

Program EVALPLOT  
(Version 2021-1)

by

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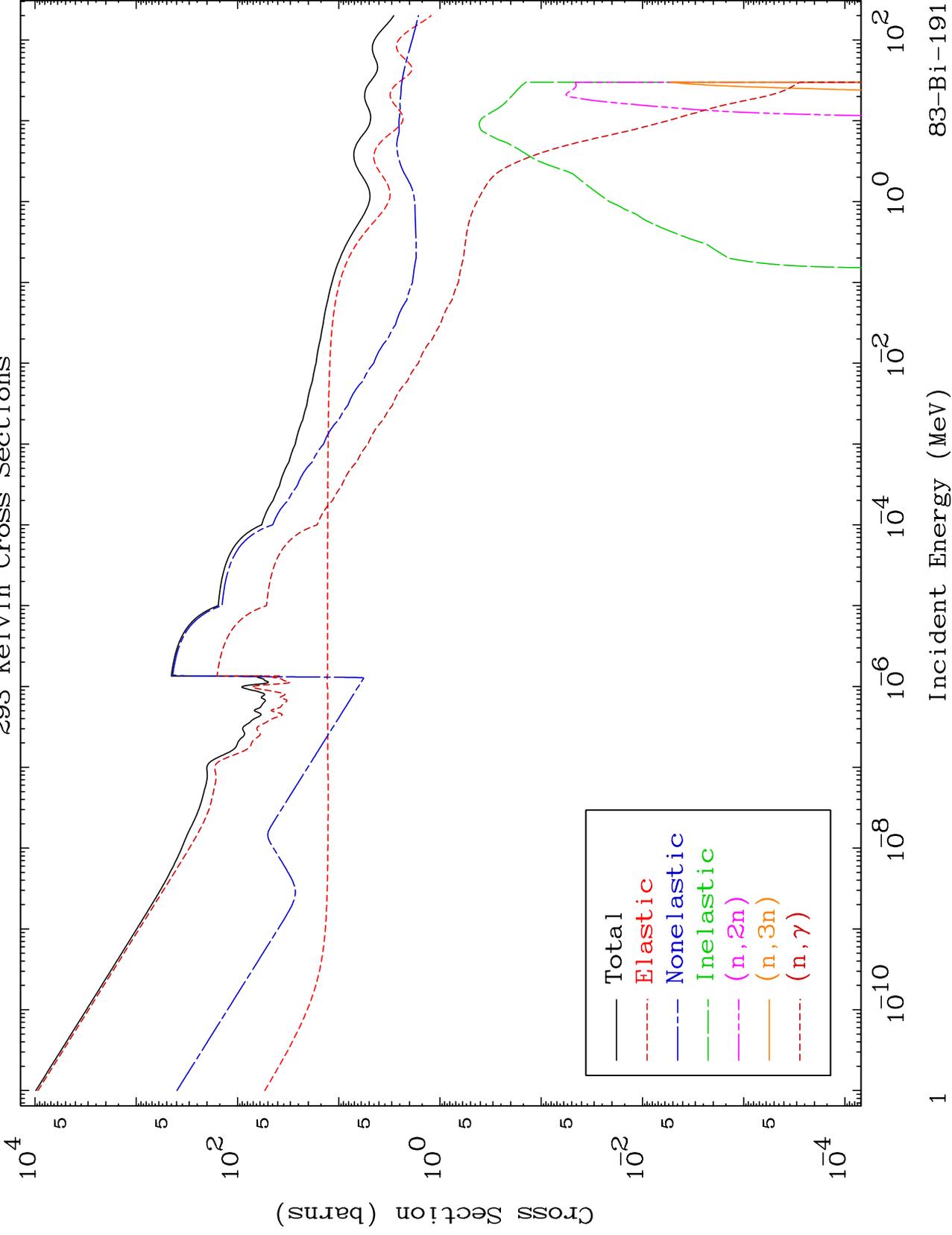
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Press Mouse Button to Start

MAT 8271

Neutron Major  
293 Kelvin Cross Sections

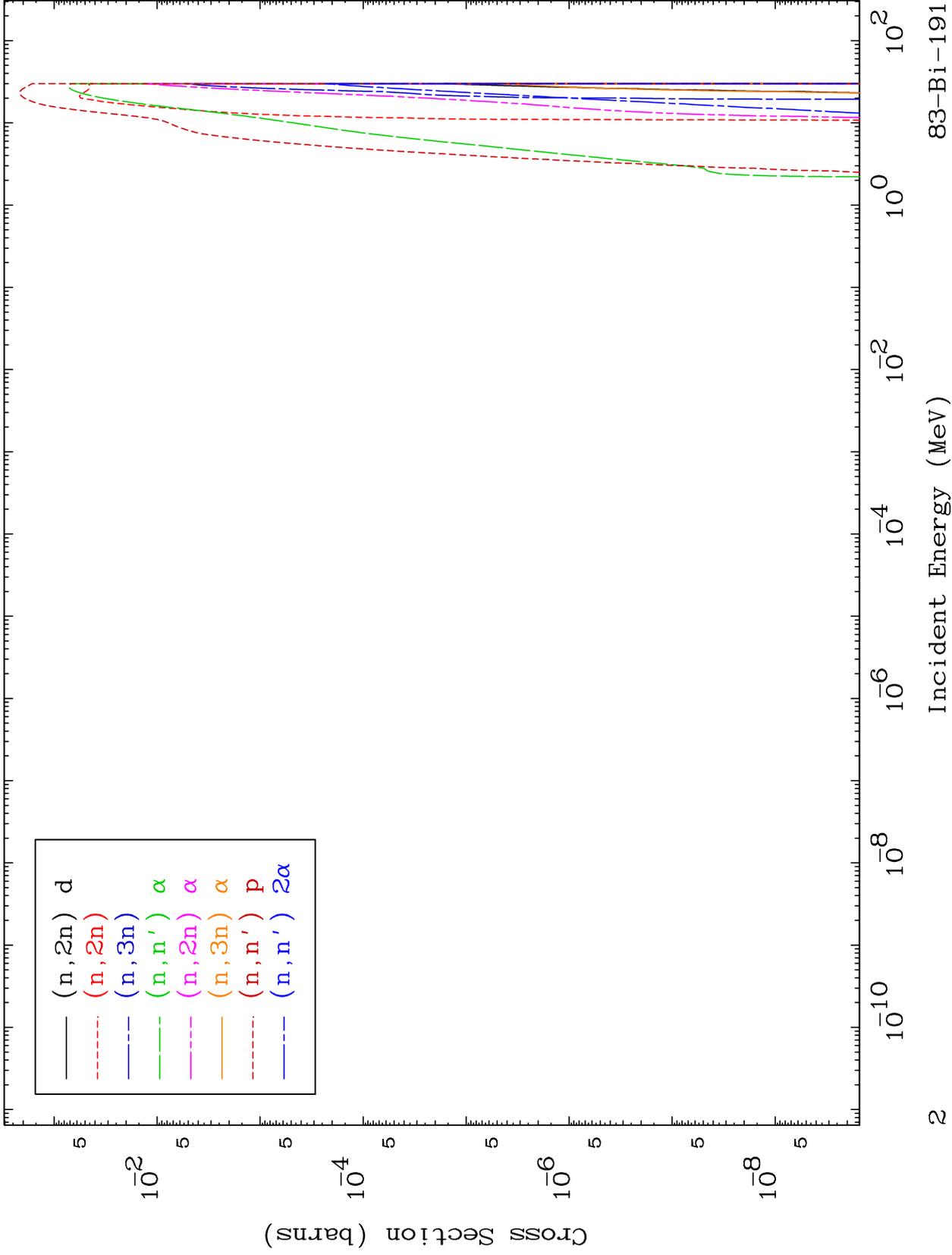
83-Bi-191



MAT 8271

Neutron Absorption  
293 Kelvin Cross Sections

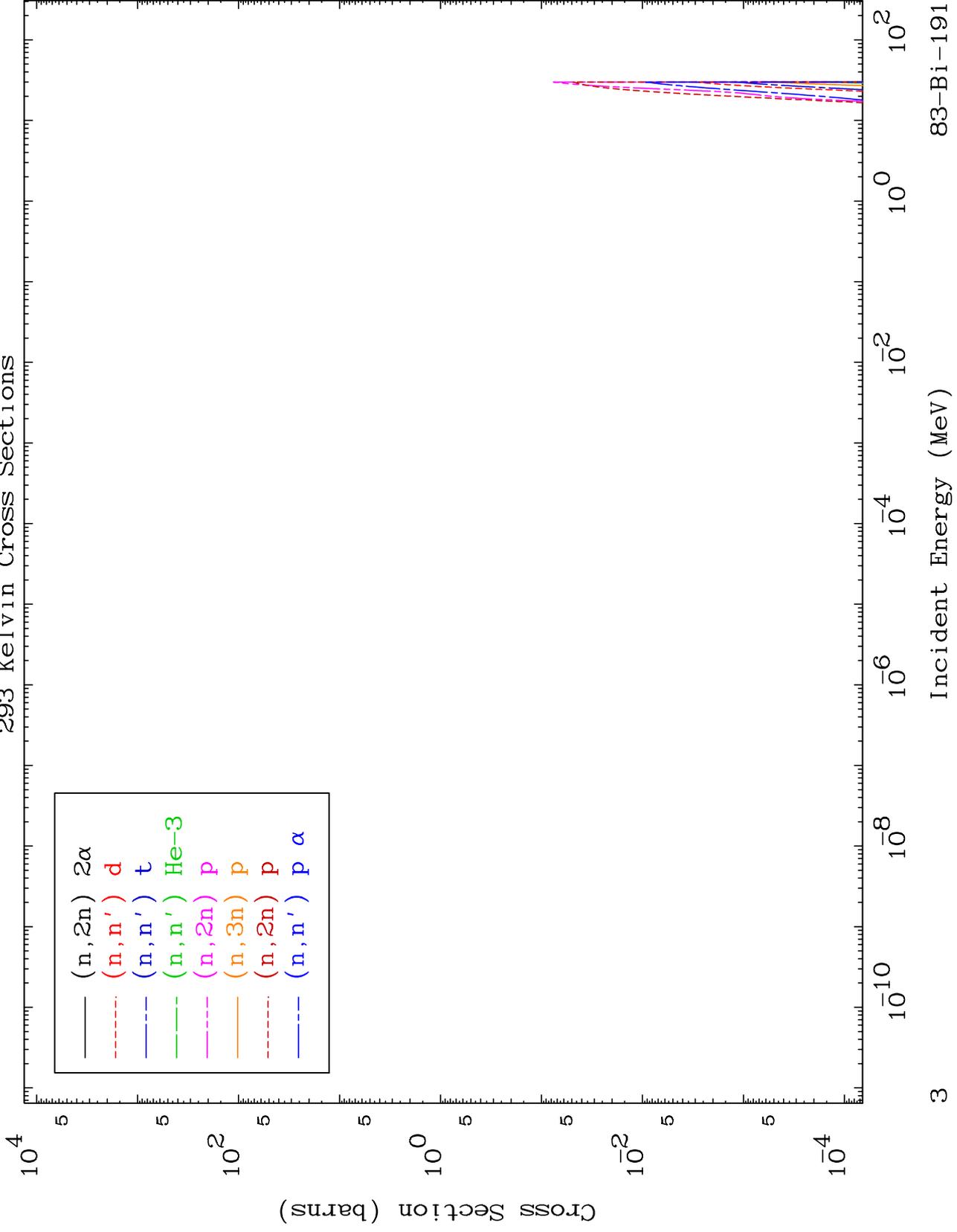
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MAT 8271

Neutron Absorption  
293 Kelvin Cross Sections

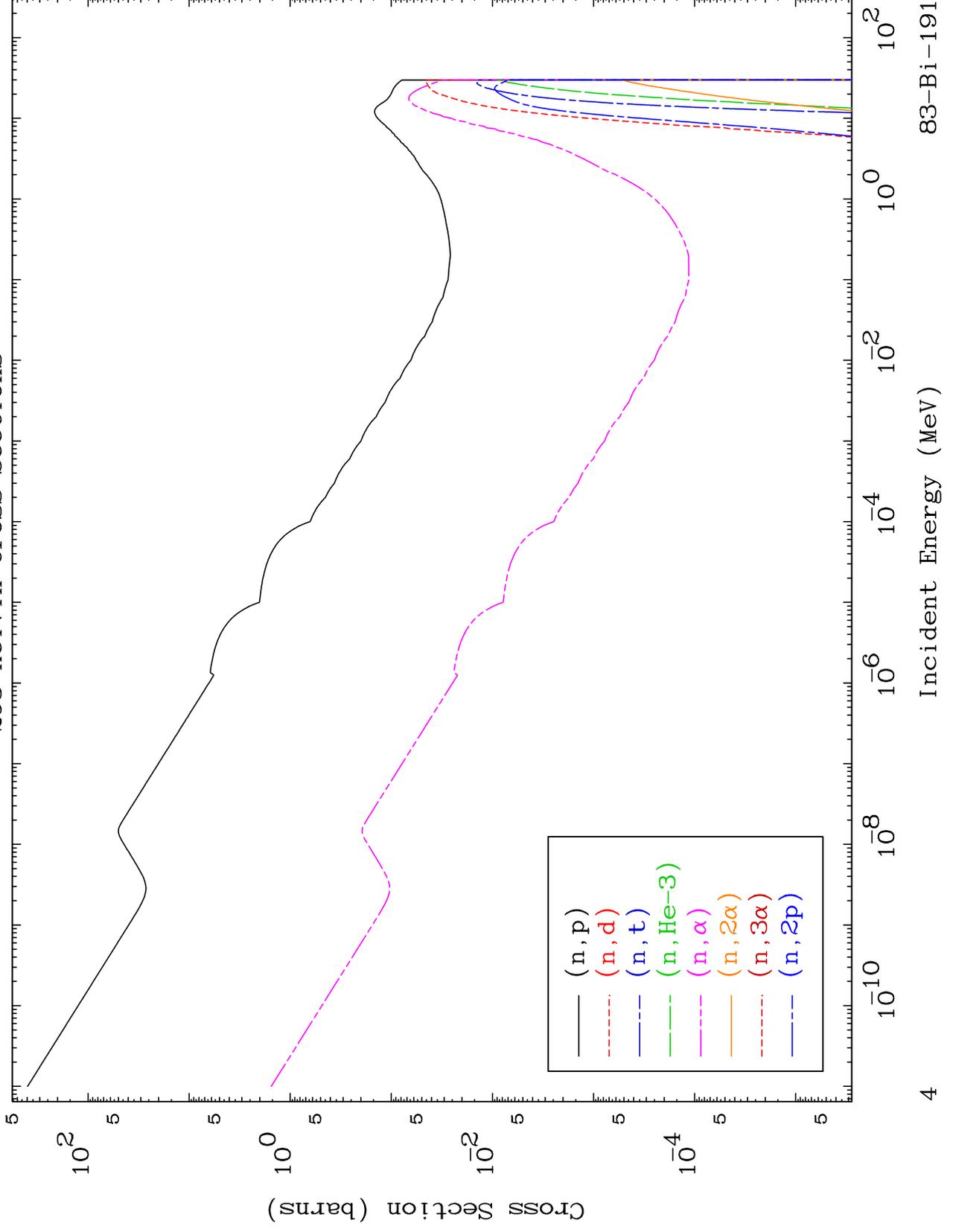
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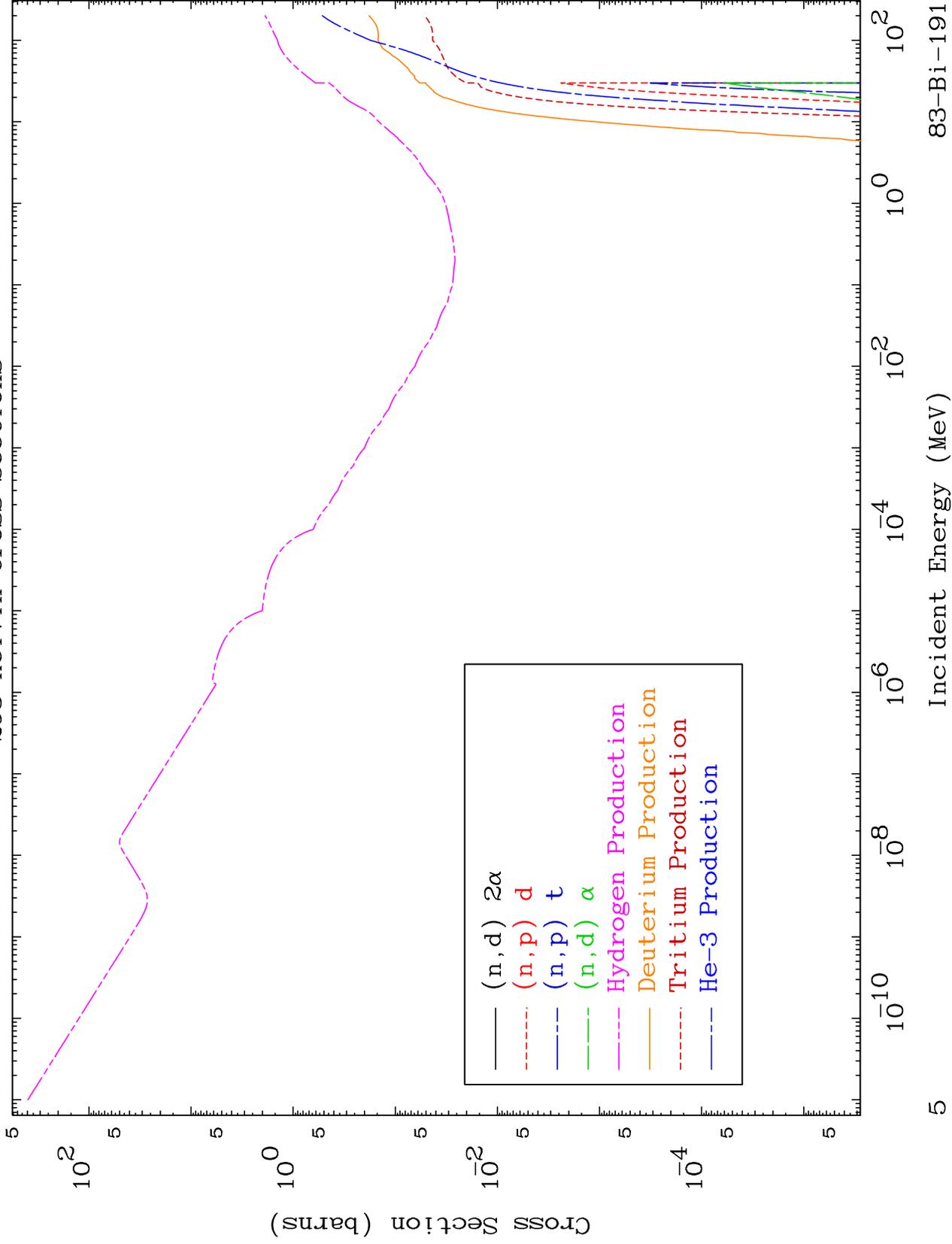


