

Program EVALPLOT  
(Version 2021-1)

by

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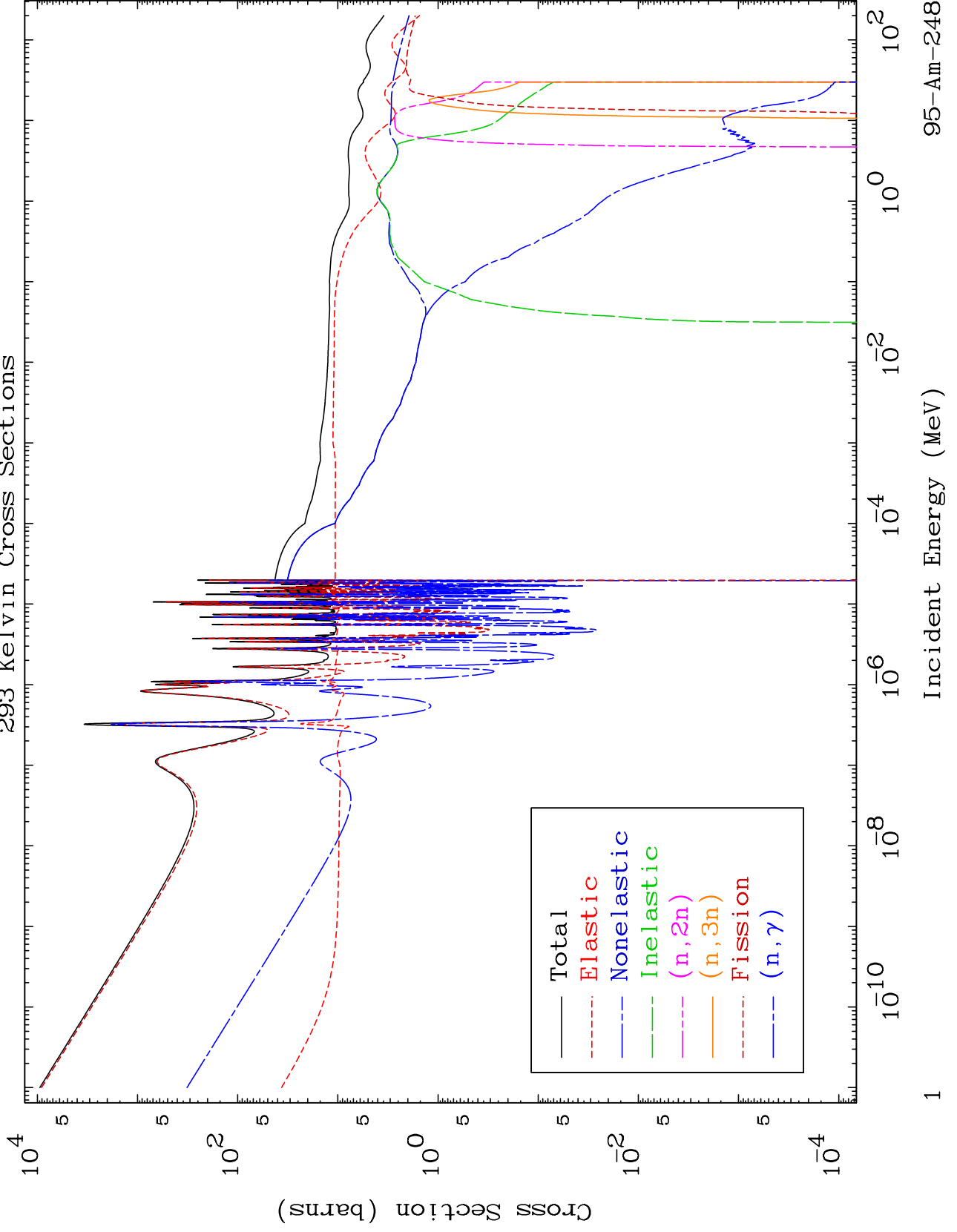
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Press Mouse Button to Start

