

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

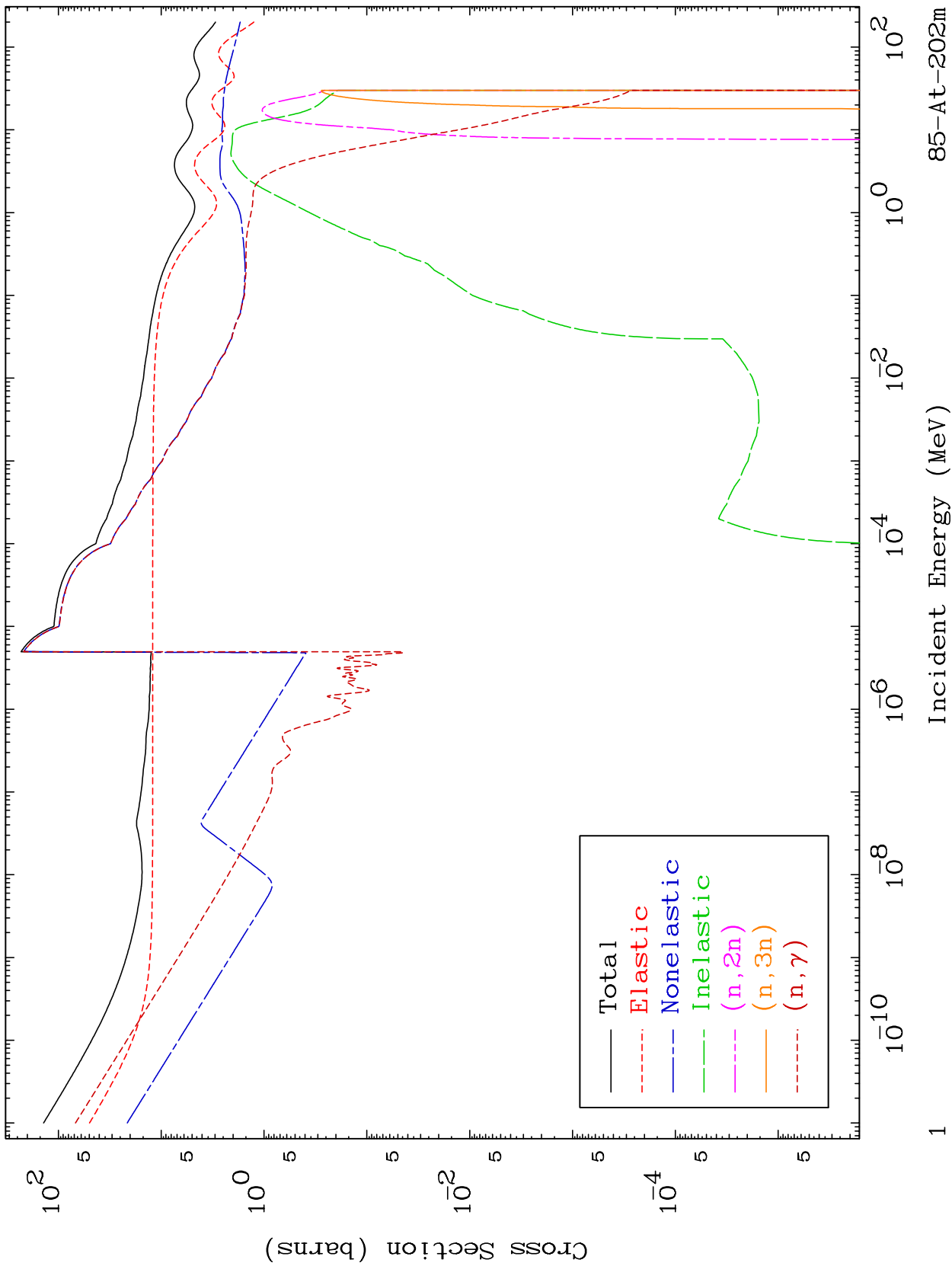
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8523