MAT 8271

Neutron Absorption  
293 Kelvin Cross Sections

83-Bi-191

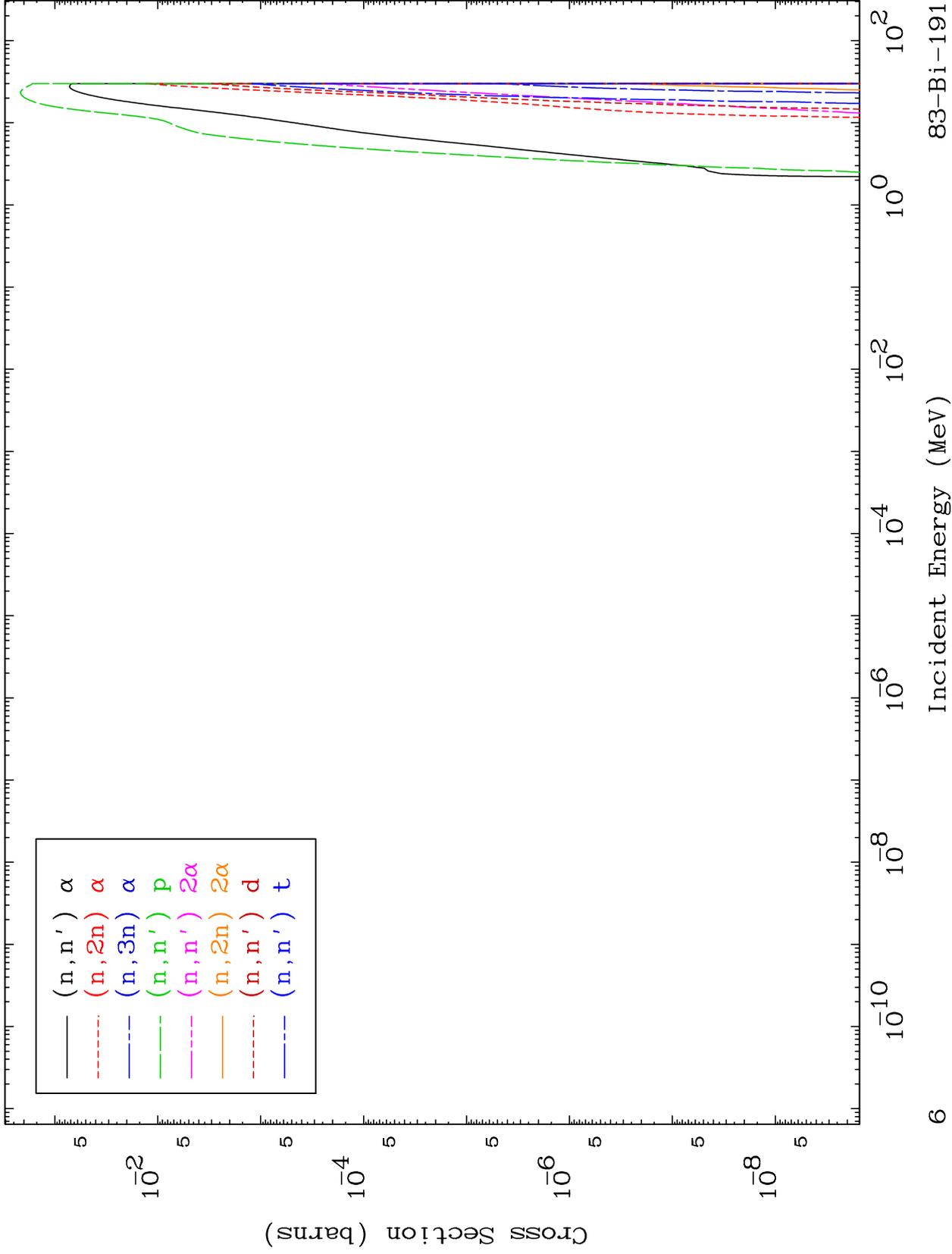




MAT 8271

Charged Particle  
293 Kelvin Cross Sections

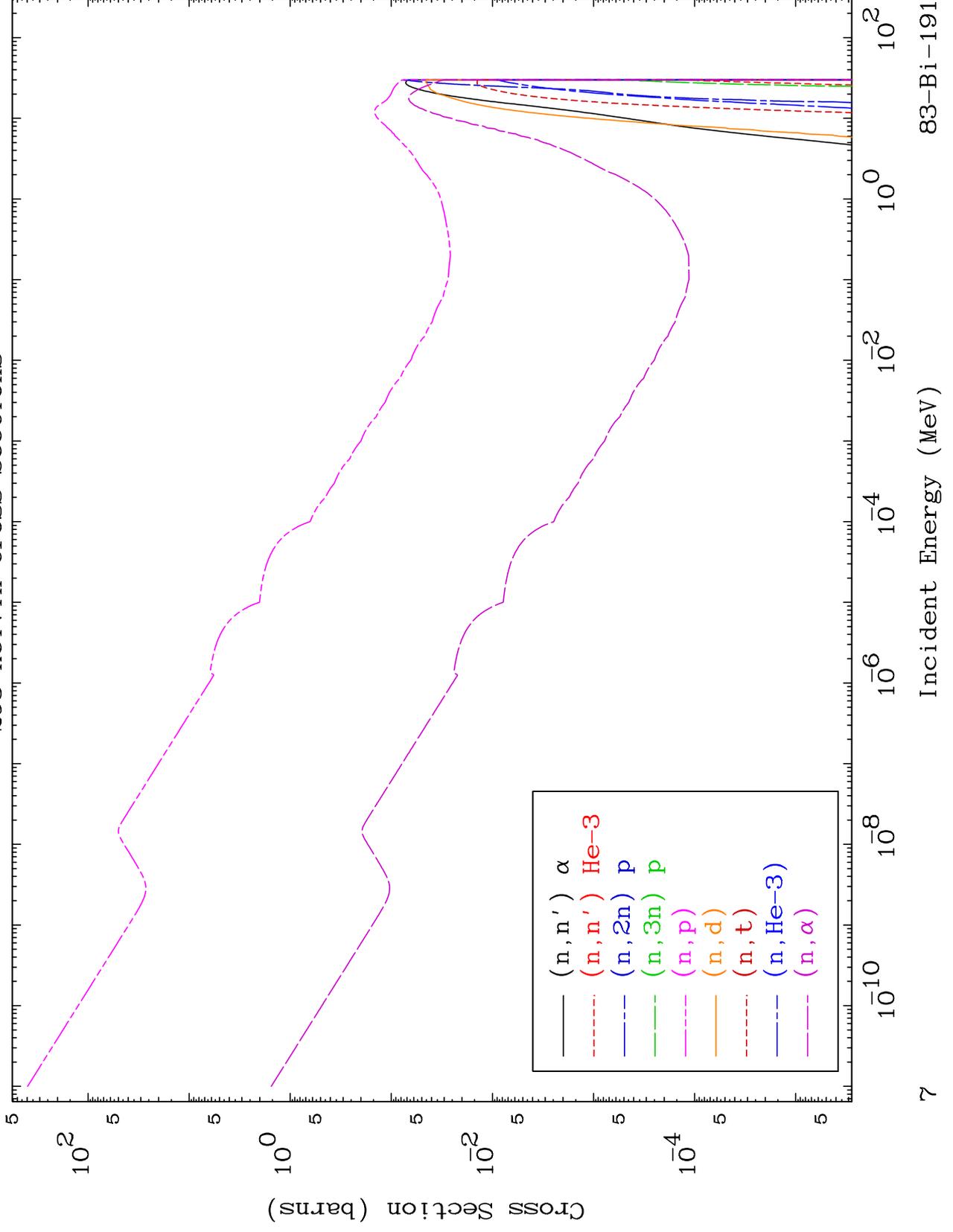
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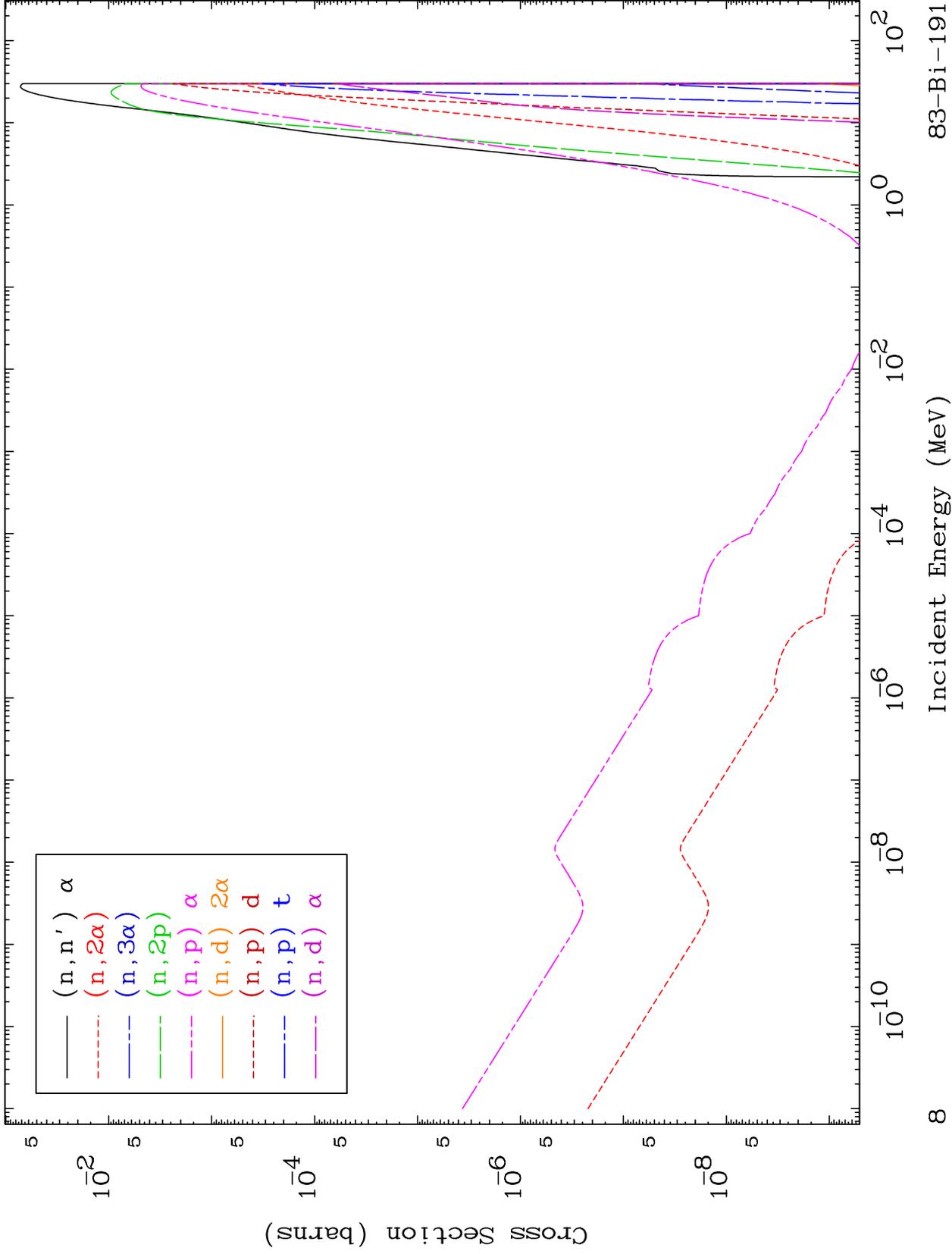


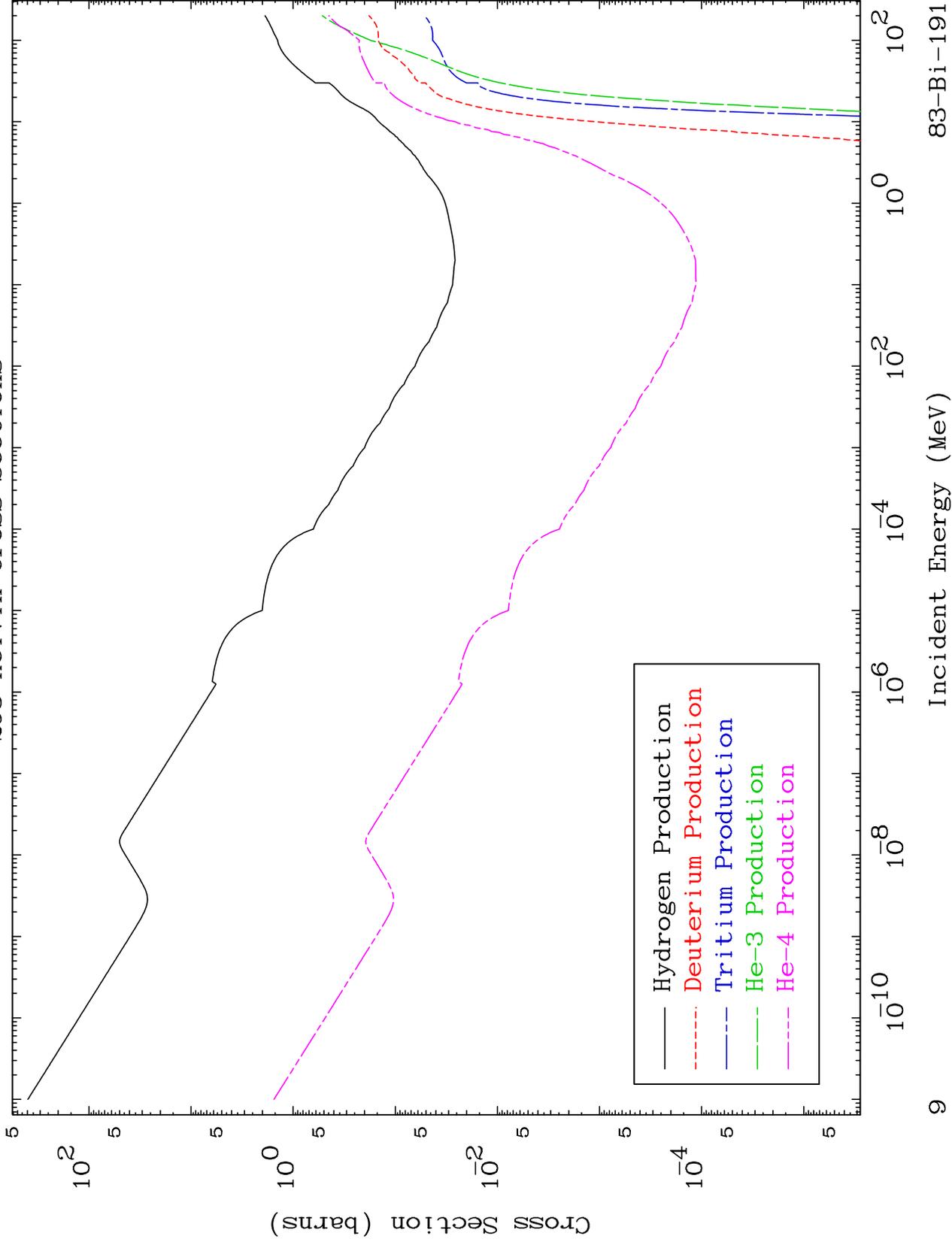
MAT 8271

Charged Particle  
293 Kelvin Cross Sections

83-Bi-191



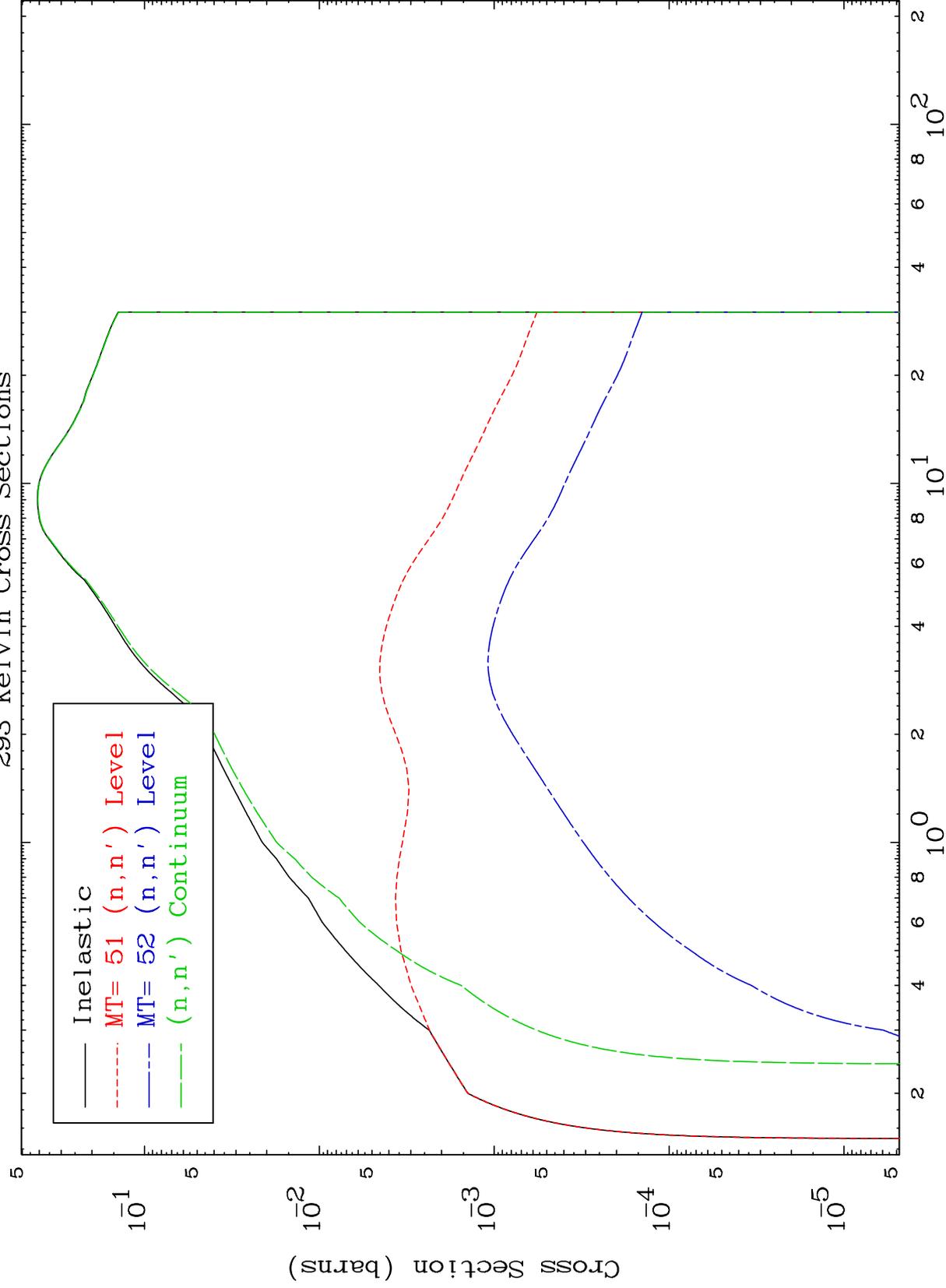




MAT 8271

83-Bi-191

(n,n') Levels  
293 Kelvin Cross Sections



10

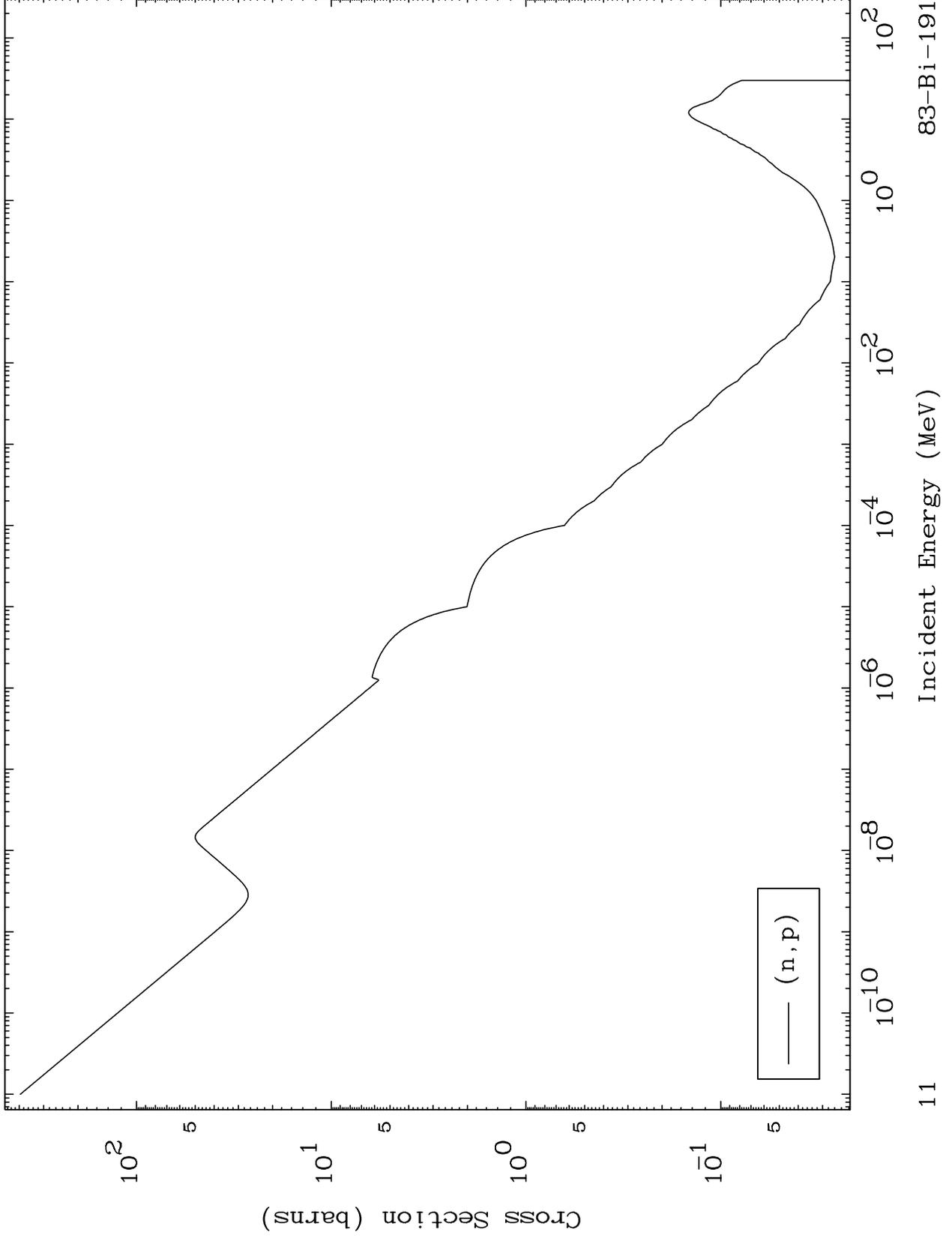
Incident Energy (MeV)

