

Program EVALPLOT
(Version 2021-1)

by

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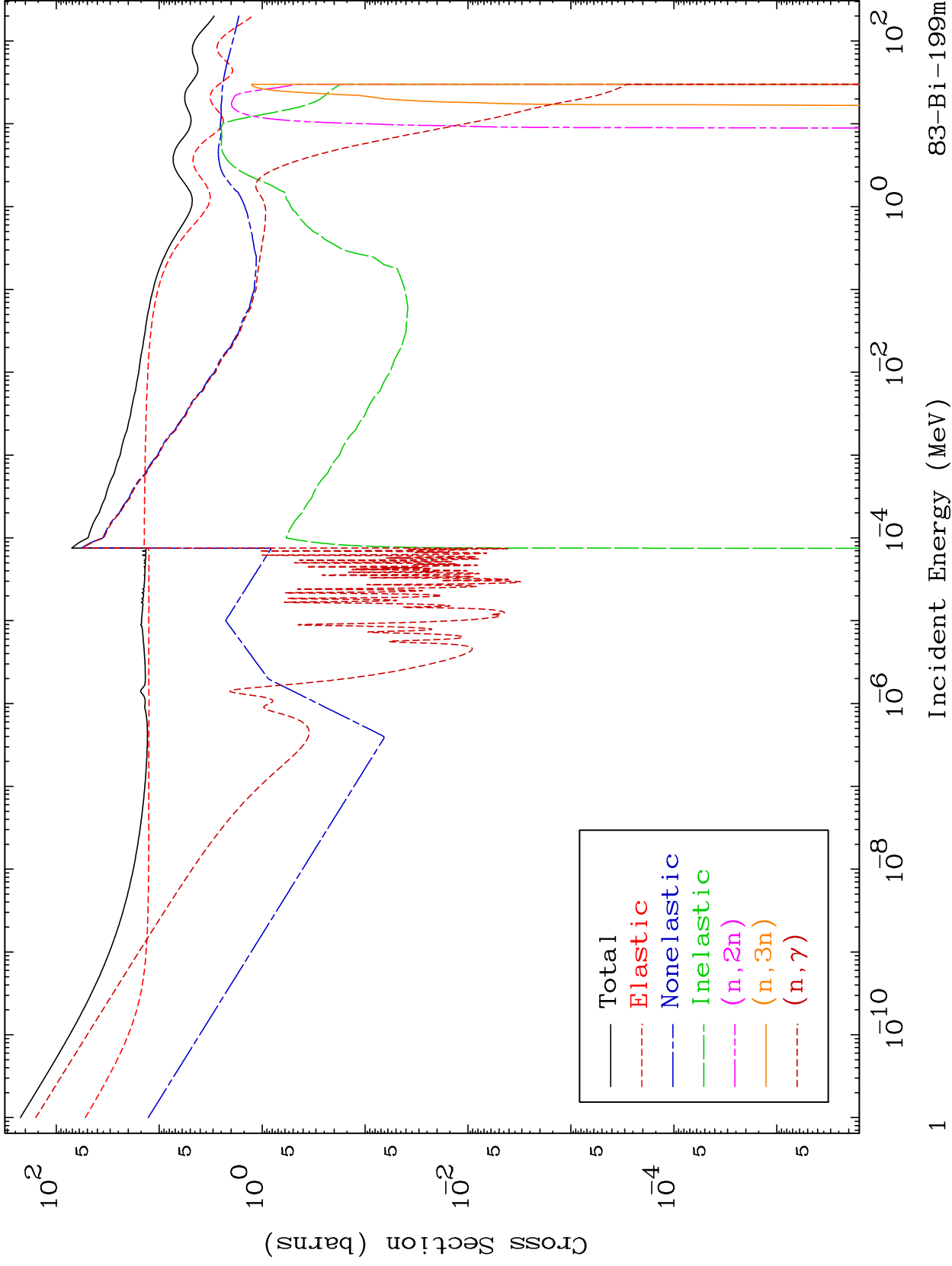
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Press Mouse Button to Start

