

Program EVALPLOT
(Version 2021-1)

by

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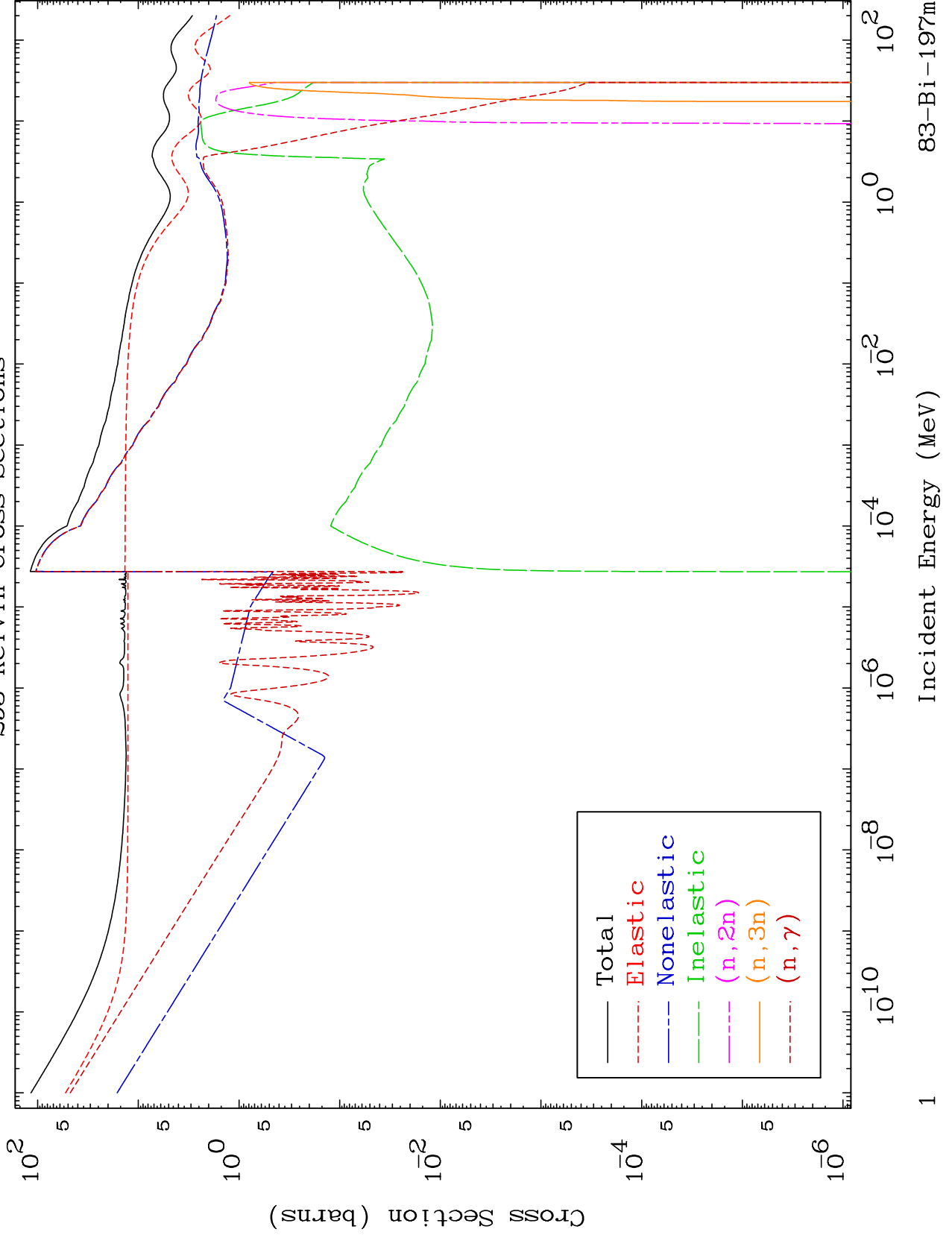
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Press Mouse Button to Start

MAT 8290

Neutron Major
293 Kelvin Cross Sections

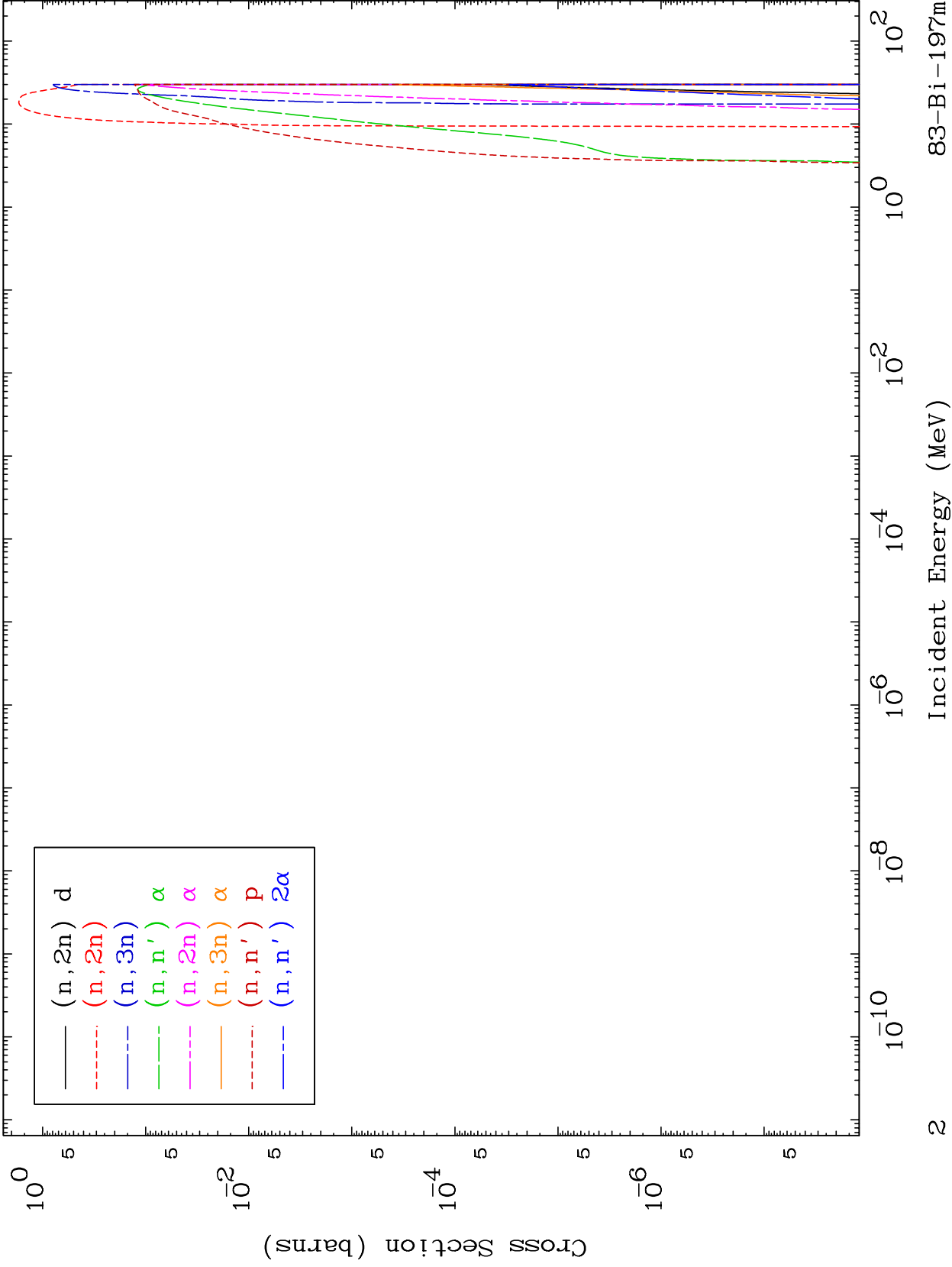
83-Bi-197m



MAT 8290

Neutron Absorption
293 Kelvin Cross Sections

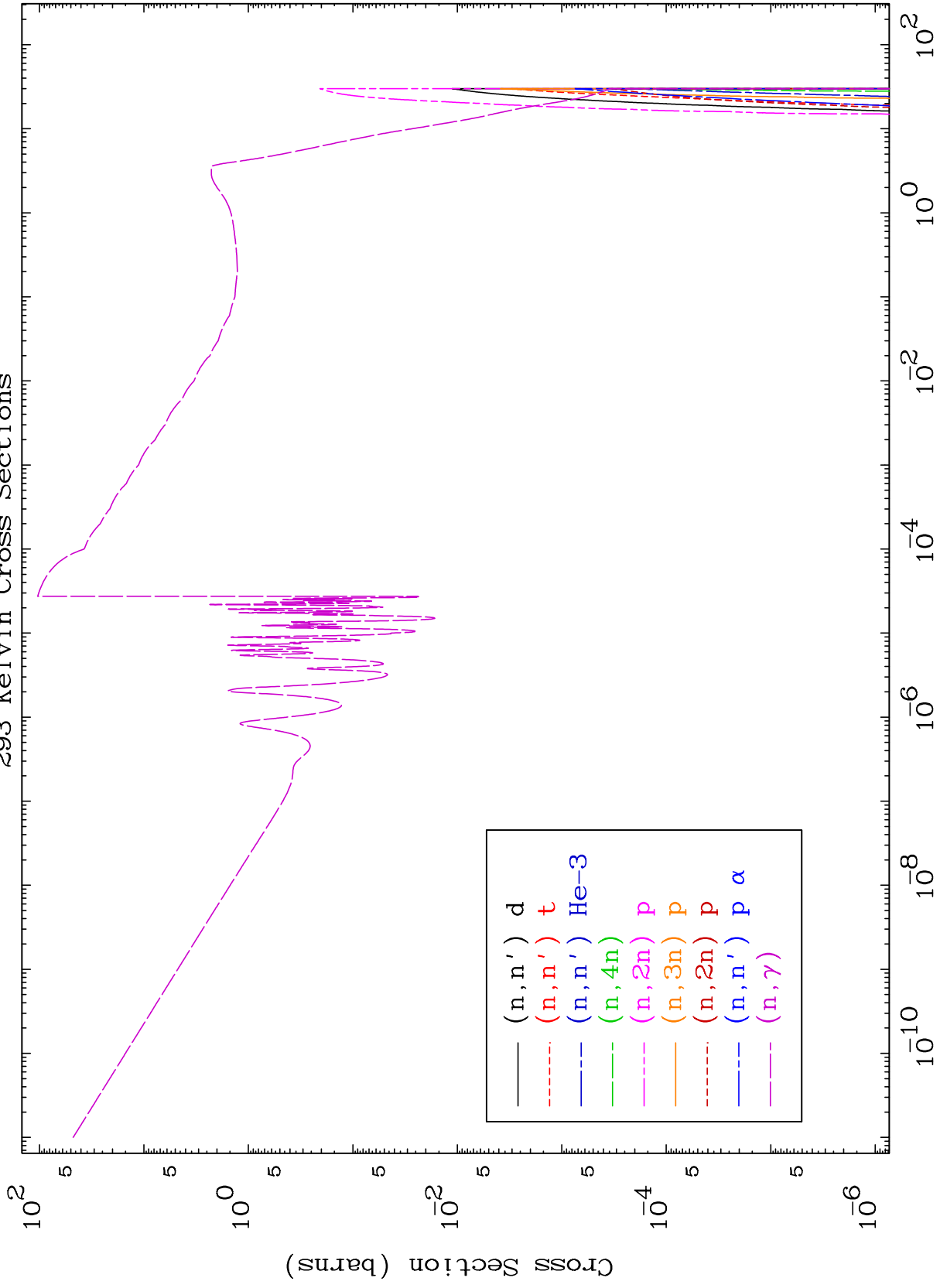
83-Bi-197m



MAT 8290

Neutron Absorption
293 Kelvin Cross Sections

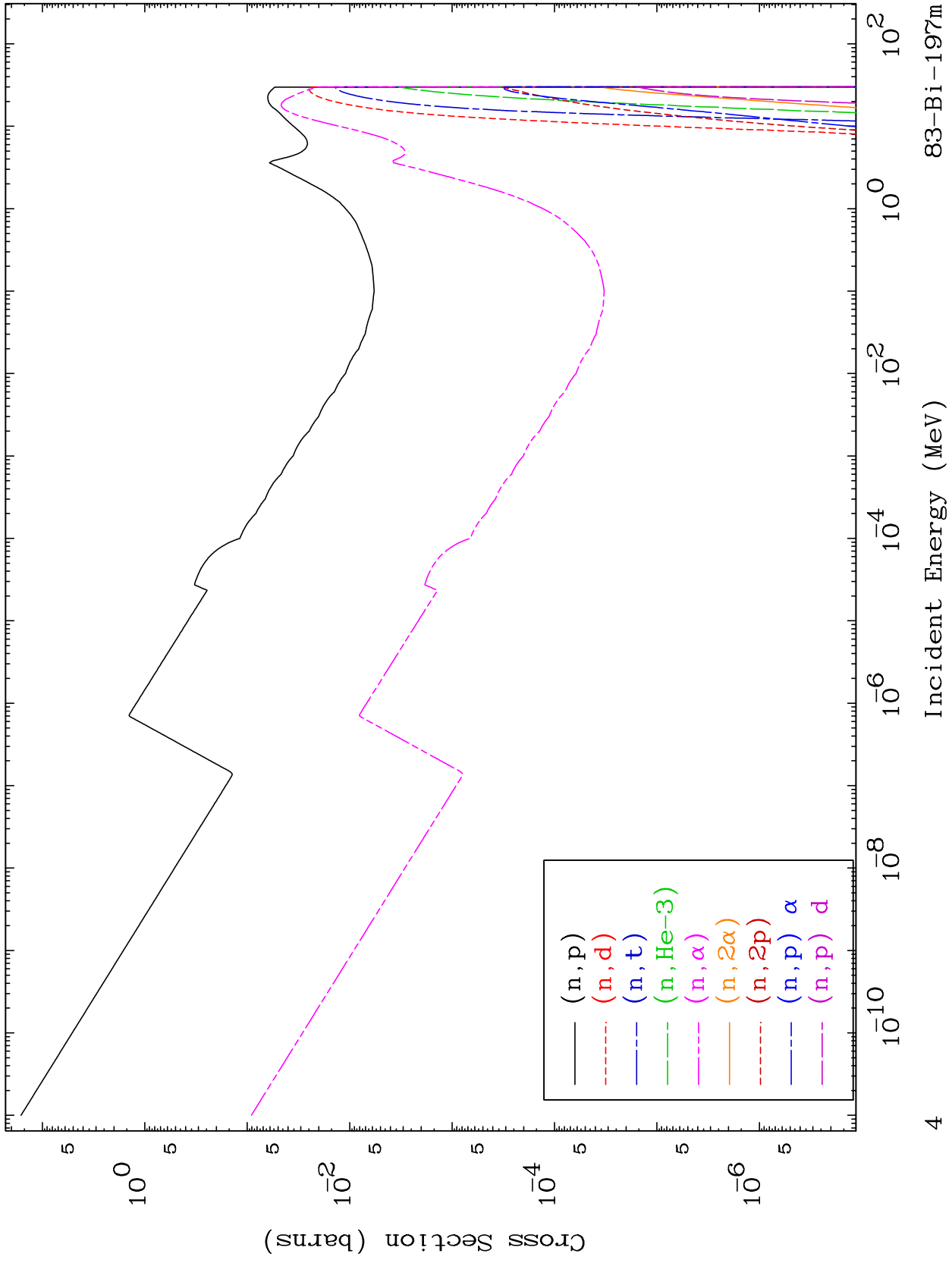
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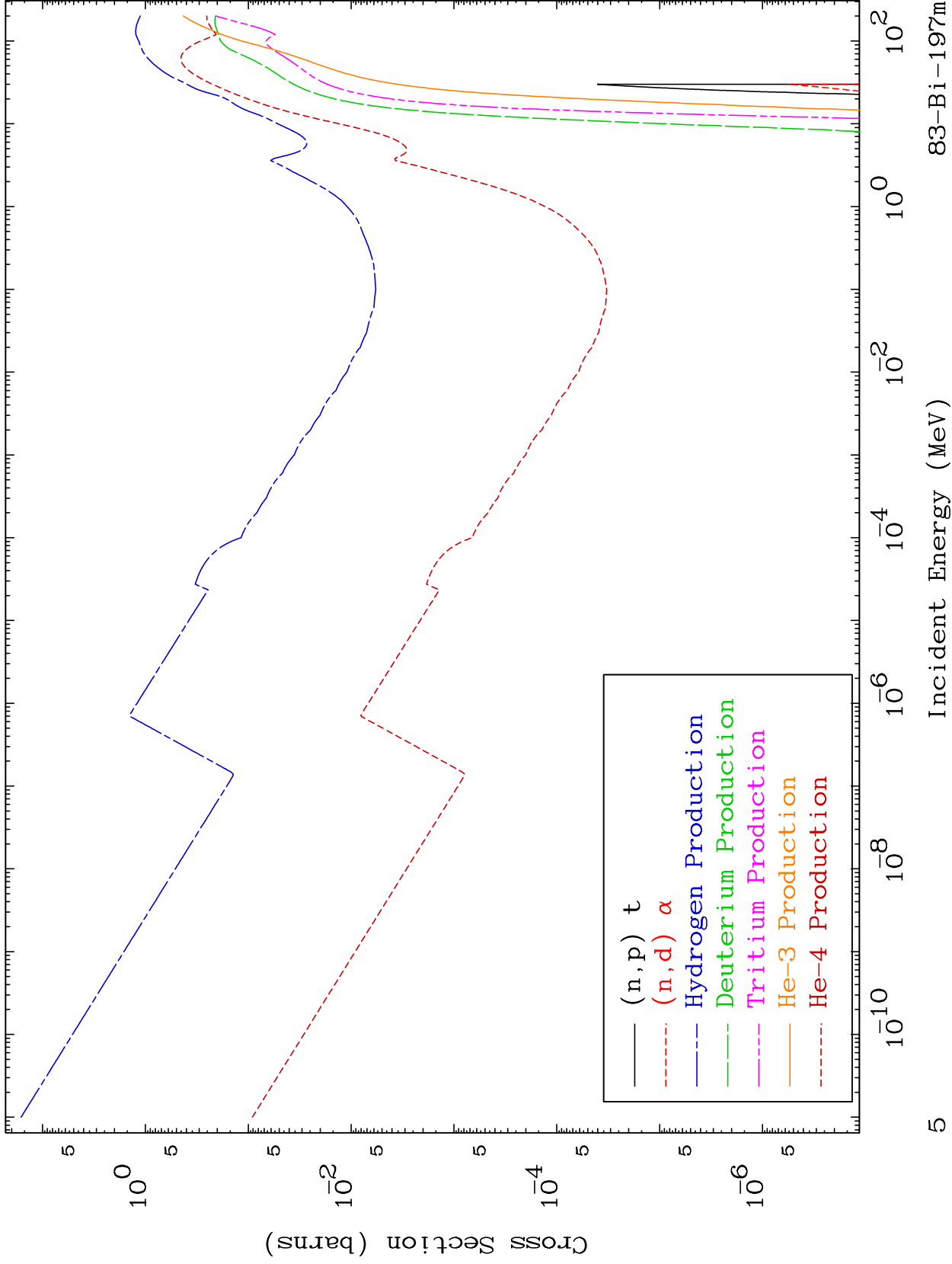