83-Bi-191

MAT 8271

(n,p) Levels  
293 Kelvin Cross Sections

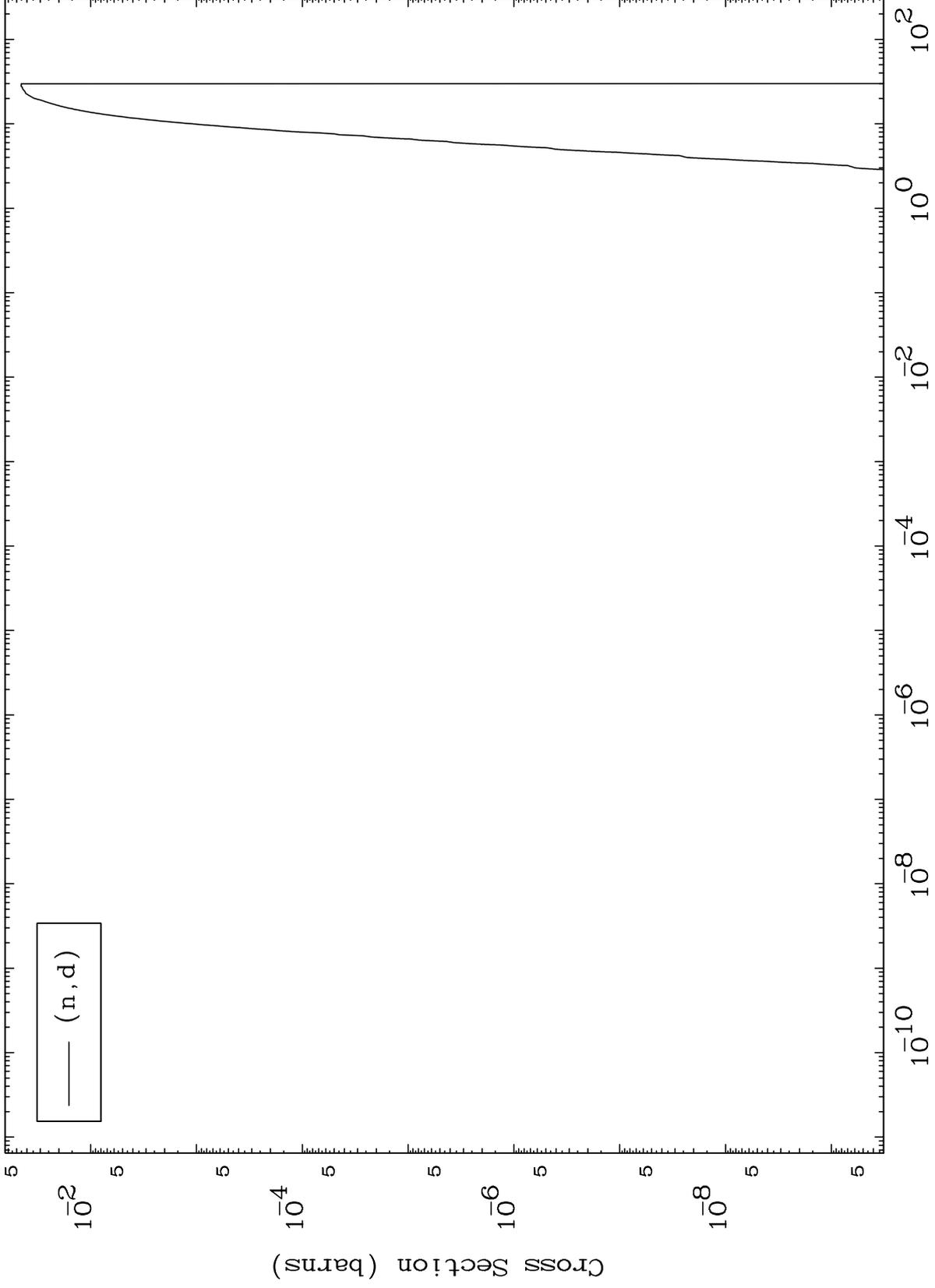
83-Bi-191



MAT 8271

(n,d) Levels  
293 Kelvin Cross Sections

83-Bi-191



12

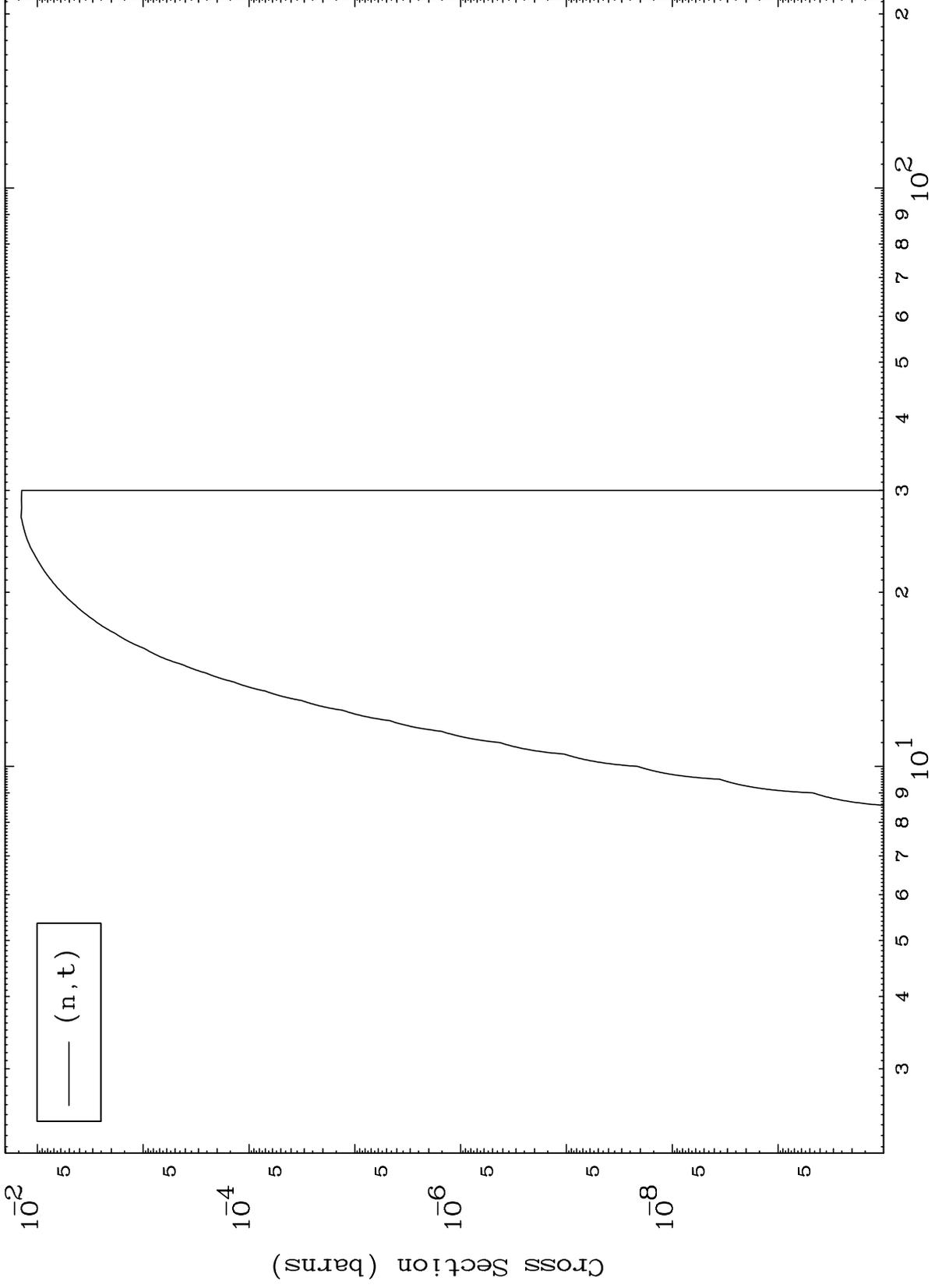
Incident Energy (MeV)

83-Bi-191

MAT 8271

(n,t) Levels  
293 Kelvin Cross Sections

83-Bi-191



13

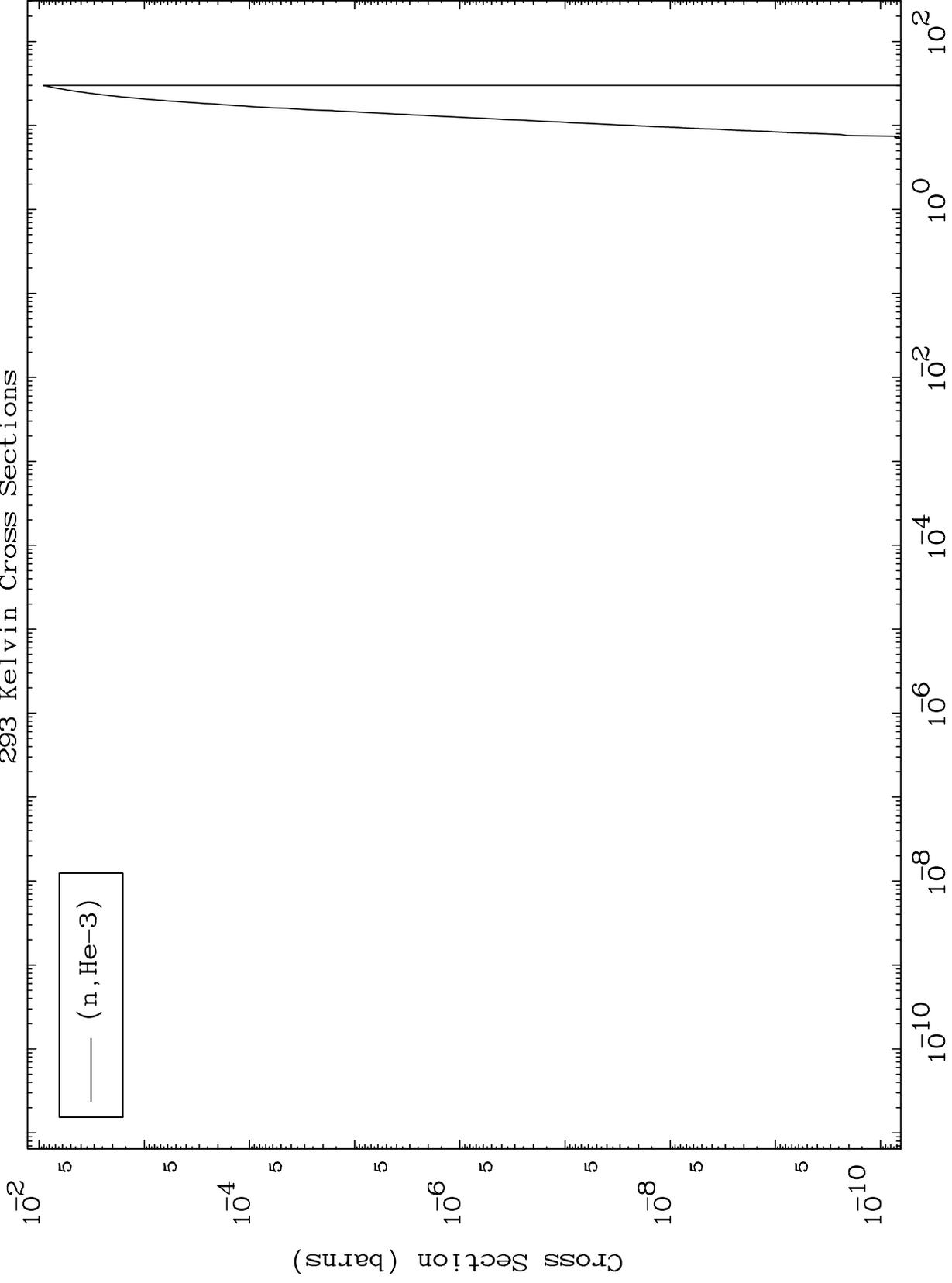
Incident Energy (MeV)

83-Bi-191

MAT 8271

(n,He3) Levels  
293 Kelvin Cross Sections

83-Bi-191



(n, He-3)

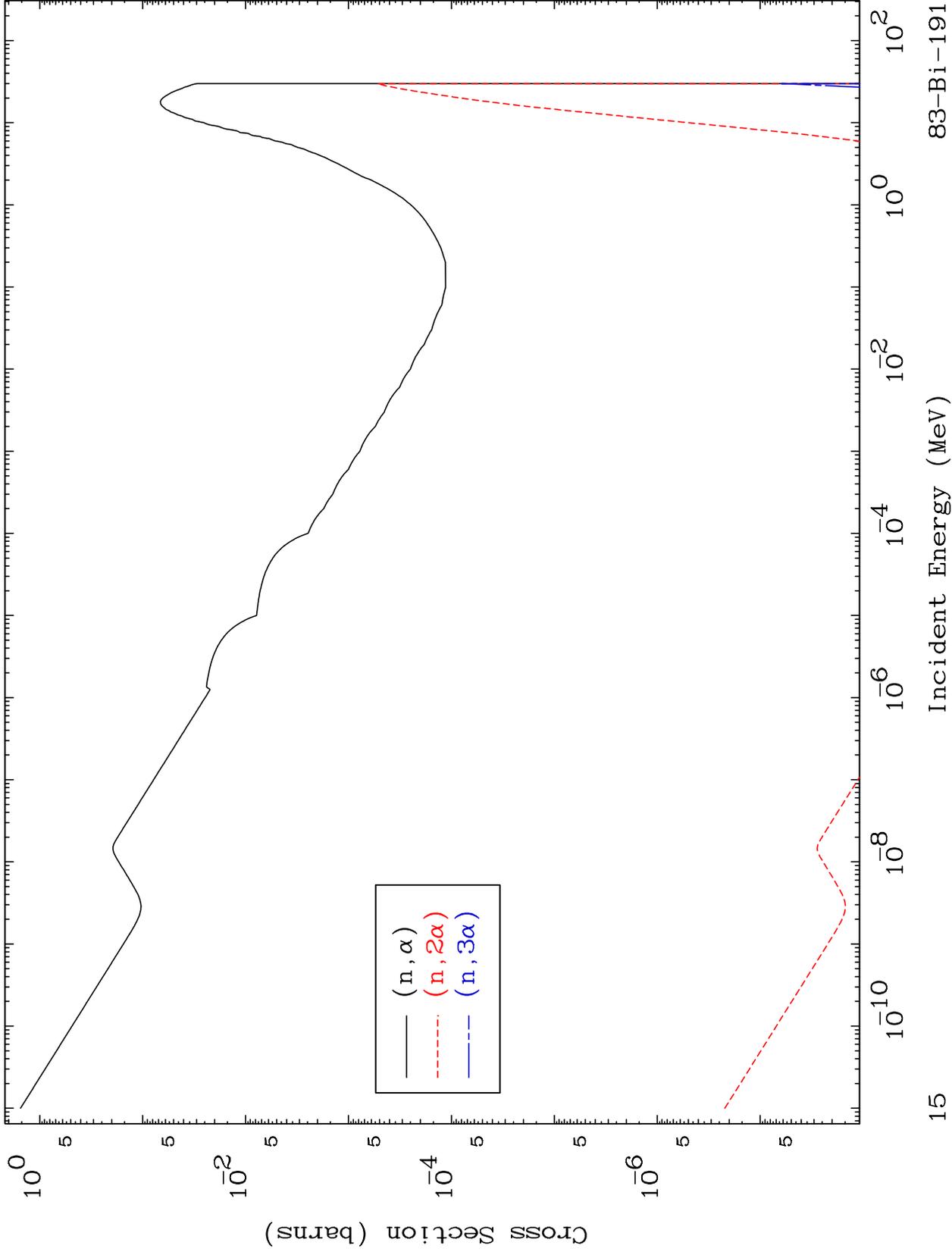
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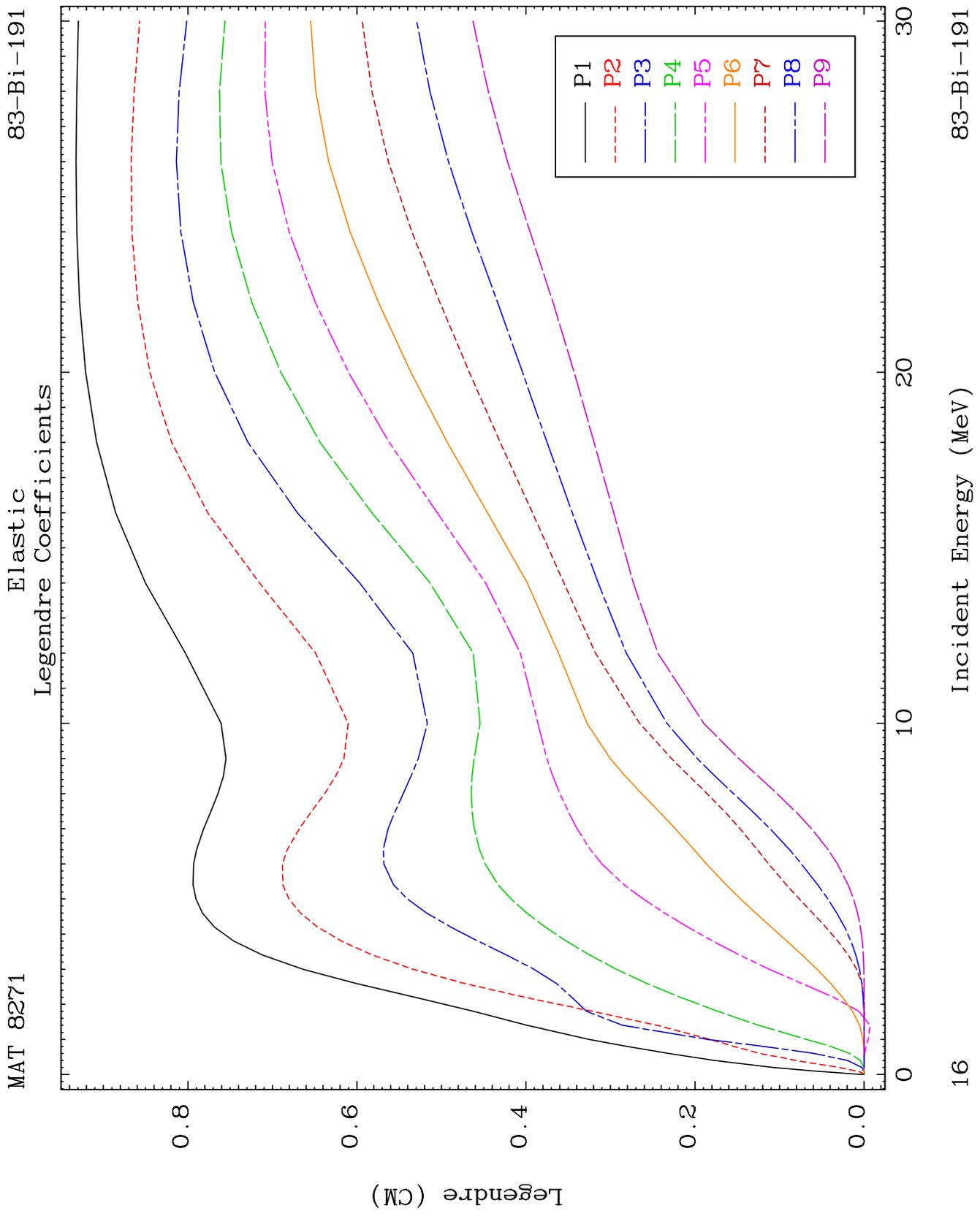
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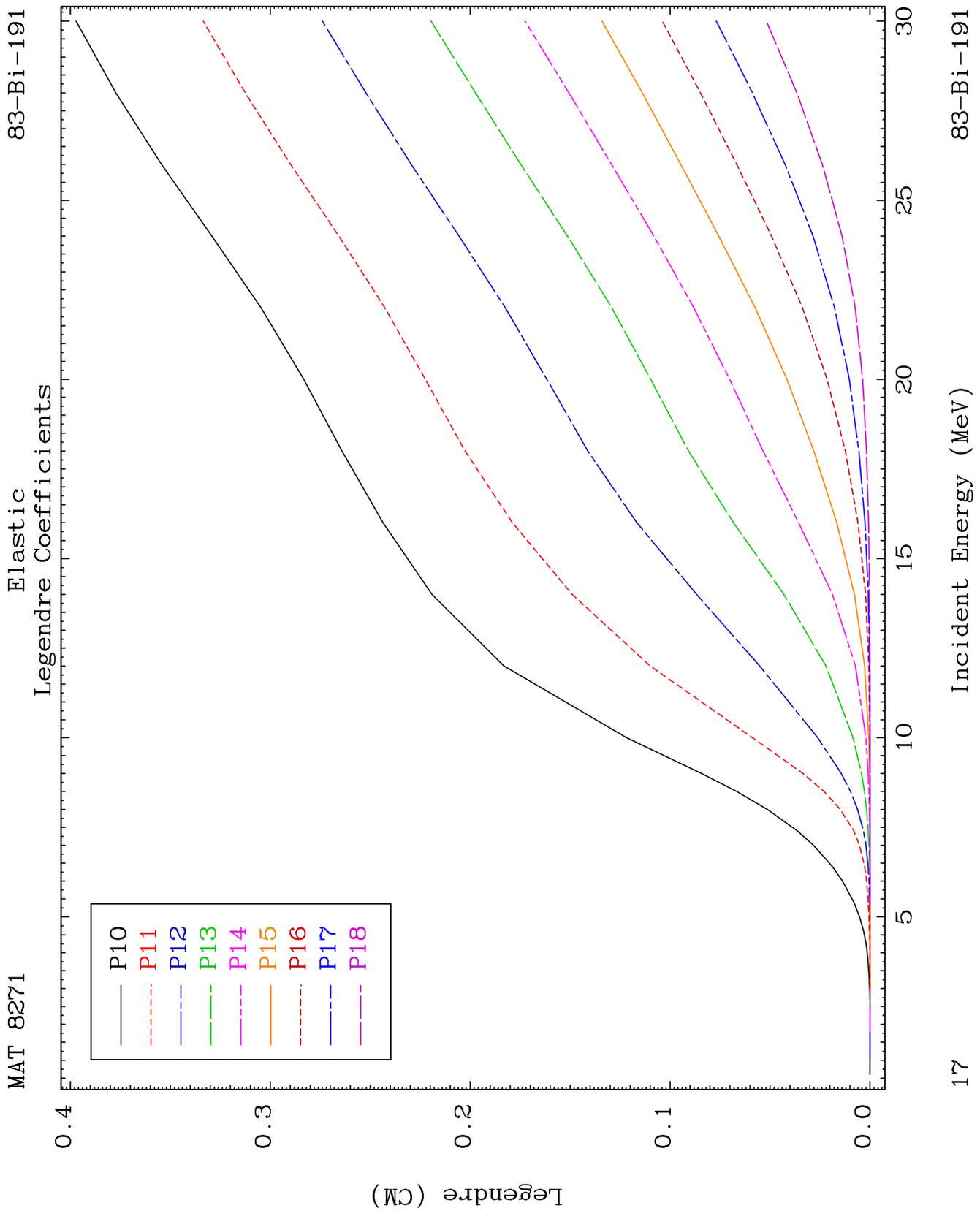
MAT 8271

