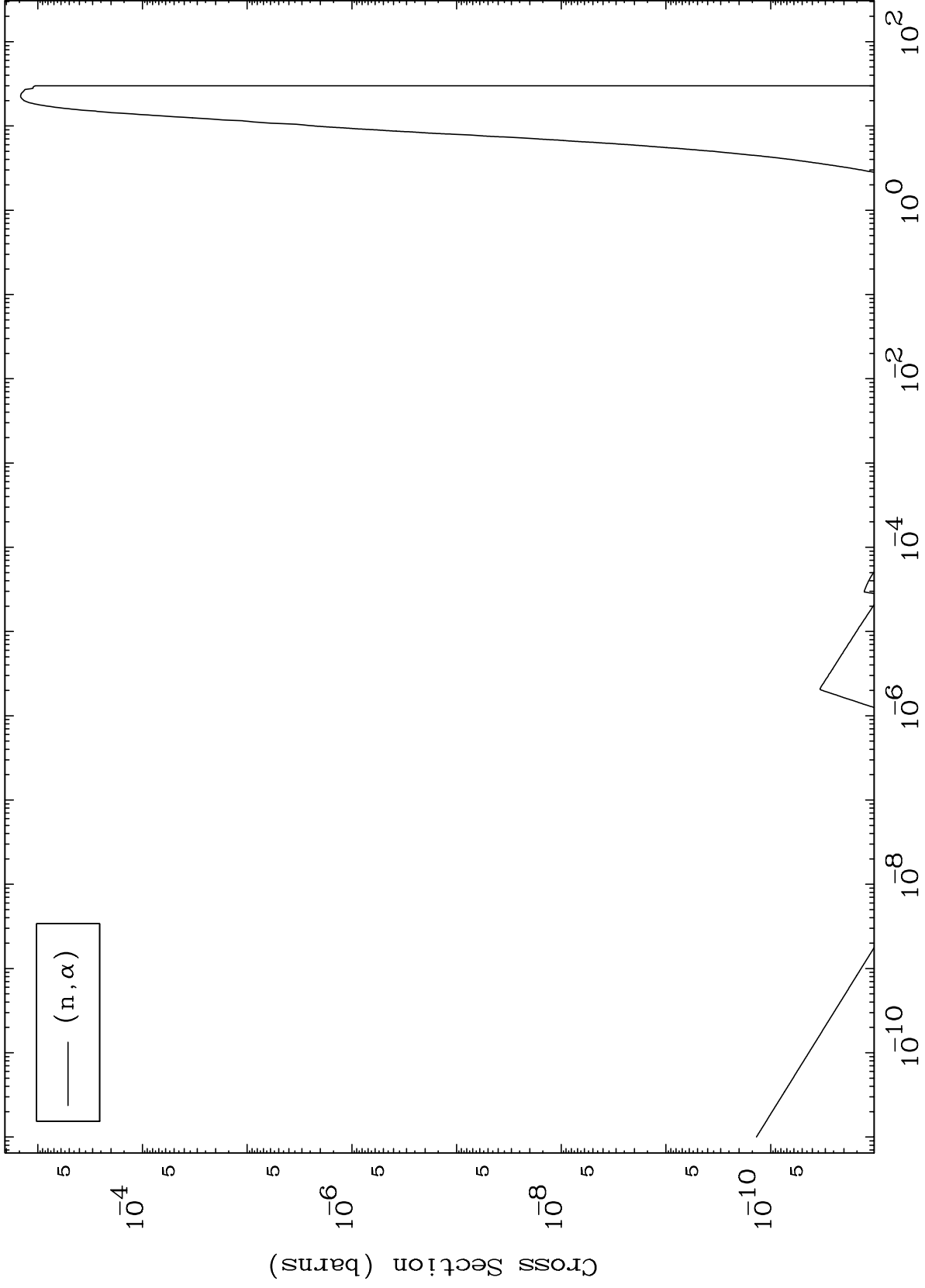
Incident Energy (MeV)

95-Am-247

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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

95-Am-247



16

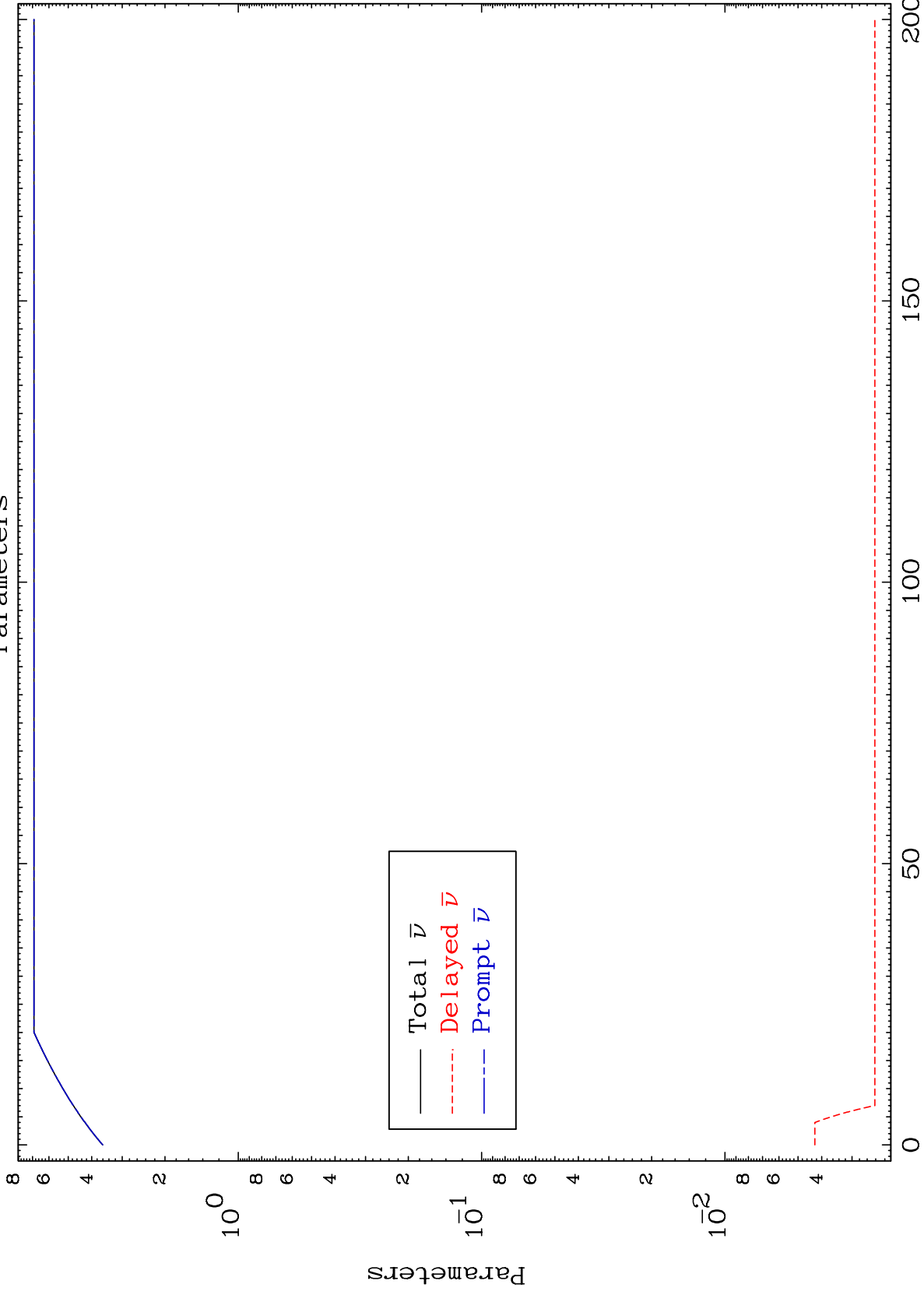
Incident Energy (MeV)