MAT 8296

Neutron Major
293 Kelvin Cross Sections

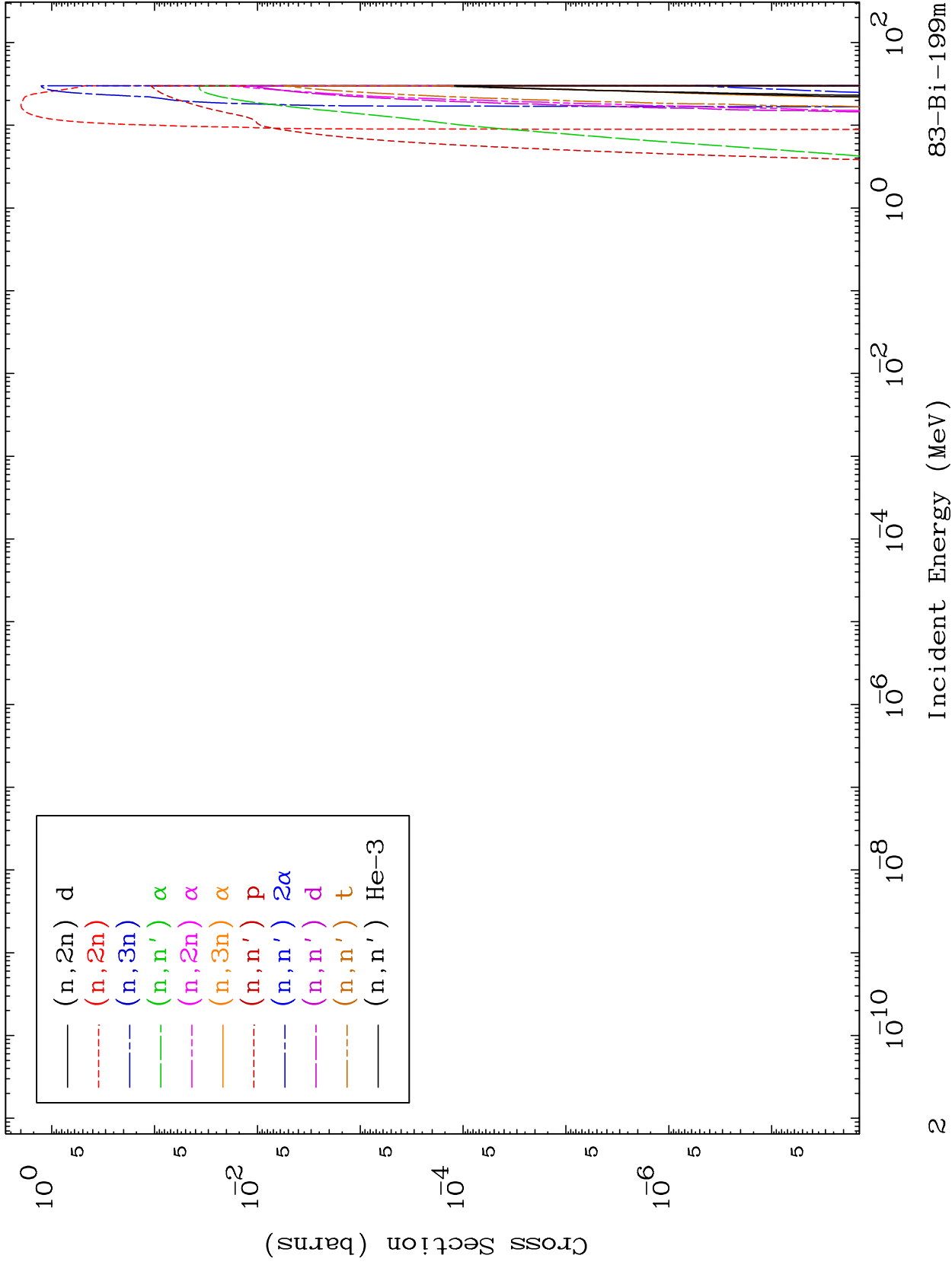
83-Bi-199m



MAT 8296

Neutron Absorption
293 Kelvin Cross Sections

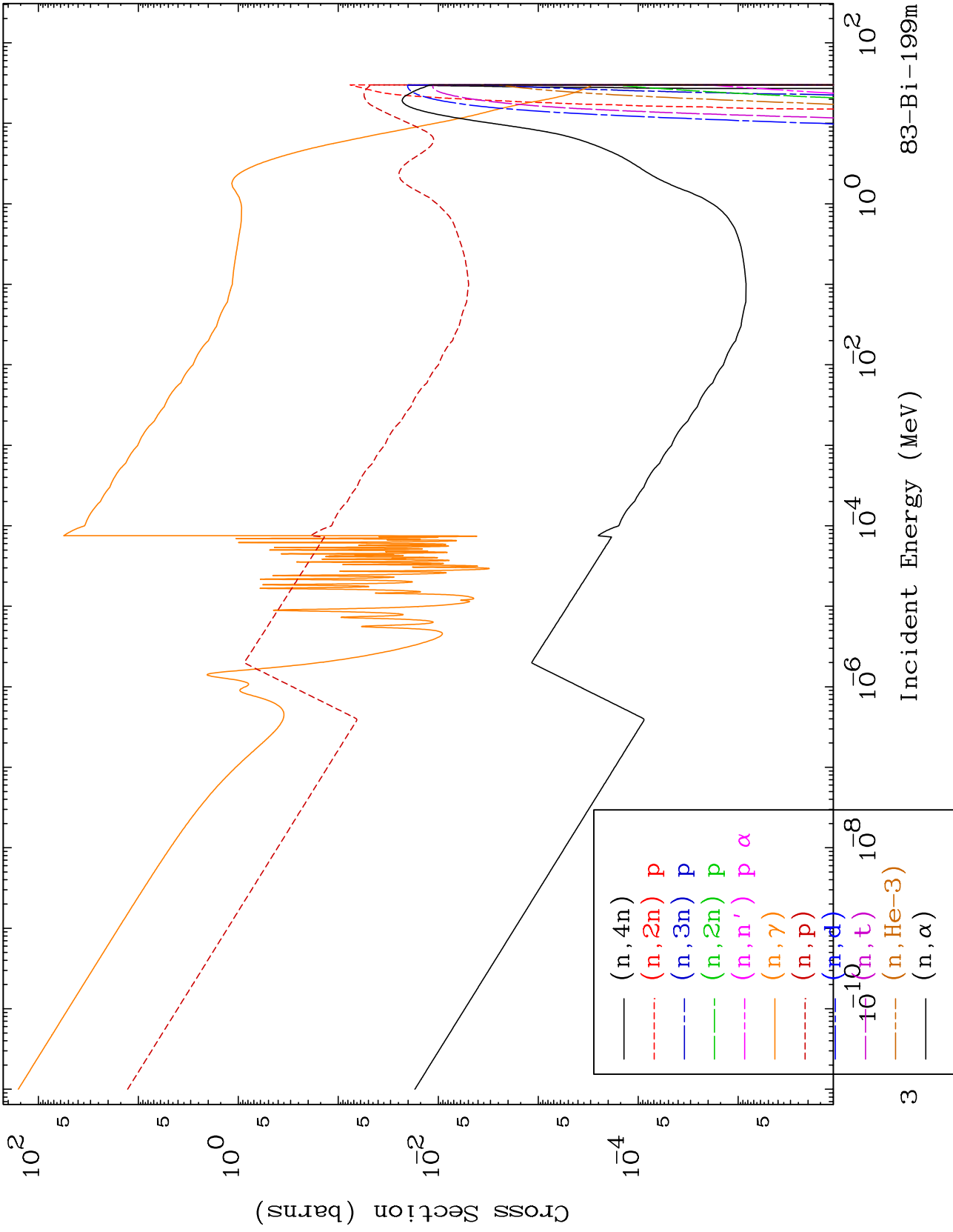
83-Bi-199m



MAT 8296

Neutron Absorption
293 Kelvin Cross Sections

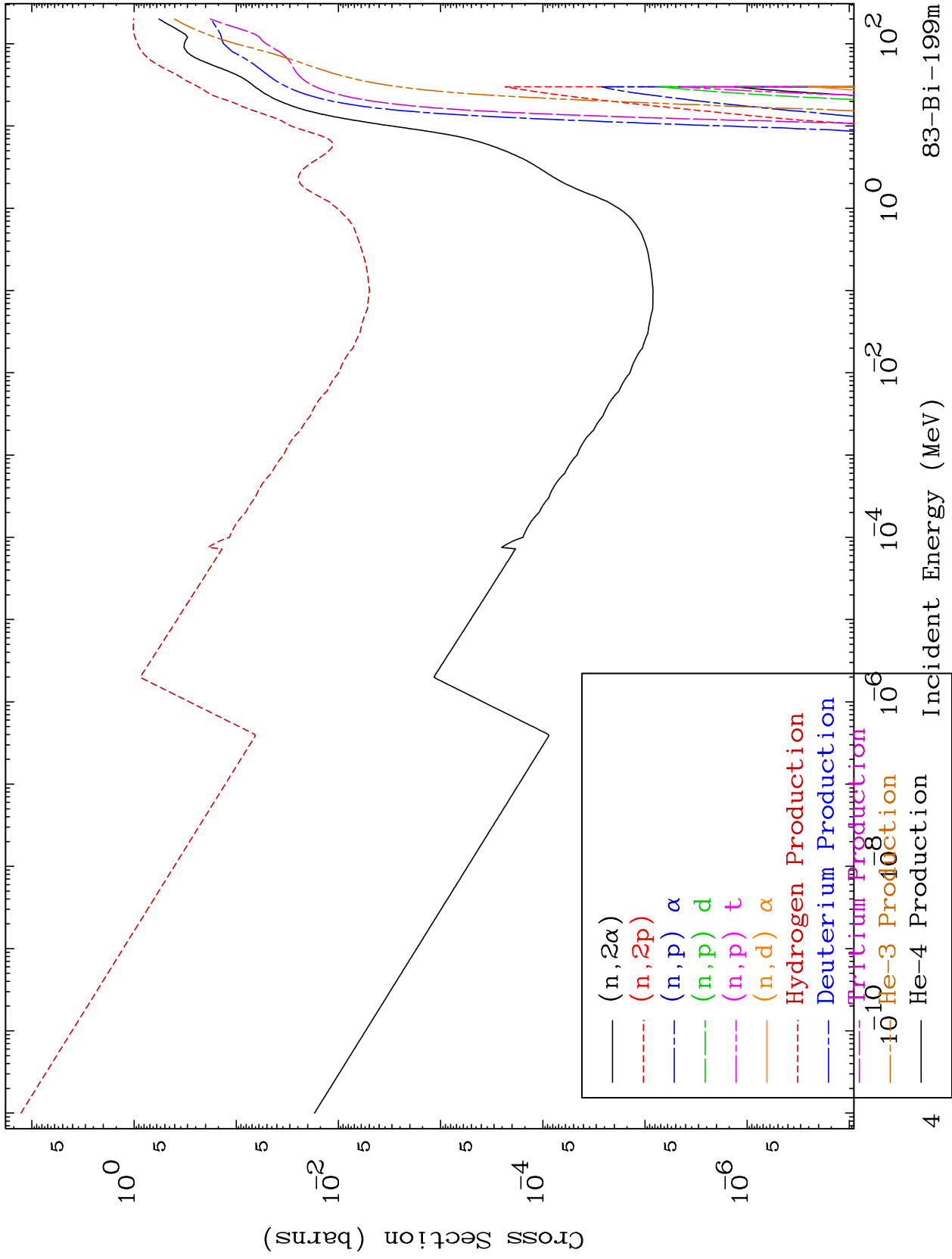
83-Bi-199m



MAT 8296

Neutron Absorption
293 Kelvin Cross Sections

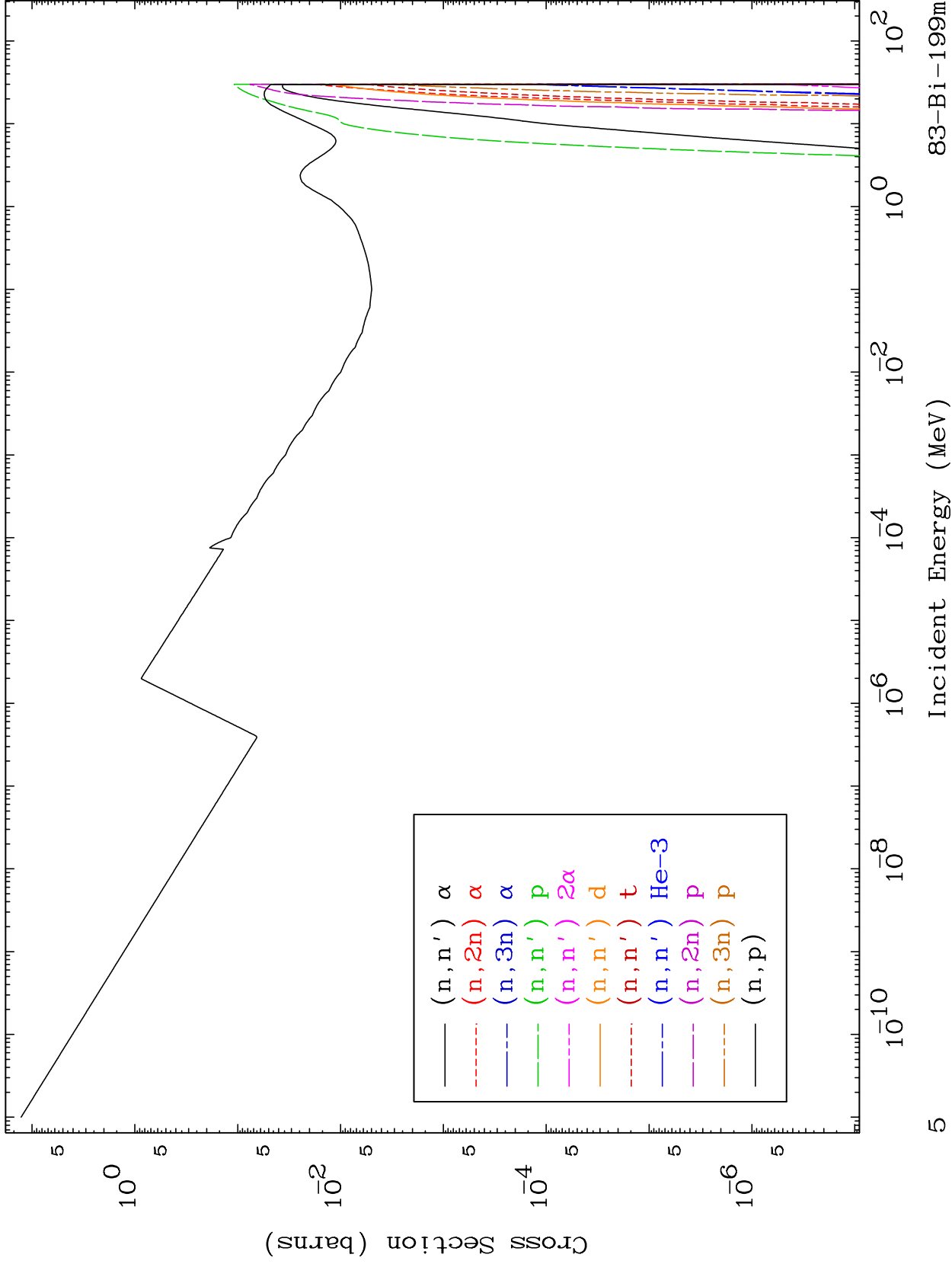
83-Bi-199m



MAT 8296

Charged Particle
293 Kelvin Cross Sections

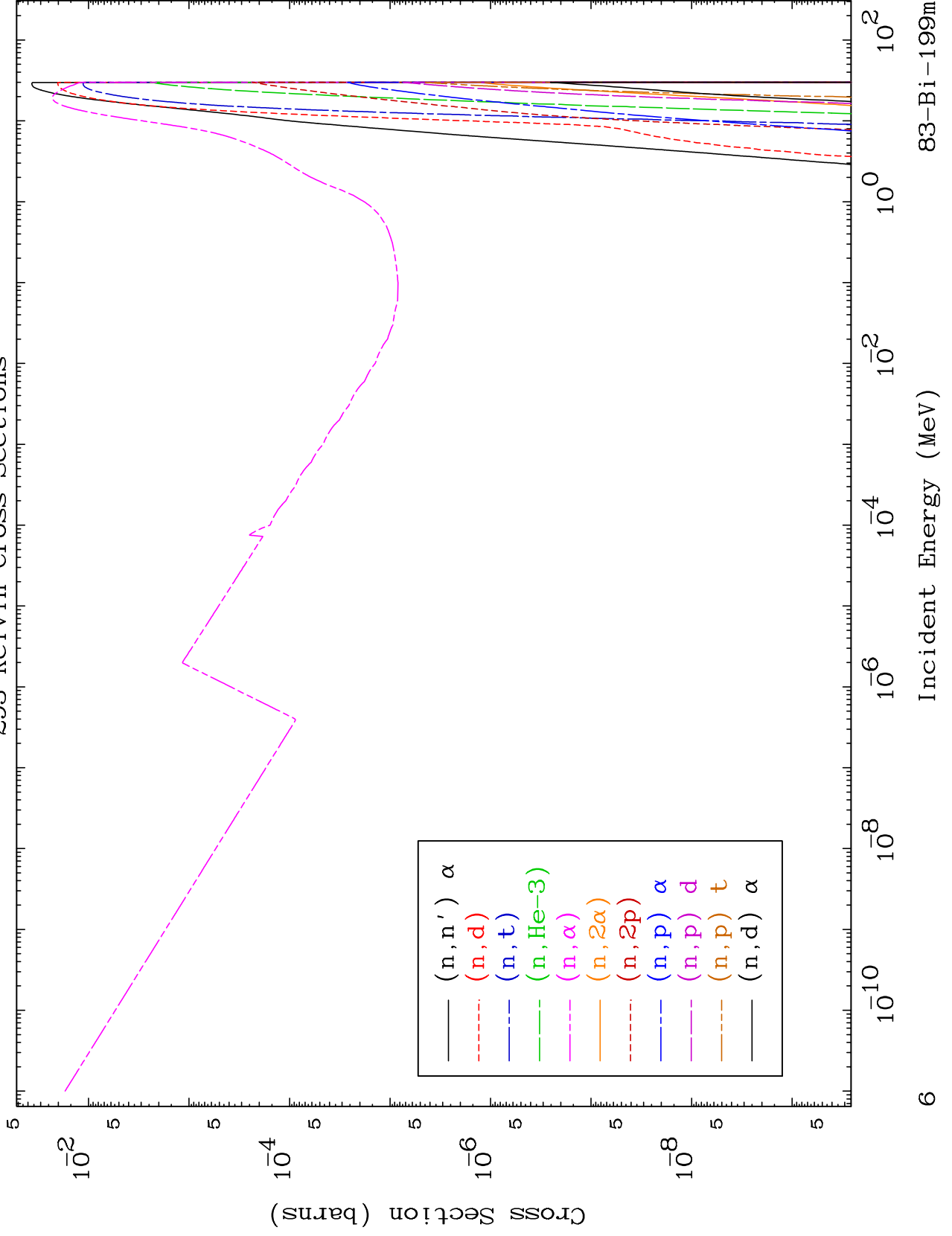
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MAT 8296

Charged Particle
293 Kelvin Cross Sections

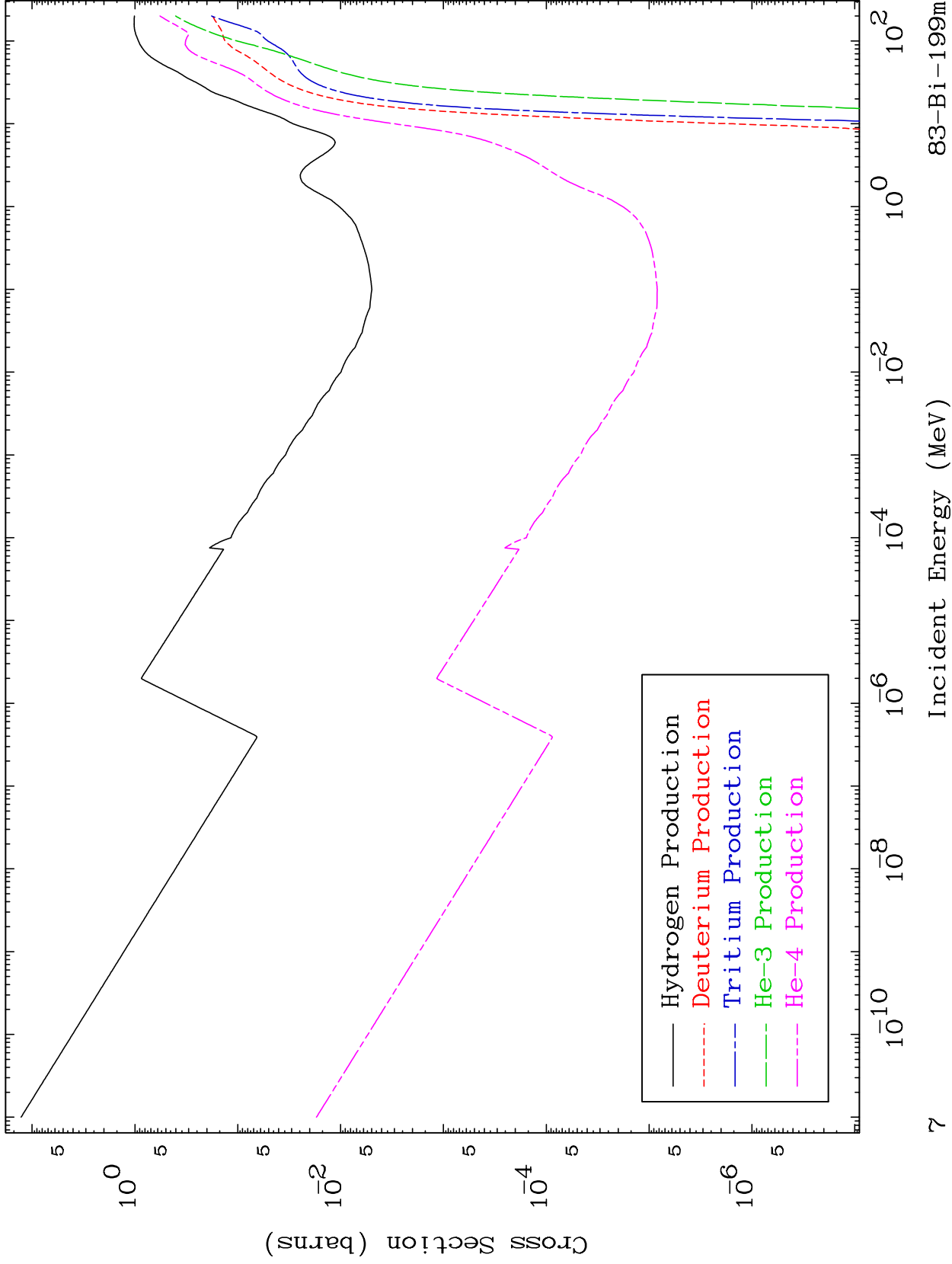
83-Bi-199m



MAT 8296

Particle Production
293 Kelvin Cross Sections

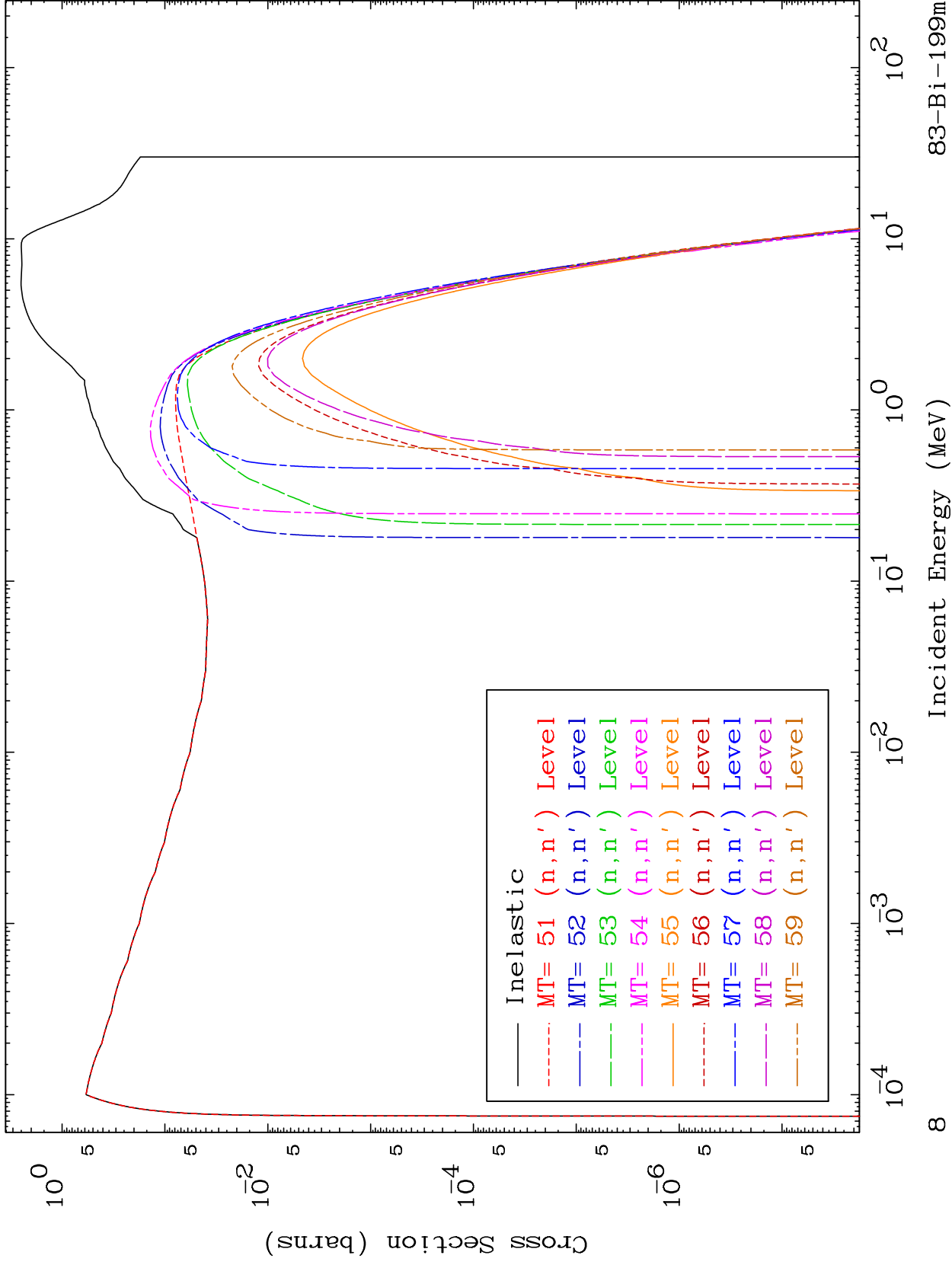
83-Bi-199m



MAT 8296

(n,n') Levels
293 Kelvin Cross Sections

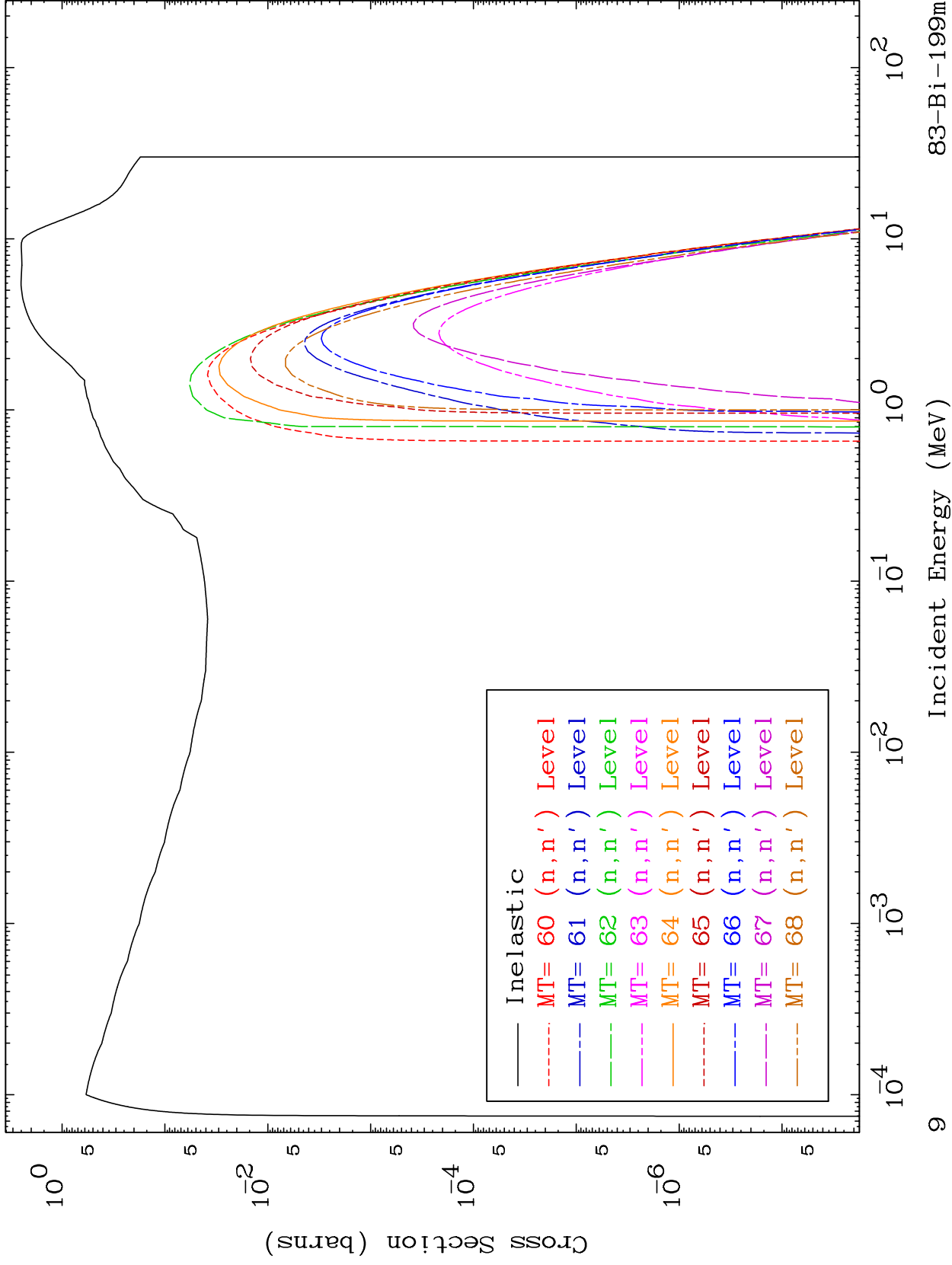
83-Bi-199m



MAT 8296

(n,n') Levels
293 Kelvin Cross Sections

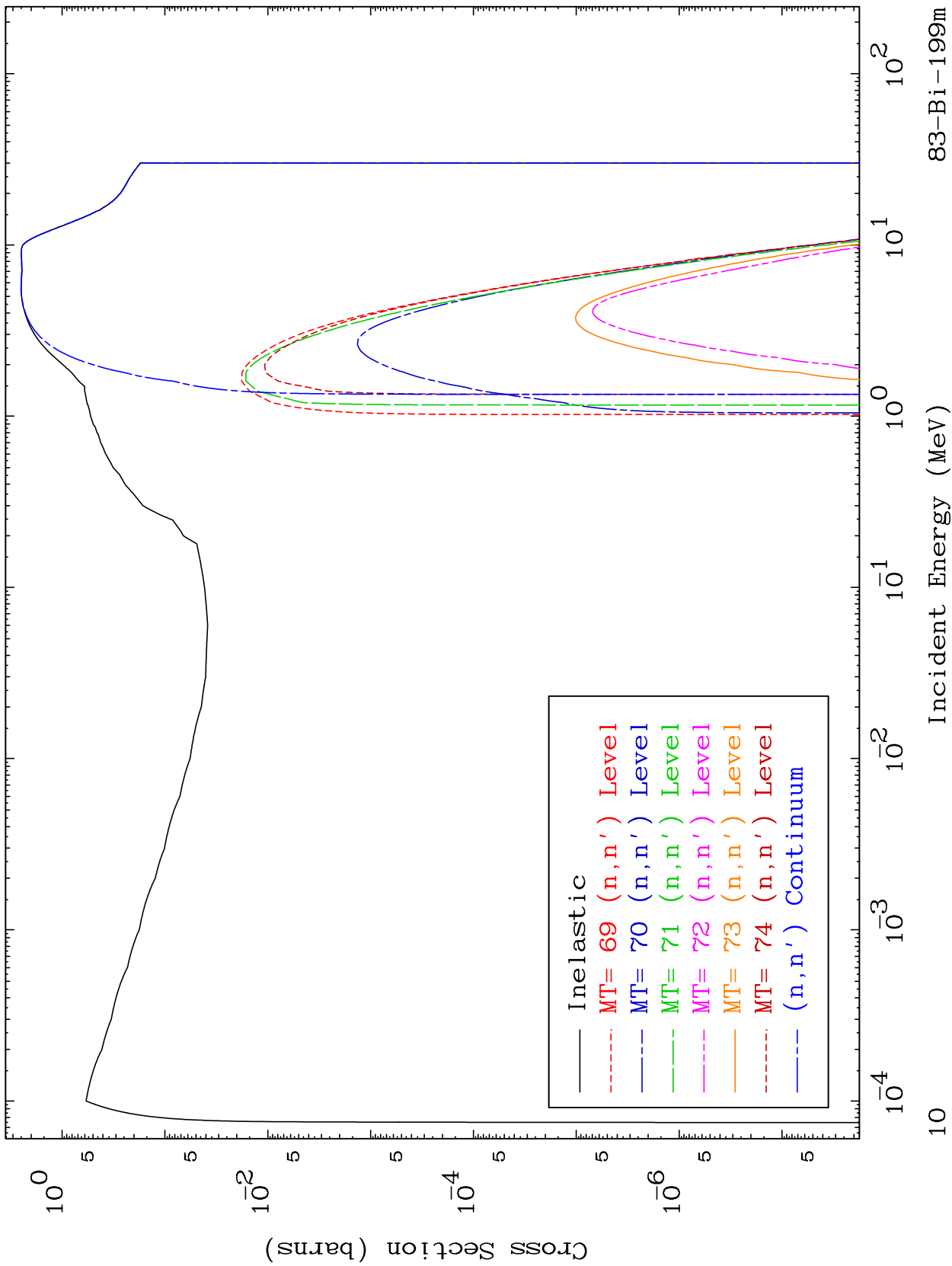
83-Bi-199m



MAT 8296

(n,n') Levels
293 Kelvin Cross Sections

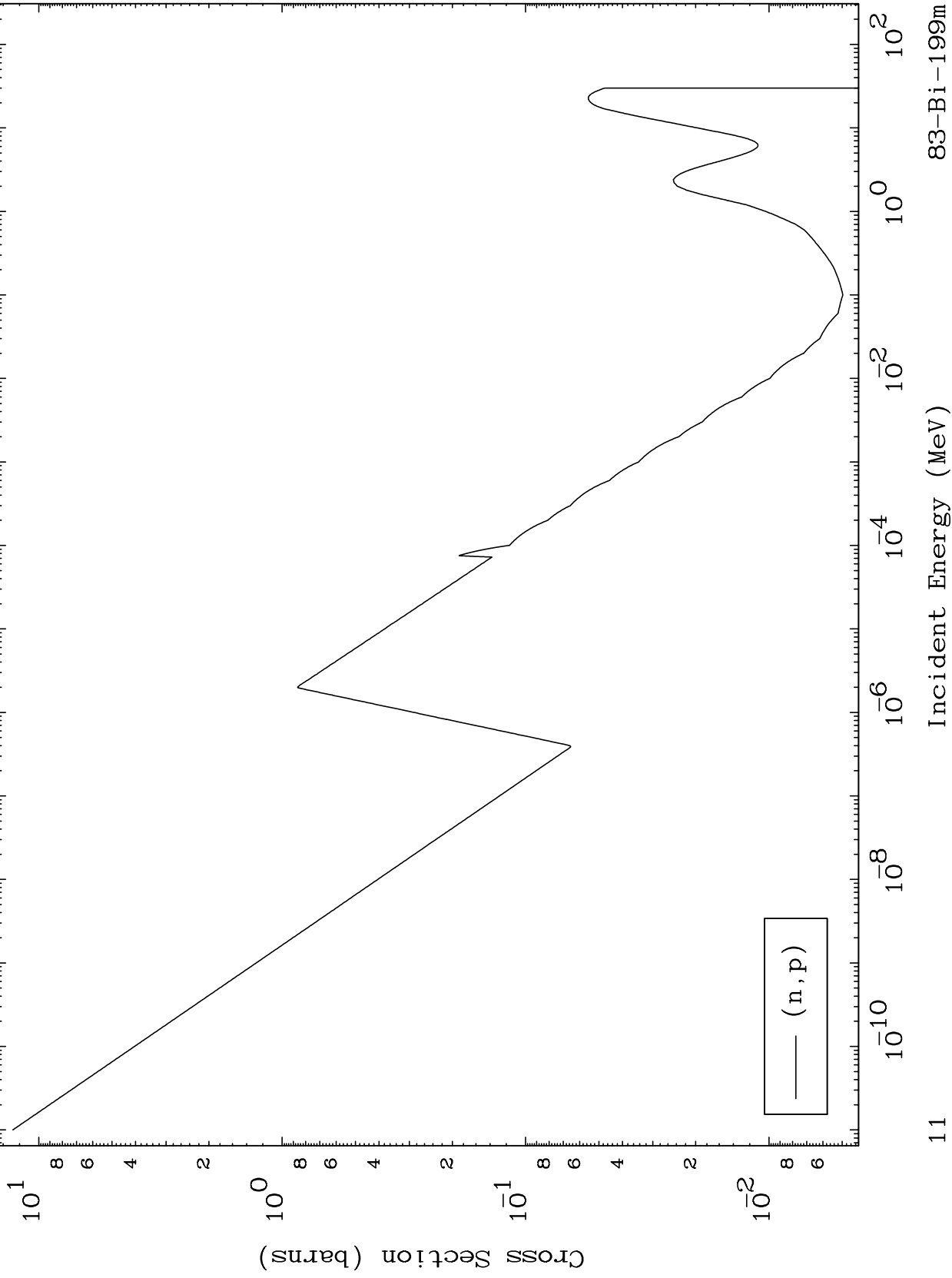
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MAT 8296