MAT 9564

Neutron Major  
293 Kelvin Cross Sections

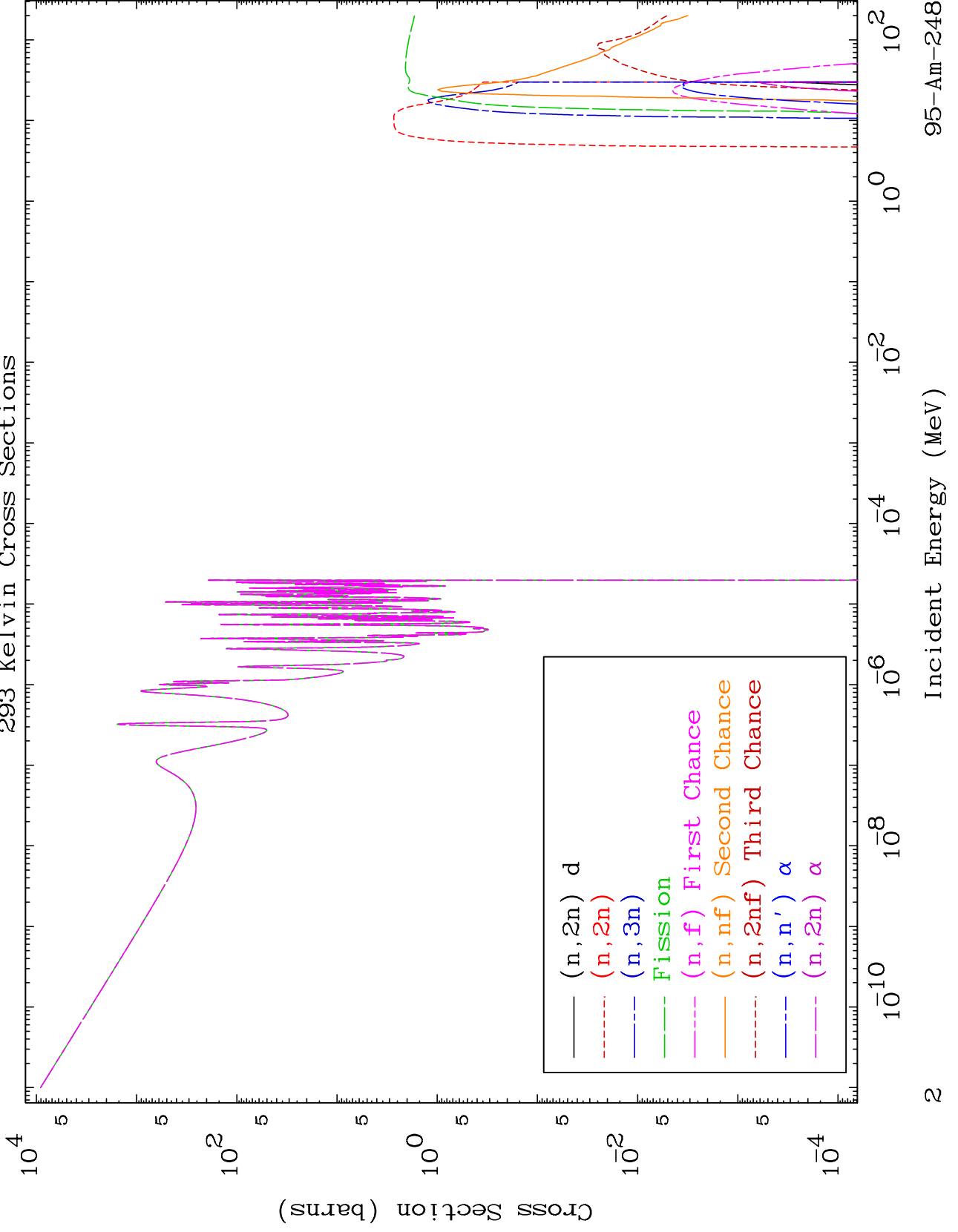
95-Am-248



MAT 9564

Neutron Absorption  
293 Kelvin Cross Sections

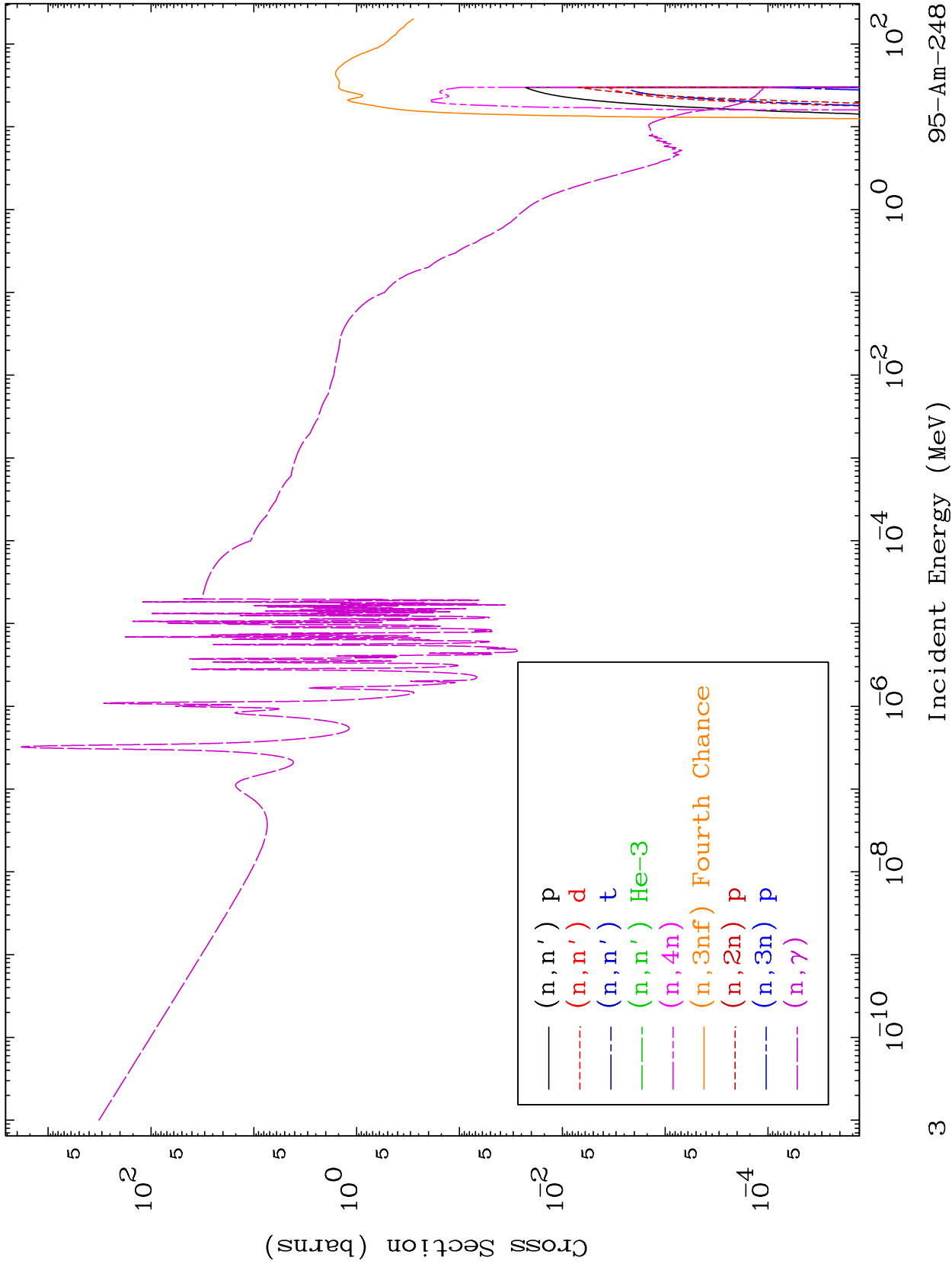
95-Am-248



MAT 9564

Neutron Absorption  
293 Kelvin Cross Sections

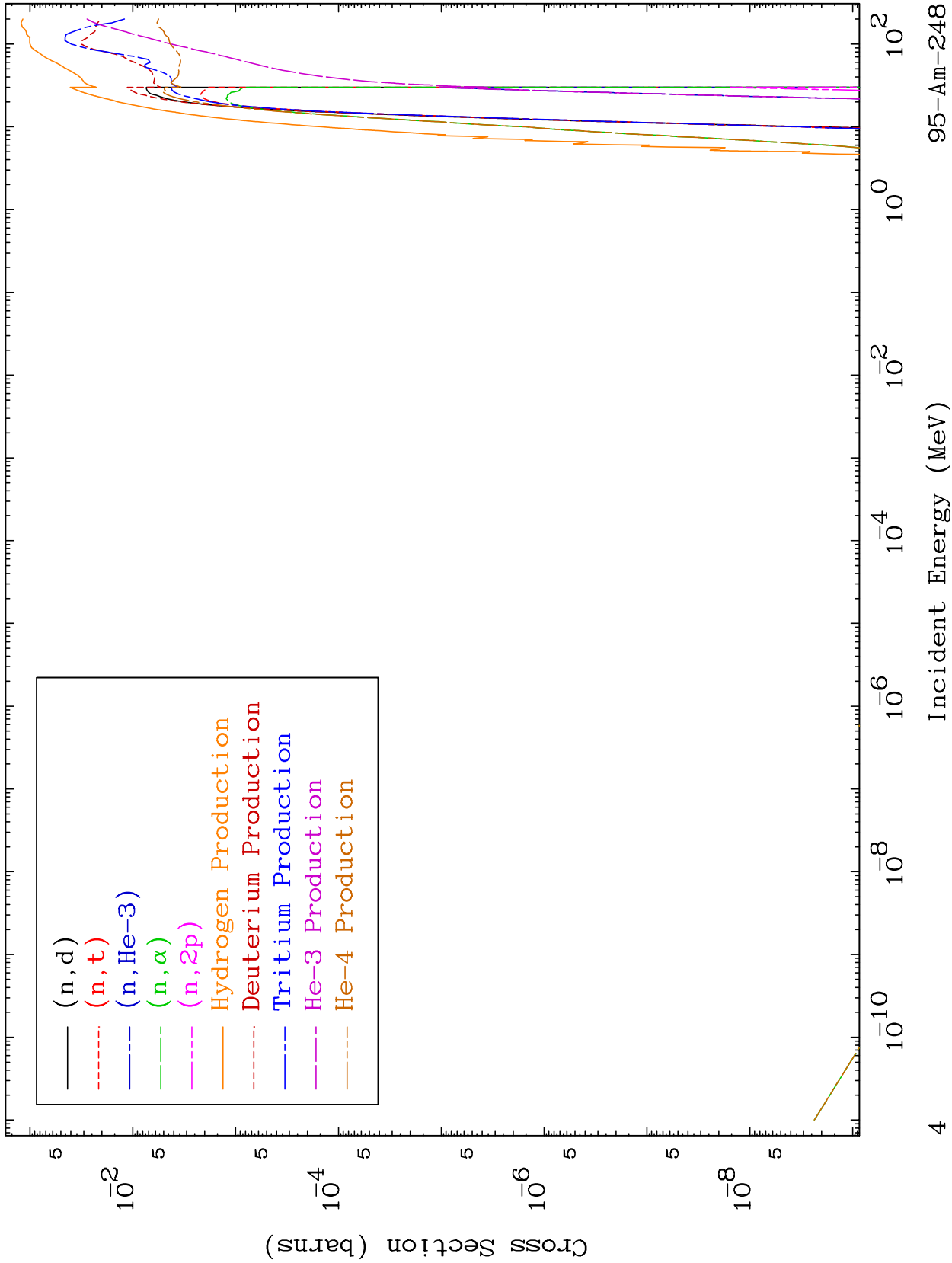
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MAT 9564

Neutron Absorption  
293 Kelvin Cross Sections

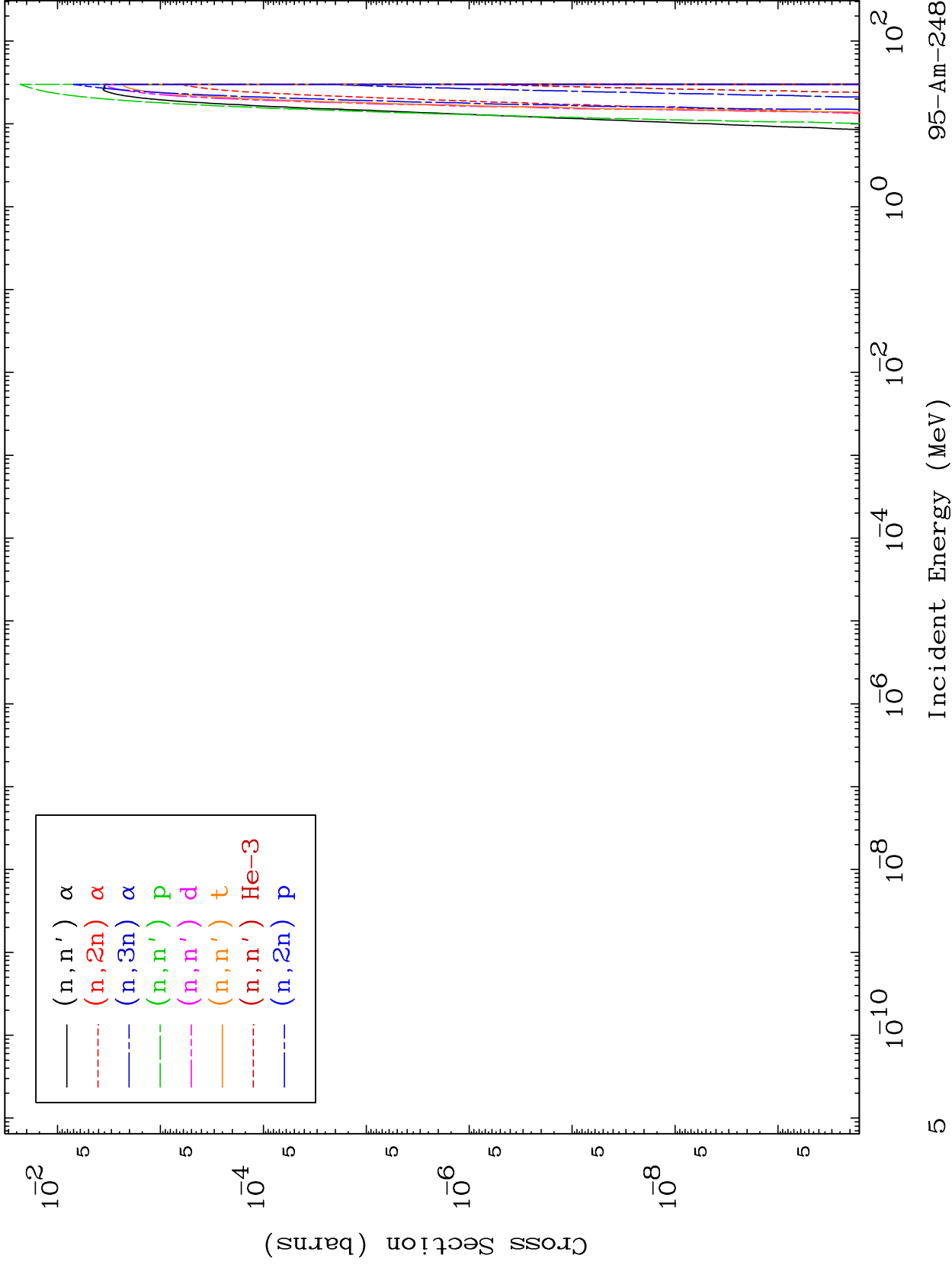
95-Am-248



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Charged Particle  
293 Kelvin Cross Sections

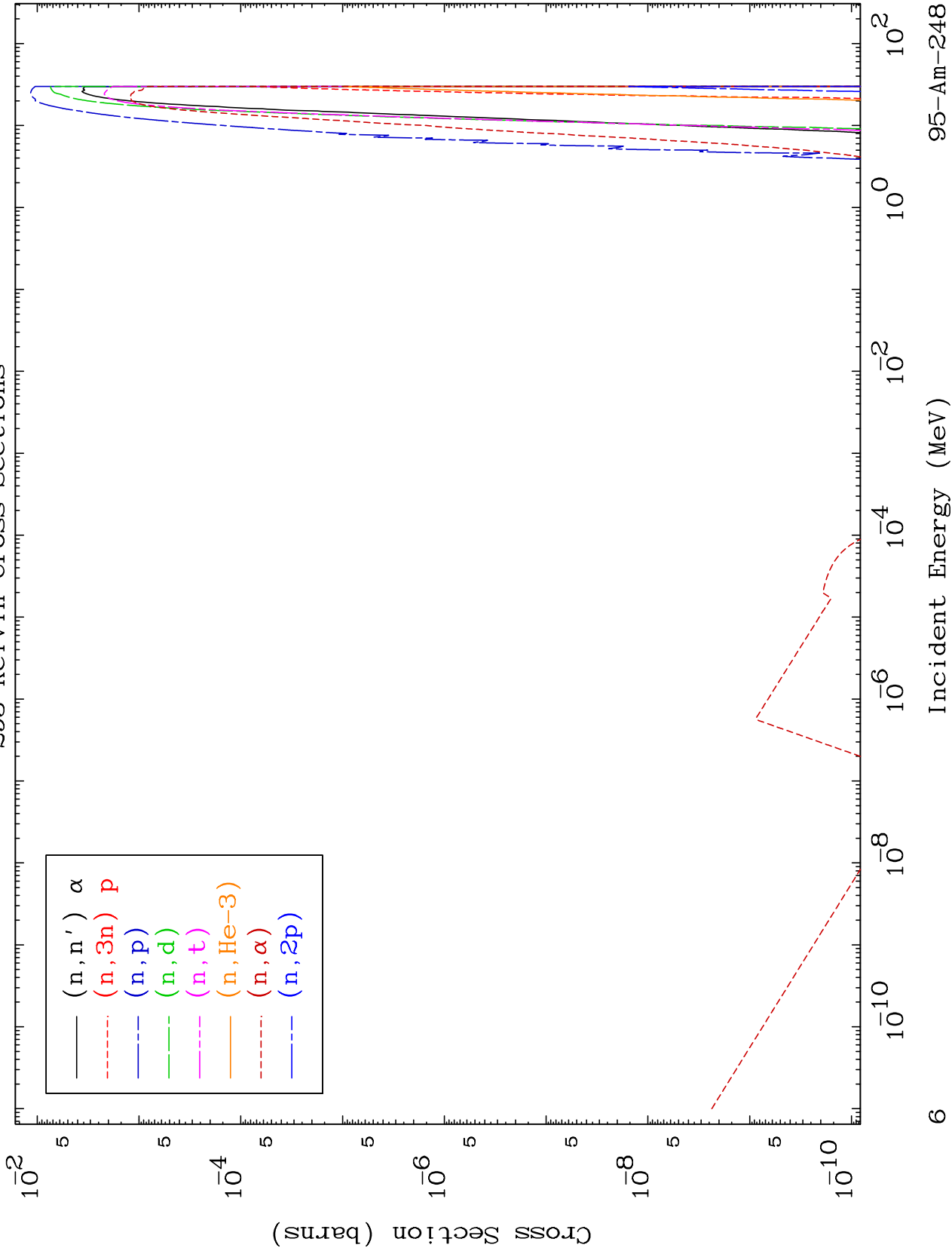
95-Am-248



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Charged Particle  
293 Kelvin Cross Sections

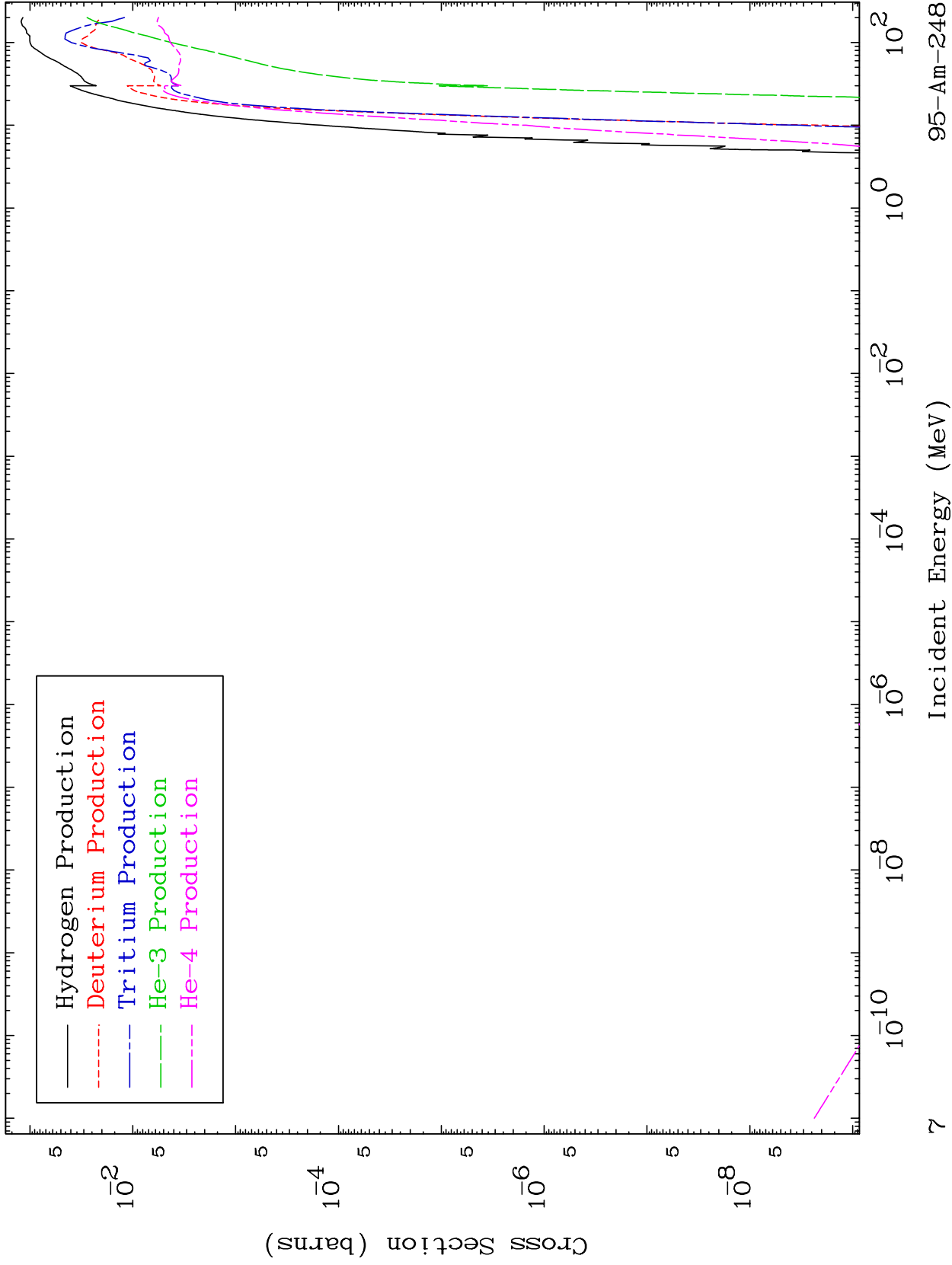
95-Am-248



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Particle Production  
293 Kelvin Cross Sections