Neutron Major  
293 Kelvin Cross Sections

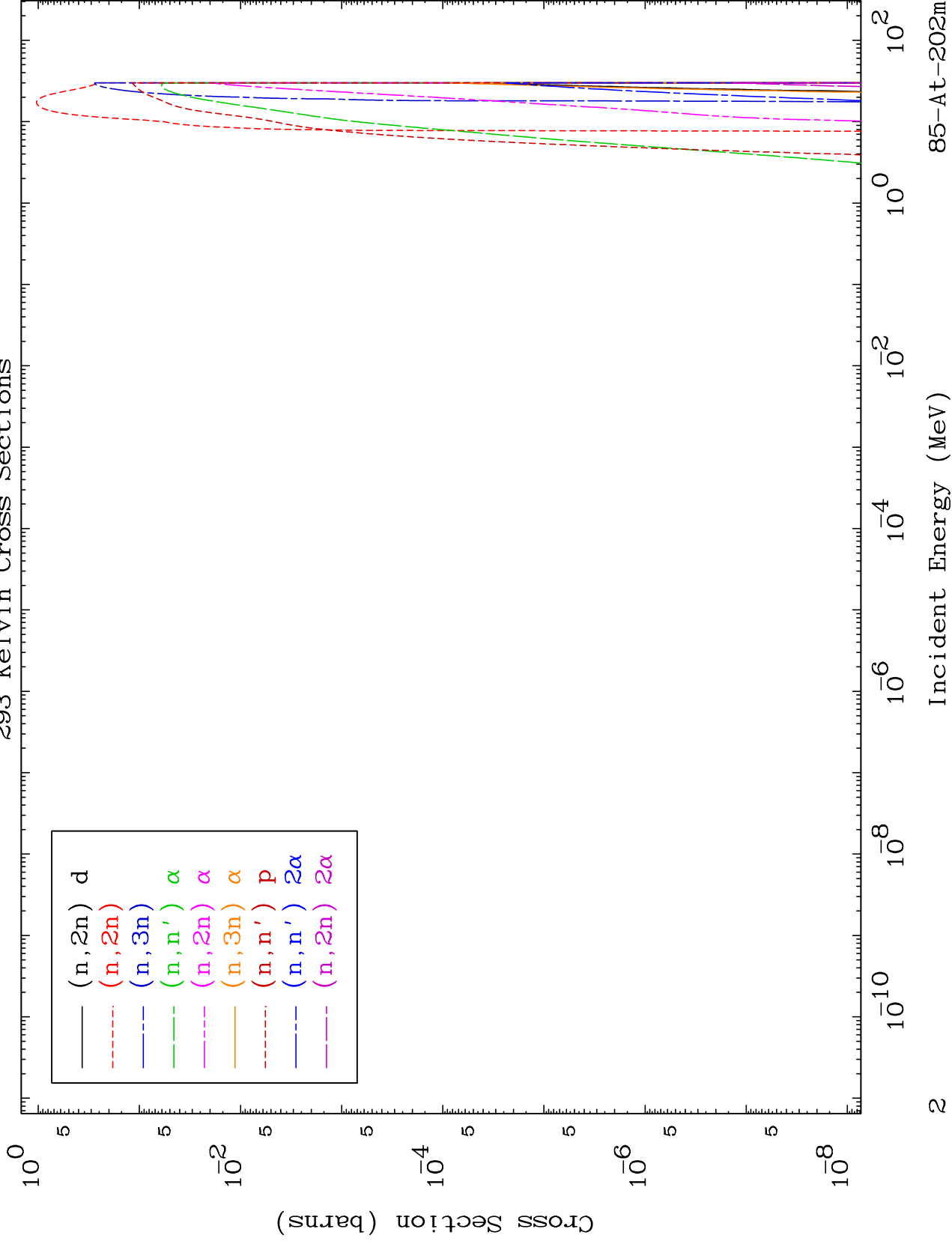
85-At-202m



MAT 8523

Neutron Absorption  
293 Kelvin Cross Sections

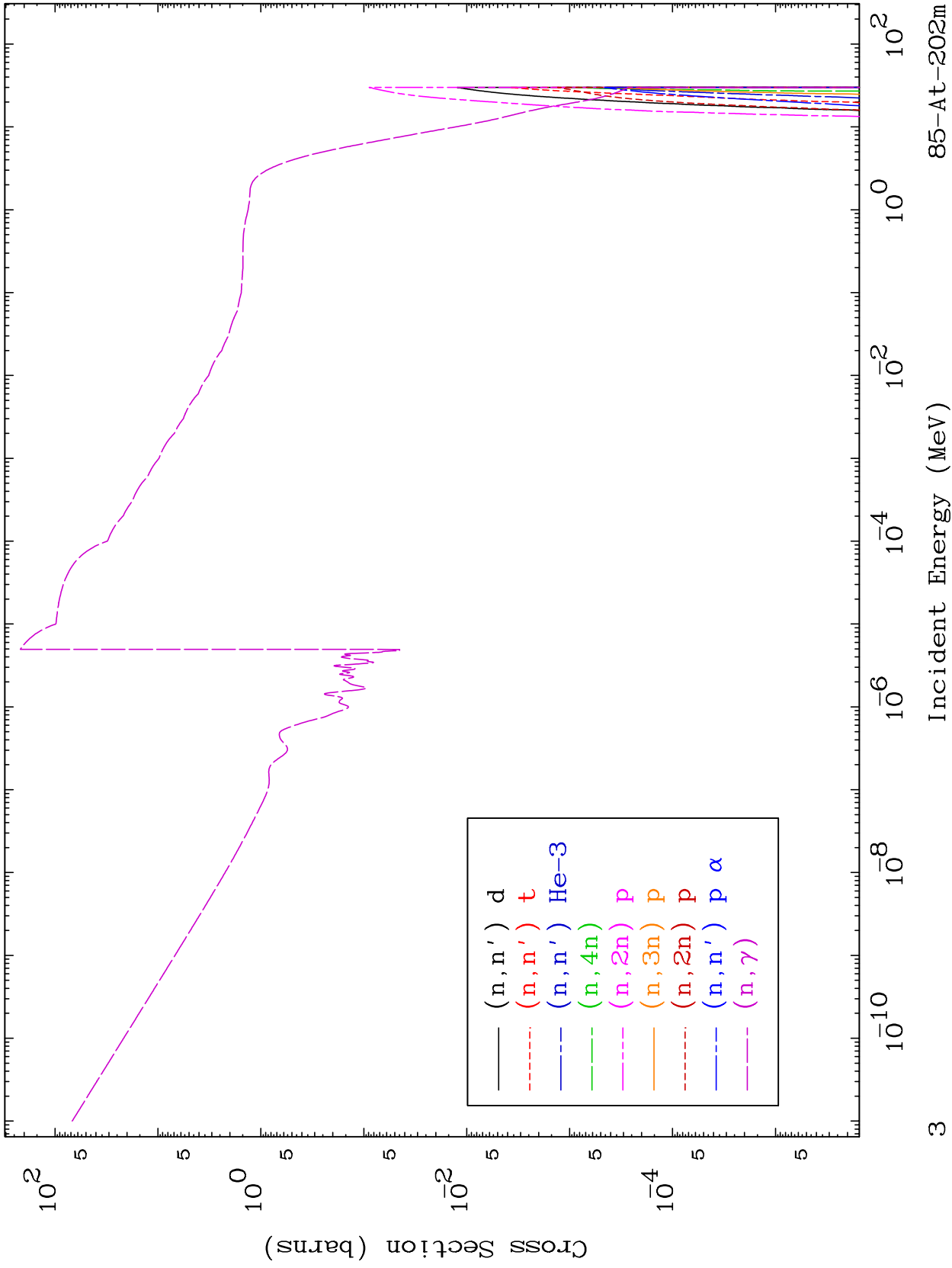
85-At-202m



MAT 8523

Neutron Absorption  
293 Kelvin Cross Sections

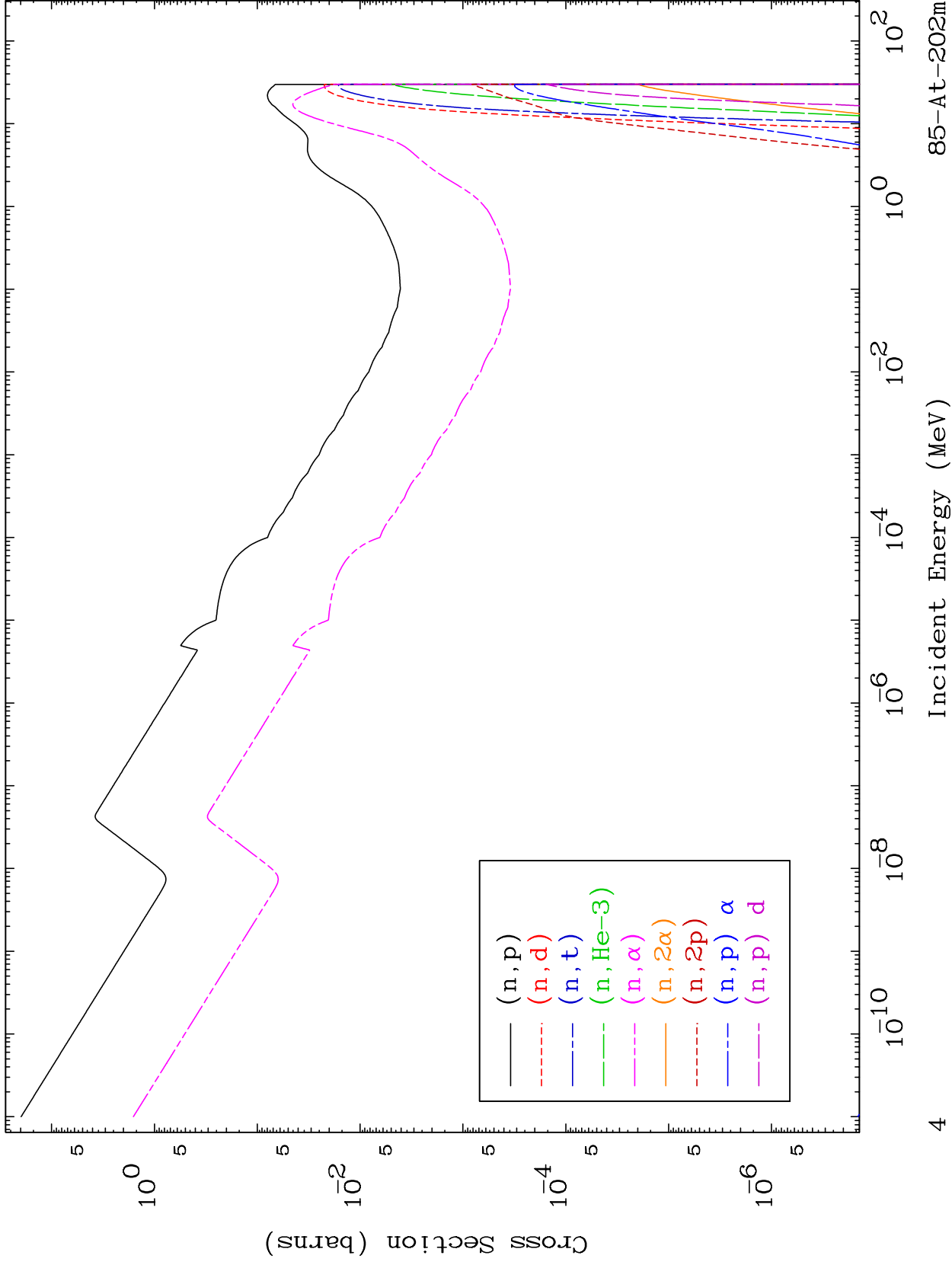
85-At-202m

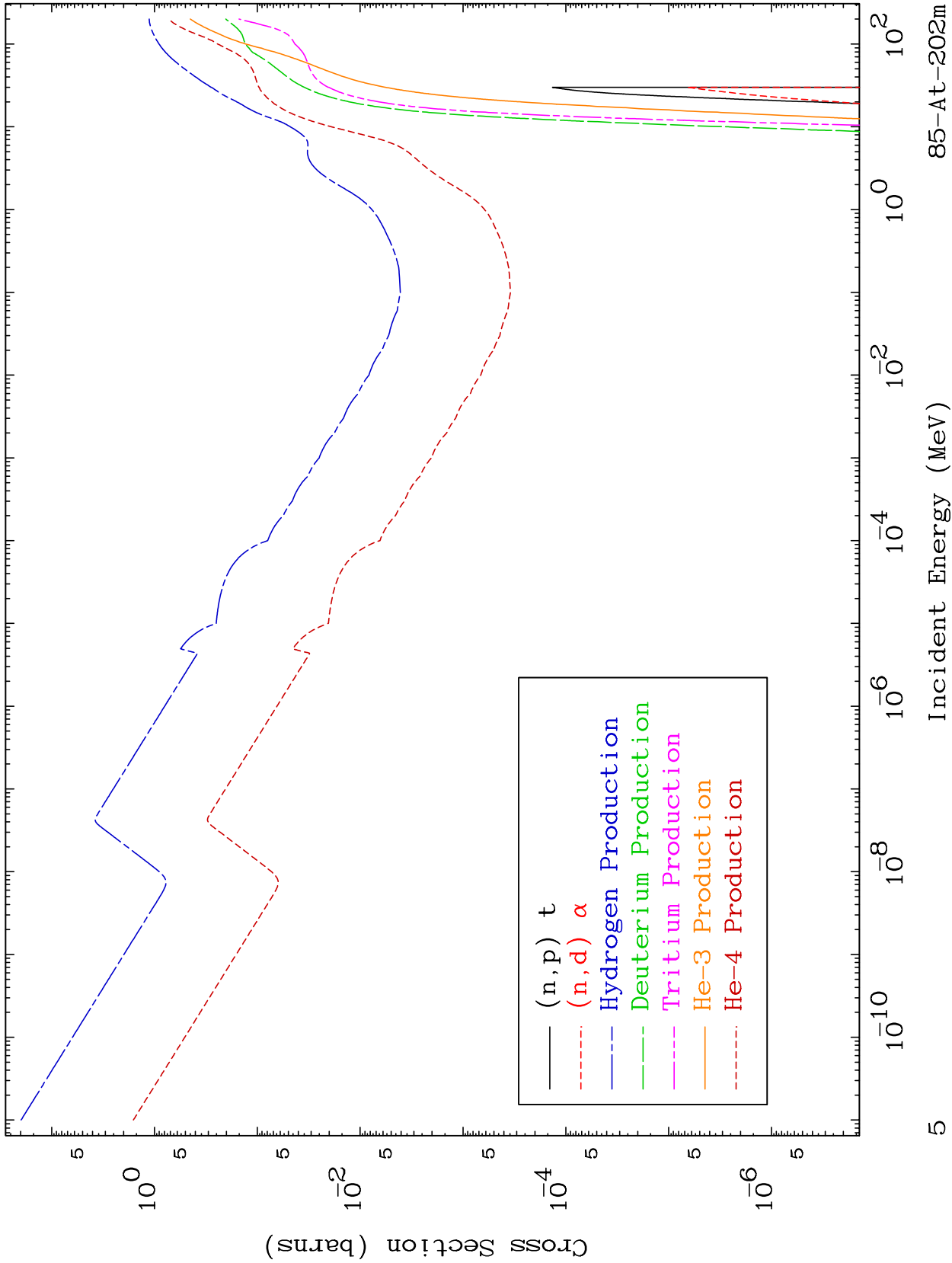


MAT 8523

Neutron Absorption  
293 Kelvin Cross Sections

85-At-202m

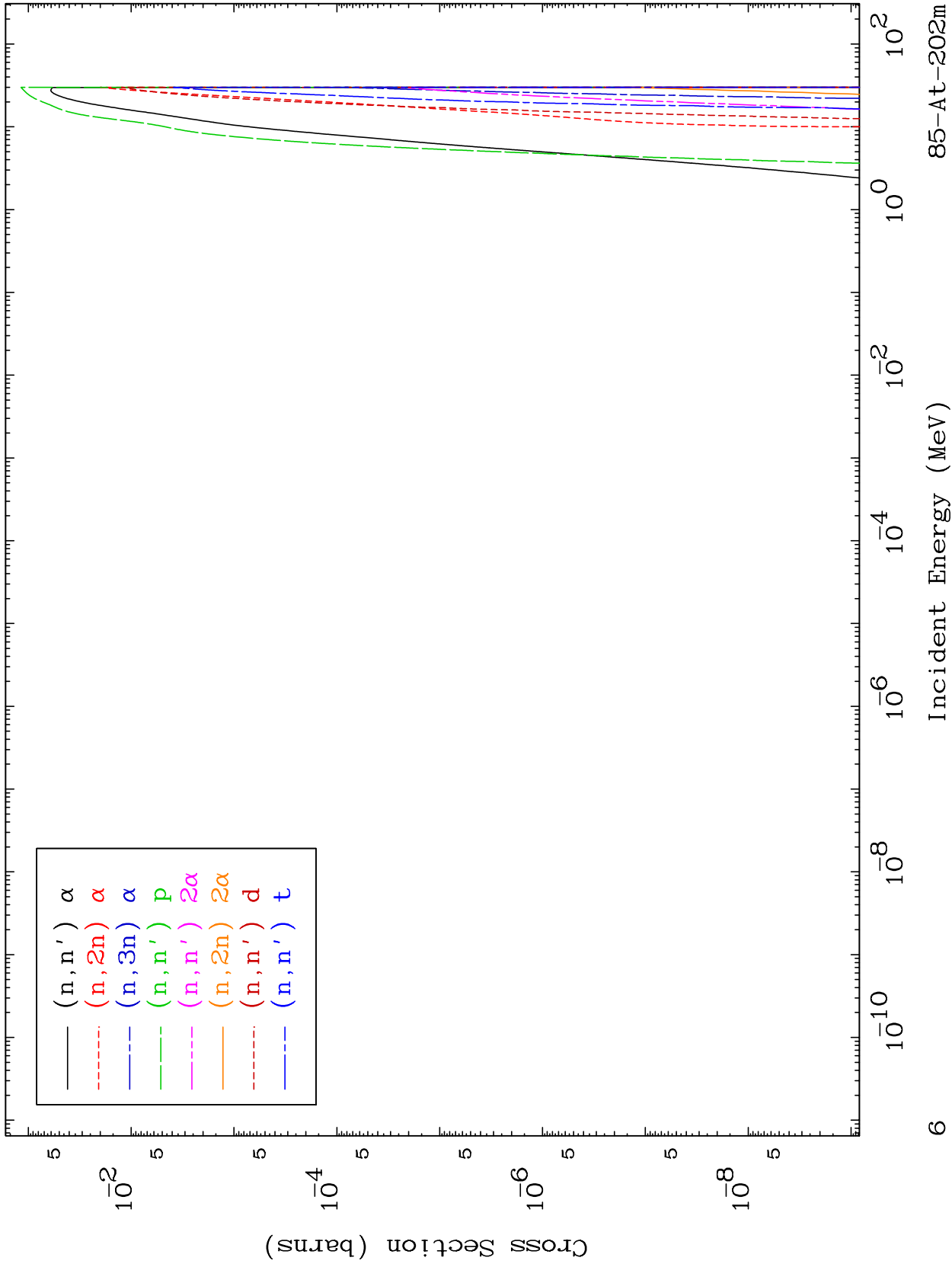




MAT 8523

Charged Particle  
293 Kelvin Cross Sections

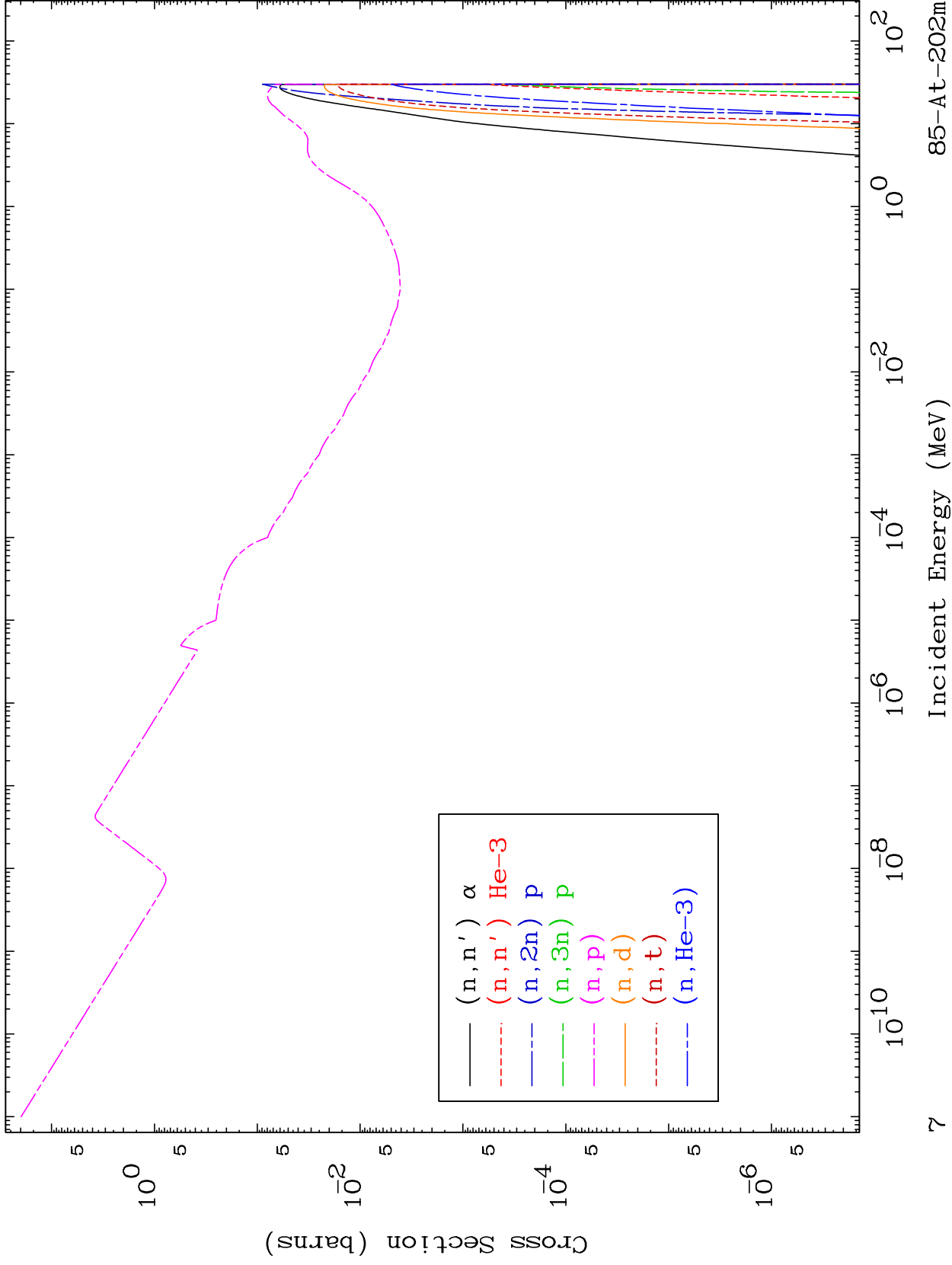
85-At-202m



MAT 8523

Charged Particle  
293 Kelvin Cross Sections

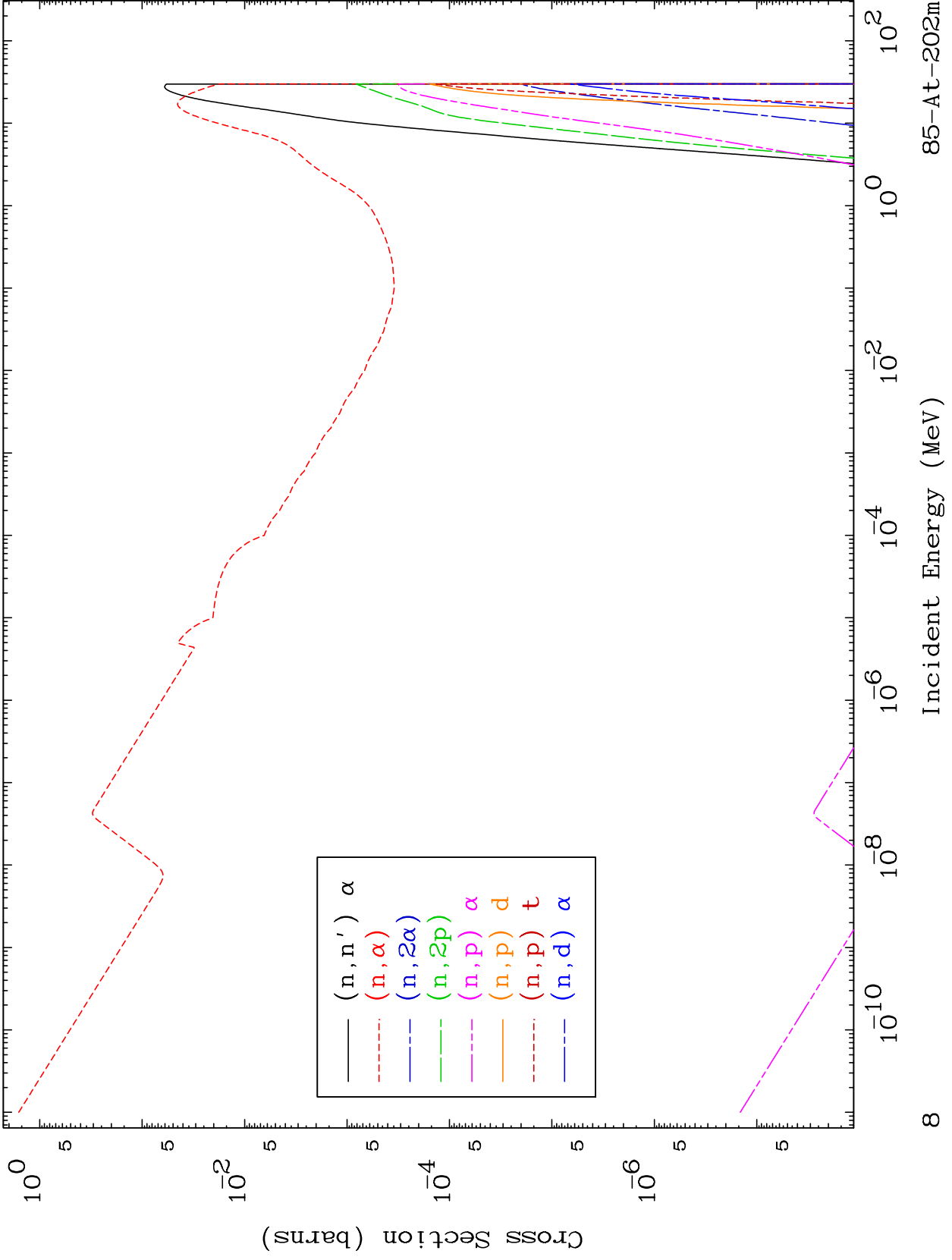
85-At-202m



MAT 8523

Charged Particle  
293 Kelvin Cross Sections

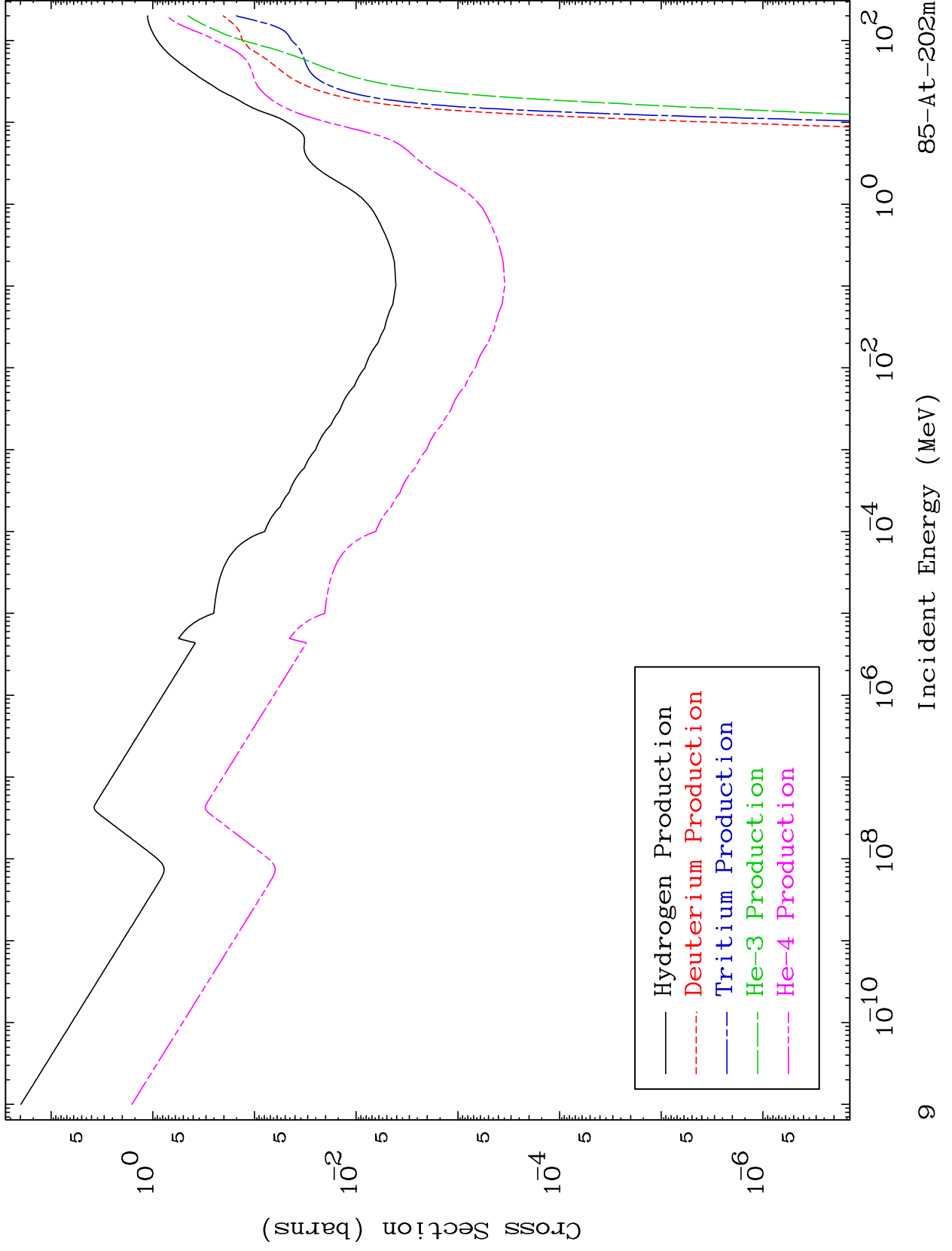
85-At-202m



MAT 8523

