

Program EVALPLOT
(Version 2021-1)

by

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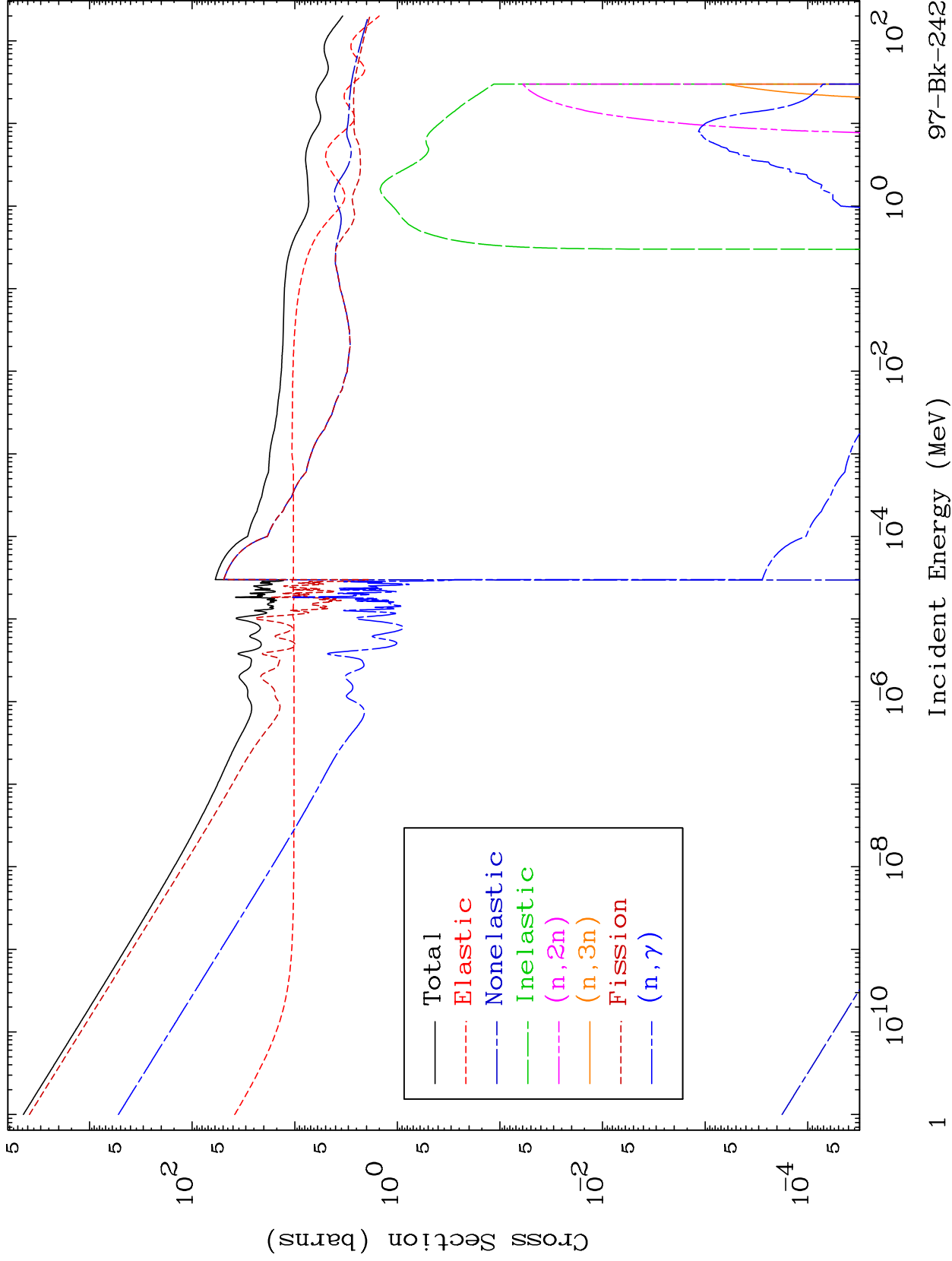
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Press Mouse Button to Start

MAT 9731

Neutron Major
293 Kelvin Cross Sections

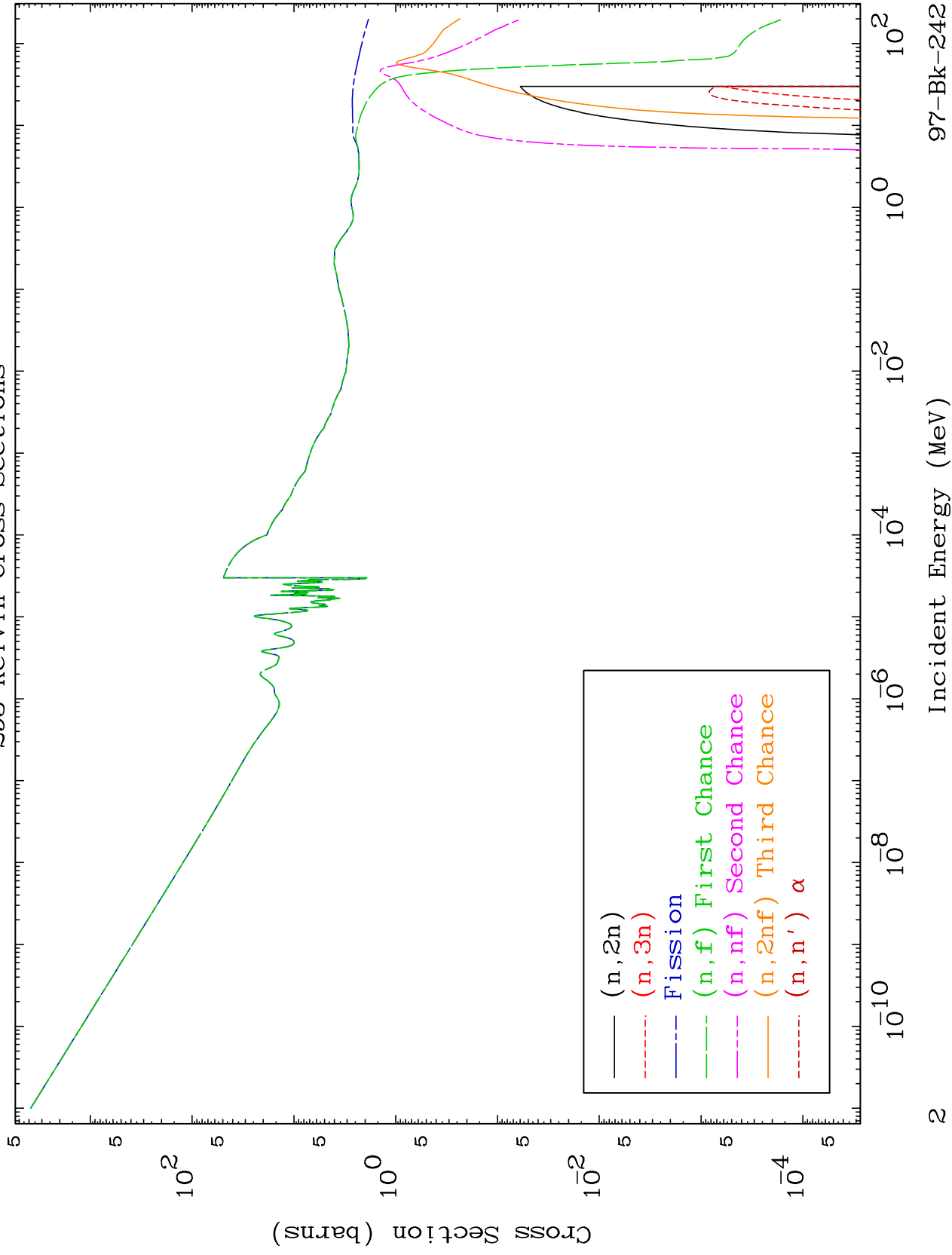
97-Bk-242



MAT 9731

Neutron Absorption
293 Kelvin Cross Sections

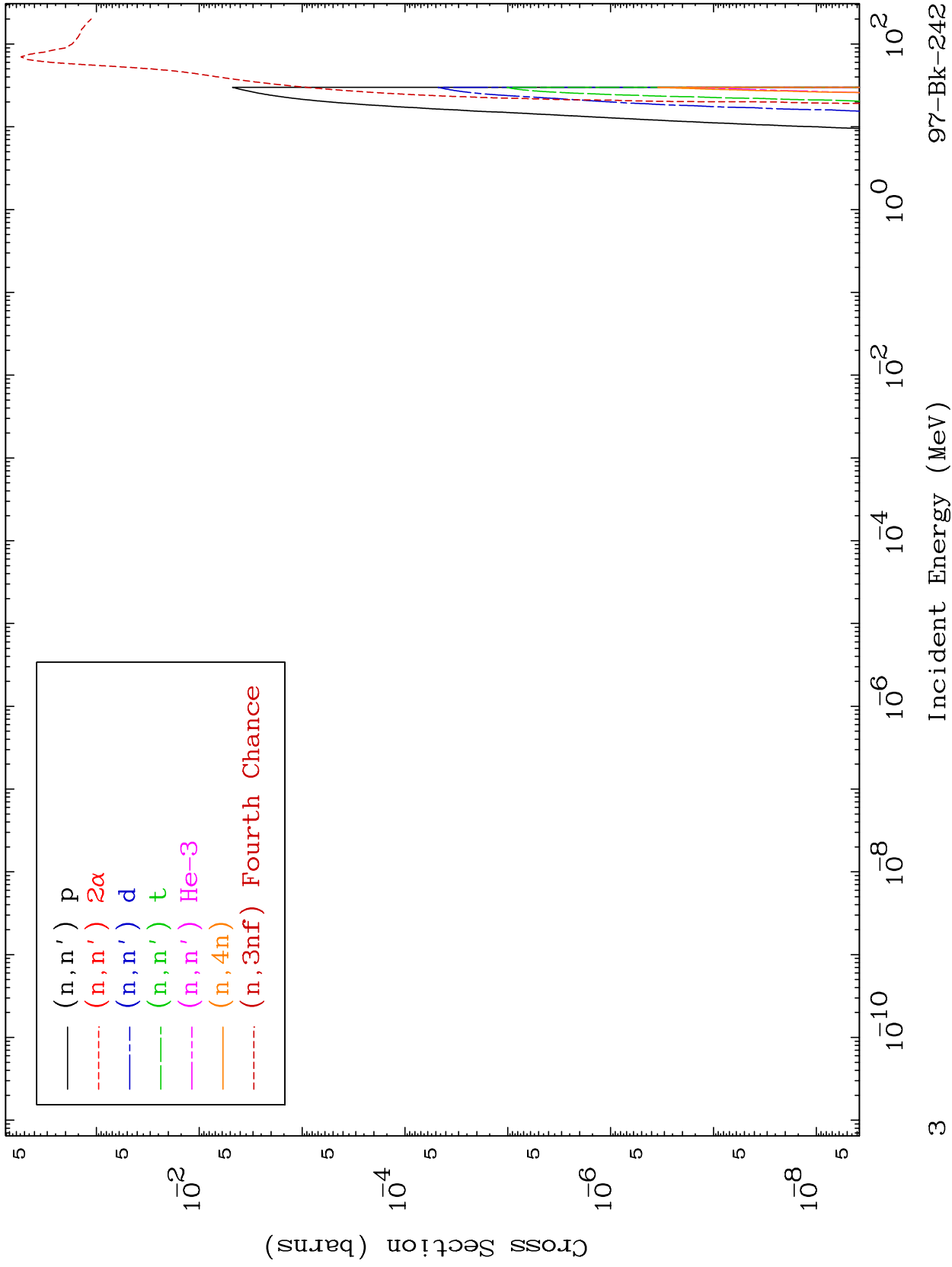
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MAT 9731