(n,p) Levels
293 Kelvin Cross Sections

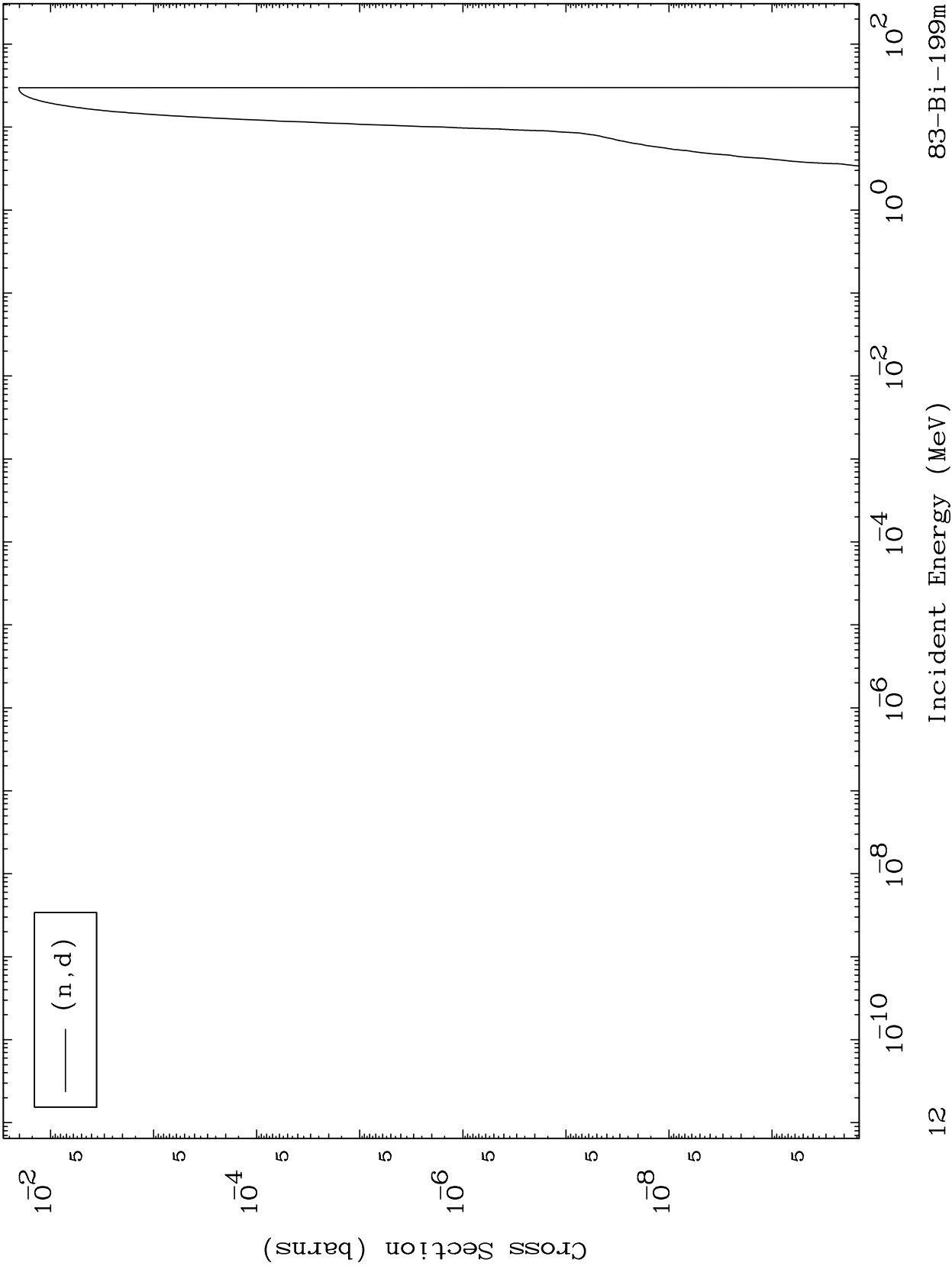
83-Bi-199m



MAT 8296

(n,d) Levels
293 Kelvin Cross Sections

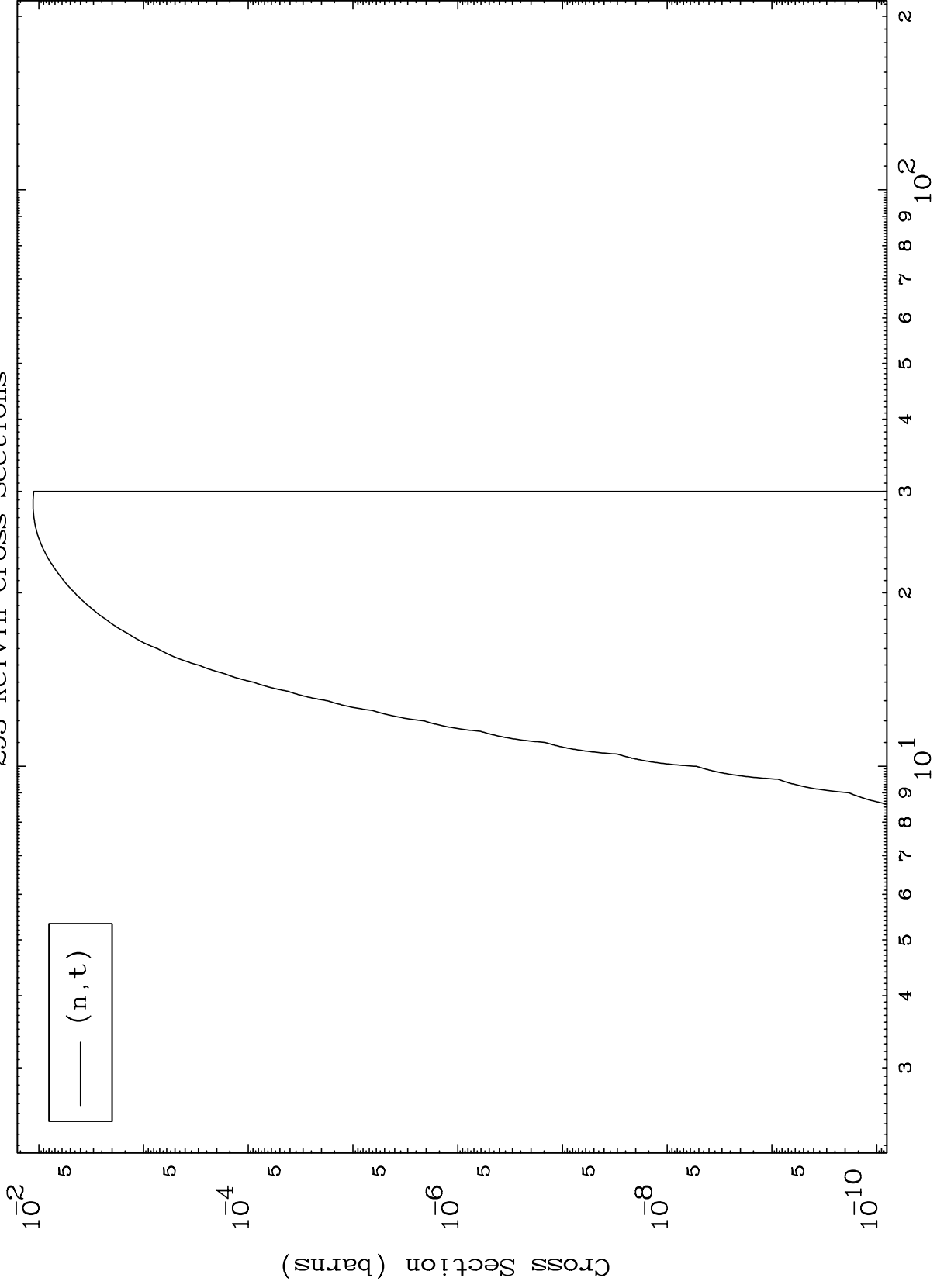
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MAT 8296

(n,t) Levels
293 Kelvin Cross Sections

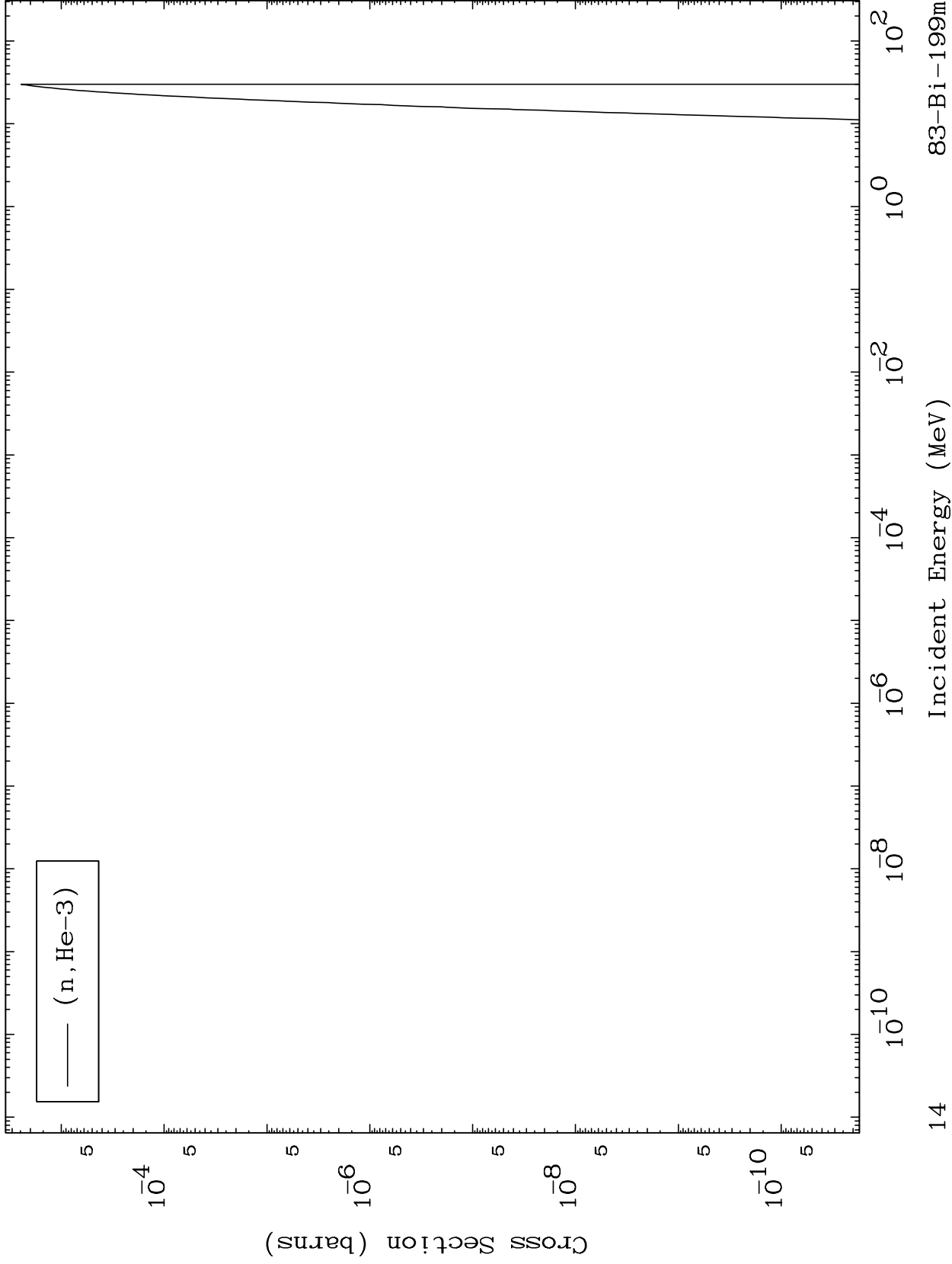
83-Bi-199m



MAT 8296

(n,He3) Levels
293 Kelvin Cross Sections

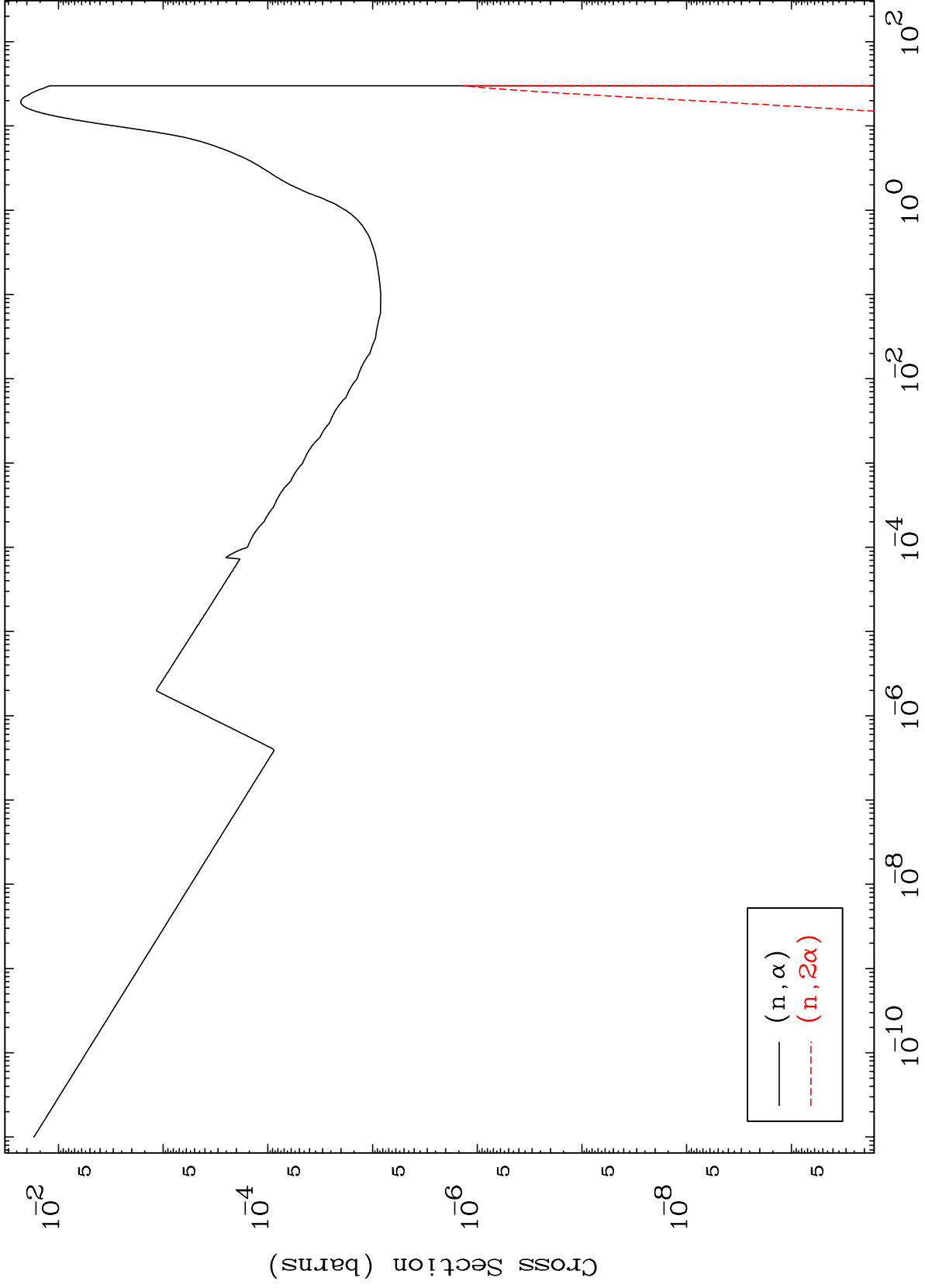
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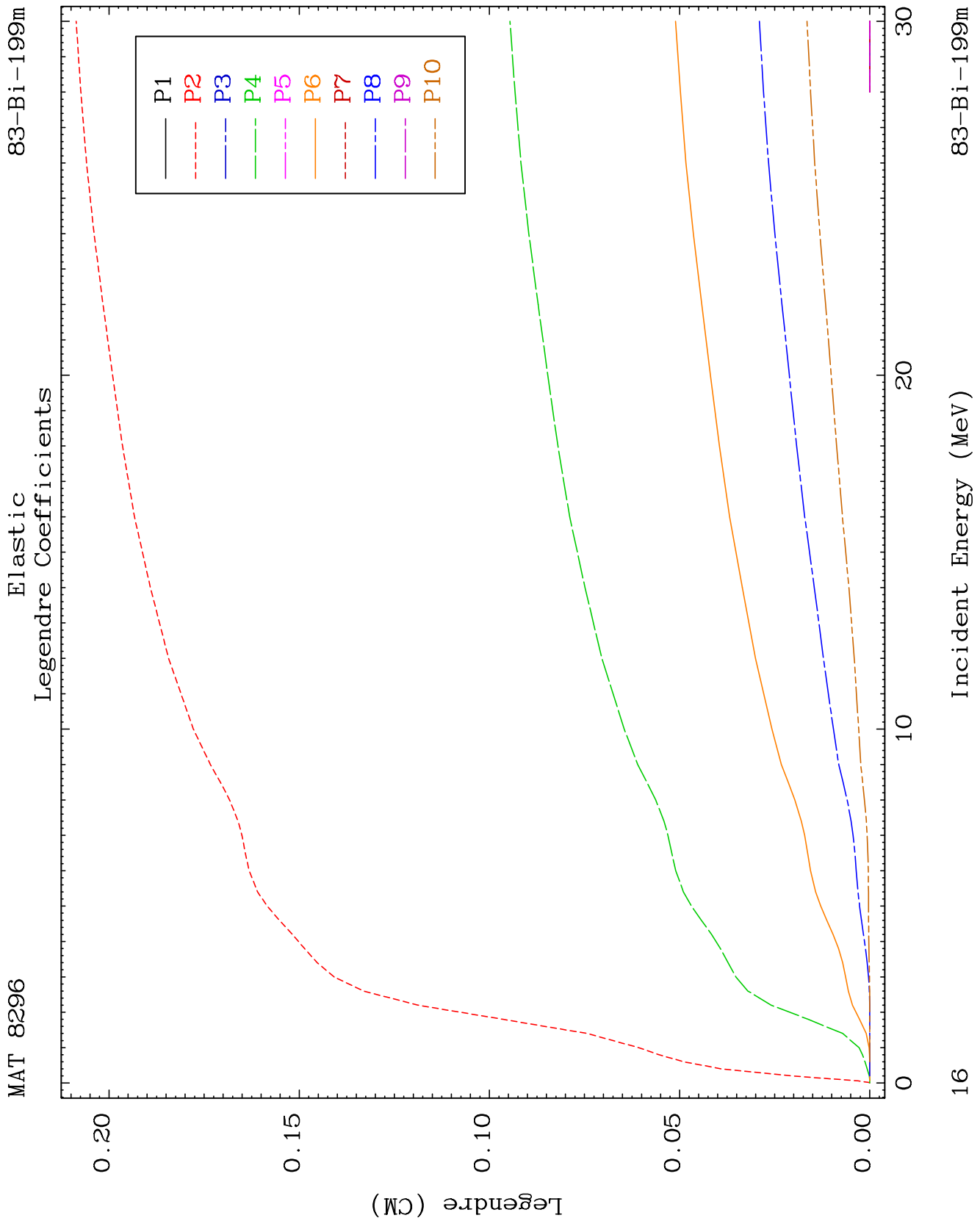