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

83-Bi-191



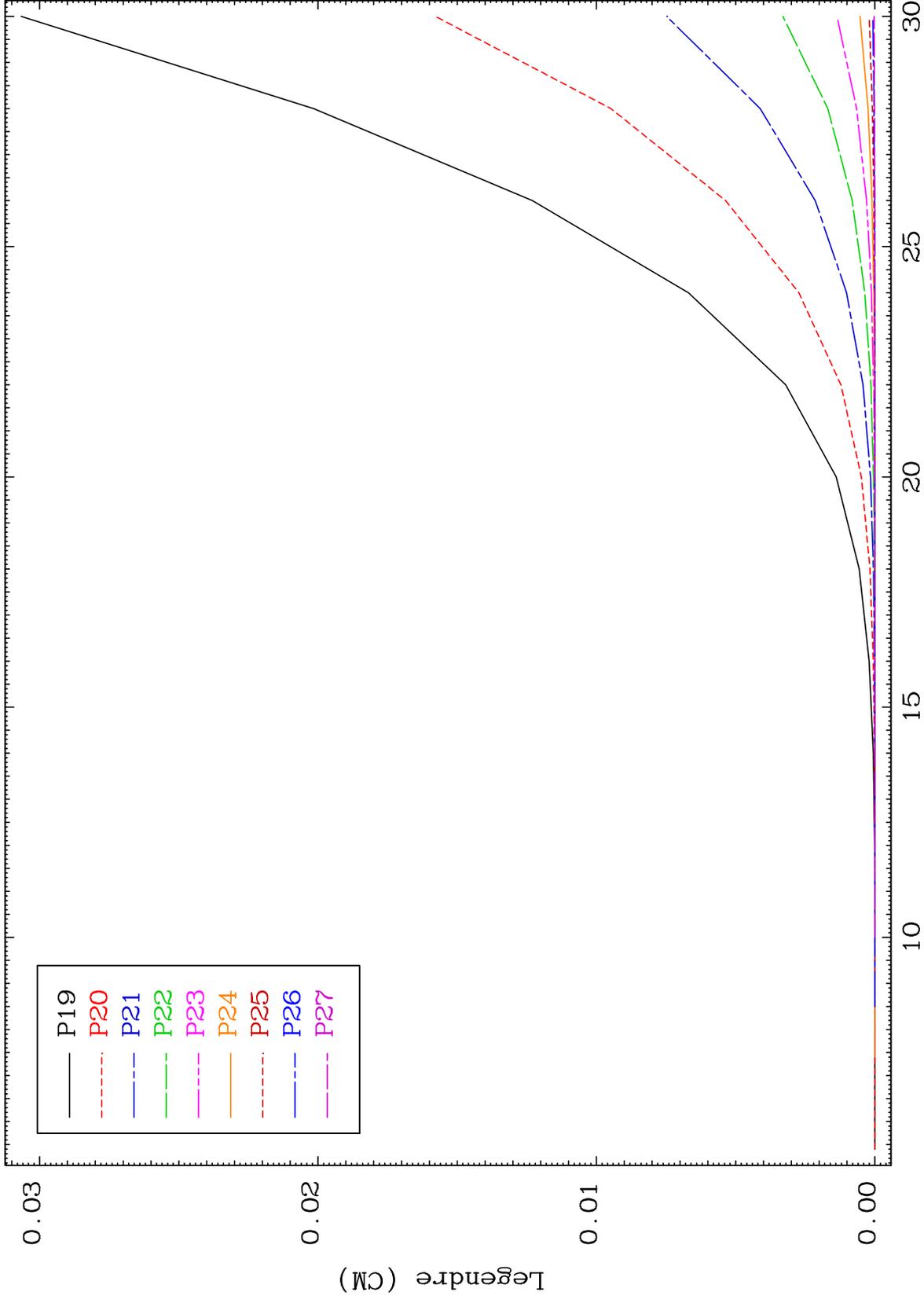




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Elastic  
Legendre Coefficients

83-Bi-191



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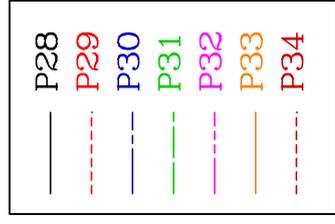
Incident Energy (MeV)

83-Bi-191

MAT 8271

Elastic  
Legendre Coefficients

83-Bi-191



$\times 10^{-6}$

Legendre (CM)

8

6

4

2

0

15

20

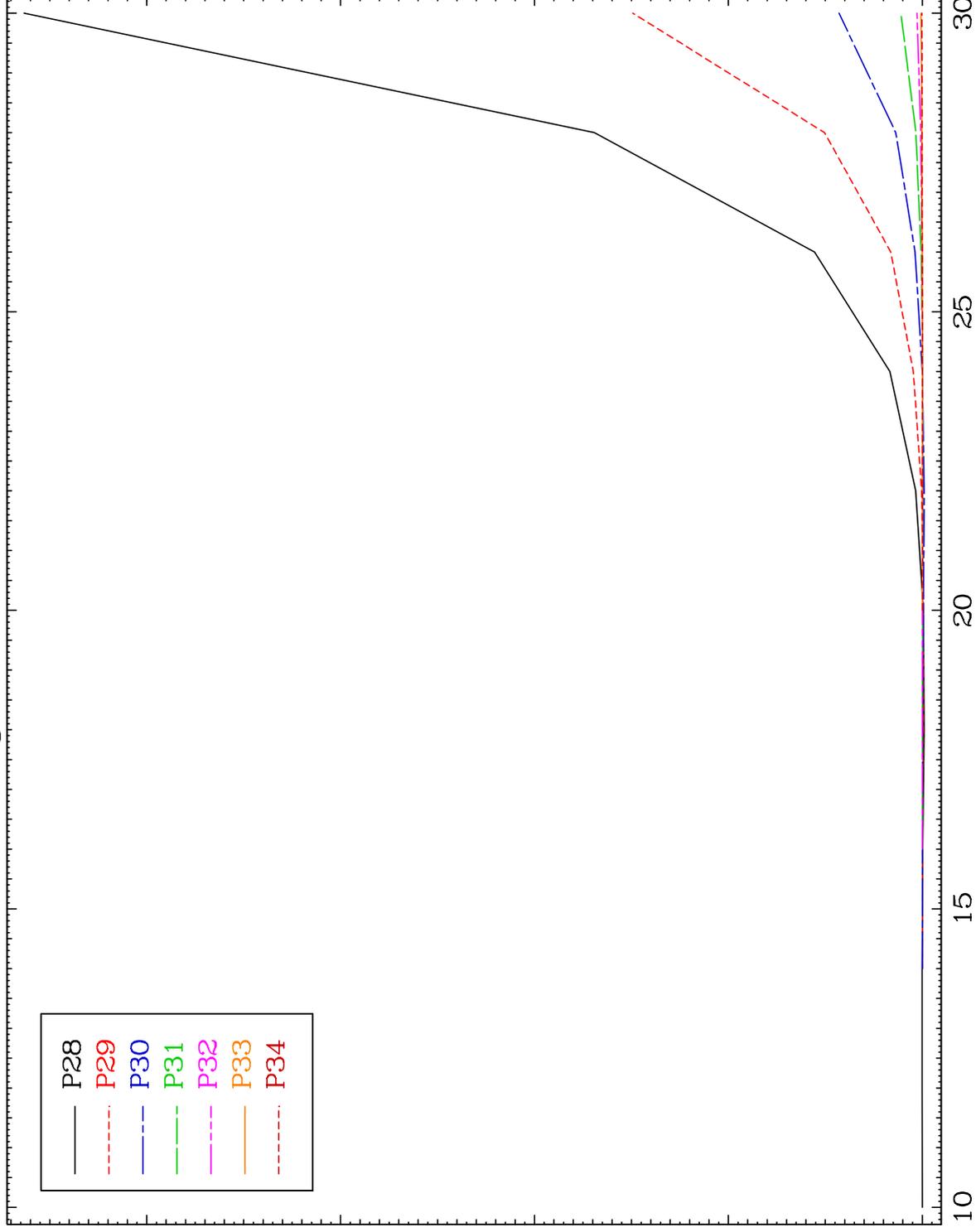
25

30

19

Incident Energy (MeV)

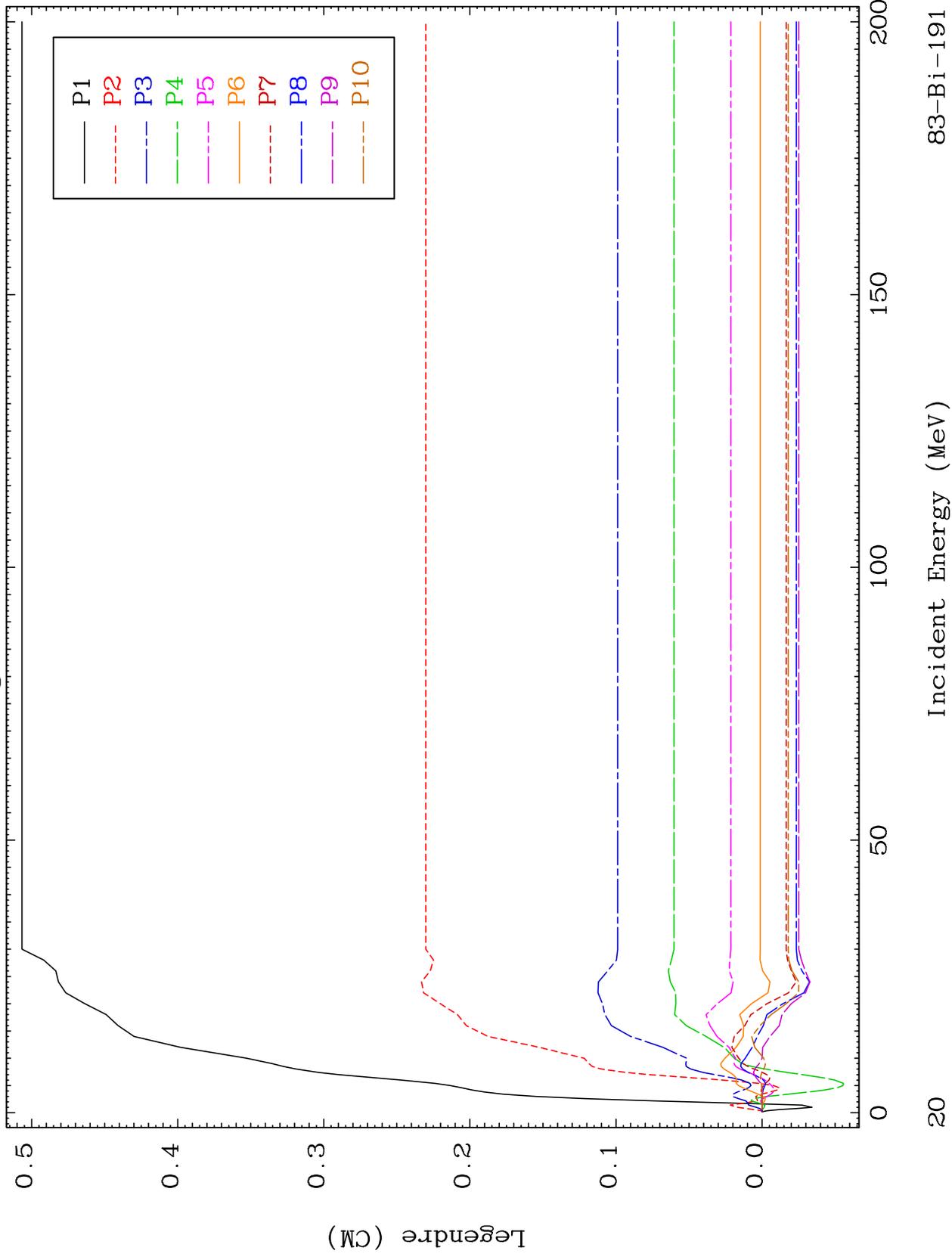
83-Bi-191



MAT 8271

MT= 51 (n,n') Level  
Legendre Coefficients

83-Bi-191



83-Bi-191

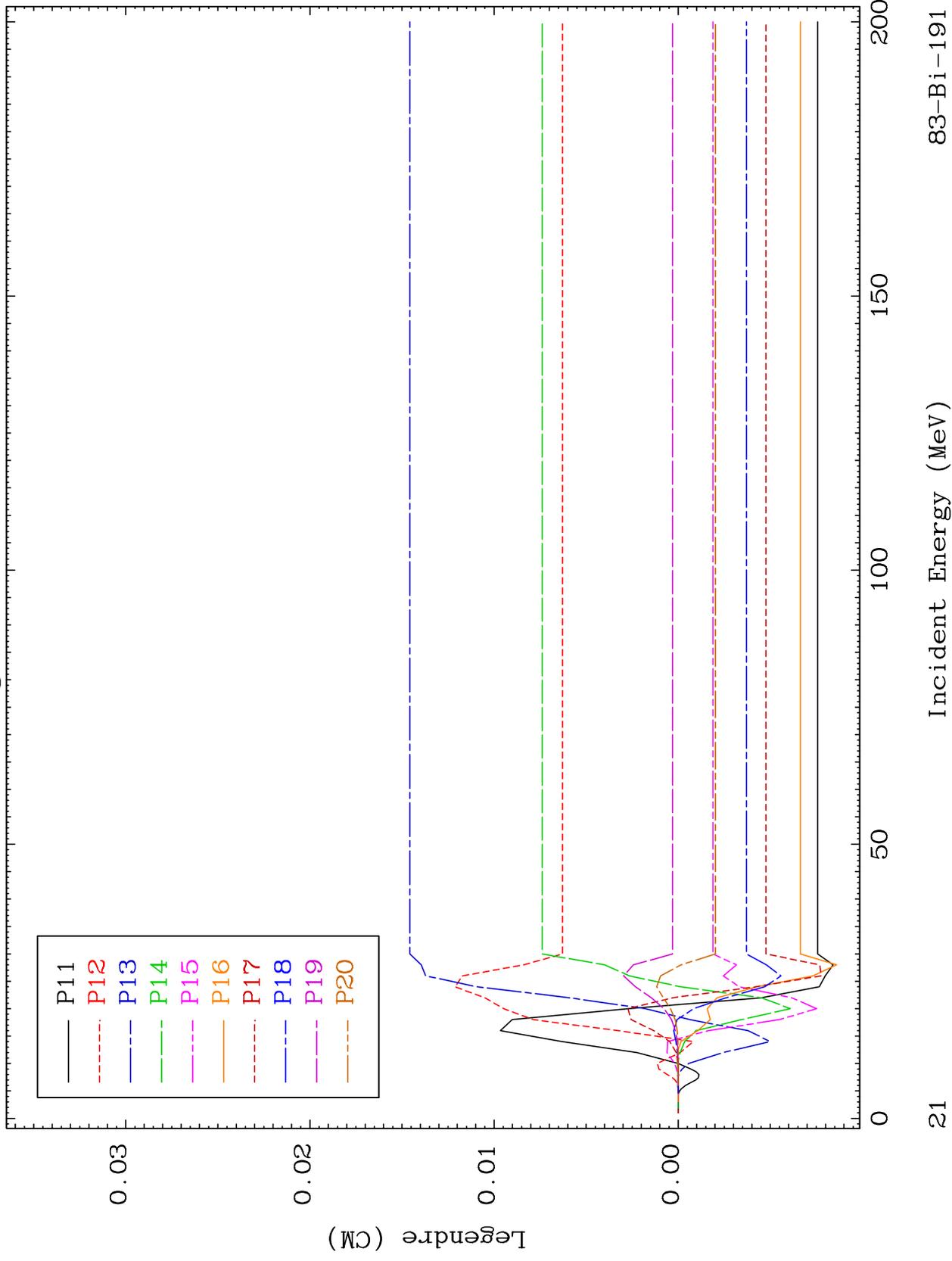
Incident Energy (MeV)

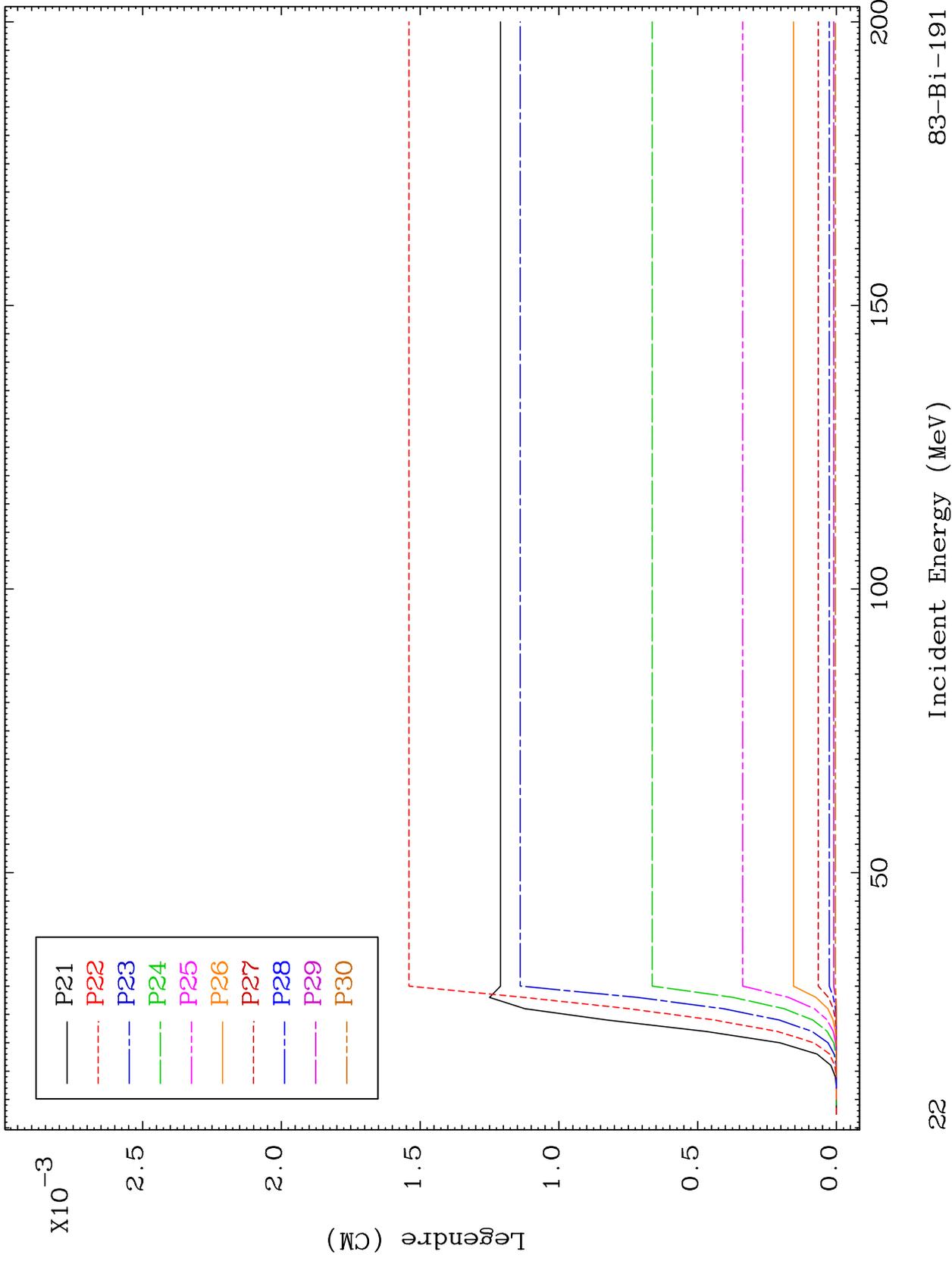
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MAT 8271

MT= 51 (n,n') Level  
Legendre Coefficients

83-Bi-191

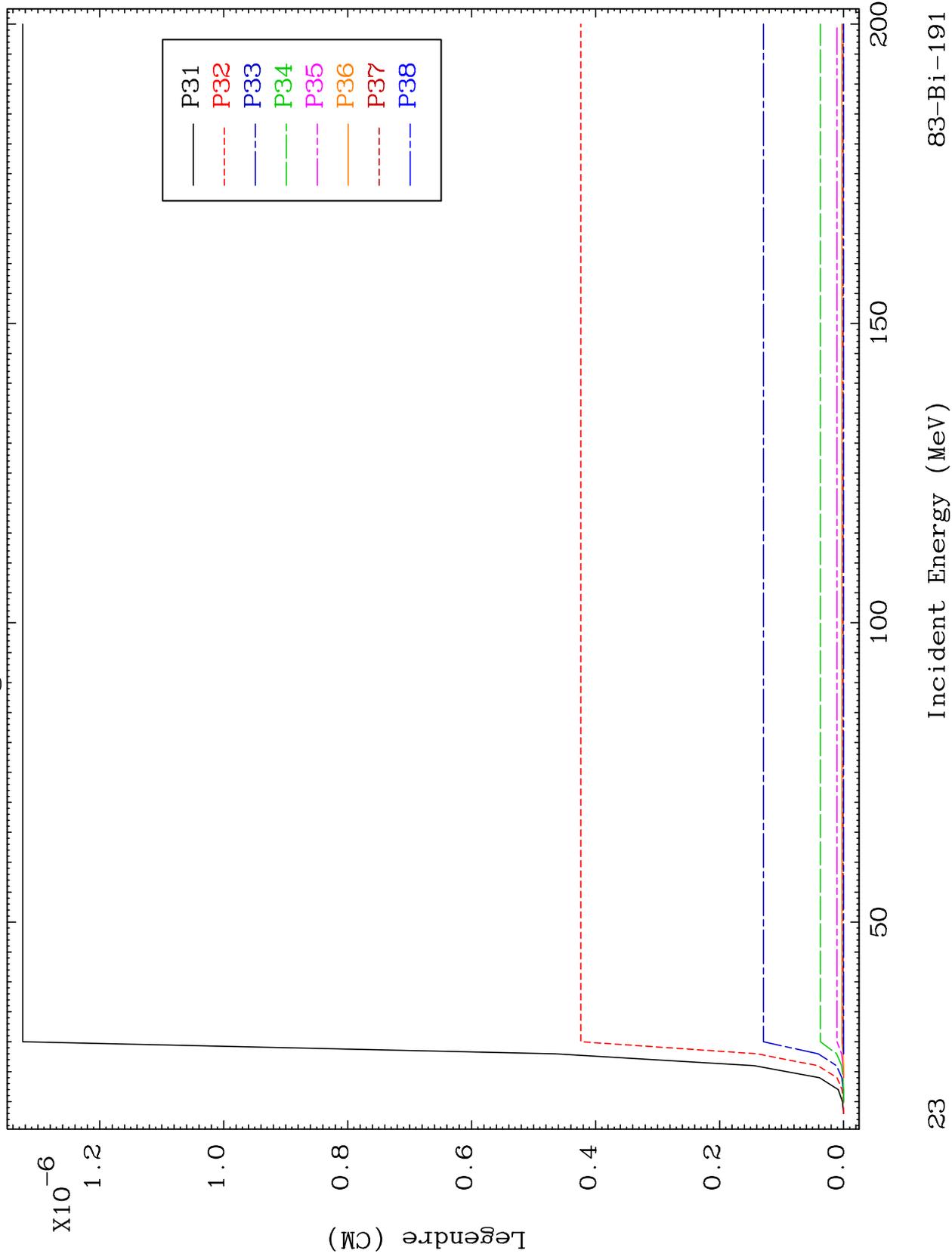




MAT 8271

MT= 51 (n,n') Level  
Legendre Coefficients

83-Bi-191



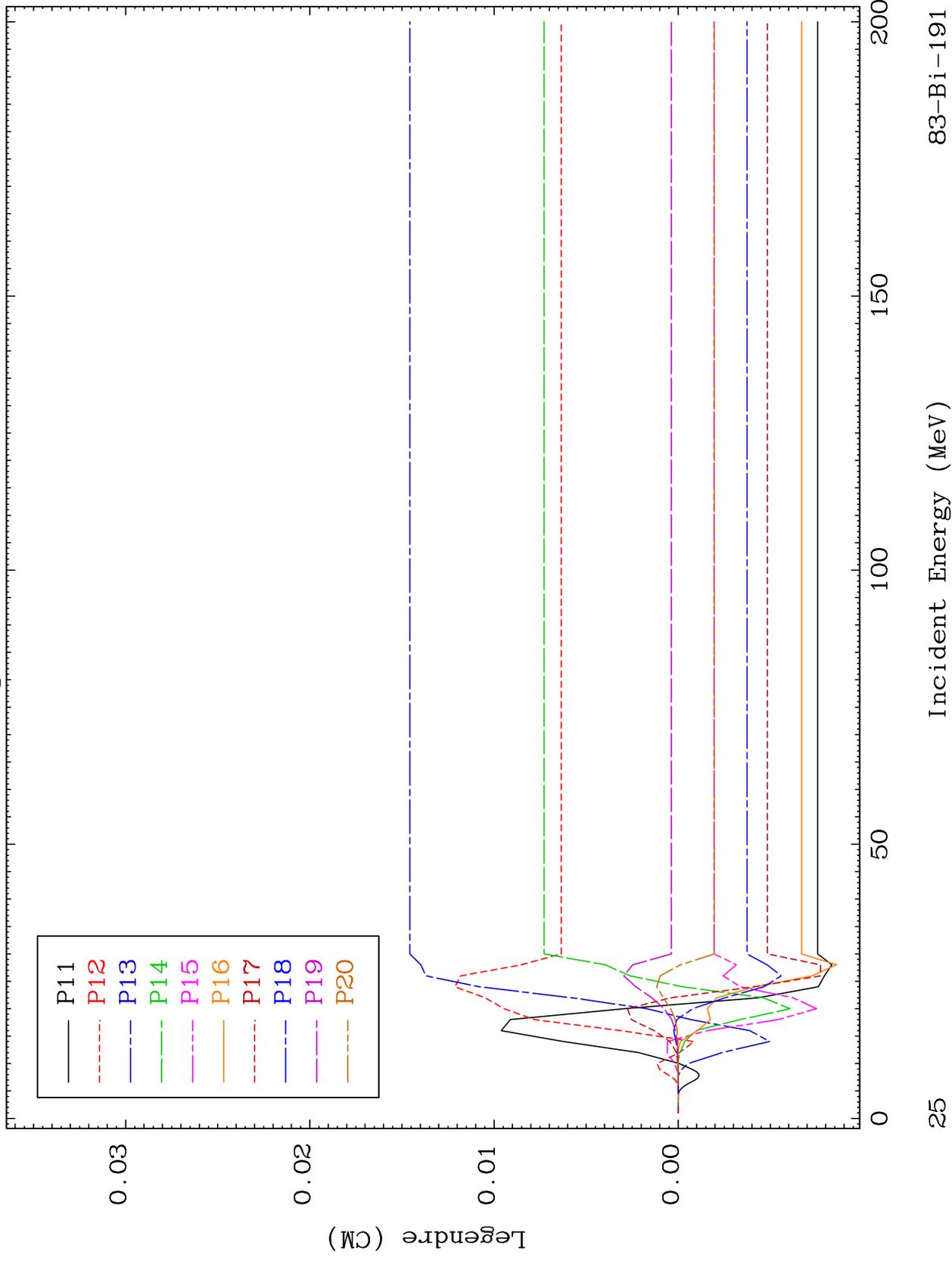
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MAT 8271