MAT 8290

Neutron Absorption
293 Kelvin Cross Sections

83-Bi-197m

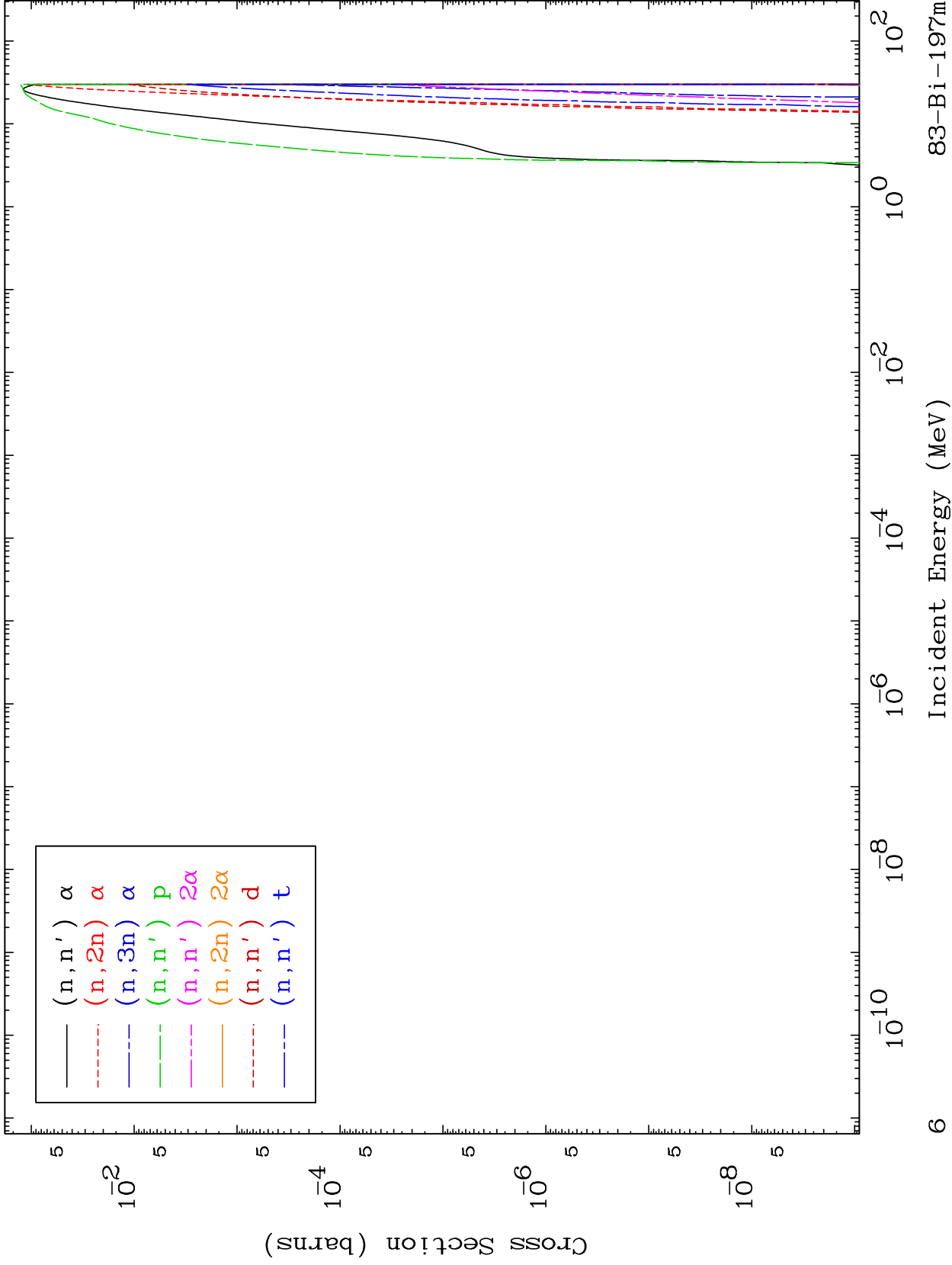




MAT 8290

Charged Particle
293 Kelvin Cross Sections

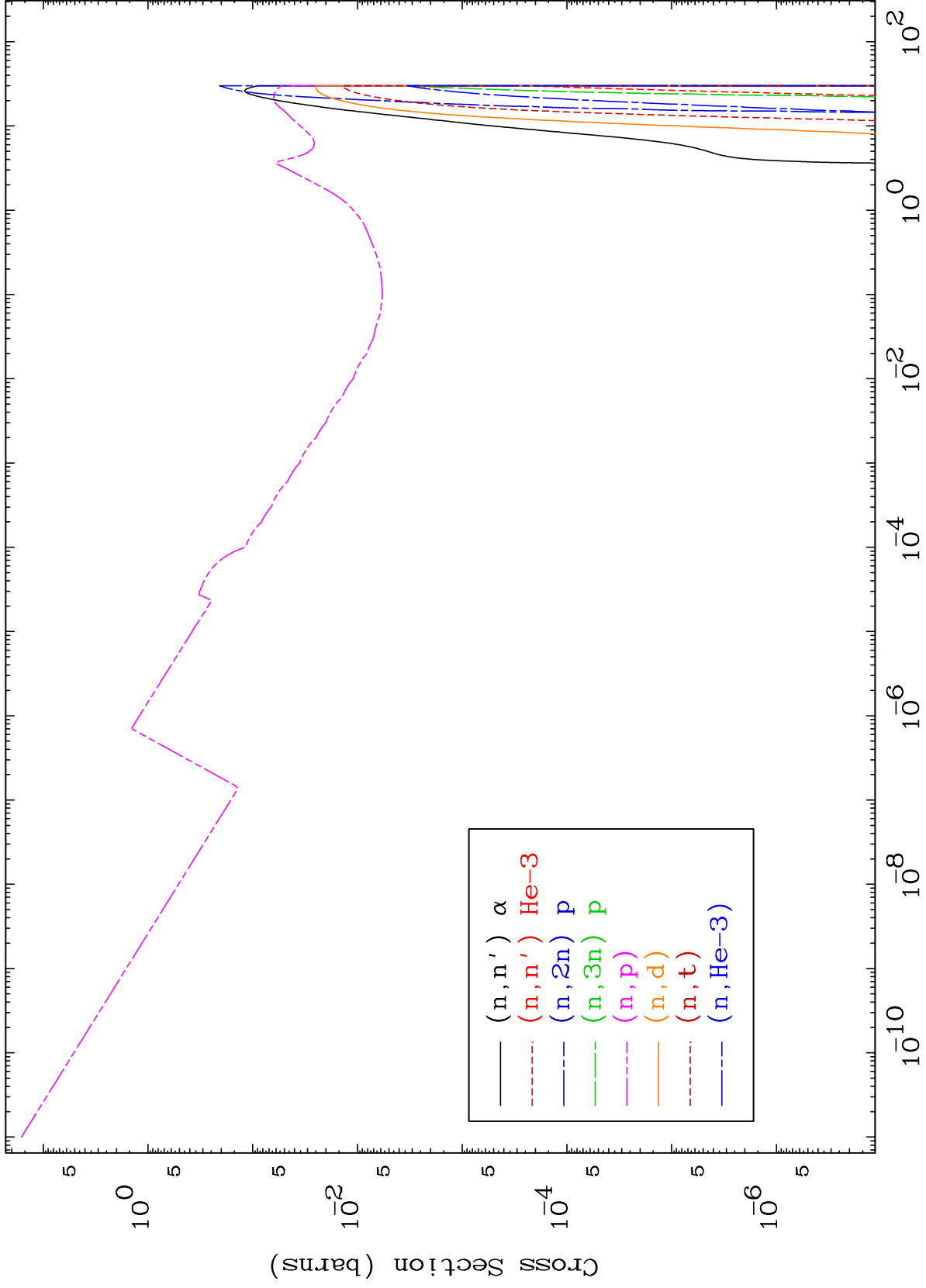
83-Bi-197m



MAT 8290

Charged Particle
293 Kelvin Cross Sections

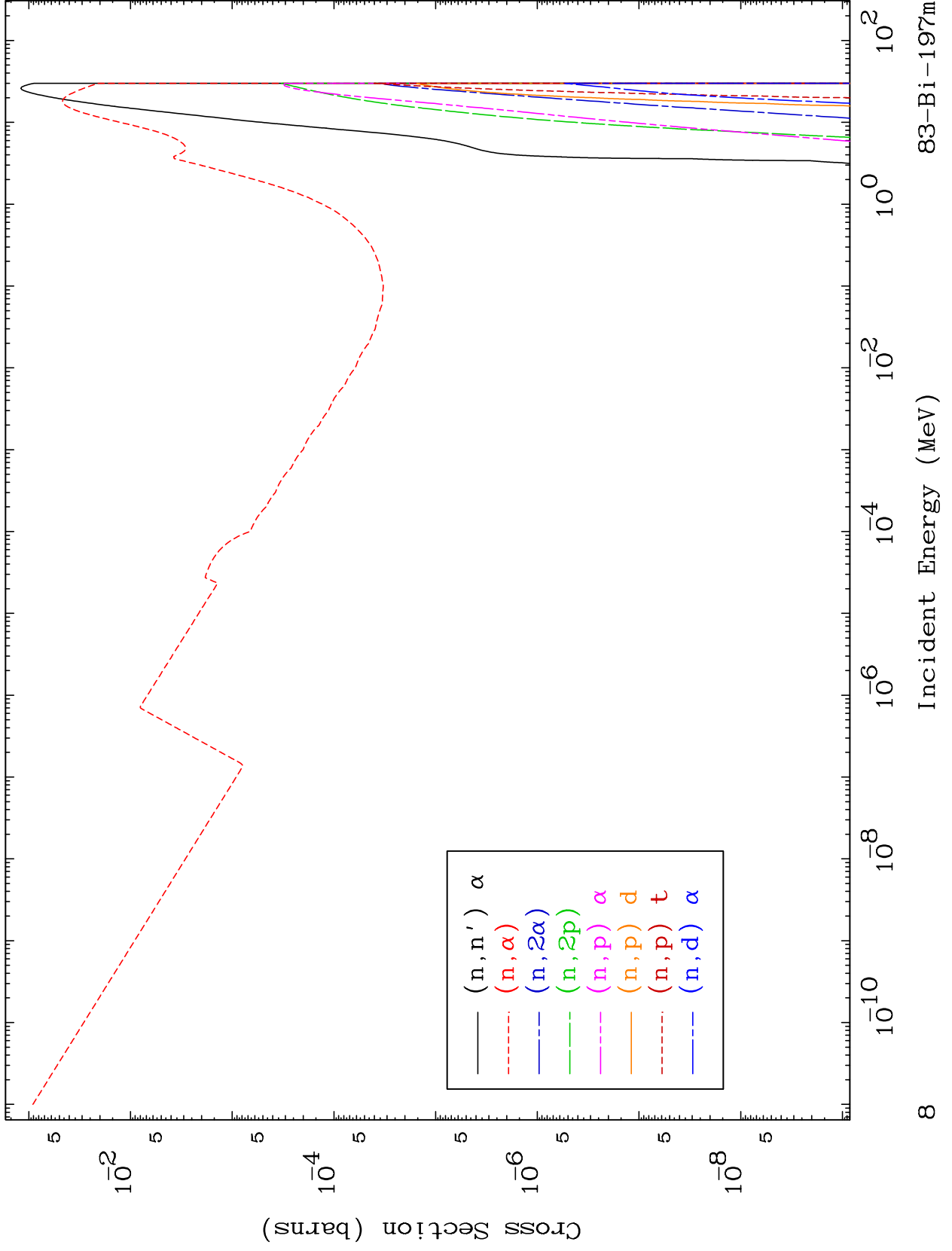
83-Bi-197m



MAT 8290

Charged Particle
293 Kelvin Cross Sections

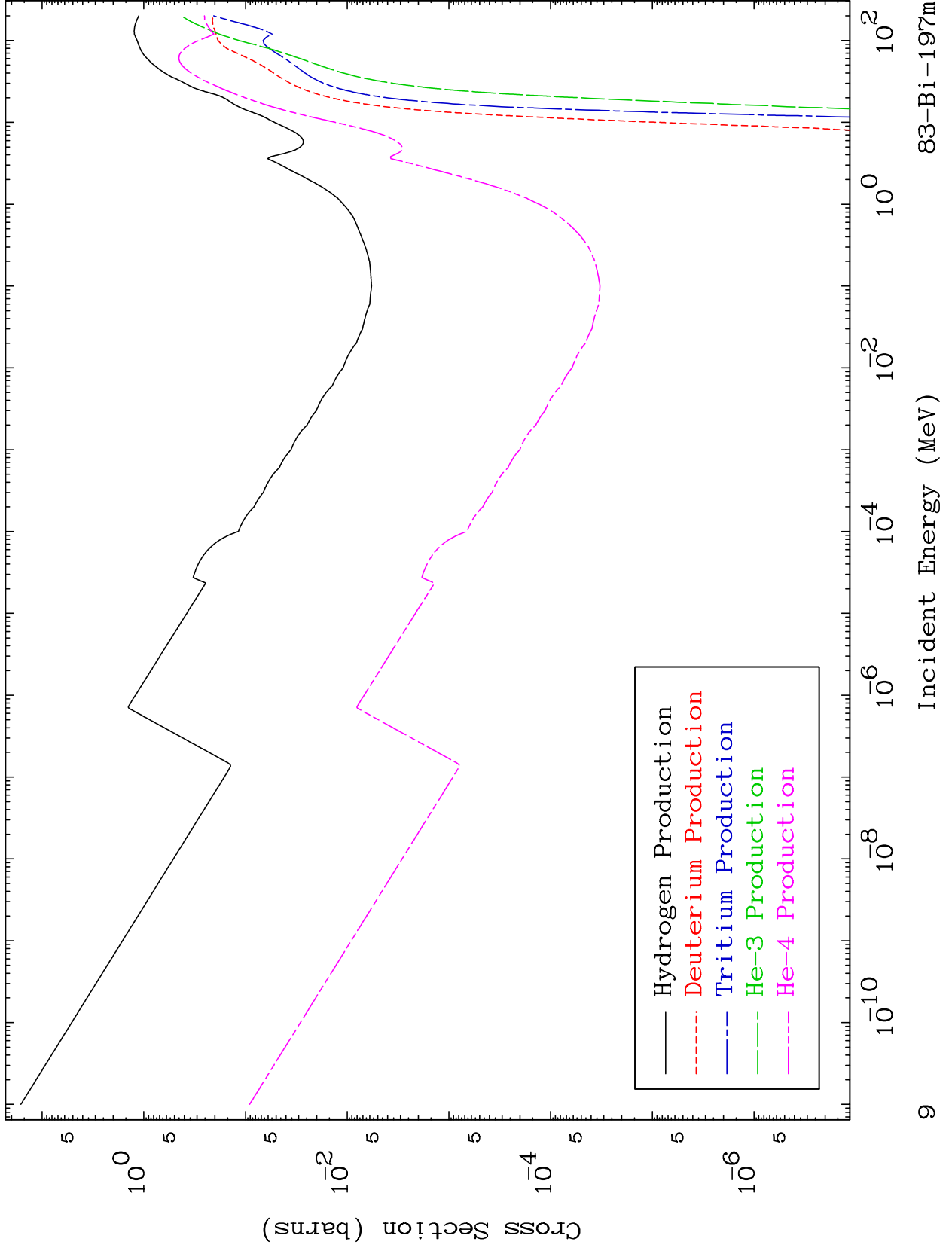
83-Bi-197m



MAT 8290

Particle Production
293 Kelvin Cross Sections

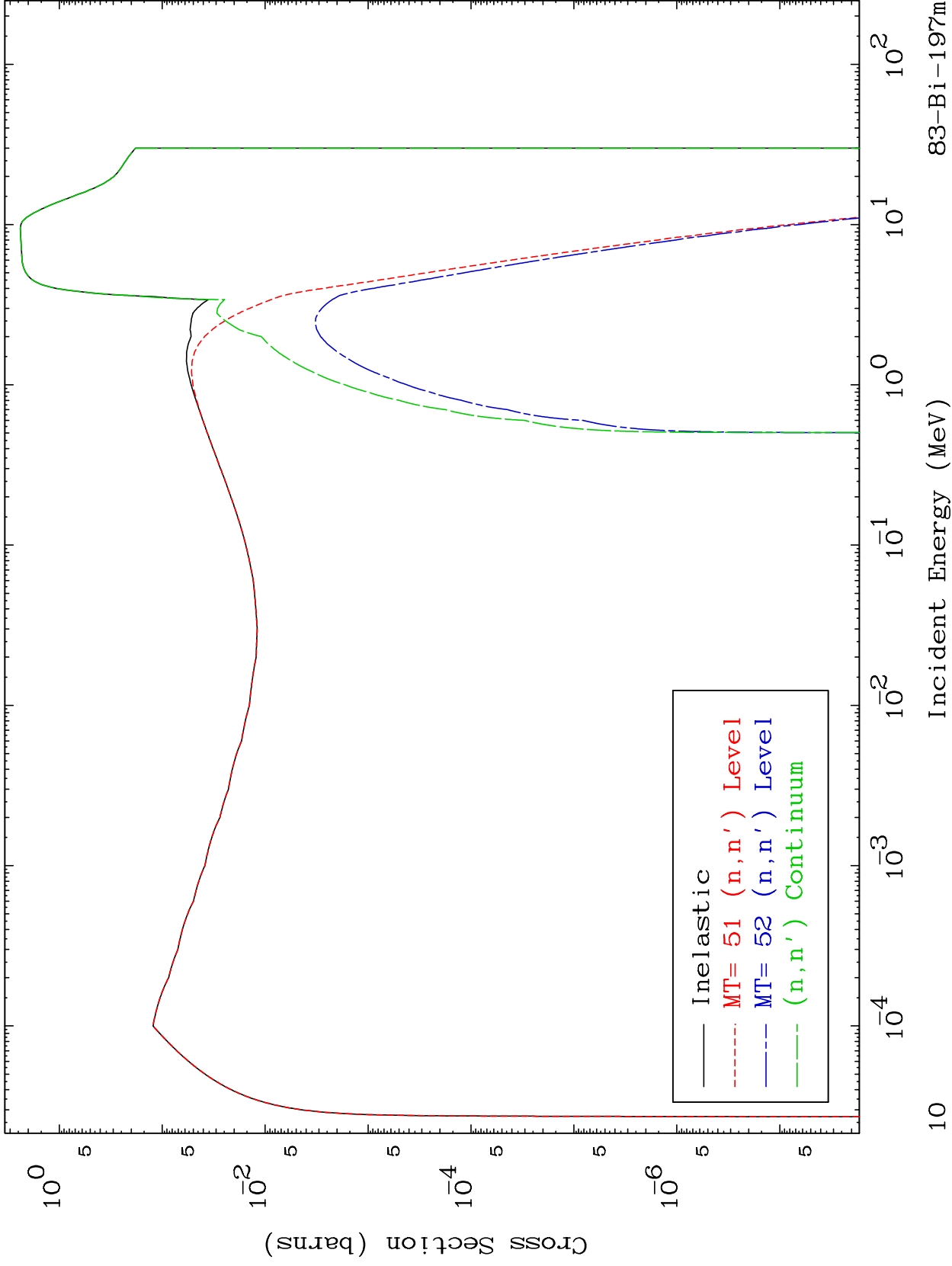
83-Bi-197m



MAT 8290

(n,n') Levels
293 Kelvin Cross Sections

83-Bi-197m



10