MAT 8296

(n,α) Levels
293 Kelvin Cross Sections

83-Bi-199m

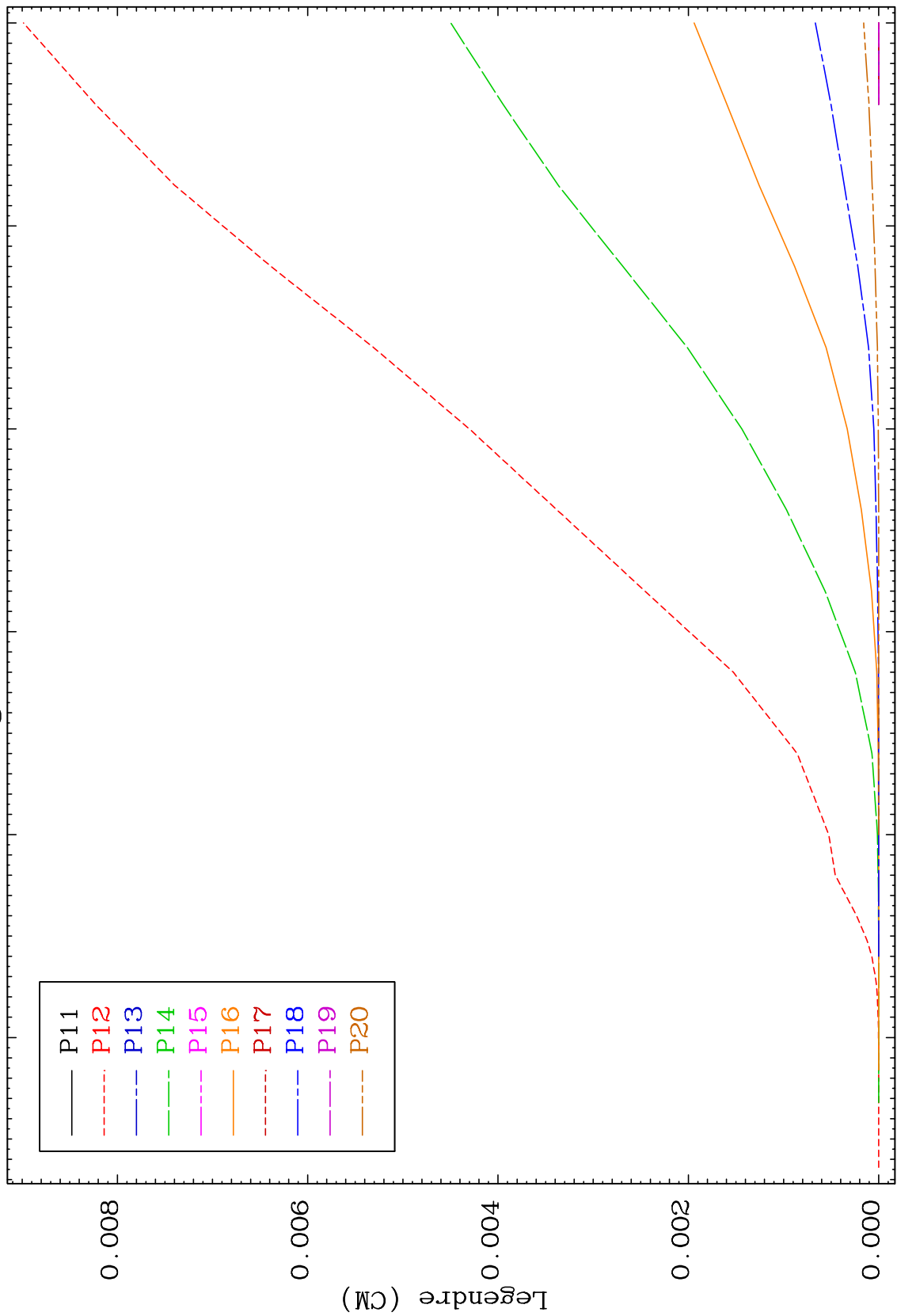




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Elastic Legendre Coefficients

83-Bi-199m



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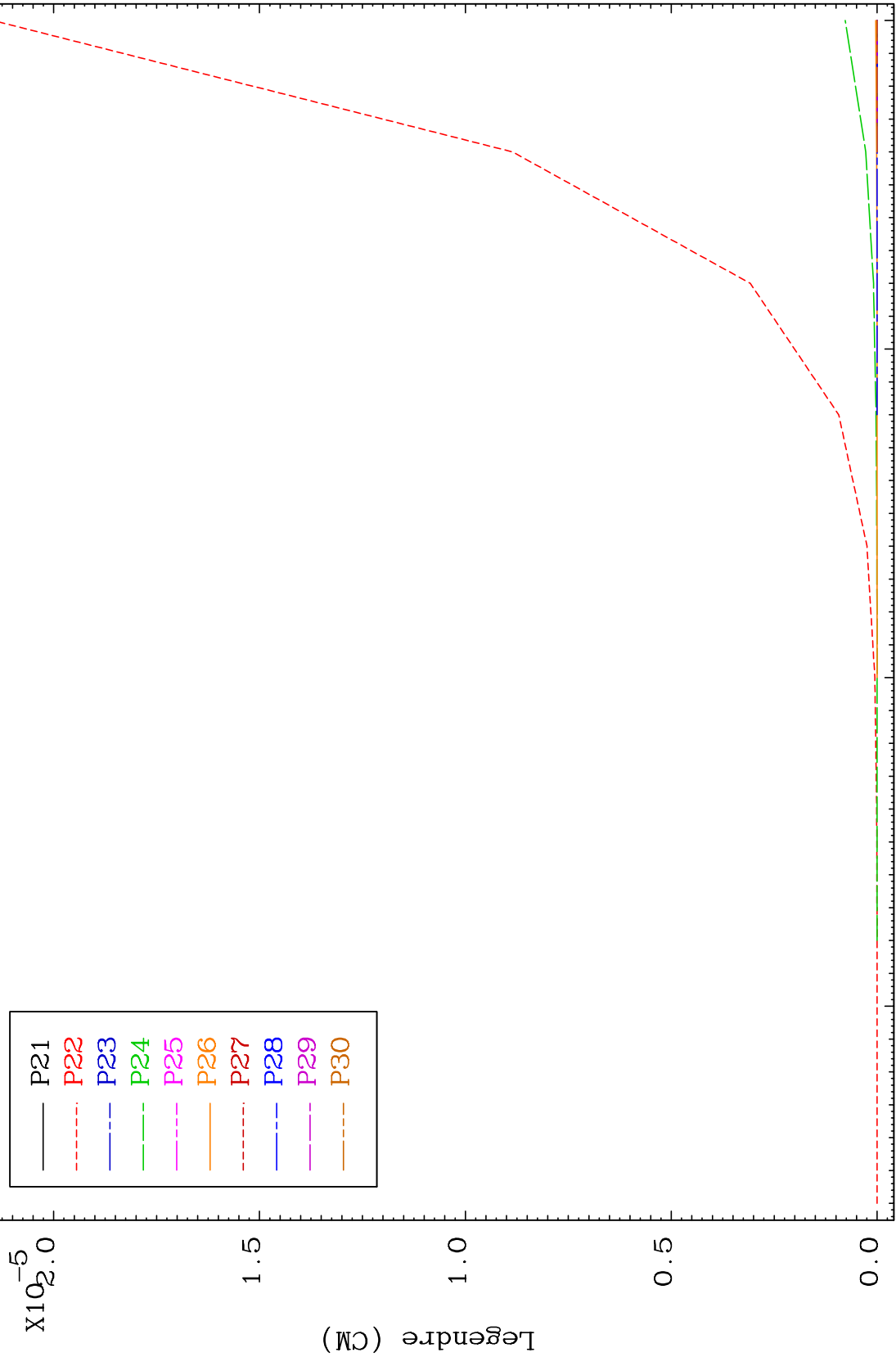
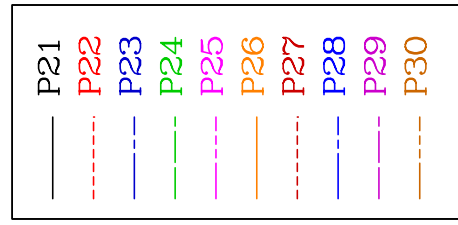
83-Bi-199m

Incident Energy (MeV)

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Elastic Legendre Coefficients

83-Bi-199m



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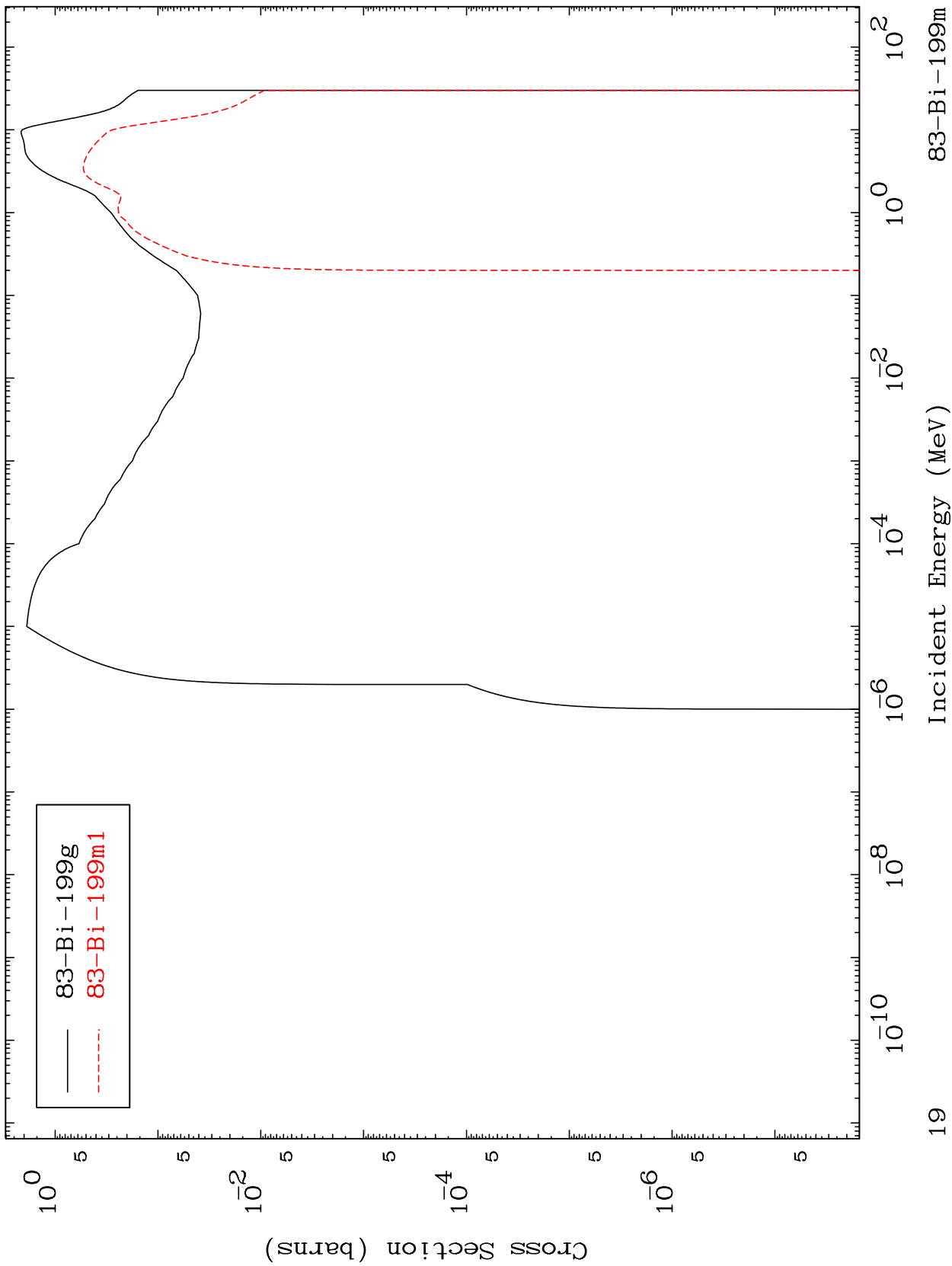
Incident Energy (MeV)

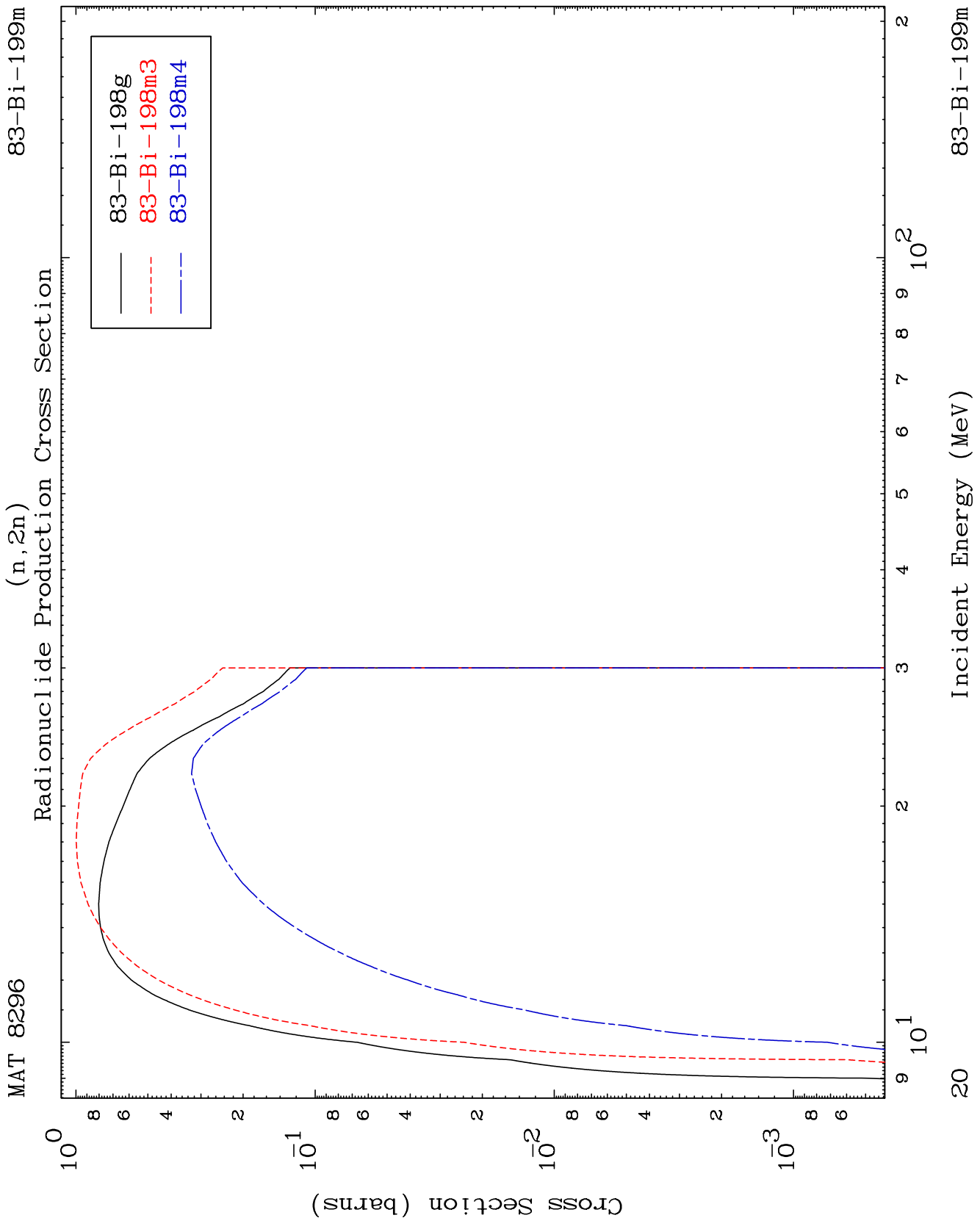
83-Bi-199m

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83-Bi-199m

Inelastic
Radionuclide Production Cross Section



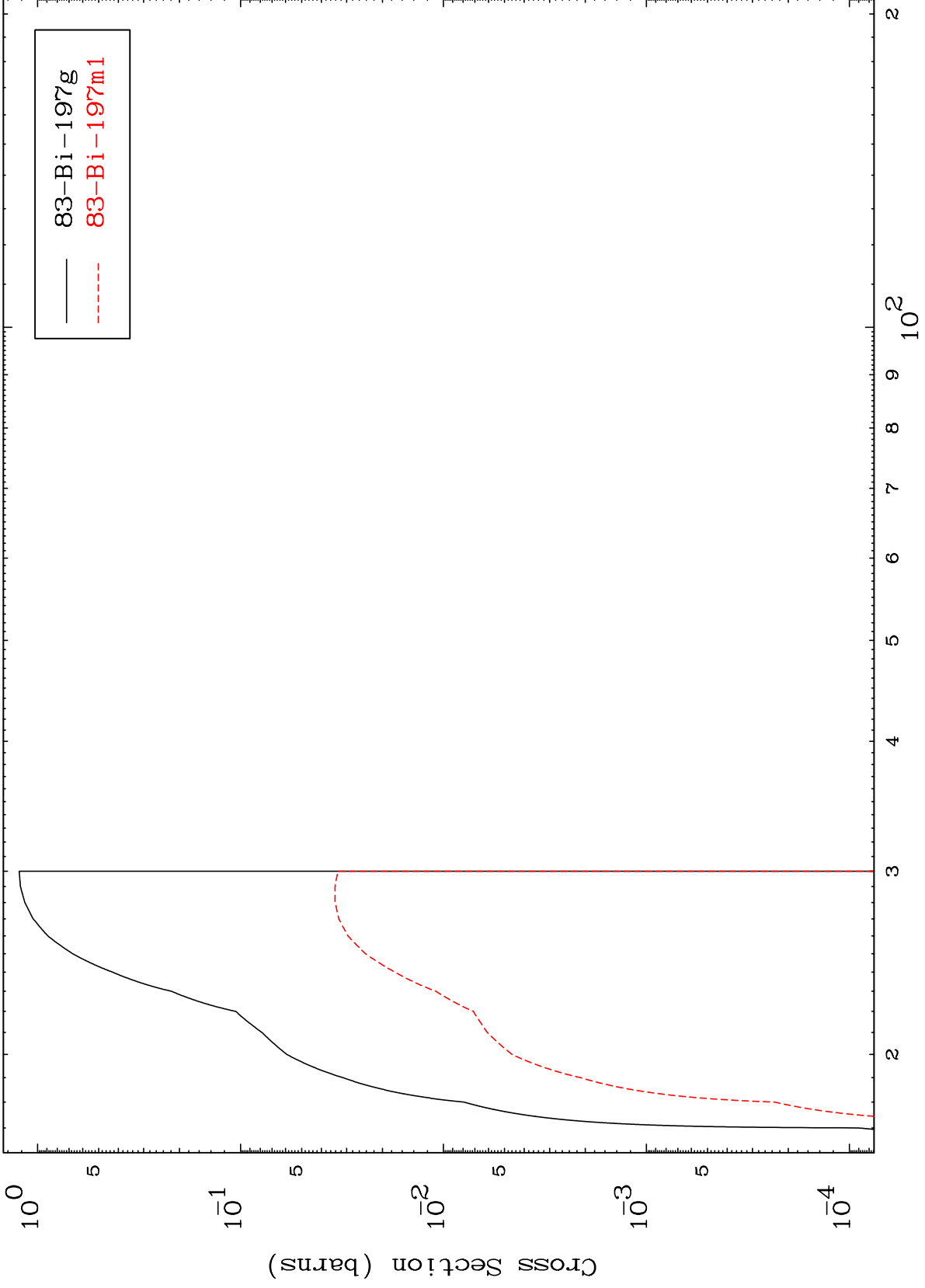


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(n,3n)