Neutron Absorption
293 Kelvin Cross Sections

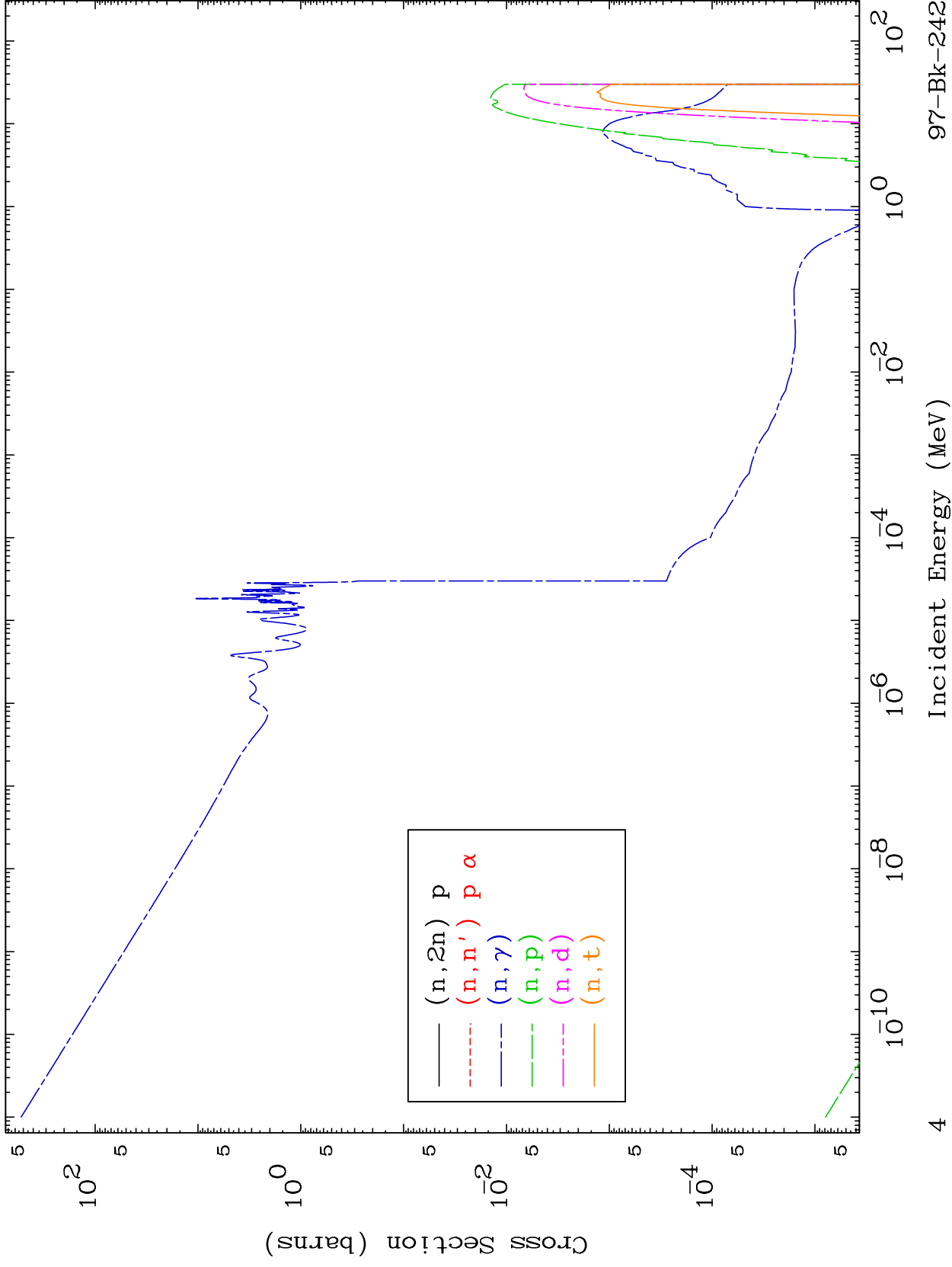
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Neutron Absorption
293 Kelvin Cross Sections

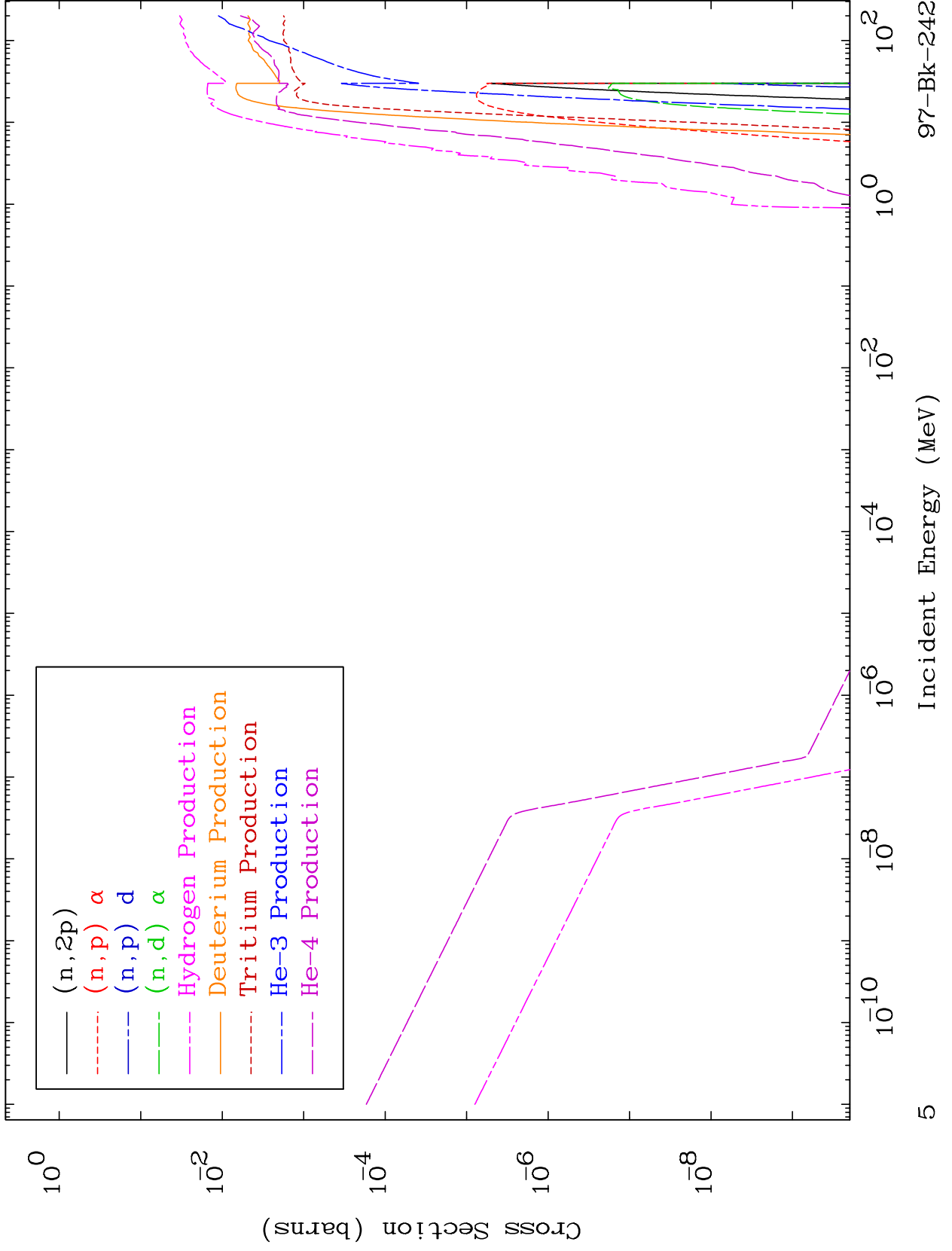
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Neutron Absorption
293 Kelvin Cross Sections

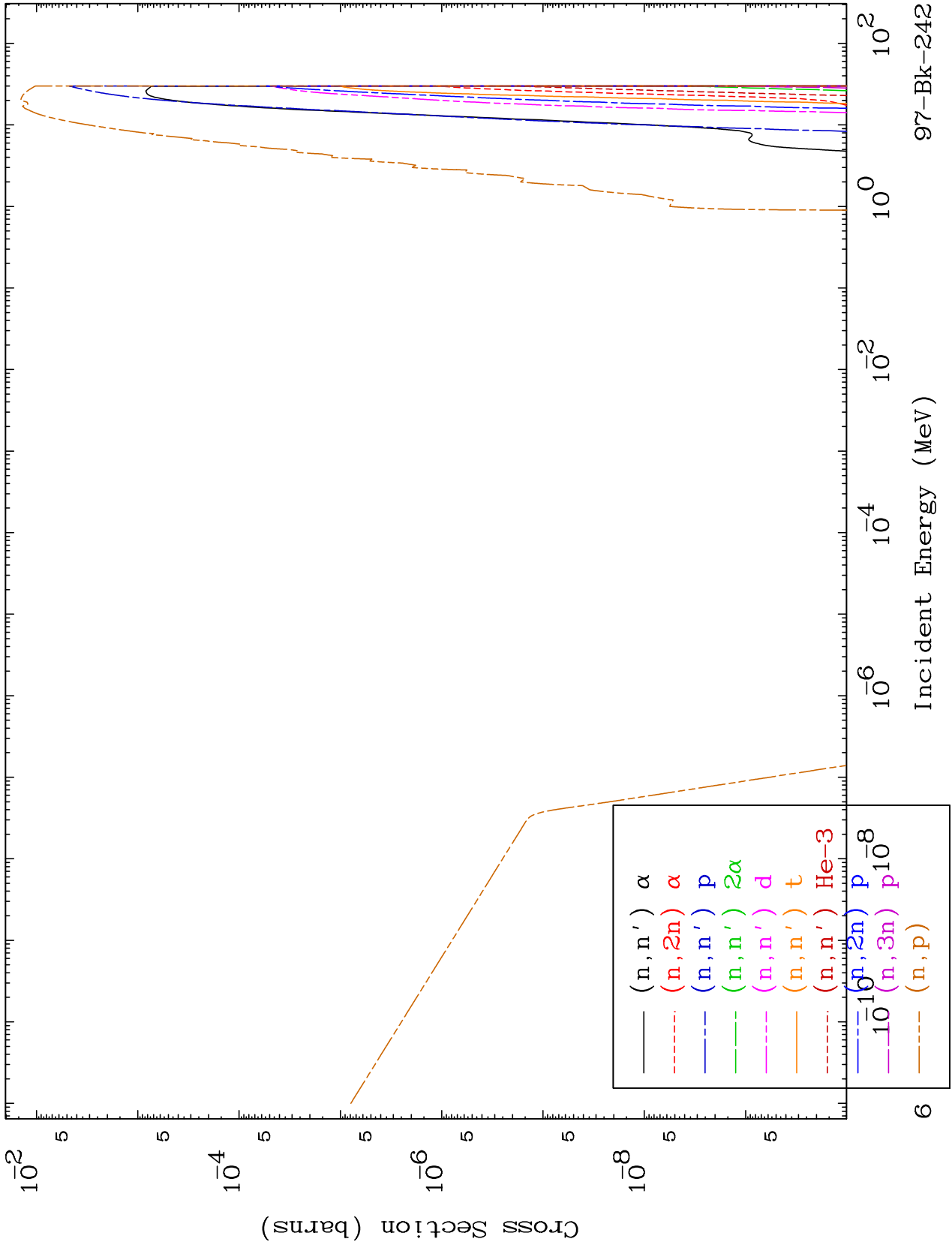
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MAT 9731

Charged Particle
293 Kelvin Cross Sections

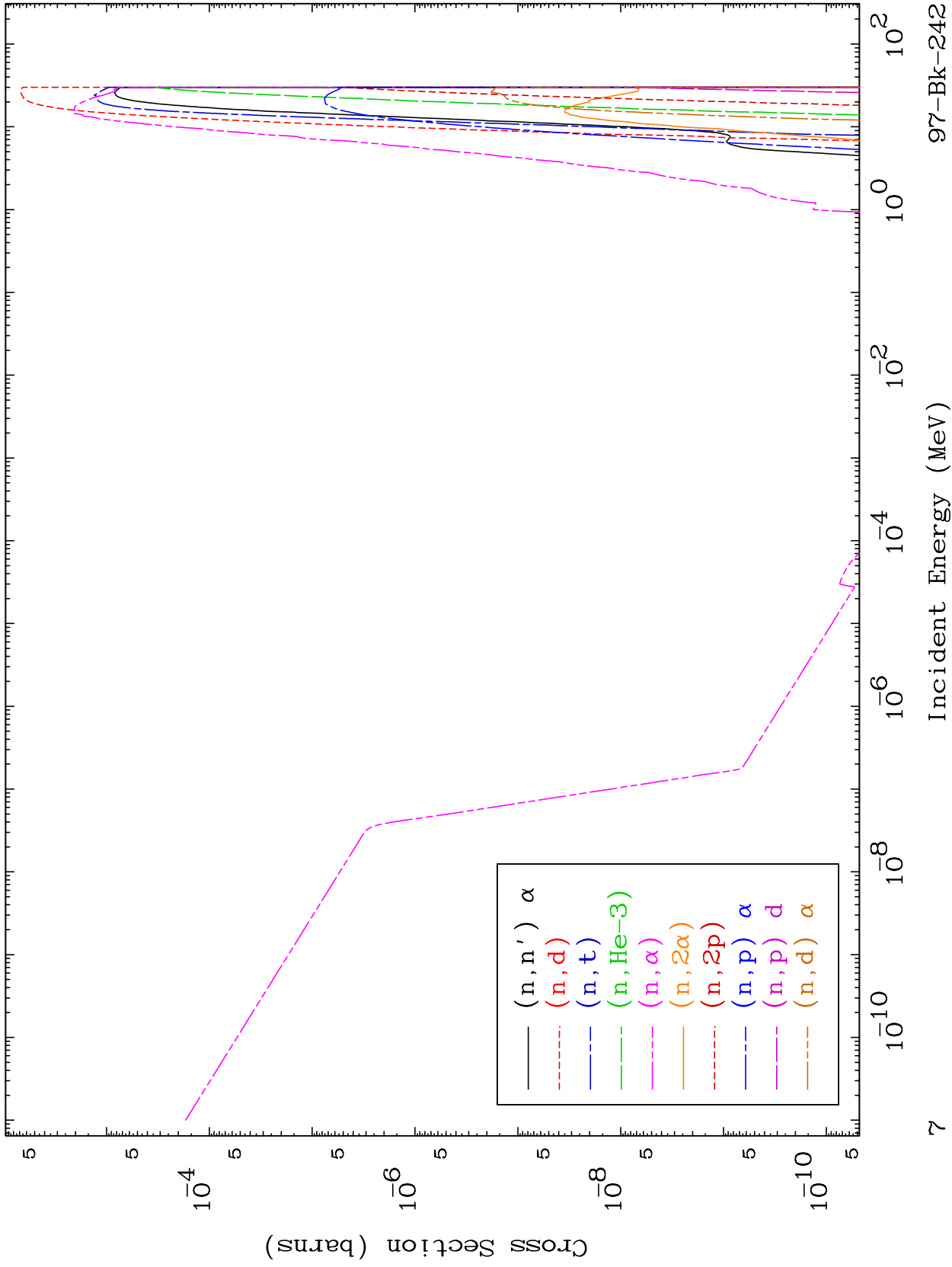
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Charged Particle
293 Kelvin Cross Sections