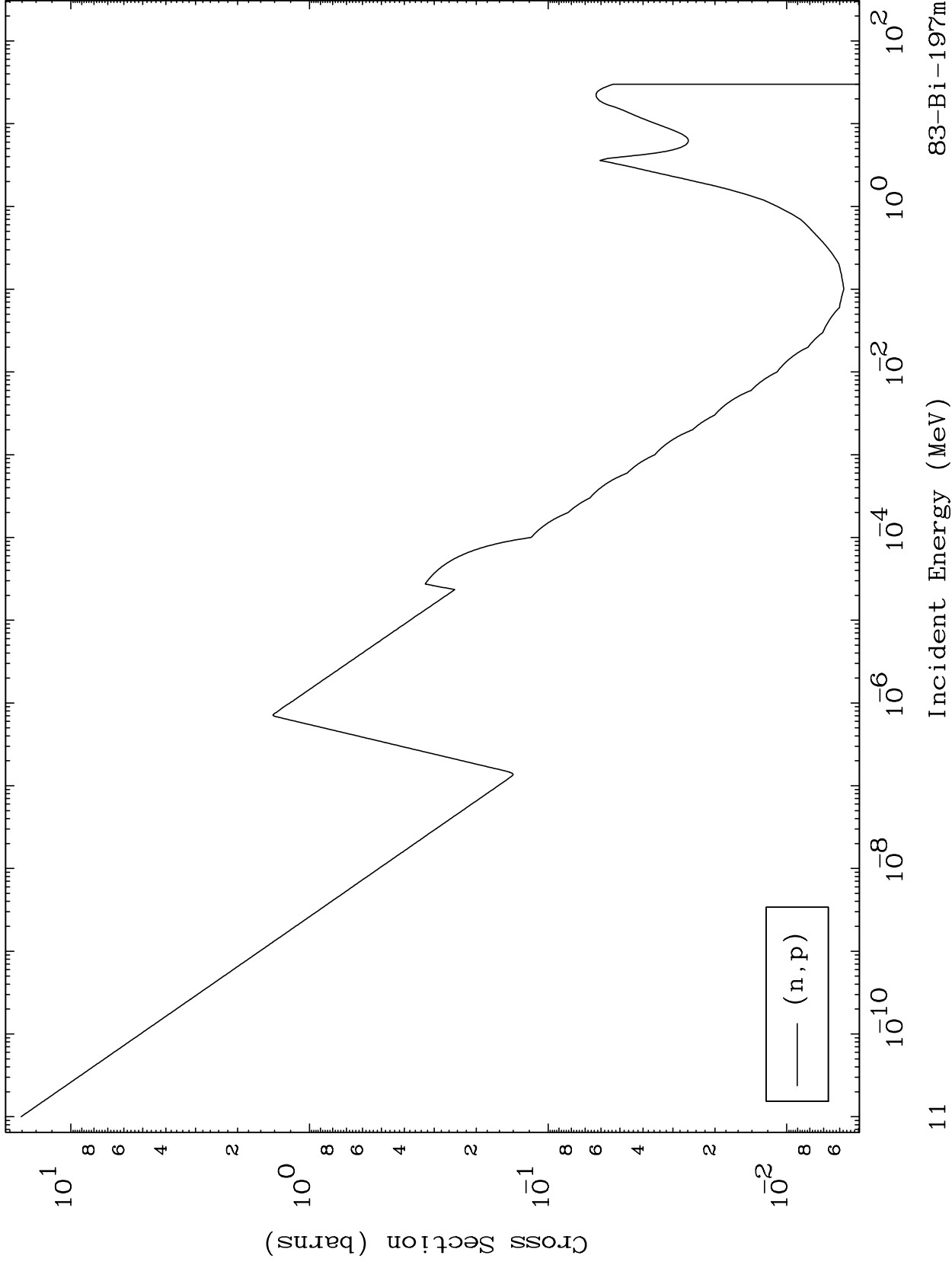
Incident Energy (MeV)

83-Bi-197m

MAT 8290

(n,p) Levels
293 Kelvin Cross Sections

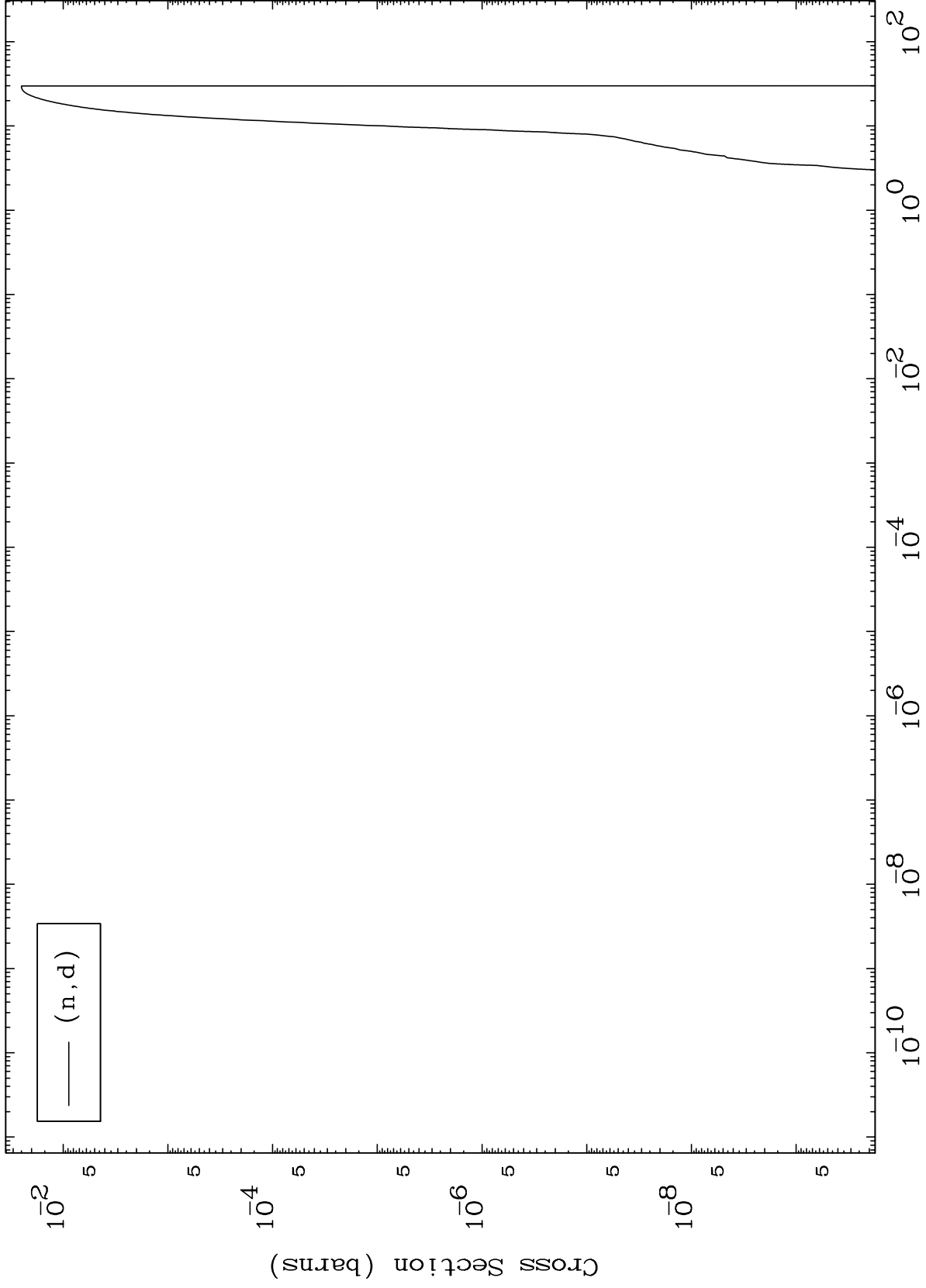
83-Bi-197m



MAT 8290

(n,d) Levels
293 Kelvin Cross Sections

83-Bi-197m



12

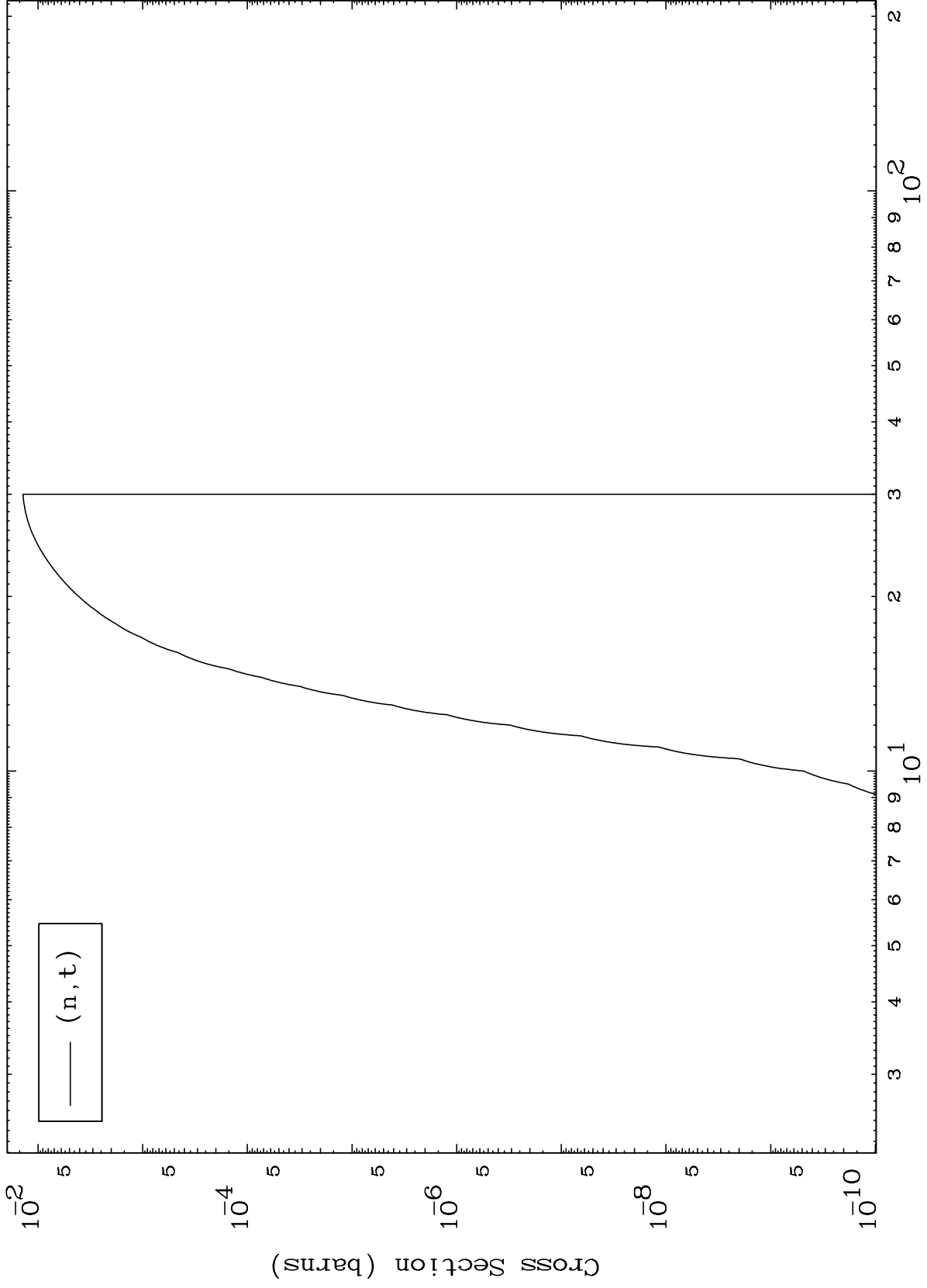
Incident Energy (MeV)

83-Bi-197m

MAT 8290

(n,t) Levels
293 Kelvin Cross Sections

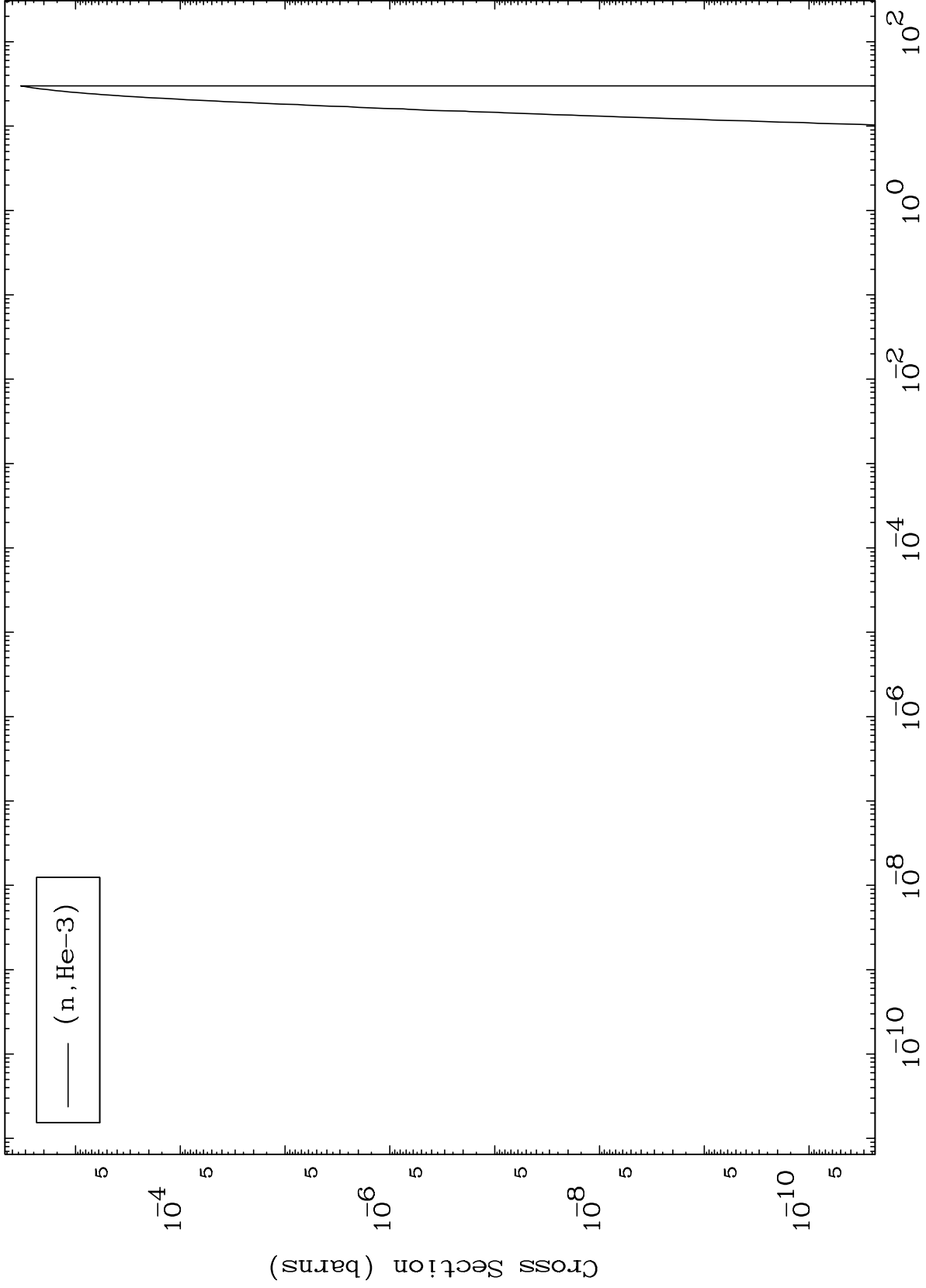
83-Bi-197m



MAT 8290

(n,He3) Levels
293 Kelvin Cross Sections

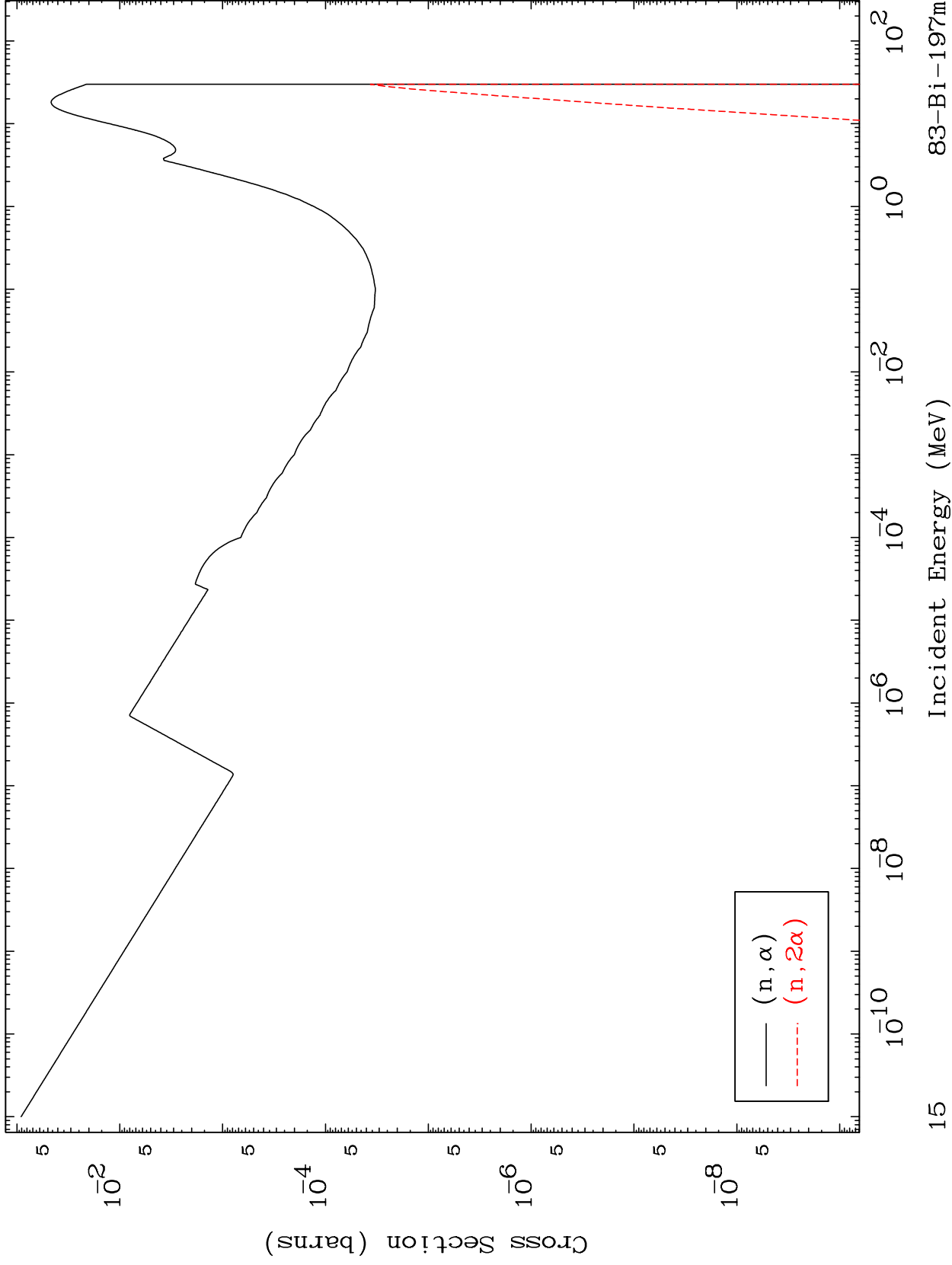
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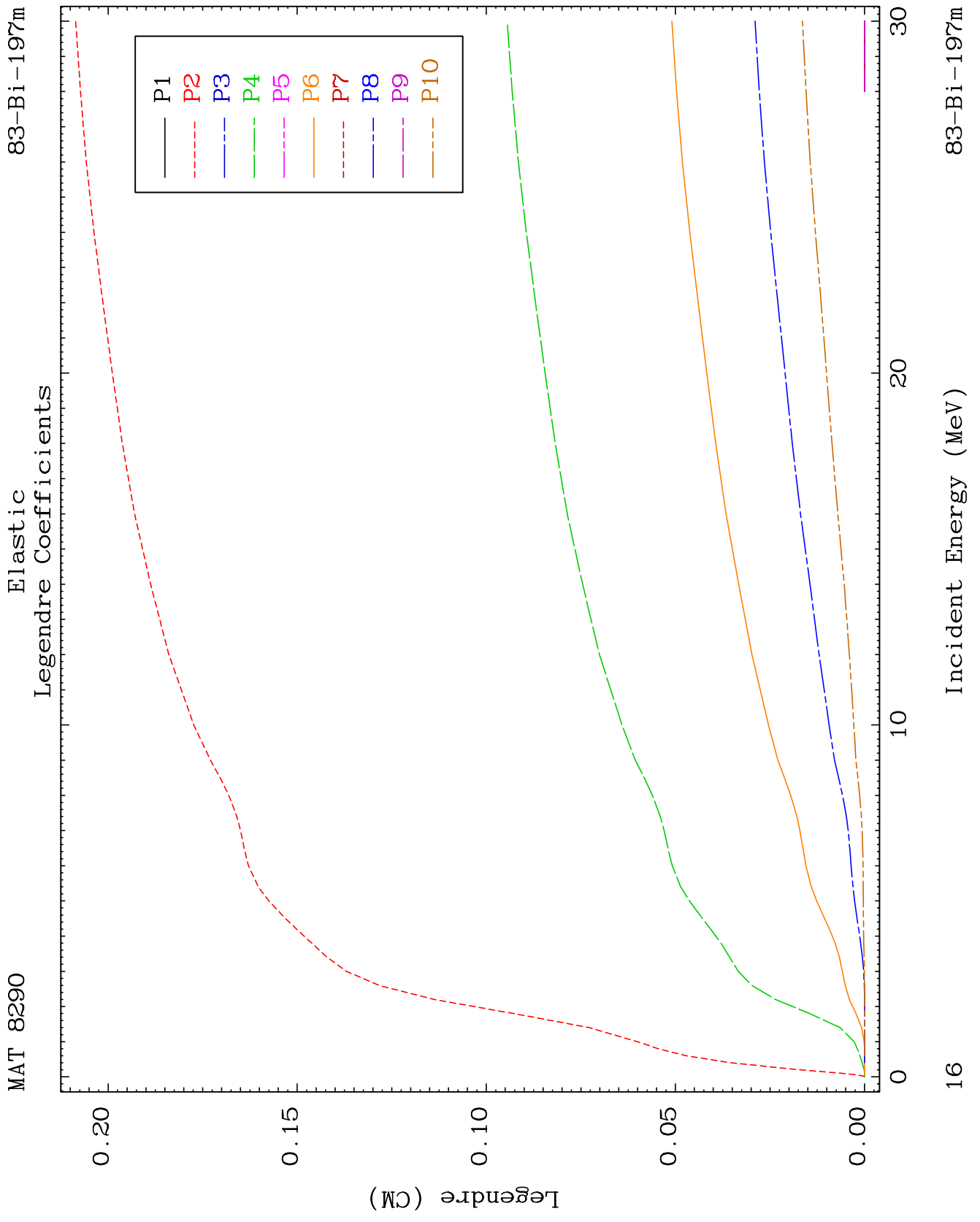