Particle Production  
293 Kelvin Cross Sections

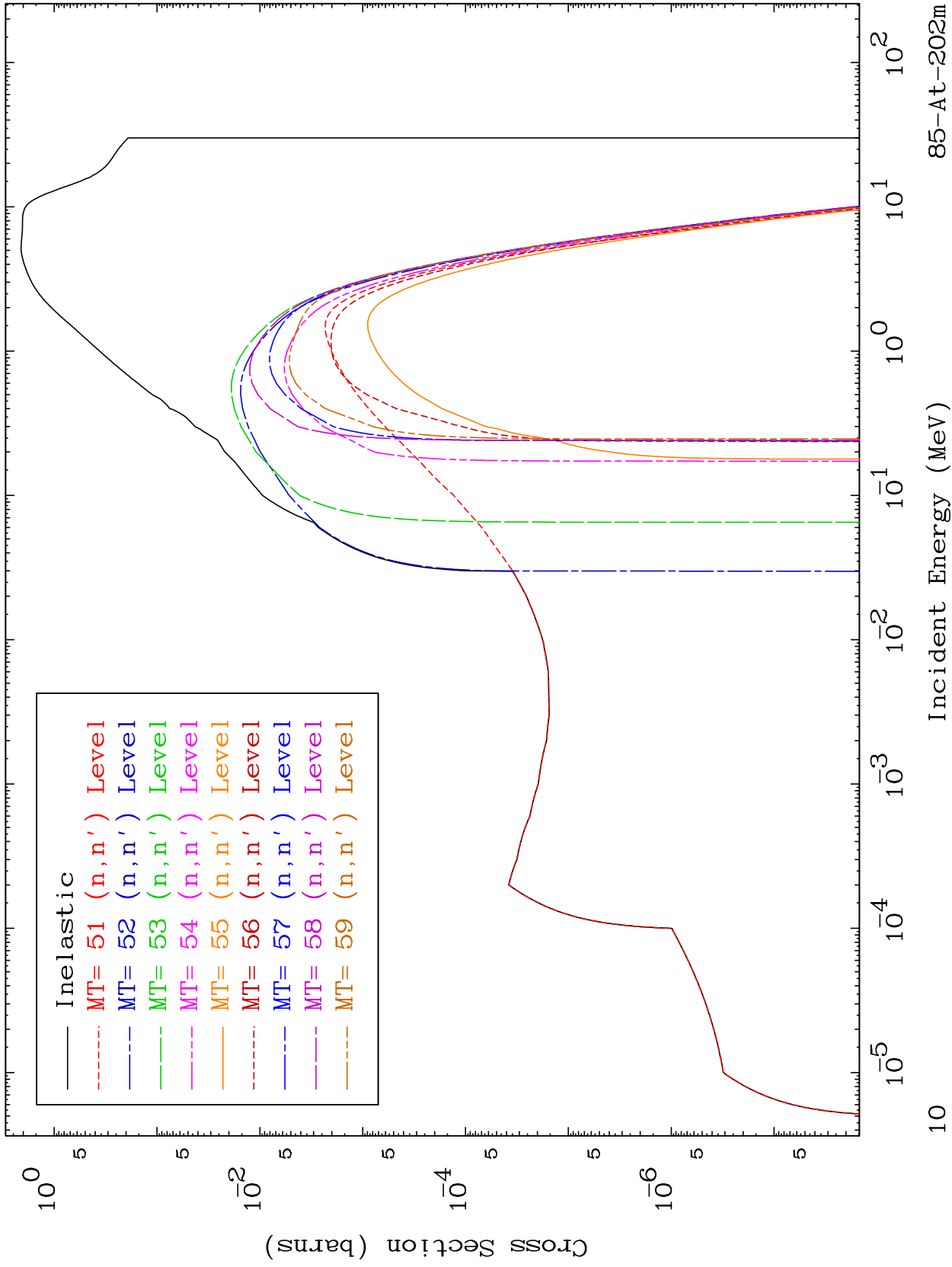
85-At-202m



MAT 8523

(n,n') Levels  
293 Kelvin Cross Sections

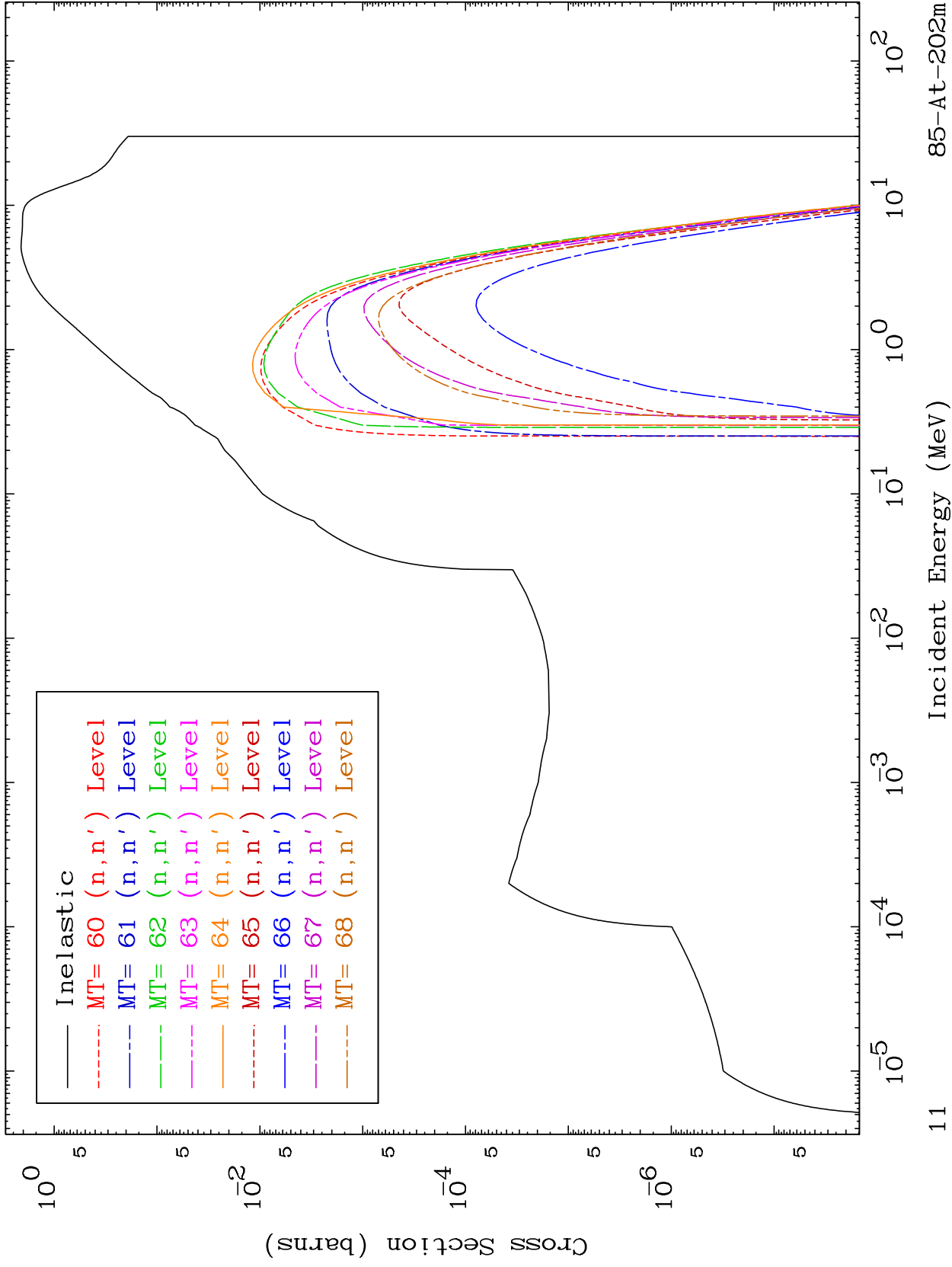
85-At-202m



MAT 8523

(n,n') Levels  
293 Kelvin Cross Sections

85-At-202m

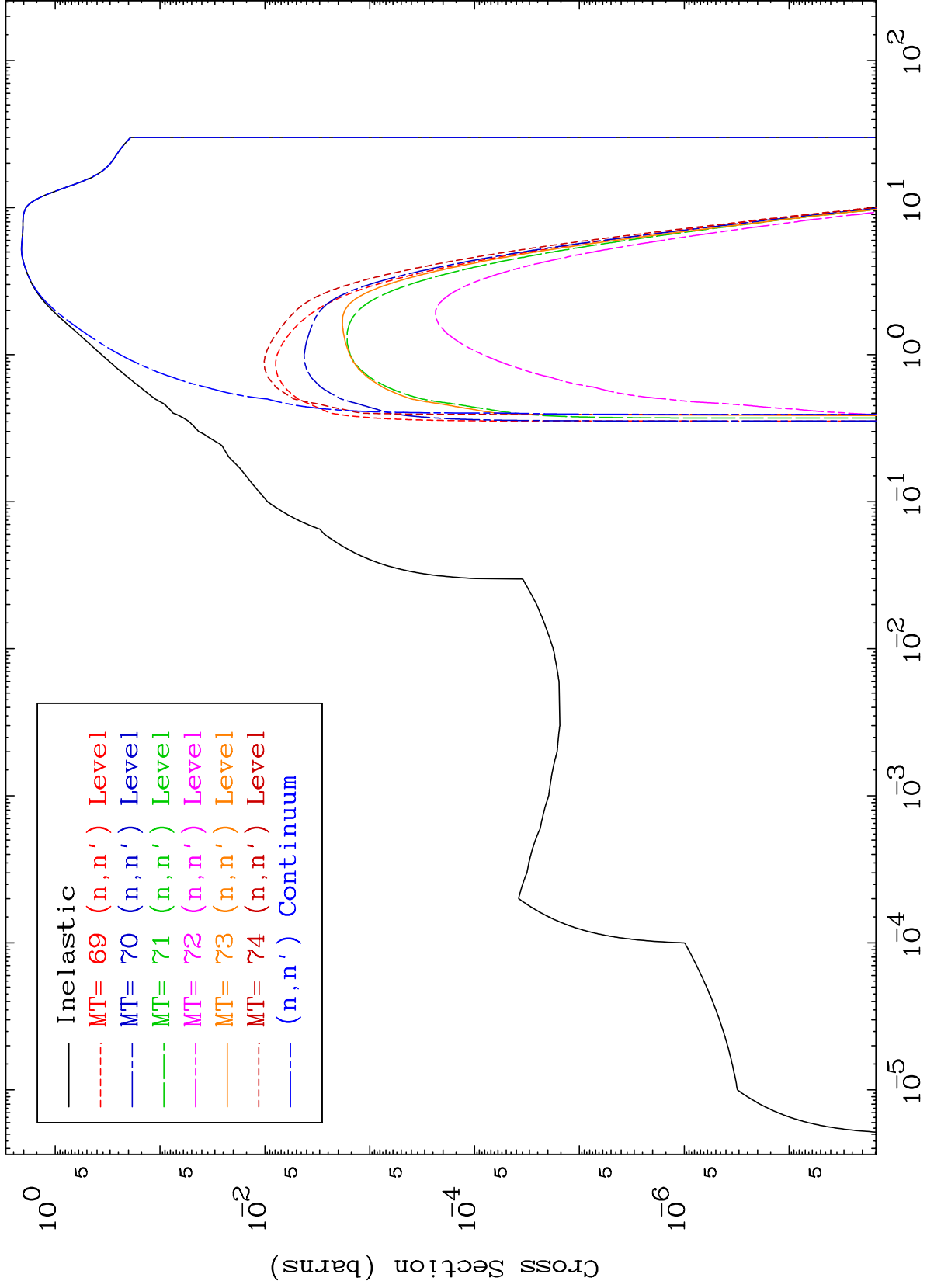


MAT 8523

(n,n') Levels

85-At-202m

293 Kelvin Cross Sections



12

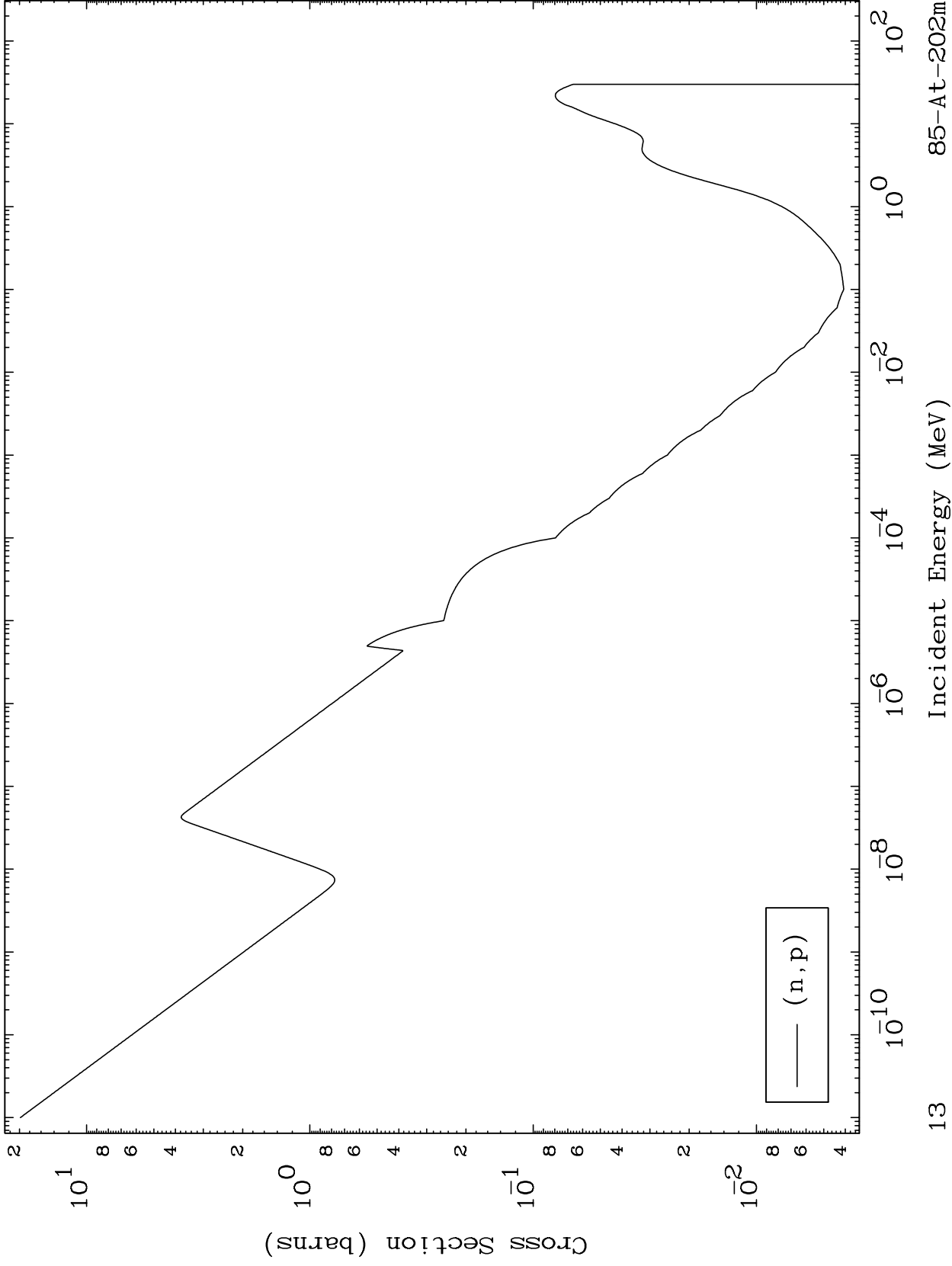
Incident Energy (MeV)

85-At-202m

MAT 8523

(n,p) Levels  
293 Kelvin Cross Sections

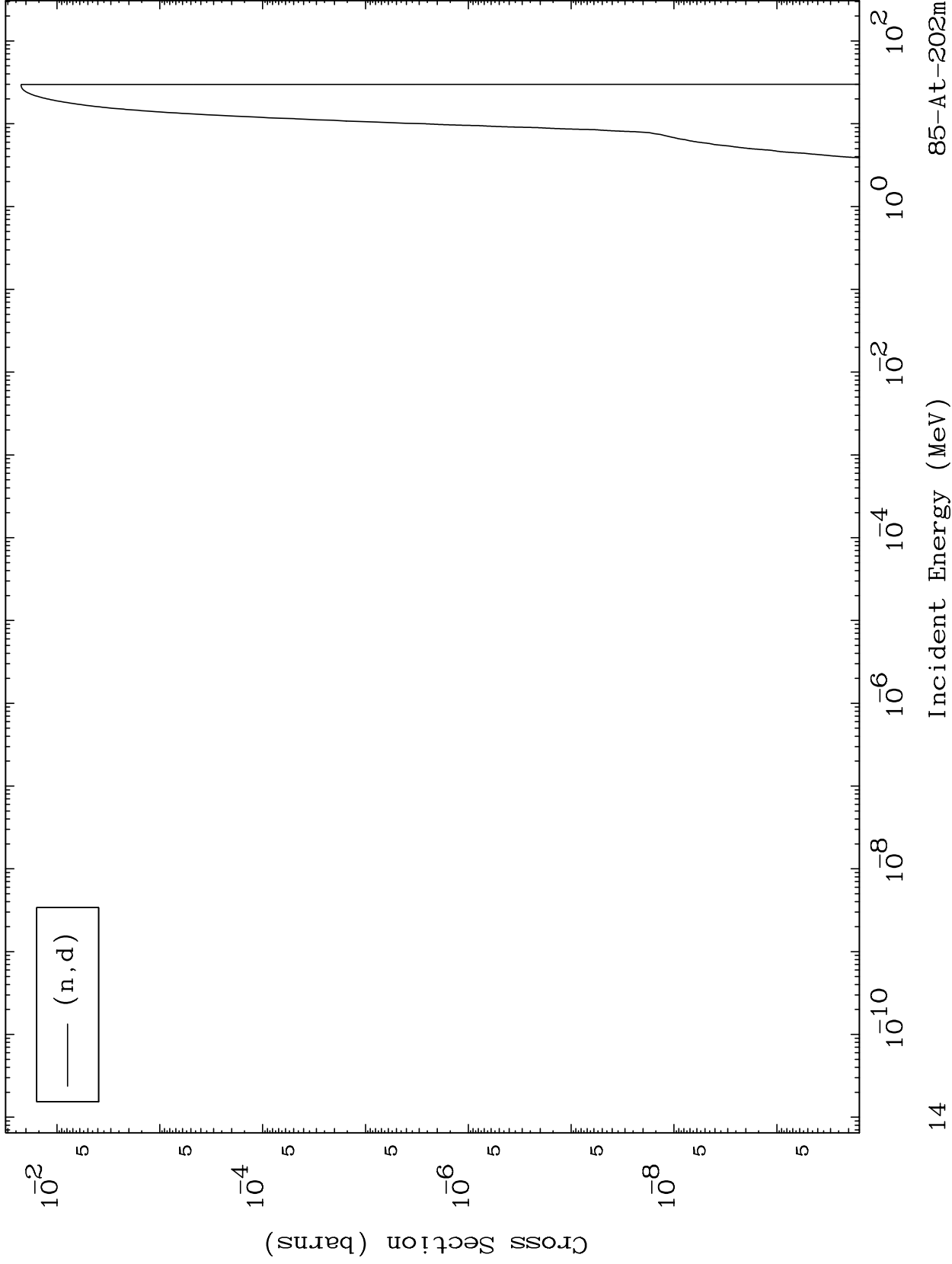
85-At-202m



MAT 8523

(n,d) Levels  
293 Kelvin Cross Sections

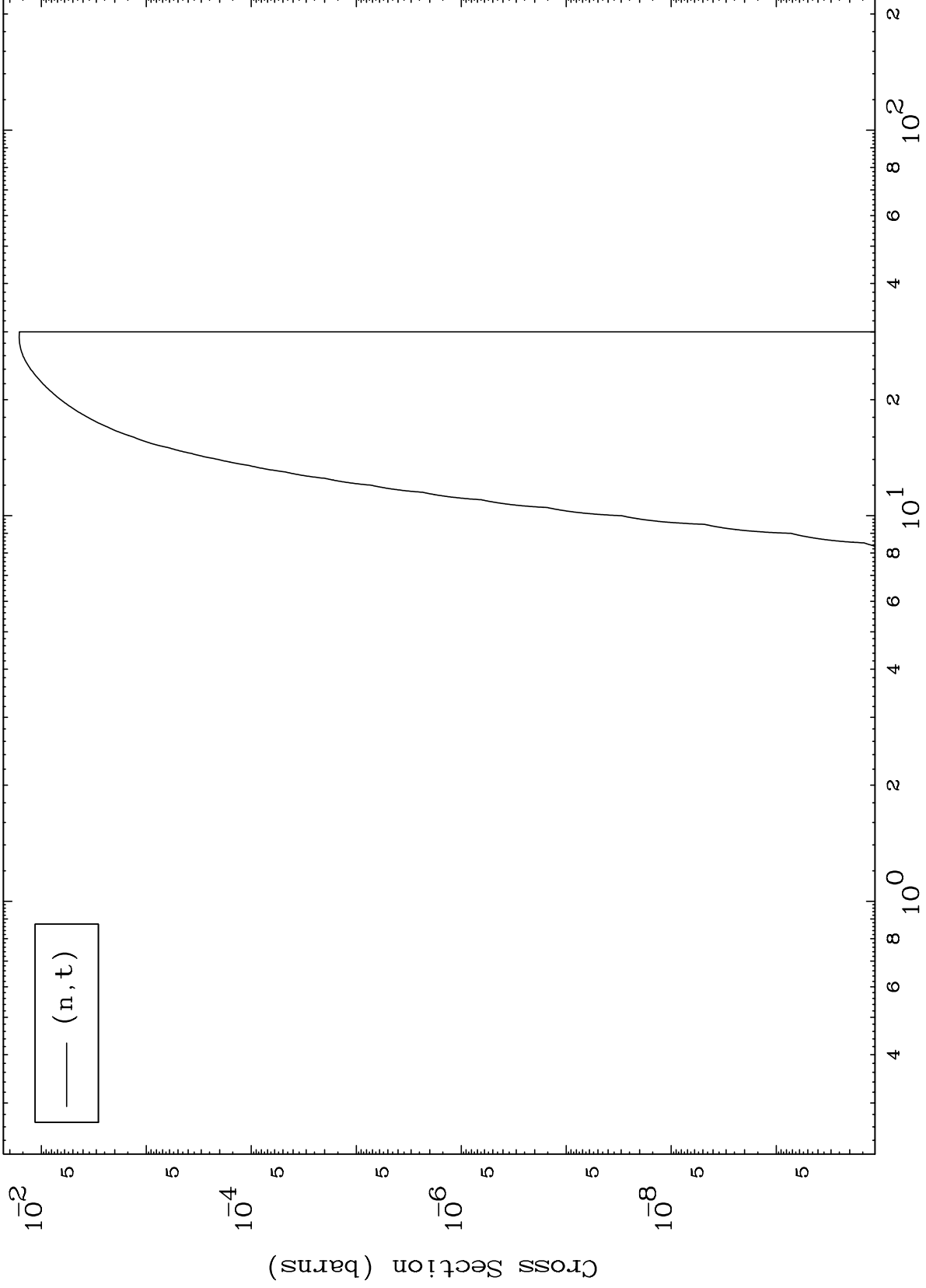
85-At-202m



MAT 8523

(n,t) Levels  
293 Kelvin Cross Sections

85-At-202m



15

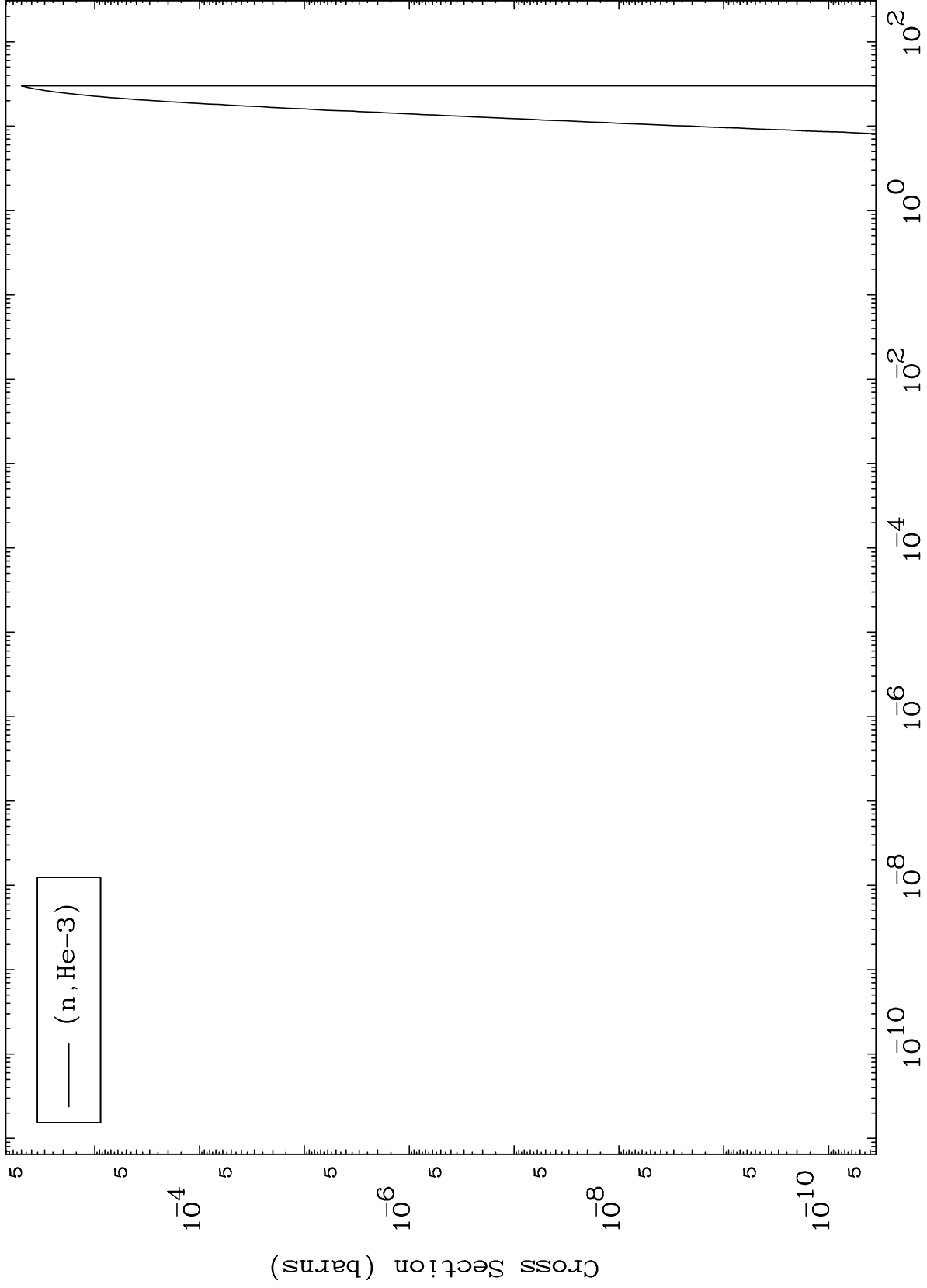
Incident Energy (MeV)