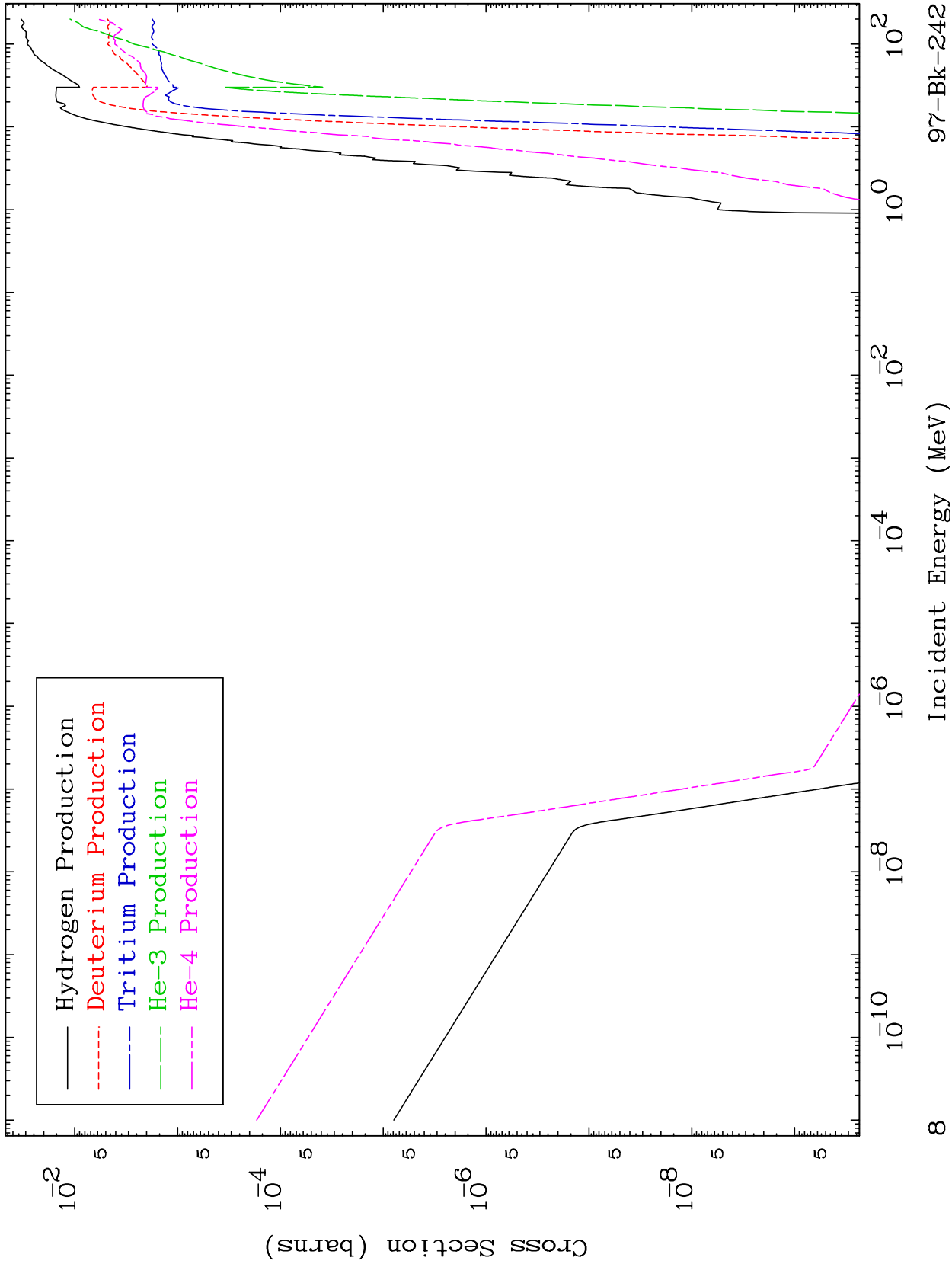
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Particle Production
293 Kelvin Cross Sections

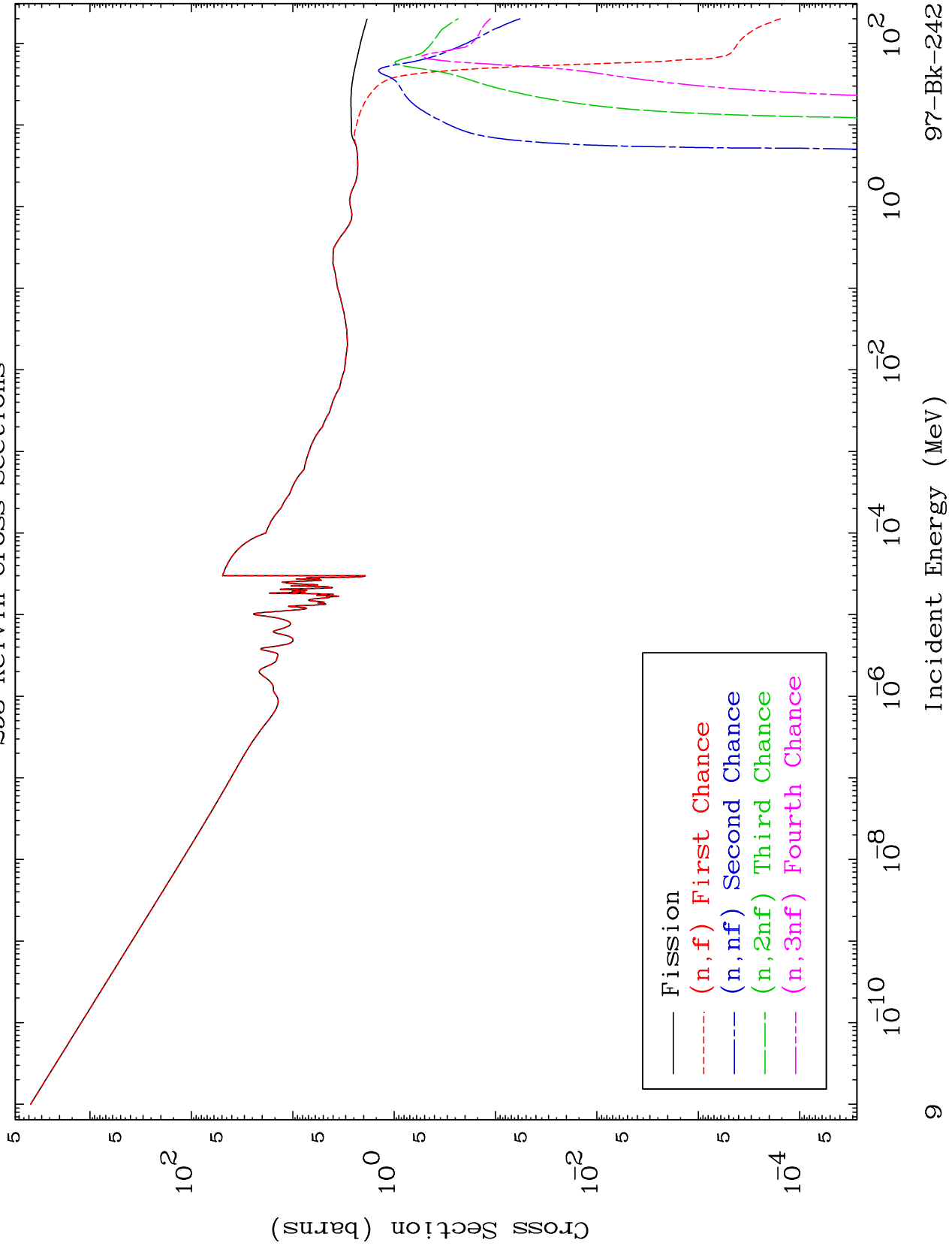
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Fission
293 Kelvin Cross Sections

97-Bk-242

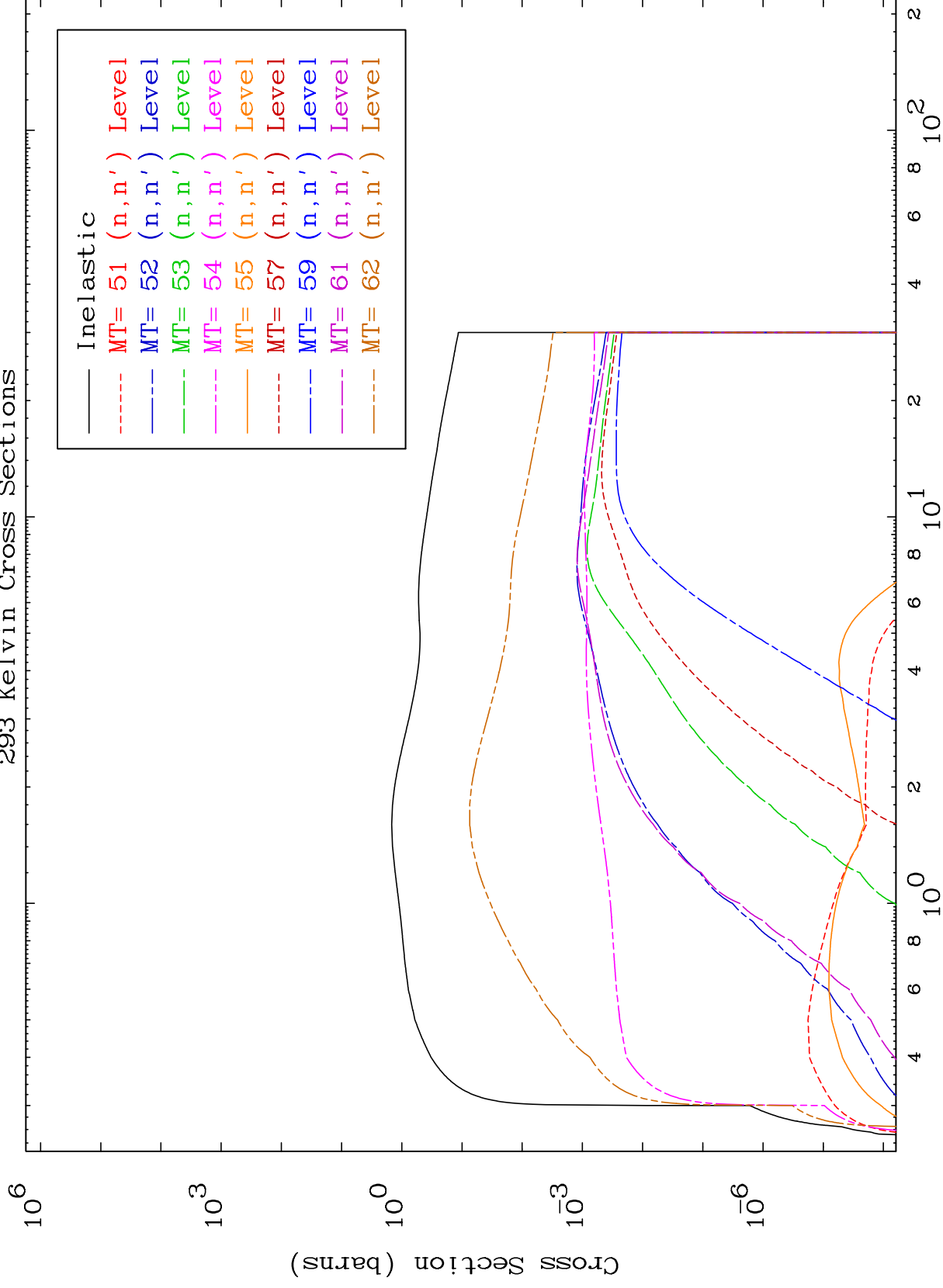


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(n,n') Levels

293 Kelvin Cross Sections

97-Bk-242



10

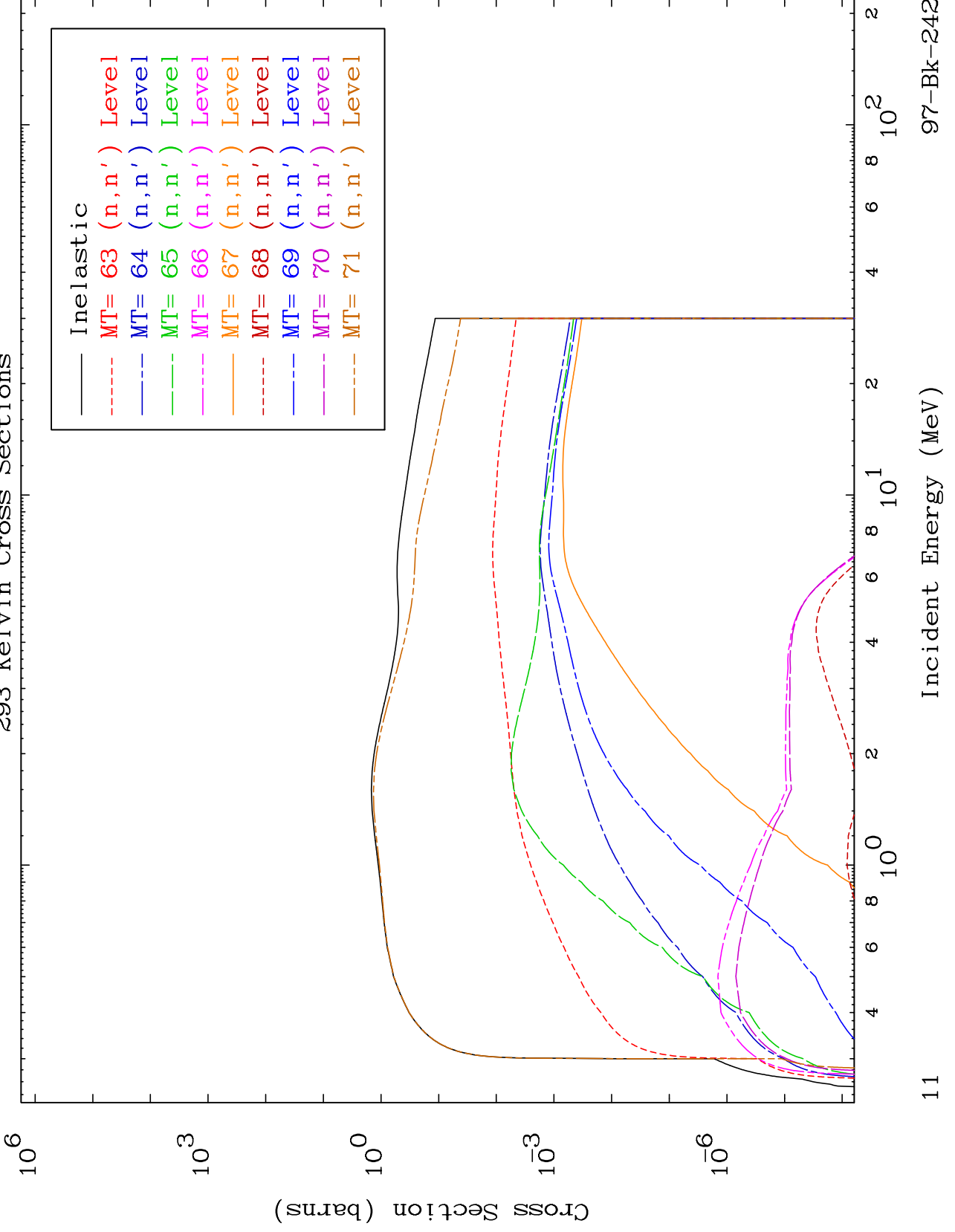
Incident Energy (MeV)