MAT 8290

(n,α) Levels
293 Kelvin Cross Sections

83-Bi-197m

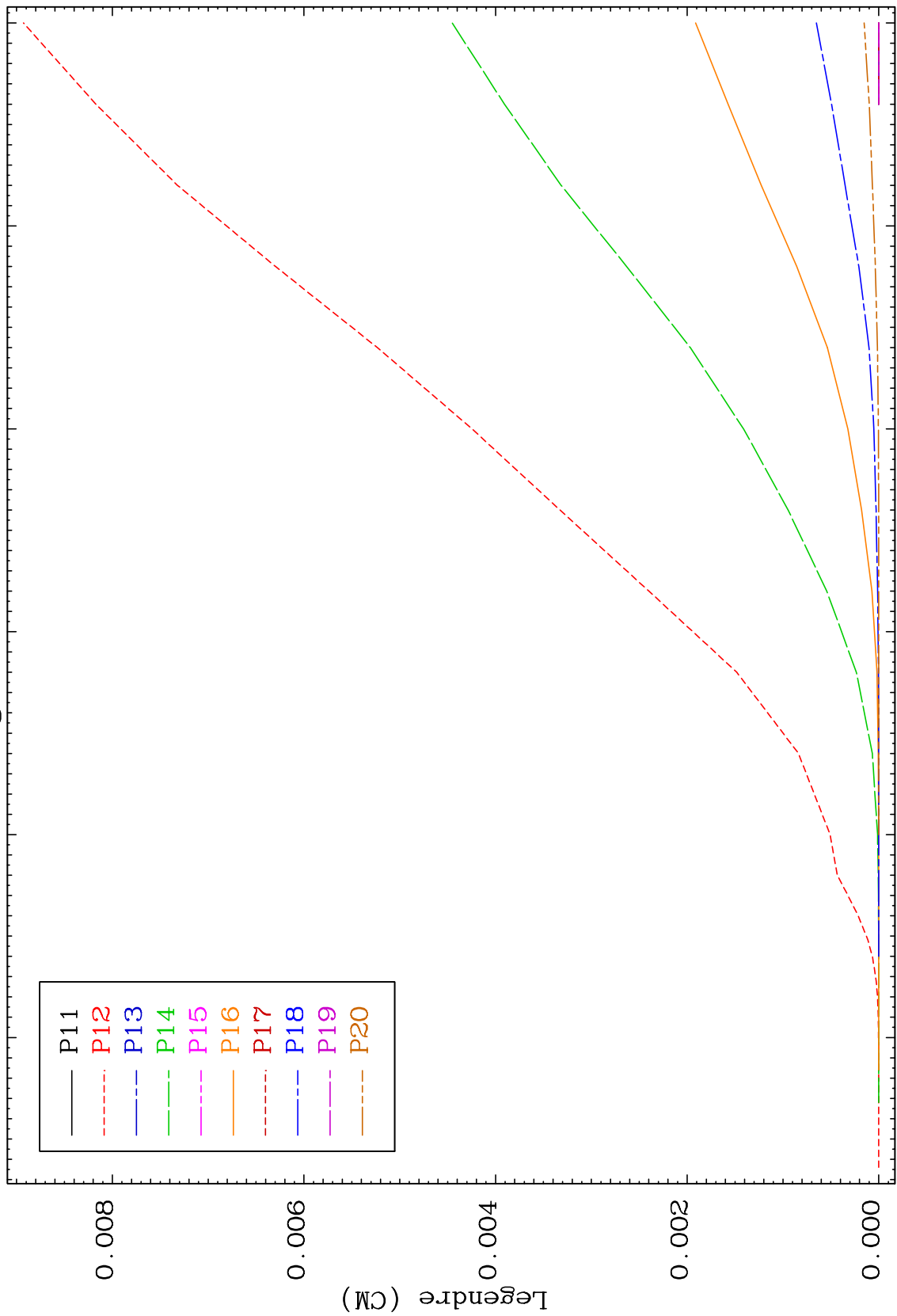




MAT 8290

Elastic Legendre Coefficients

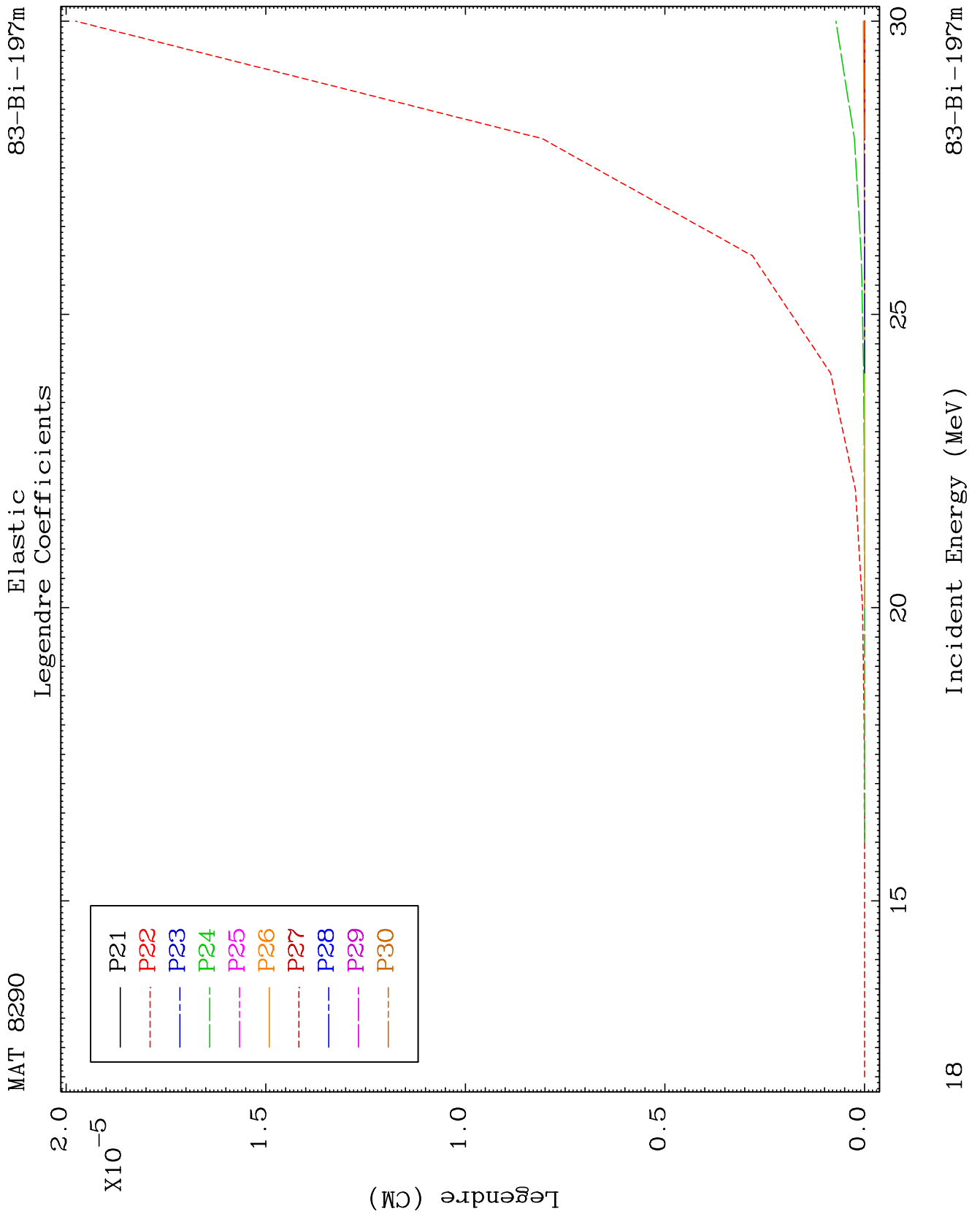
83-Bi-197m



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83-Bi-197m

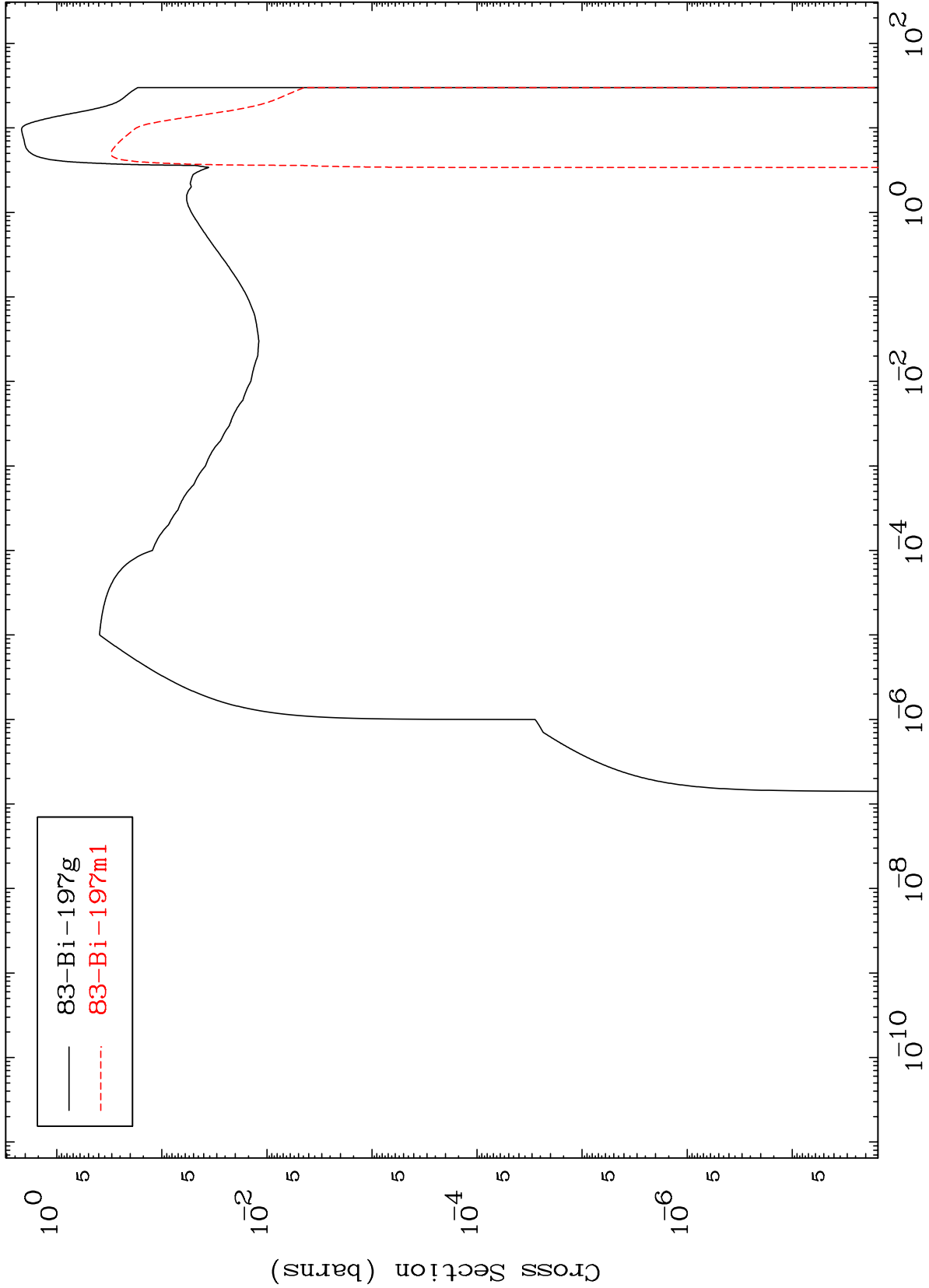
Incident Energy (MeV)



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Inelastic
Radionuclide Production Cross Section

83-Bi-197m

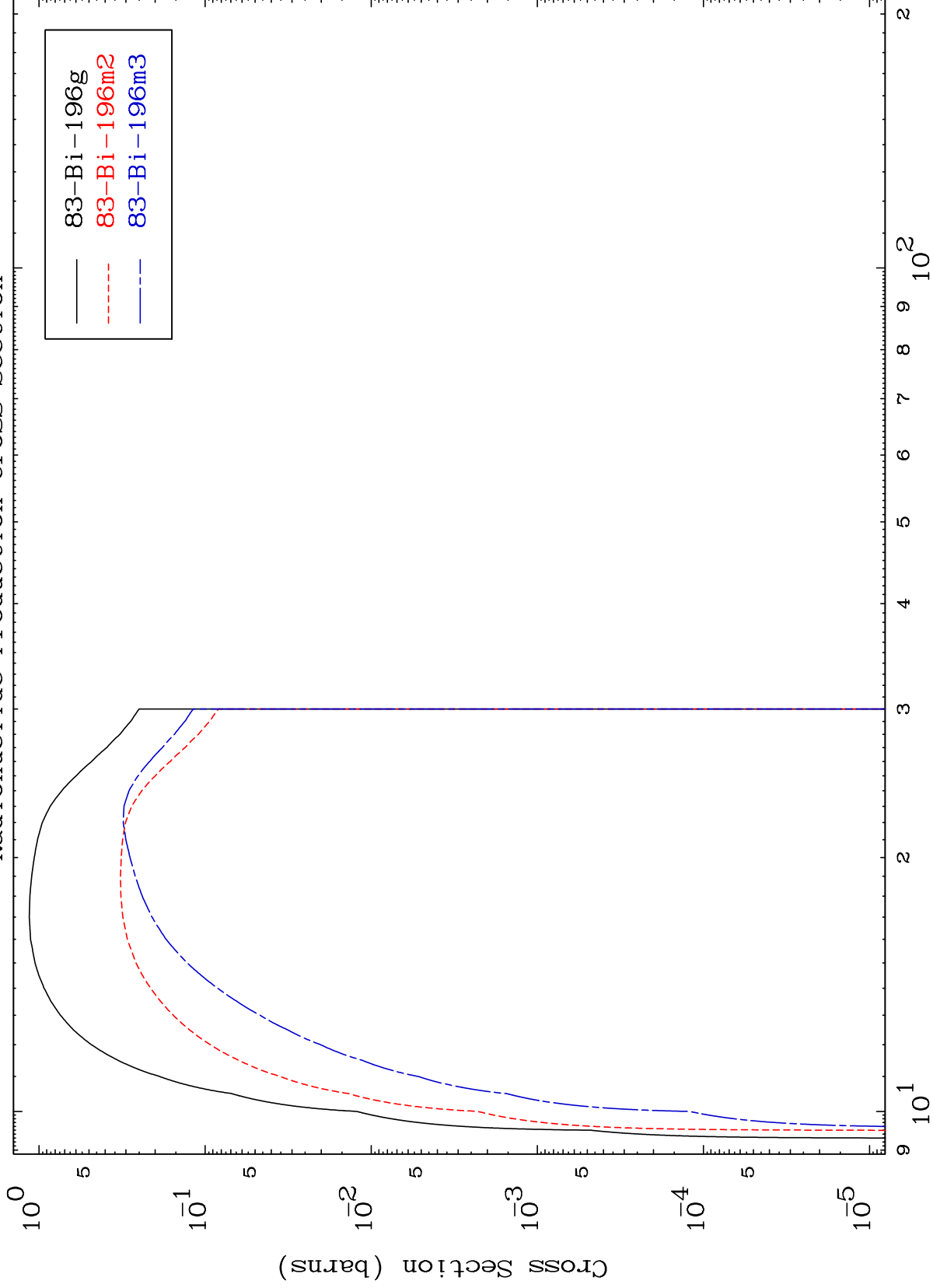


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(n,2n)