85-At-202m

MAT 8523

(n,He3) Levels  
293 Kelvin Cross Sections

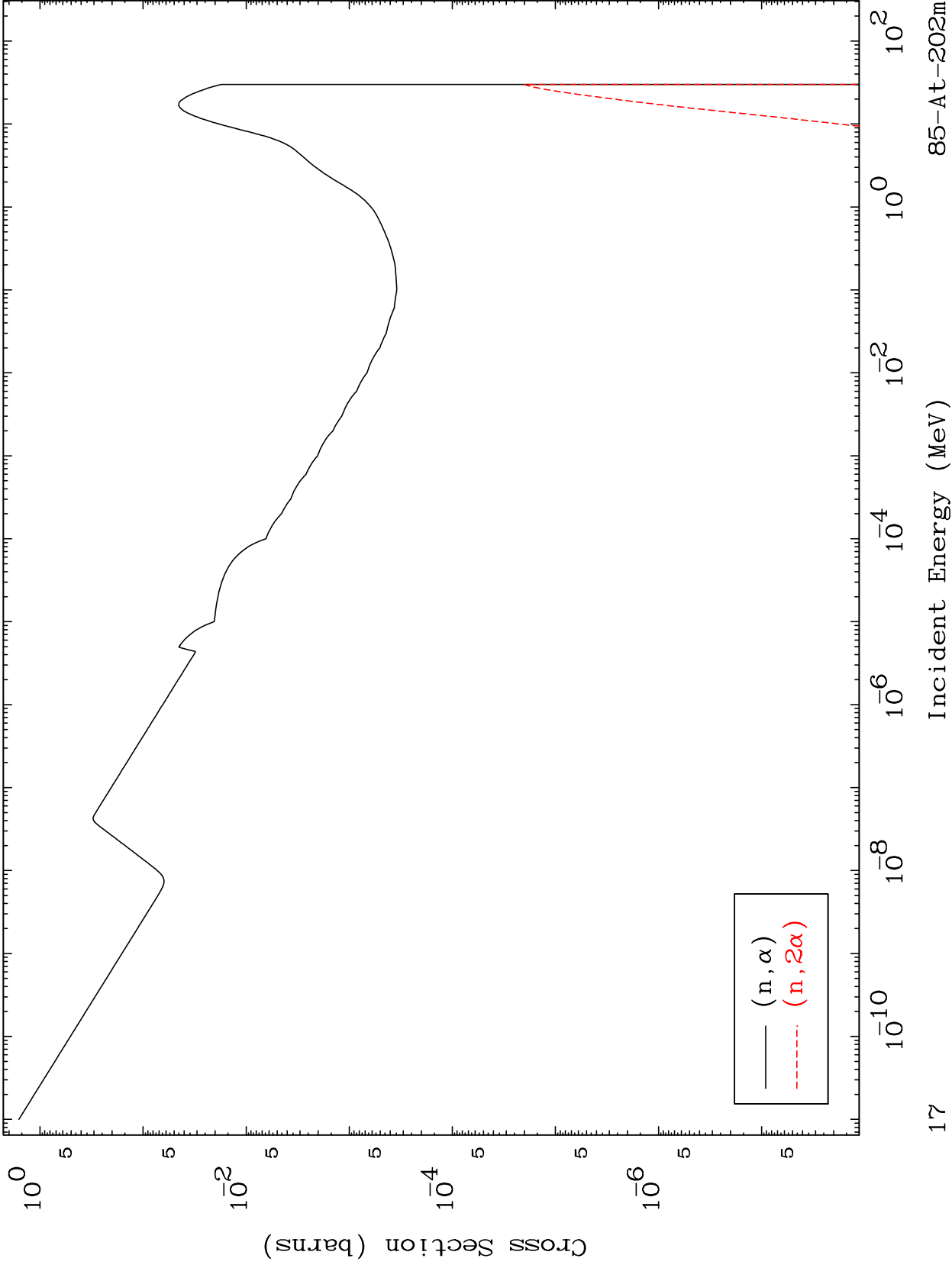
85-At-202m



MAT 8523

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

85-At-202m

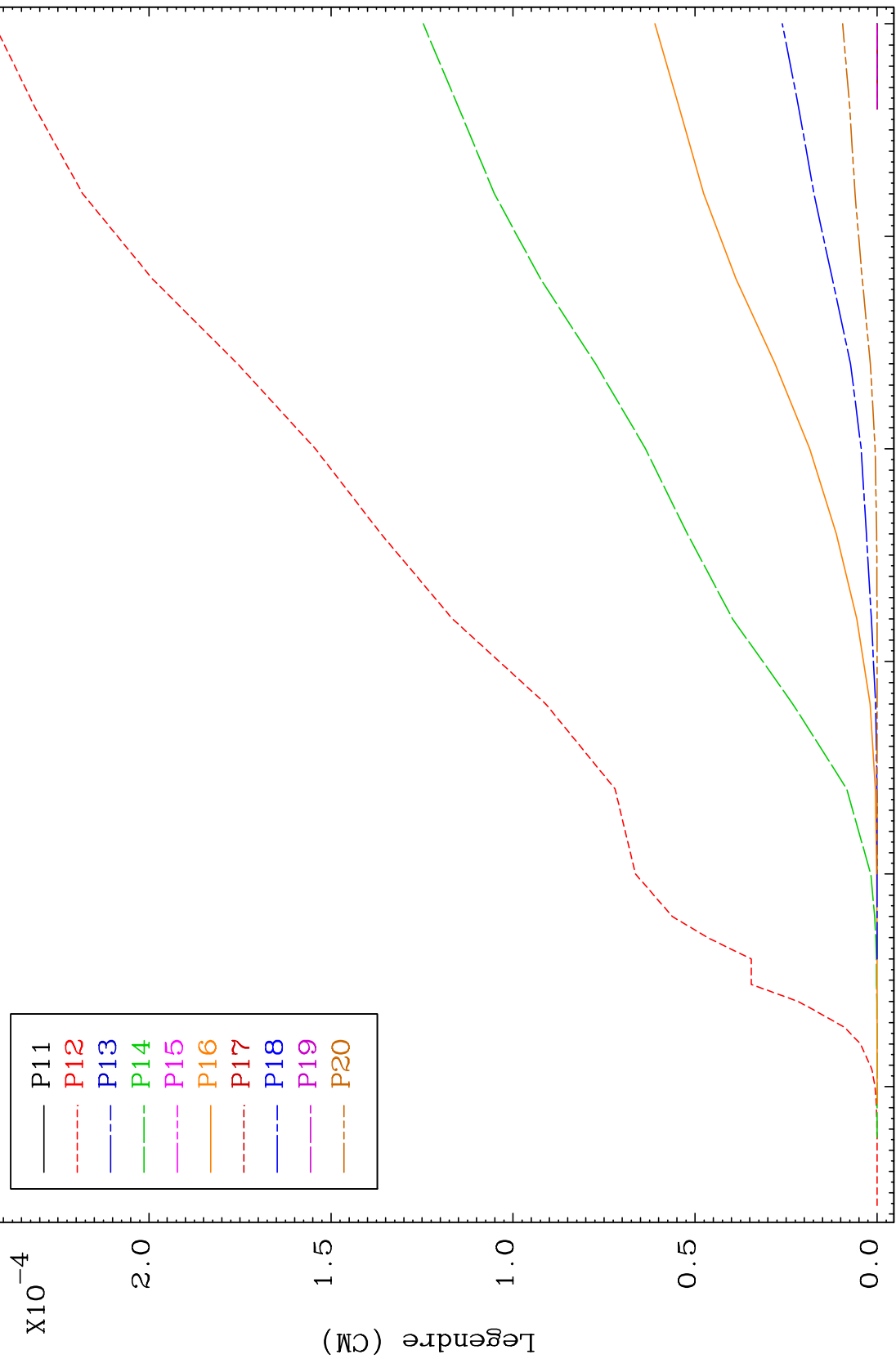
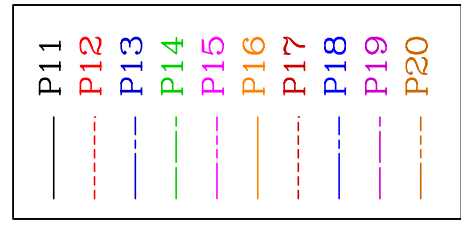




MAT 8523

Elastic Legendre Coefficients

85-At-202m



19

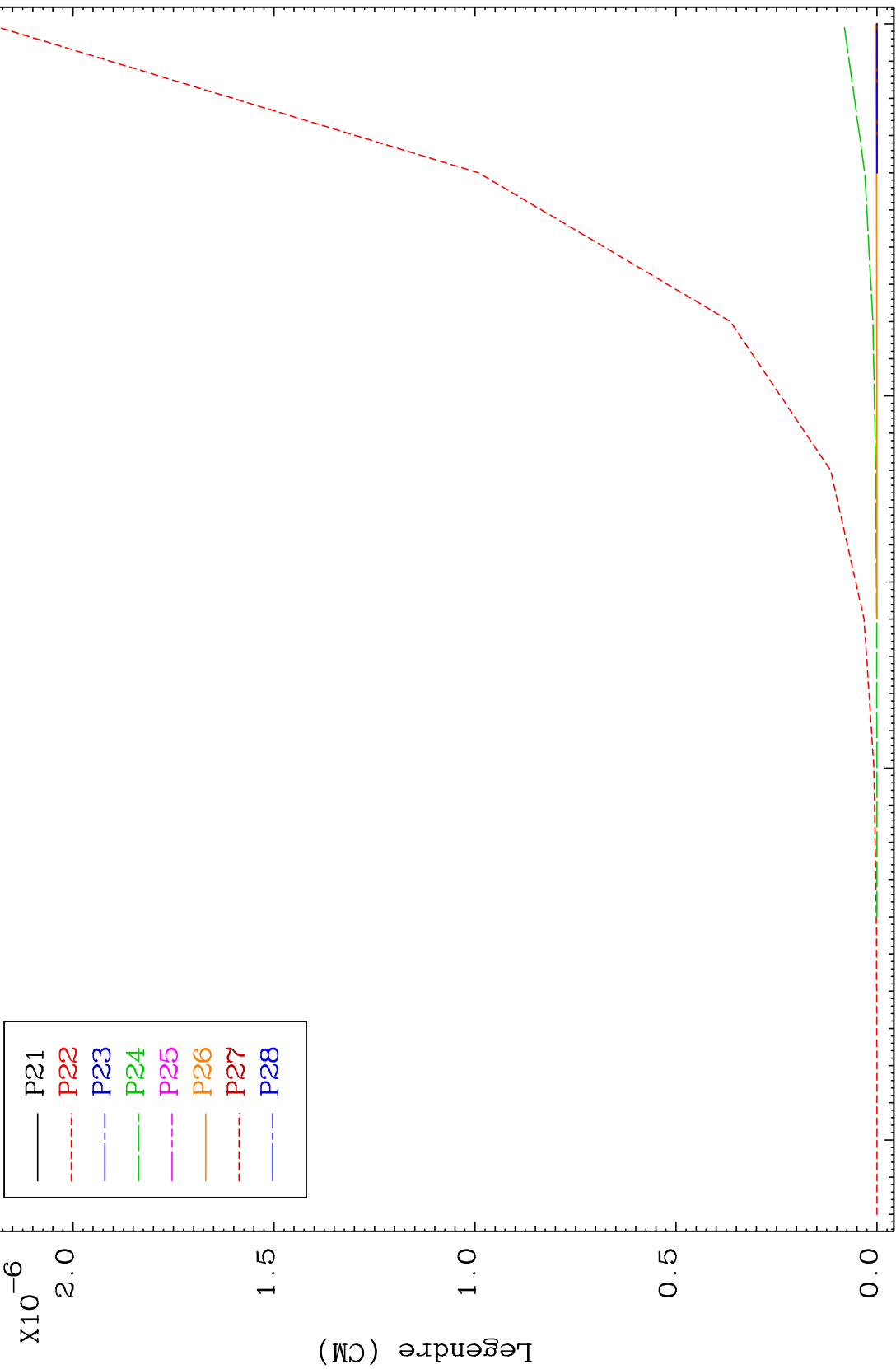
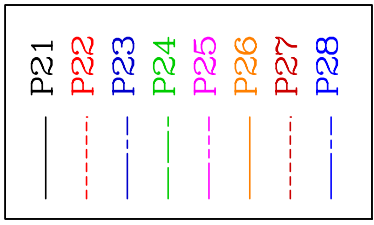
85-At-202m

Incident Energy (MeV)

MAT 8523

Elastic Legendre Coefficients

85-At-202m



20

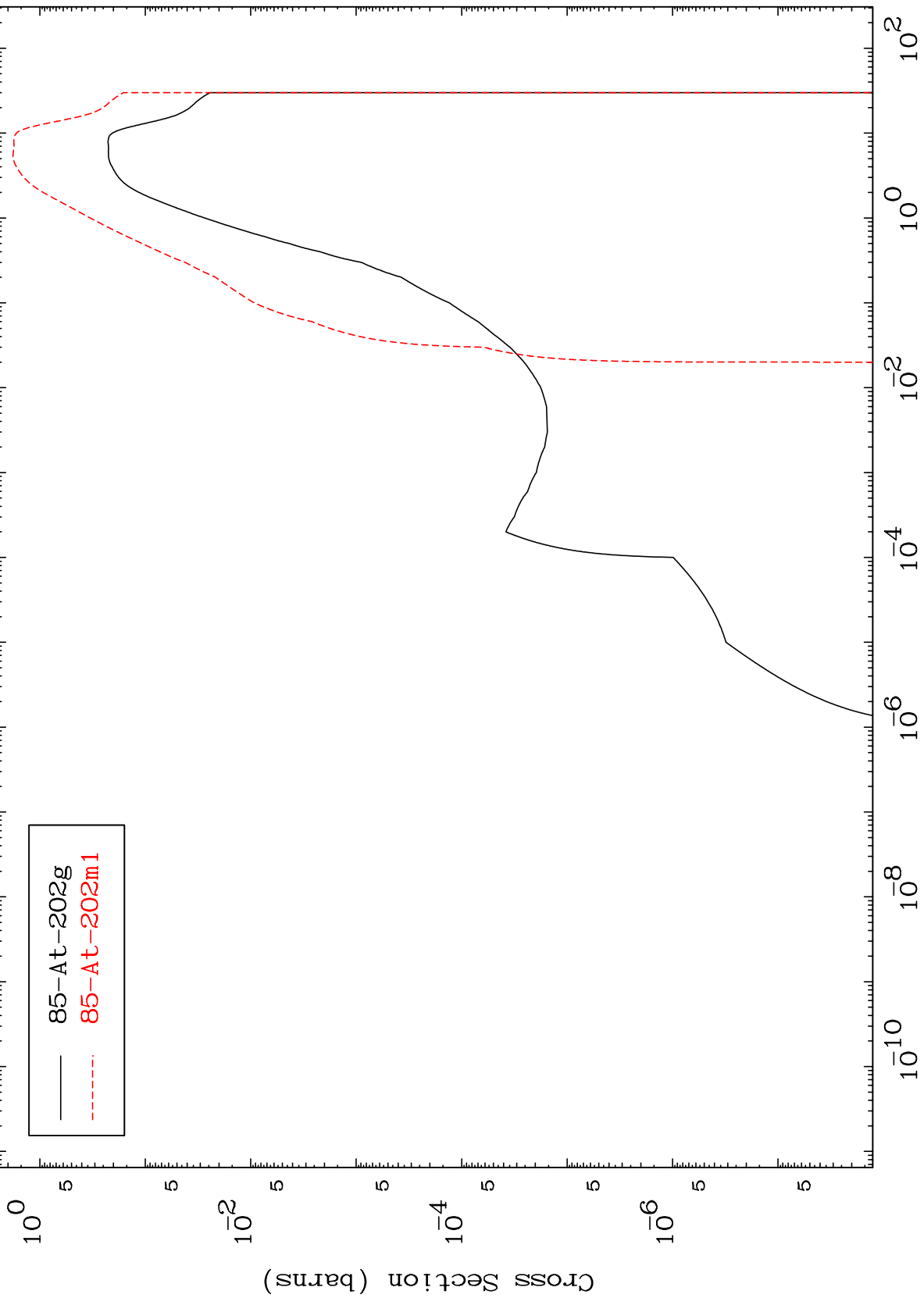
Incident Energy (MeV)

85-At-202m

MAT 8523

85-At-202m

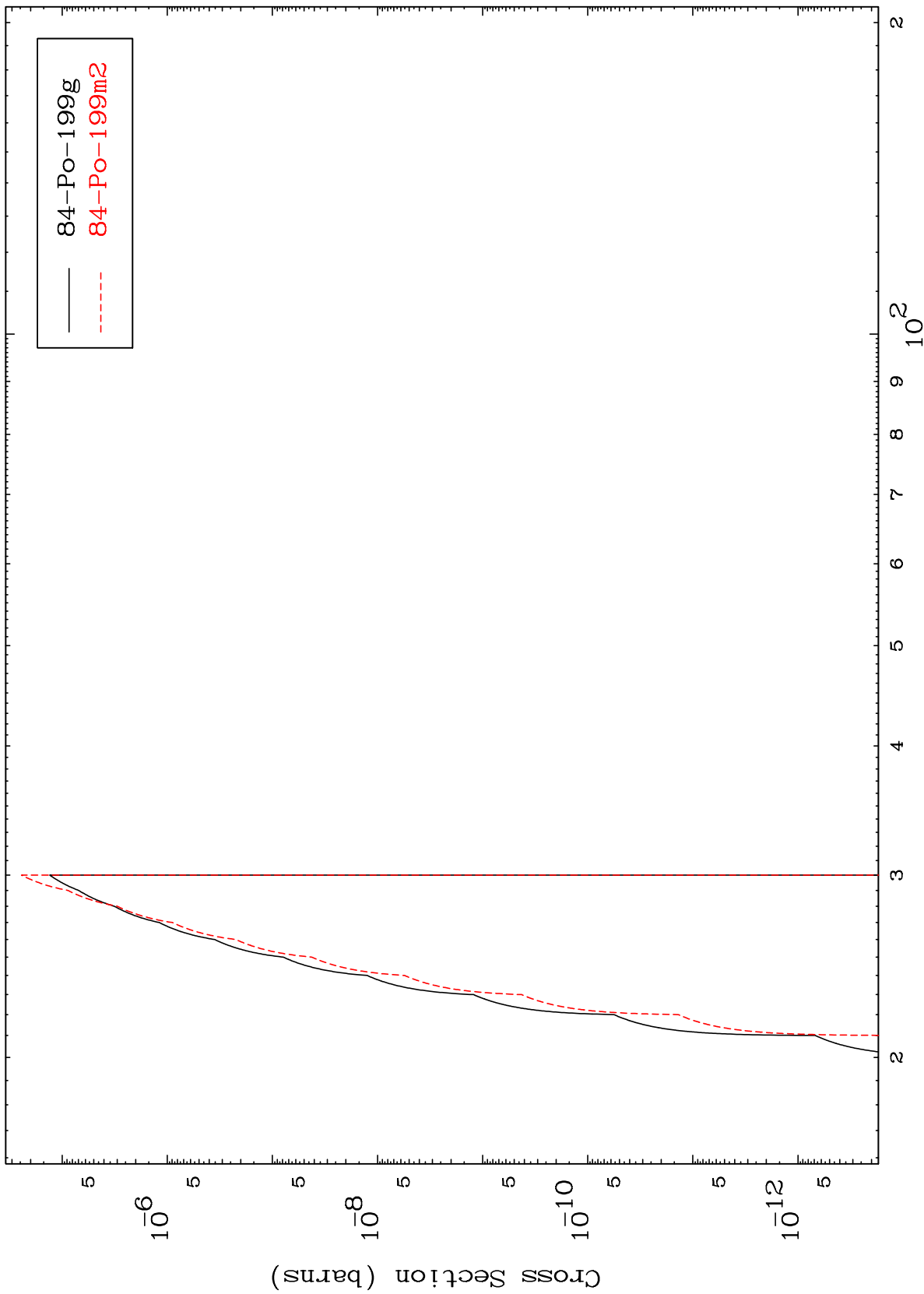
Inelastic  
Radionuclide Production Cross Section