83-Bi-199m

Radionuclide Production Cross Section



21

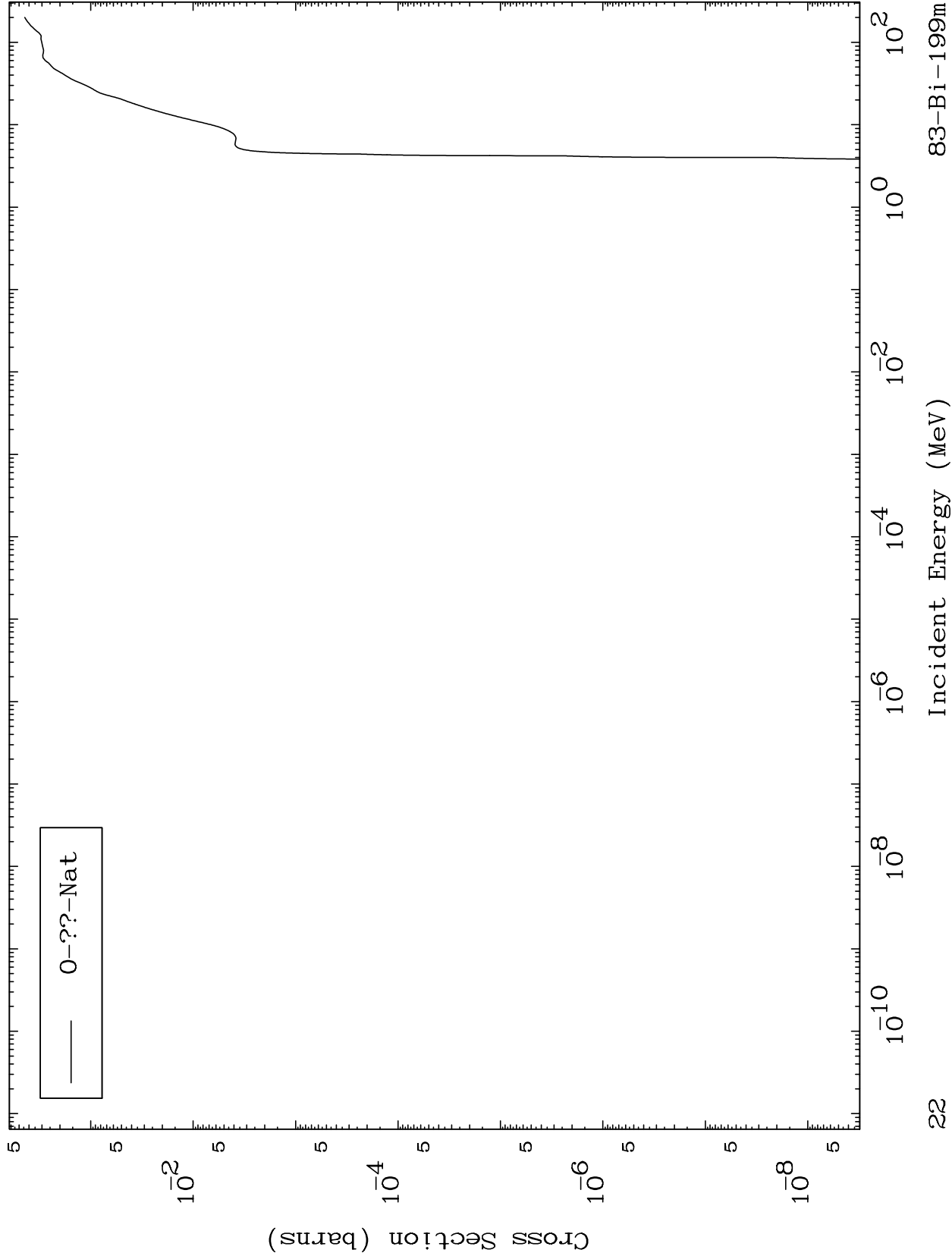
Incident Energy (MeV)