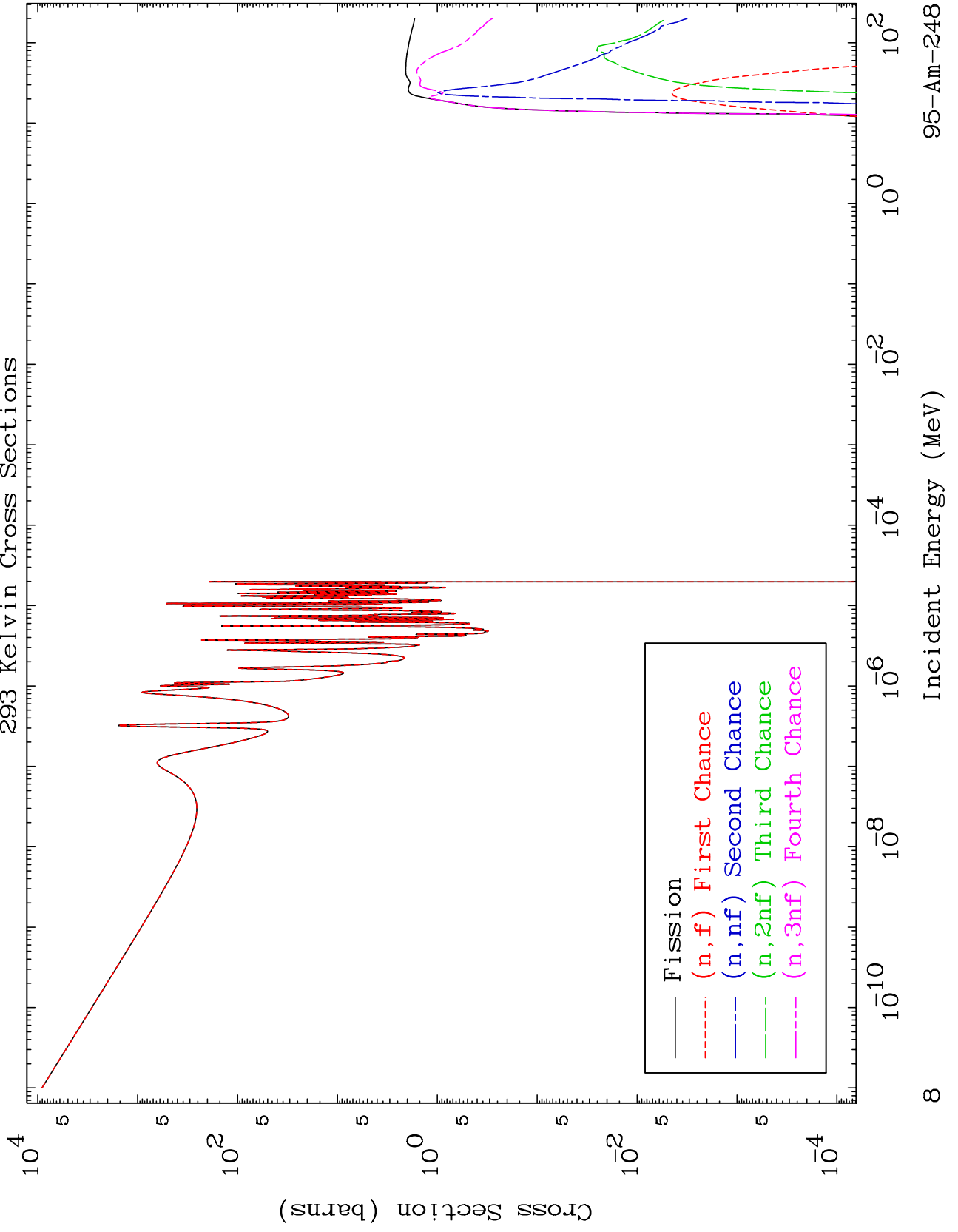
95-Am-248



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Fission  
293 Kelvin Cross Sections

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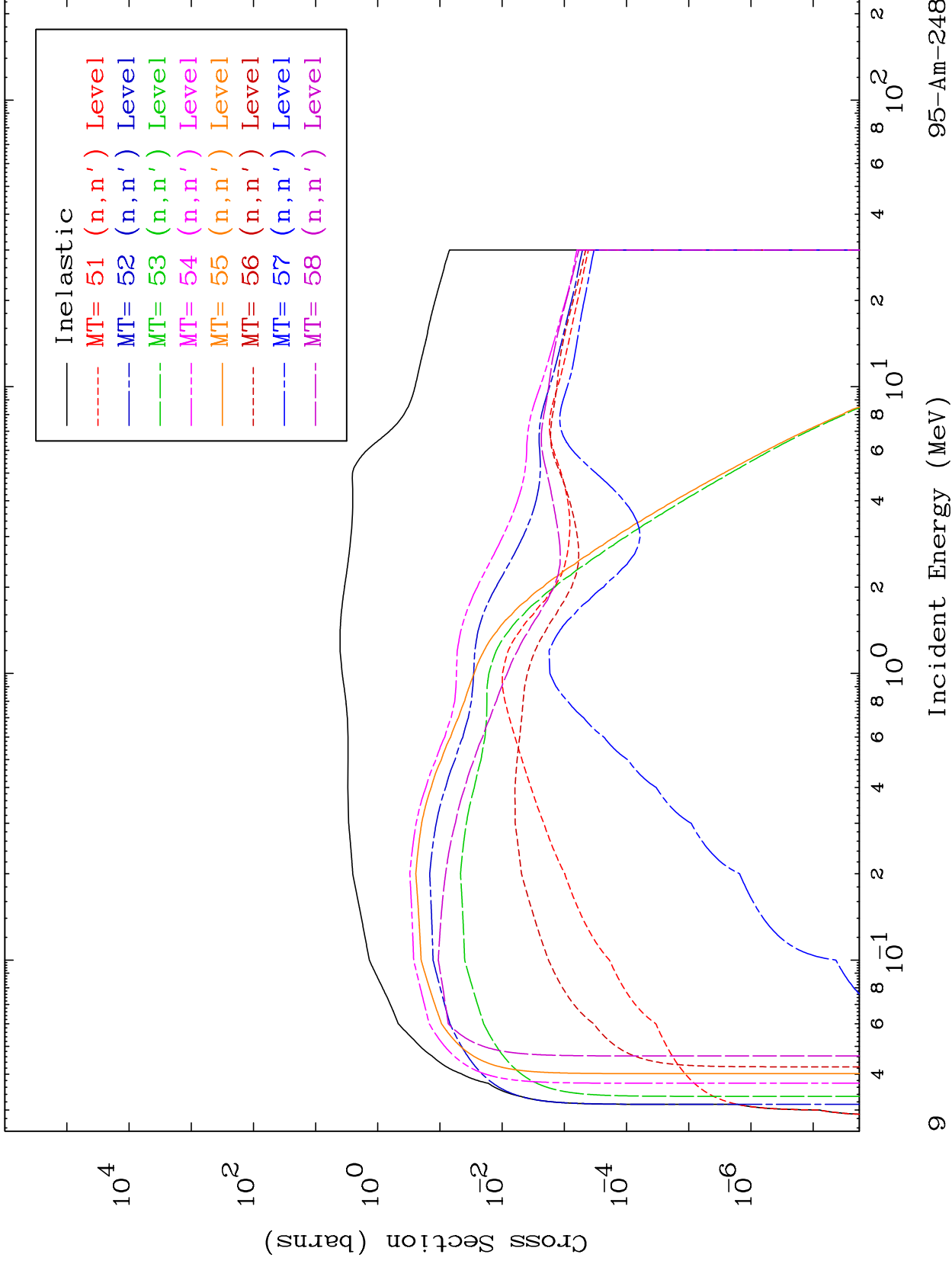


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(n,n') Levels

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293 Kelvin Cross Sections

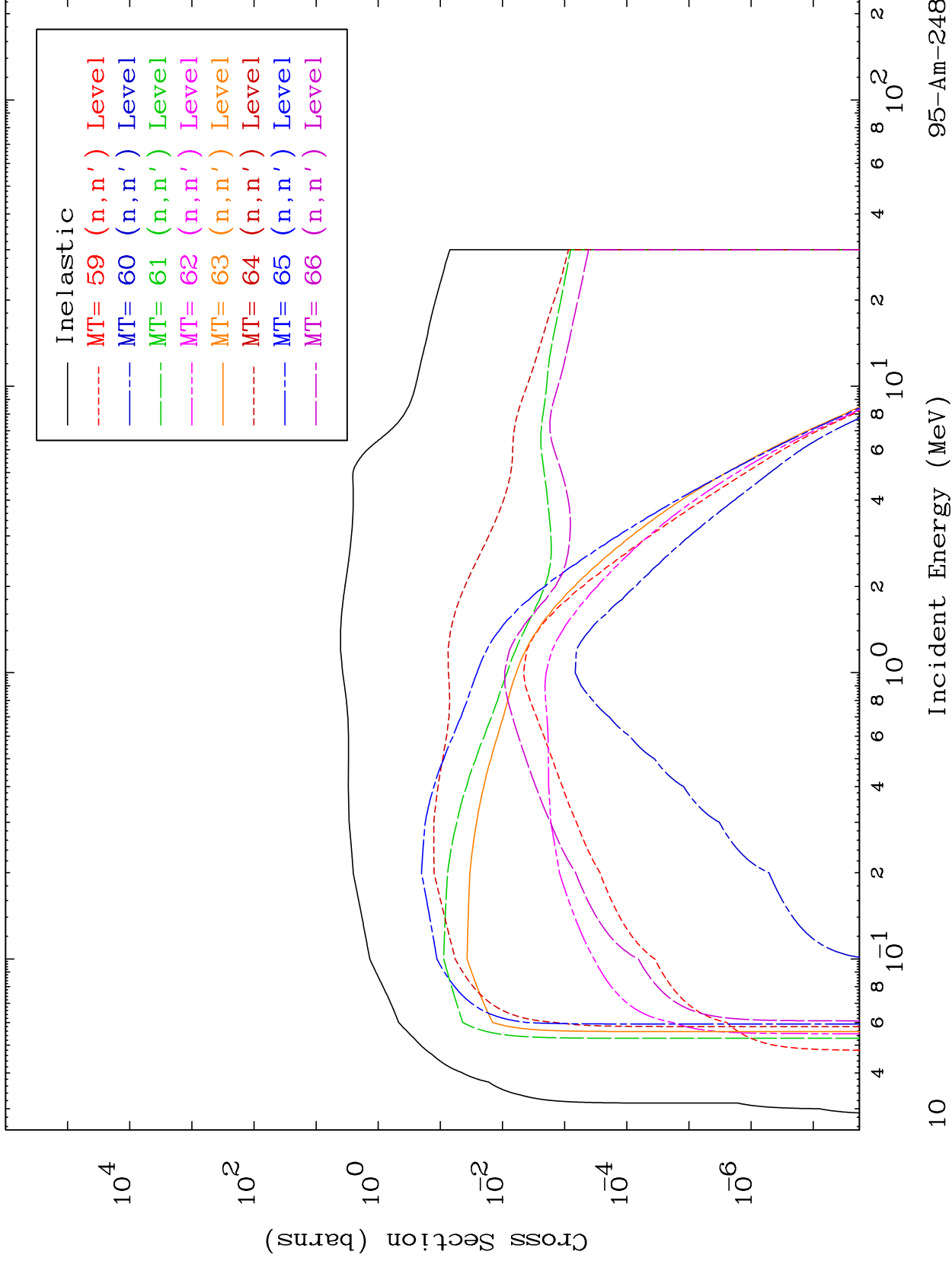


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(n,n') Levels

95-Am-248

293 Kelvin Cross Sections



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Incident Energy (MeV)