95-Am-247

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Energy Release  
Parameters

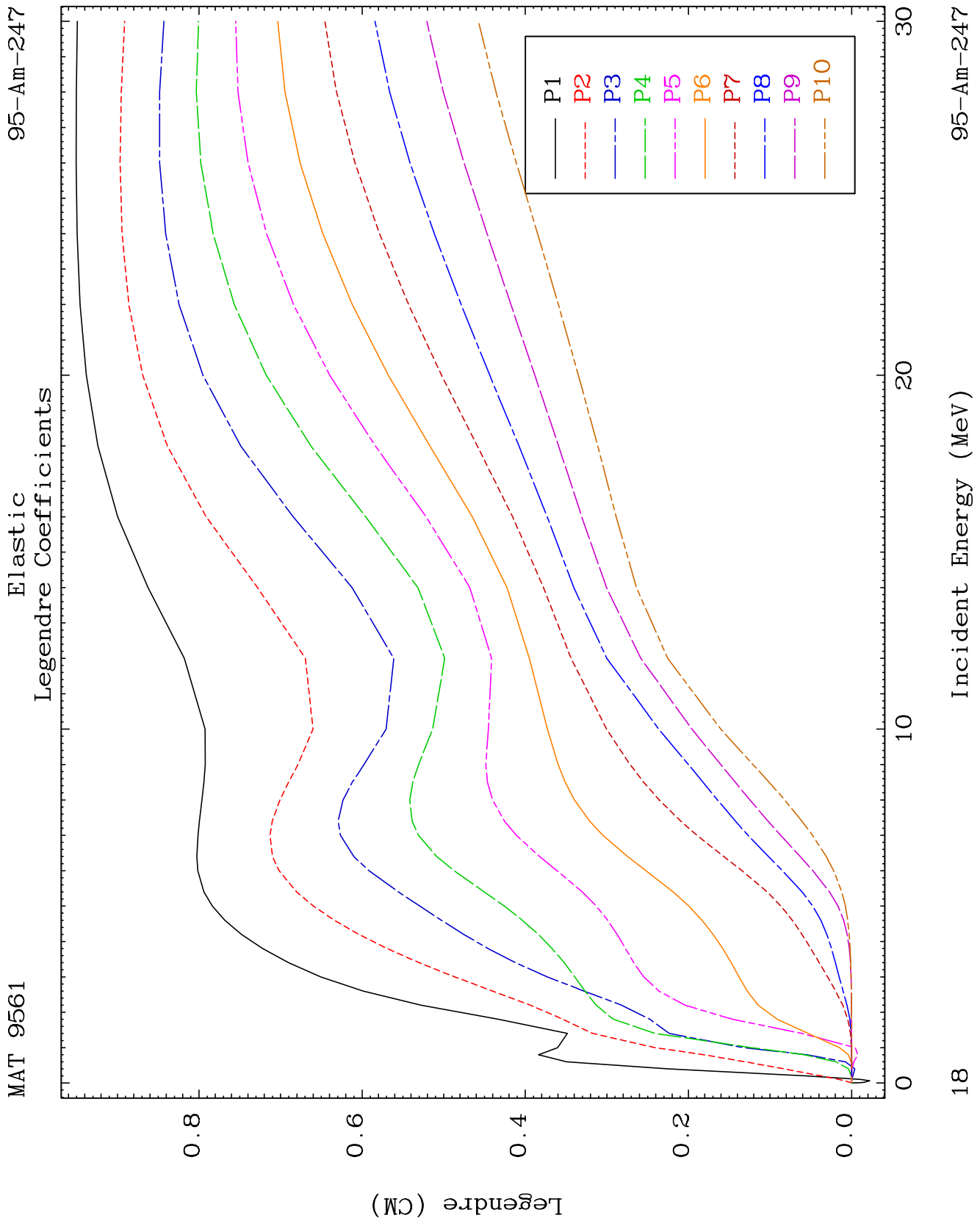
95-Am-247

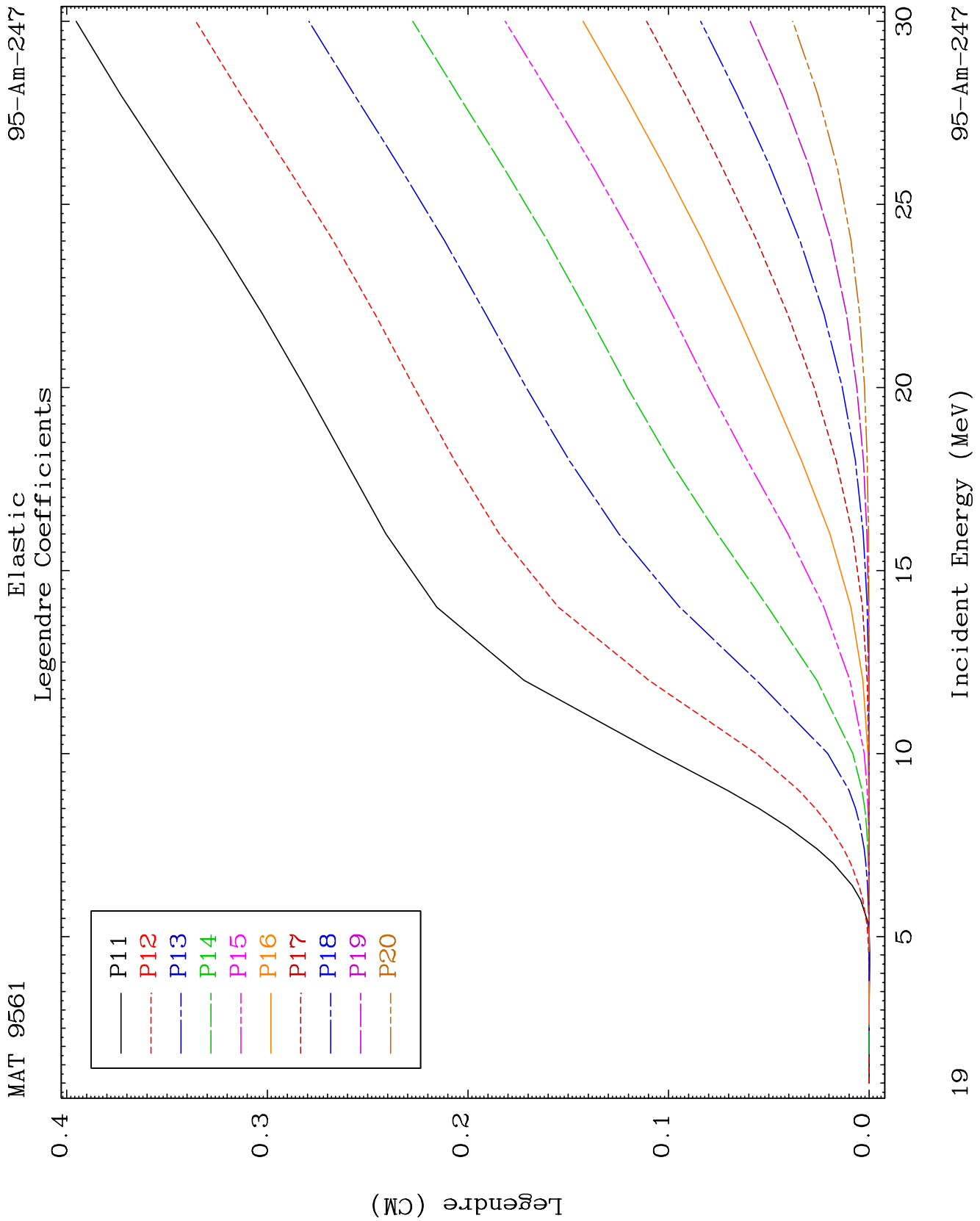


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Incident Energy (MeV)