97-Bk-242

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(n,n') Levels
293 Kelvin Cross Sections

97-Bk-242

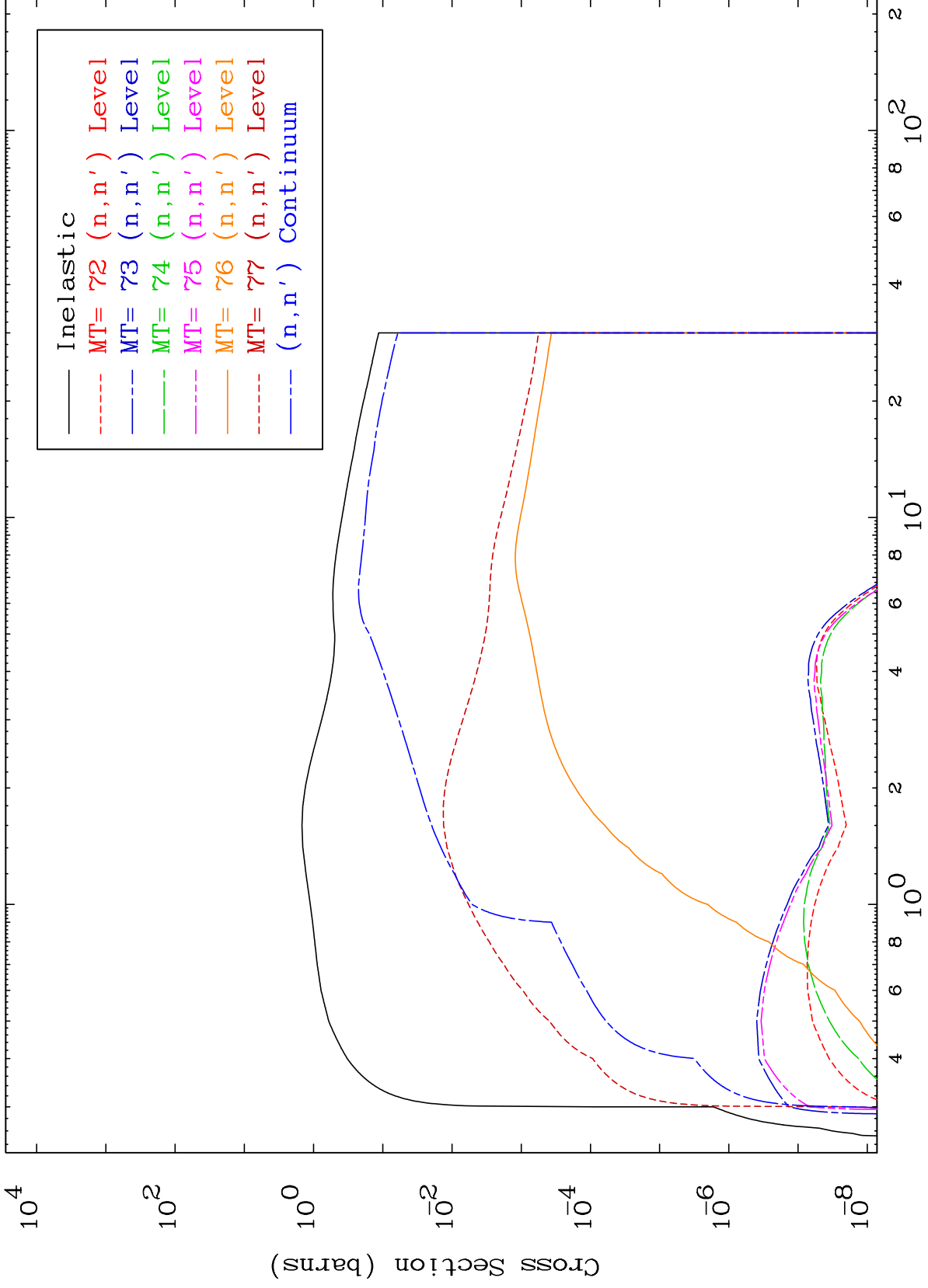


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(n,n') Levels

97-Bk-242

293 Kelvin Cross Sections



12

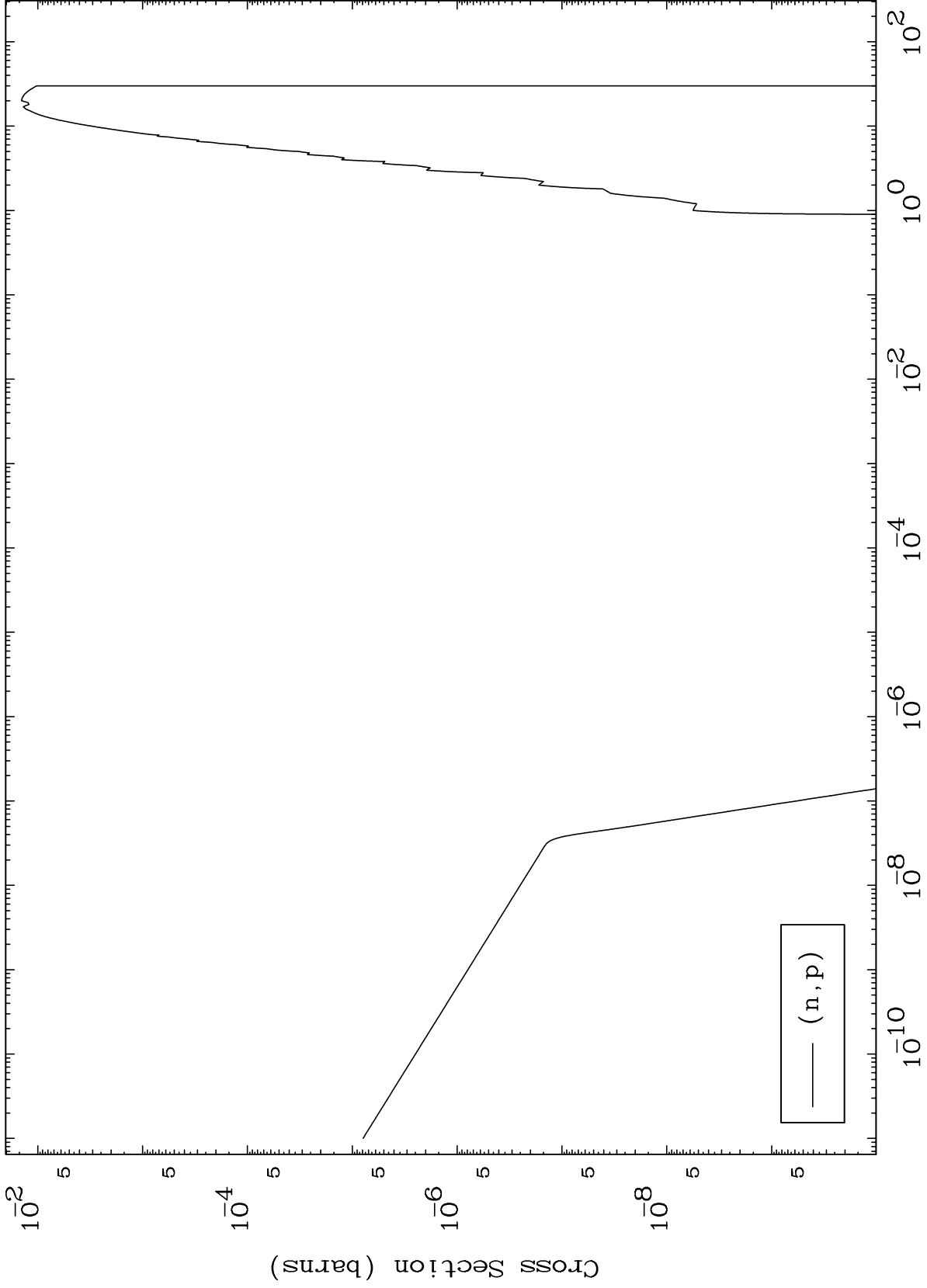
Incident Energy (MeV)

97-Bk-242

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(n,p) Levels
293 Kelvin Cross Sections

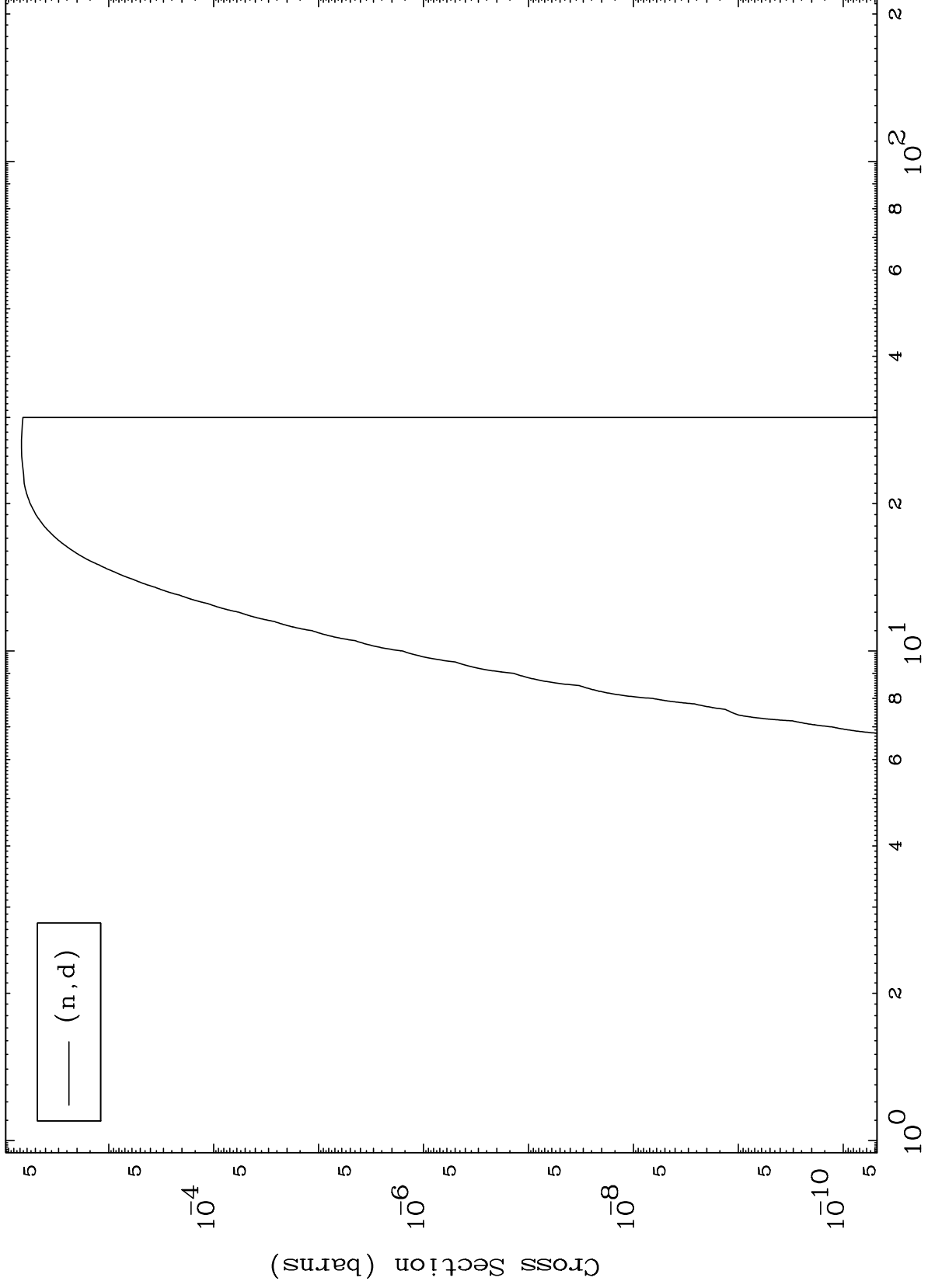
97-Bk-242



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(n,d) Levels
293 Kelvin Cross Sections