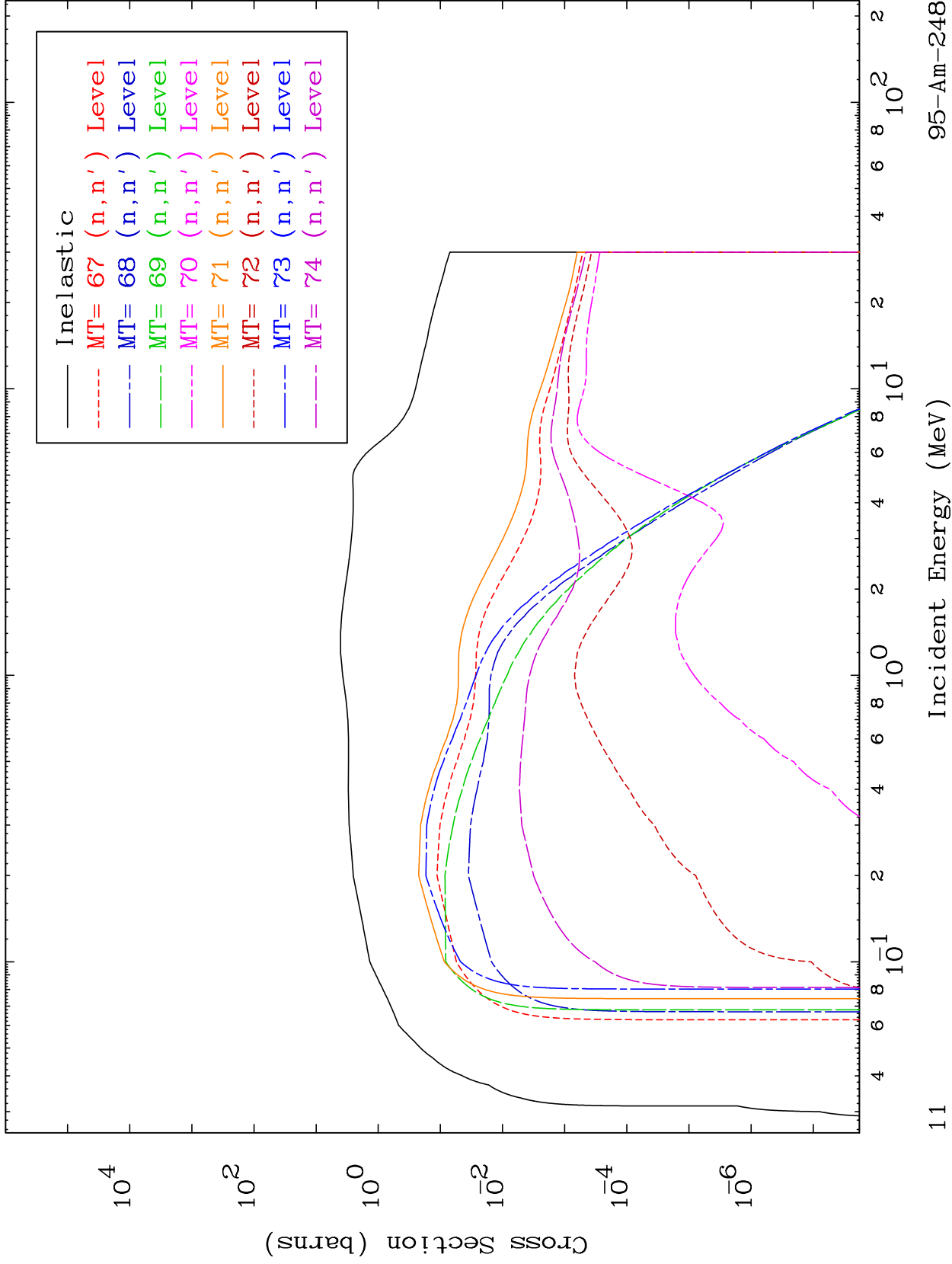
95-Am-248

MAT 9564

(n,n') Levels

95-Am-248

293 Kelvin Cross Sections

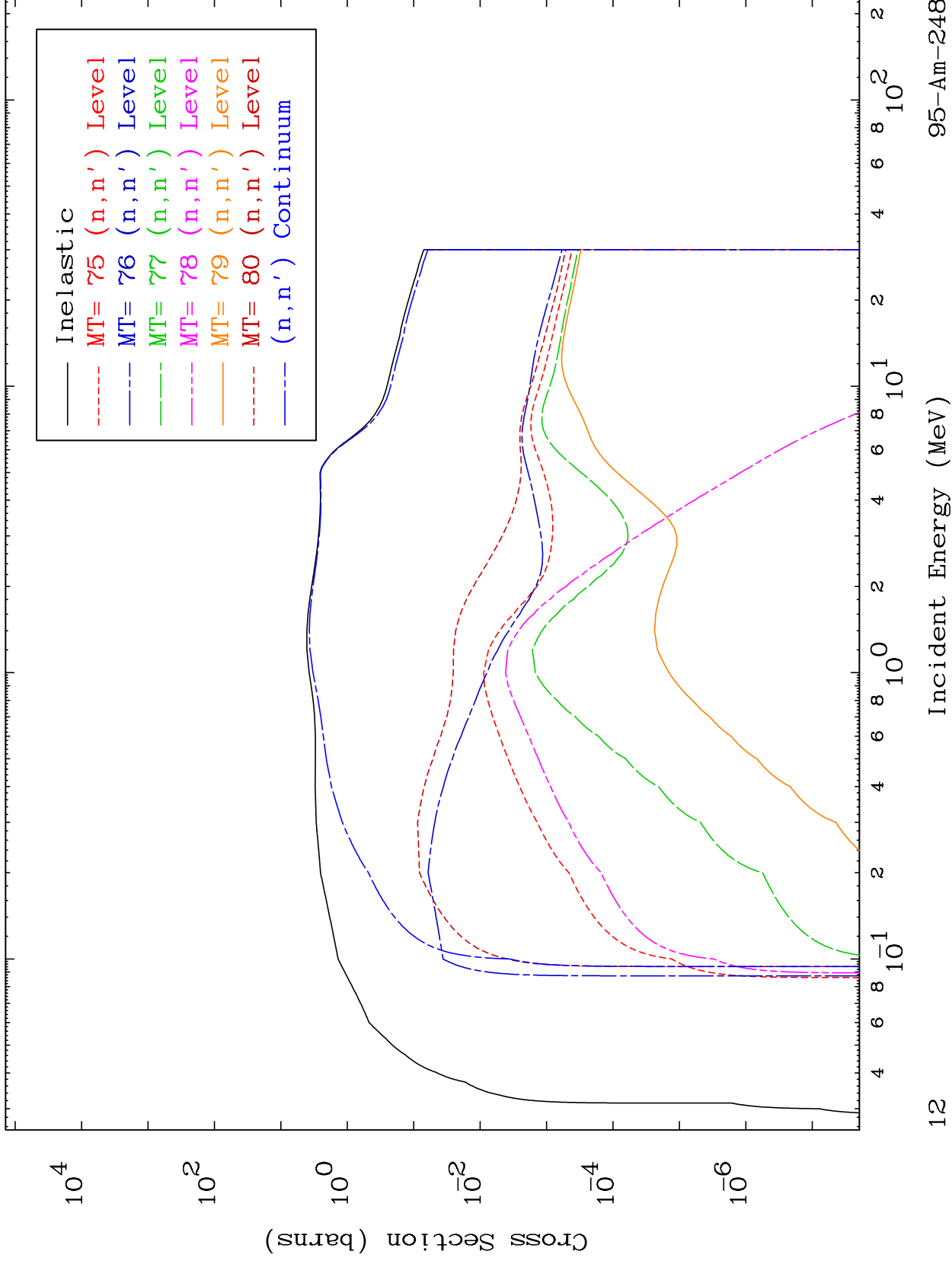


MAT 9564

(n,n') Levels

95-Am-248

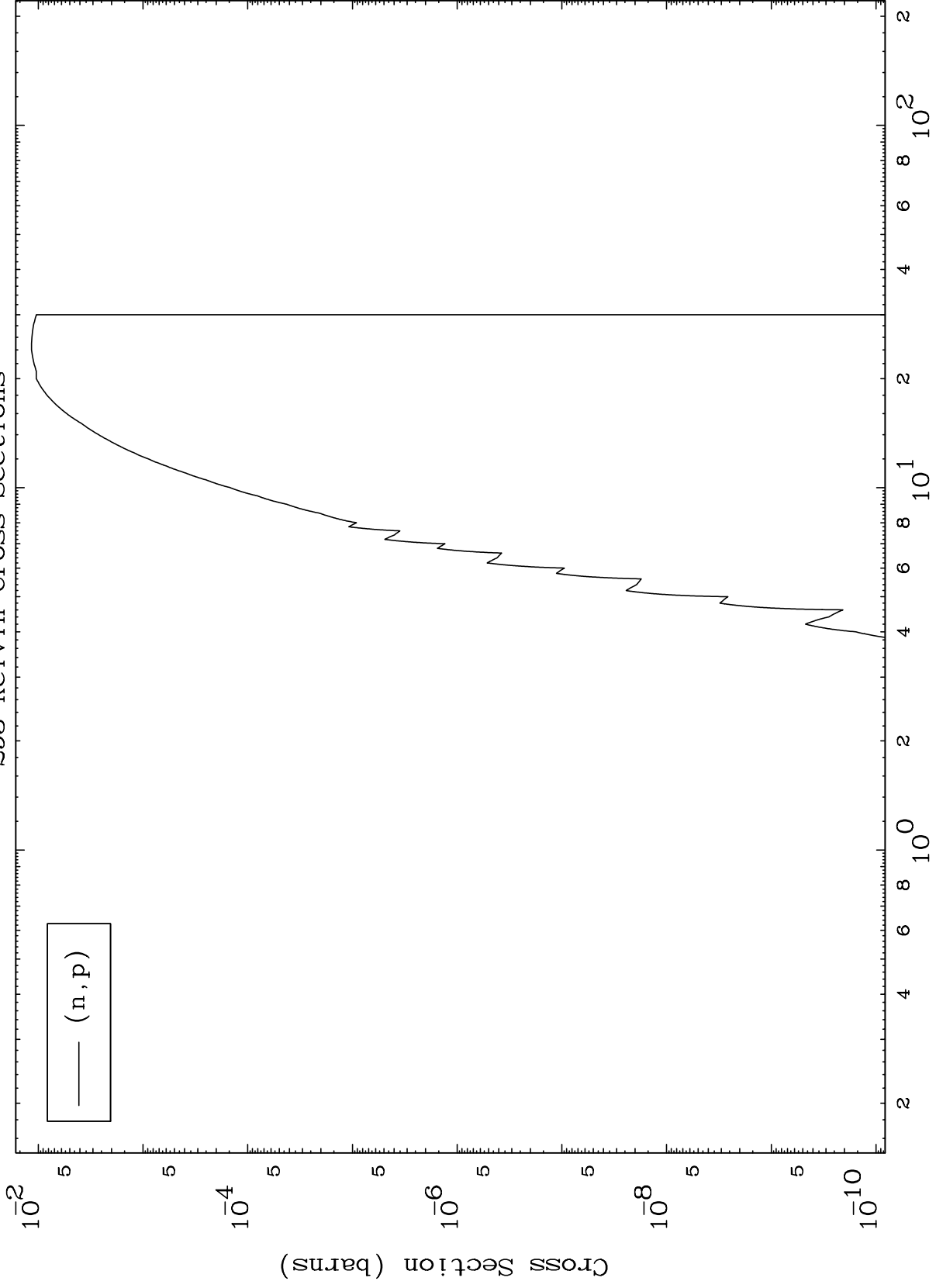
293 Kelvin Cross Sections



MAT 9564

(n,p) Levels  
293 Kelvin Cross Sections

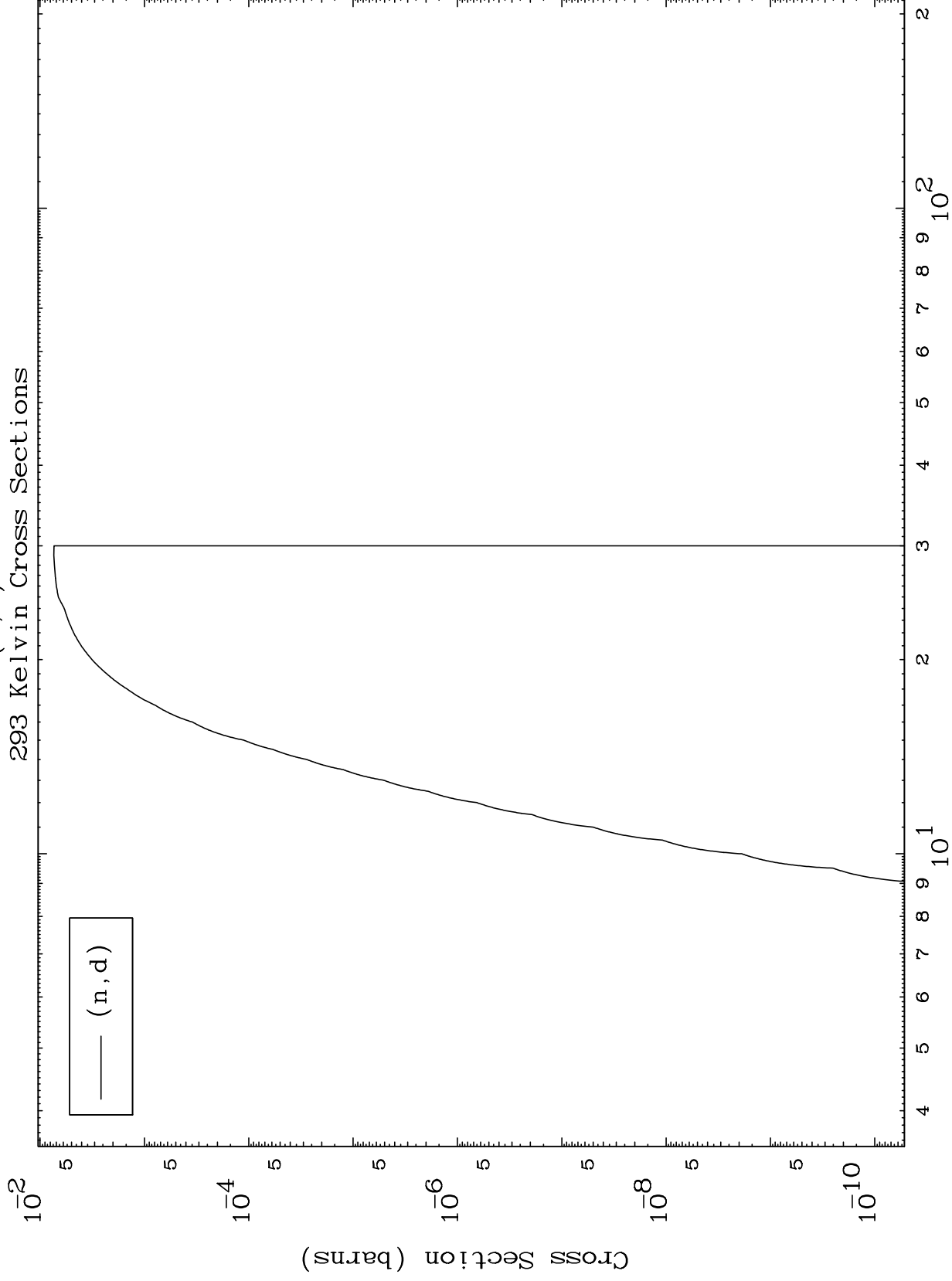
95-Am-248



MAT 9564

(n,d) Levels  
293 Kelvin Cross Sections

95-Am-248