83-Bi-199m

MAT 8296

Fission
Radionuclide Production Cross Section

83-Bi-199m

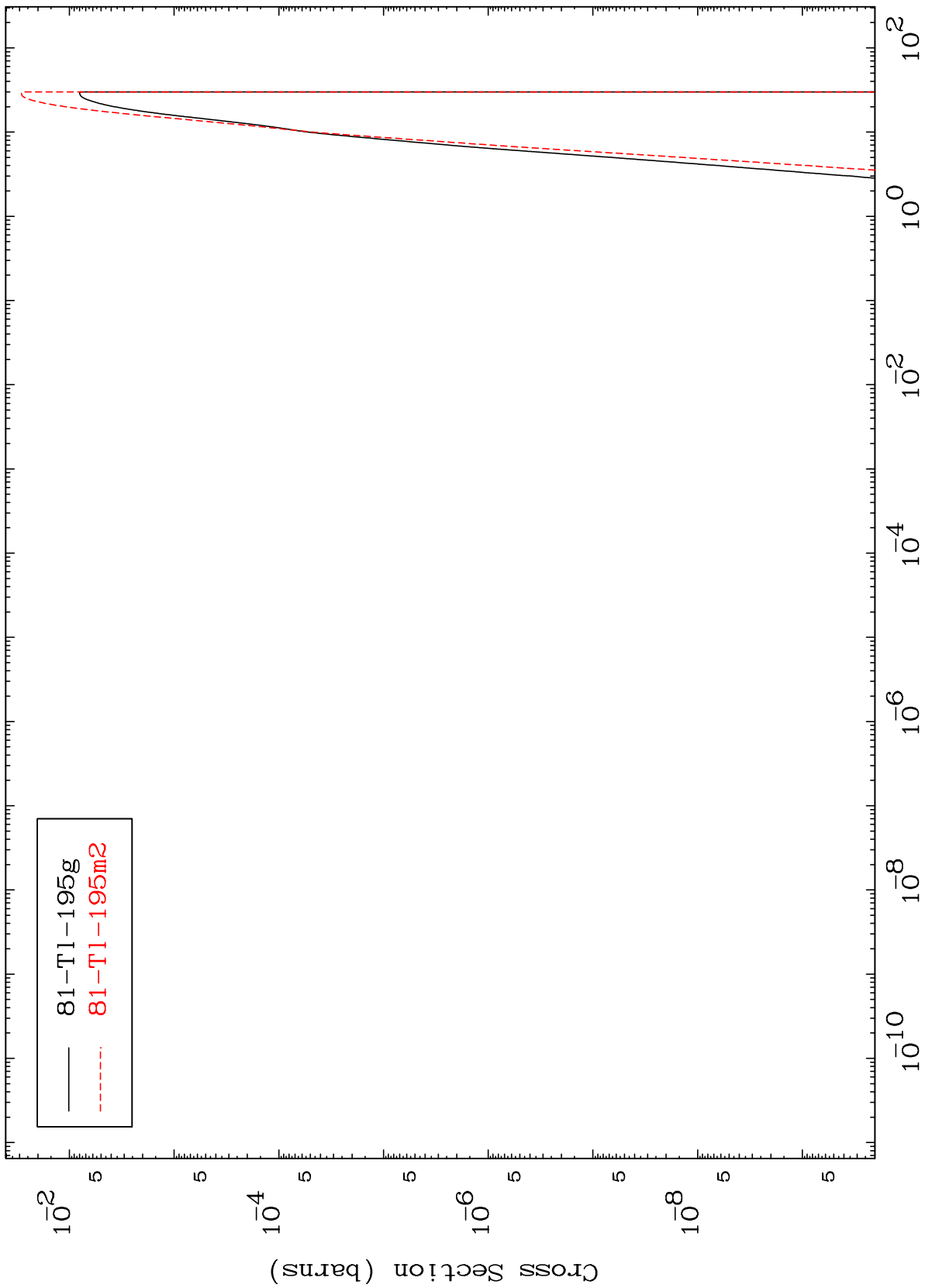


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$(n, n') \alpha$

83-Bi-199m

Radionuclide Production Cross Section

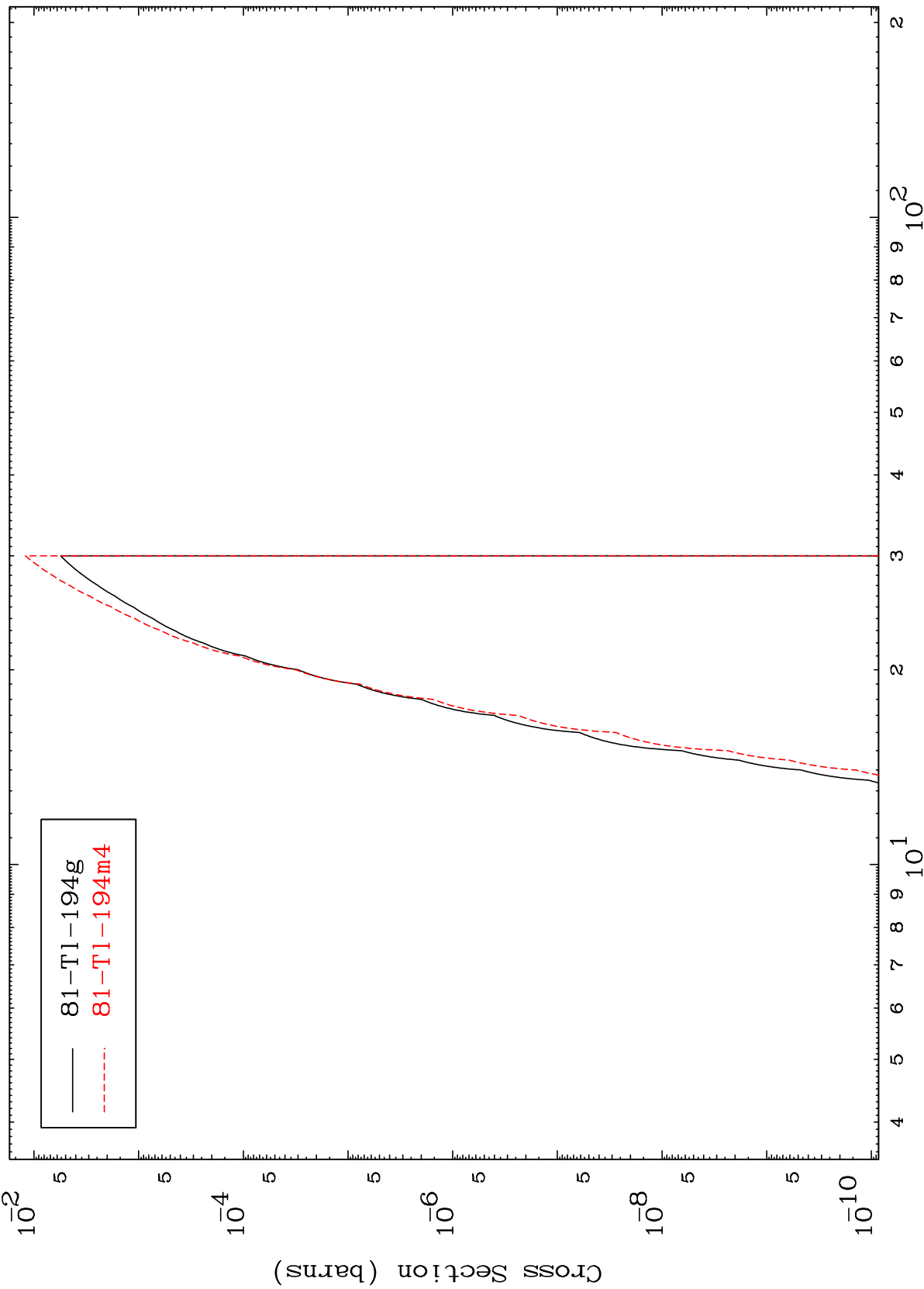


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$(n,2n) \alpha$

$^{83}\text{Bi}-199\text{m}$

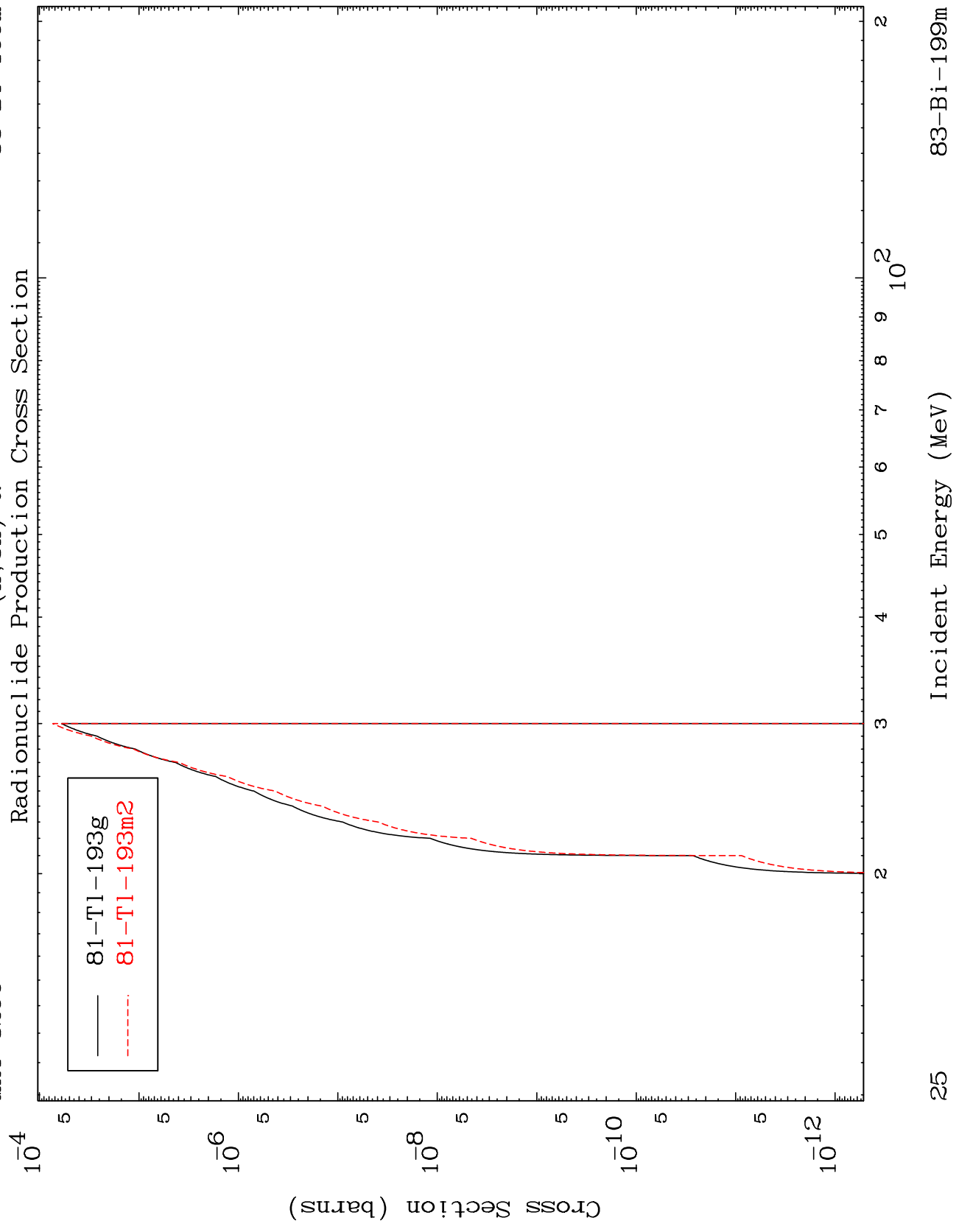
Radionuclide Production Cross Section



MAT 8296

(n,3n) α

83-Bi-199m

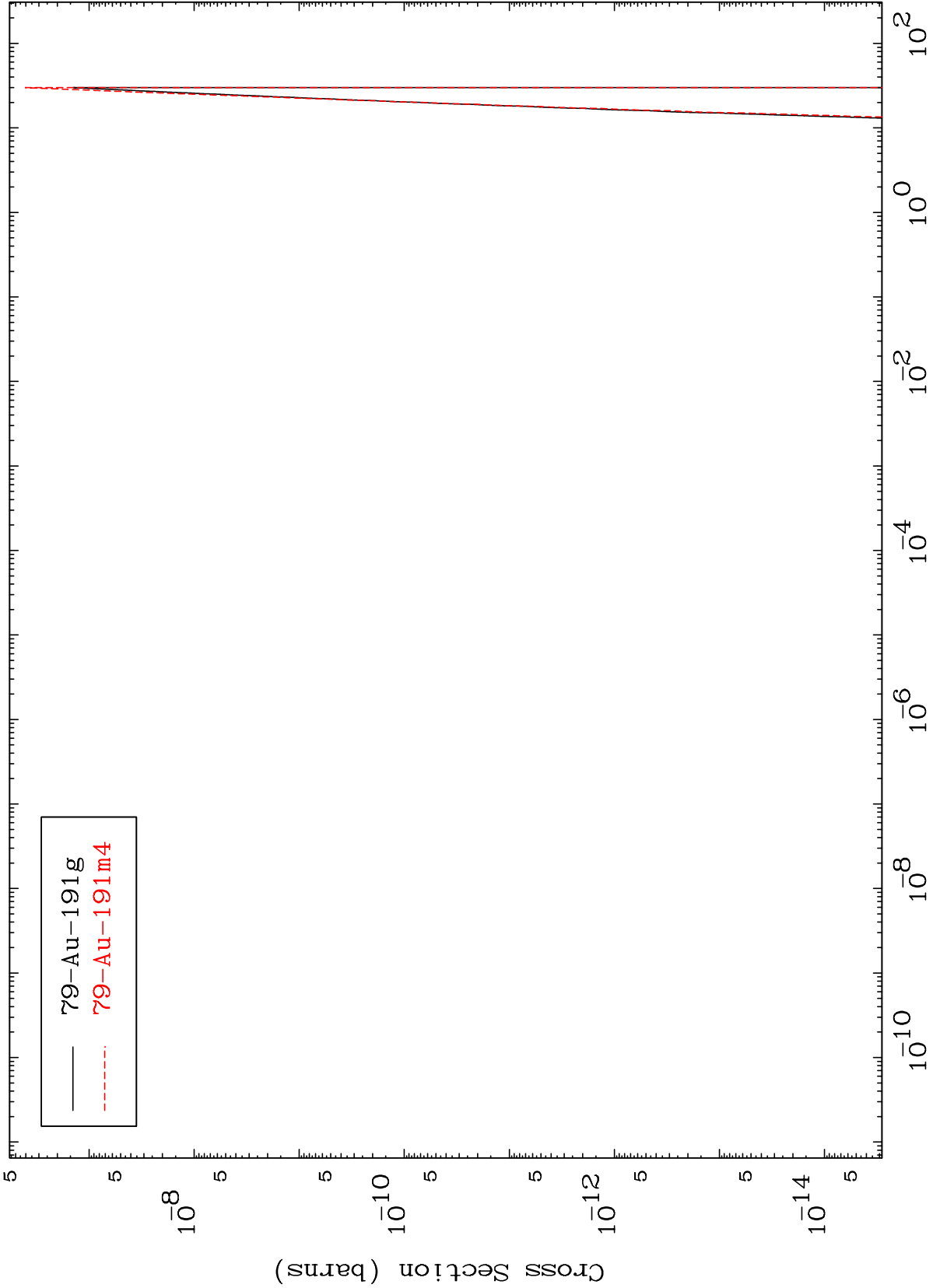


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(n,n') 2 α

83-Bi-199m

Radionuclide Production Cross Section



26

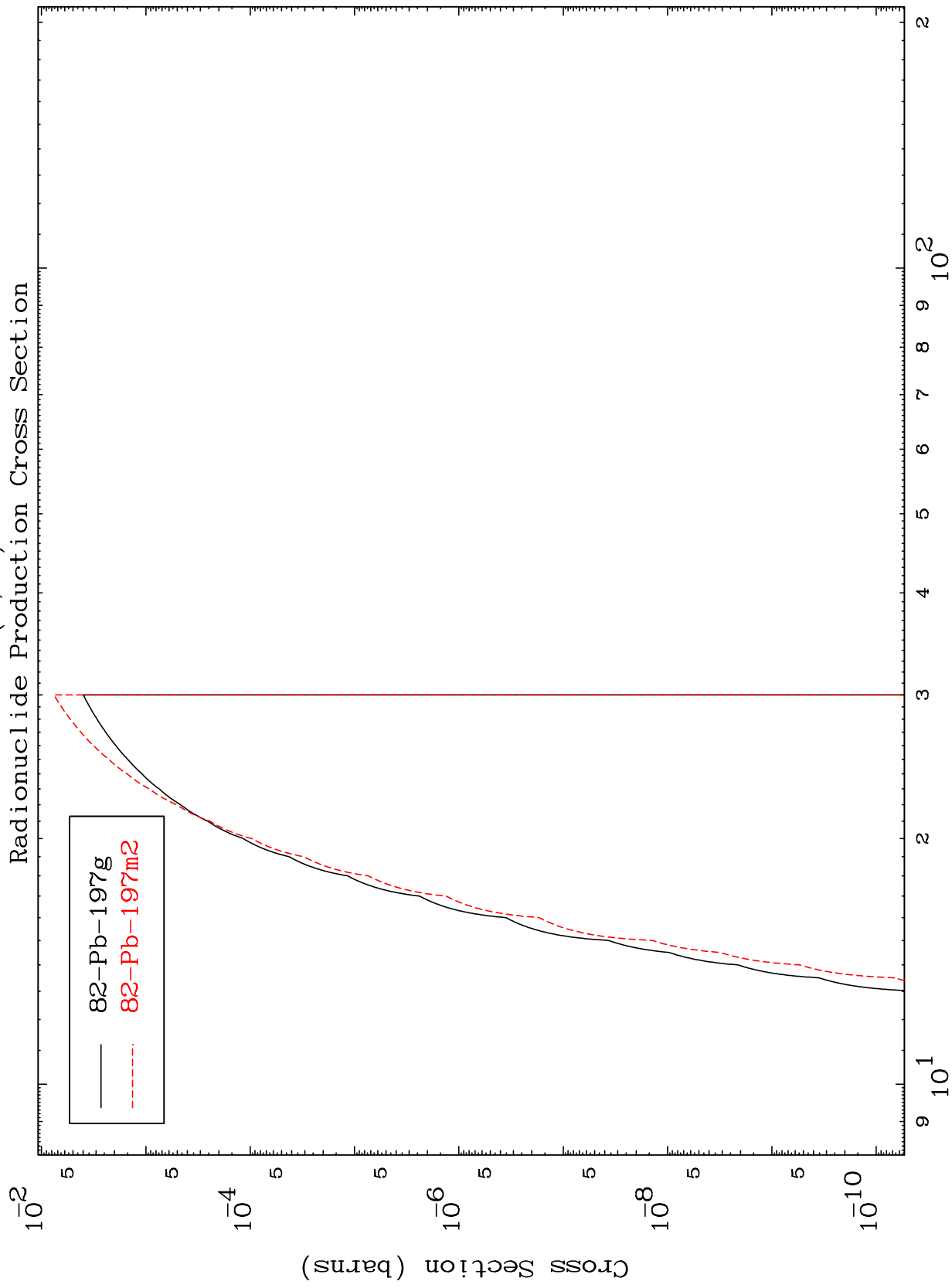
Incident Energy (MeV)

83-Bi-199m

MAT 8296

(n,n') d

83-Bi-199m



27

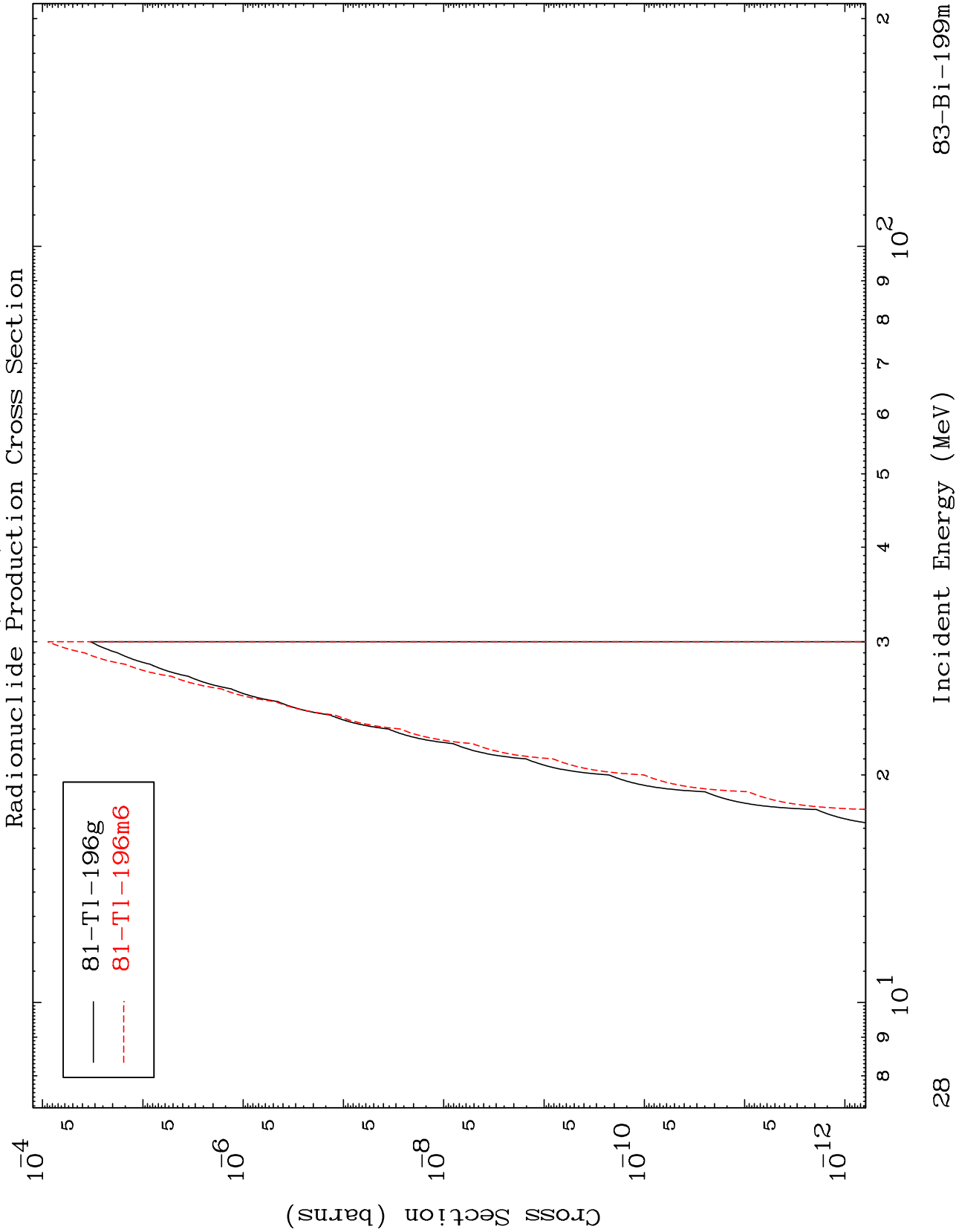
Incident Energy (MeV)

83-Bi-199m

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(n,n') He-3

83-Bi-199m



28

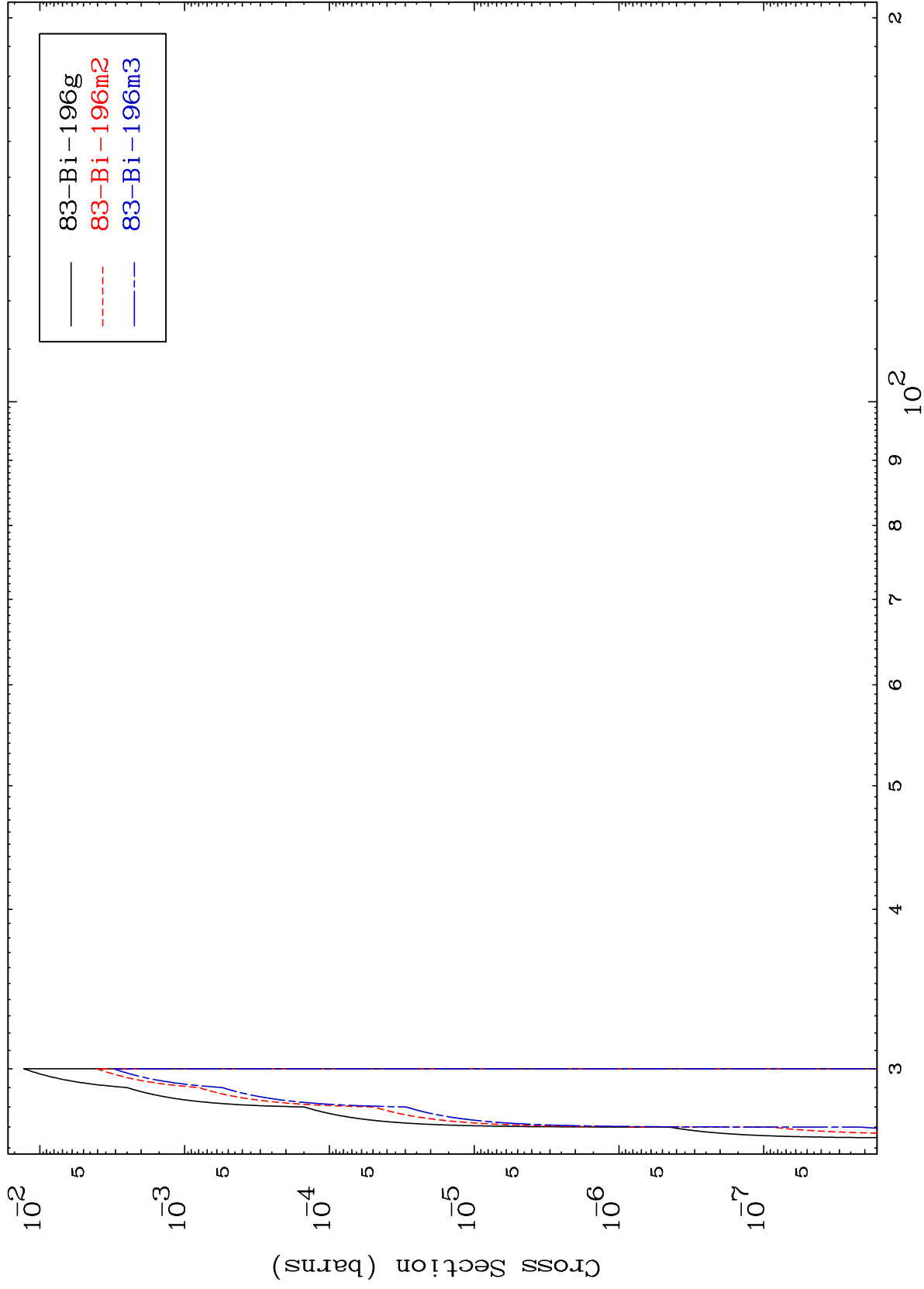
Incident Energy (MeV)

83-Bi-199m

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83-Bi-199m

(n,4n)
Radionuclide Production Cross Section



83-Bi-199m

Incident Energy (MeV)

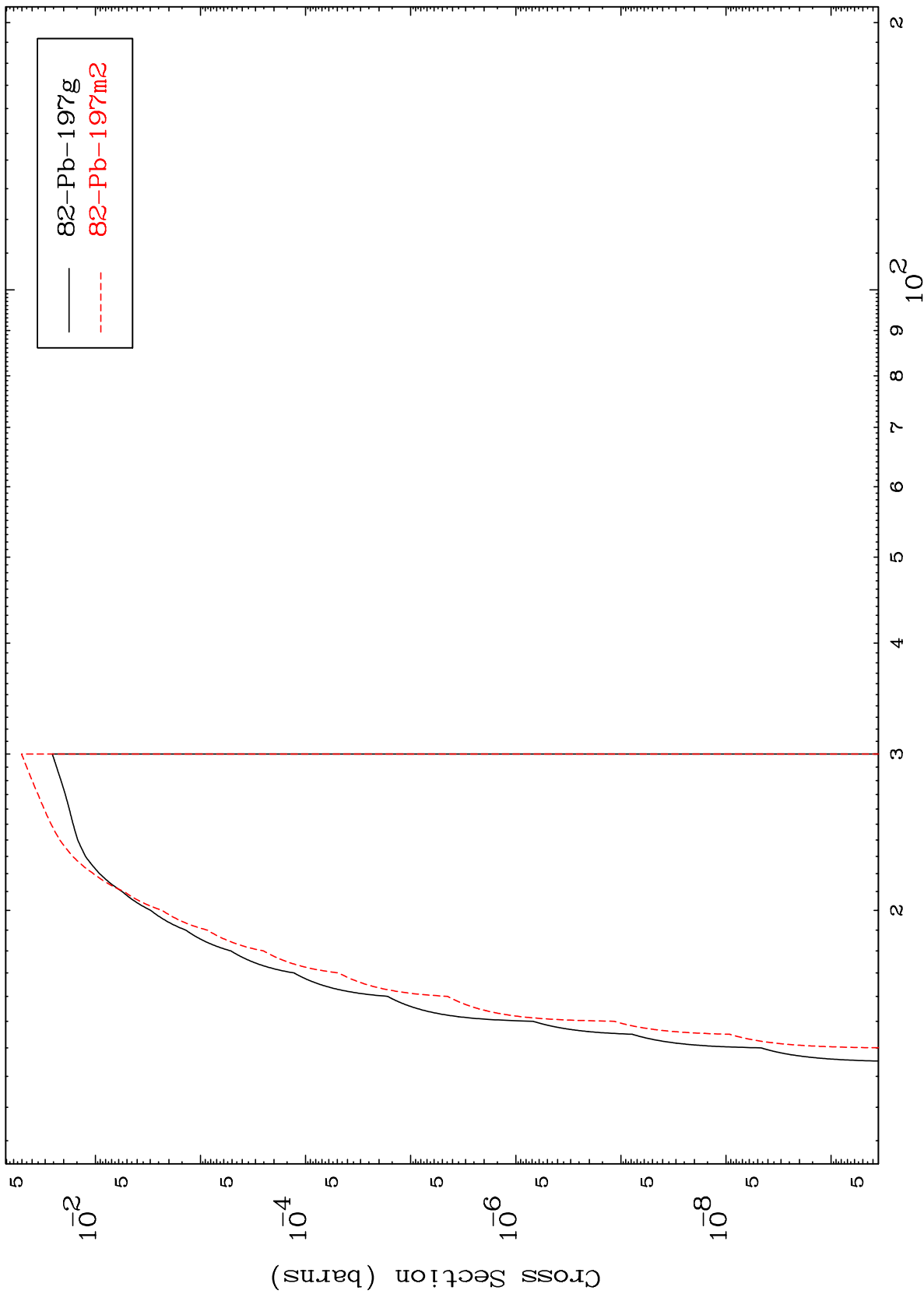
29

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(n,2n) p

83-Bi-199m

Radionuclide Production Cross Section



30

Incident Energy (MeV)