95-Am-247

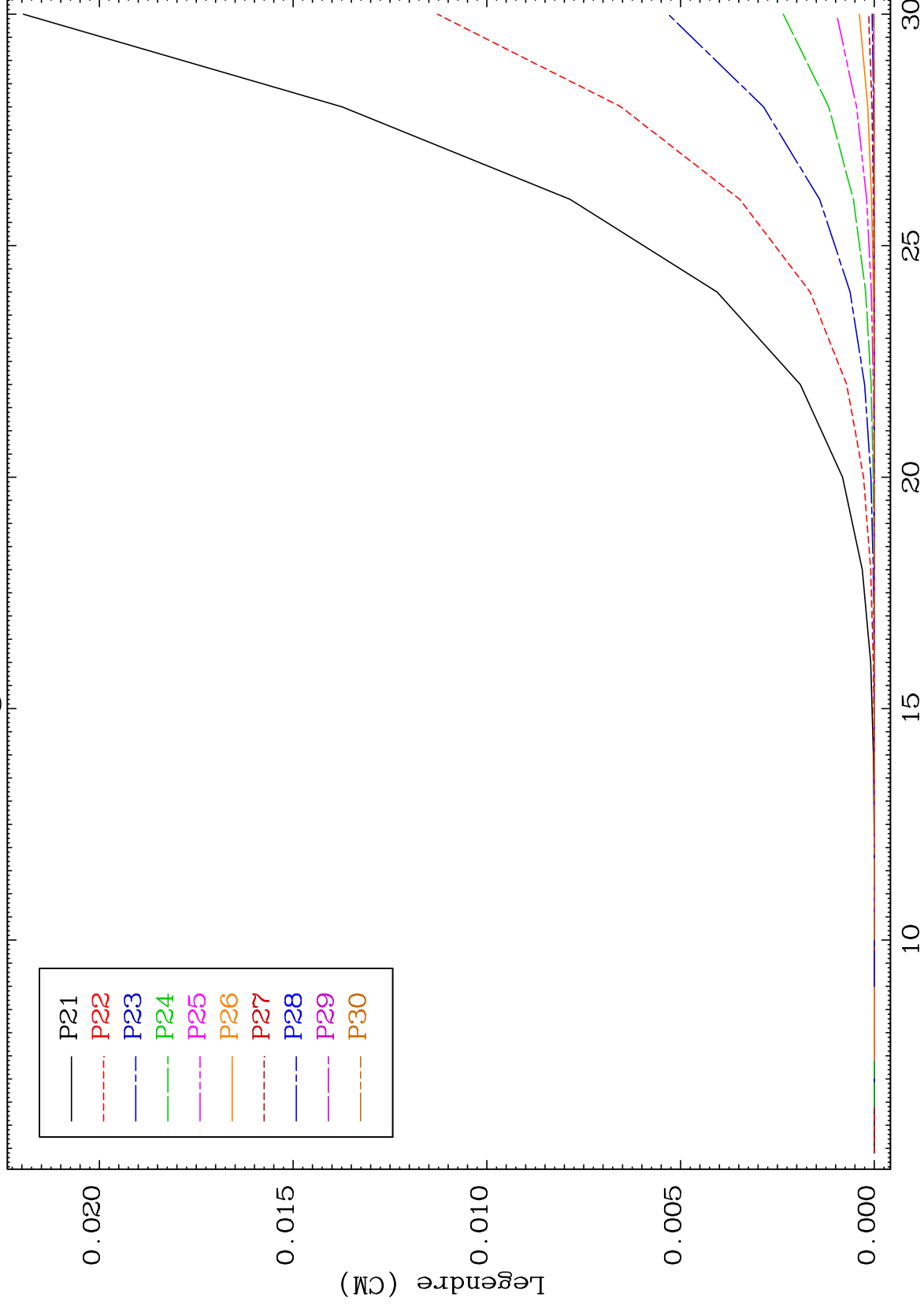




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### Elastic Legendre Coefficients

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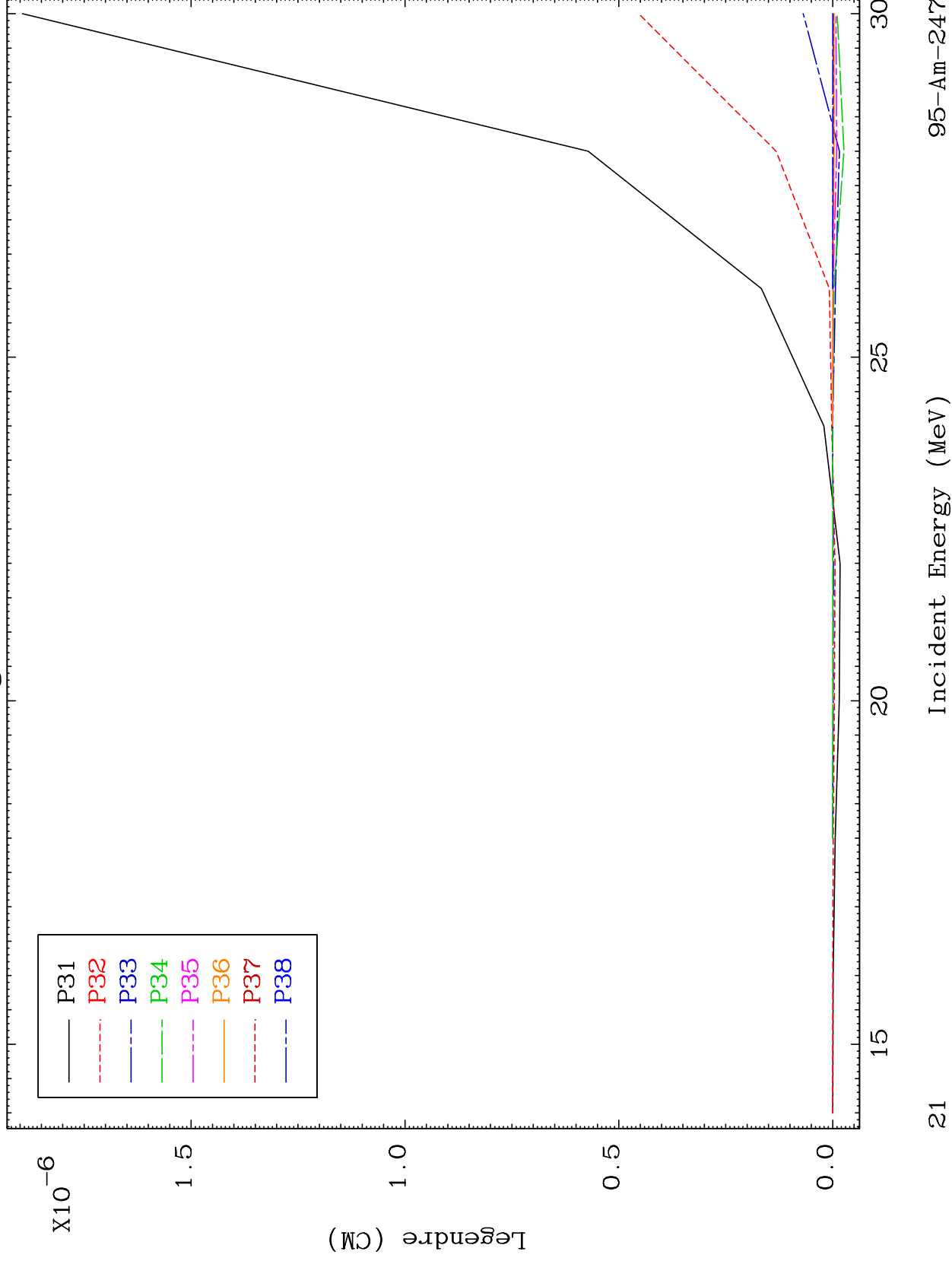
Incident Energy (MeV)

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Elastic Legendre Coefficients