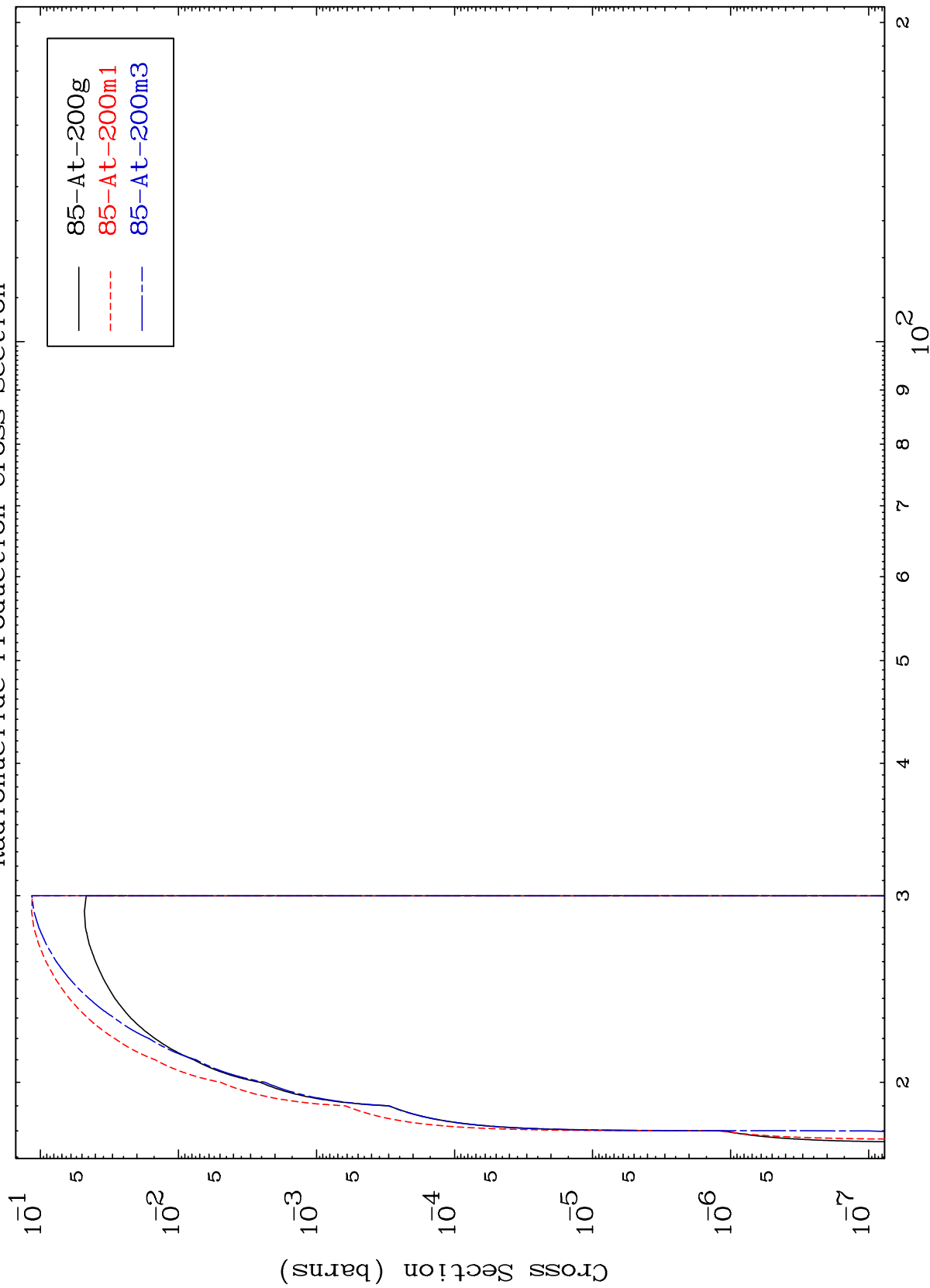
85-At-202m

Incident Energy (MeV)

Radionuclide Production Cross Section



(n,3n)  
Radionuclide Production Cross Section

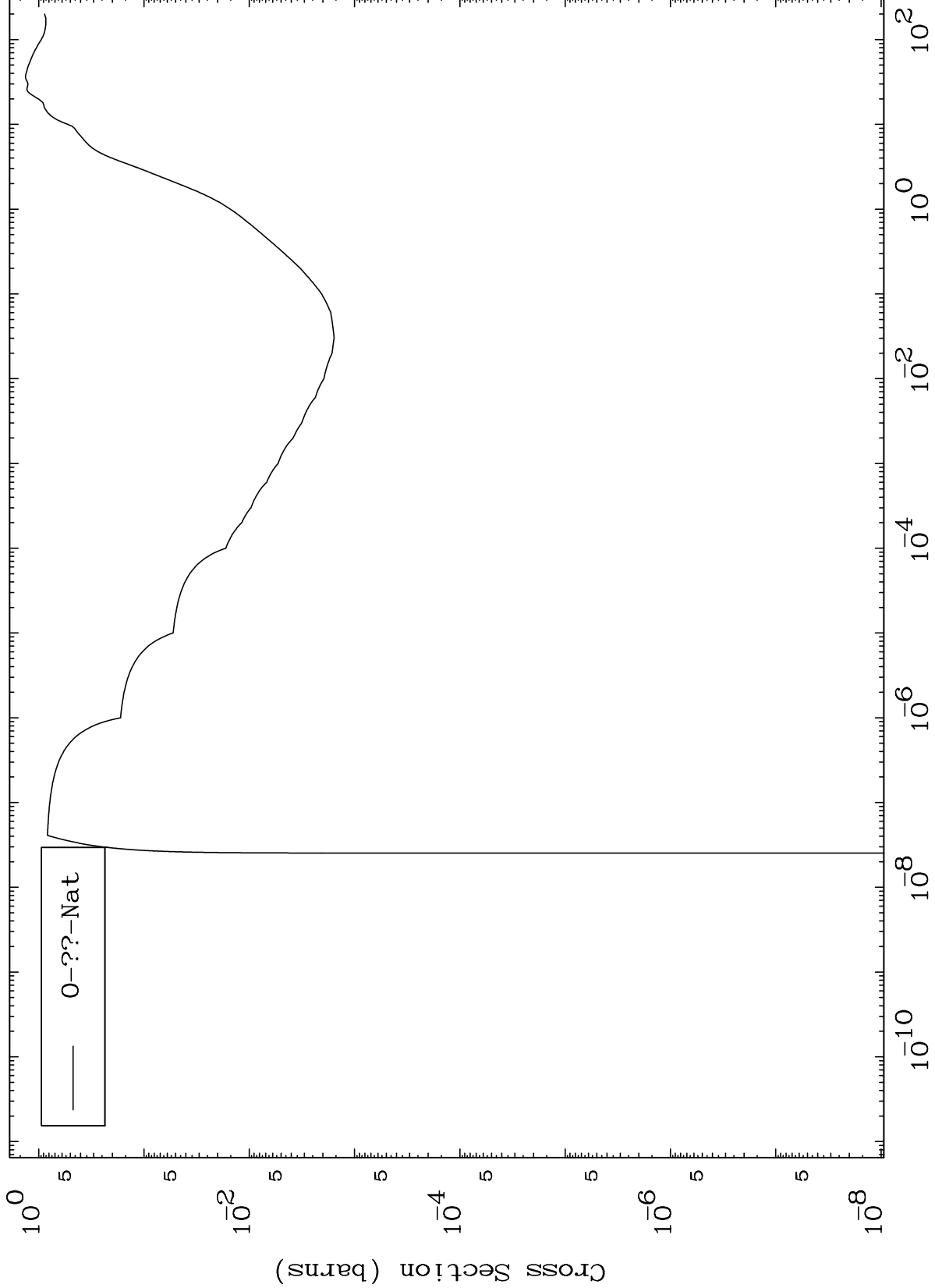


MAT 8523

Fission

85-At-202m

Radionuclide Production Cross Section

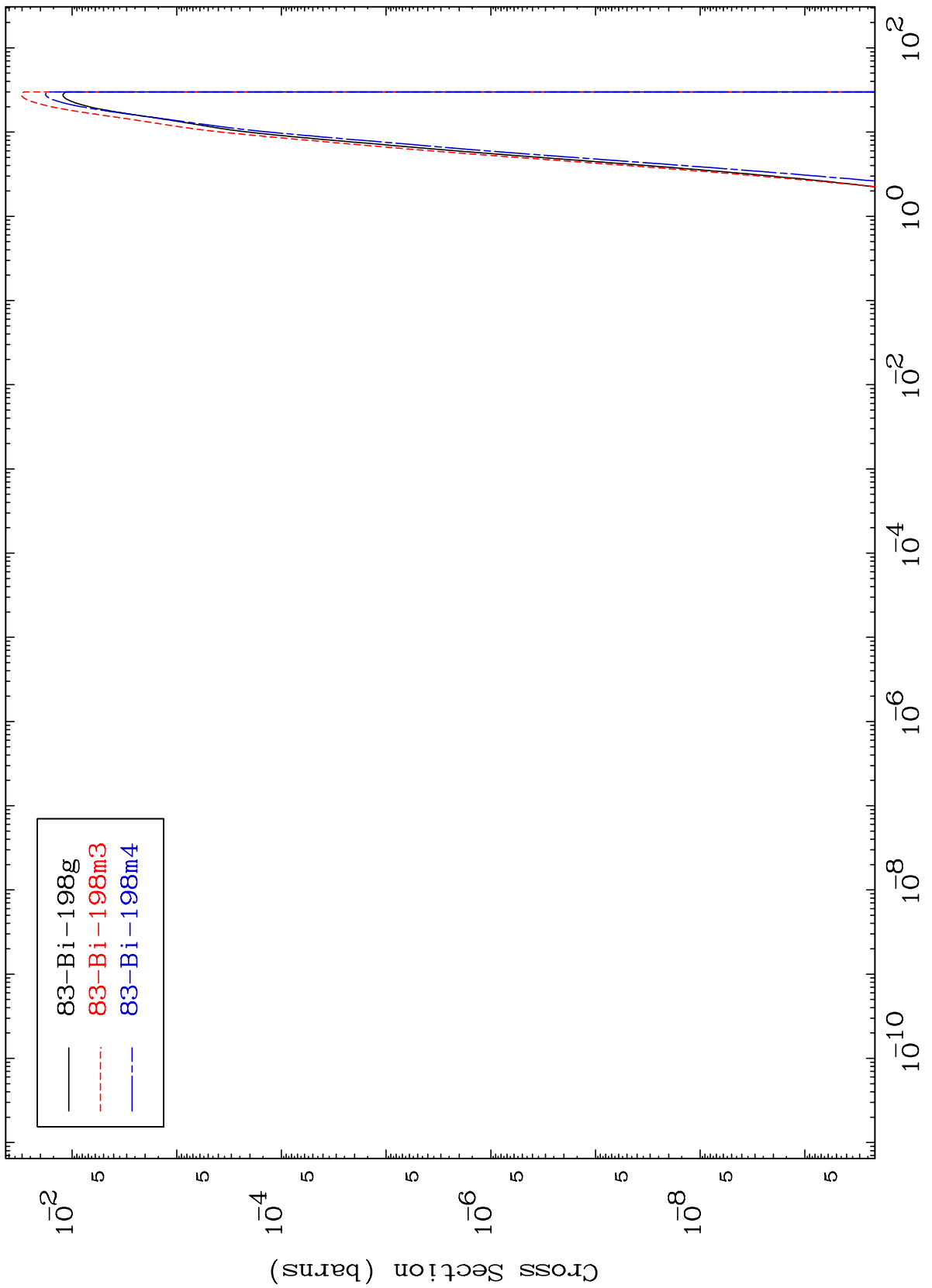


MAT 8523

$(n, n') \alpha$

85-At-202m

Radionuclide Production Cross Section

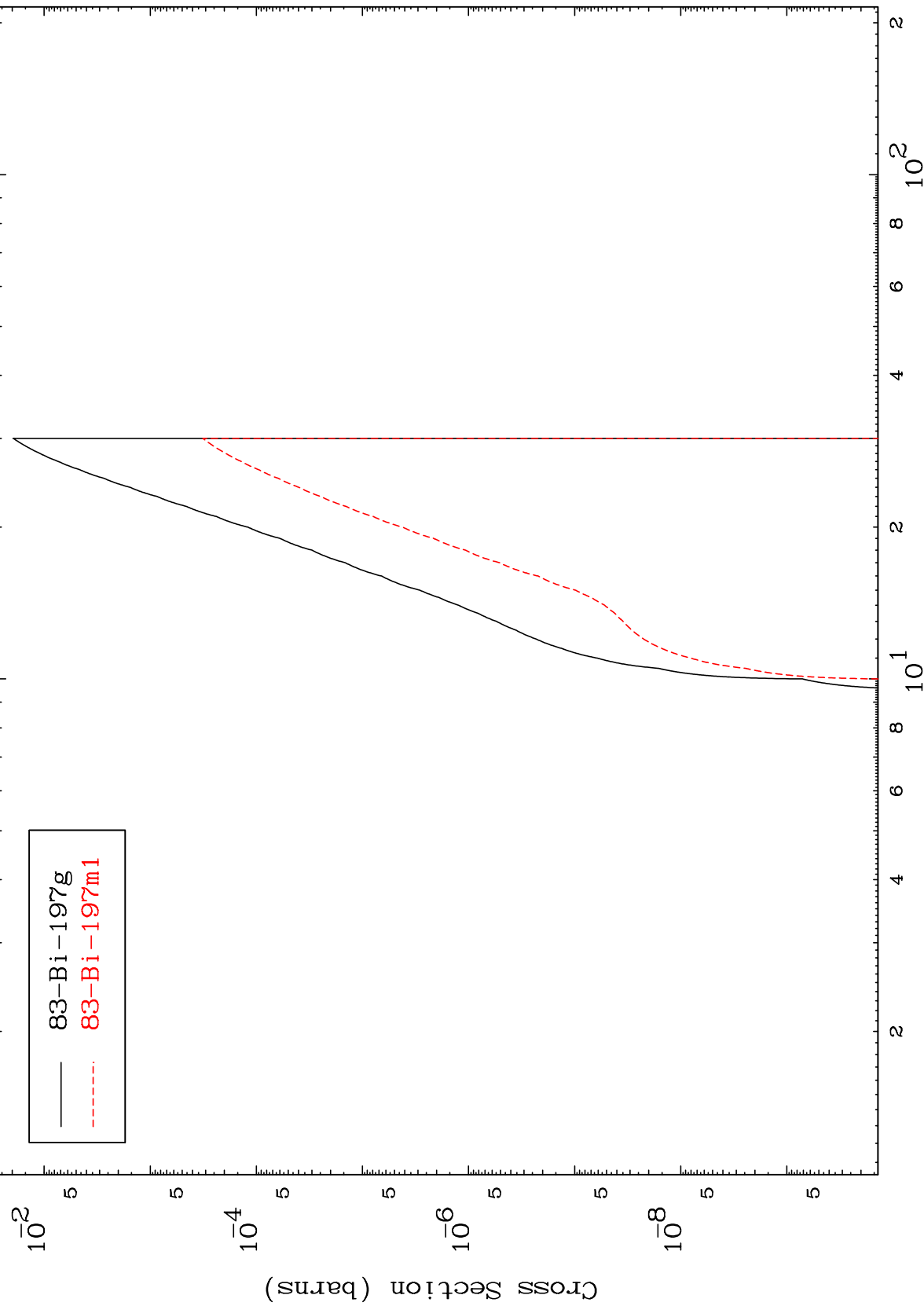


MAT 8523

(n,2n)  $\alpha$

85-At-202m

Radionuclide Production Cross Section



— 83-Bi-197g  
- - - 83-Bi-197m1

Incident Energy (MeV)

85-At-202m

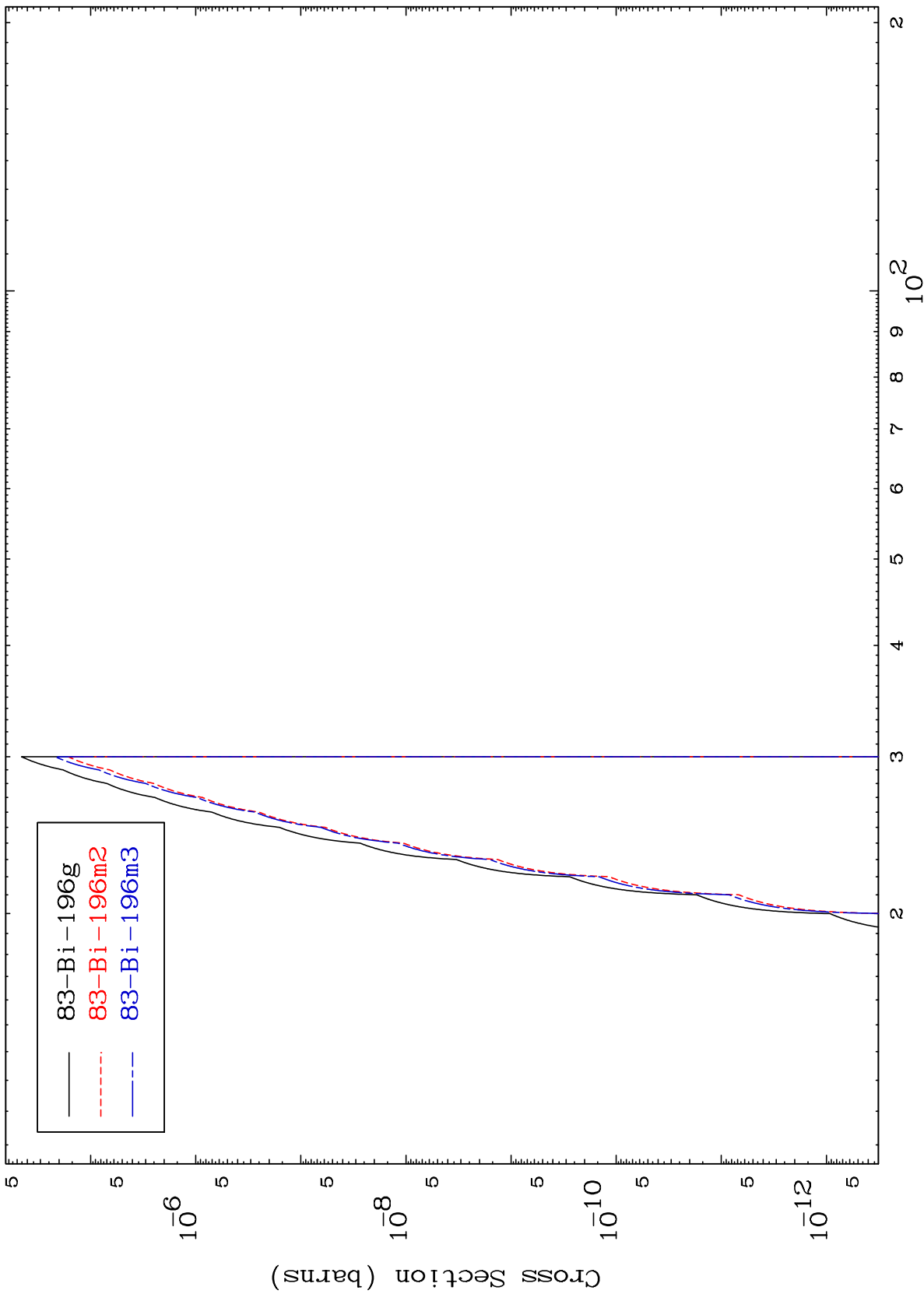
26

MAT 8523

(n,3n)  $\alpha$

85-At-202m

Radionuclide Production Cross Section



Incident Energy (MeV)