MT= 52 (n,n') Level  
Legendre Coefficients

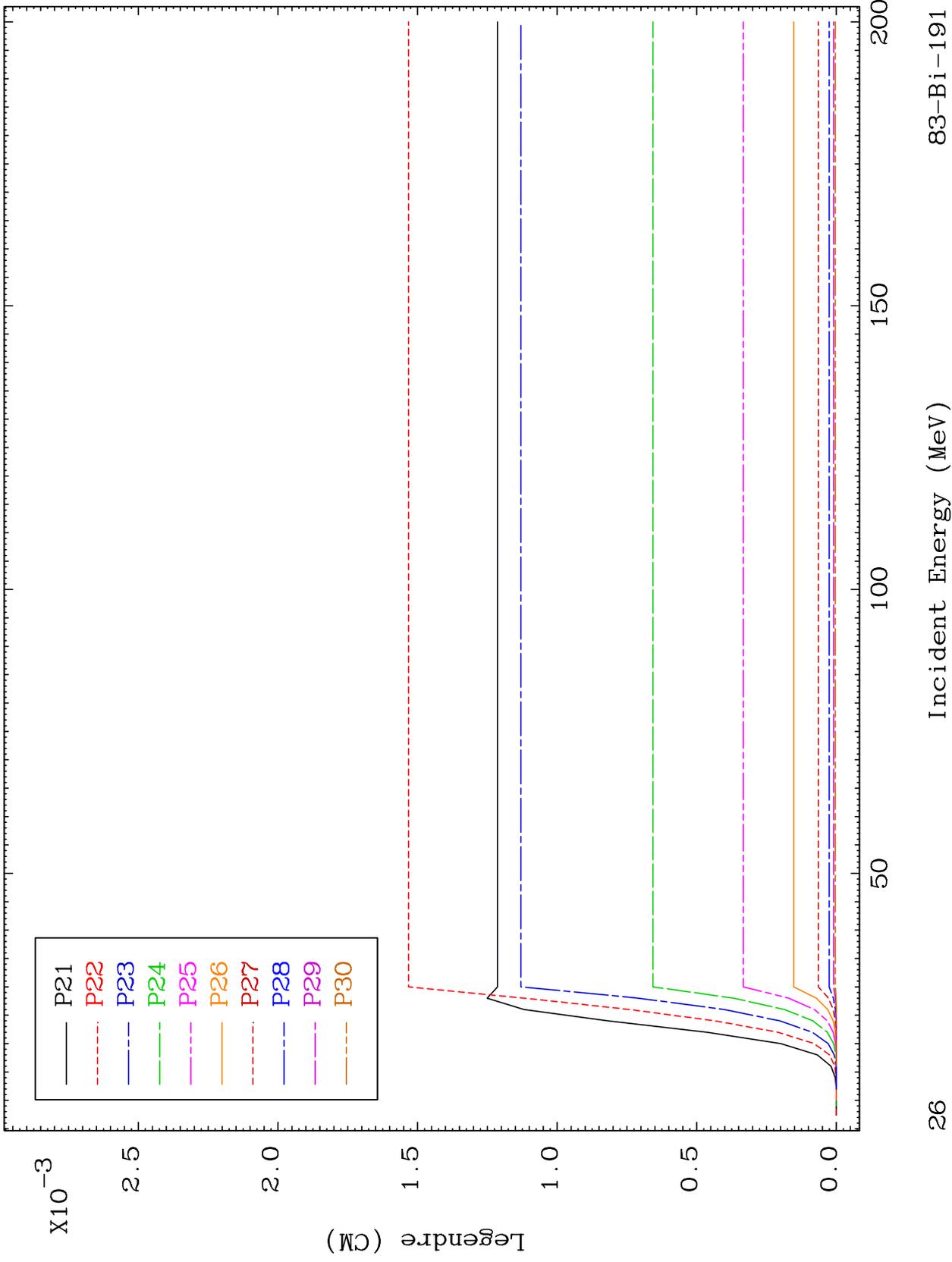
83-Bi-191



83-Bi-191

Incident Energy (MeV)

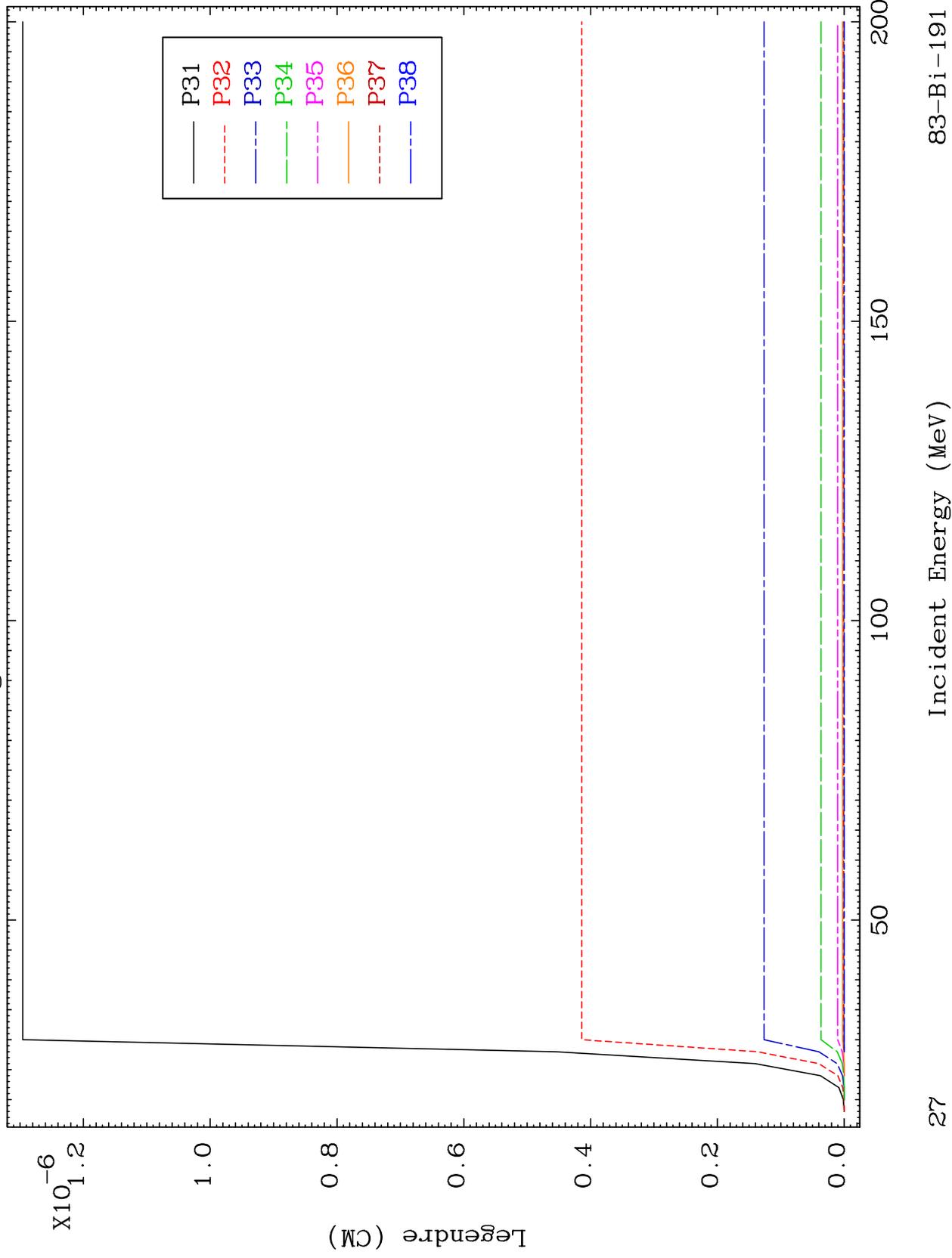
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MAT 8271

MT= 52 (n,n') Level  
Legendre Coefficients

83-Bi-191

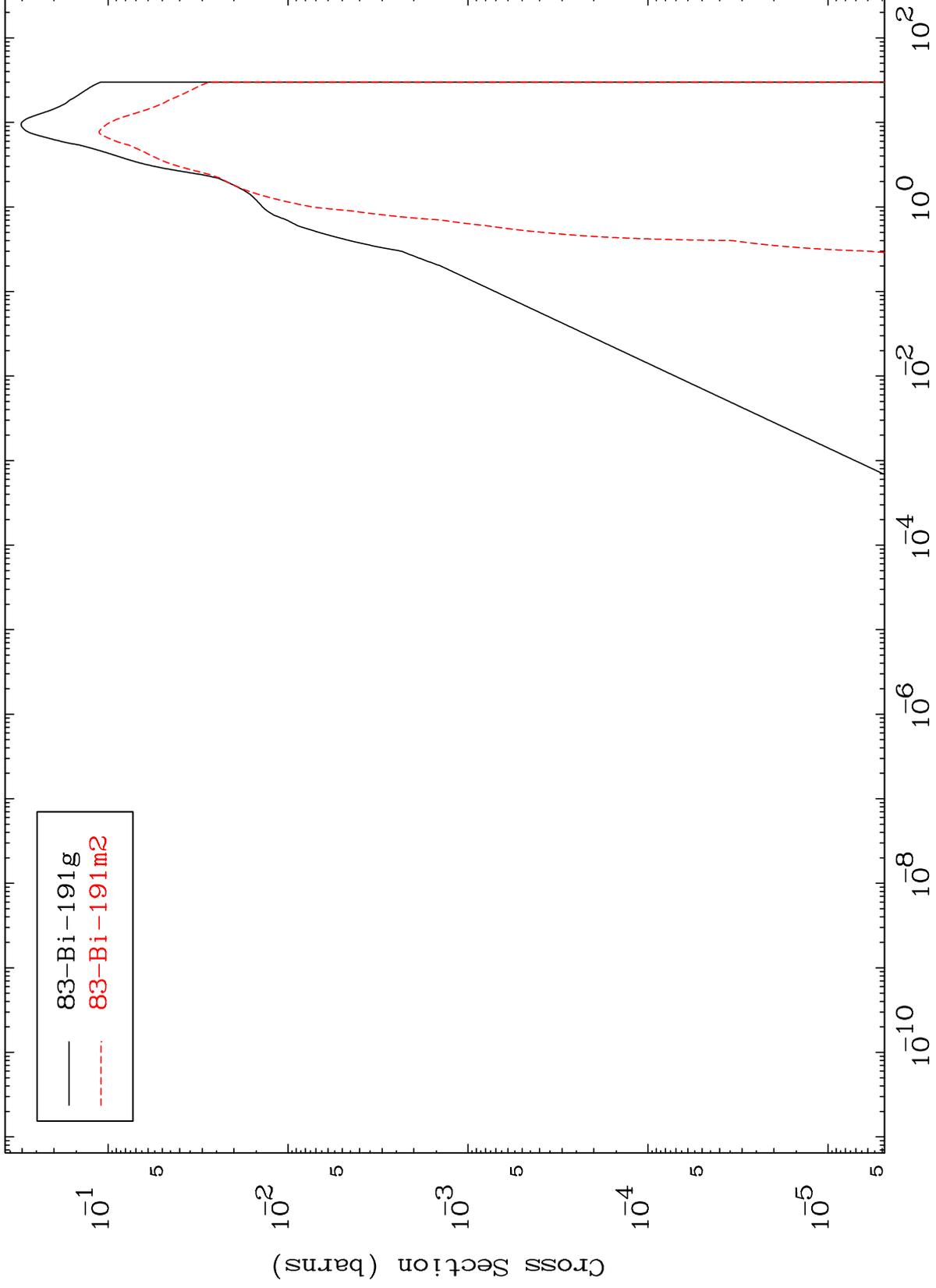


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Inelastic  
Radionuclide Production Cross Section

83-Bi-191

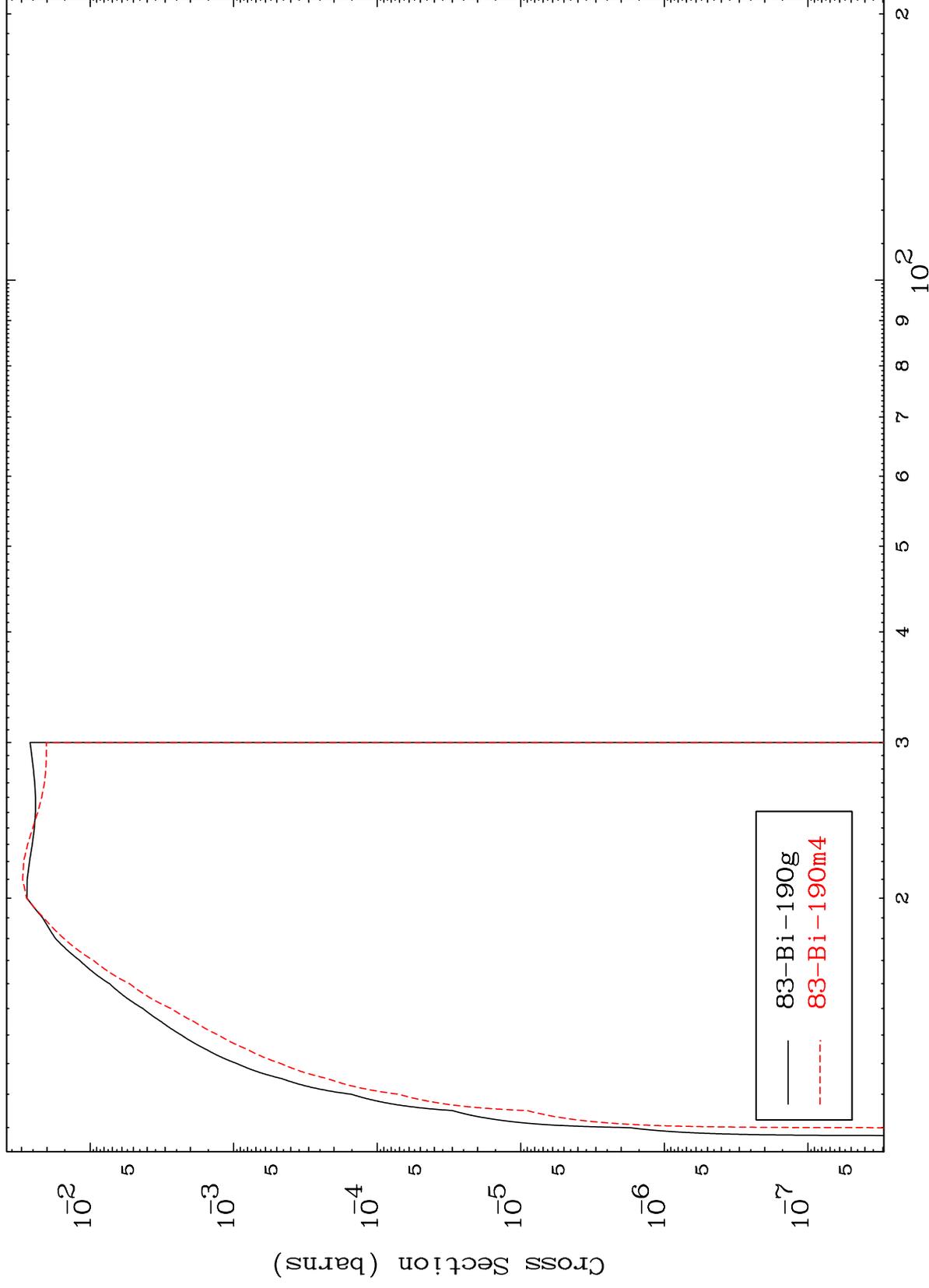


83-Bi-191

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83-Bi-191

(n,2n)  
Radionuclide Production Cross Section



29

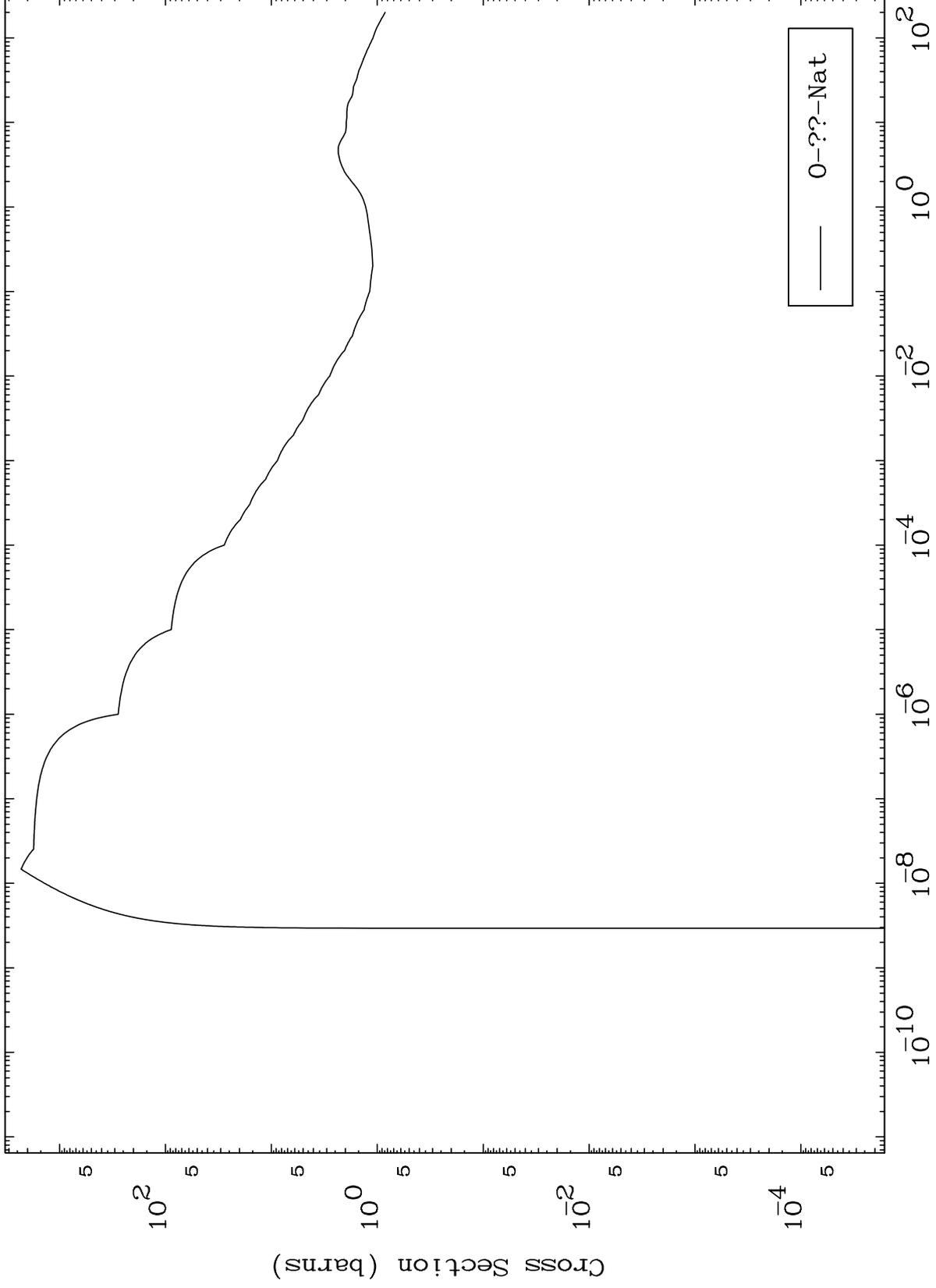
Incident Energy (MeV)