95-Am-247



21

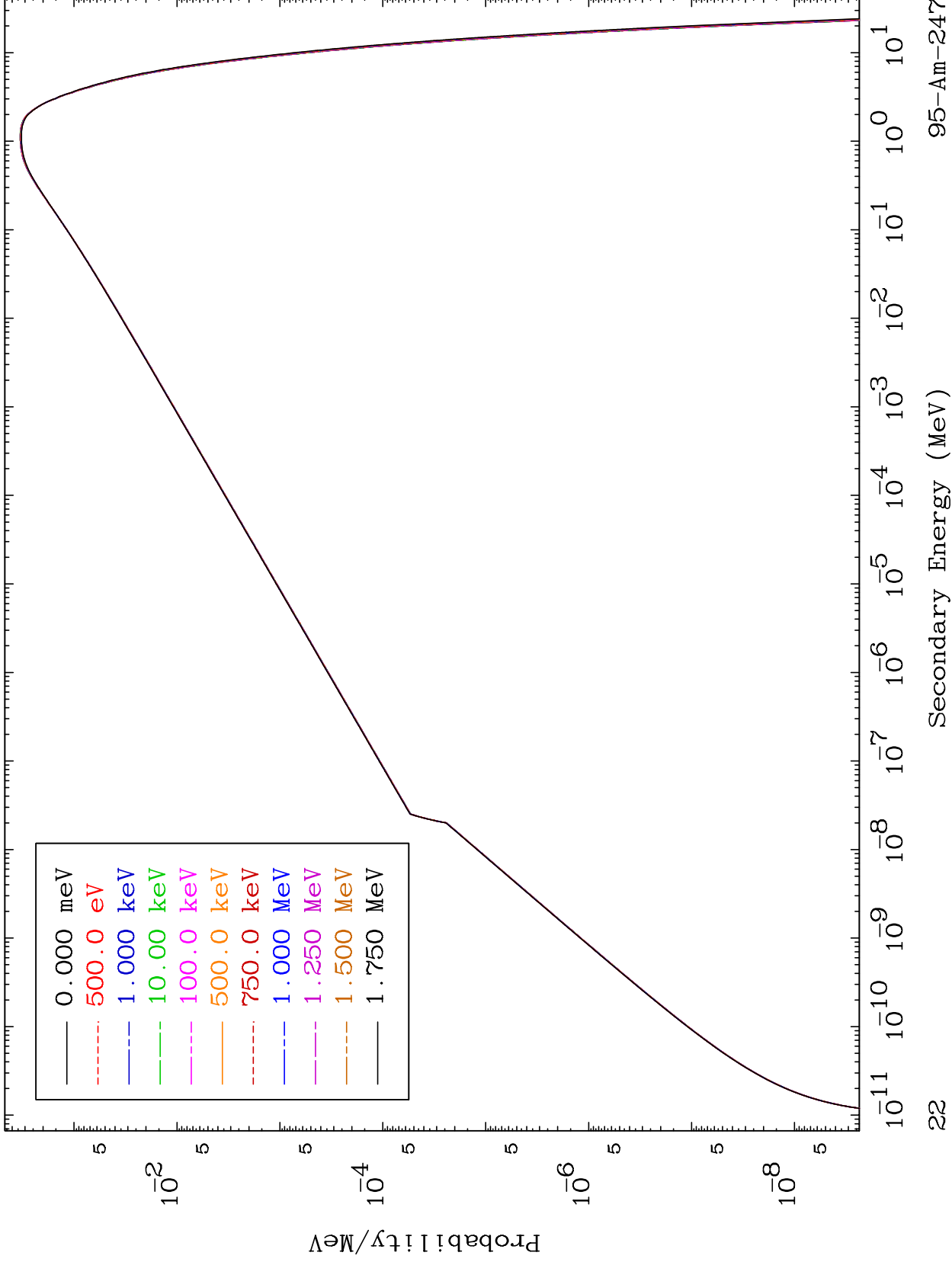
Incident Energy (MeV)

95-Am-247

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### Fission Energy Distributions

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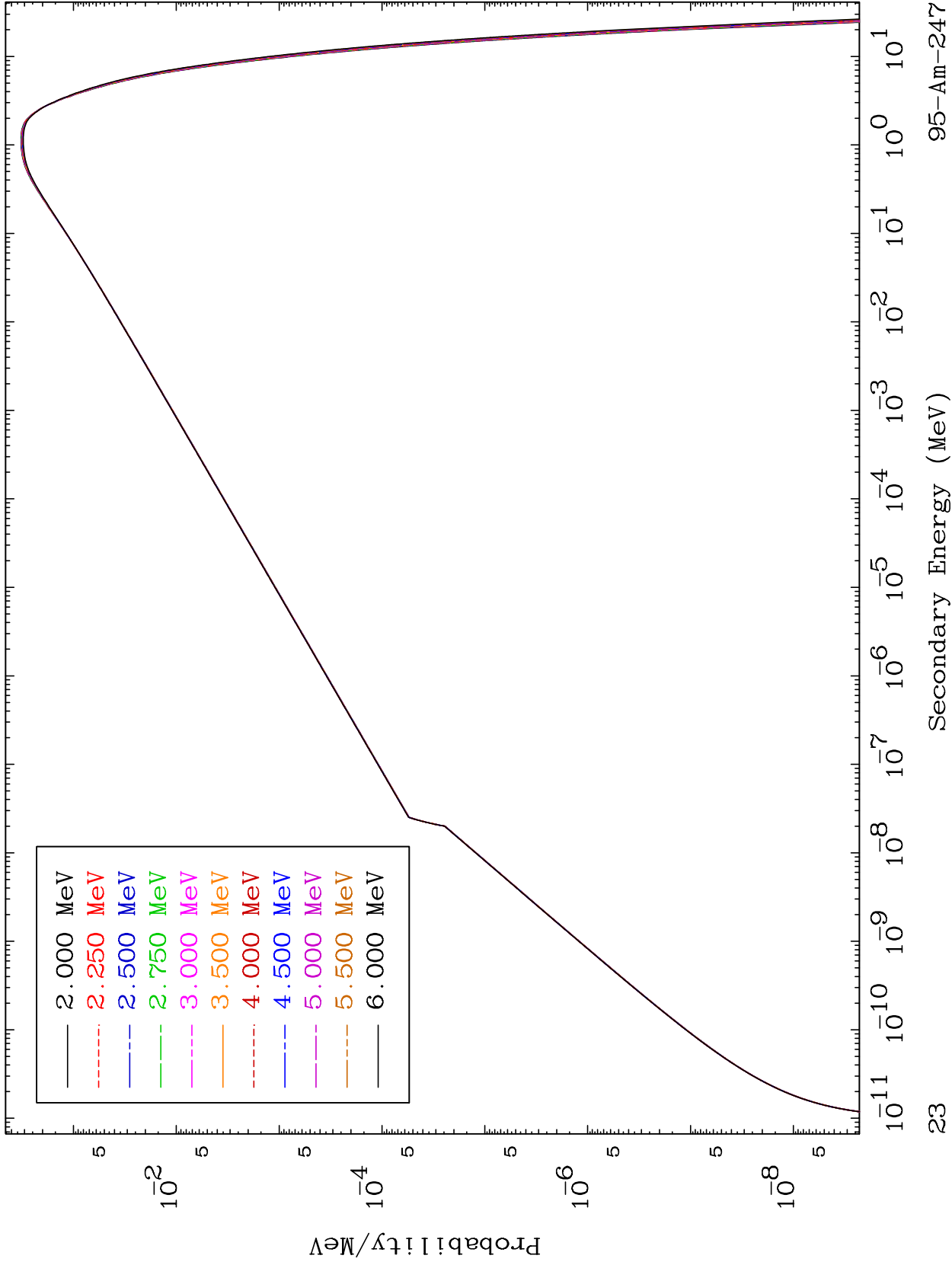
Secondary Energy (MeV)

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### Fission Energy Distributions

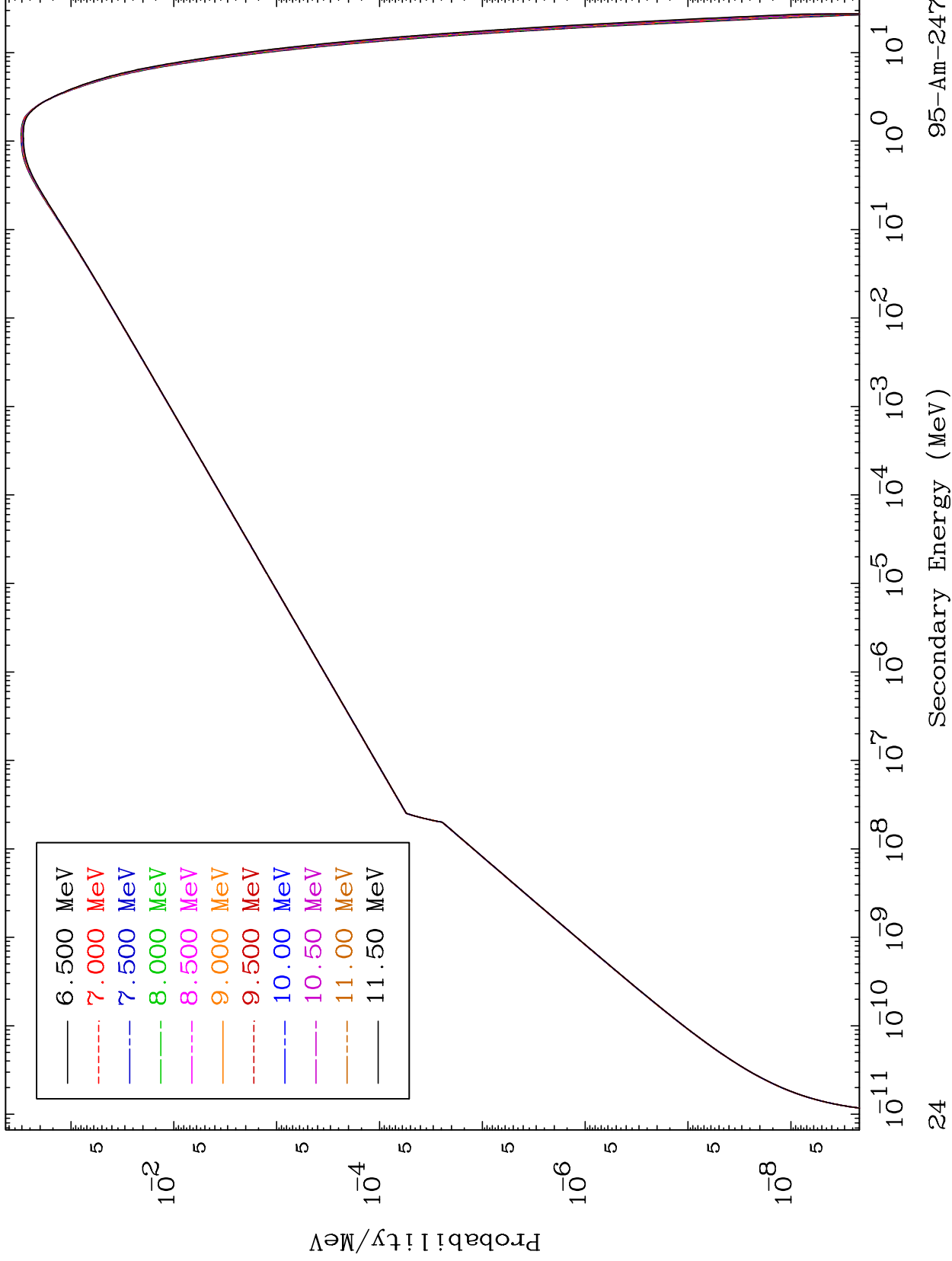
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### Fission Energy Distributions