97-Bk-242



14

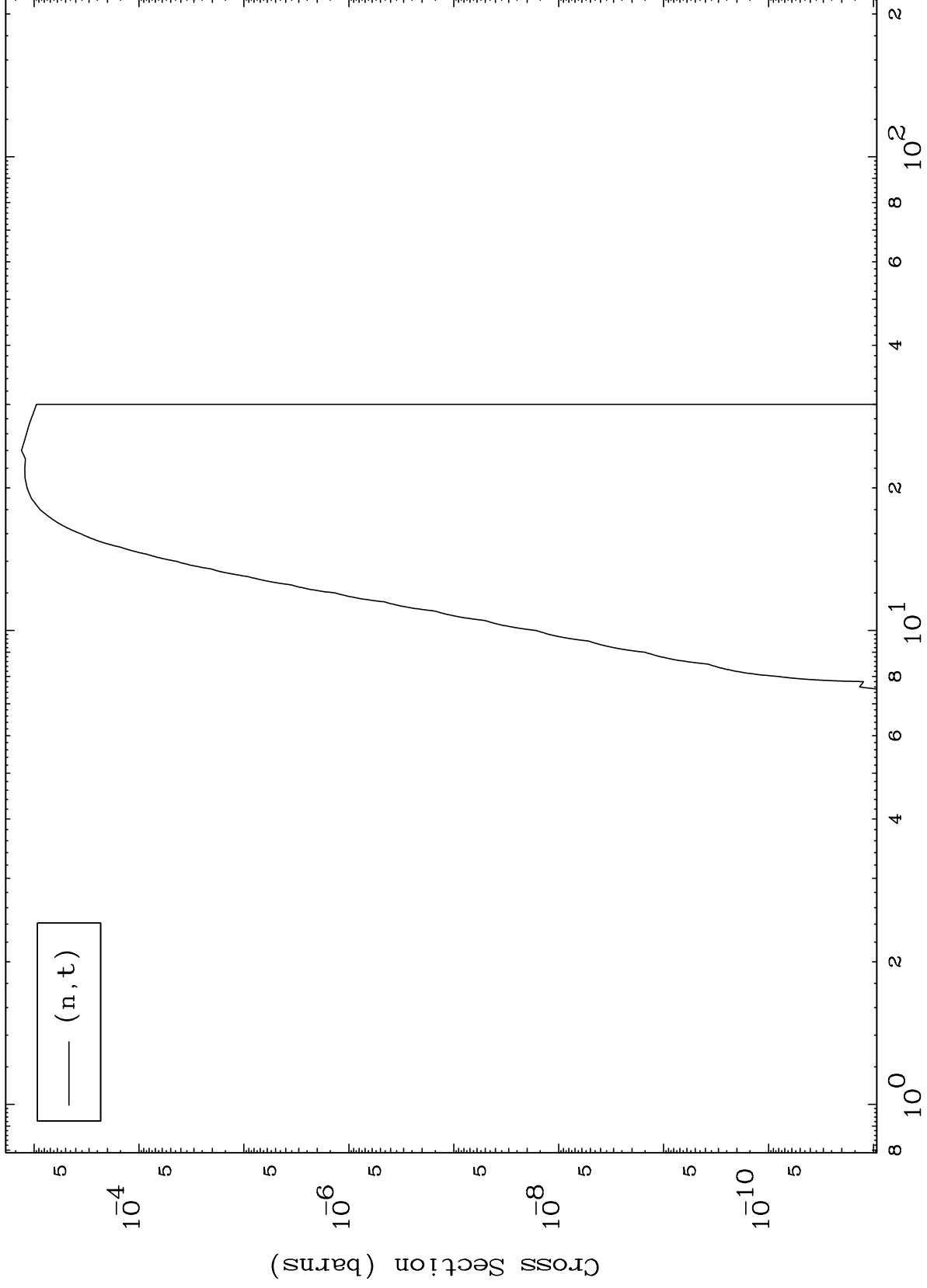
Incident Energy (MeV)

97-Bk-242

MAT 9731

(n,t) Levels
293 Kelvin Cross Sections

97-Bk-242



15

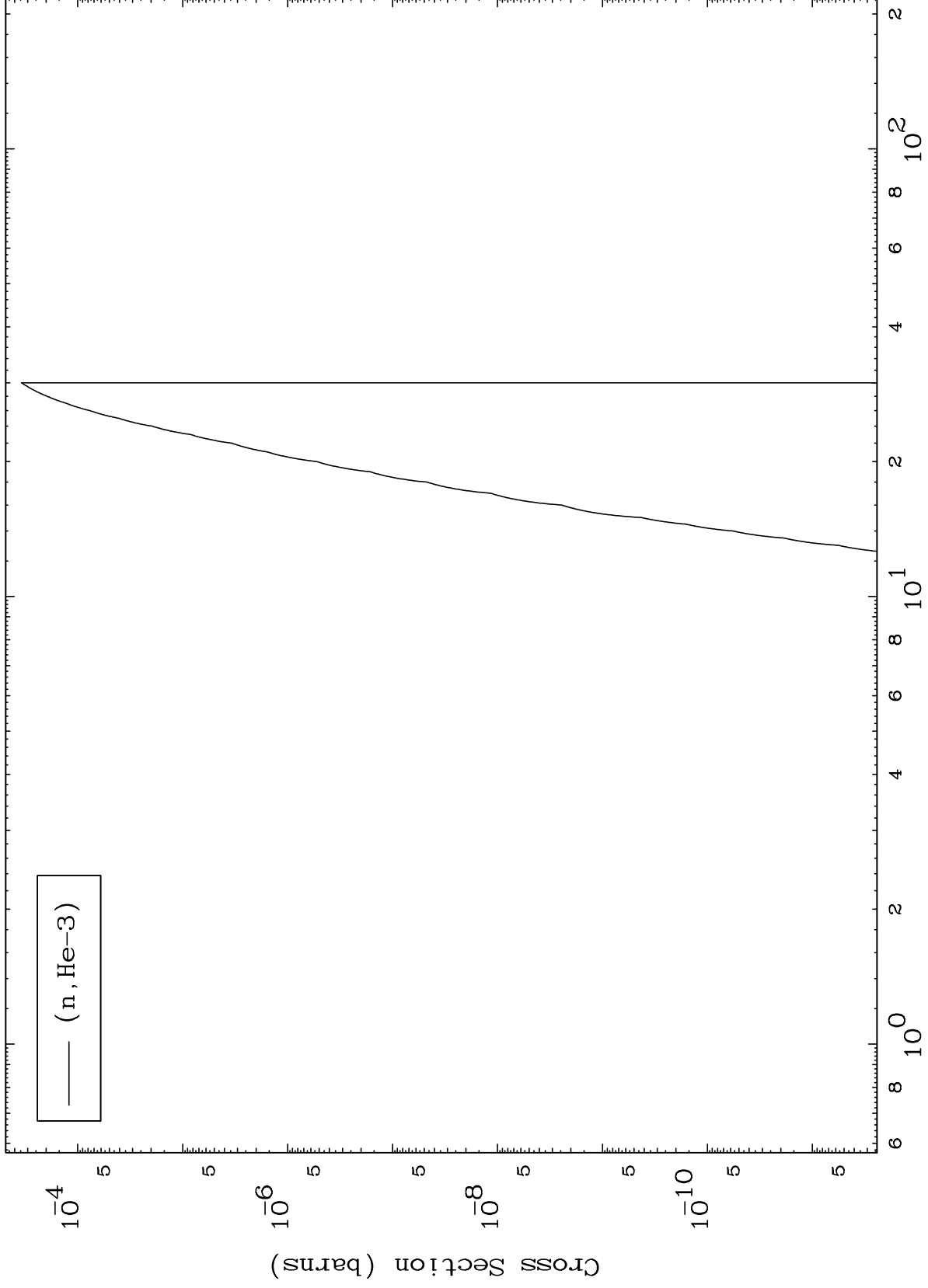
Incident Energy (MeV)

97-Bk-242

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(n,He3) Levels
293 Kelvin Cross Sections

97-Bk-242



16

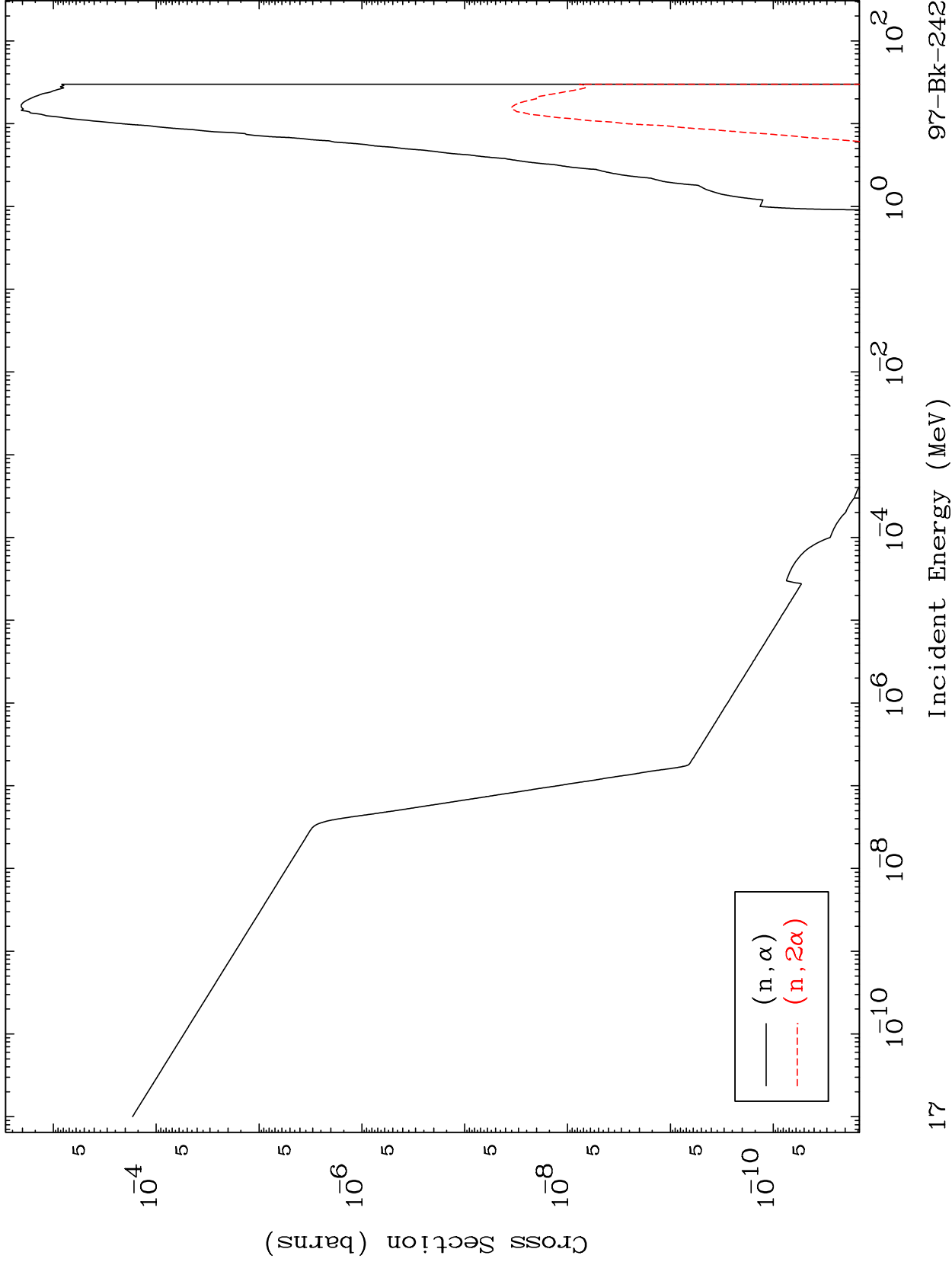
Incident Energy (MeV)

97-Bk-242

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(n, α) Levels
293 Kelvin Cross Sections