83-Bi-191

MAT 8271

83-Bi-191

Fission  
Radionuclide Production Cross Section



30

83-Bi-191

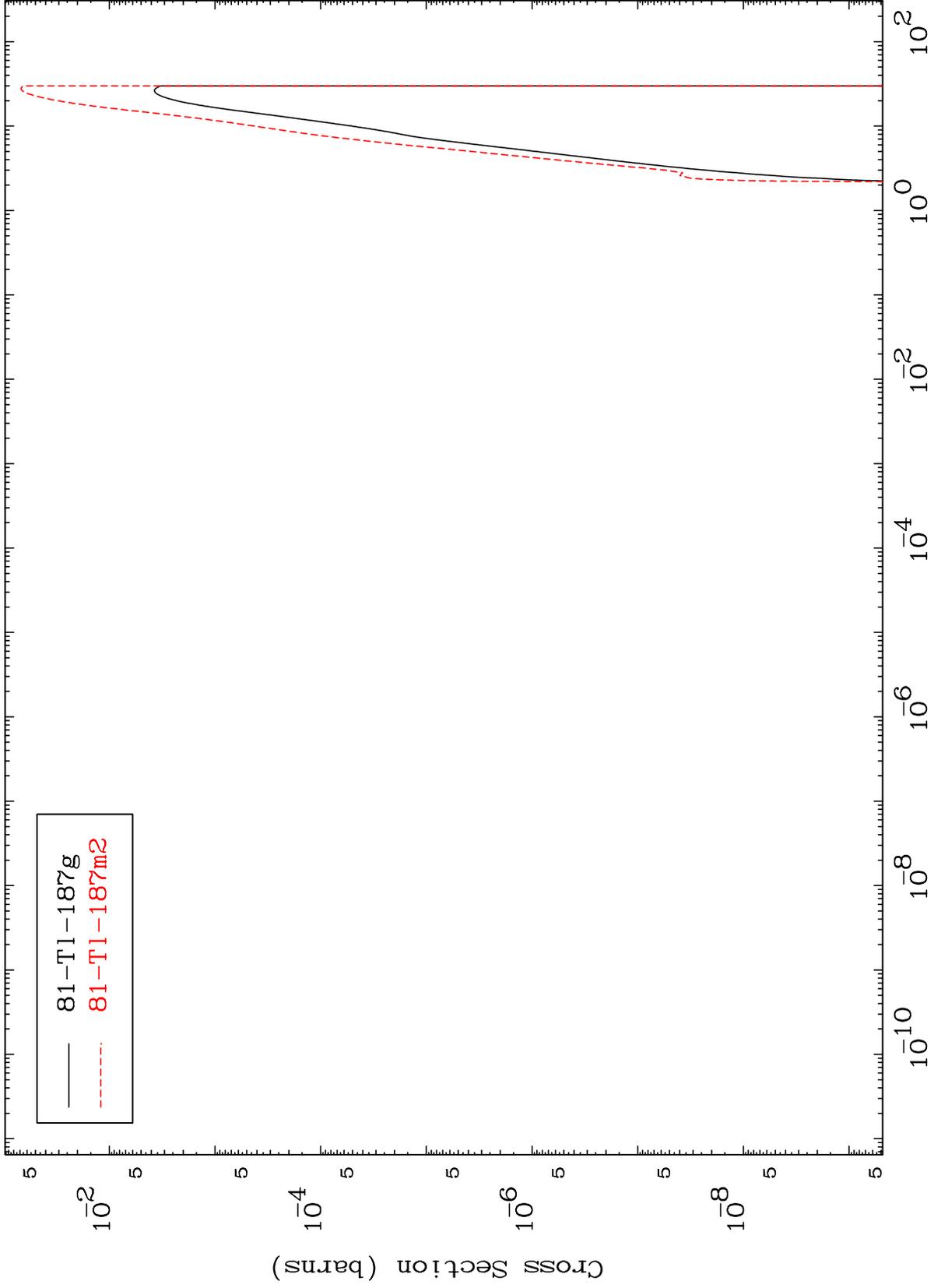
Incident Energy (MeV)

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$(n, n') \alpha$

83-Bi-191

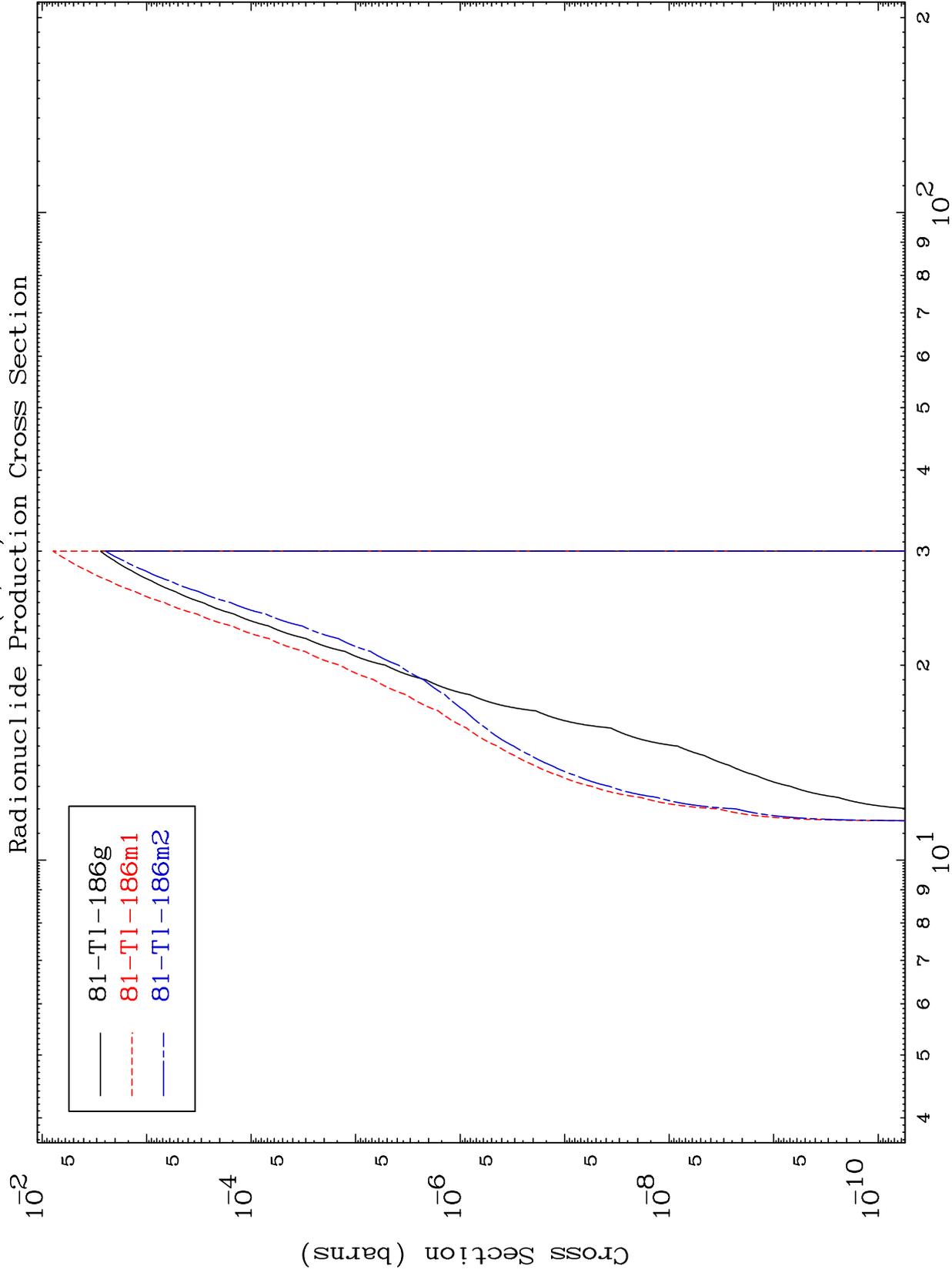
Radionuclide Production Cross Section



MAT 8271

$(n,2n) \alpha$

83-Bi-191

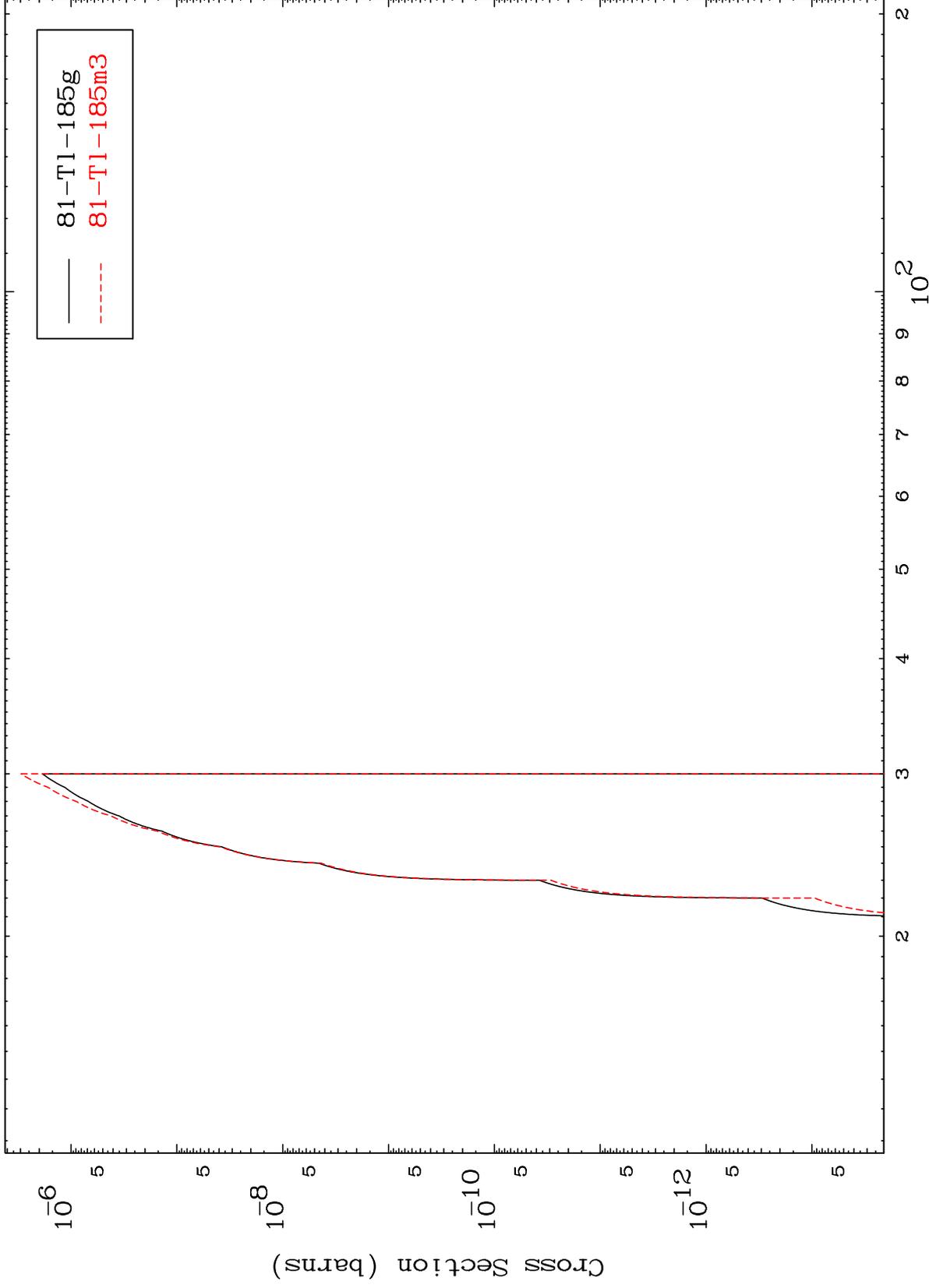


32

Incident Energy (MeV)

83-Bi-191

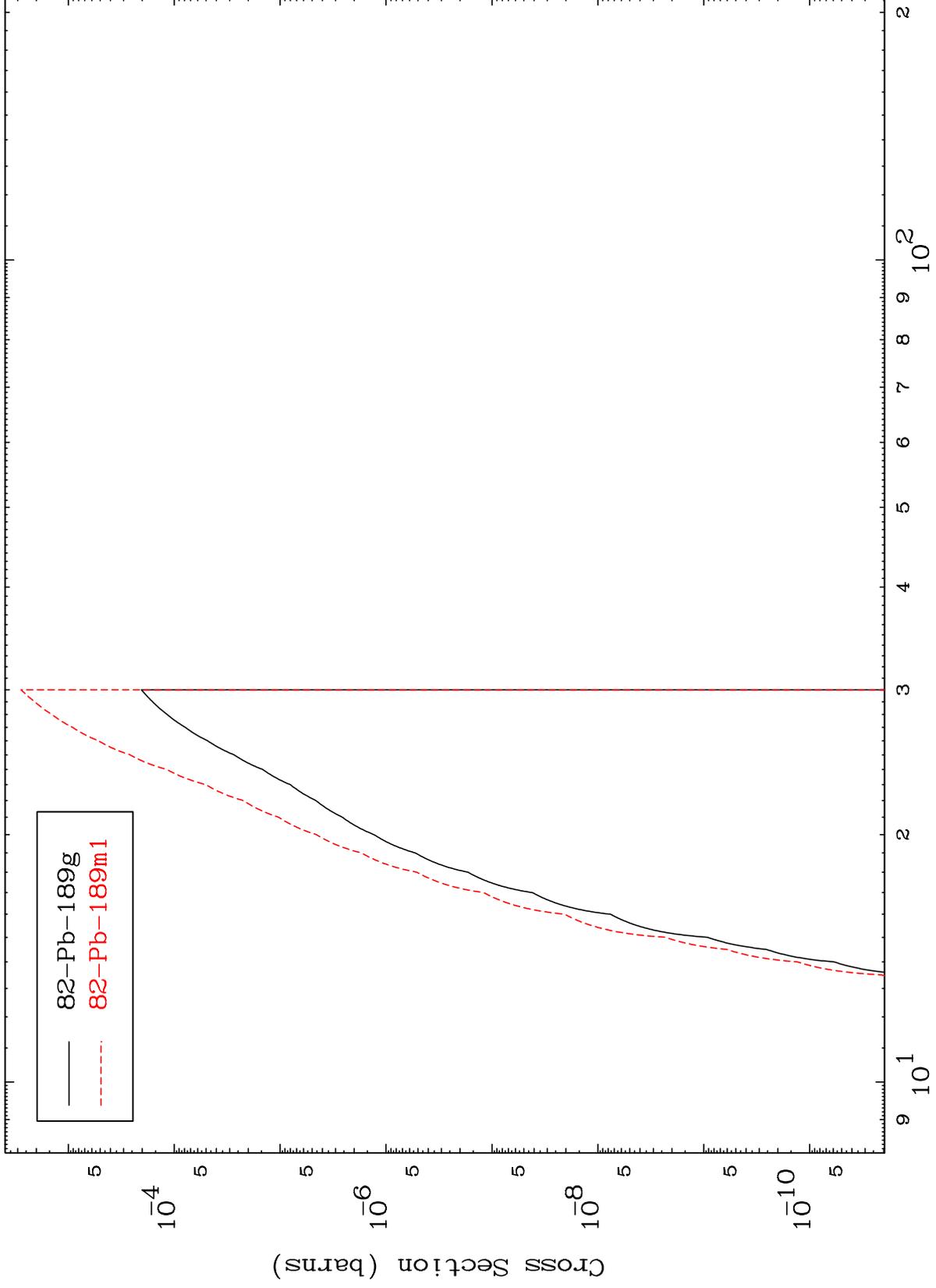
Radionuclide Production Cross Section



MAT 8271

83-Bi-191

(n,n') d  
Radionuclide Production Cross Section



34

Incident Energy (MeV)

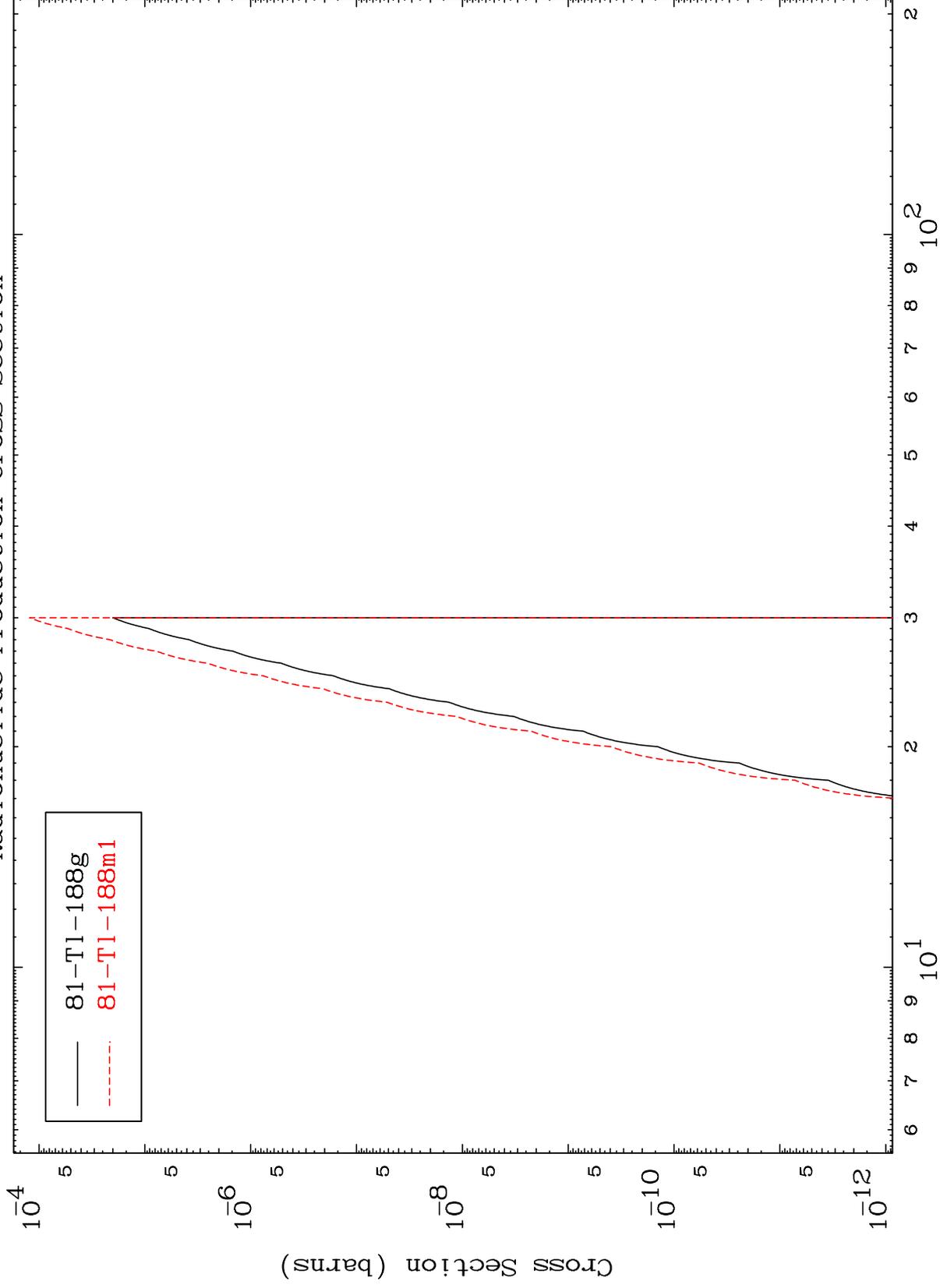
83-Bi-191

MAT 8271

(n,n') He-3

83-Bi-191

Radionuclide Production Cross Section



35

Incident Energy (MeV)

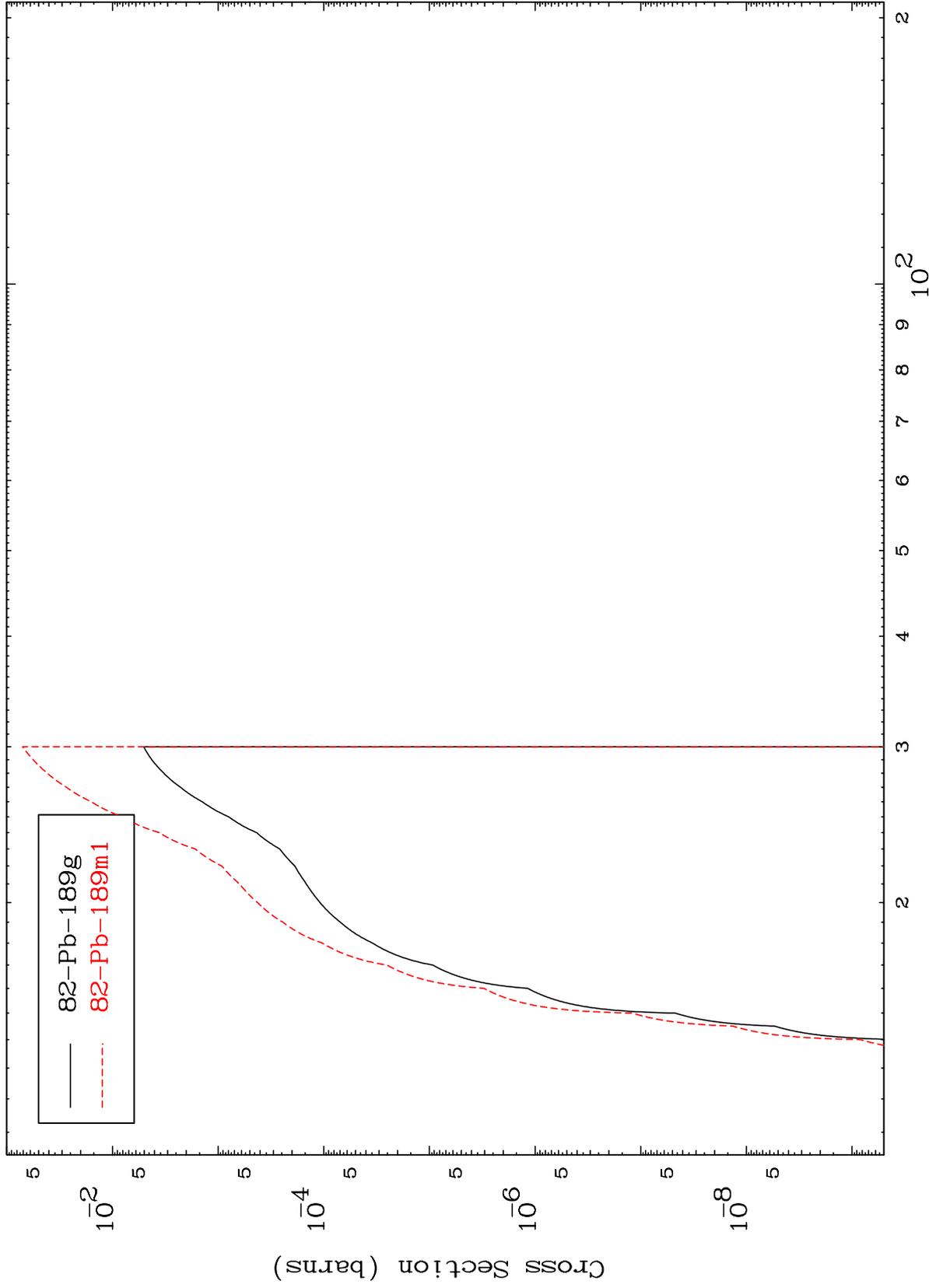
83-Bi-191

MAT 8271

(n,2n) p

83-Bi-191

Radionuclide Production Cross Section



36

Incident Energy (MeV)