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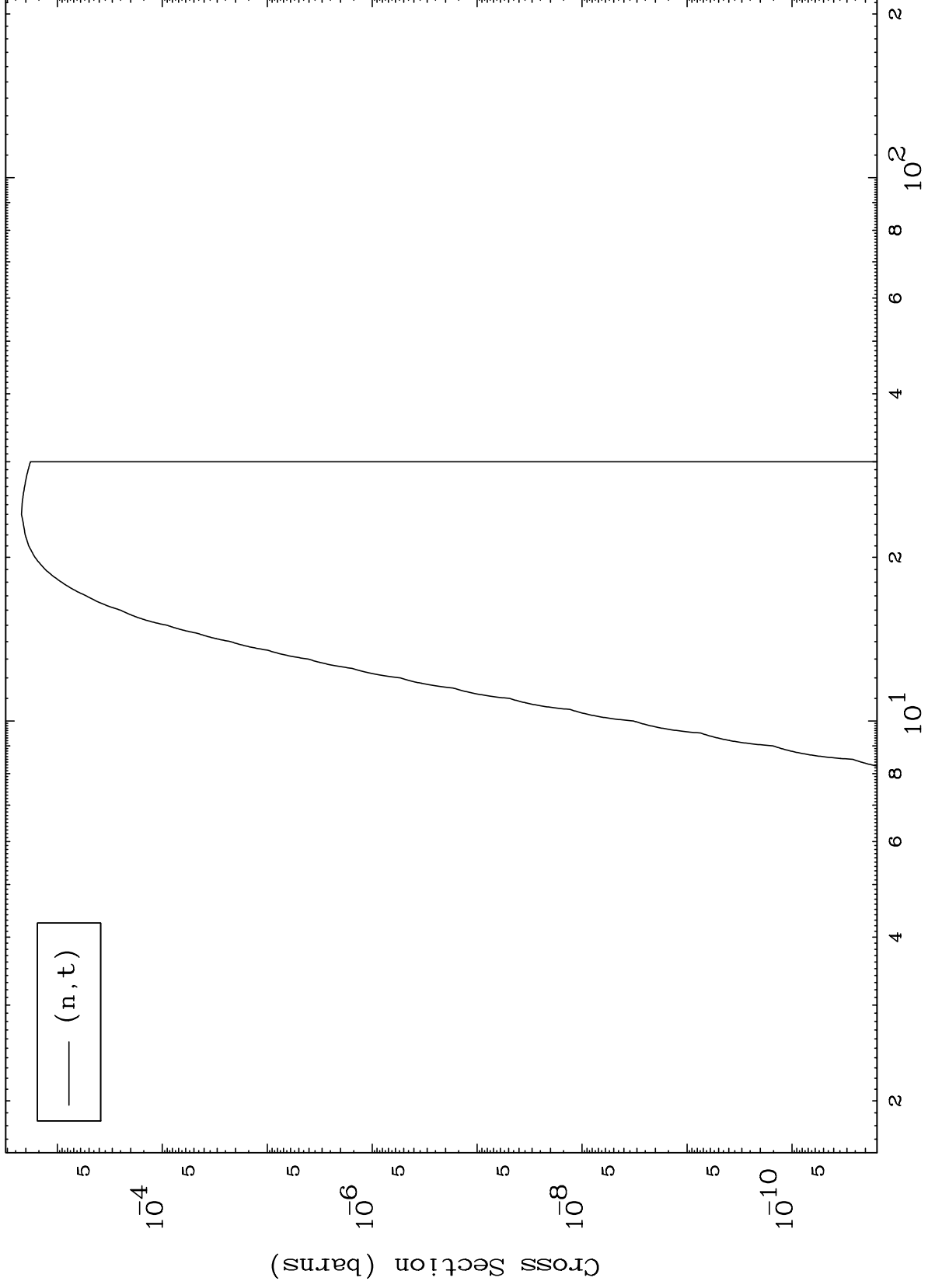
Incident Energy (MeV)

95-Am-248

MAT 9564

(n,t) Levels  
293 Kelvin Cross Sections

95-Am-248



15

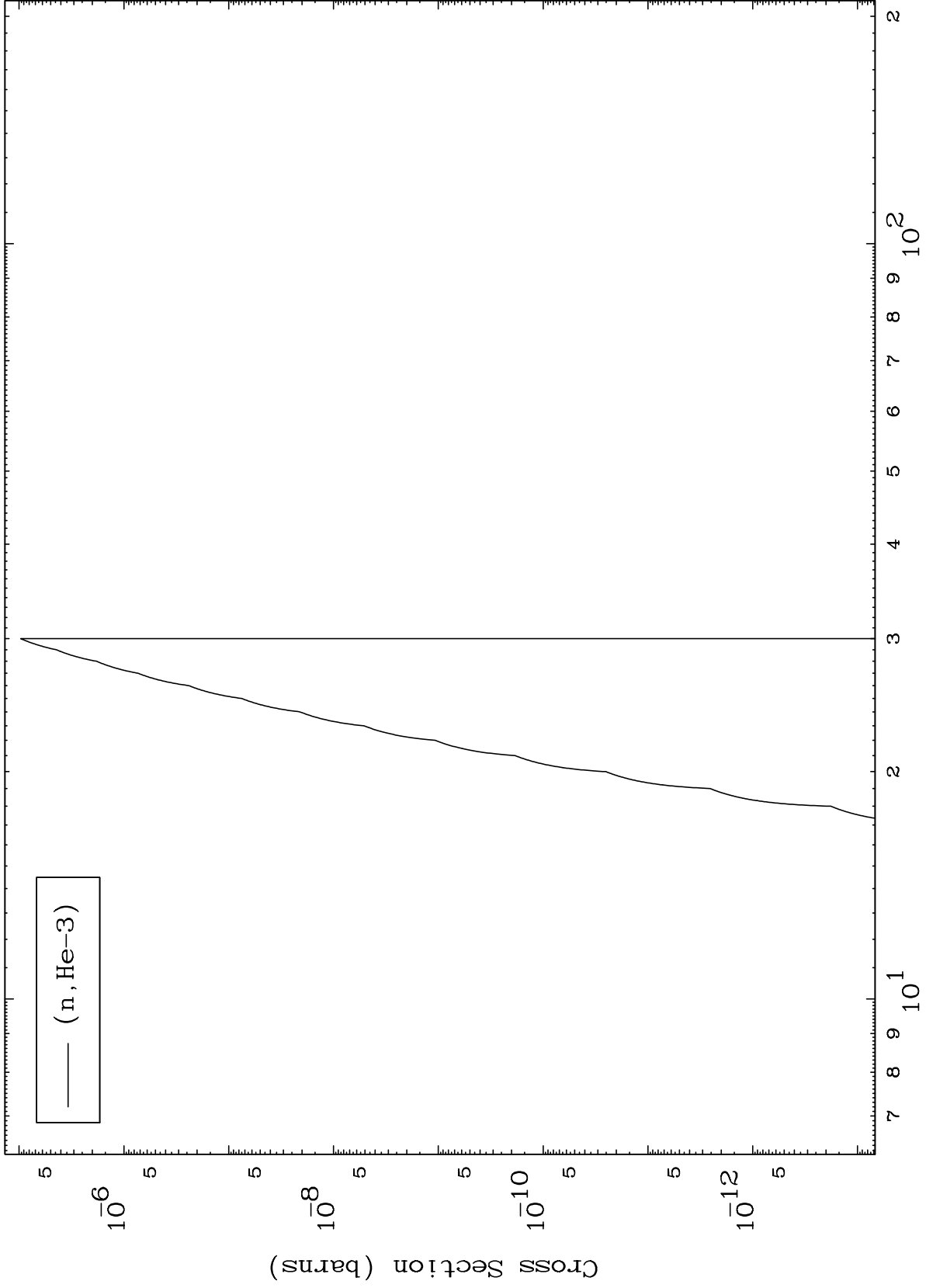
Incident Energy (MeV)

95-Am-248

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(n,He3) Levels  
293 Kelvin Cross Sections

95-Am-248



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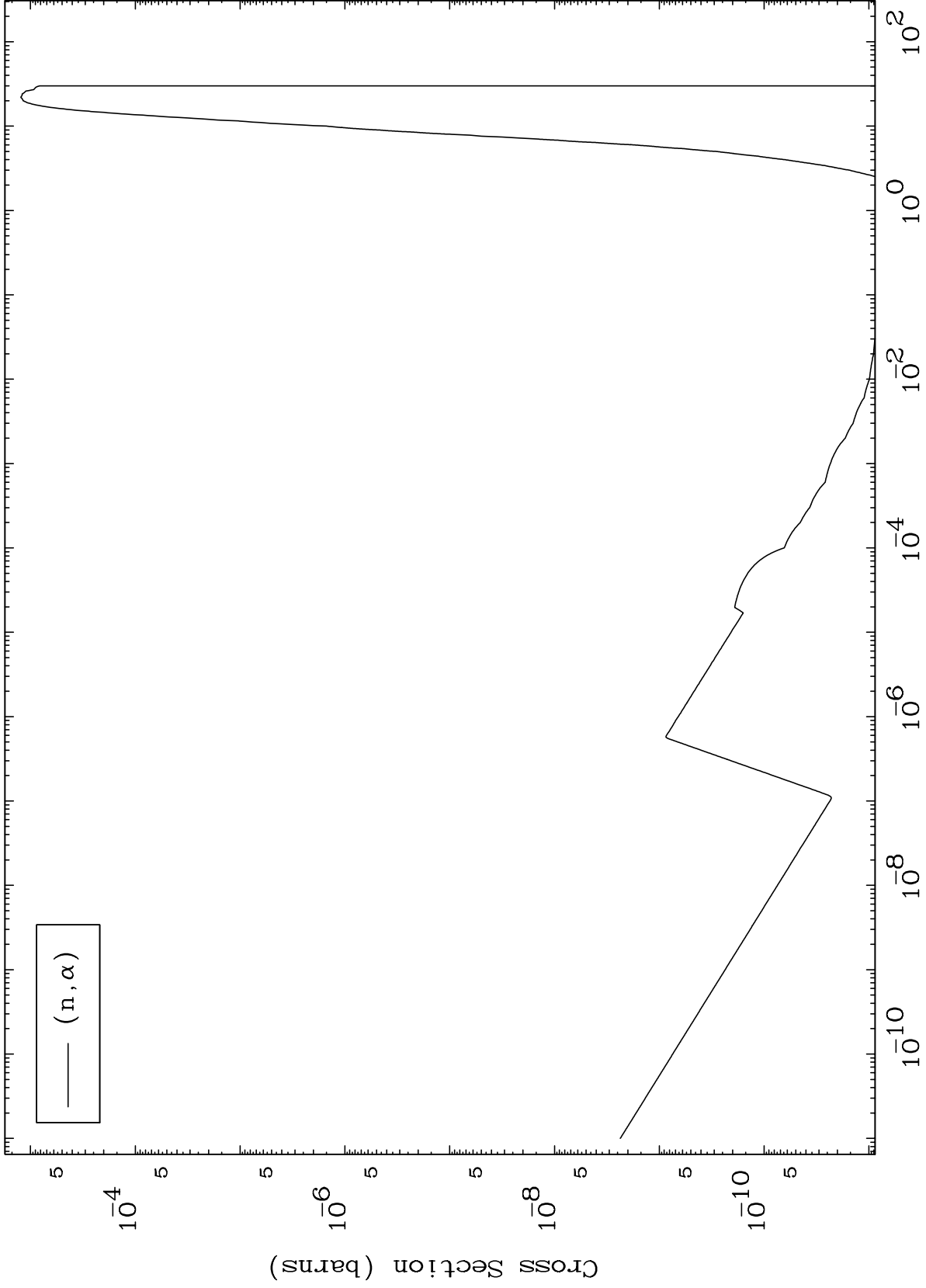
Incident Energy (MeV)