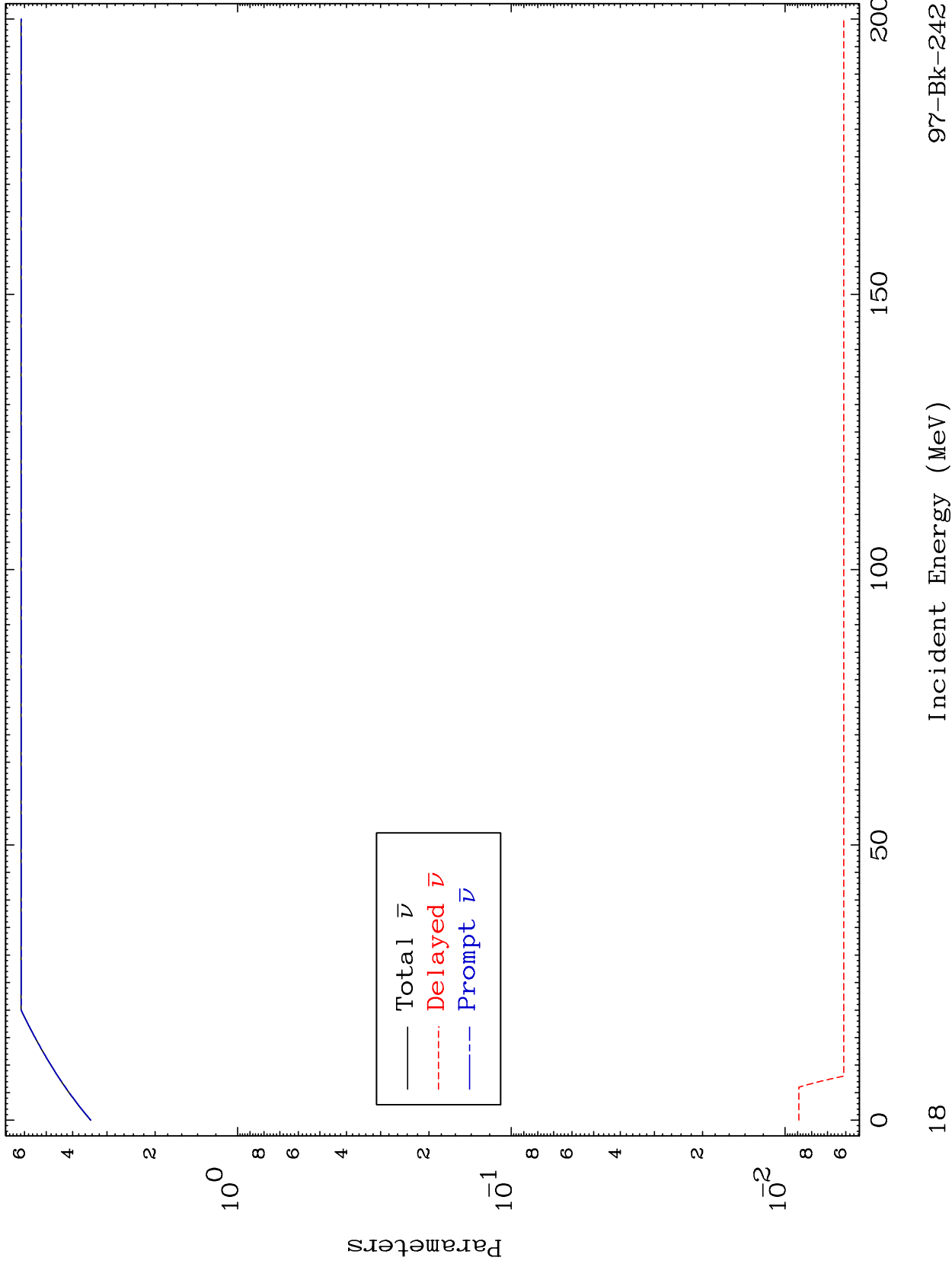
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Energy Release
Parameters

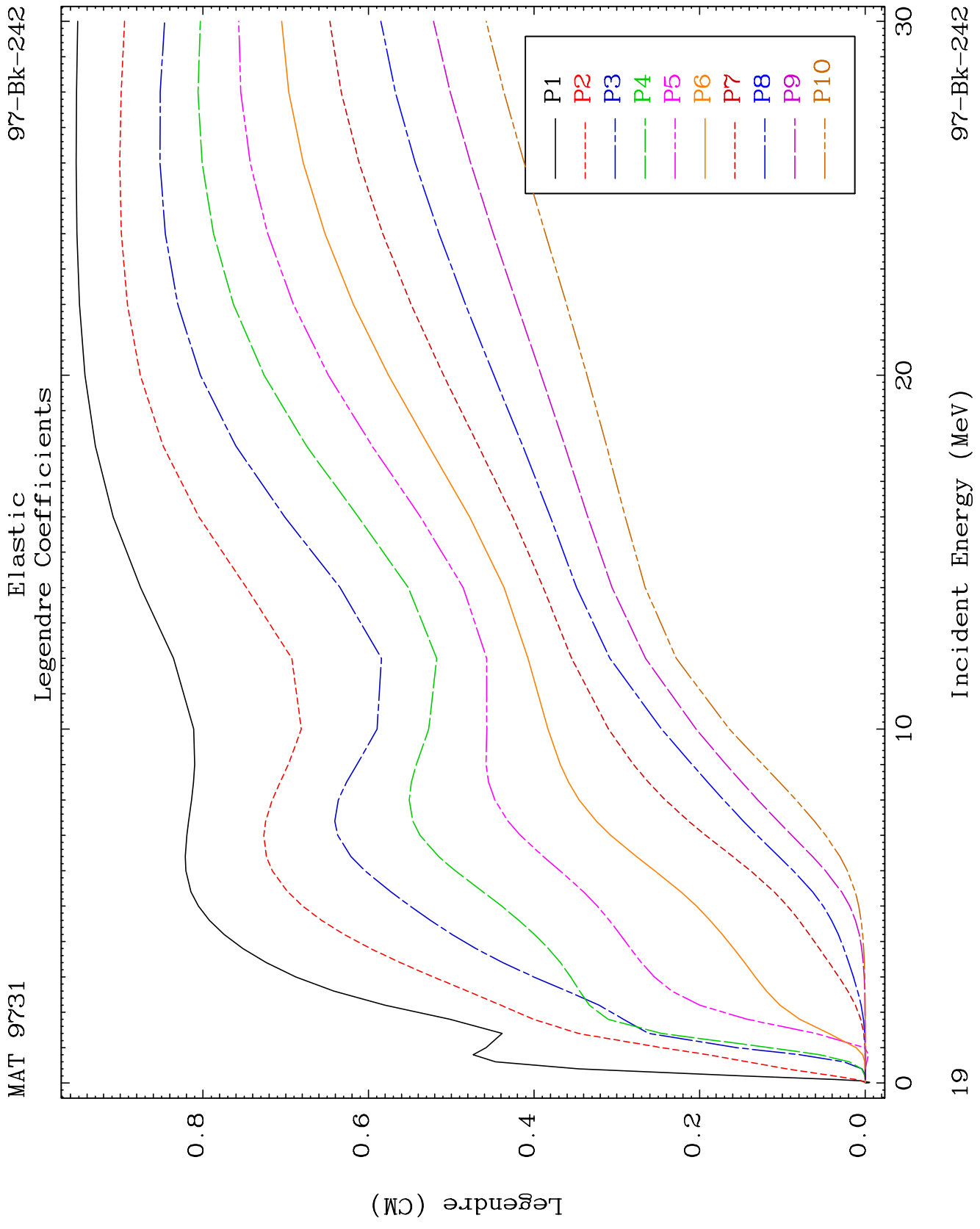
97-Bk-242



18

Incident Energy (MeV)

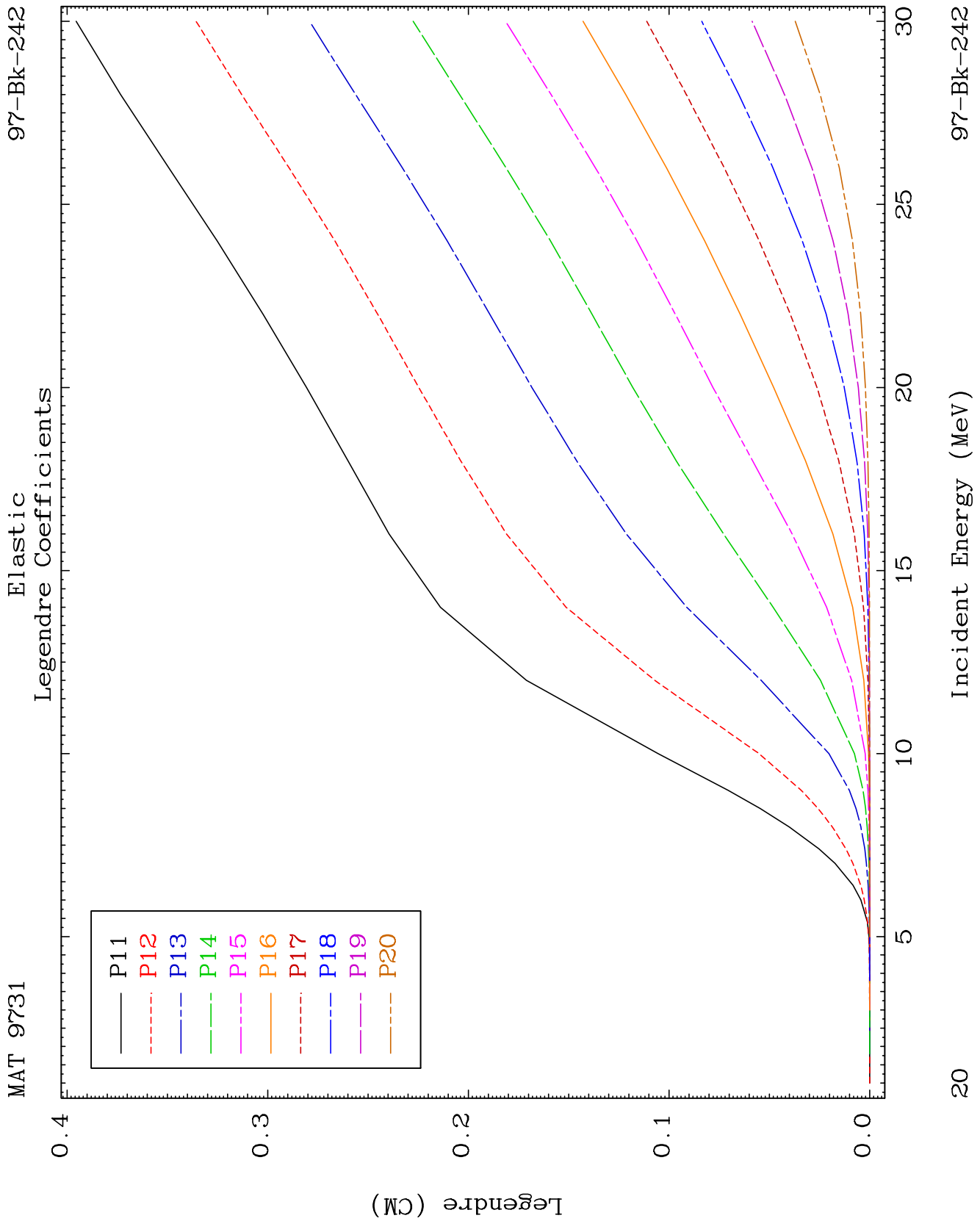
97-Bk-242



97-Bk-242

Incident Energy (MeV)

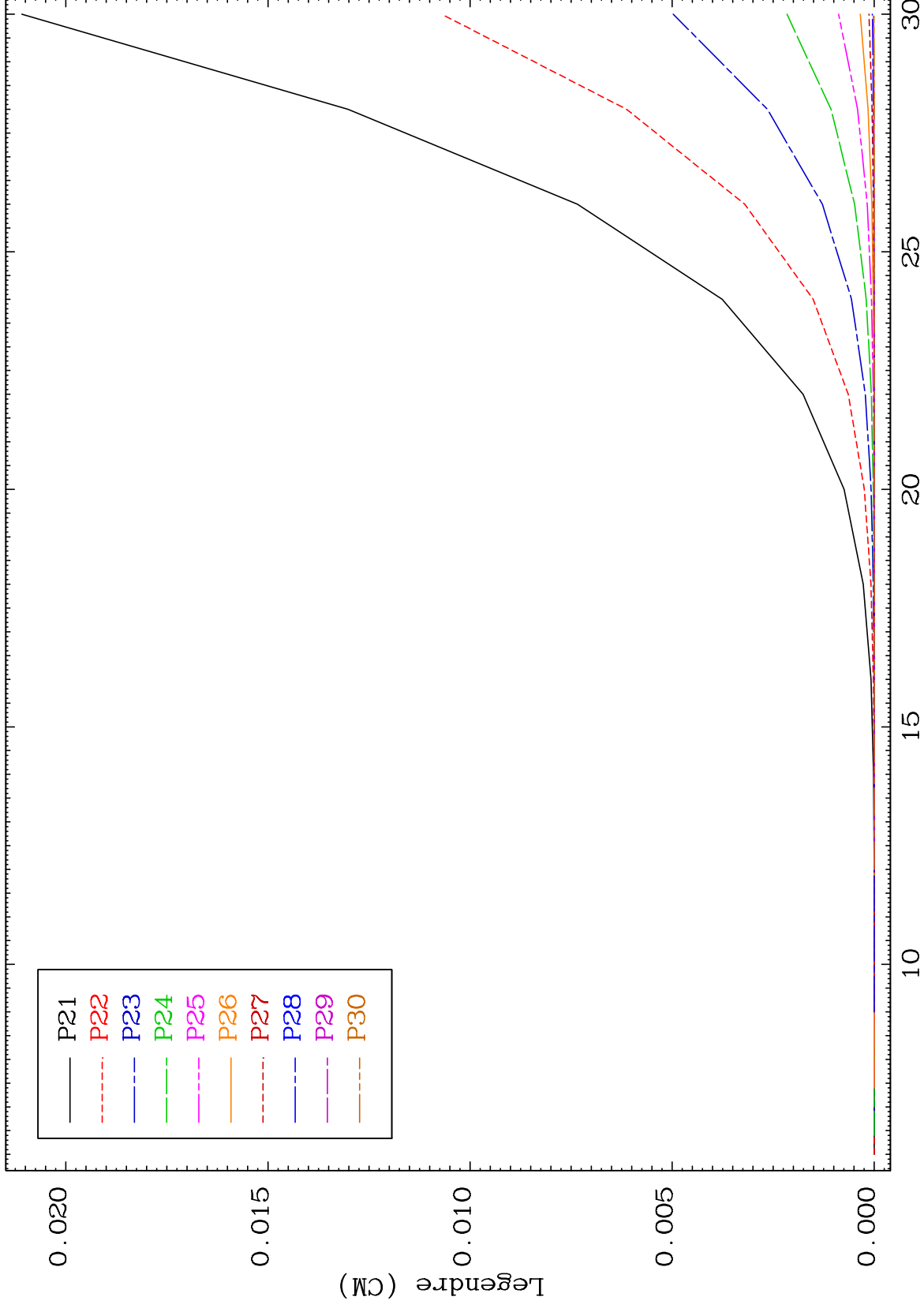
19



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Elastic Legendre Coefficients