85-At-202m

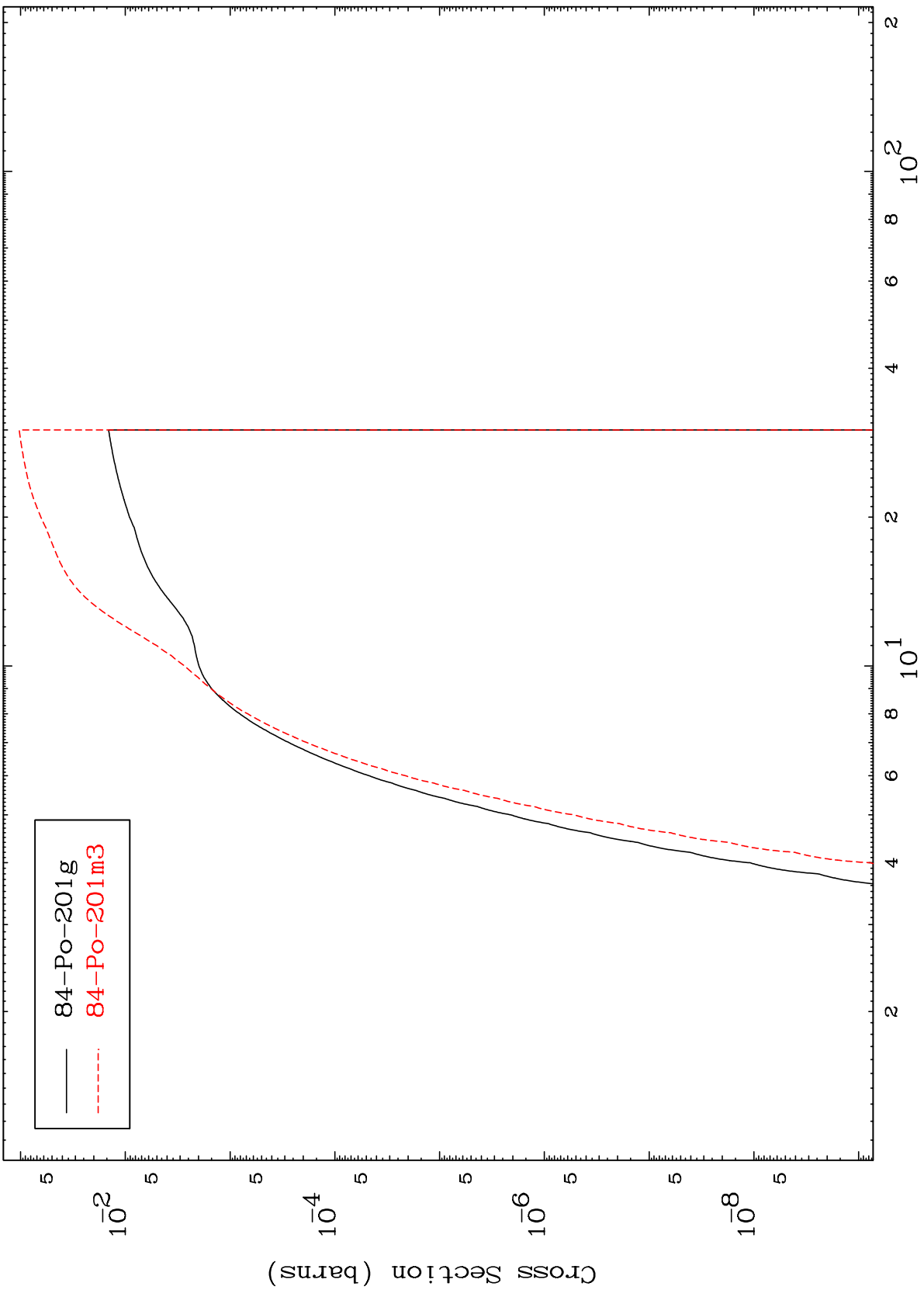
27

MAT 8523

(n,n') p

85-At-202m

Radionuclide Production Cross Section



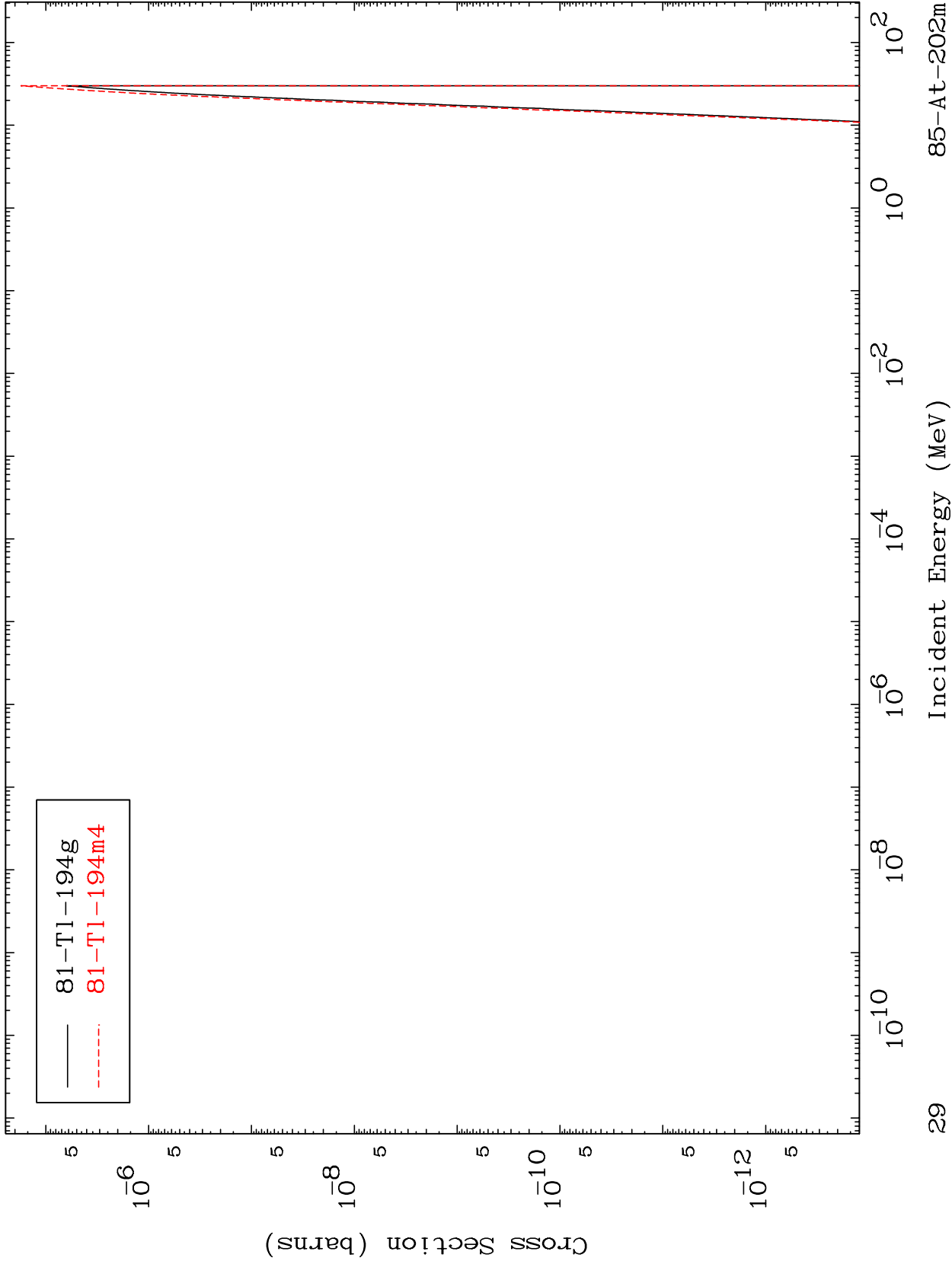
84-Po-201g  
84-Po-201m3

MAT 8523

(n,n') 2α

85-At-202m

Radionuclide Production Cross Section

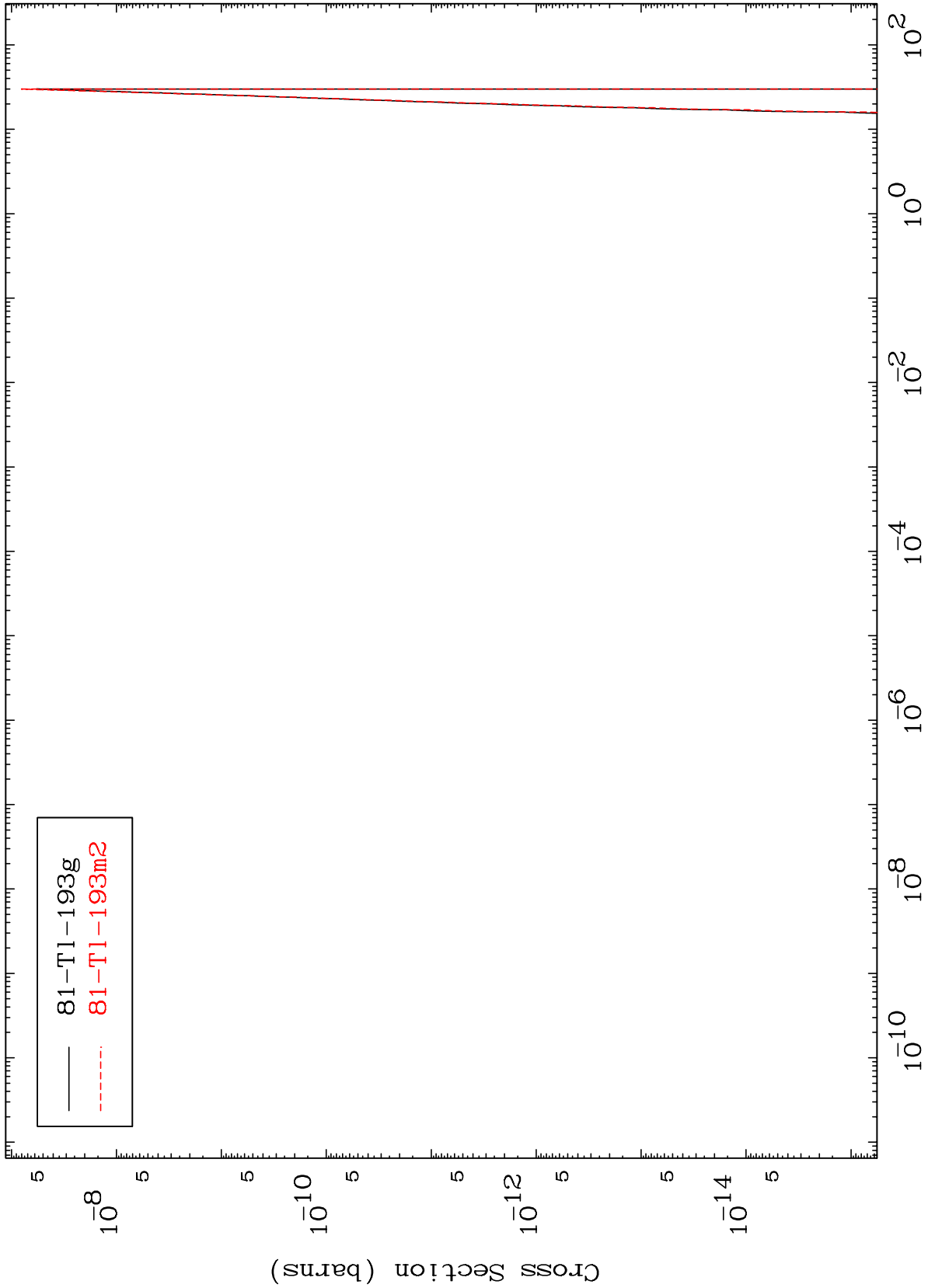


MAT 8523

(n,2n) 2α

85-At-202m

Radionuclide Production Cross Section

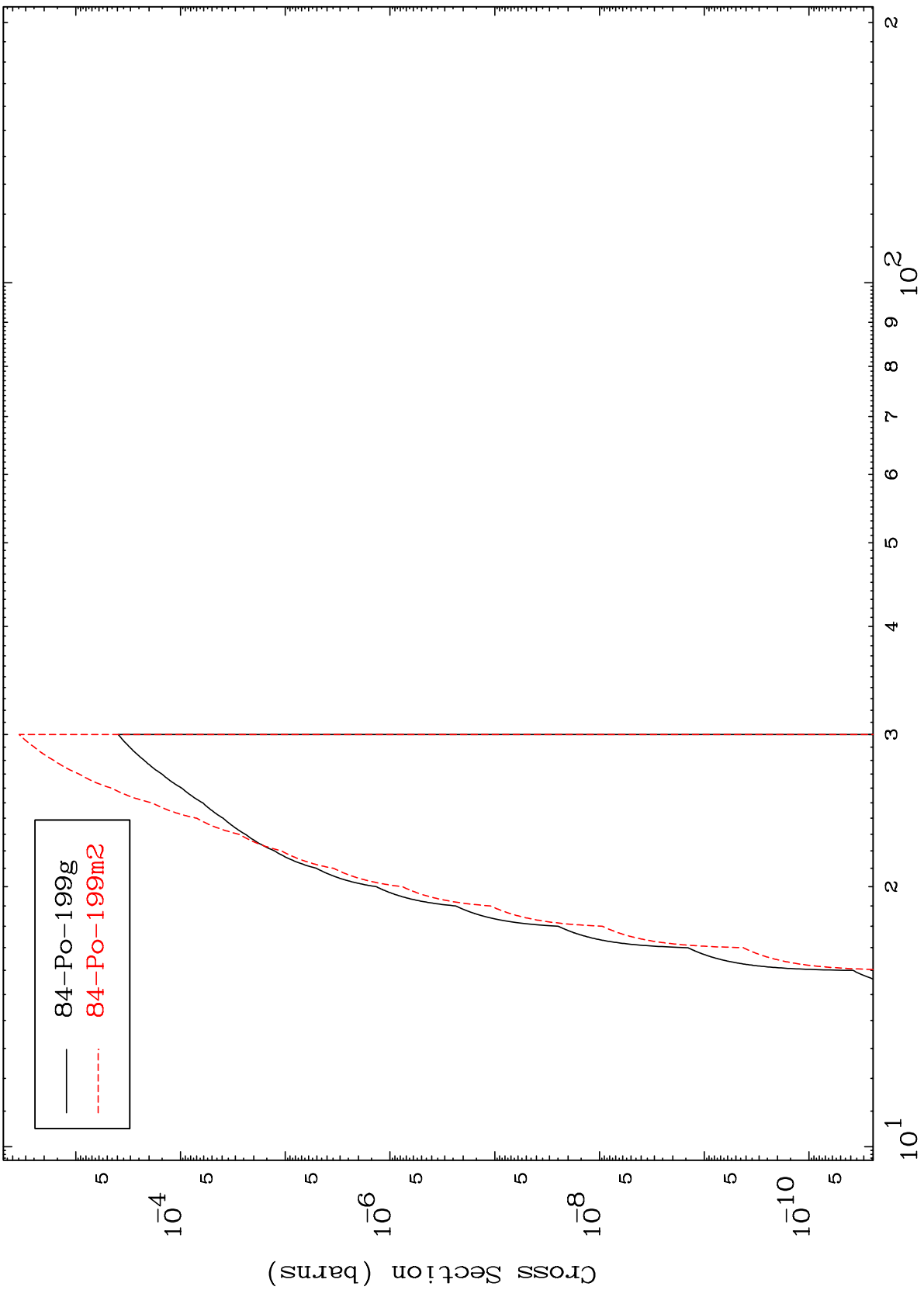


MAT 8523

(n,n') t

85-At-202m

Radionuclide Production Cross Section



Incident Energy (MeV)