95-Am-248

MAT 9564

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

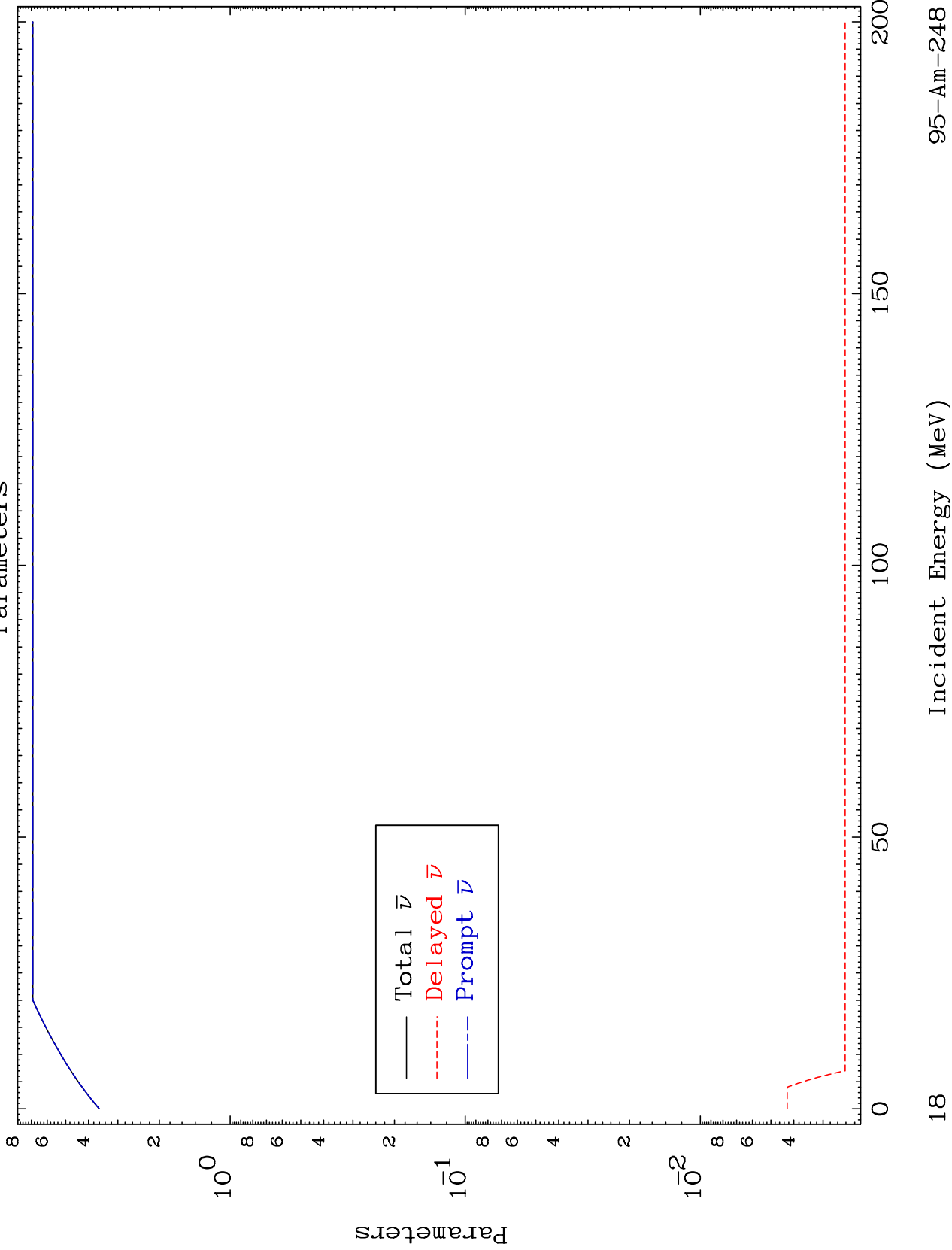
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Energy Release  
Parameters

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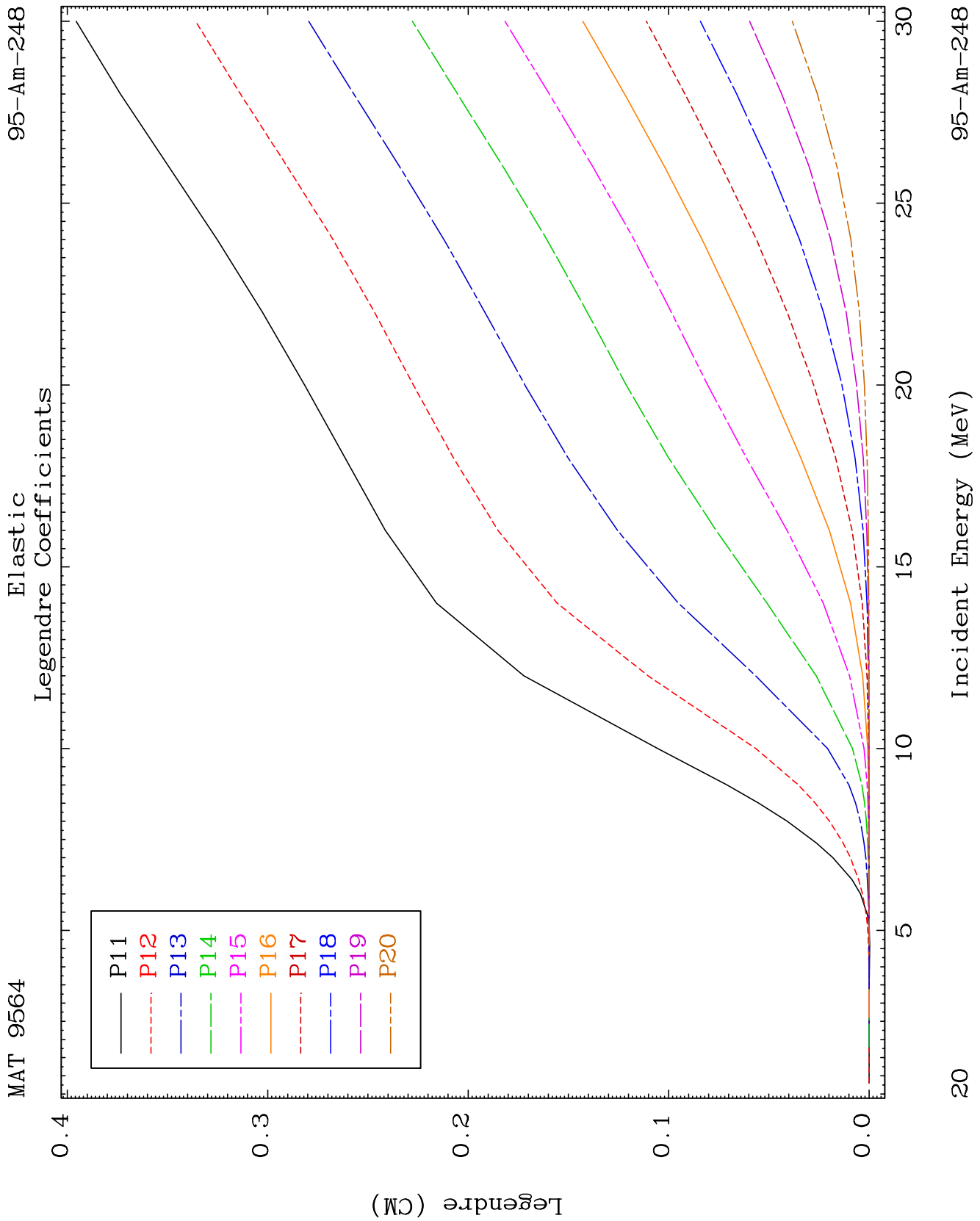


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Incident Energy (MeV)