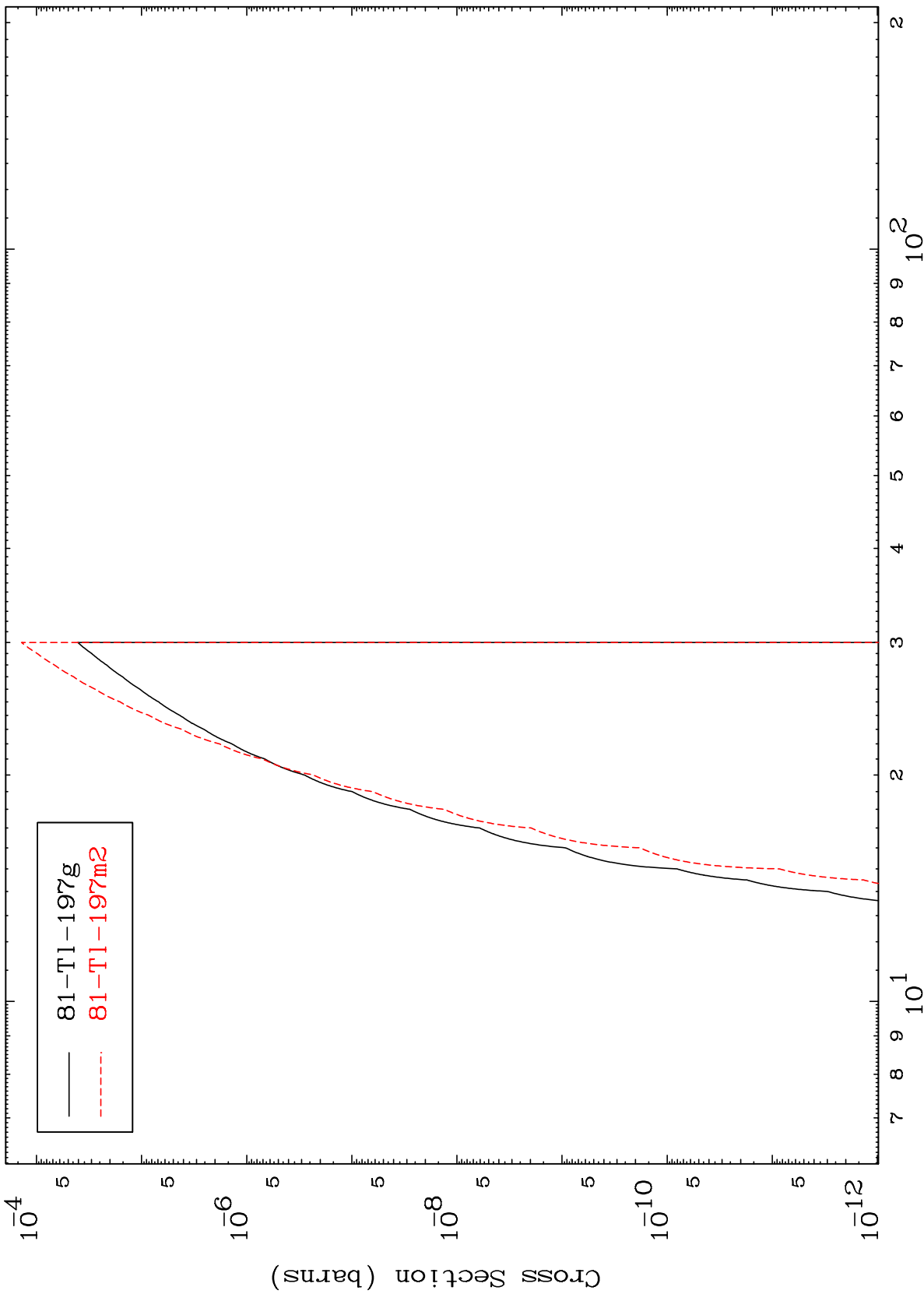
83-Bi-199m

MAT 8296

(n,2n) p

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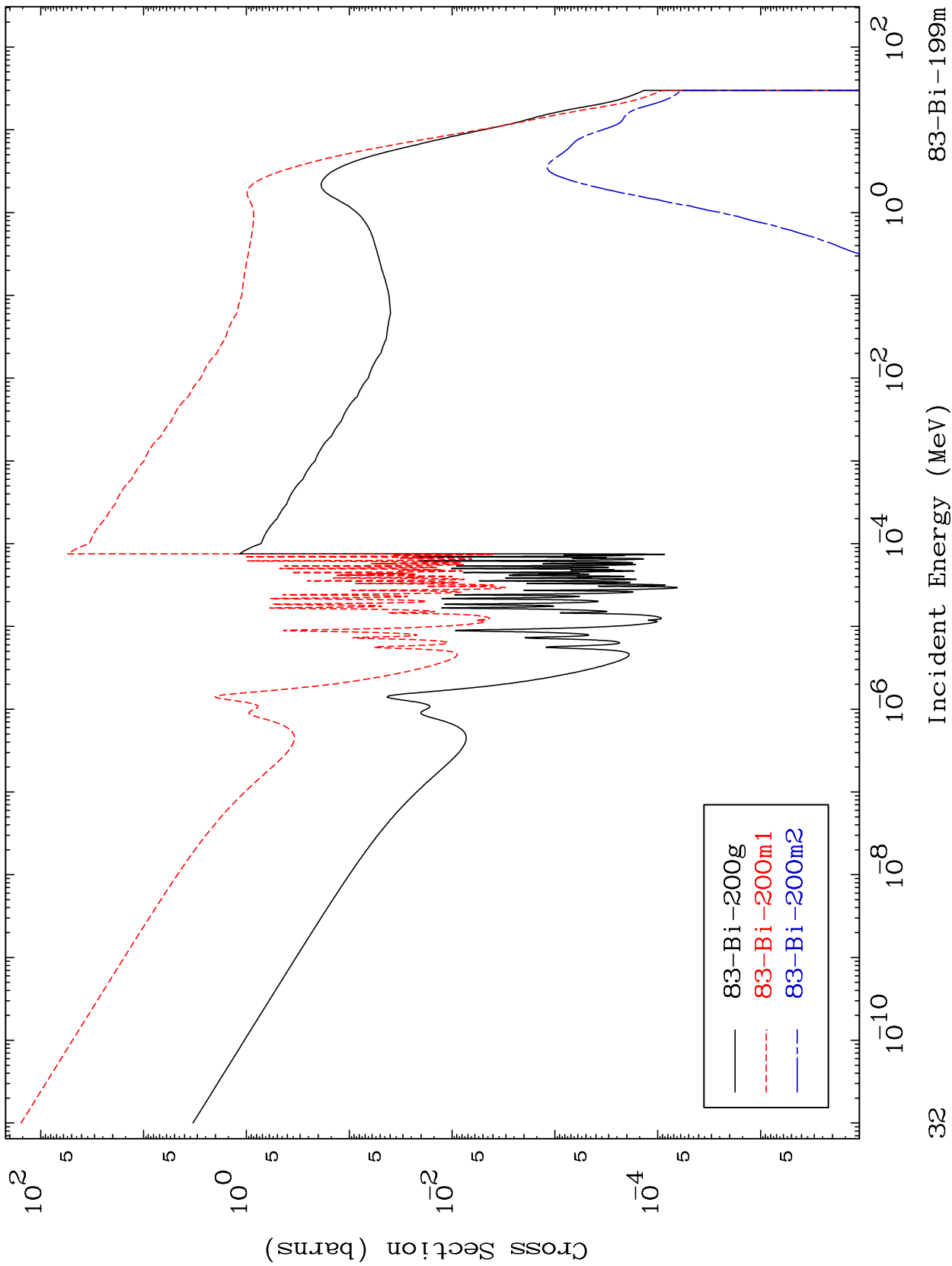
Radionuclide Production Cross Section



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83-Bi-199m

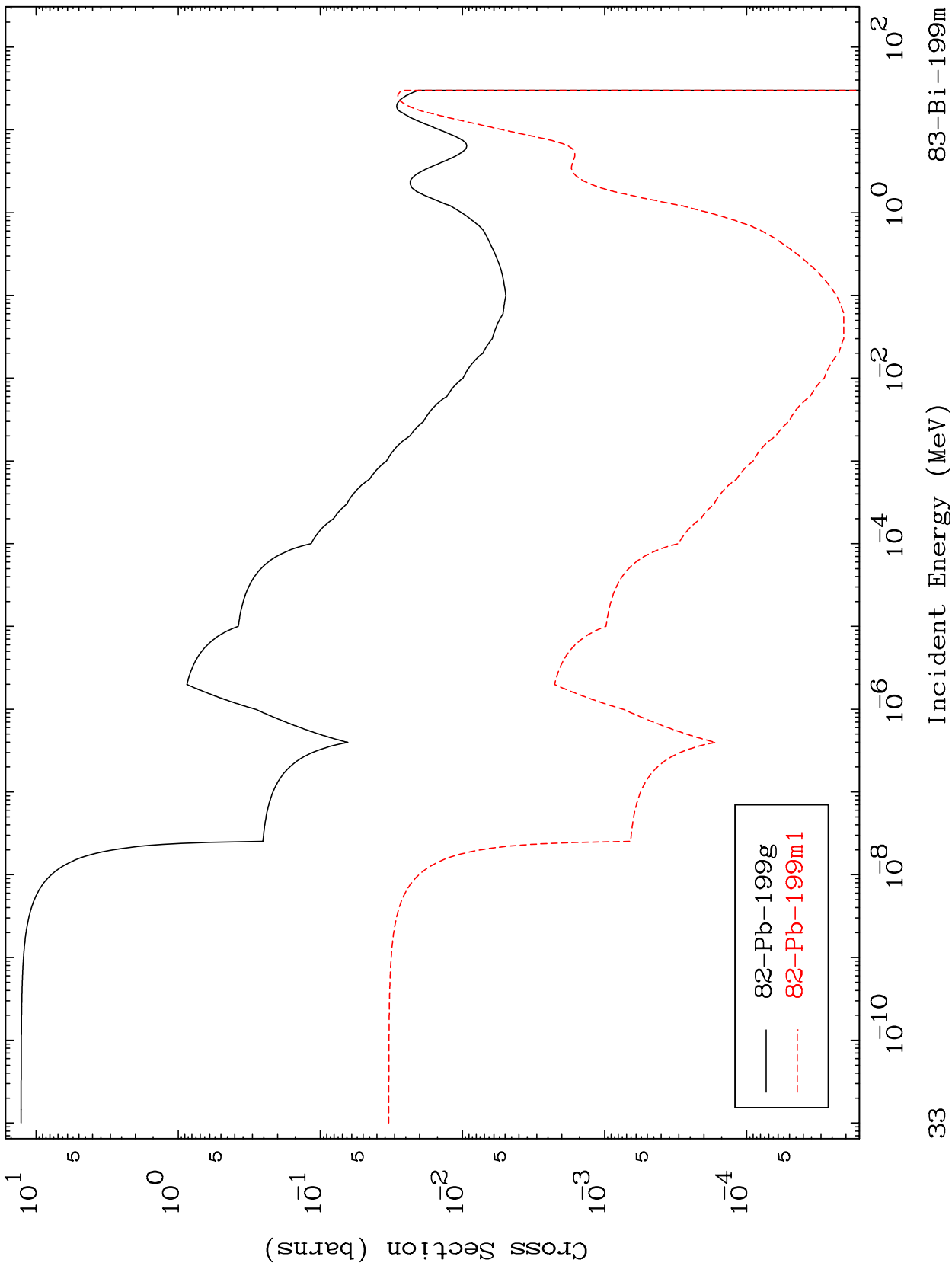
Radionuclide Production Cross Section
(n, γ)



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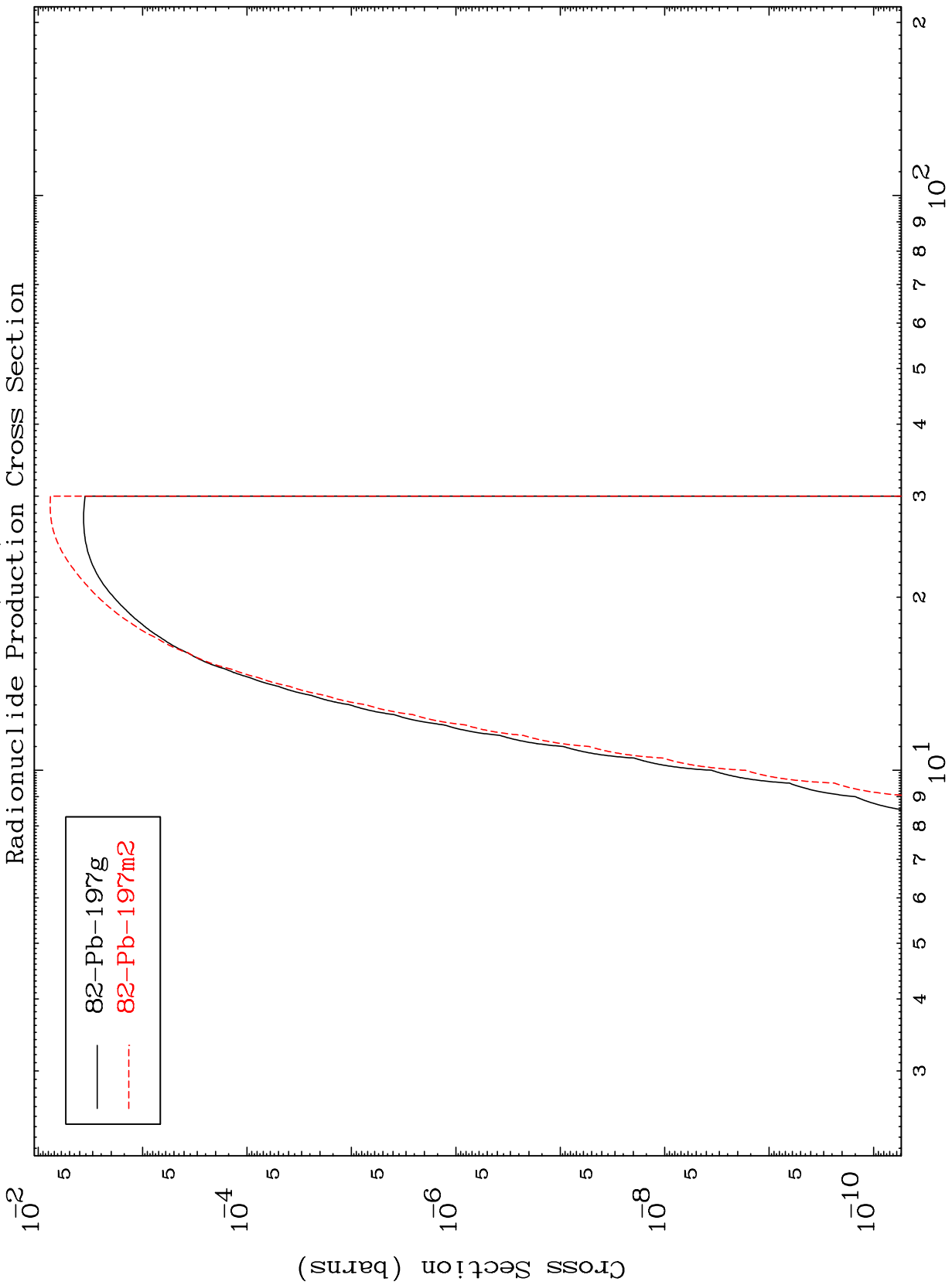
83-Bi-199m

Radionuclide Production Cross Section



MAT 8296

83-Bi-199m

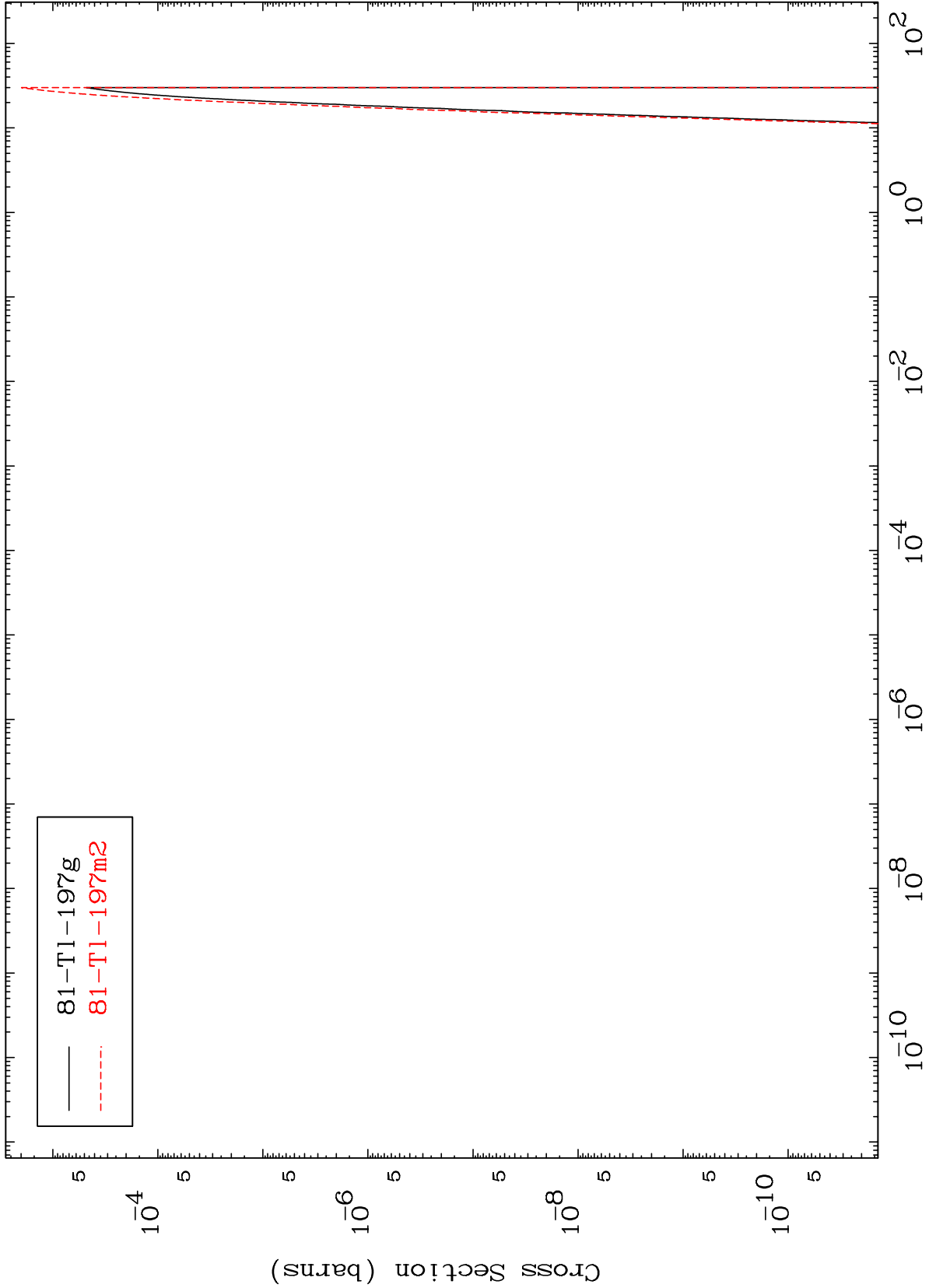


MAT 8296

(n,He-3)

83-Bi-199m

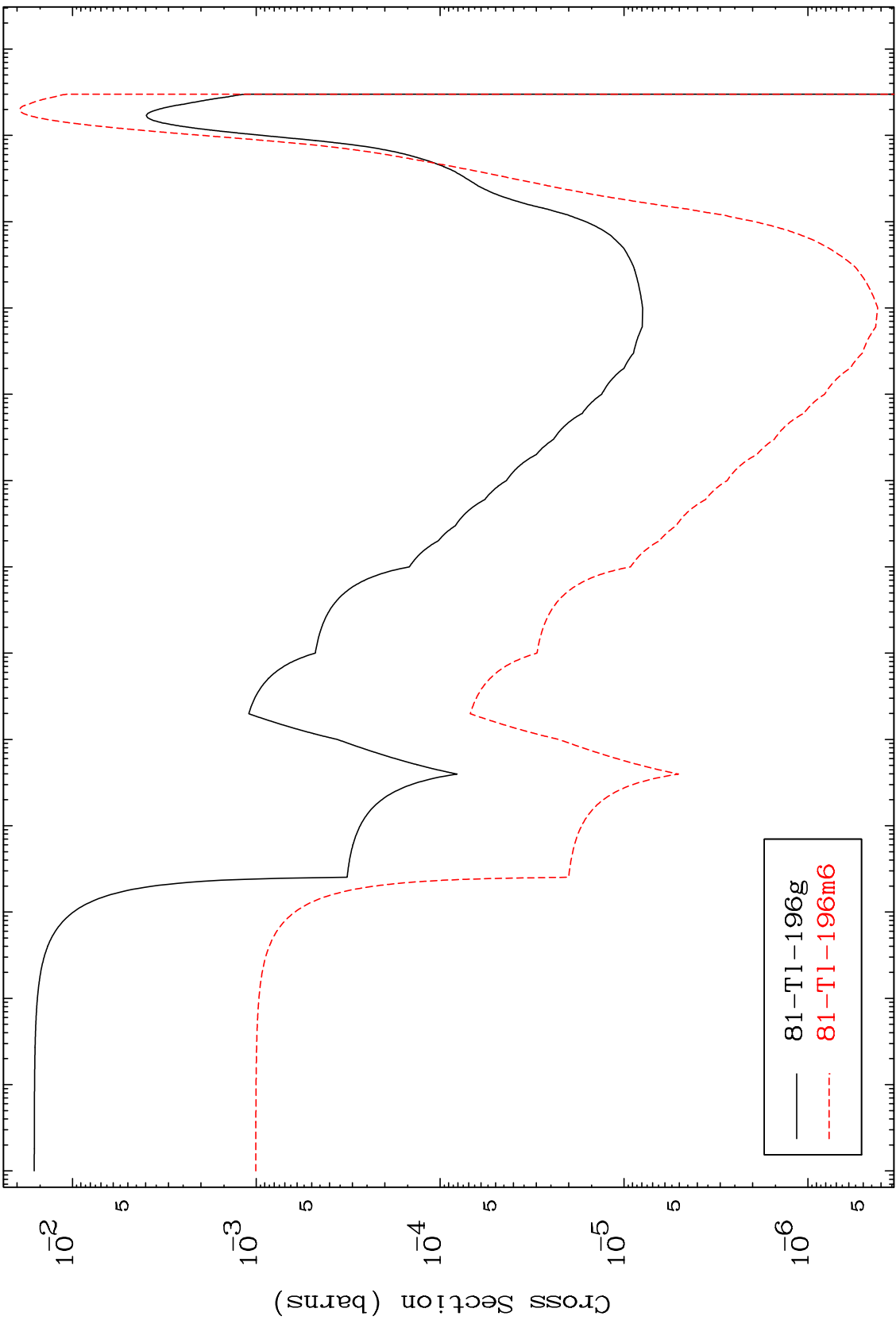
Radionuclide Production Cross Section



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83-Bi-199m

Radionuclide Production Cross Section
(n, α)