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21

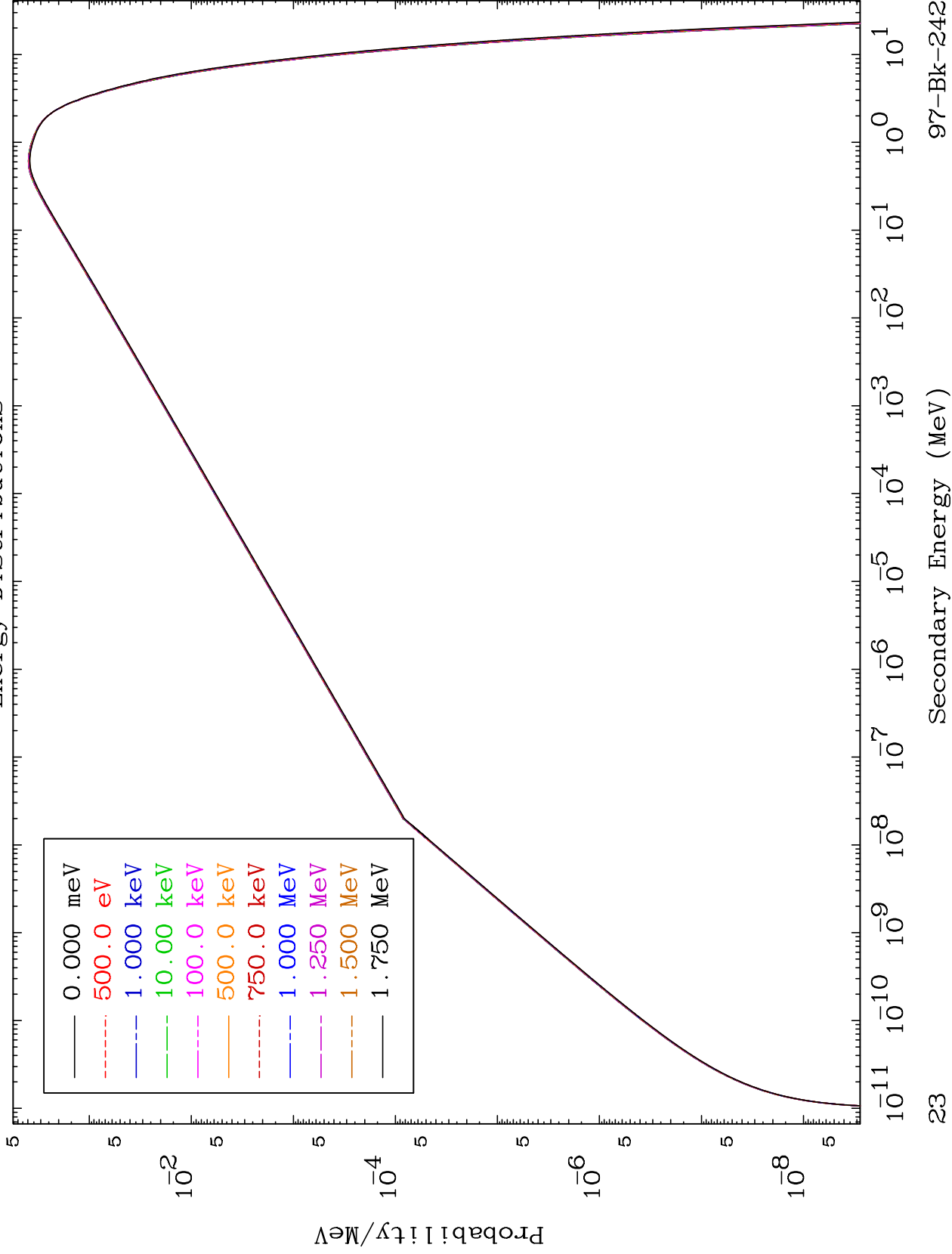
Incident Energy (MeV)

97-Bk-242

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Fission Energy Distributions

97-Bk-242



97-Bk-242

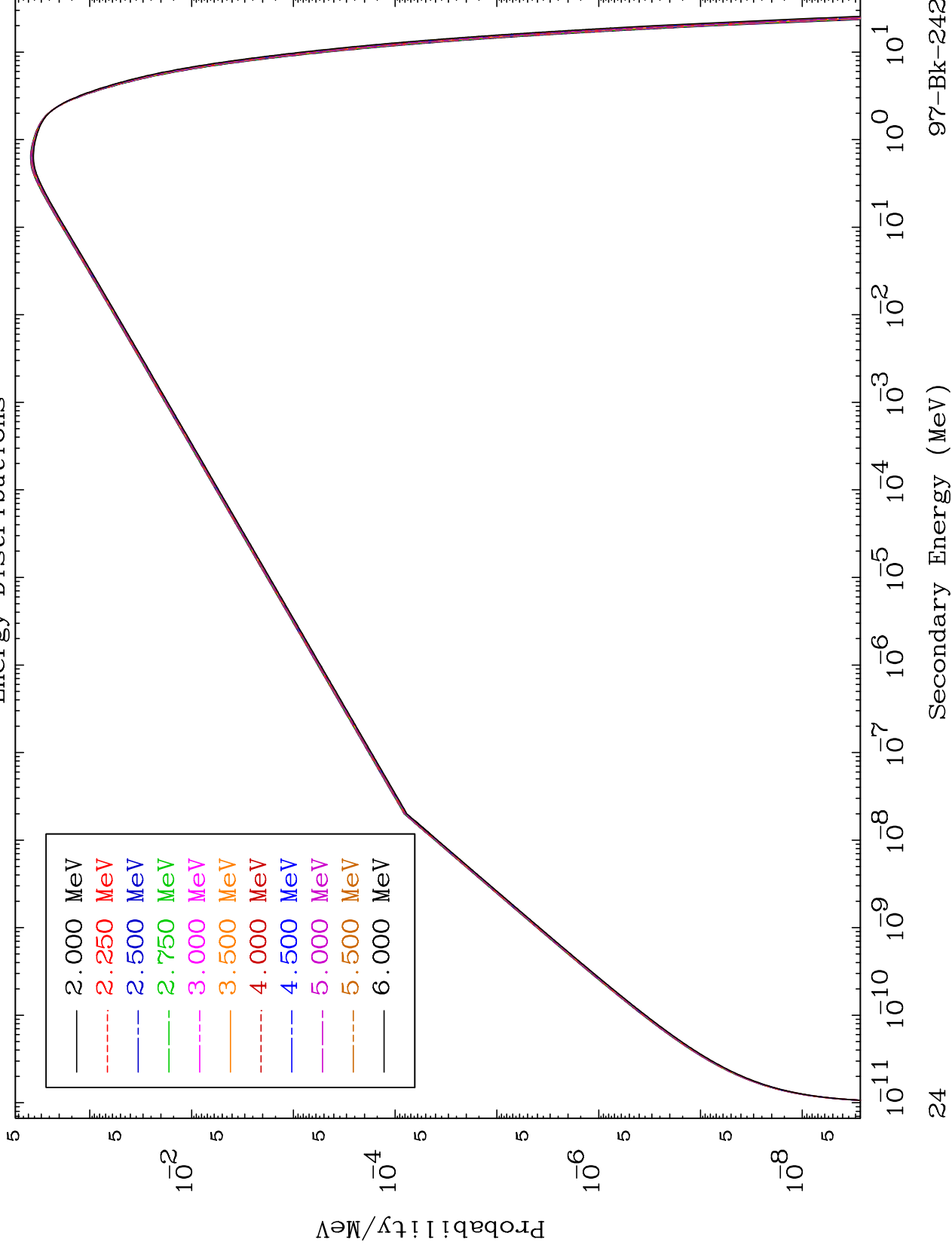
Secondary Energy (MeV)

23

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Fission Energy Distributions

97-Bk-242



97-Bk-242

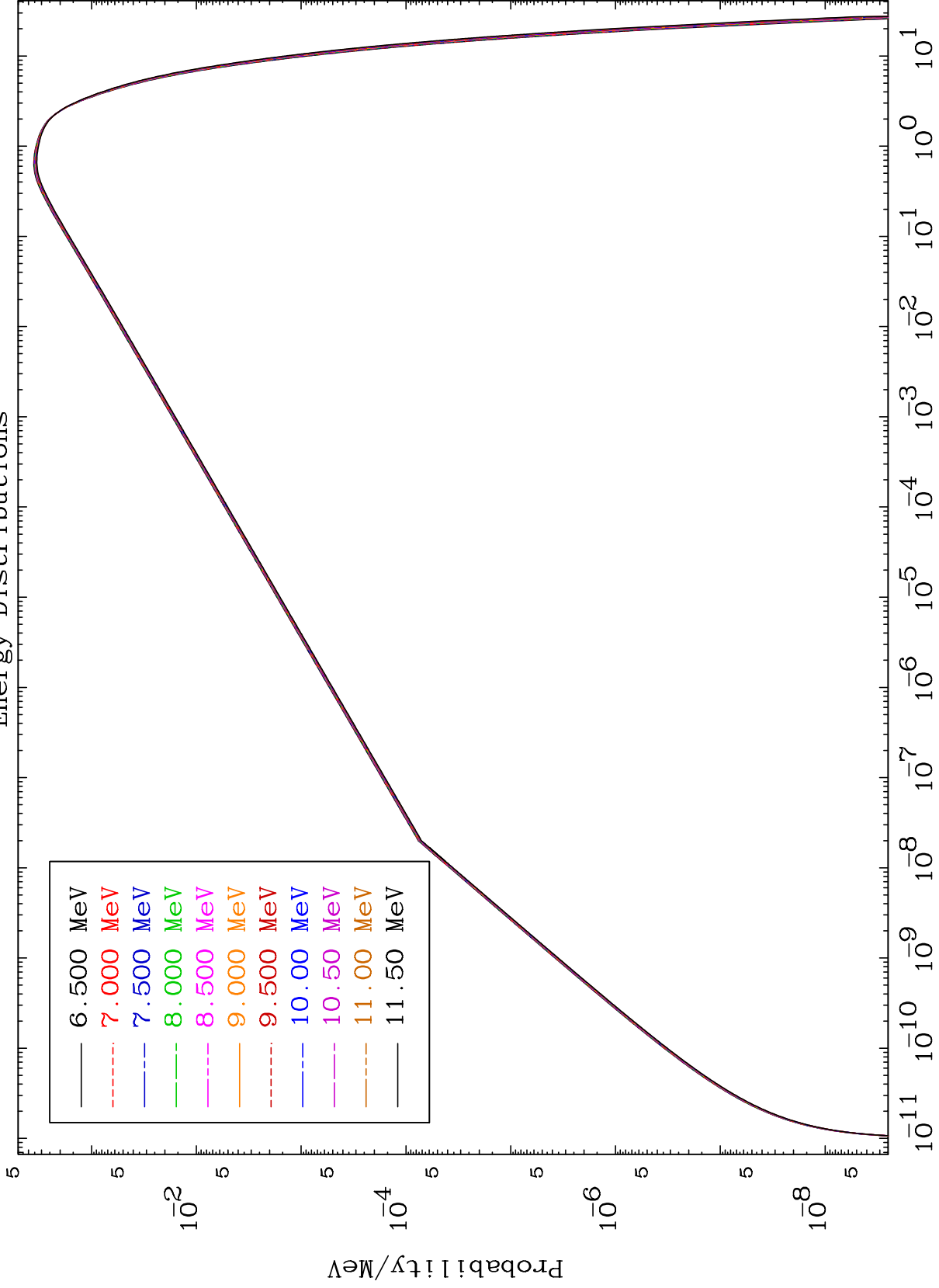
Secondary Energy (MeV)

24

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Fission Energy Distributions

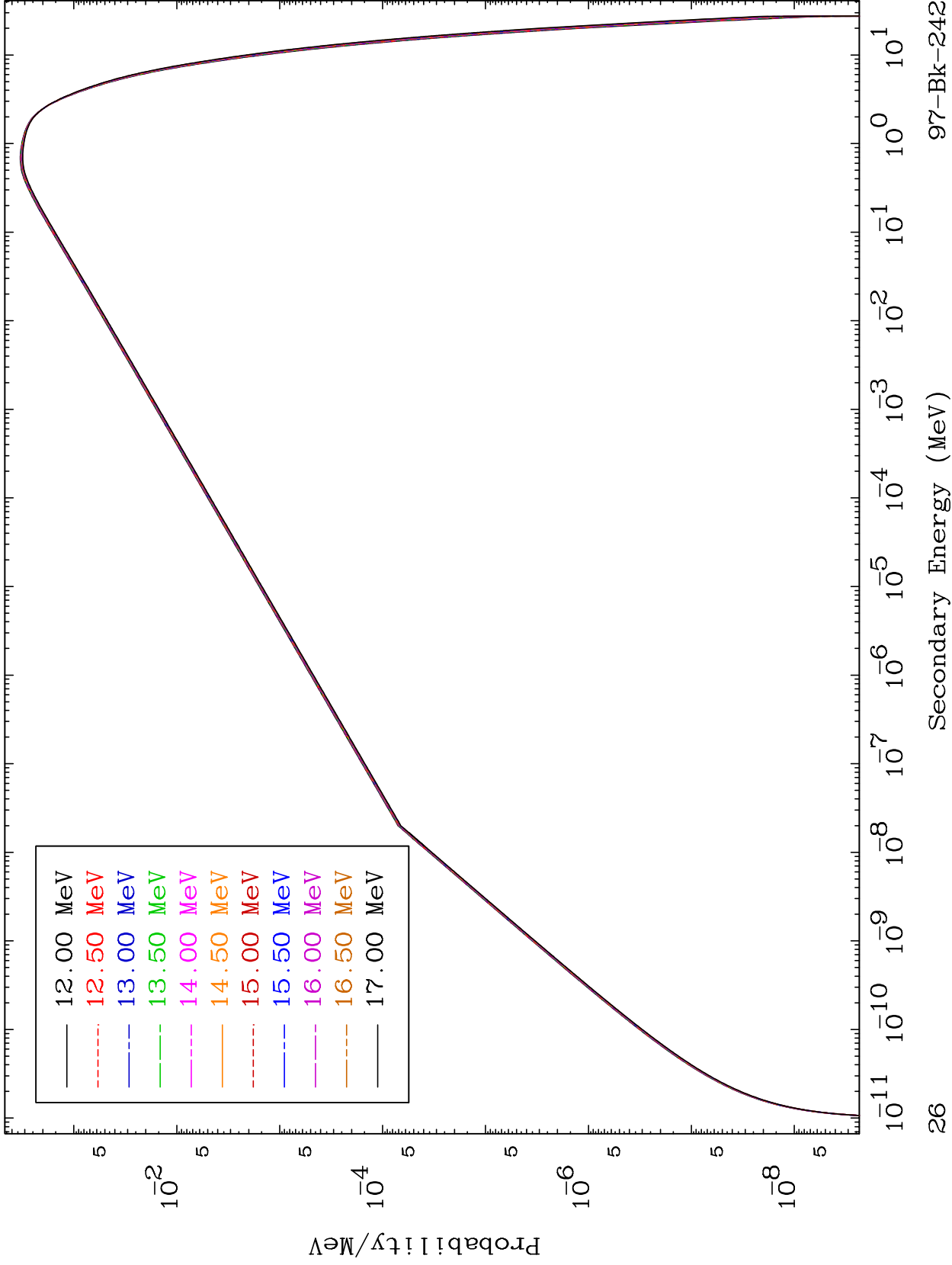
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25