85-At-202m

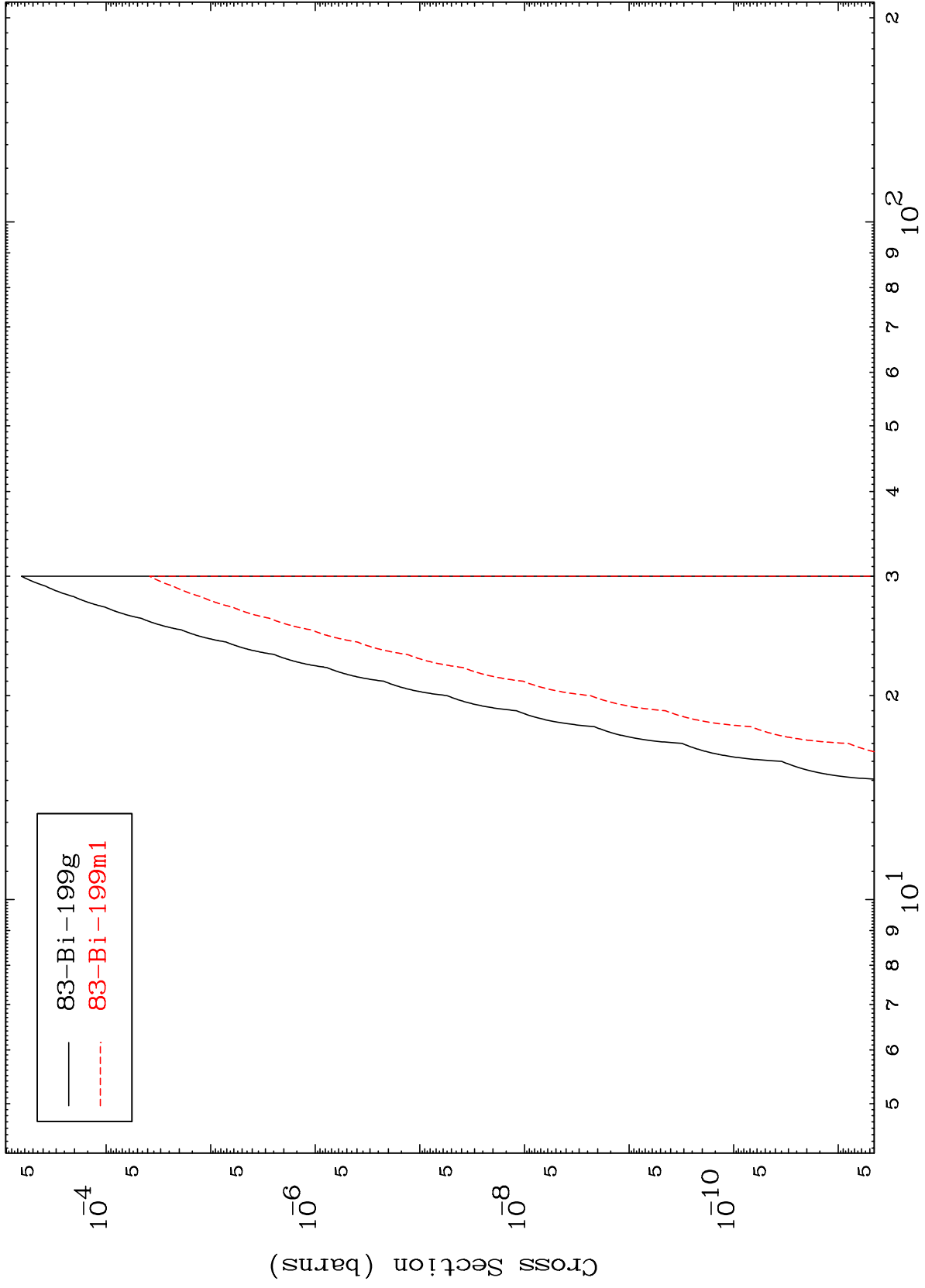
31

MAT 8523

(n,n') He-3

85-At-202m

Radionuclide Production Cross Section



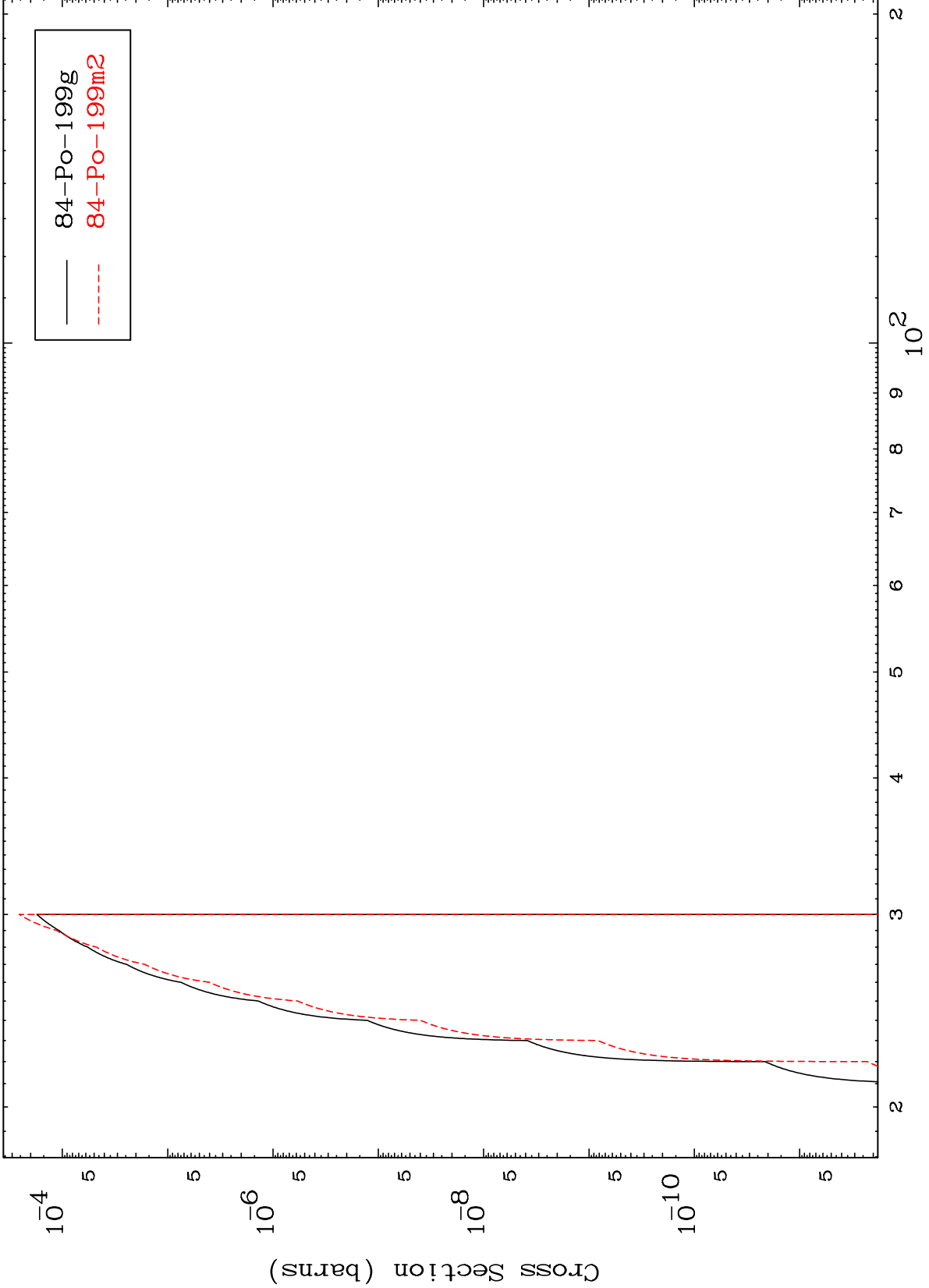
83-Bi-199g  
83-Bi-199m1

32

Incident Energy (MeV)

85-At-202m

Radionuclide Production Cross Section

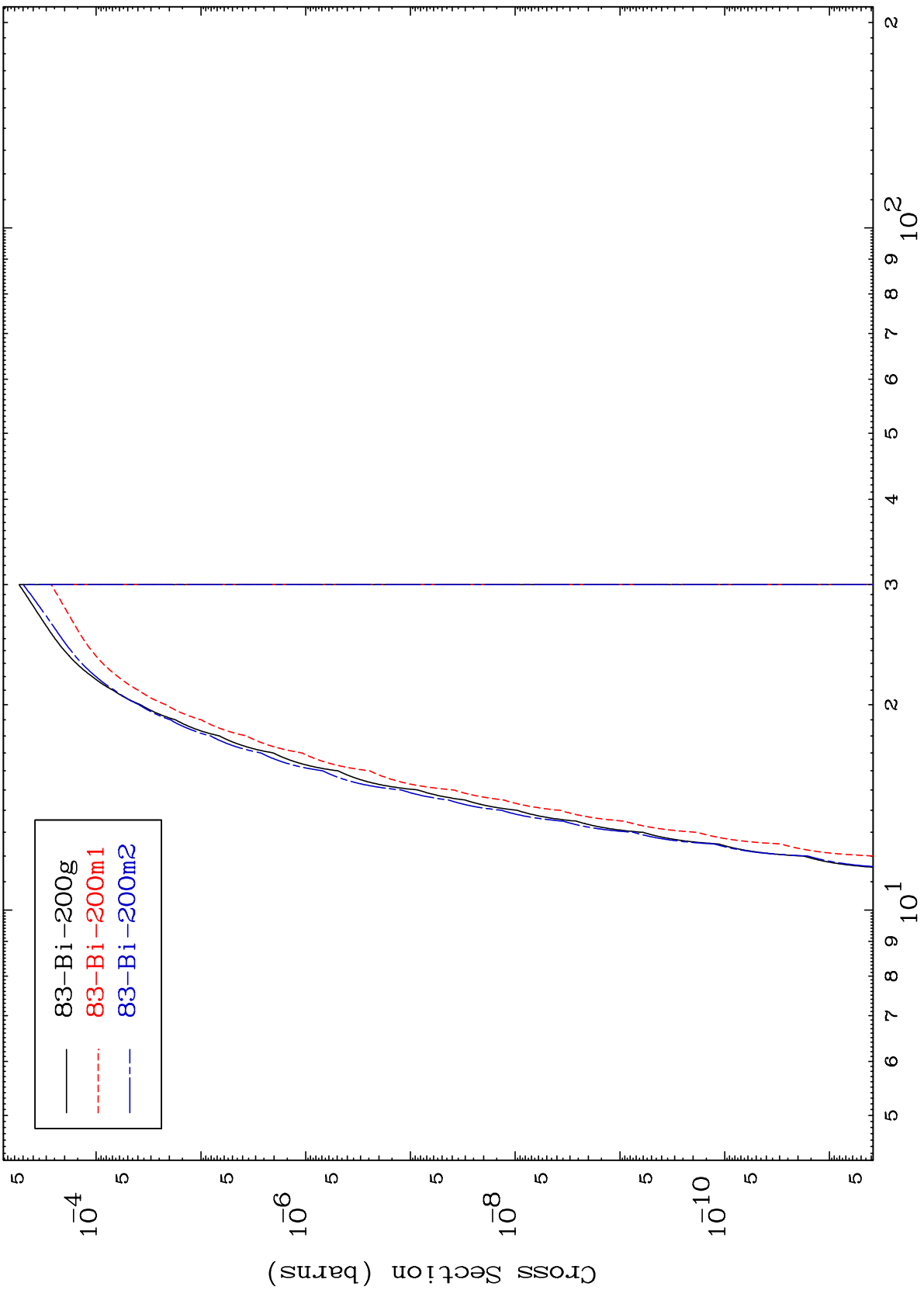


MAT 8523

(n,2n) p

85-At-202m

Radionuclide Production Cross Section



34

Incident Energy (MeV)

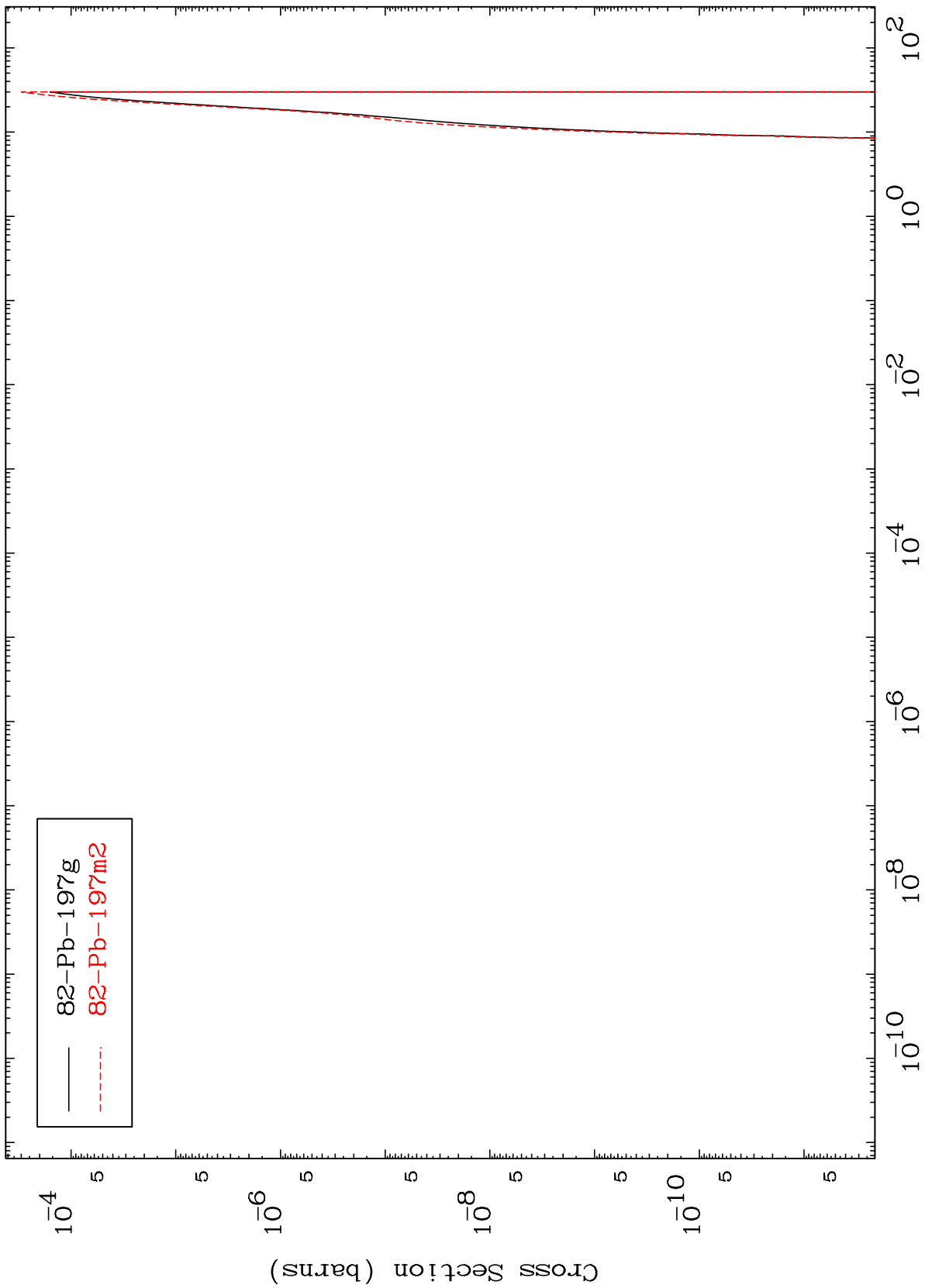
85-At-202m

MAT 8523

(n,n') p  $\alpha$

85-At-202m

Radionuclide Production Cross Section

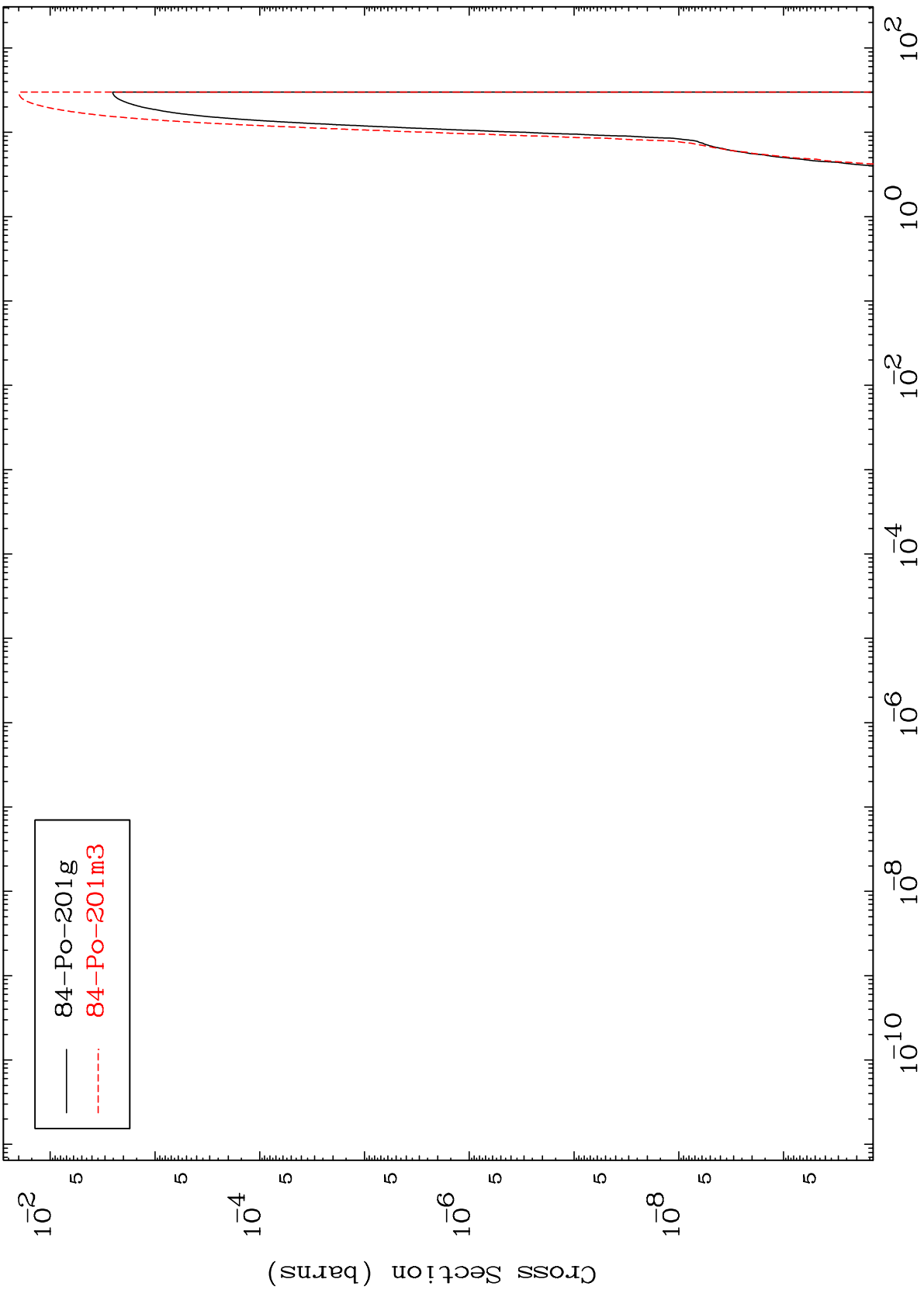


MAT 8523

(n,d)

85-At-202m

Radionuclide Production Cross Section

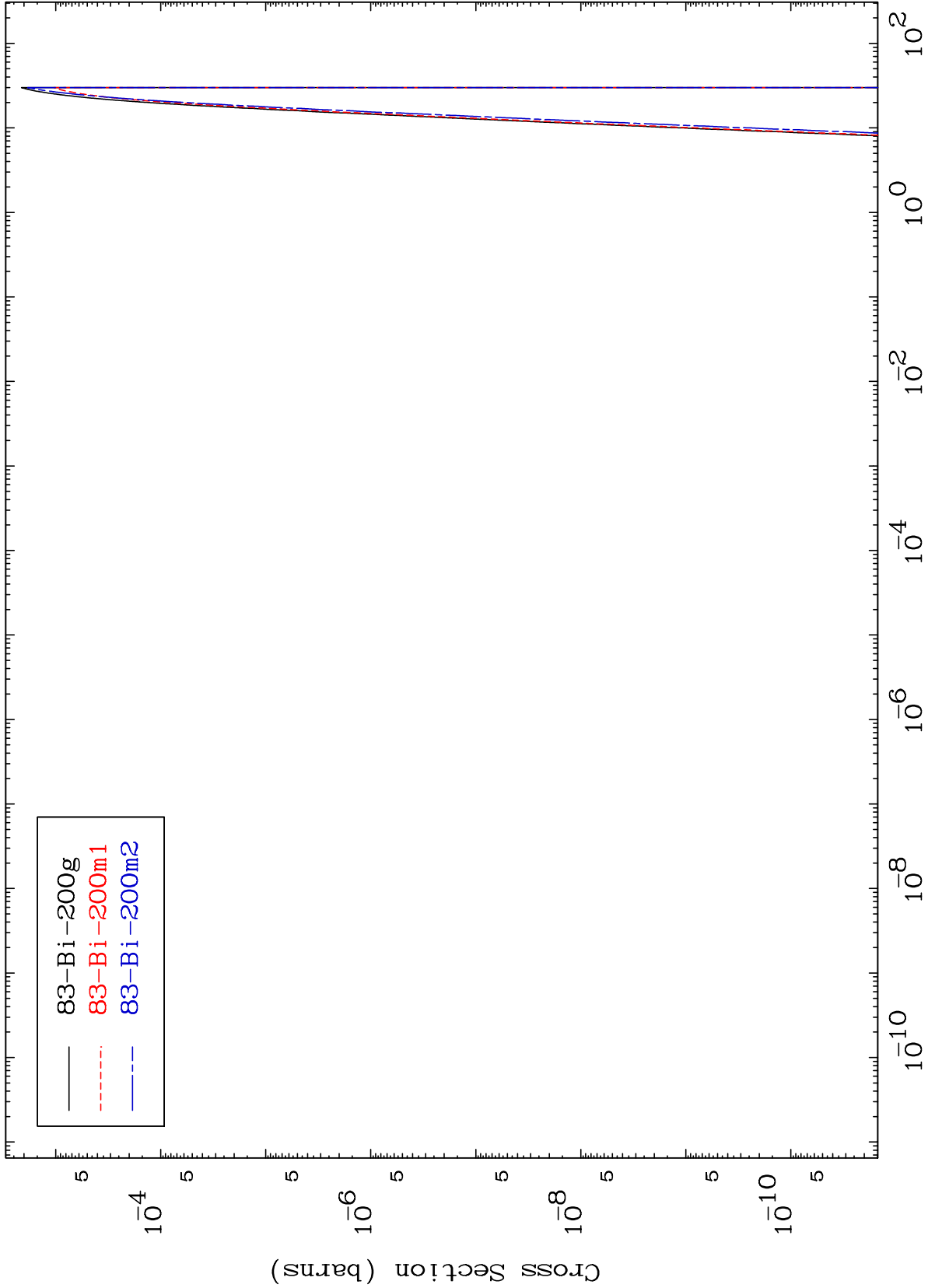


MAT 8523

(n,He-3)