95-Am-247

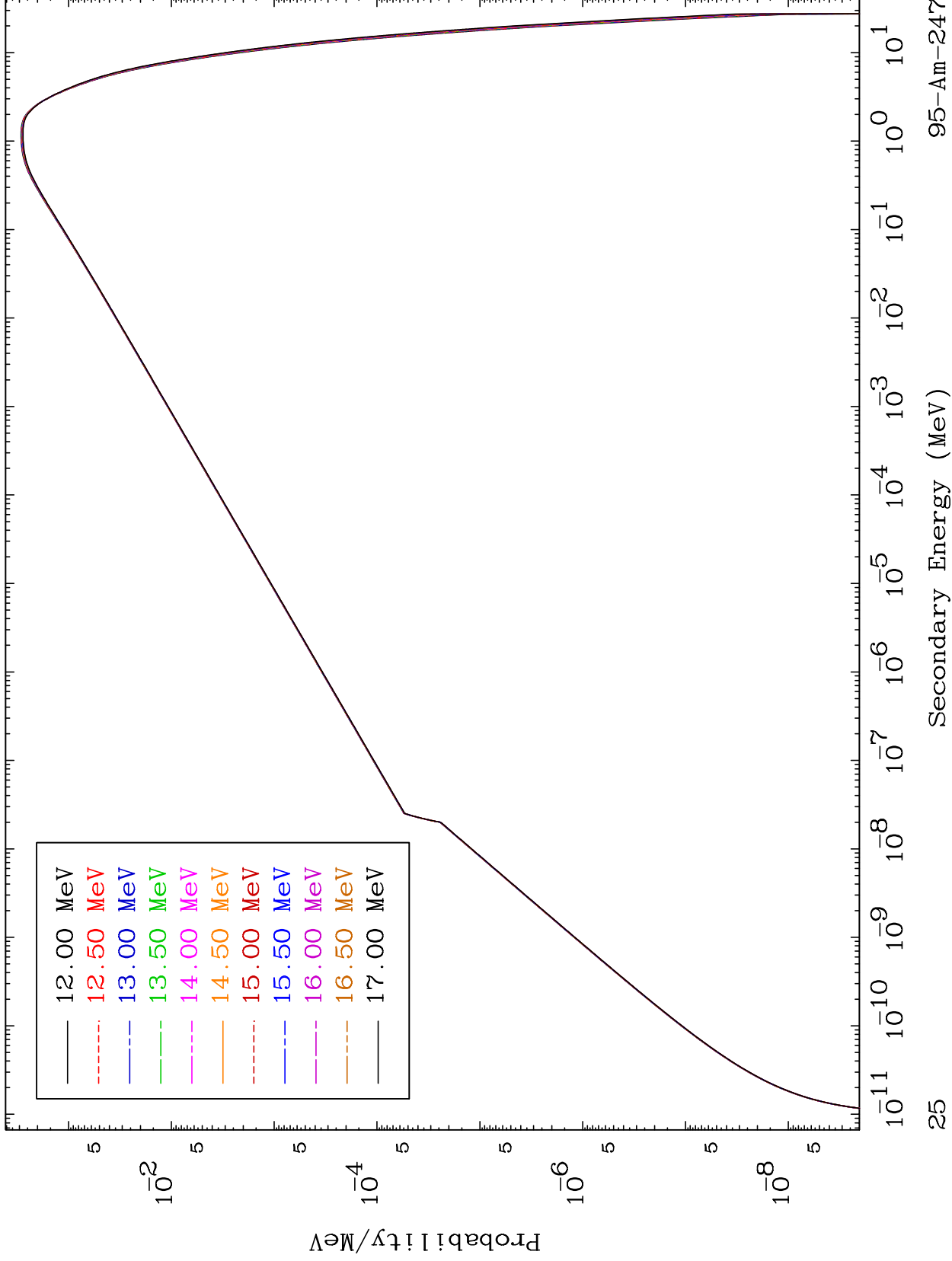


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### Fission Energy Distributions

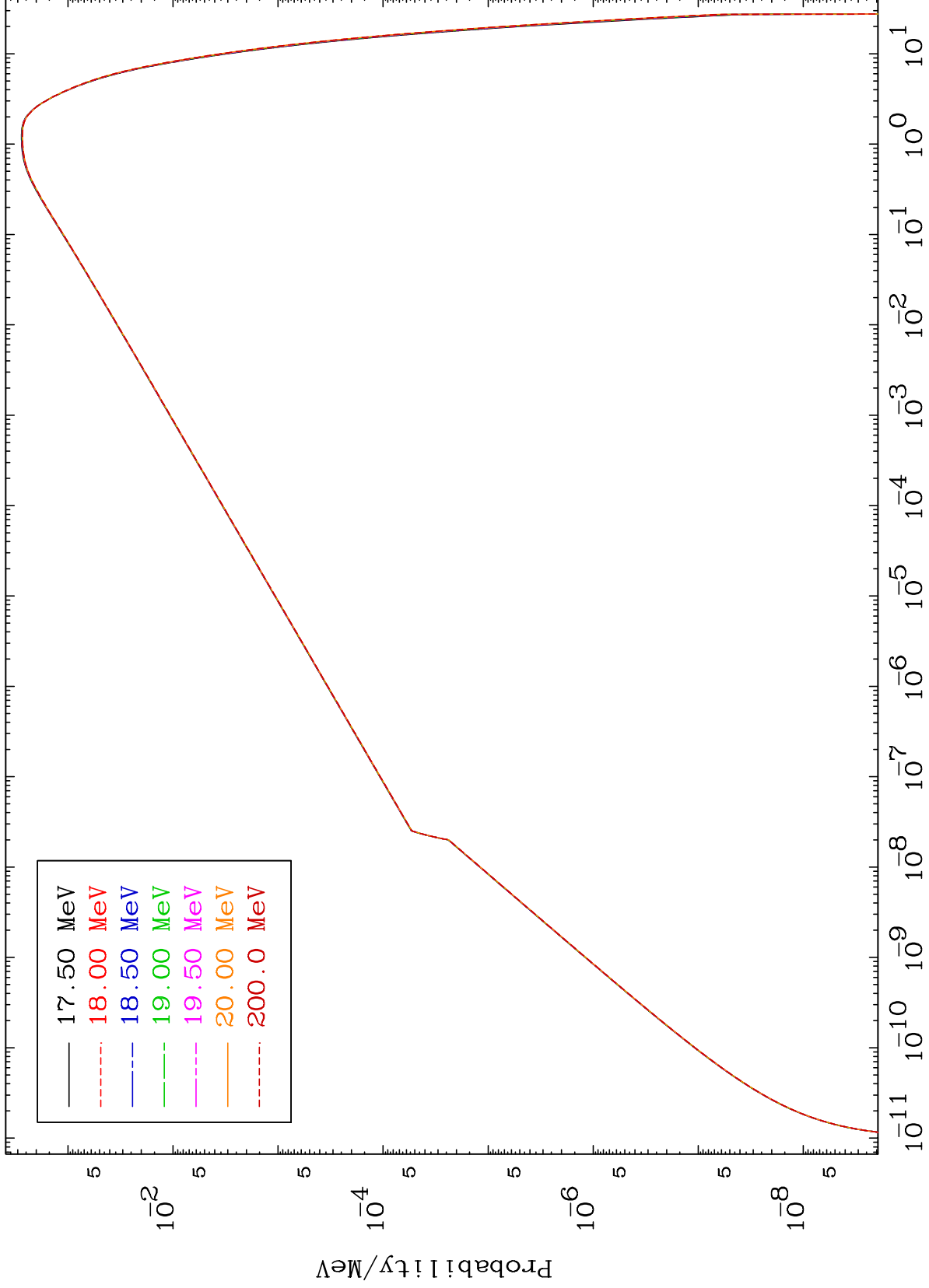
95-Am-247



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### Fission Energy Distributions