83-Bi-197m

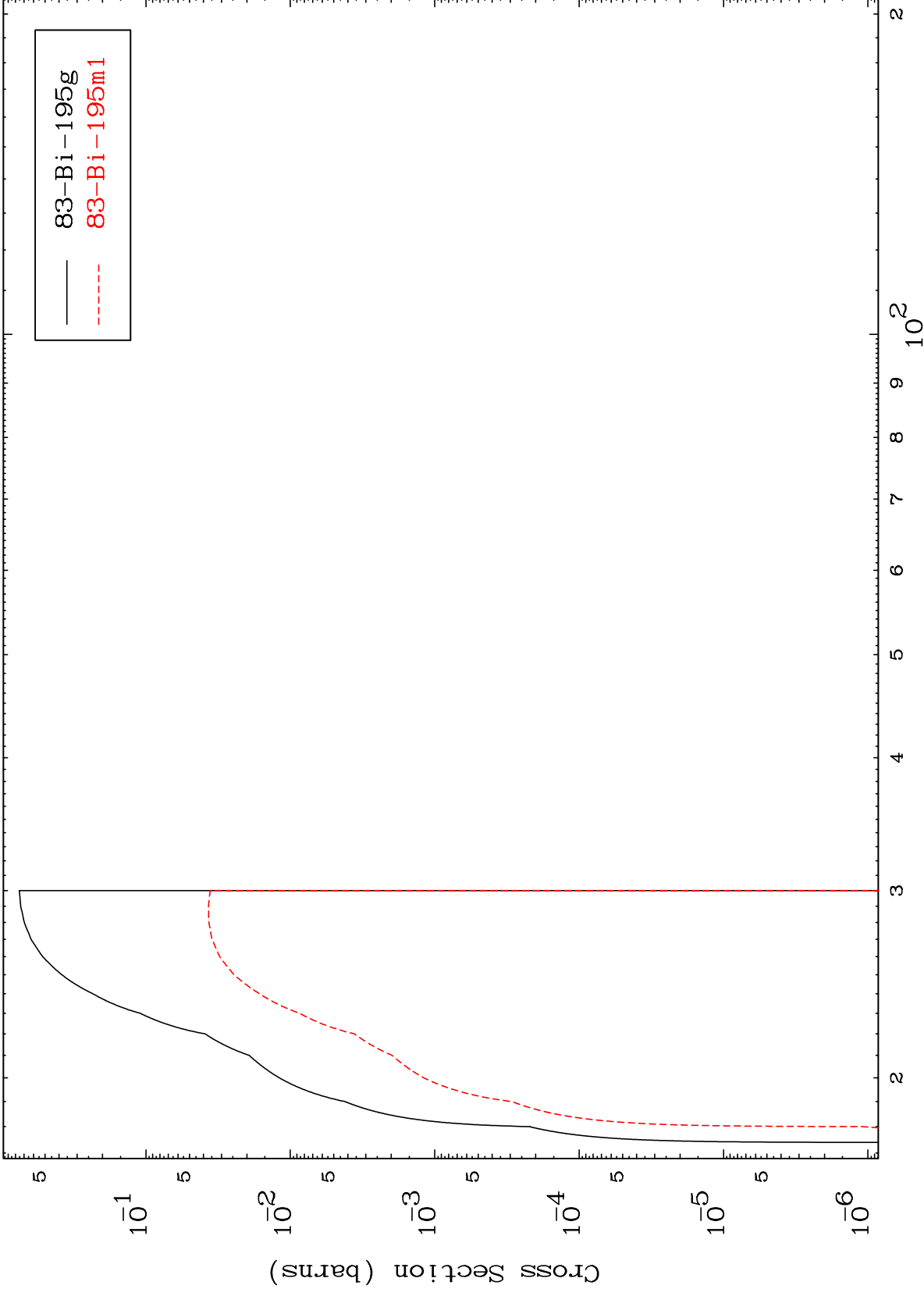
Radionuclide Production Cross Section



Incident Energy (MeV)

83-Bi-197m

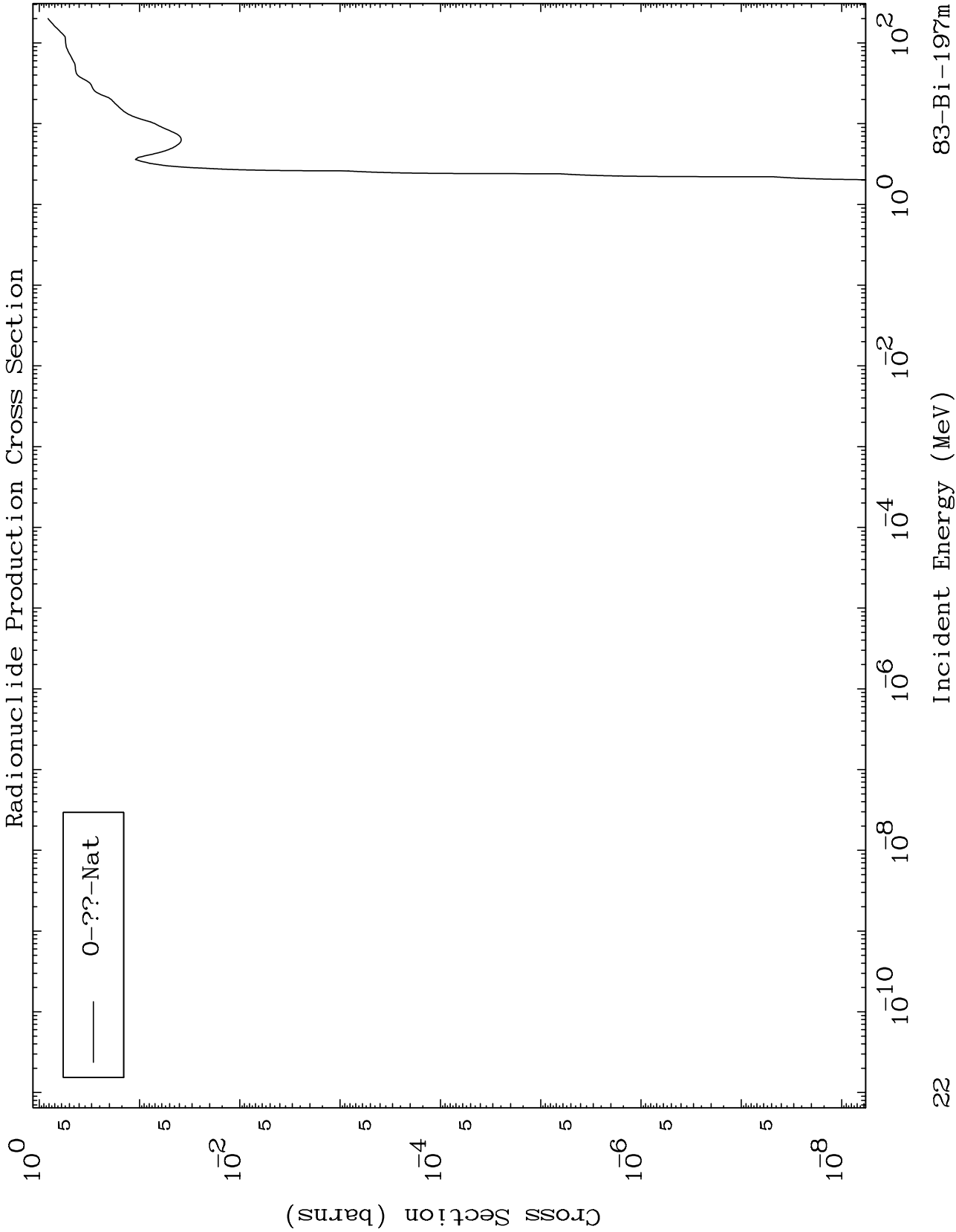
Radionuclide Production Cross Section



MAT 8290

Fission

83-Bi-197m

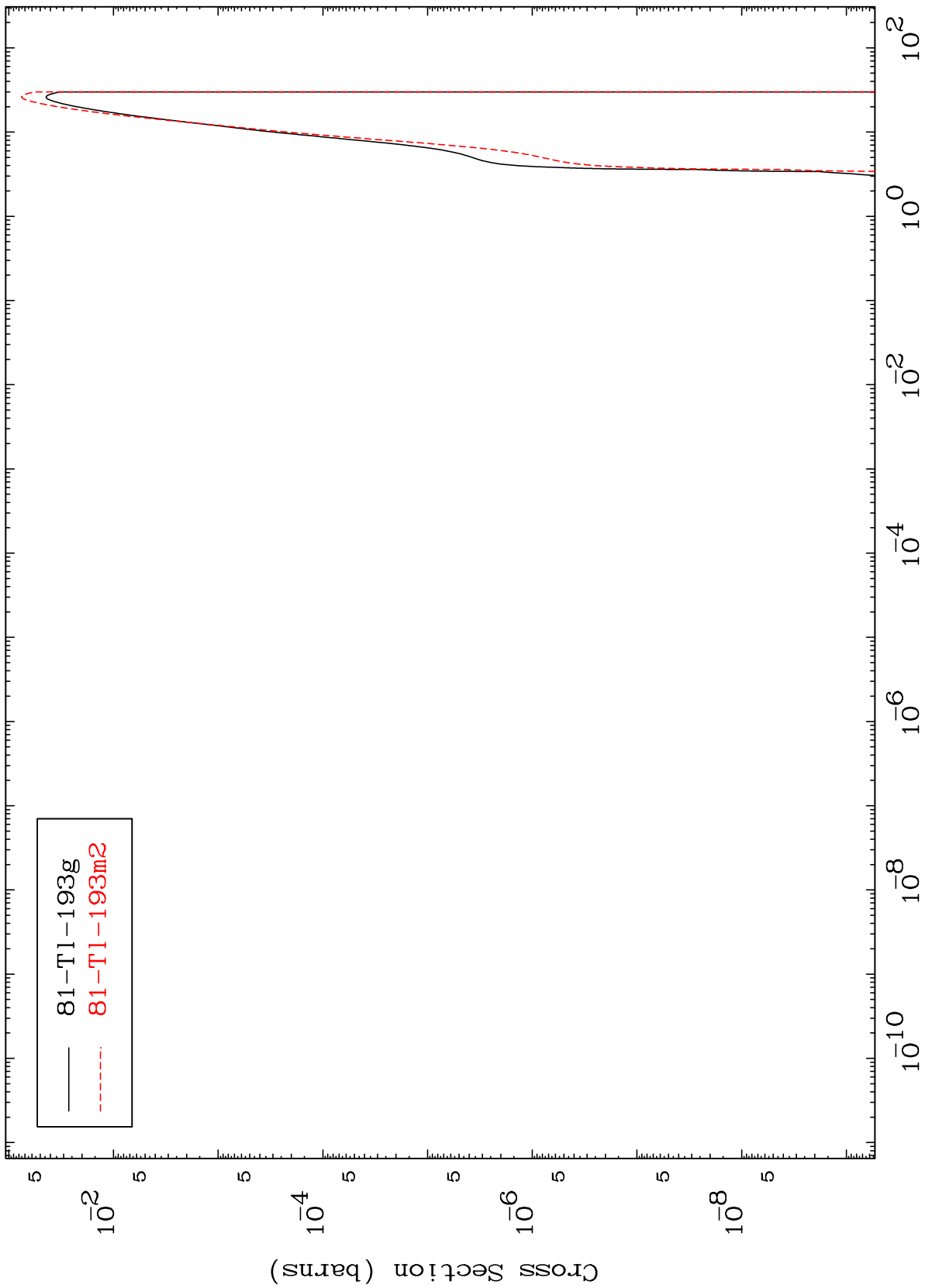


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$(n, n') \alpha$

83-Bi-197m

Radionuclide Production Cross Section

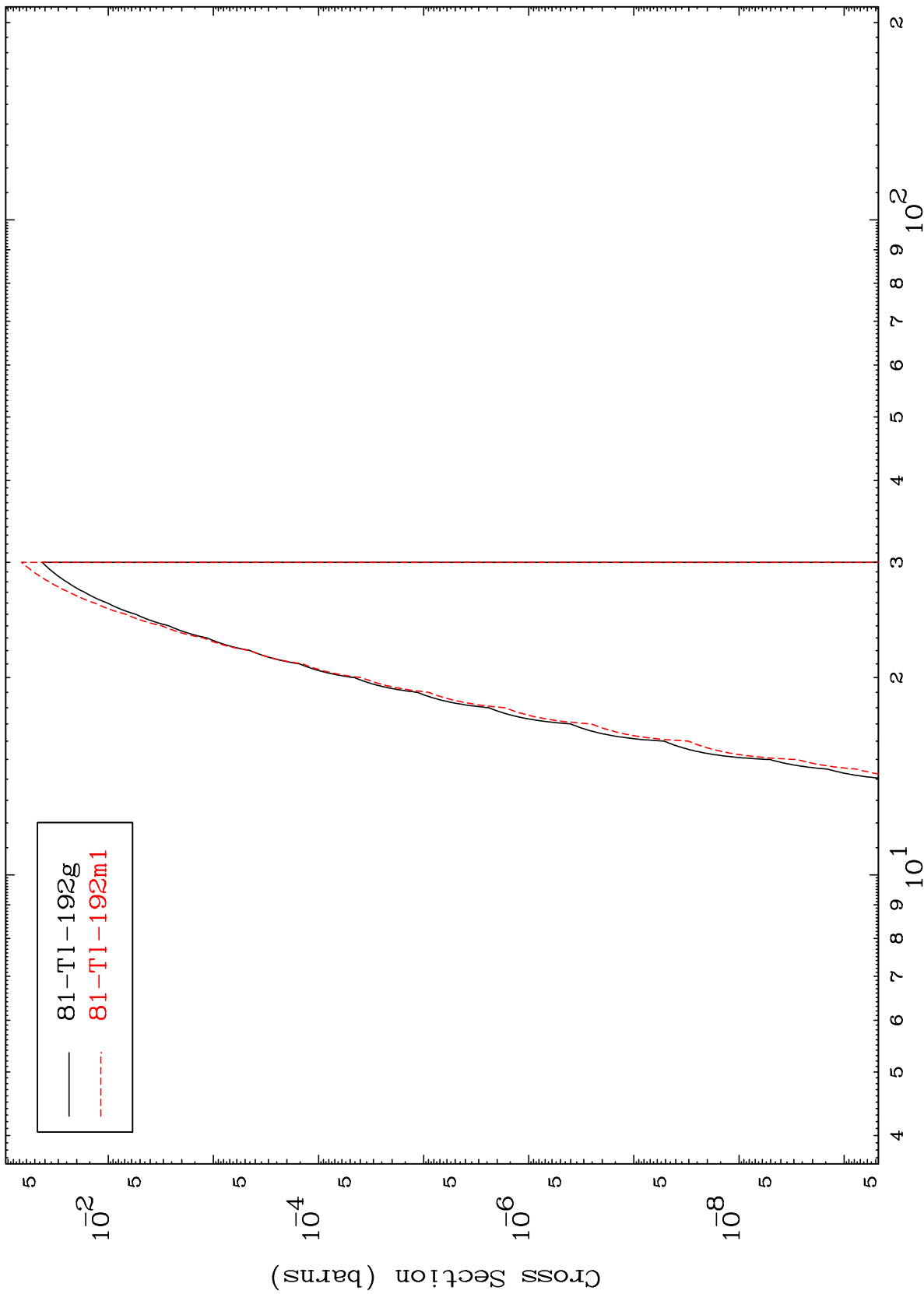


MAT 8290

$(n,2n) \alpha$

$^{83}\text{Bi}-197\text{m}$

Radionuclide Production Cross Section



81-Tl-192g
81-Tl-192m1

24

Incident Energy (MeV)