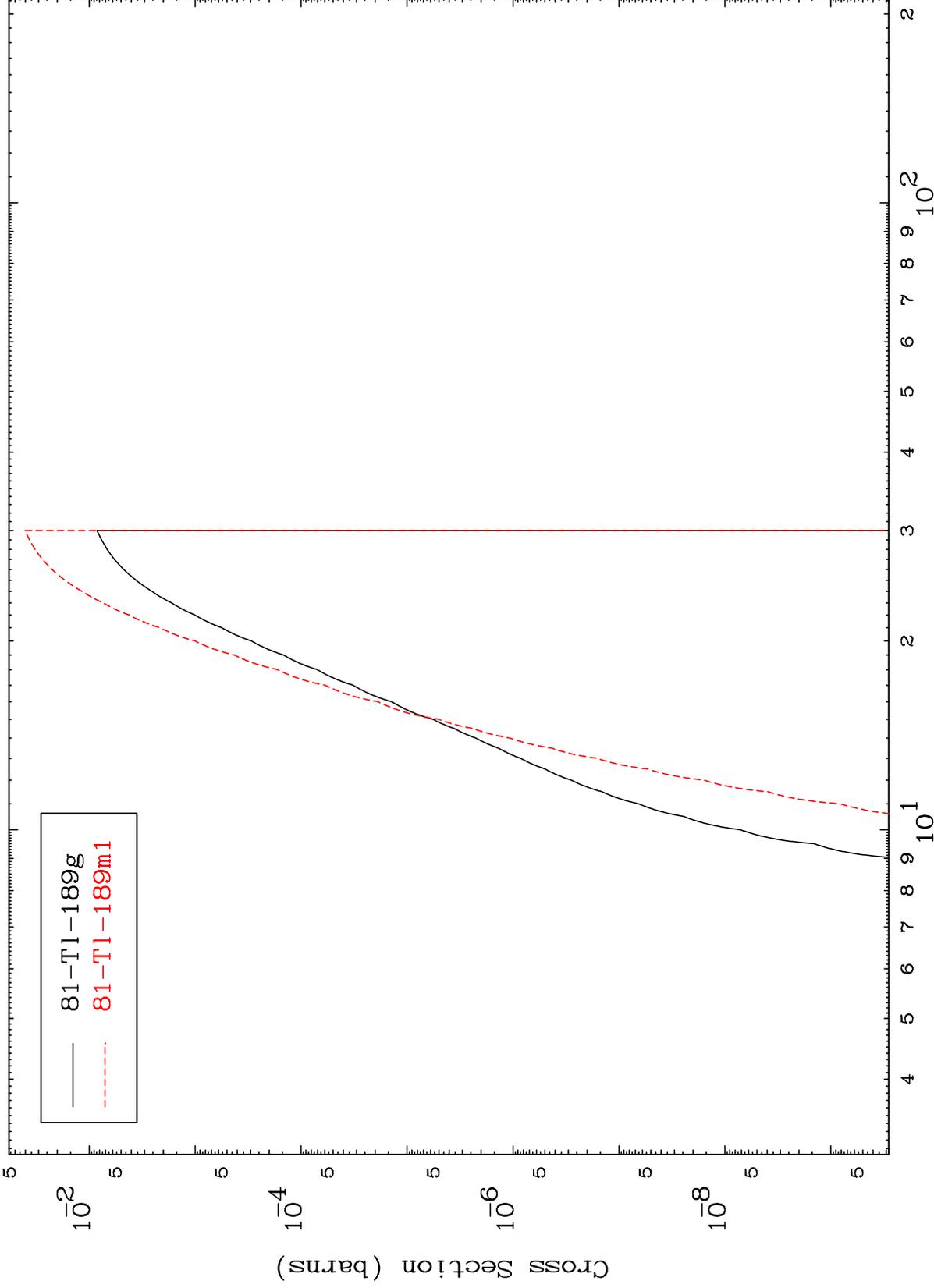
83-Bi-191

MAT 8271

(n,2n) p

83-Bi-191

Radionuclide Production Cross Section



37

Incident Energy (MeV)

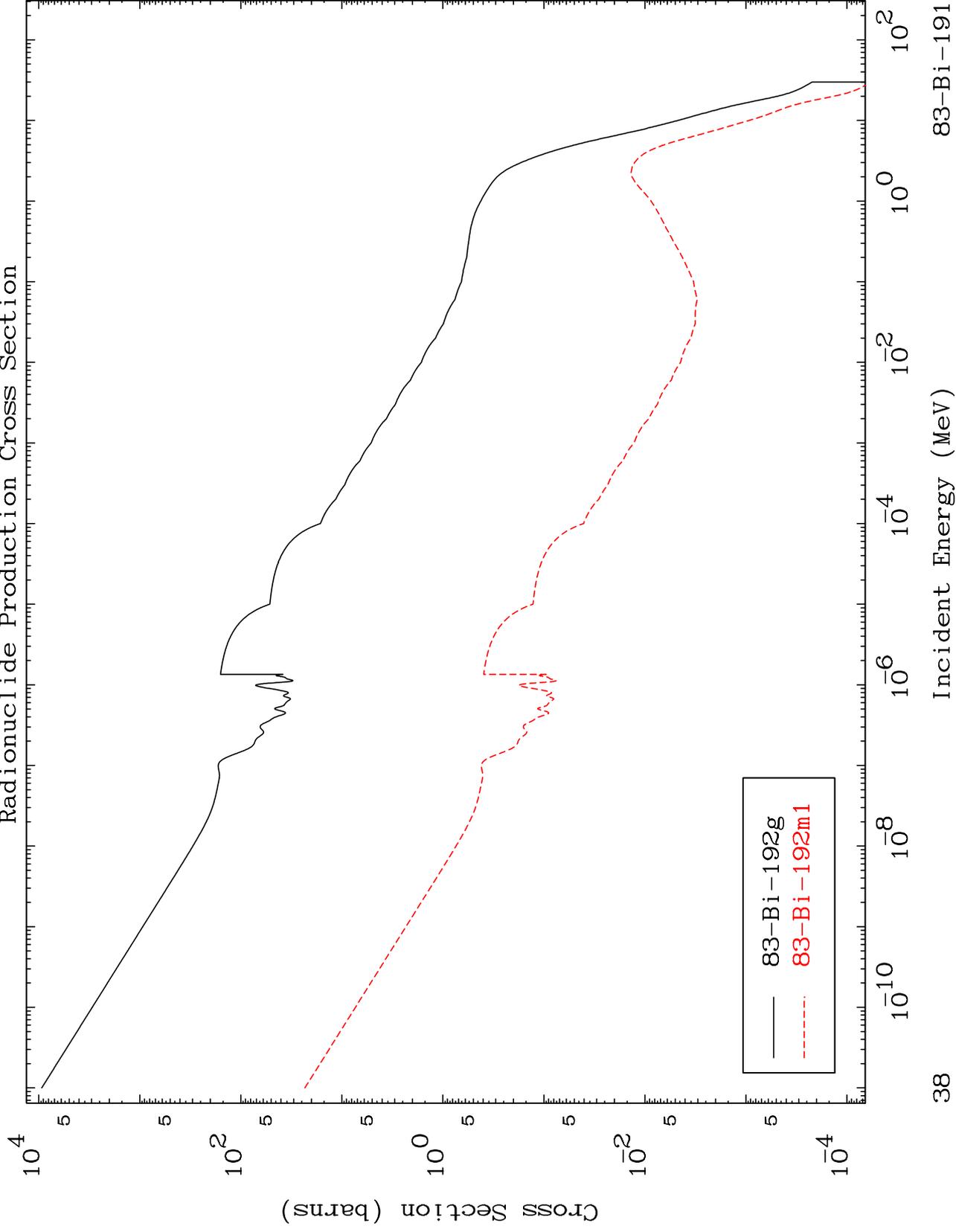
83-Bi-191

MAT 8271

83-Bi-191

Radionuclide Production Cross Section

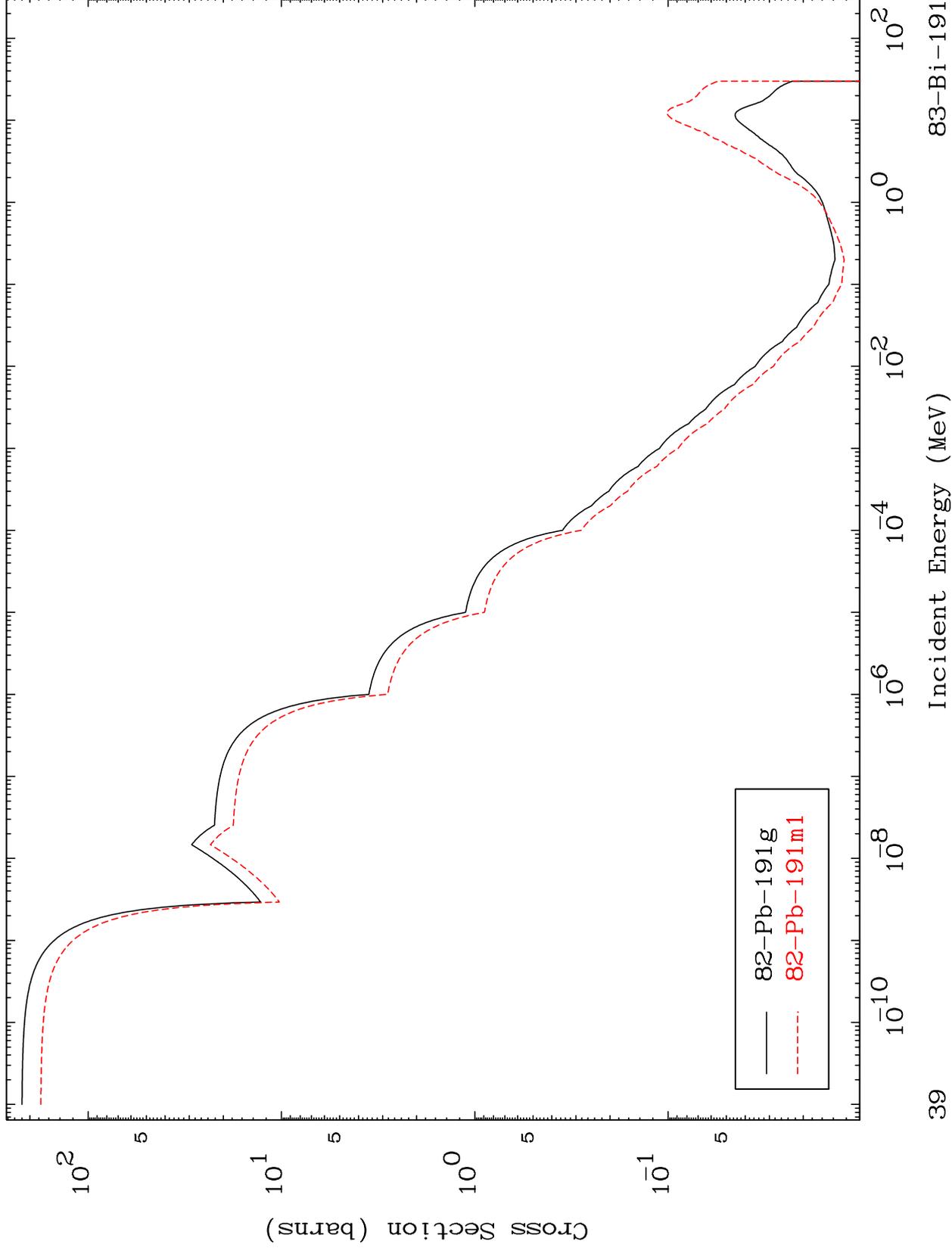
(n,  $\gamma$ )



MAT 8271

83-Bi-191

(n,p)  
Radionuclide Production Cross Section

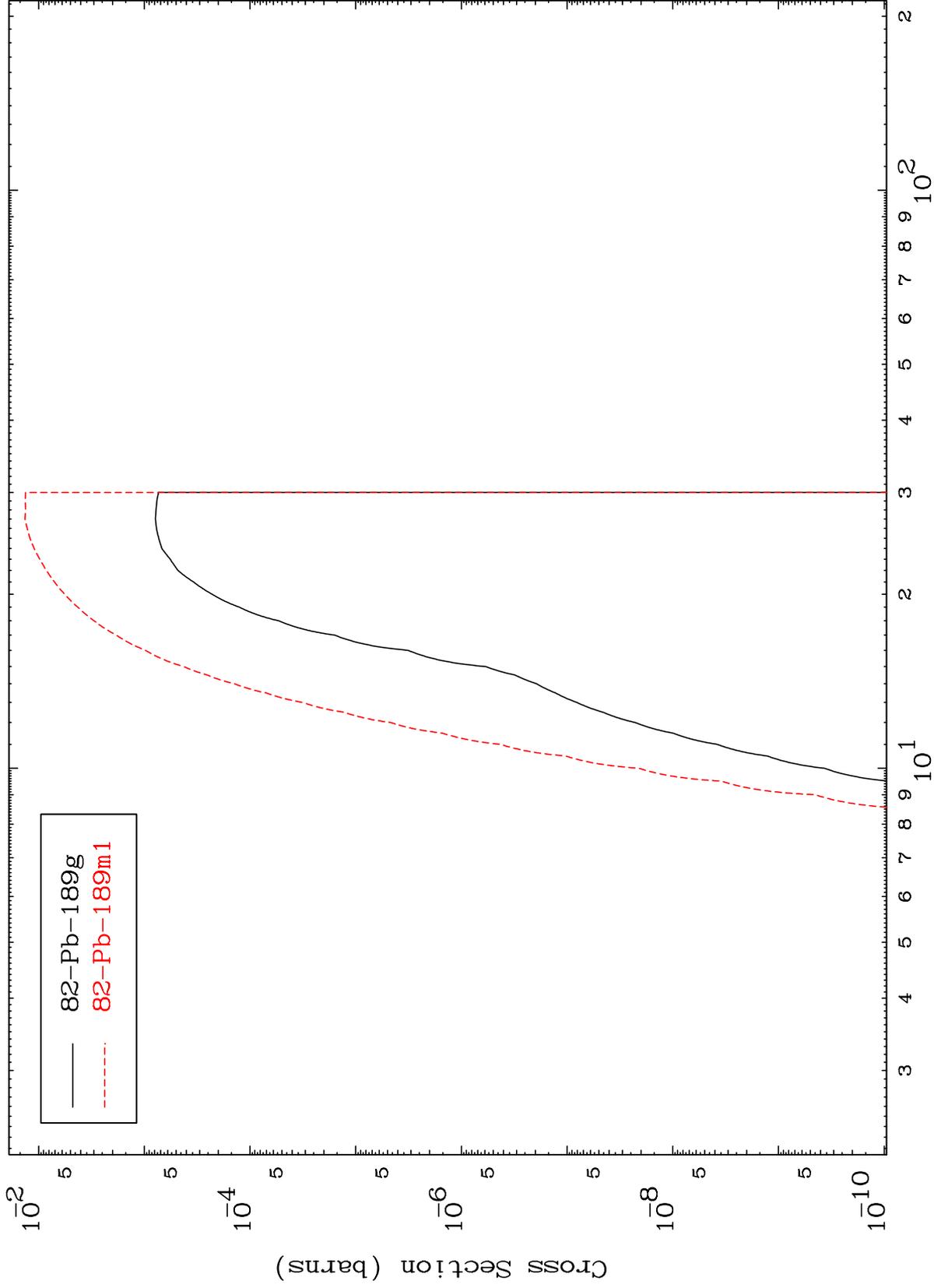


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83-Bi-191

(n,t)  
Radionuclide Production Cross Section



40

Incident Energy (MeV)

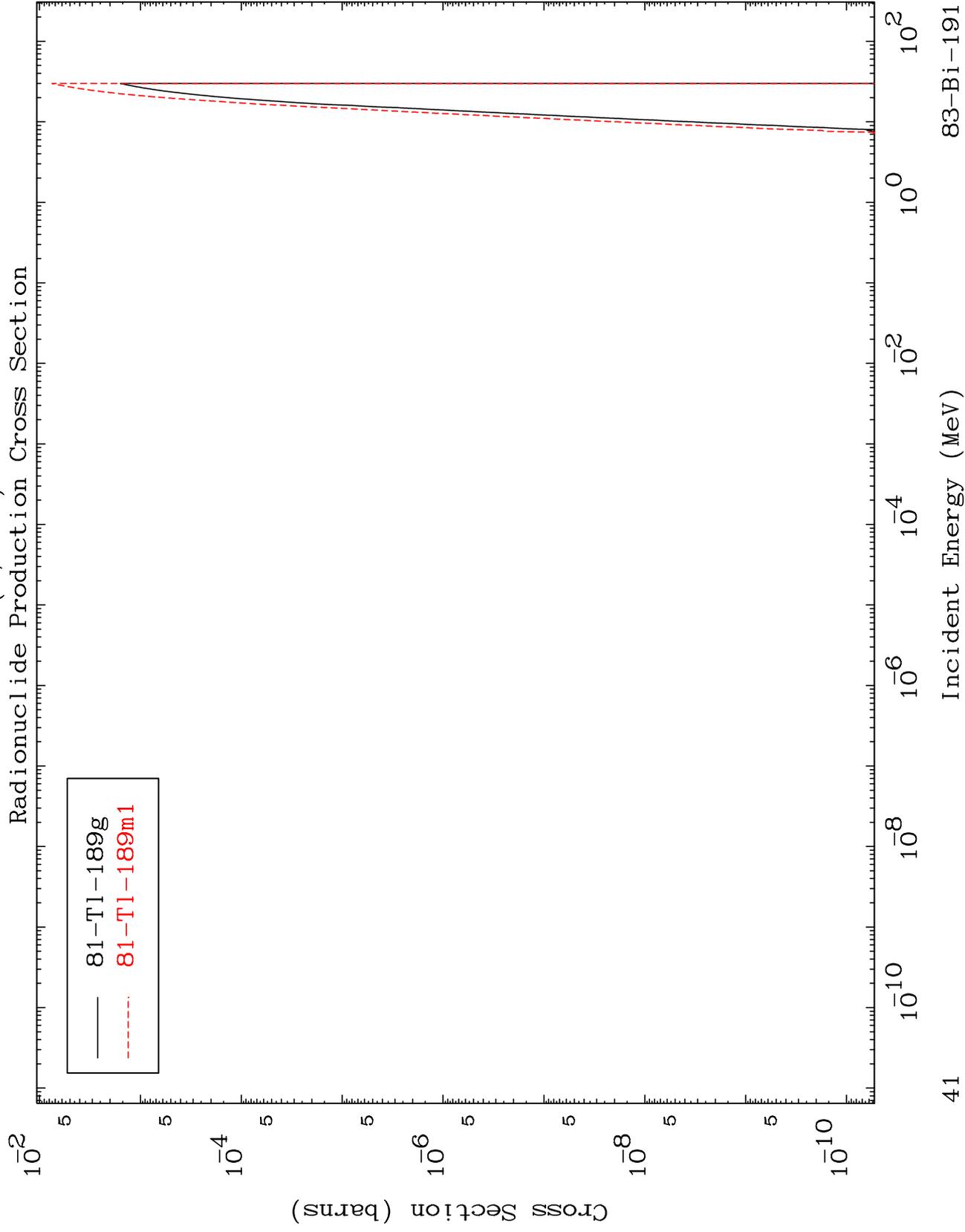
83-Bi-191

MAT 8271

(n,He-3)