95-Am-247



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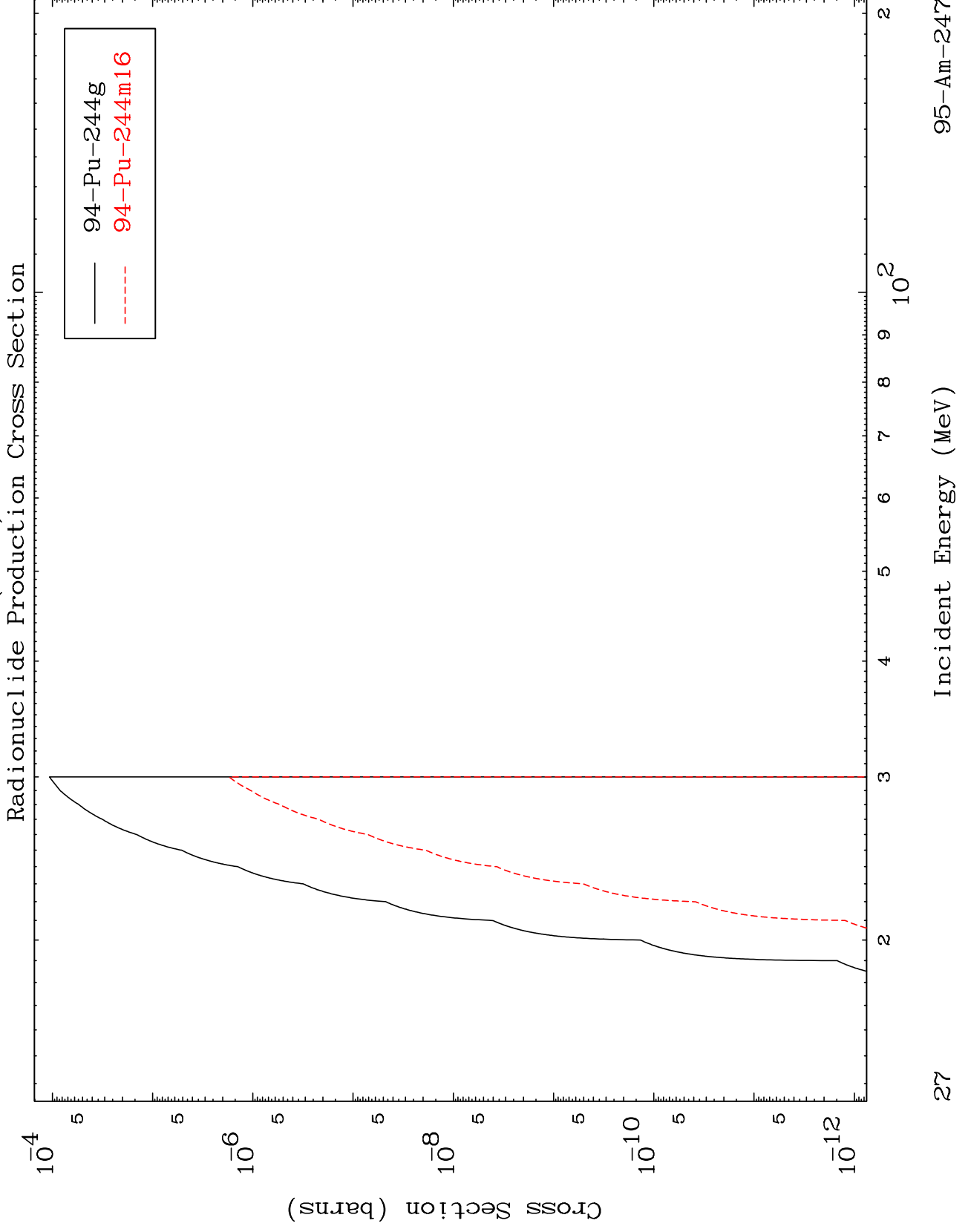
Secondary Energy (MeV)

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(n,2n) d

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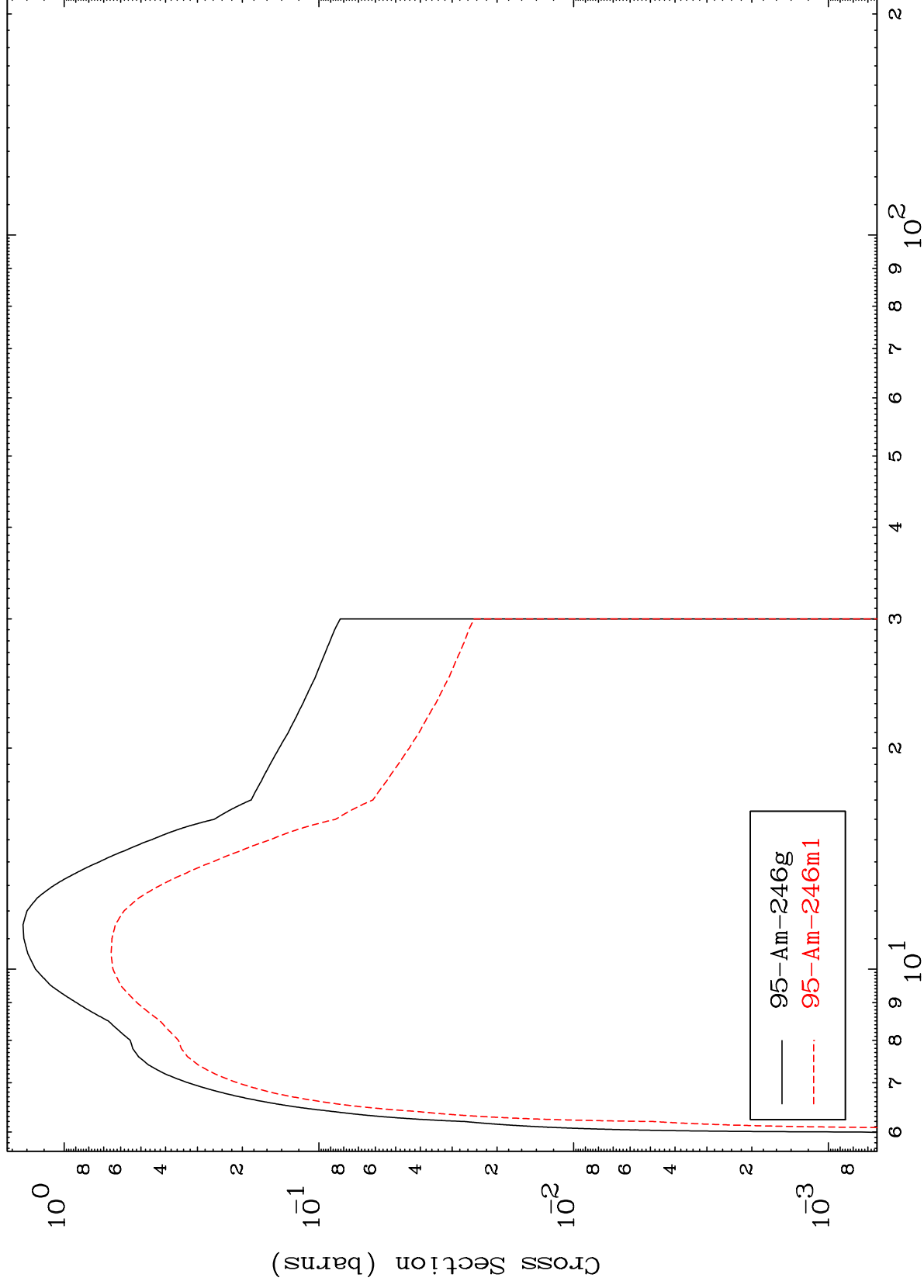
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95-Am-247