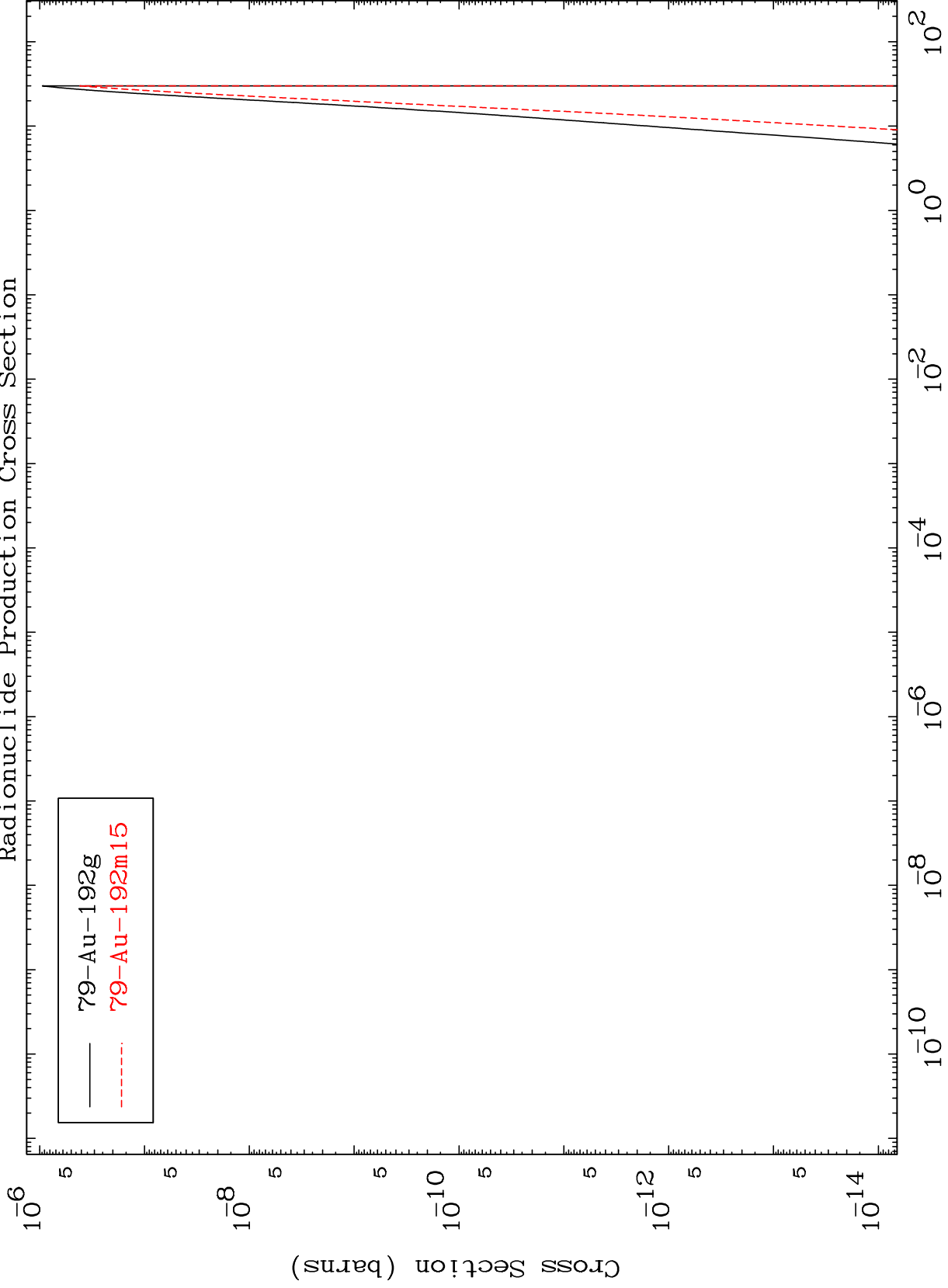
— 81-Tl-196g
- - - 81-Tl-196m6

MAT 8296

(n,2α)

83-Bi-199m

Radionuclide Production Cross Section



37

Incident Energy (MeV)

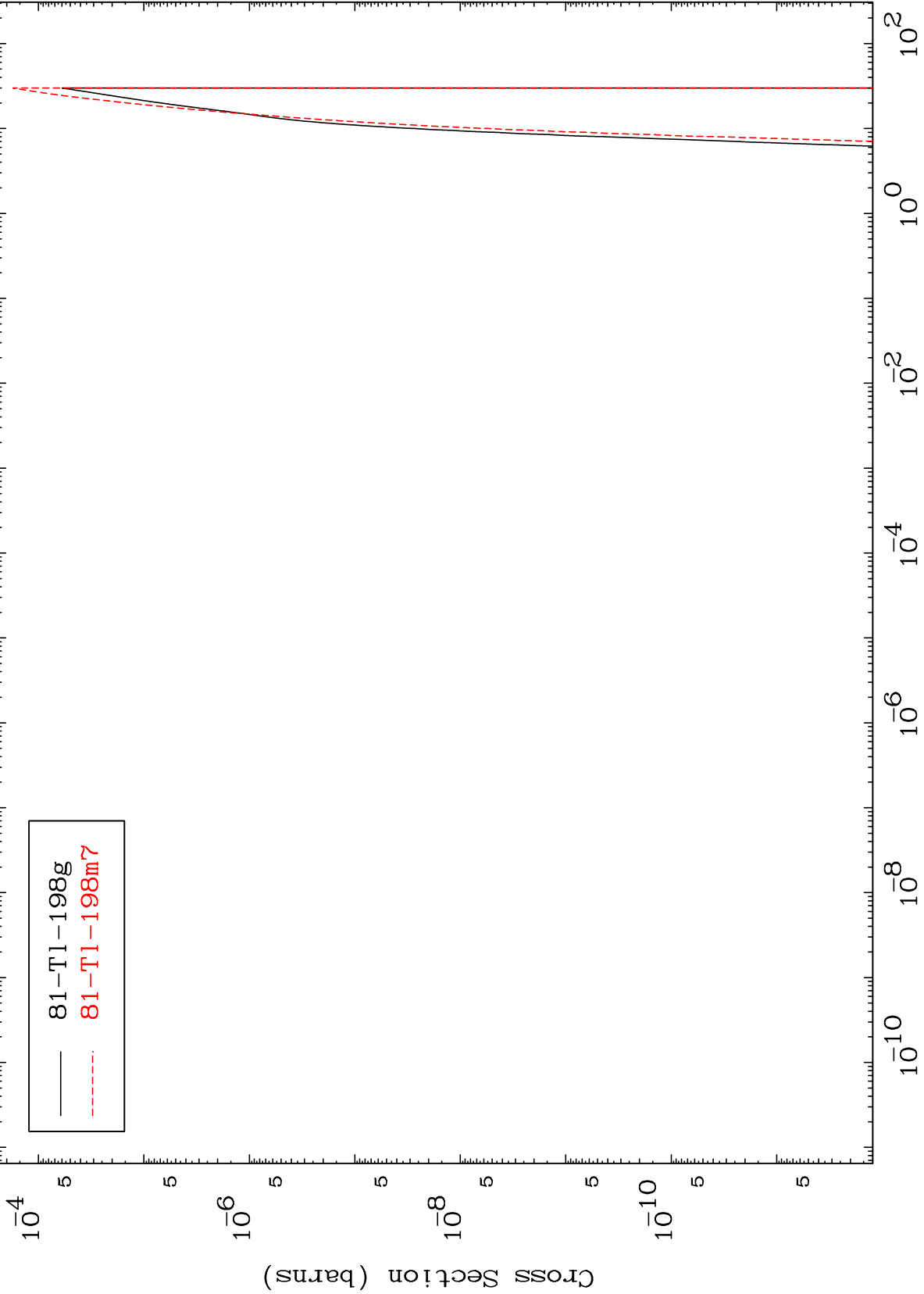
83-Bi-199m

MAT 8296

(n,2p)

83-Bi-199m

Radionuclide Production Cross Section

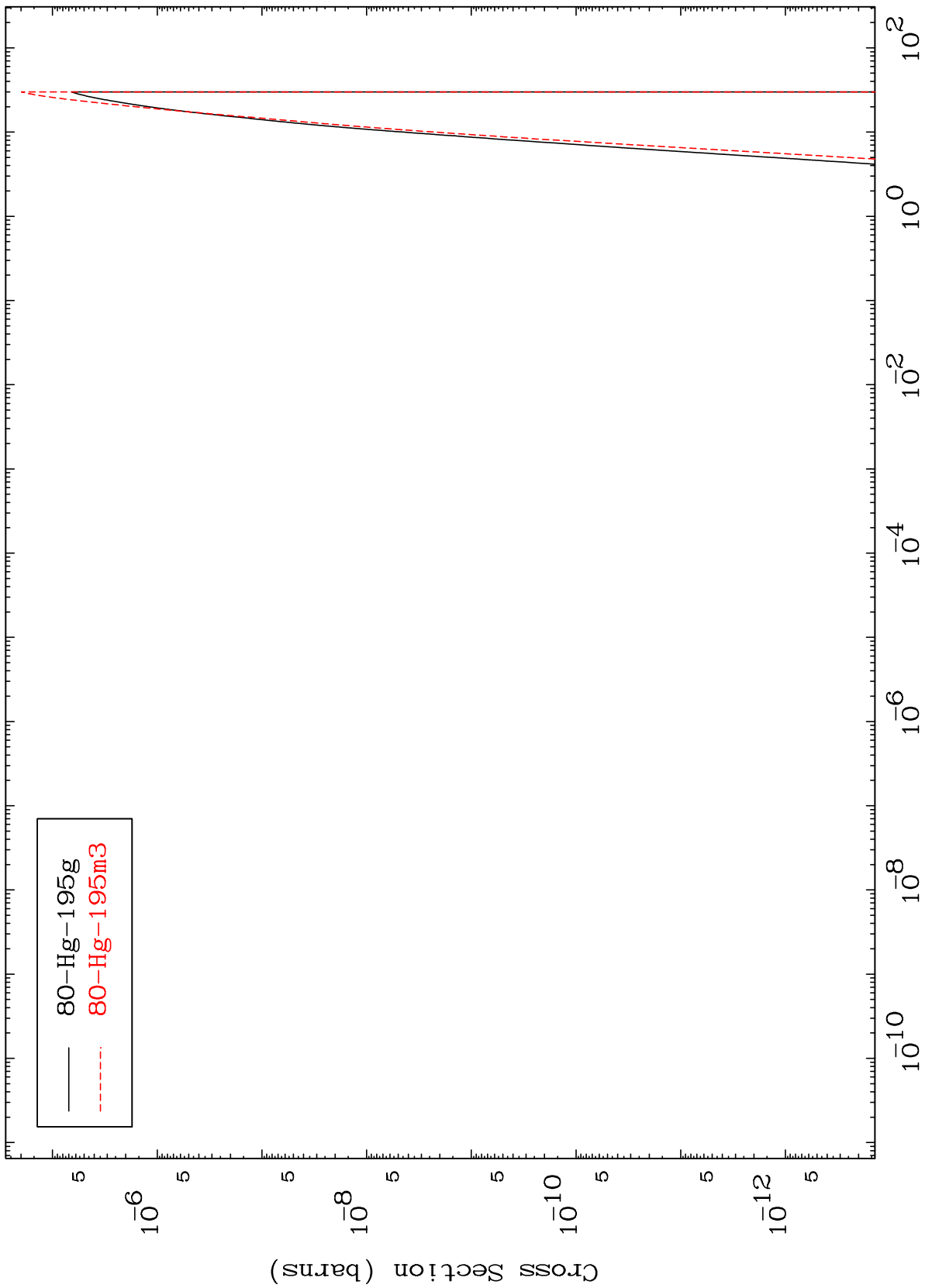


MAT 8296

(n,p) α

83-Bi-199m

Radionuclide Production Cross Section

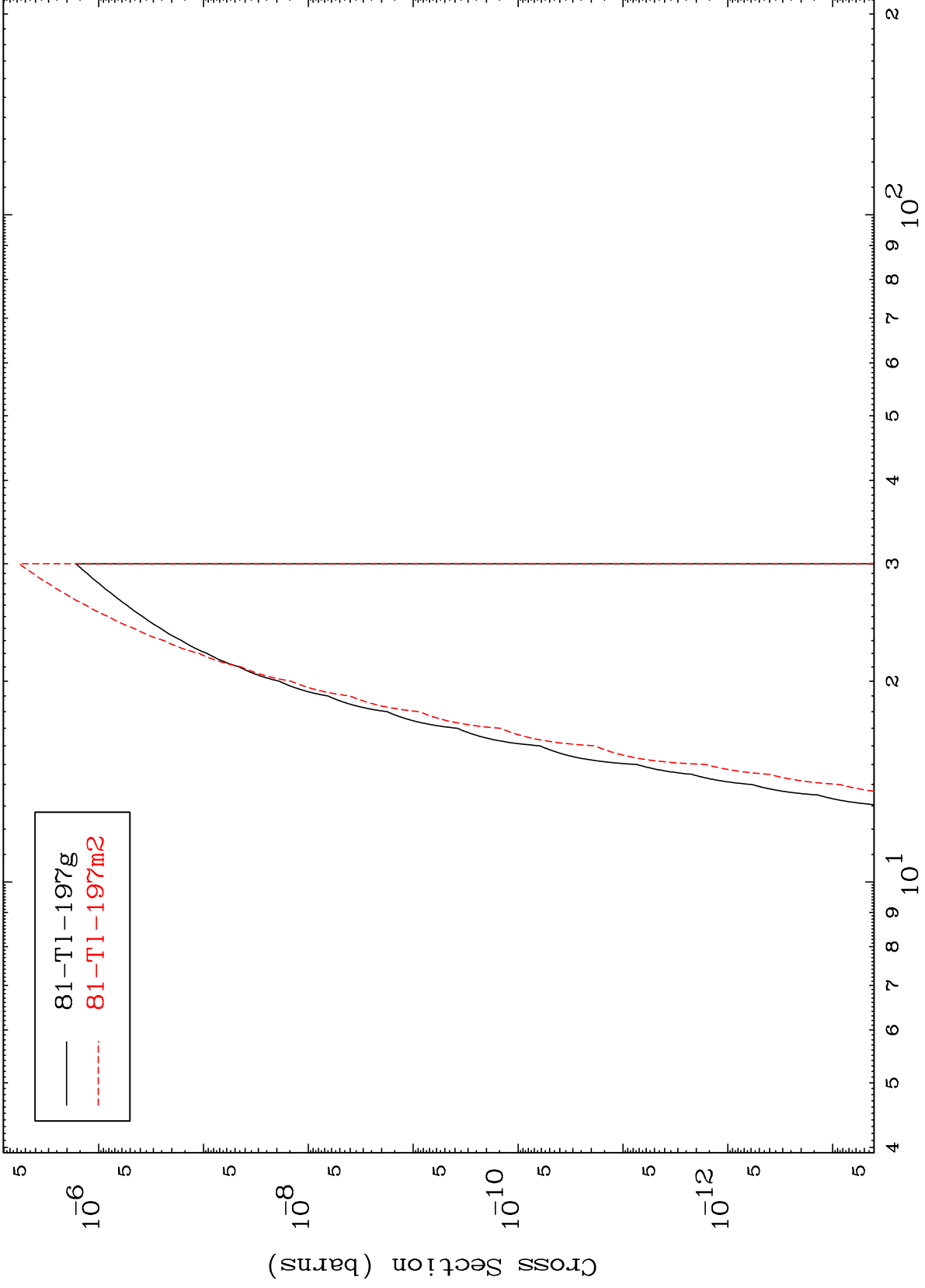


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(n,p) d

83-Bi-199m

Radionuclide Production Cross Section



81-Tl-197g
81-Tl-197m2

40

Incident Energy (MeV)

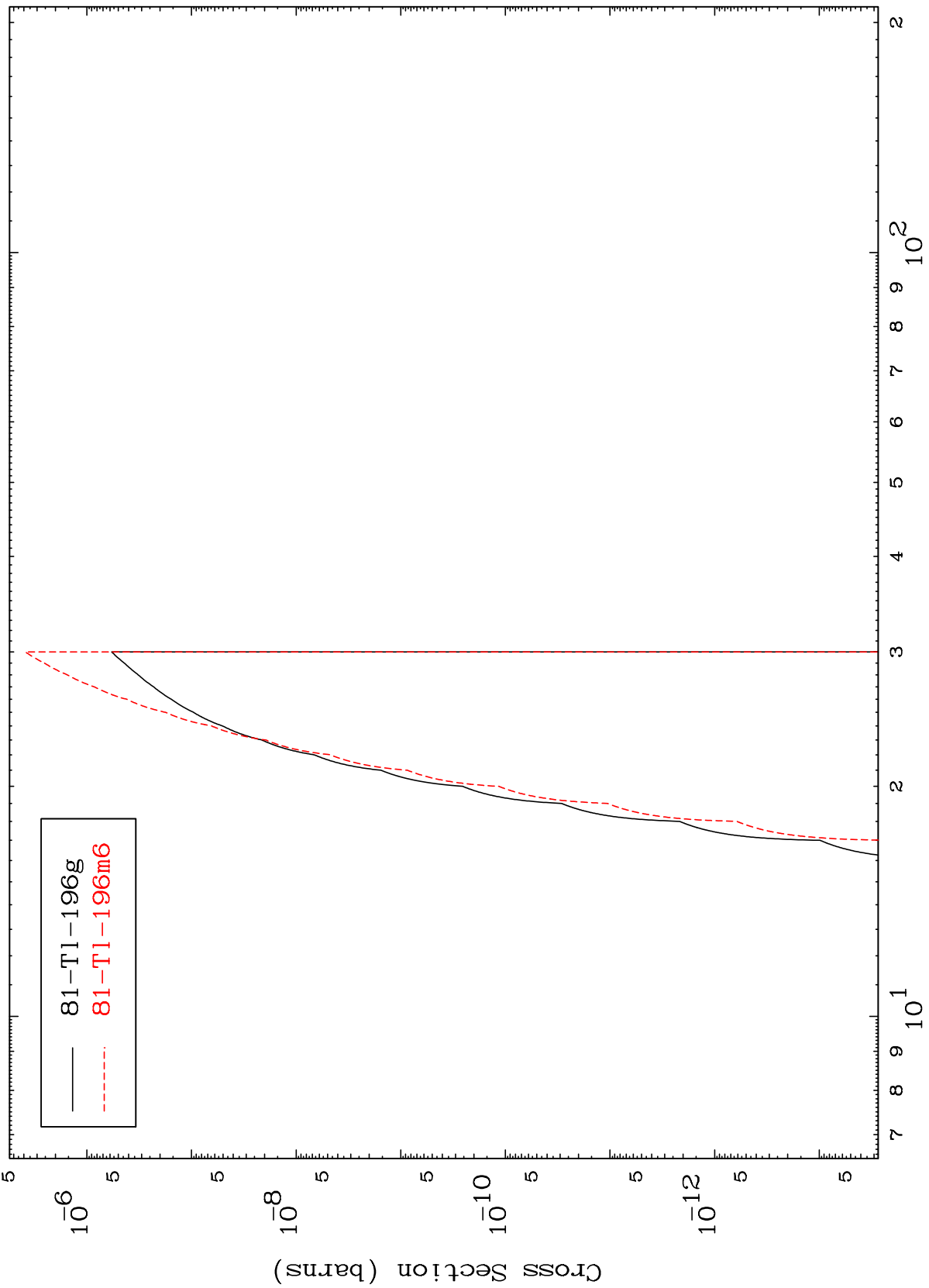
83-Bi-199m

MAT 8296

(n,p) t

83-Bi-199m

Radionuclide Production Cross Section



41

Incident Energy (MeV)

83-Bi-199m