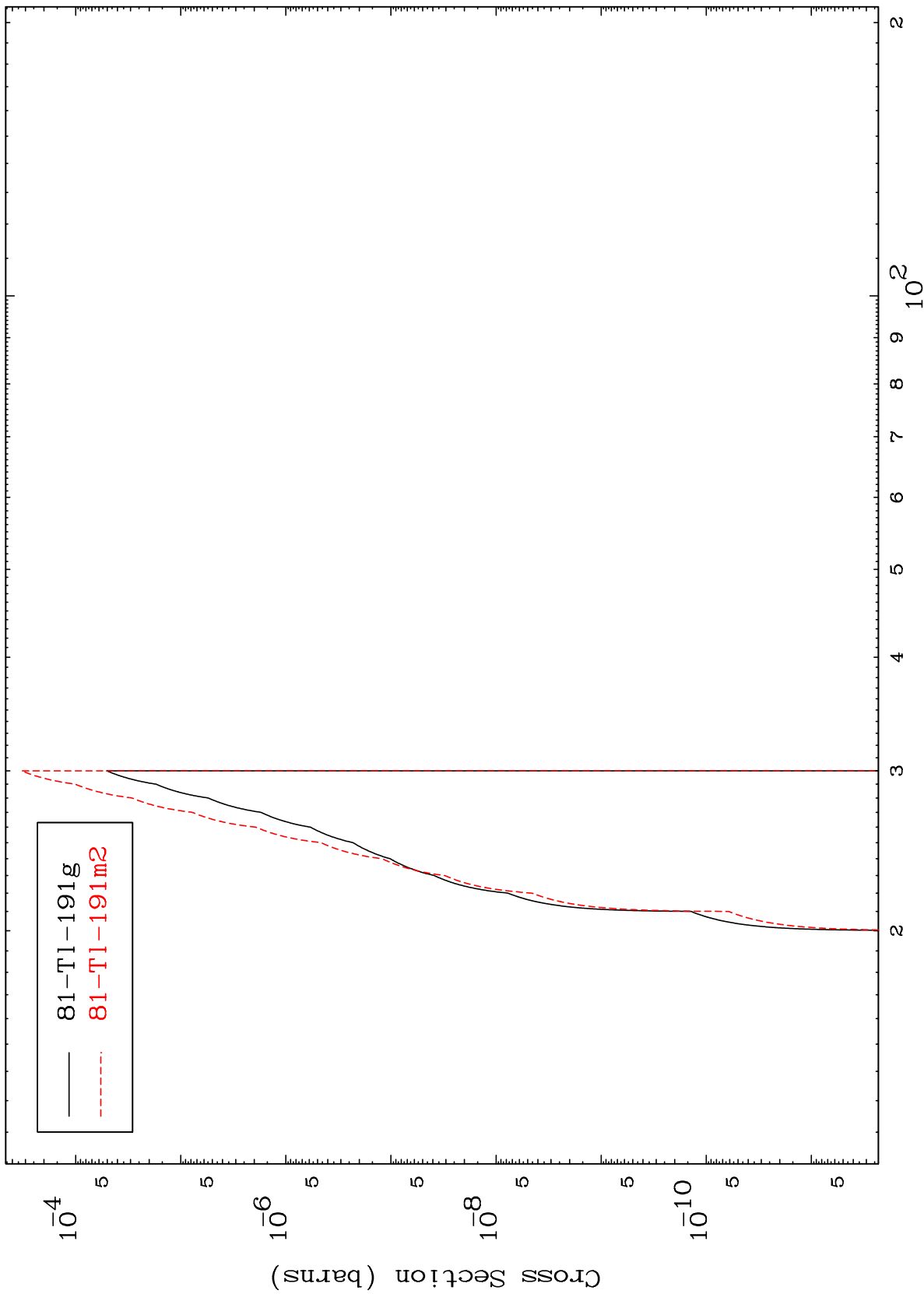
$^{83}\text{Bi}-197\text{m}$

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(n,3n) α

83-Bi-197m

Radionuclide Production Cross Section

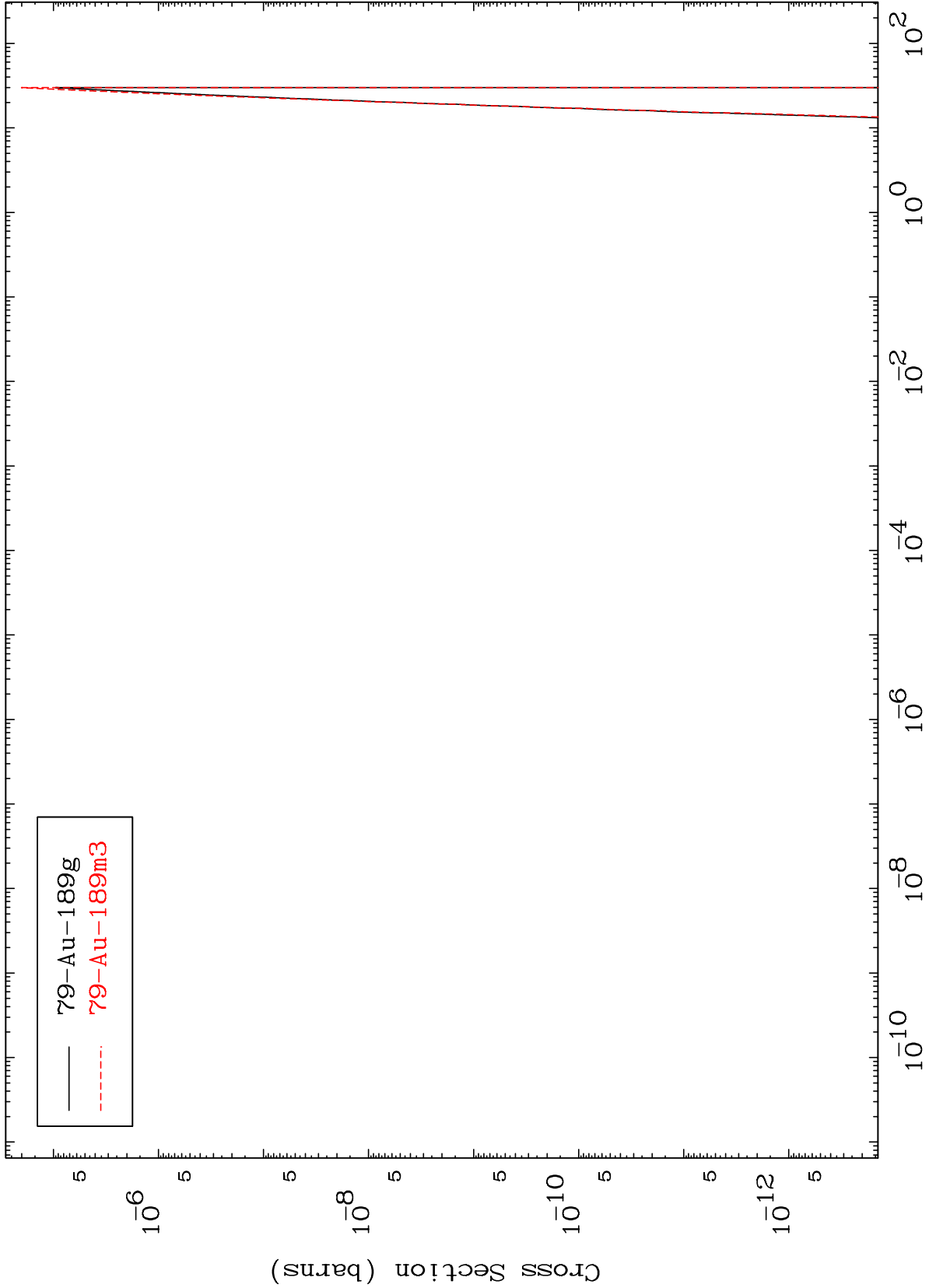


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(n,n') 2 α

83-Bi-197m

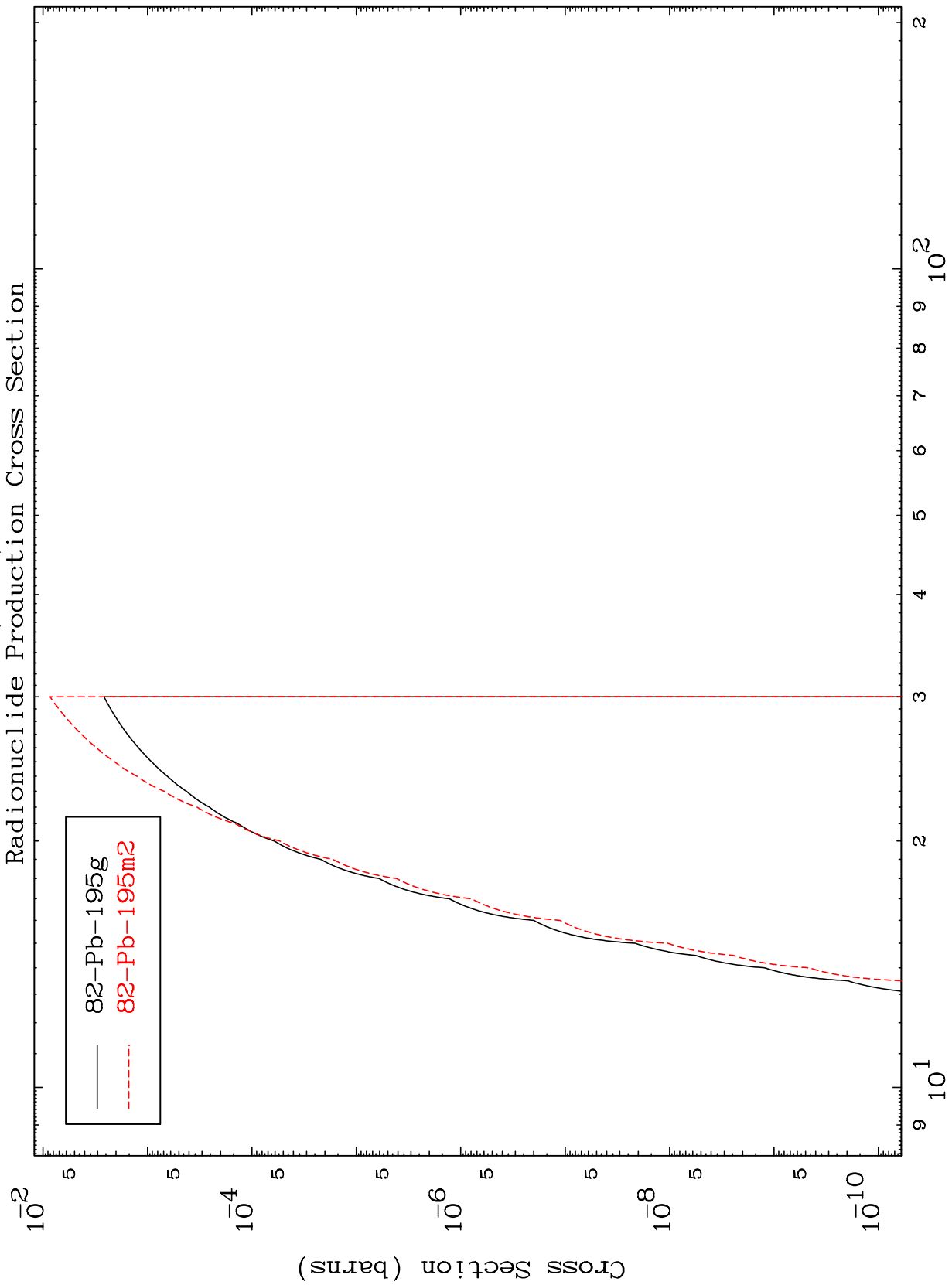
Radionuclide Production Cross Section



MAT 8290

(n,n') d

83-Bi-197m



27

Incident Energy (MeV)

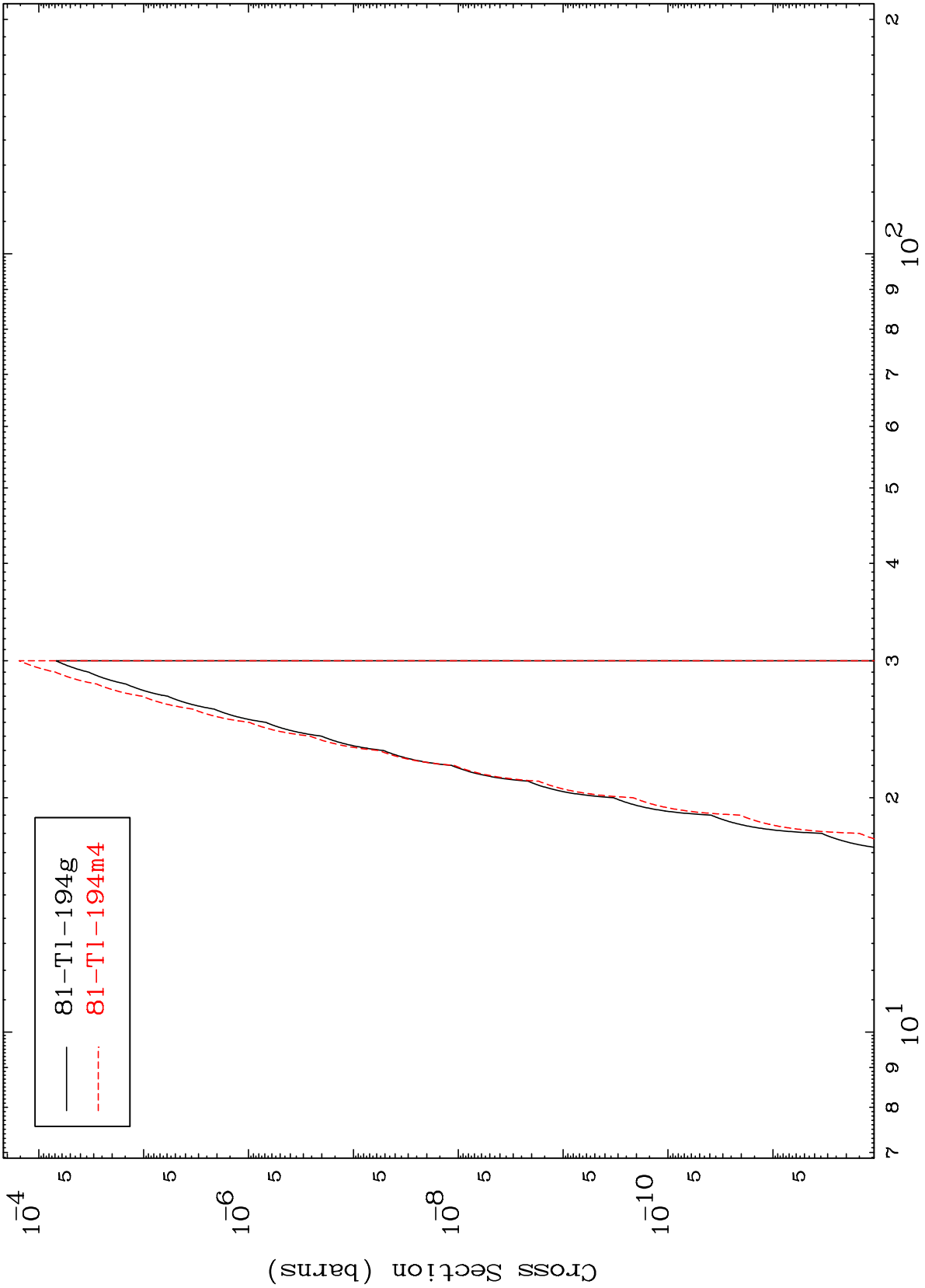
83-Bi-197m

MAT 8290

(n,n') He-3

83-Bi-197m

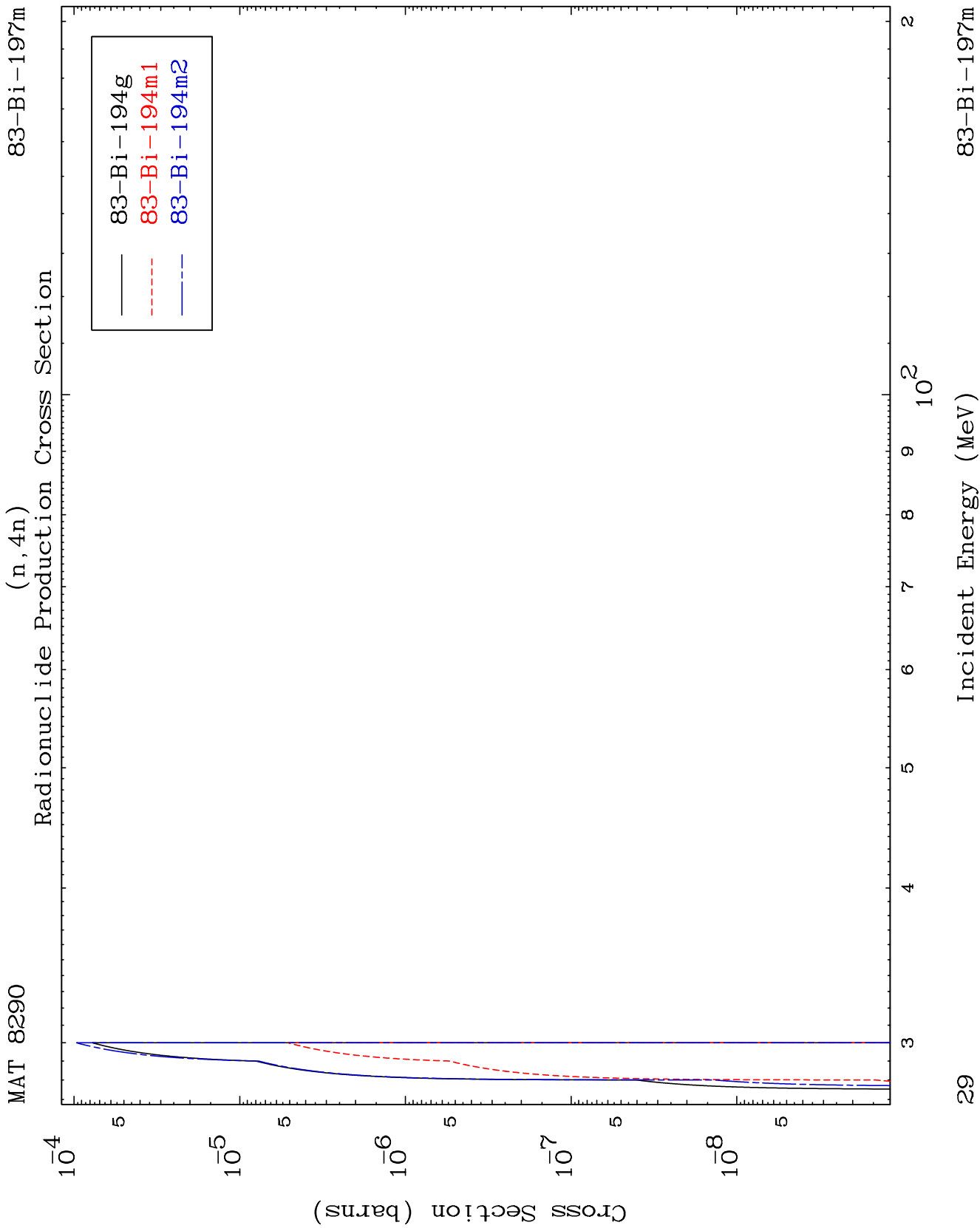
Radionuclide Production Cross Section



28

Incident Energy (MeV)

83-Bi-197m

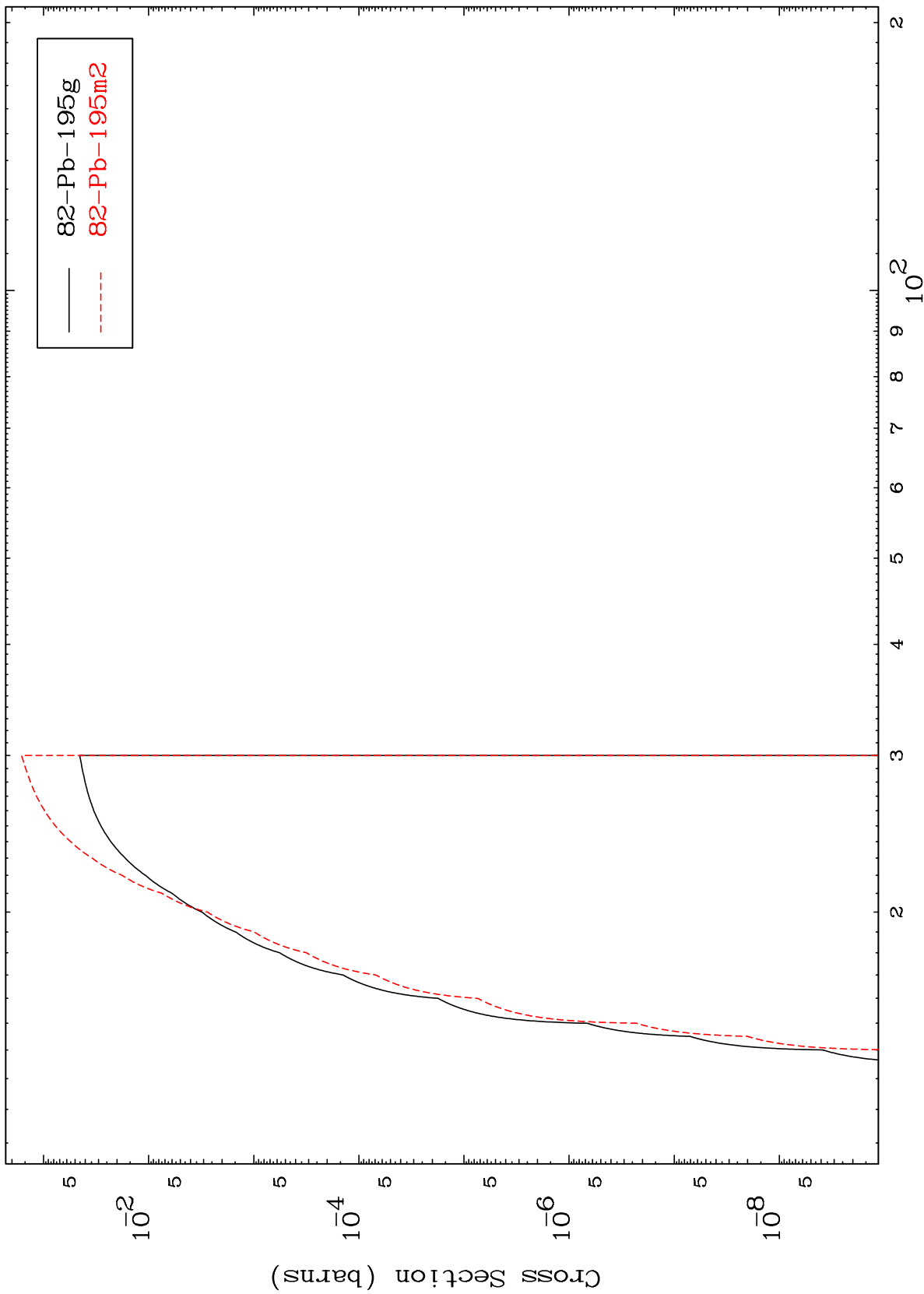


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(n,2n) p

83-Bi-197m

Radionuclide Production Cross Section



30

Incident Energy (MeV)