(n,2n)  
Radionuclide Production Cross Section



28

Incident Energy (MeV)

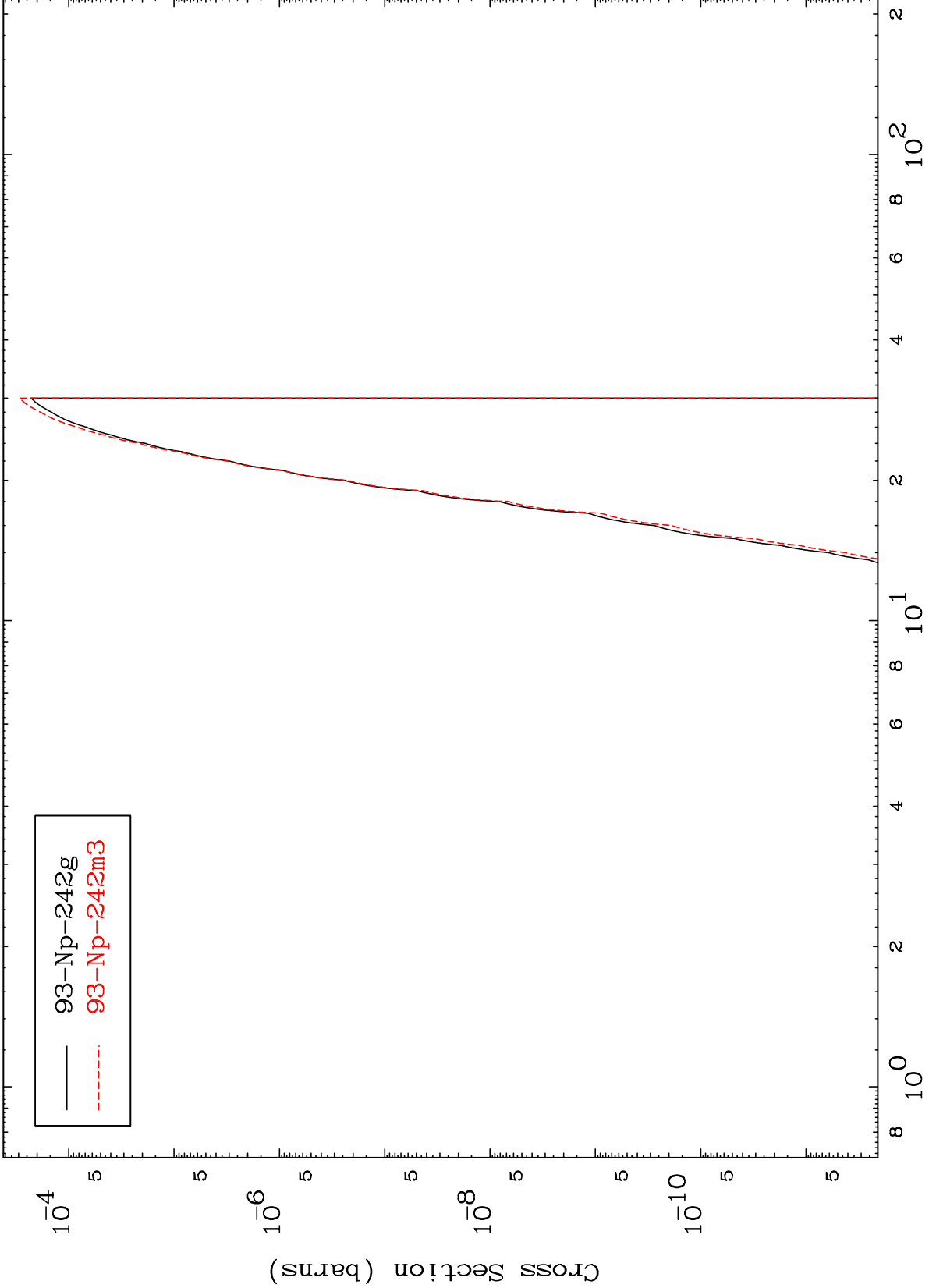
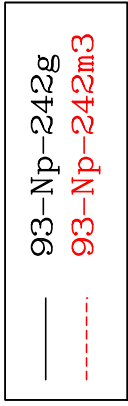
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(n,2n)  $\alpha$

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Radionuclide Production Cross Section



29

Incident Energy (MeV)

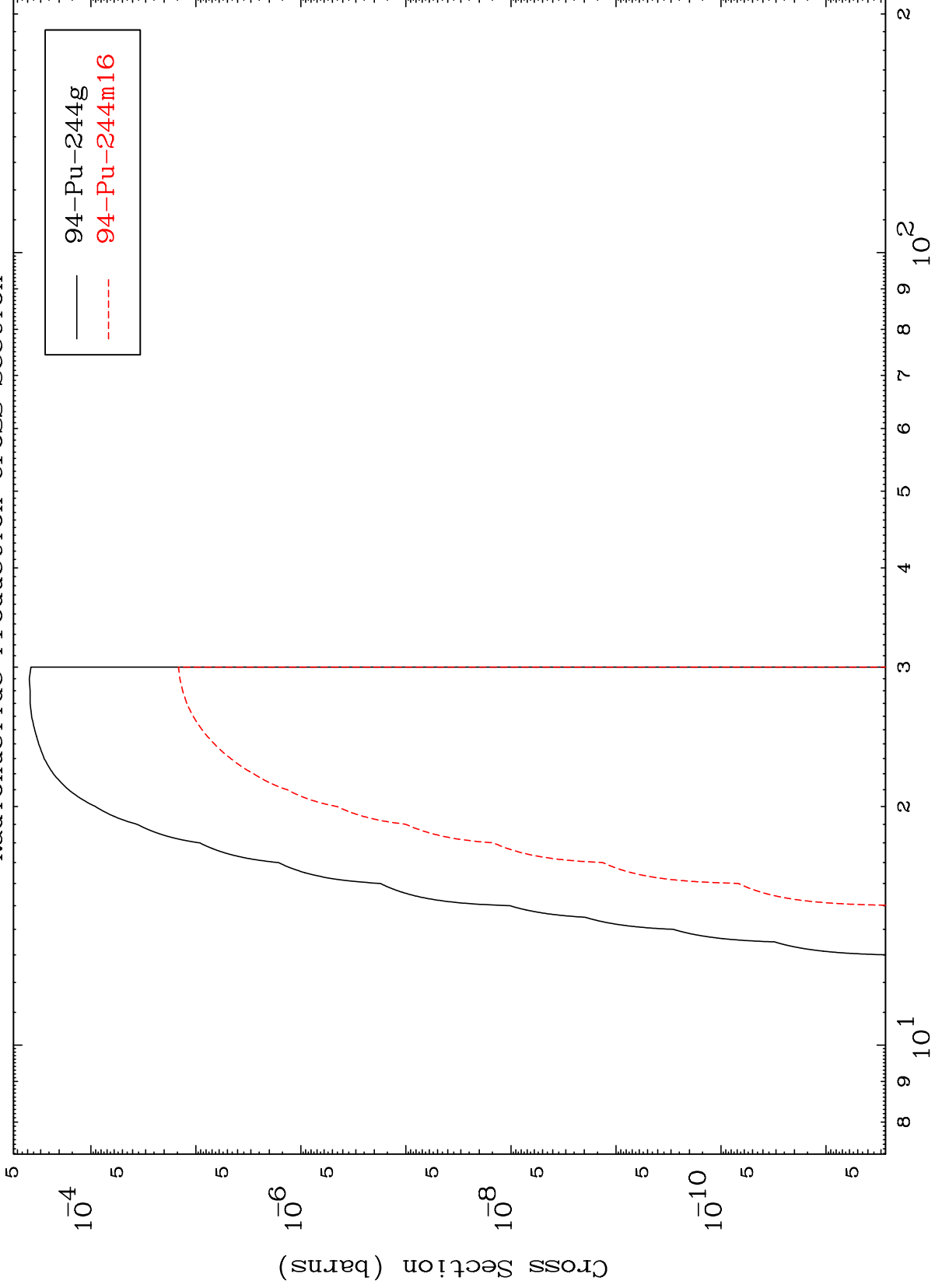
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(n,n') t