83-Bi-191

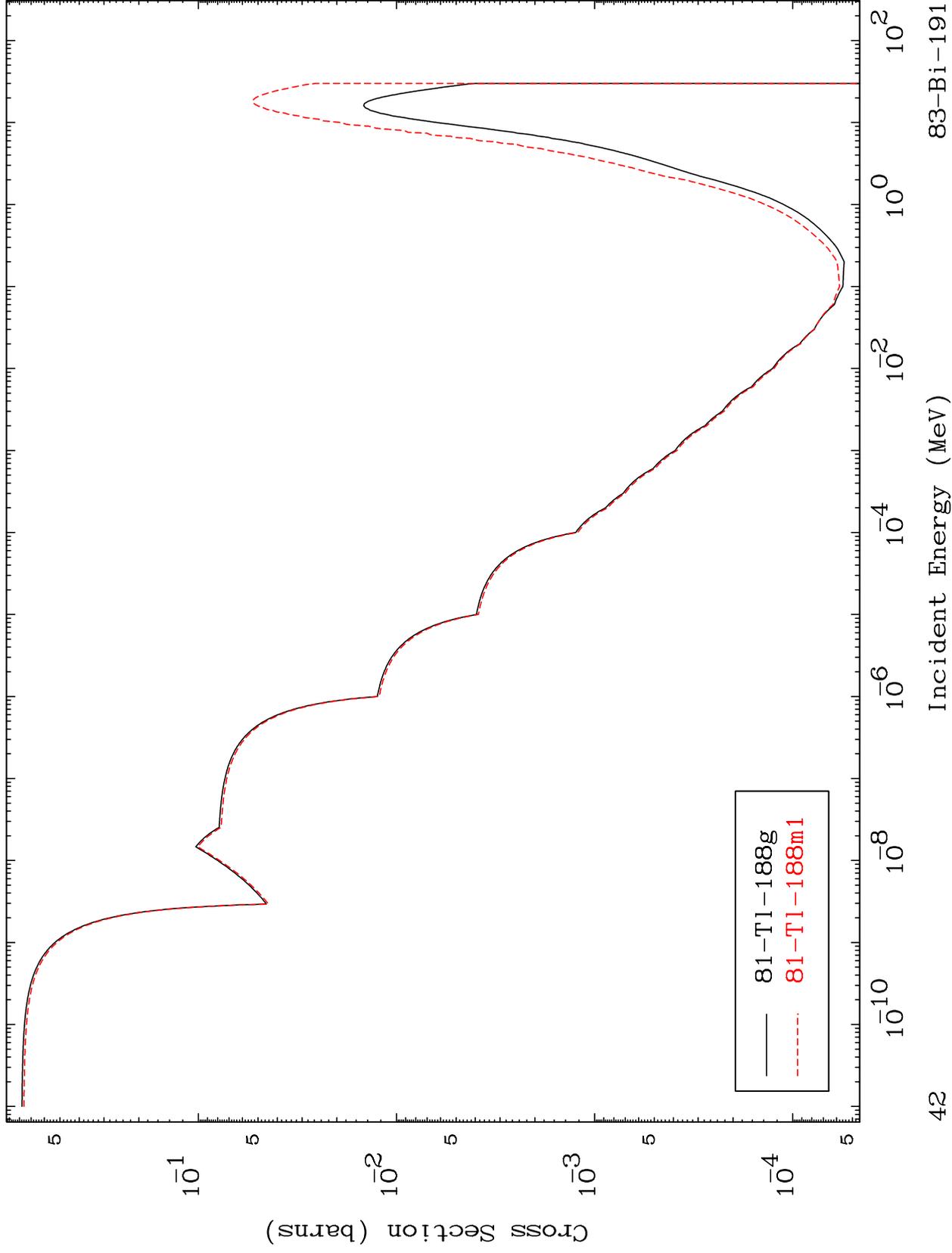
Radionuclide Production Cross Section



MAT 8271

83-Bi-191

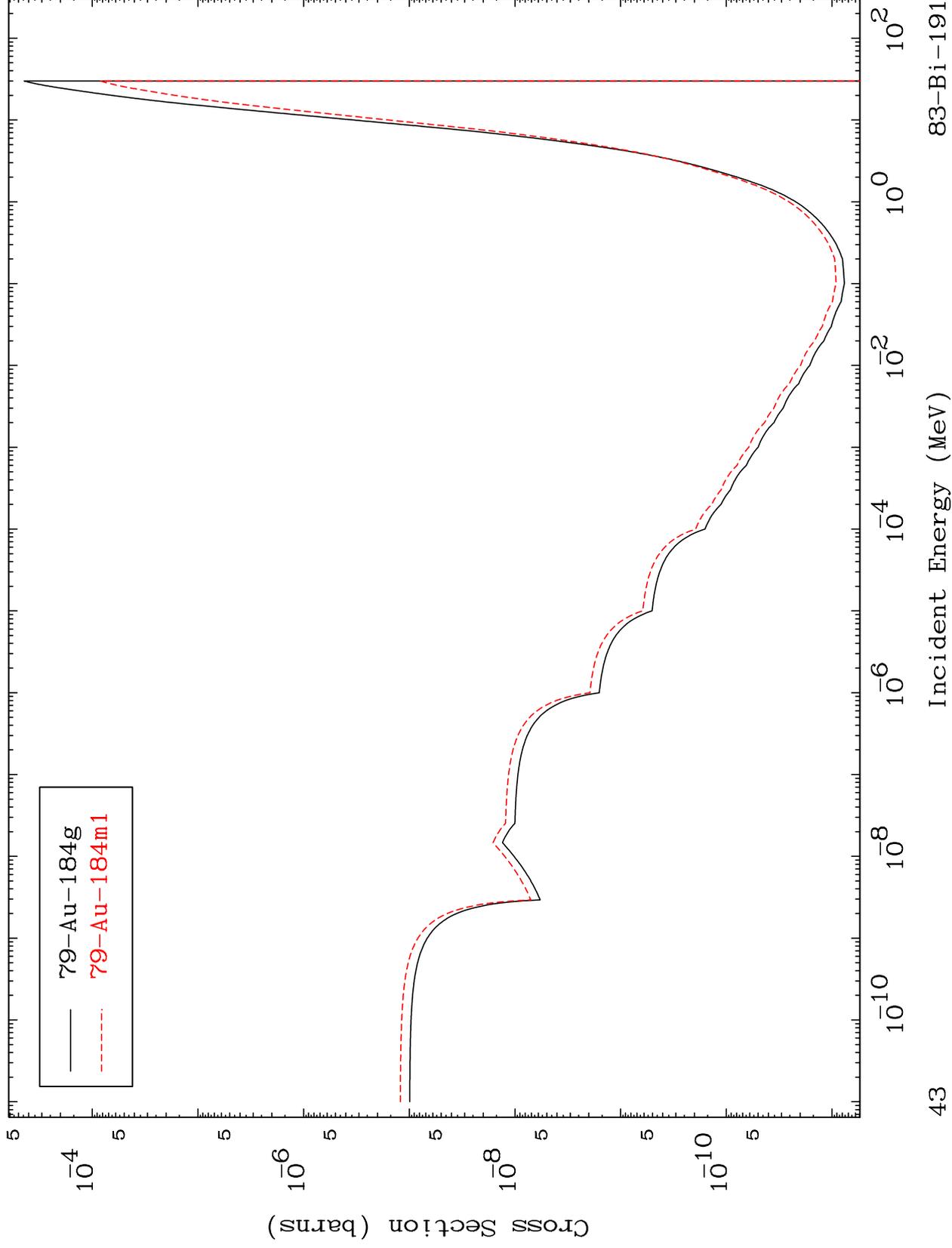
(n,  $\alpha$ )  
Radionuclide Production Cross Section



MAT 8271

83-Bi-191

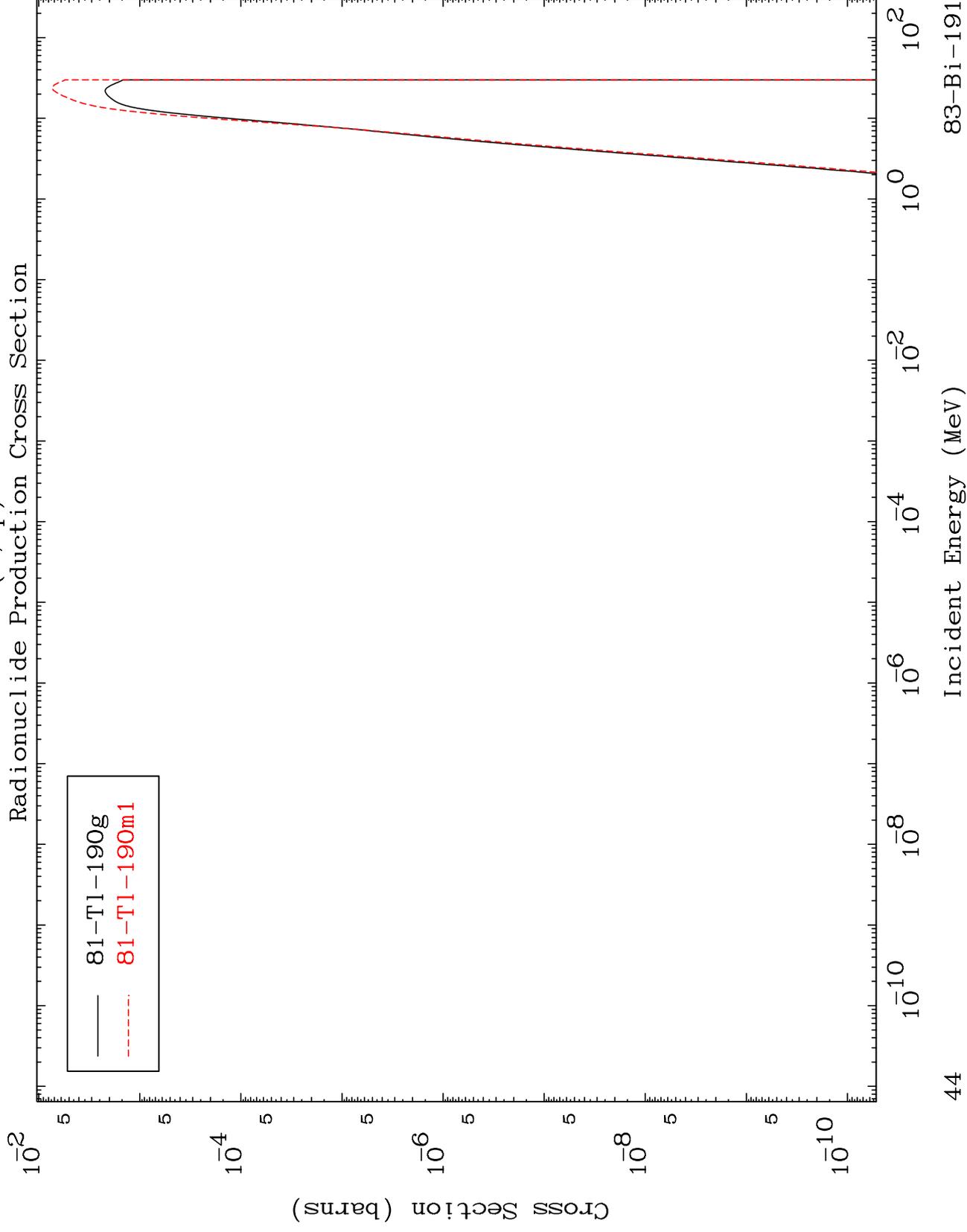
Radionuclide Production Cross Section  
(n,2 $\alpha$ )



MAT 8271

83-Bi-191

(n,2p)  
Radionuclide Production Cross Section

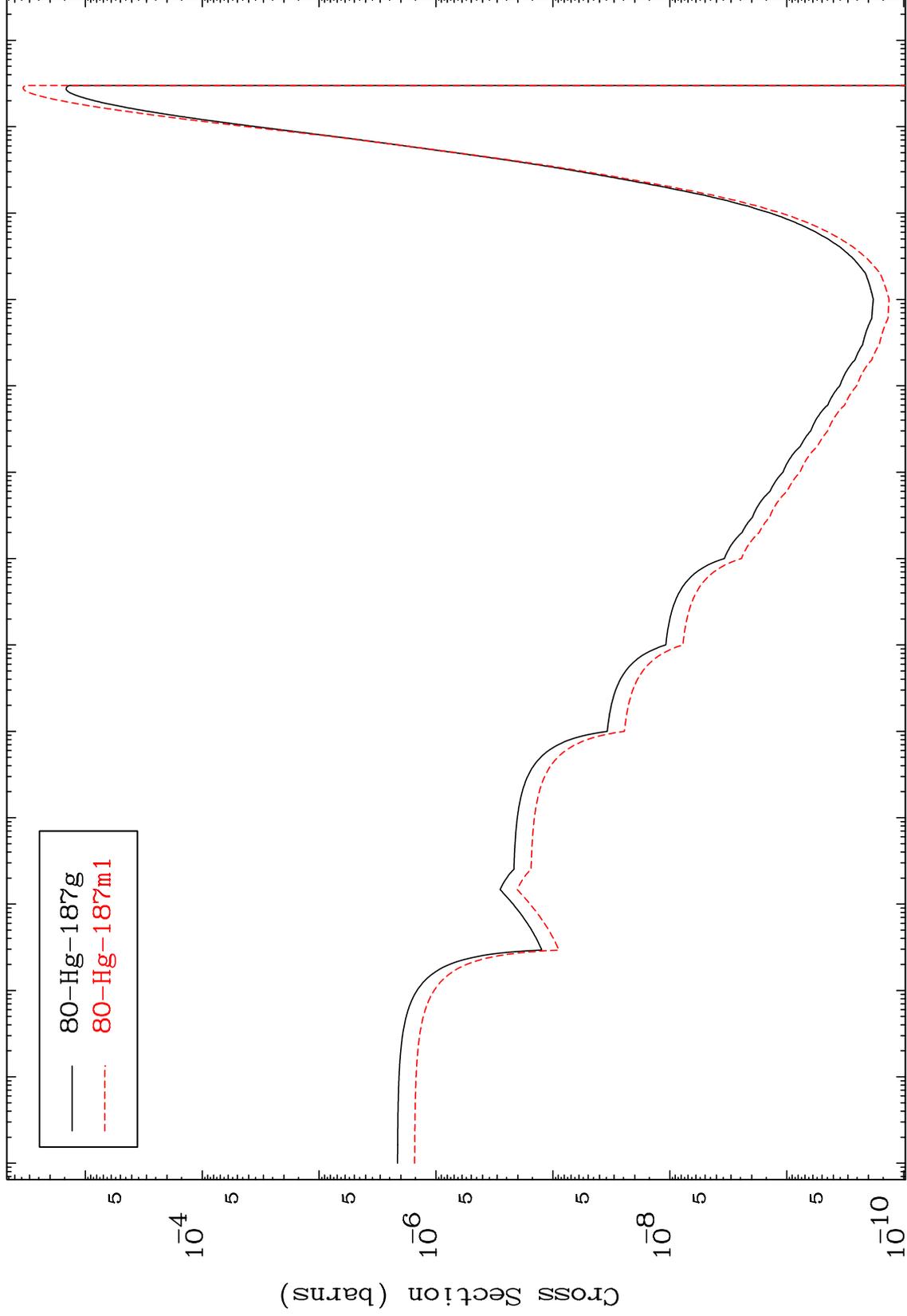


MAT 8271

(n,p)  $\alpha$

83-Bi-191

Radionuclide Production Cross Section



45

Incident Energy (MeV)

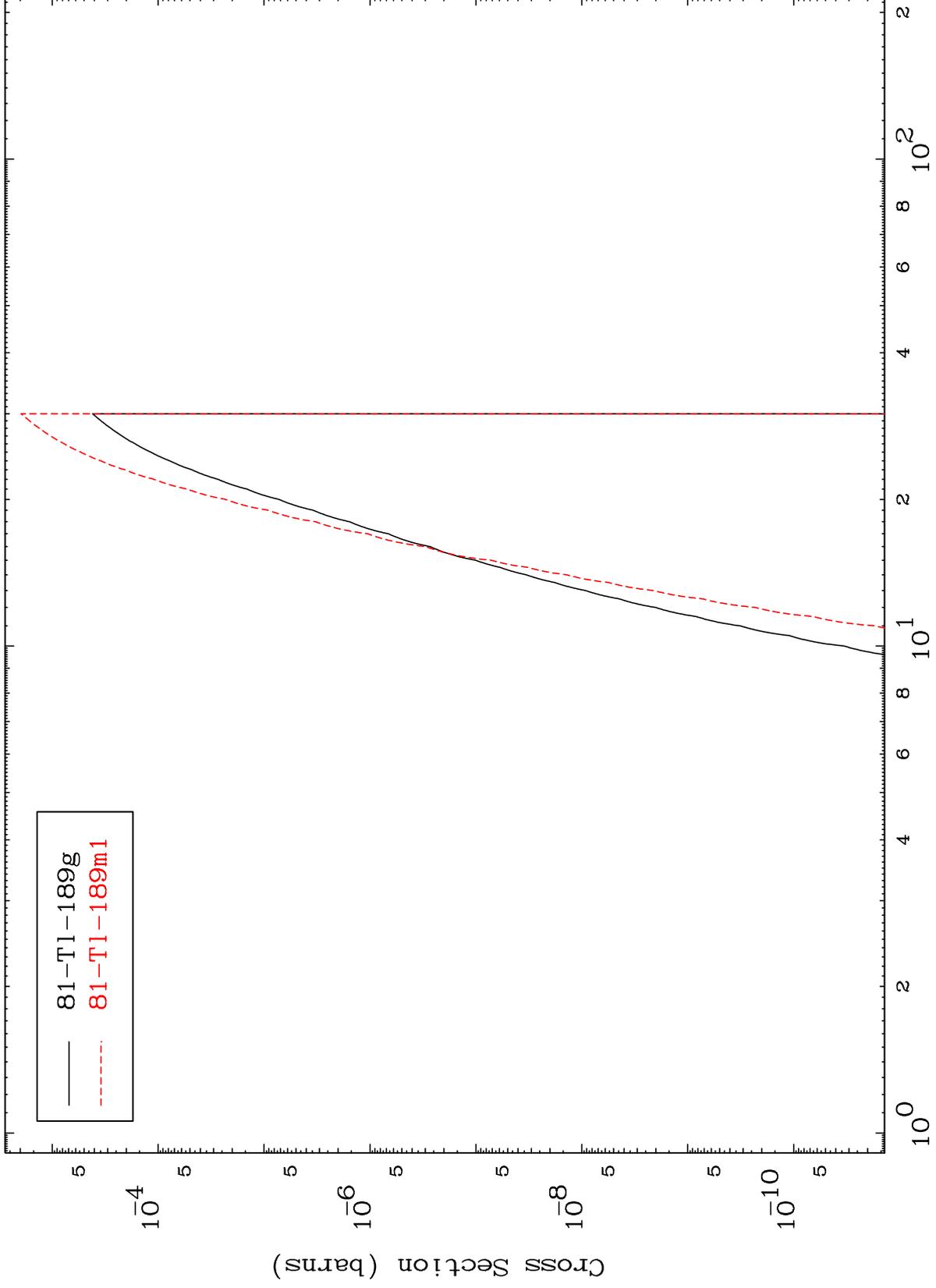
83-Bi-191

MAT 8271

(n,p) d

83-Bi-191

Radionuclide Production Cross Section



46

Incident Energy (MeV)

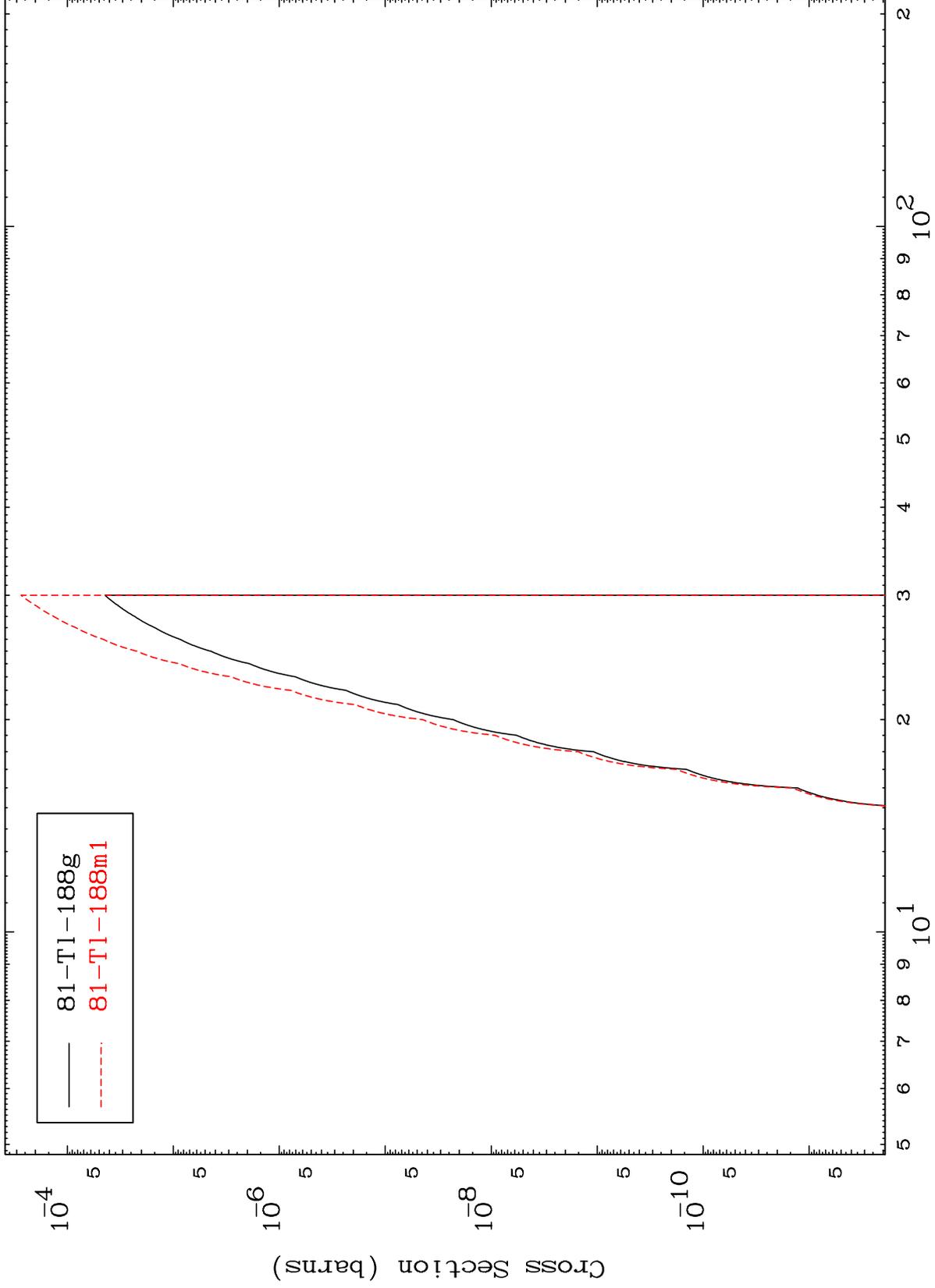
83-Bi-191

MAT 8271

(n,p) t

83-Bi-191

Radionuclide Production Cross Section



47

Incident Energy (MeV)

83-Bi-191