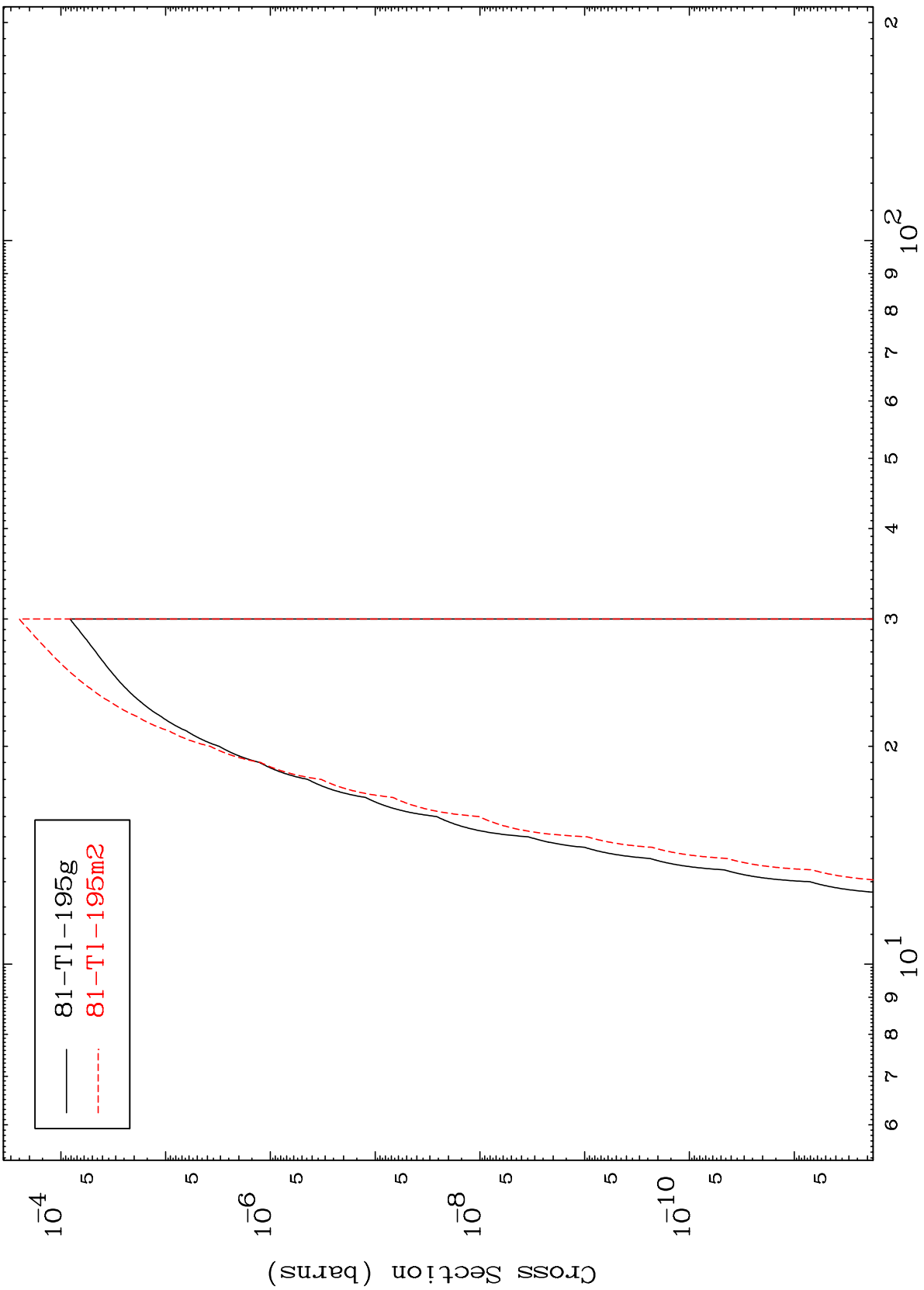
83-Bi-197m

MAT 8290

(n,2n) p

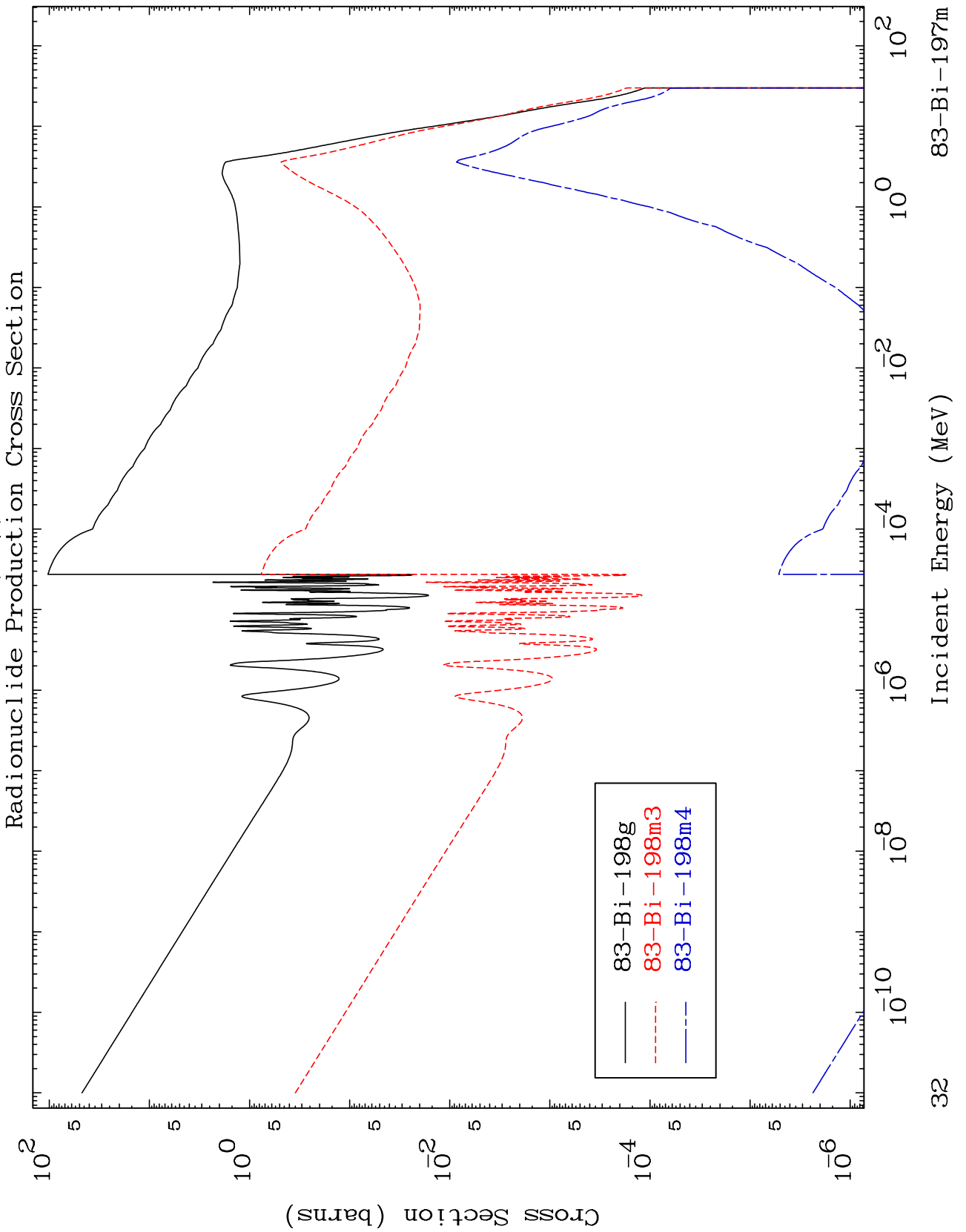
83-Bi-197m

Radionuclide Production Cross Section



MAT 8290

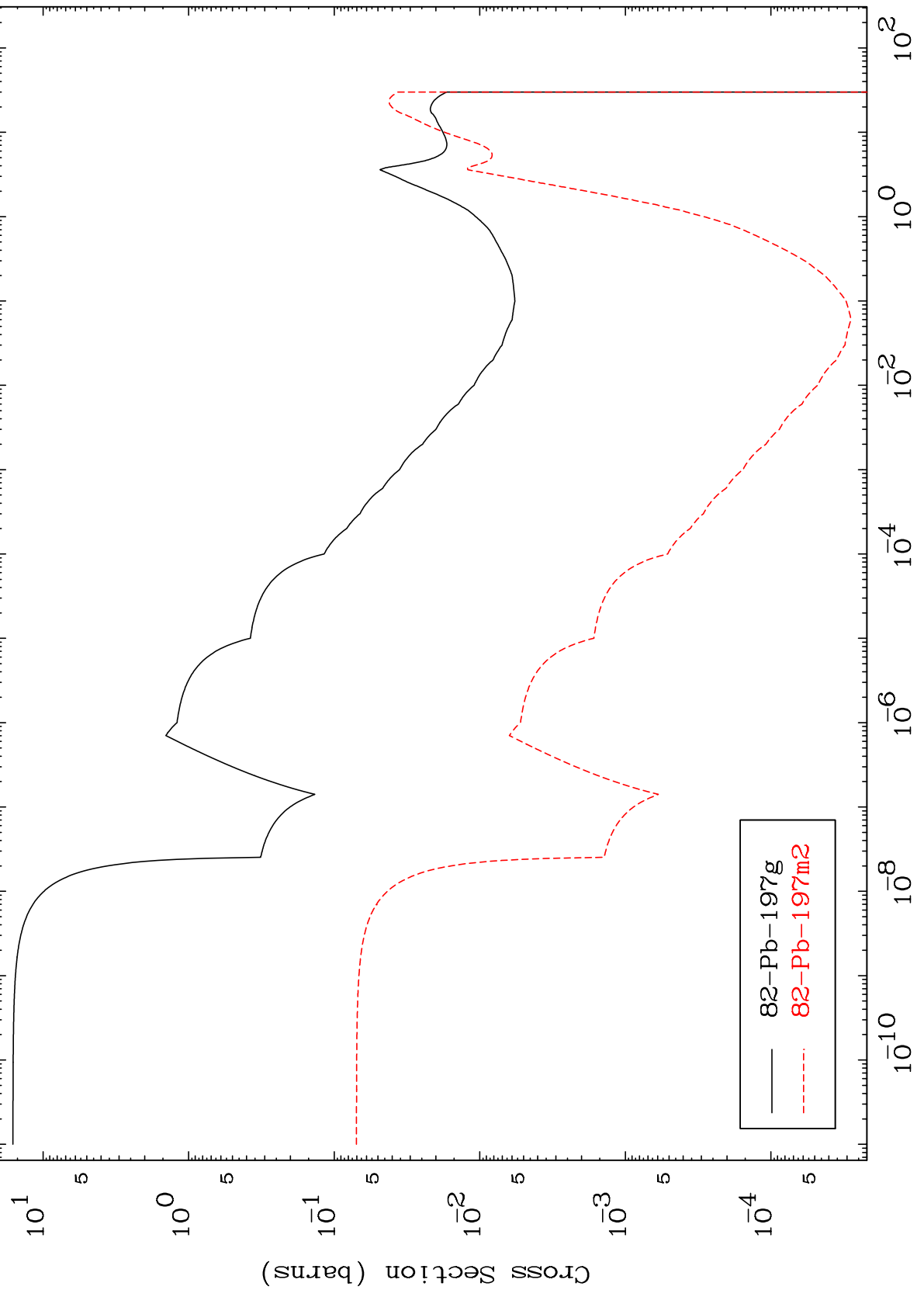
83-Bi-197m



MAT 8290

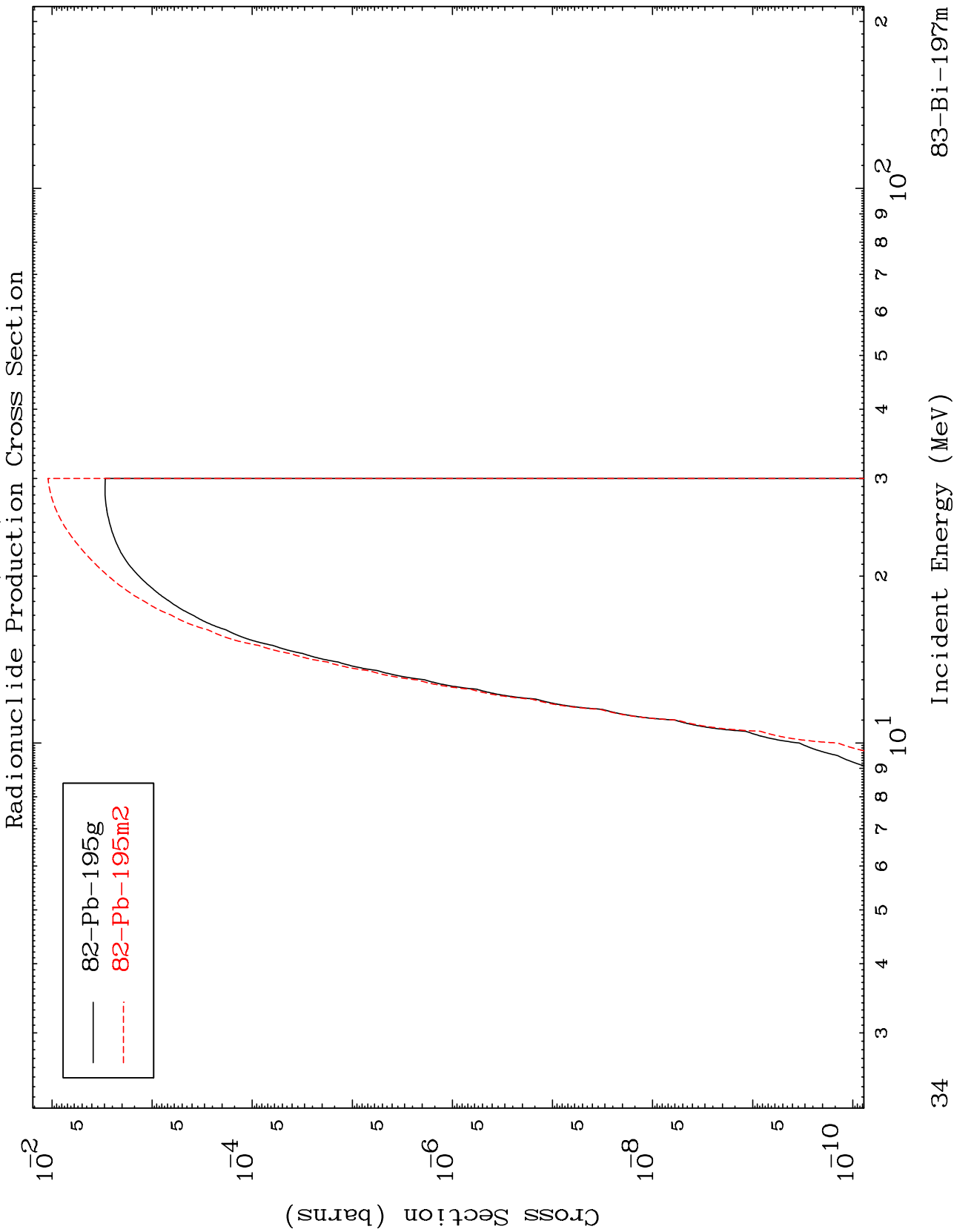
83-Bi-197m

(n,p)
Radionuclide Production Cross Section



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⁸³Bi-197m

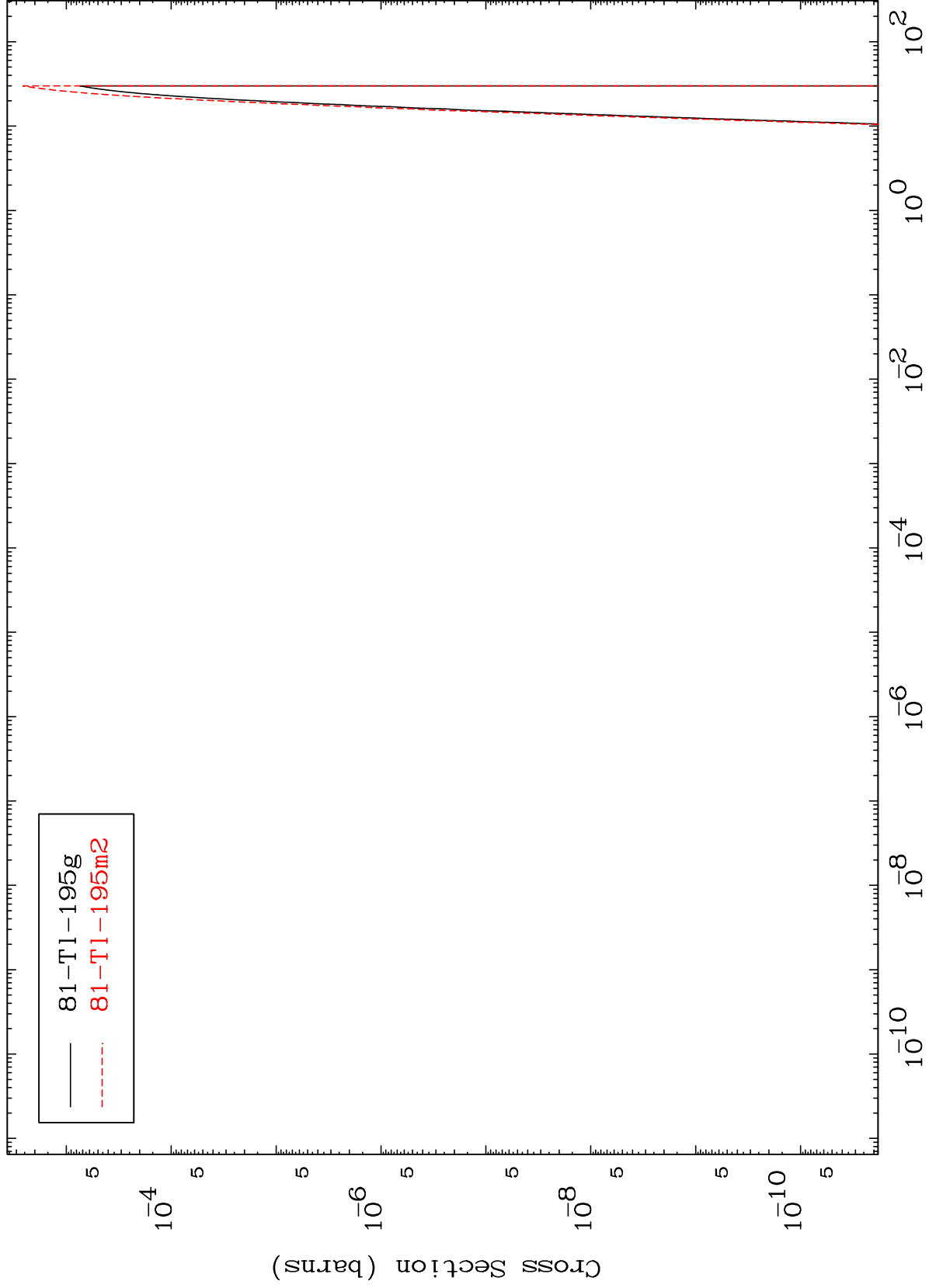


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(n,He-3)

83-Bi-197m

Radionuclide Production Cross Section



35

Incident Energy (MeV)