Secondary Energy (MeV)

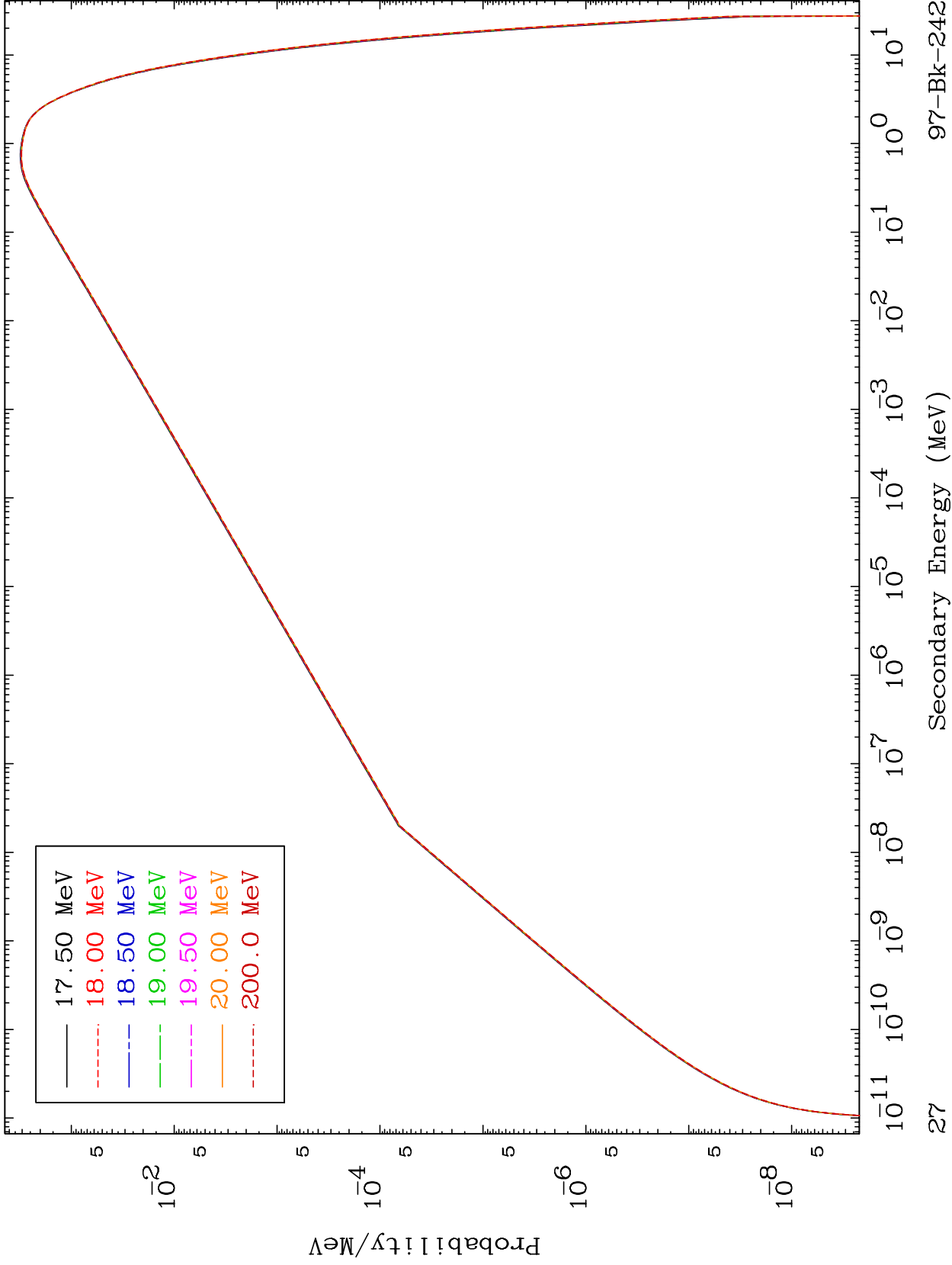
97-Bk-242



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Fission Energy Distributions

97-Bk-242

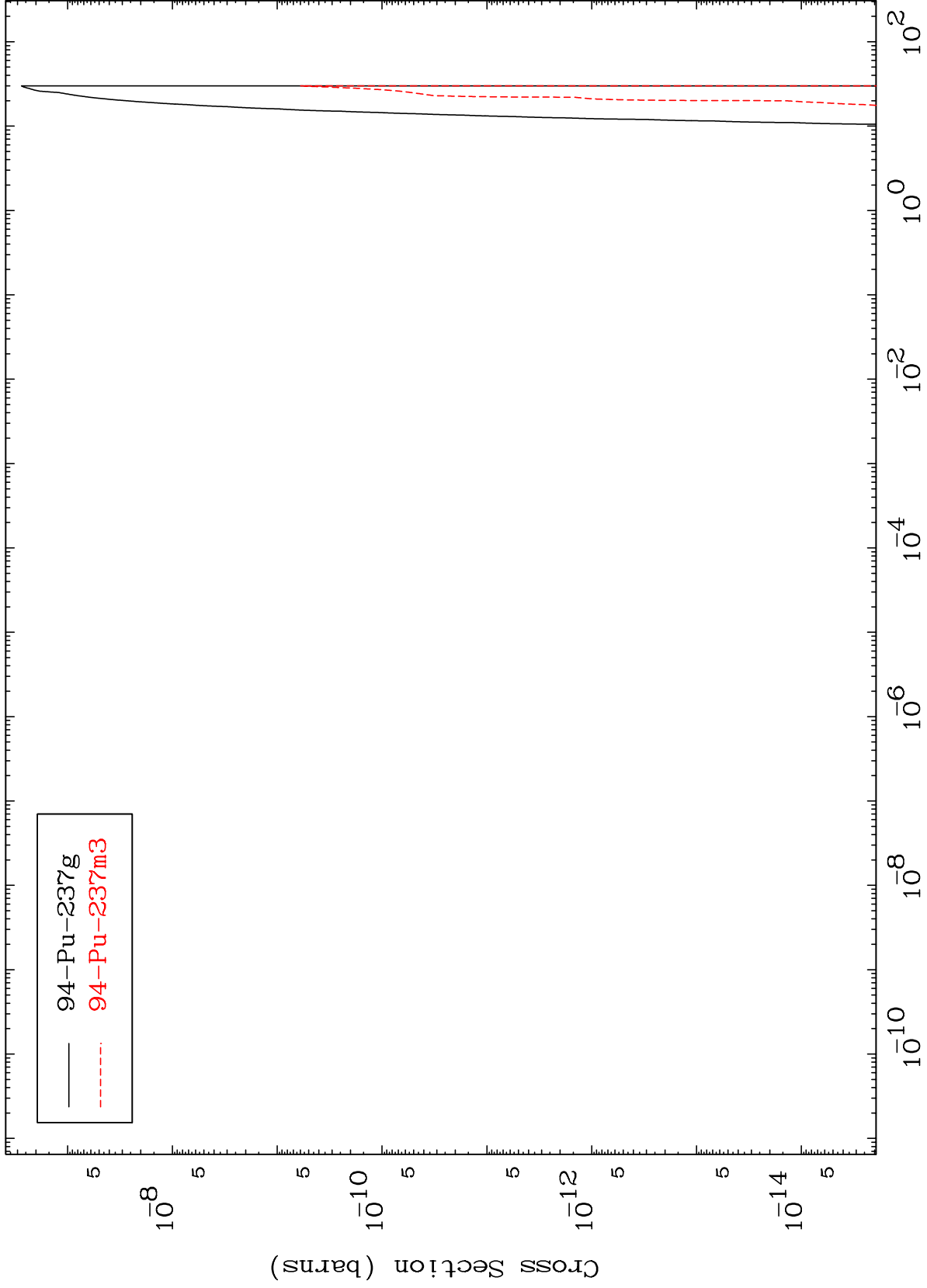


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(n,n') p α

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Radionuclide Production Cross Section



28

Incident Energy (MeV)

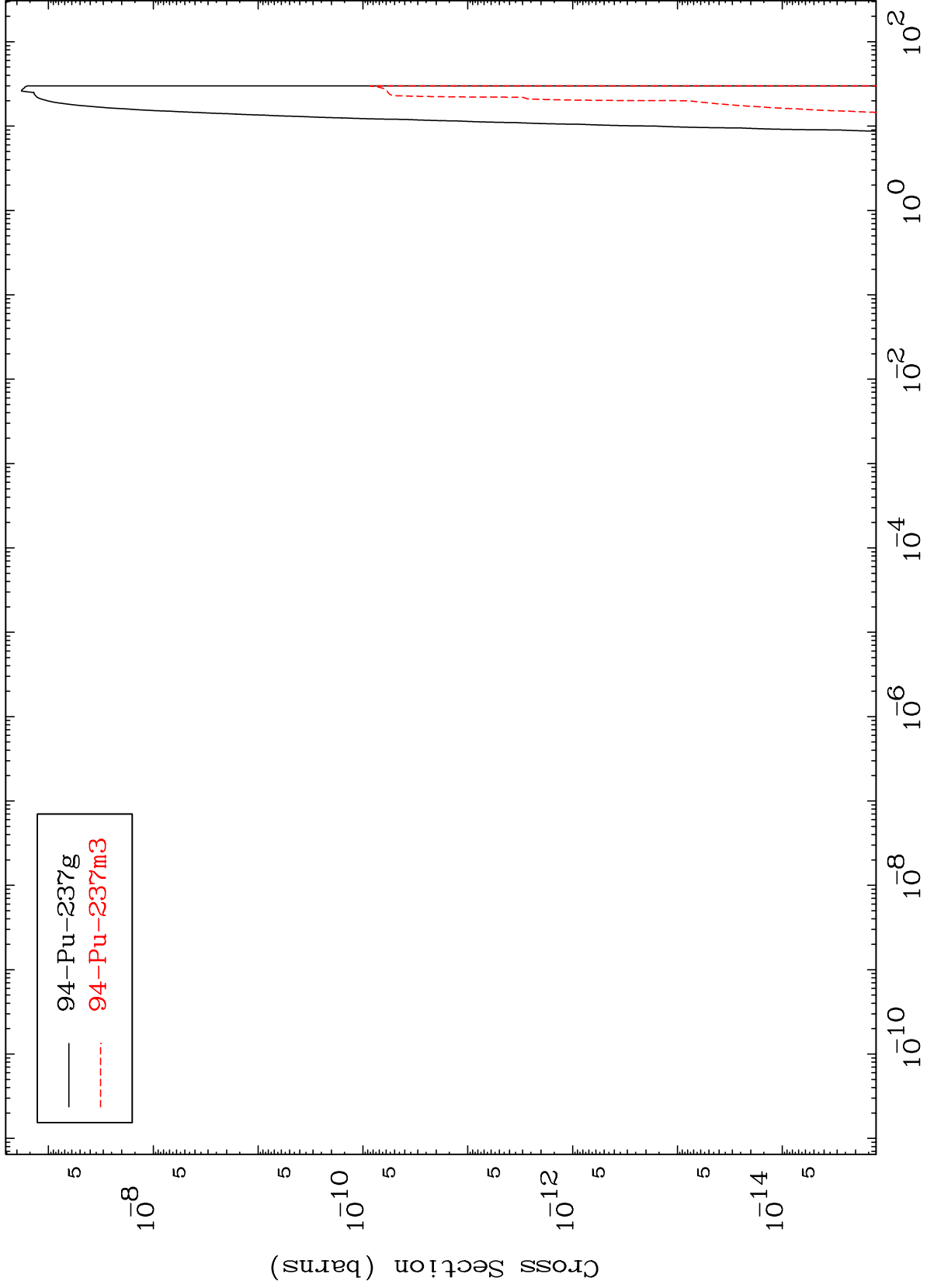
97-Bk-242

MAT 9731

(n,d) α

97-Bk-242

Radionuclide Production Cross Section



29

Incident Energy (MeV)

97-Bk-242