85-At-202m

Radionuclide Production Cross Section

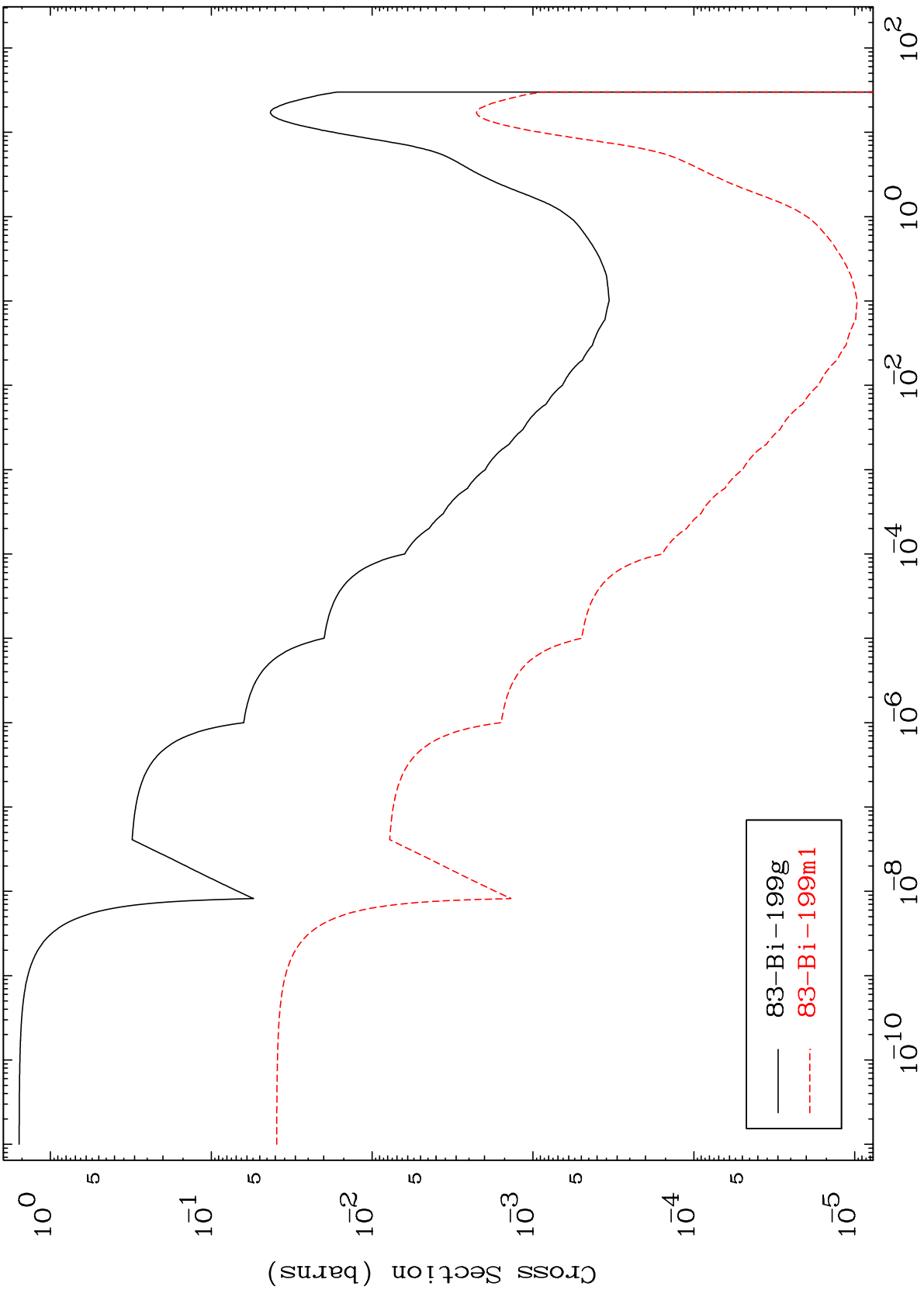


MAT 8523

85-At-202m

Radionuclide Production Cross Section

(n,  $\alpha$ )

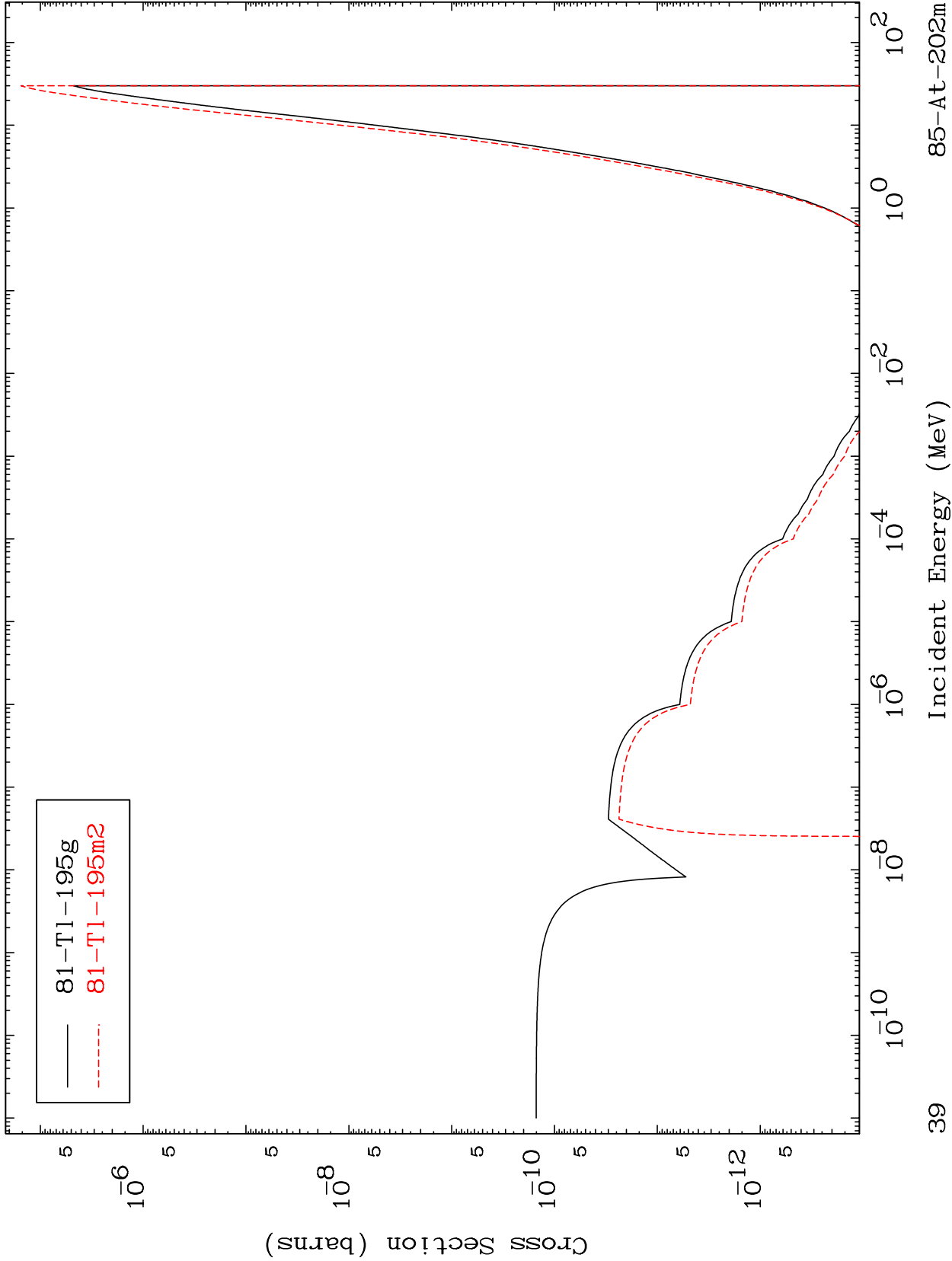


MAT 8523

(n,2α)

85-At-202m

Radionuclide Production Cross Section

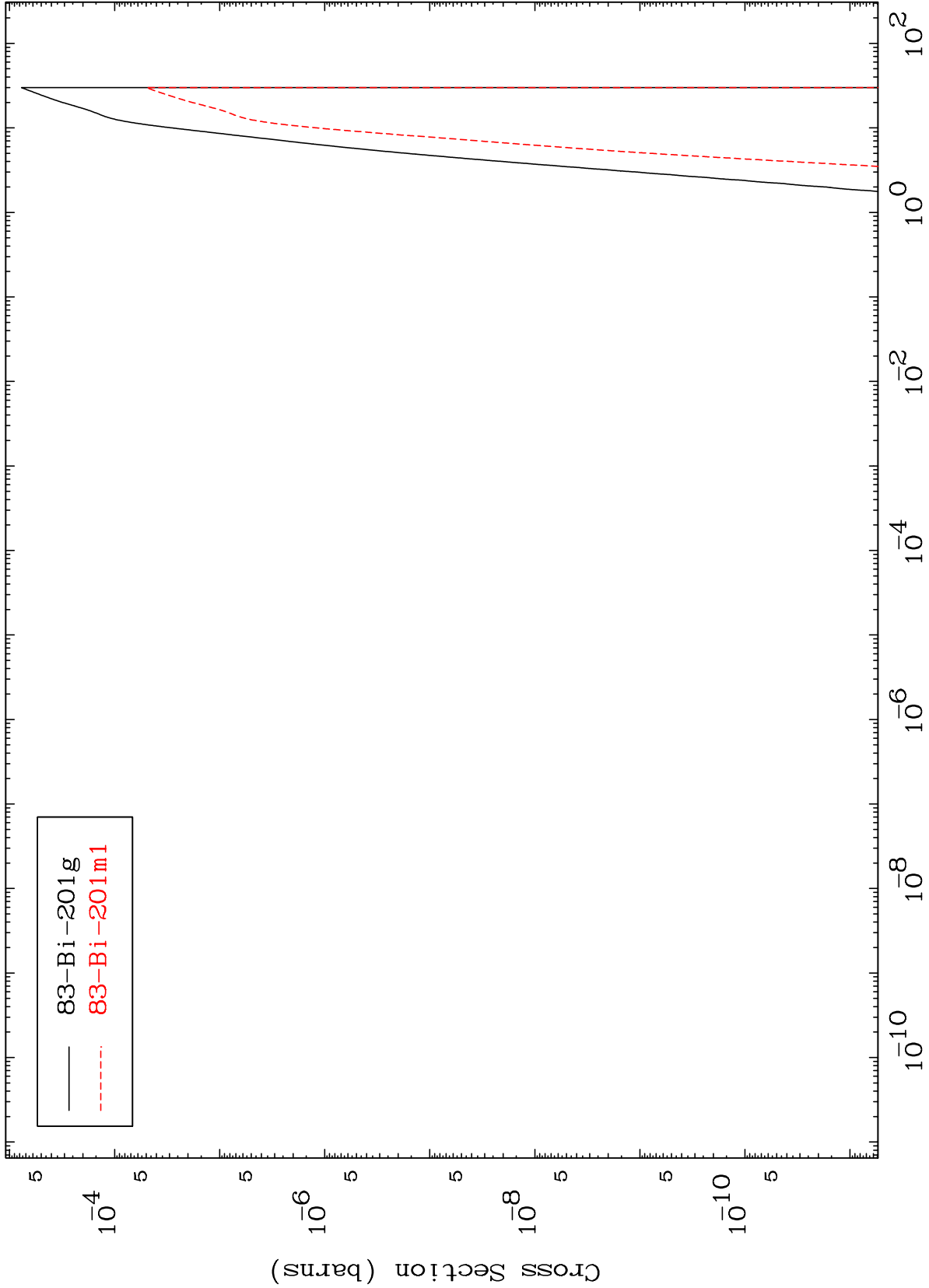


MAT 8523

(n,2p)

85-At-202m

Radionuclide Production Cross Section



40

Incident Energy (MeV)

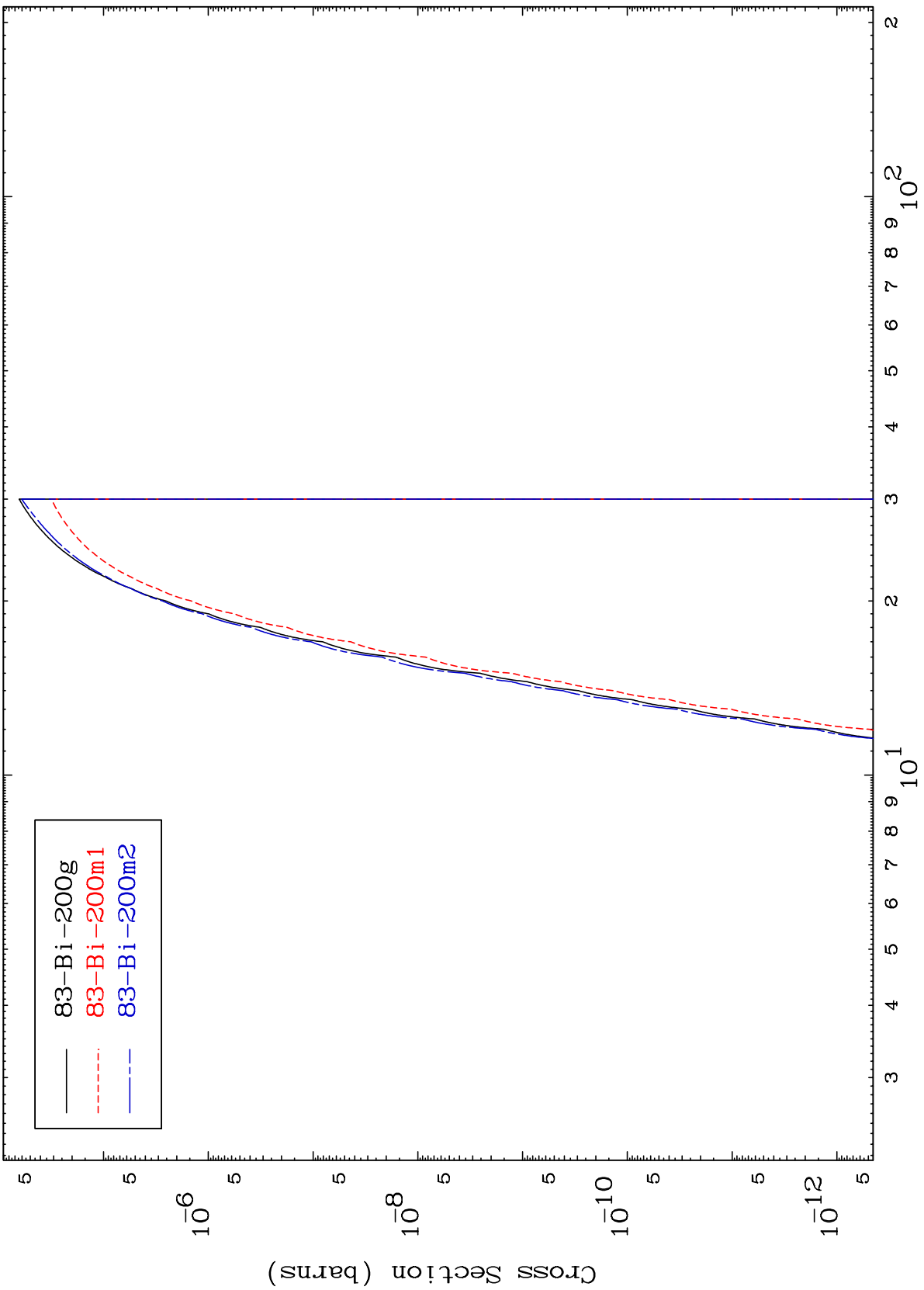
85-At-202m

MAT 8523

(n,p) d

85-At-202m

Radionuclide Production Cross Section

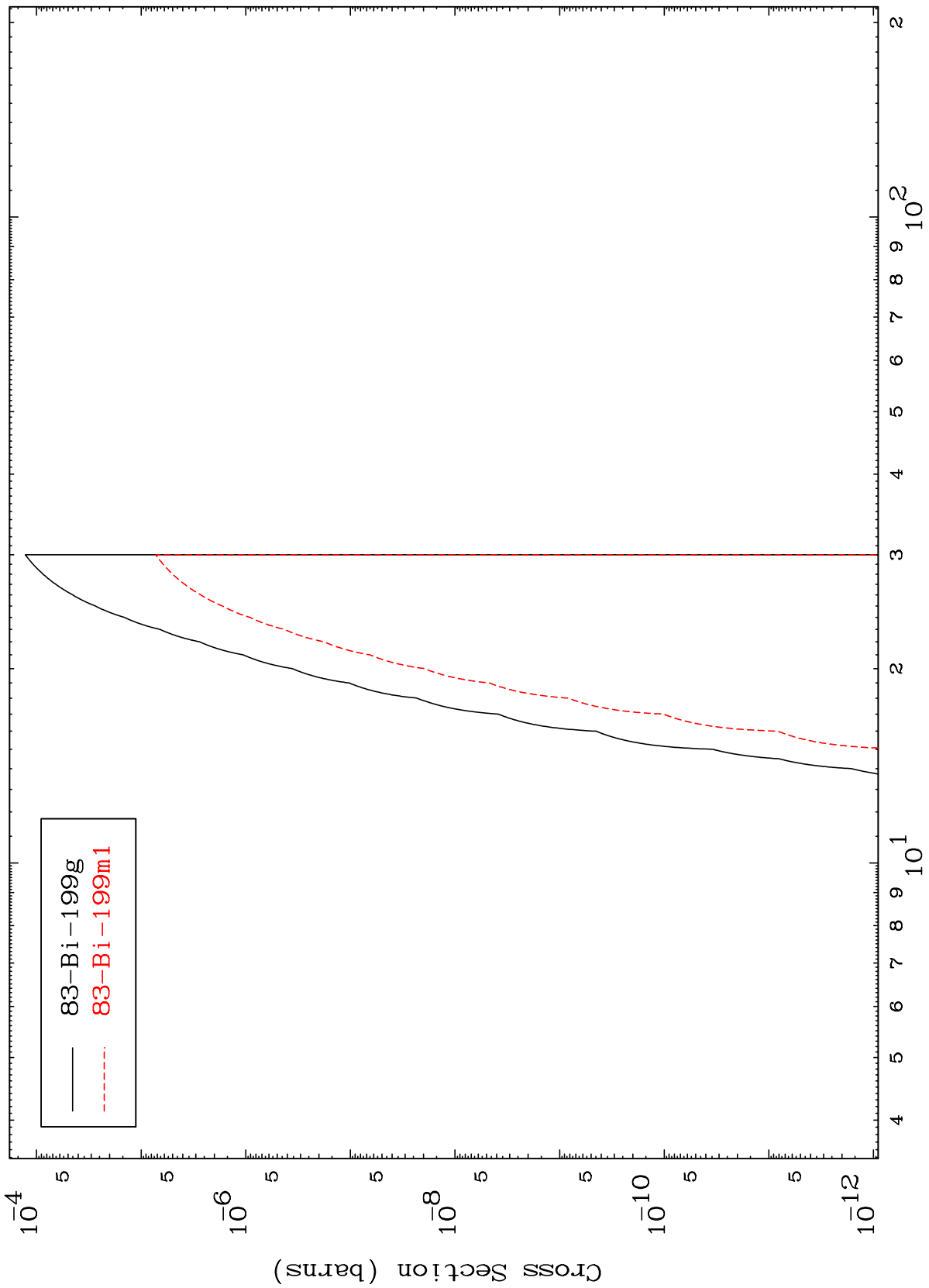


MAT 8523

(n,p) t

85-At-202m

Radionuclide Production Cross Section



83-Bi-199g  
83-Bi-199m1

Incident Energy (MeV)

85-At-202m

42

MAT 8523

(n,d)  $\alpha$

85-At-202m

Radionuclide Production Cross Section