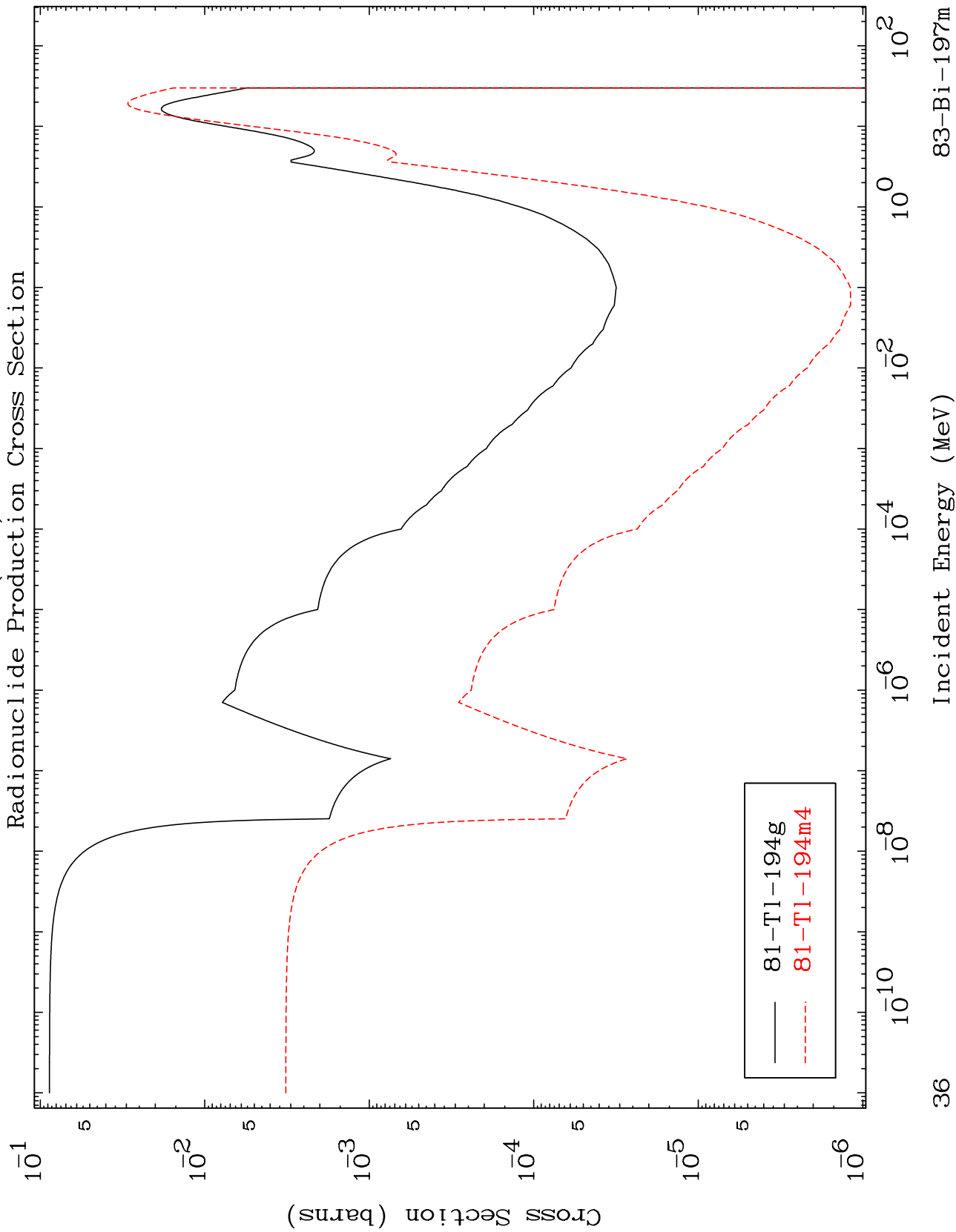
83-Bi-197m

MAT 8290

83-Bi-197m

Radionuclide Production Cross Section

(n, α)

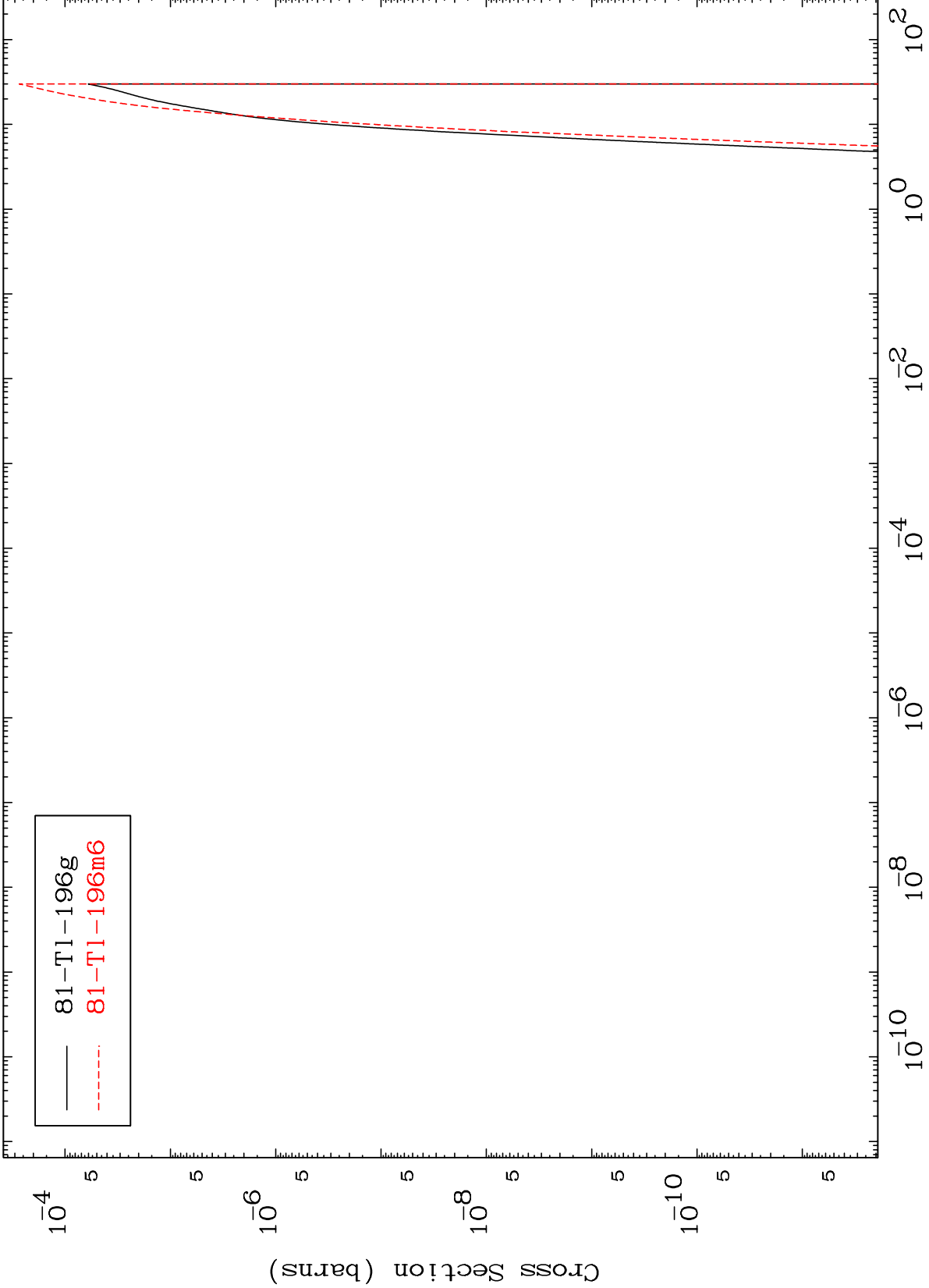
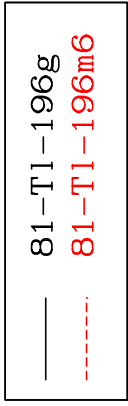


MAT 8290

(n,2p)

83-Bi-197m

Radionuclide Production Cross Section

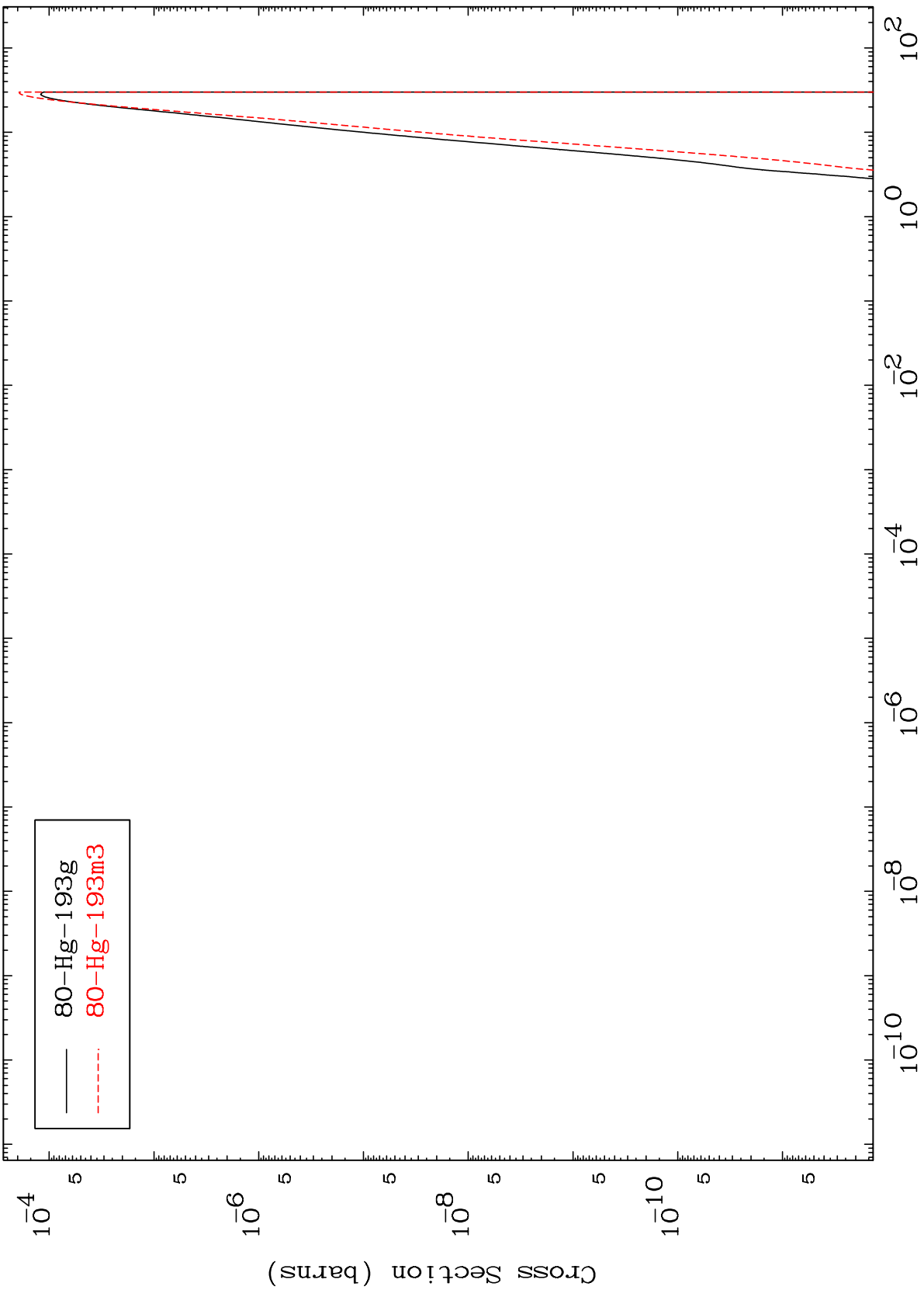


MAT 8290

(n,p) α

83-Bi-197m

Radionuclide Production Cross Section

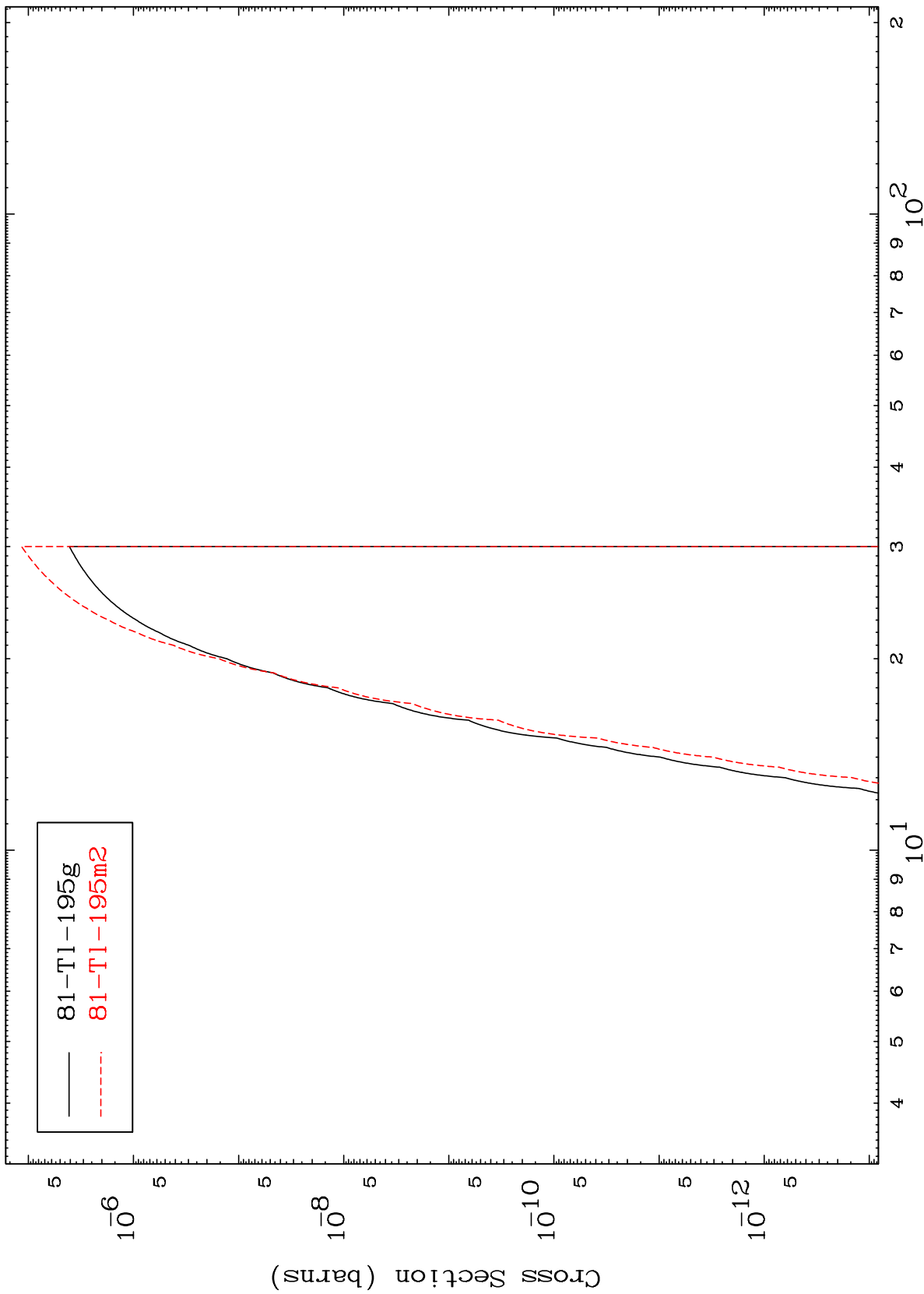


MAT 8290

(n,p) d

83-Bi-197m

Radionuclide Production Cross Section



81-Tl-195g
81-Tl-195m2

Incident Energy (MeV)

83-Bi-197m

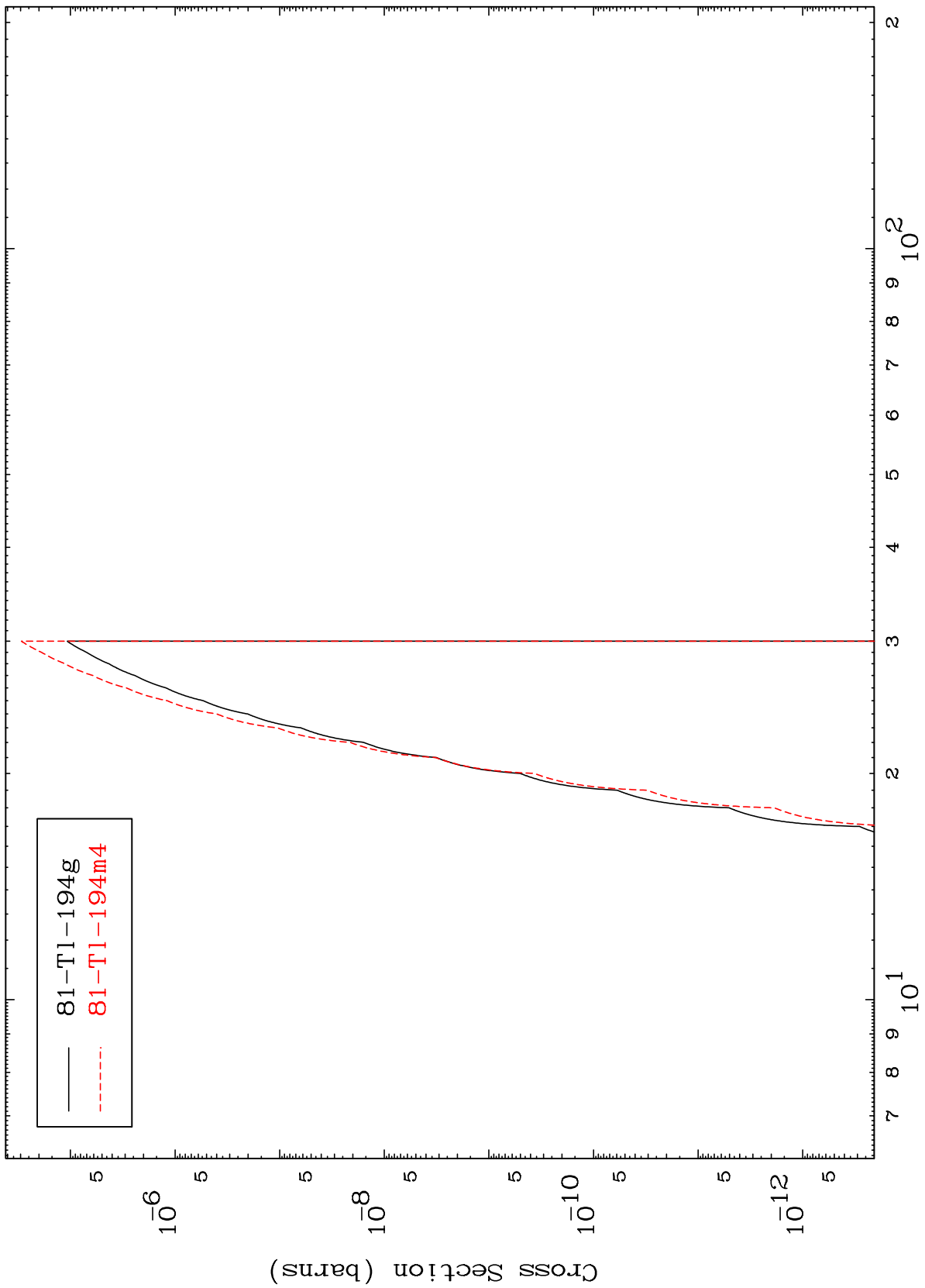
39

MAT 8290

(n,p) t

83-Bi-197m

Radionuclide Production Cross Section



40

Incident Energy (MeV)

83-Bi-197m