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Radionuclide Production Cross Section



30

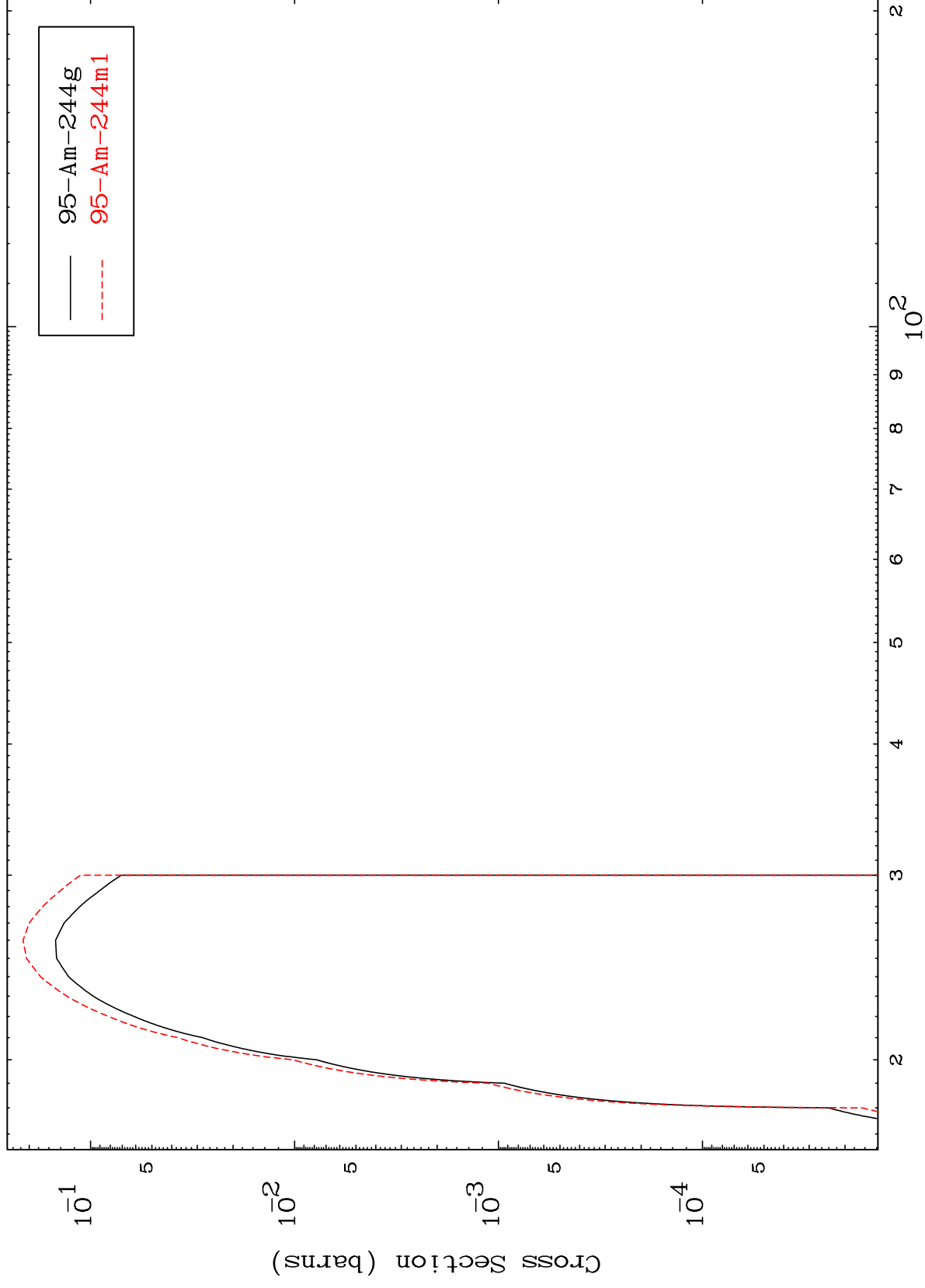
Incident Energy (MeV)

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(n,4n)  
Radionuclide Production Cross Section



31

Incident Energy (MeV)

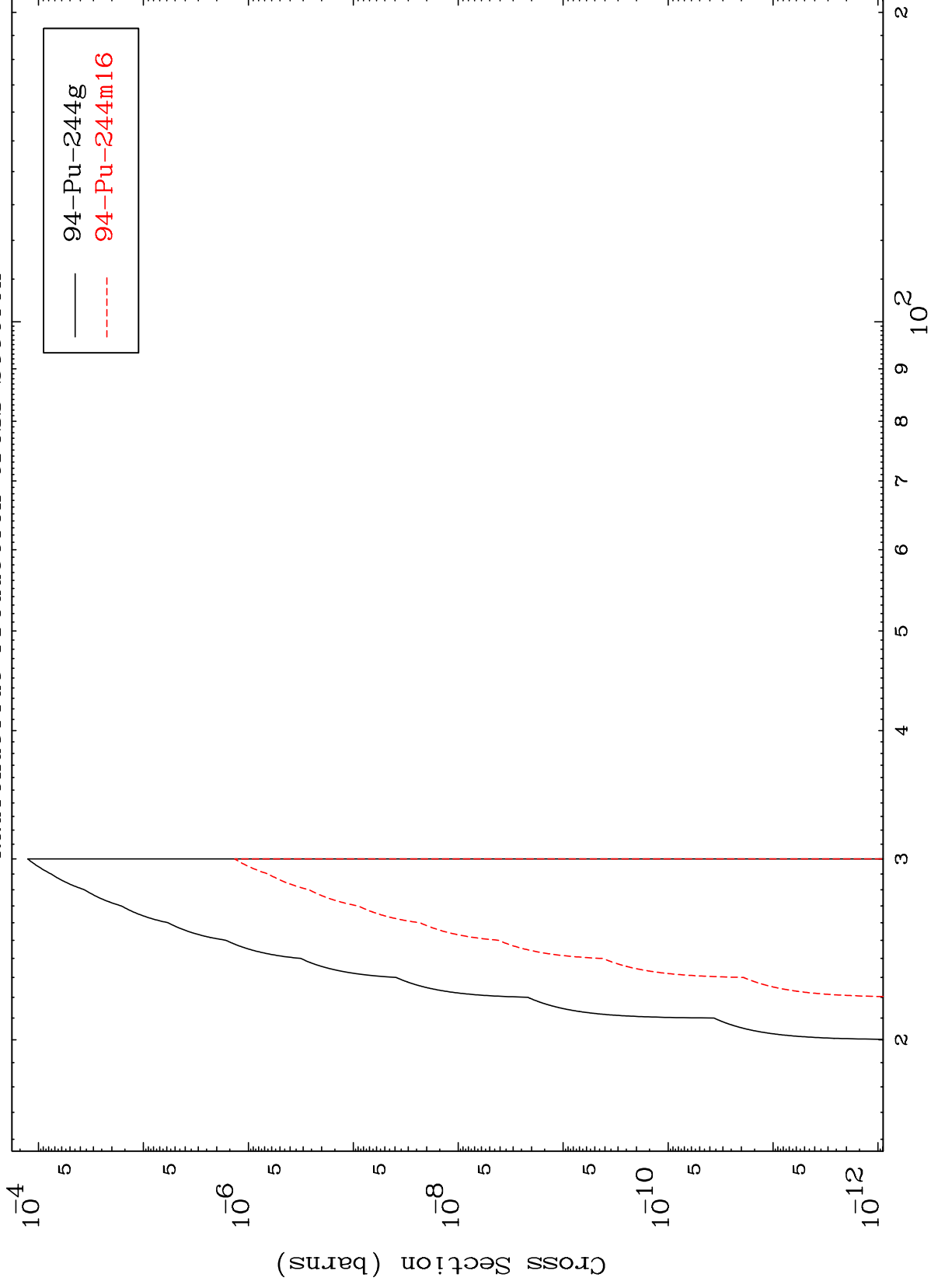
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(n,3n) p

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Radionuclide Production Cross Section



32

Incident Energy (MeV)

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