95-Am-248

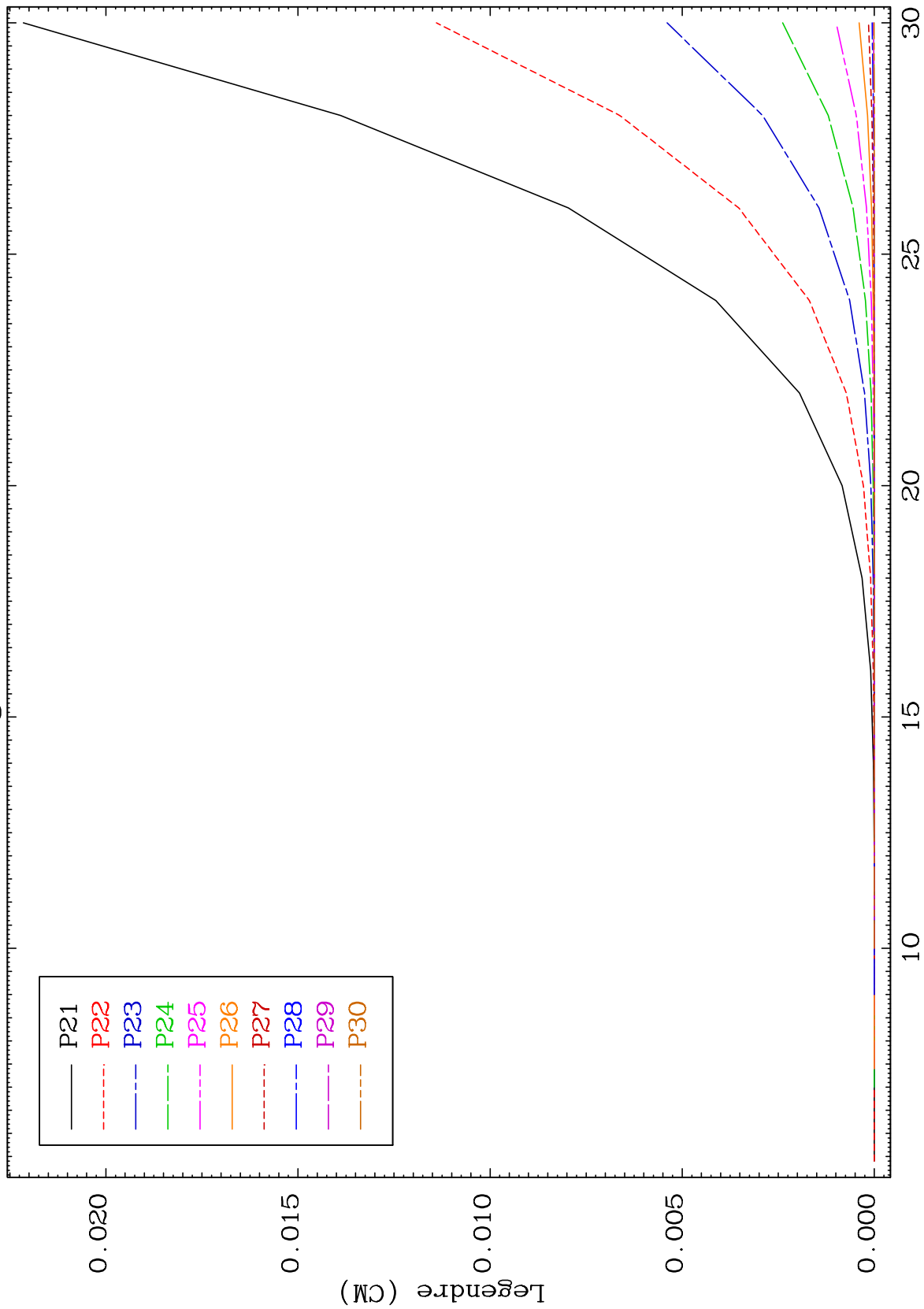




MAT 9564

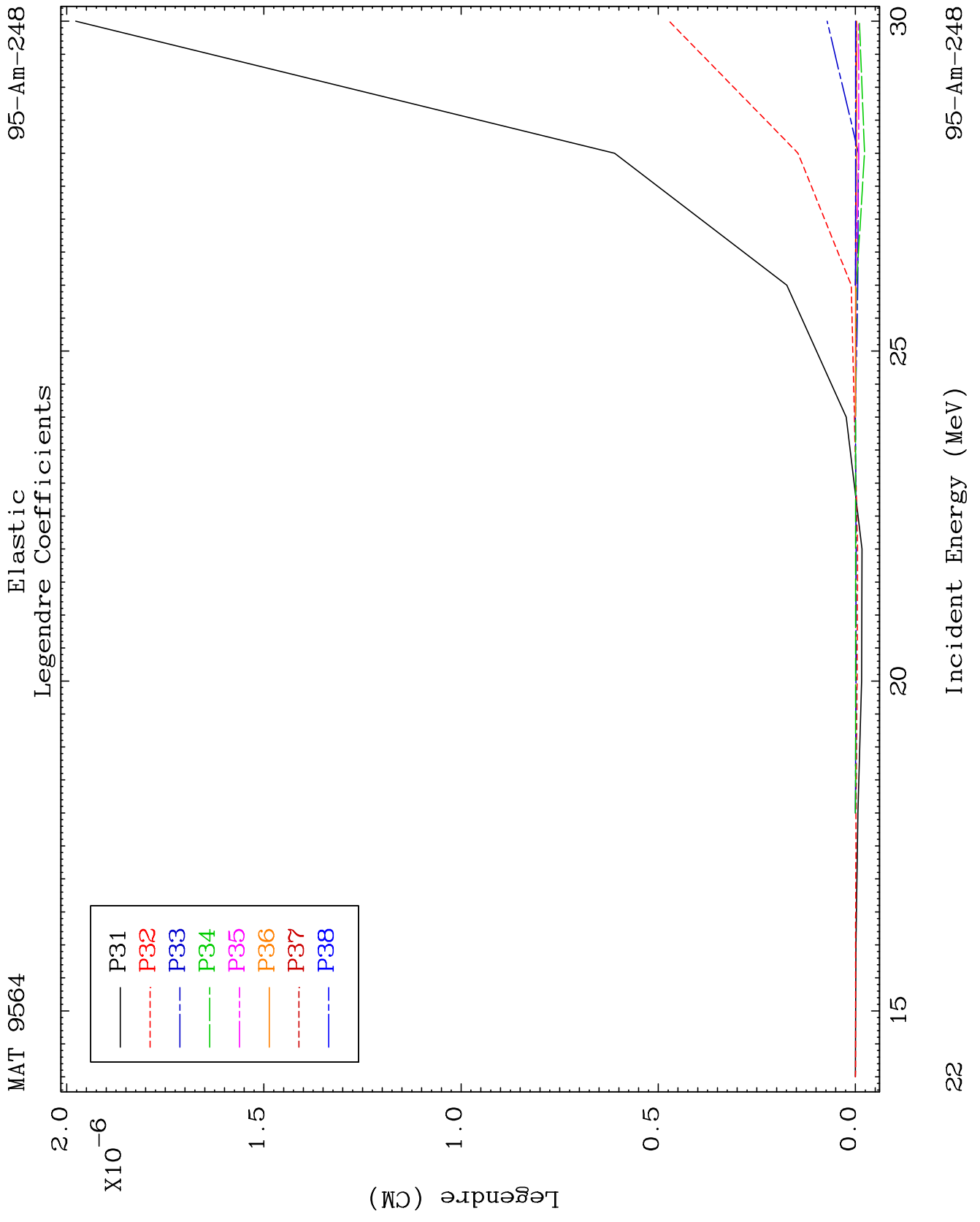
Elastic Legendre Coefficients

95-Am-248



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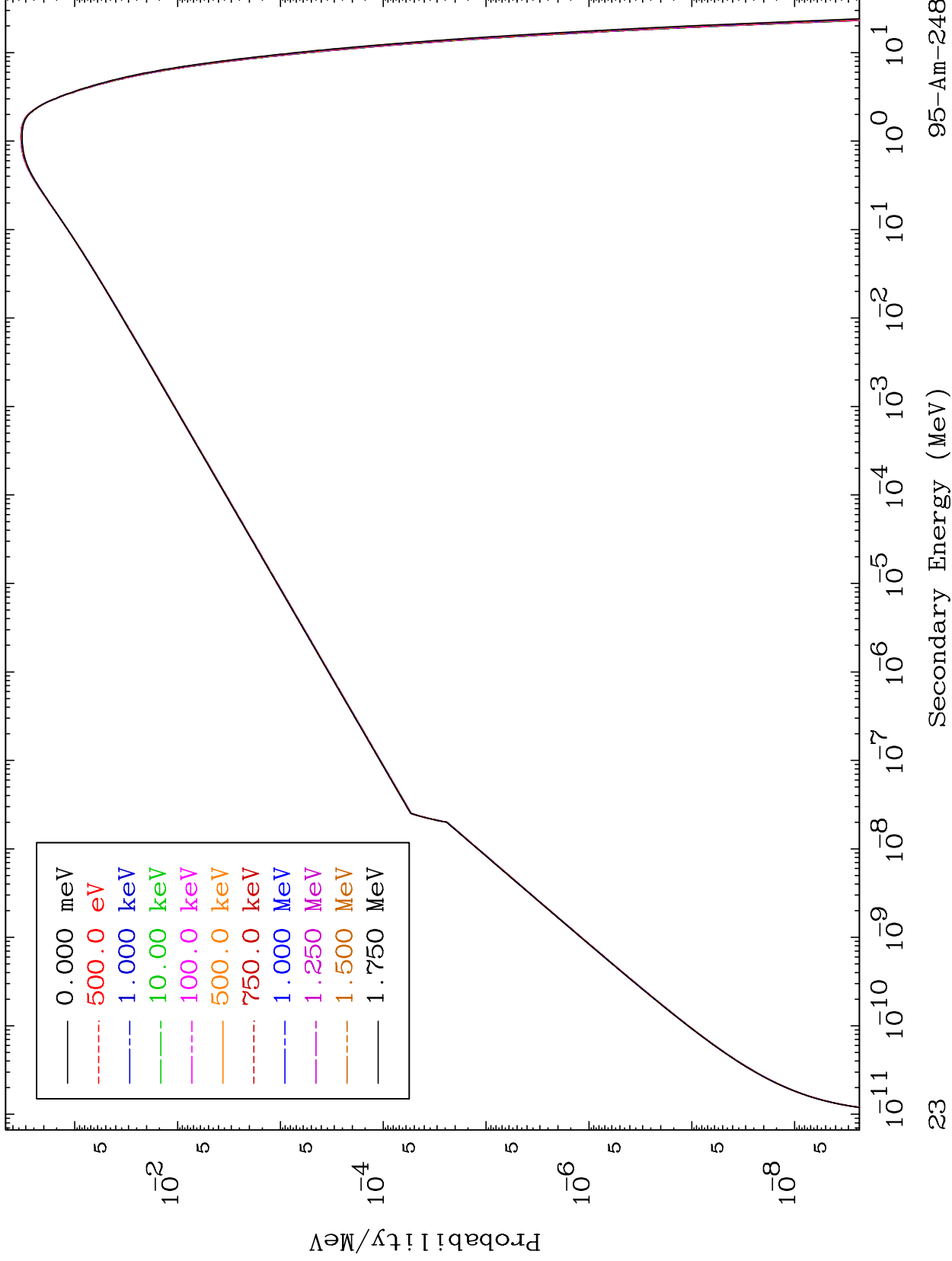
95-Am-248



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### Fission Energy Distributions

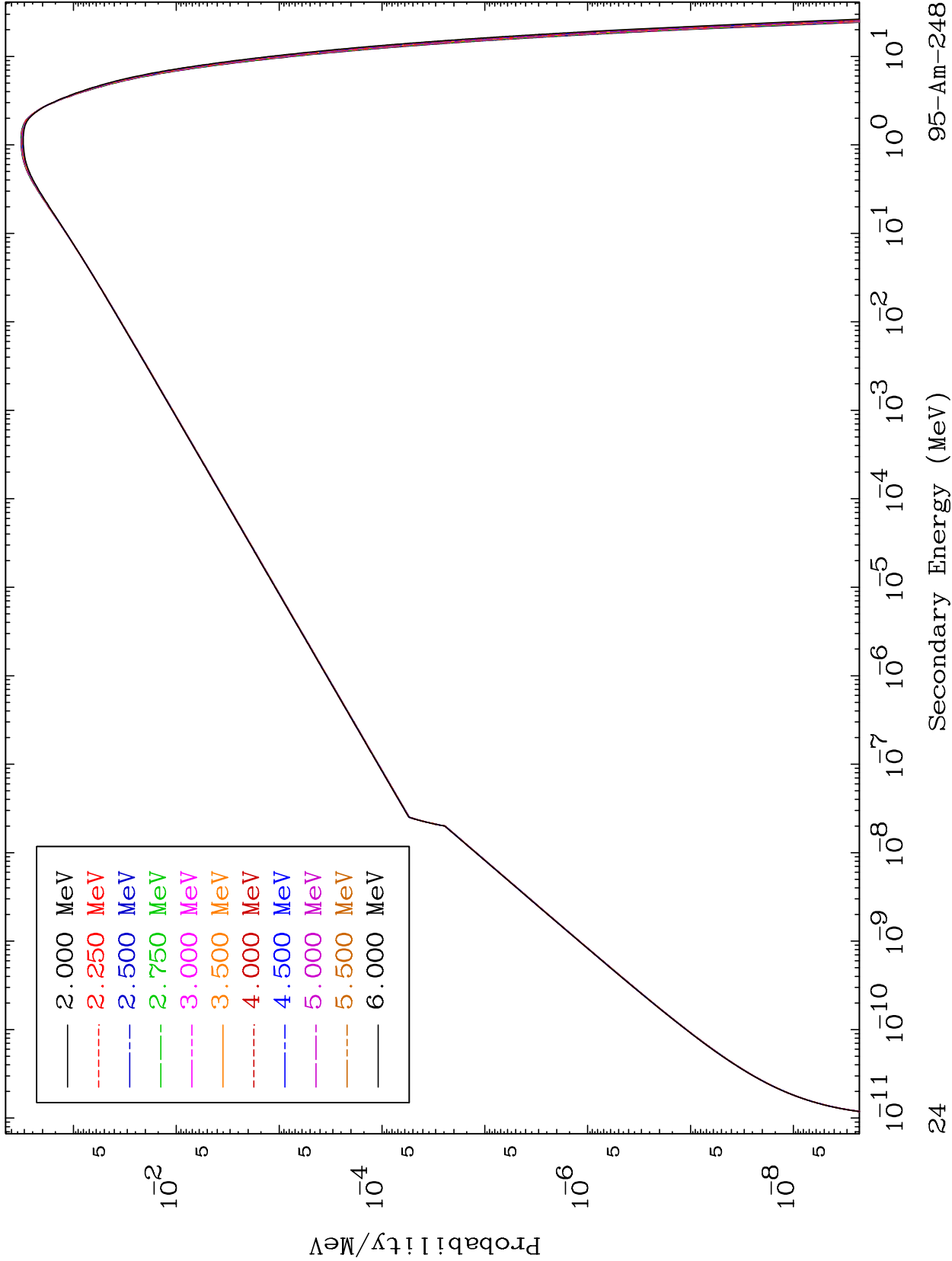
95-Am-248



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### Fission Energy Distributions

95-Am-248



95-Am-248

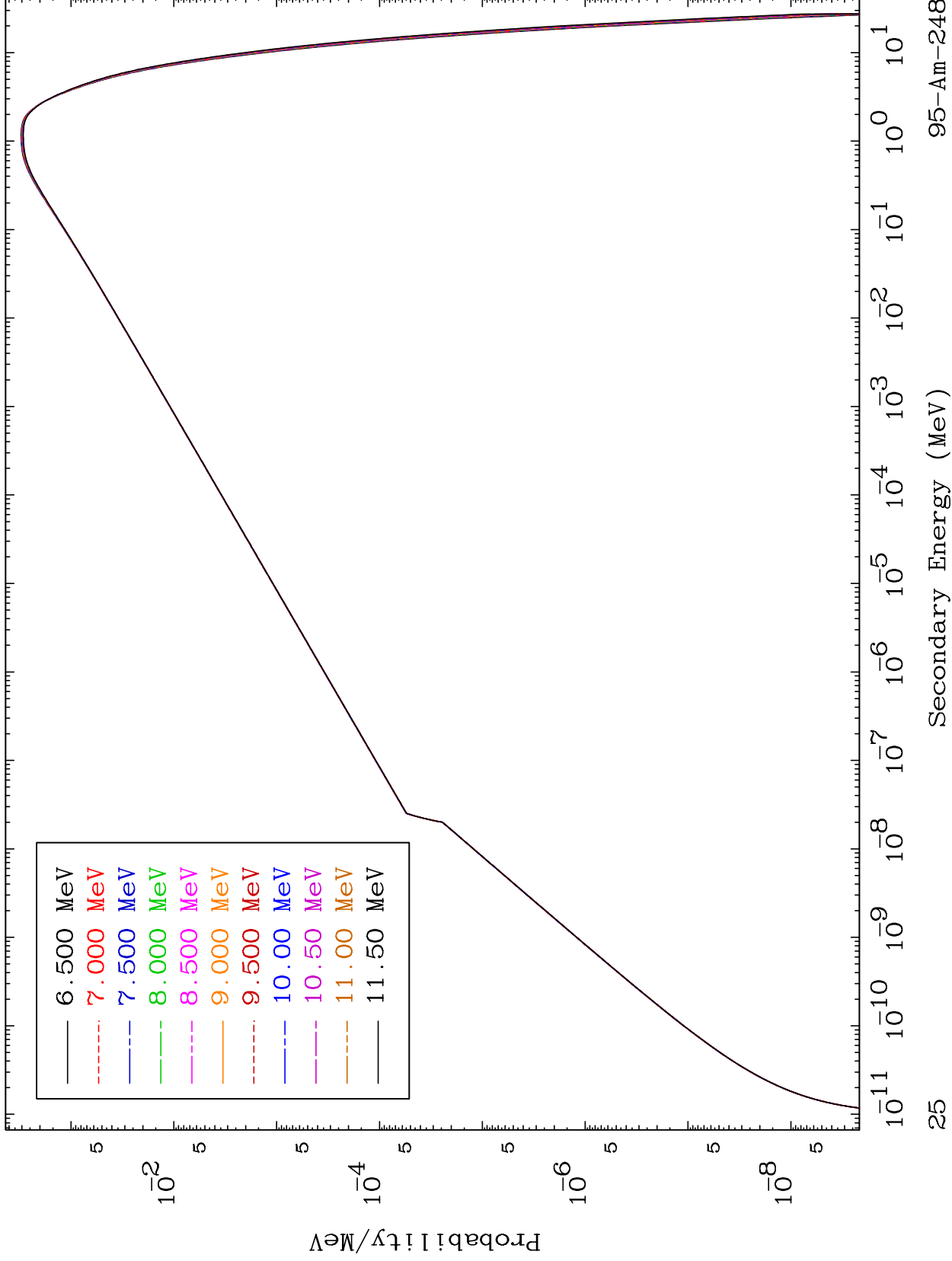
Secondary Energy (MeV)

24

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### Fission Energy Distributions

95-Am-248



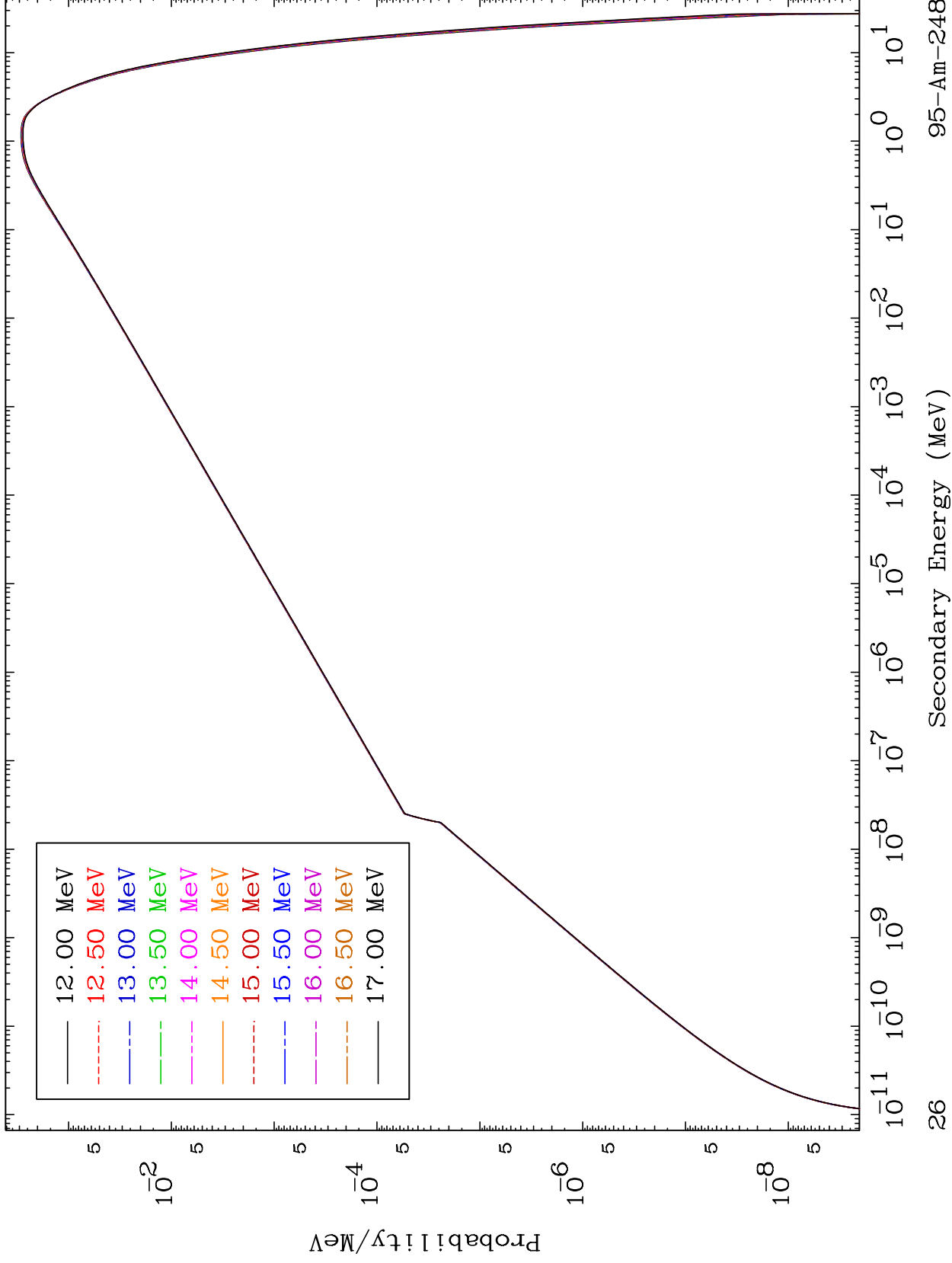
95-Am-248

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### Fission Energy Distributions

95-Am-248



95-Am-248

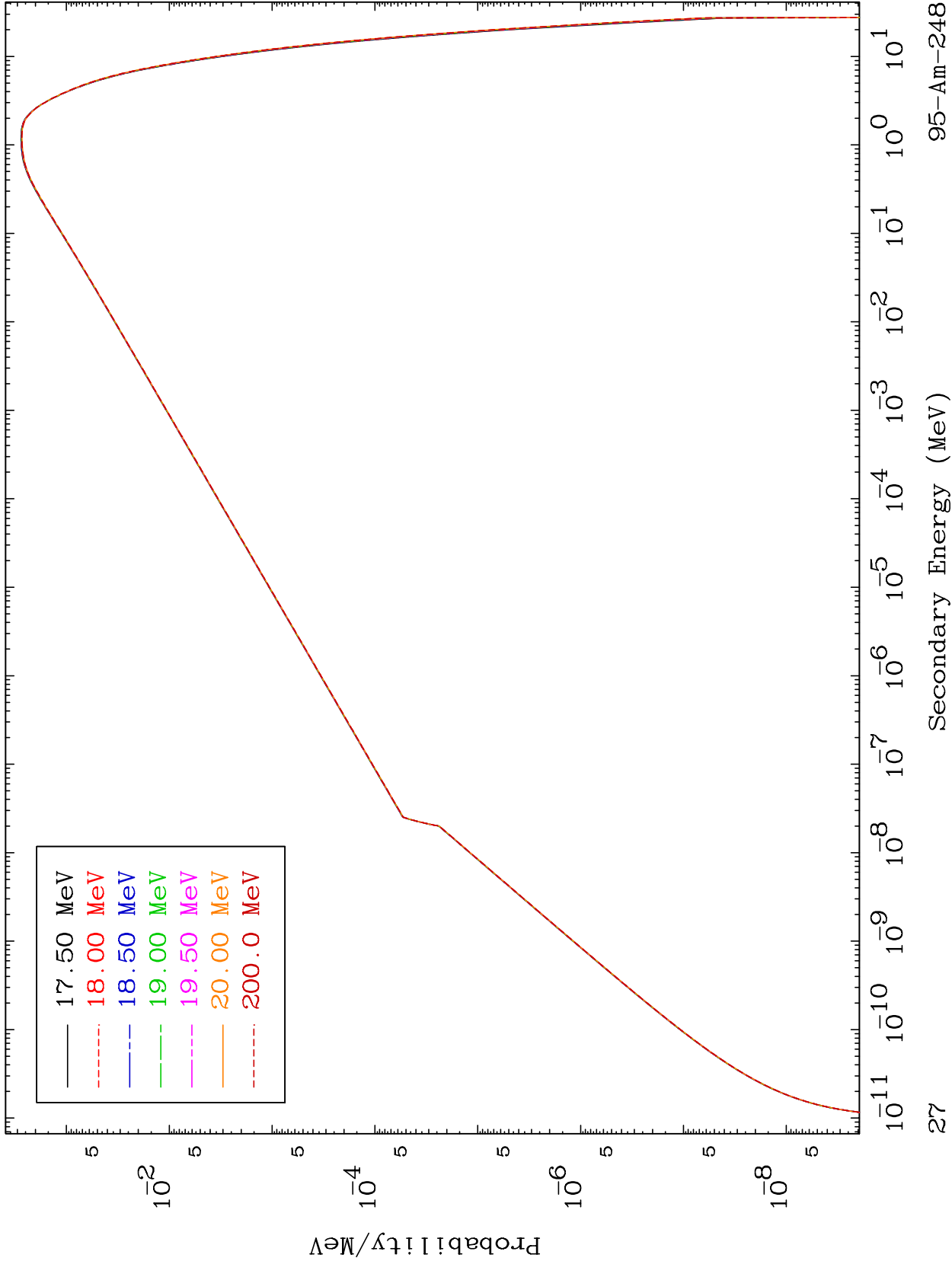
Secondary Energy (MeV)

26

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### Fission Energy Distributions

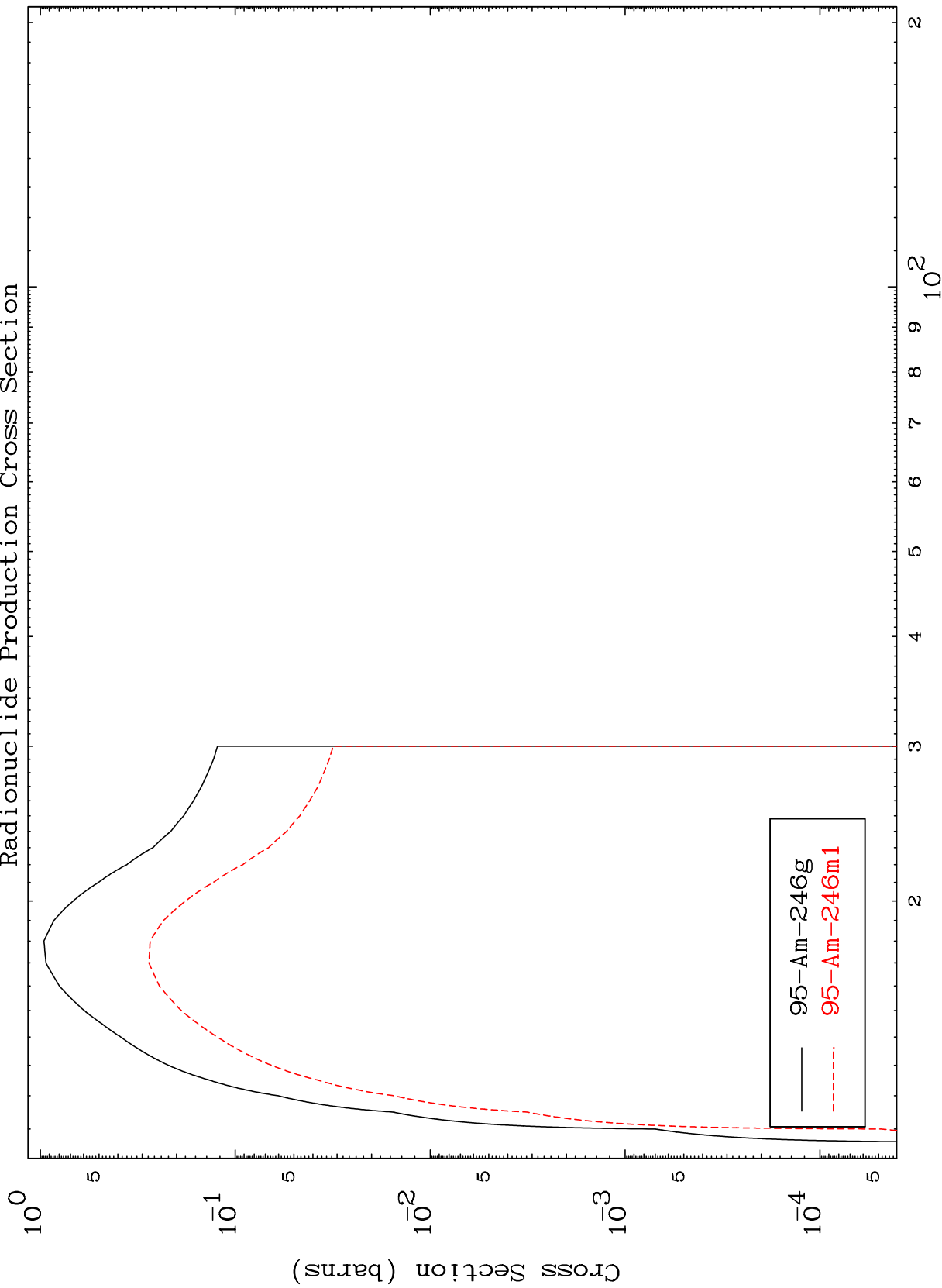
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(n,3n)  
Radionuclide Production Cross Section



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Incident Energy (MeV)

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(n,3n)  $\alpha$

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Radionuclide Production Cross Section

