

Program EVALPLOT
(Version 2021-1)

by

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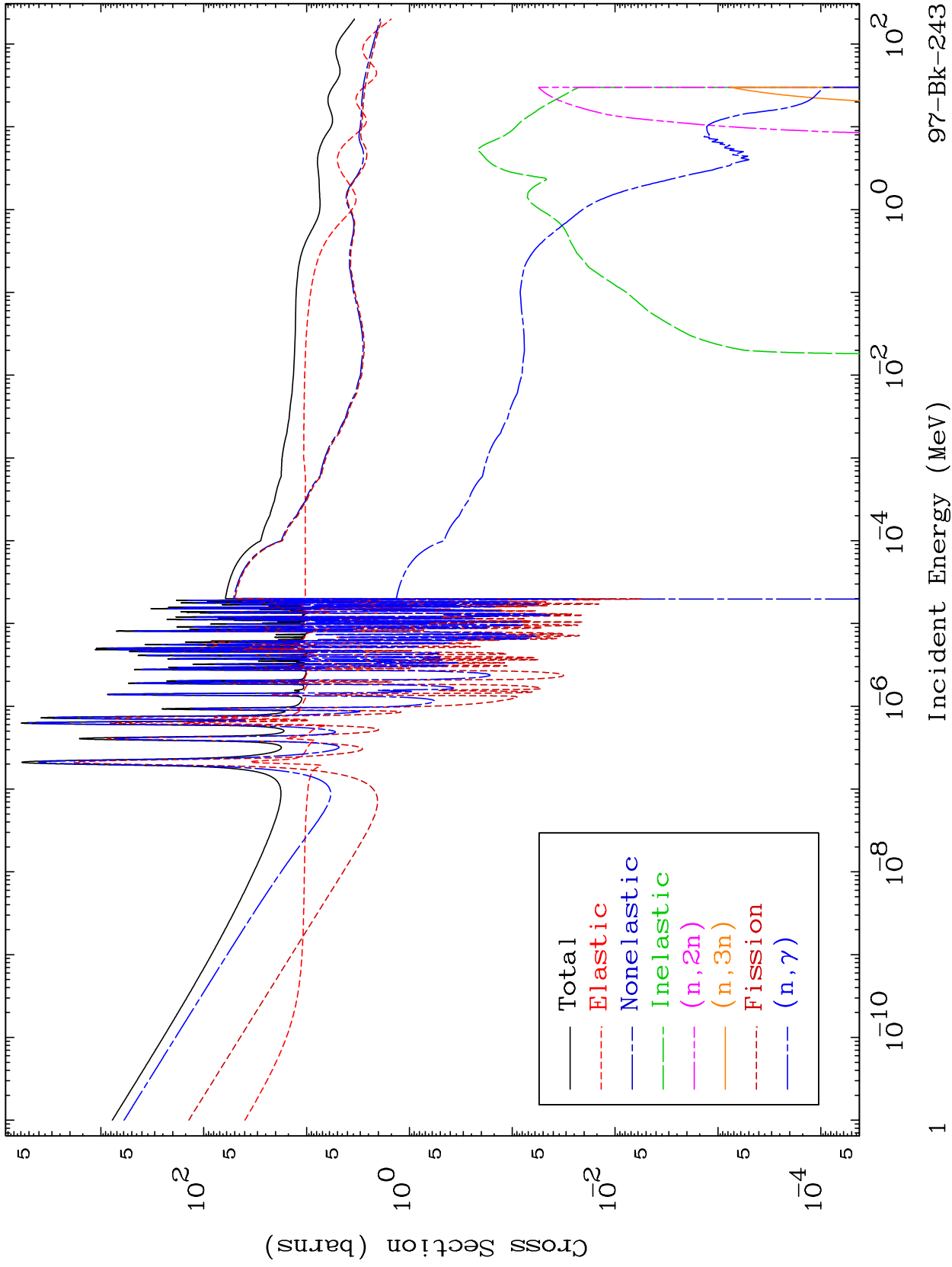
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9734

Neutron Major
293 Kelvin Cross Sections

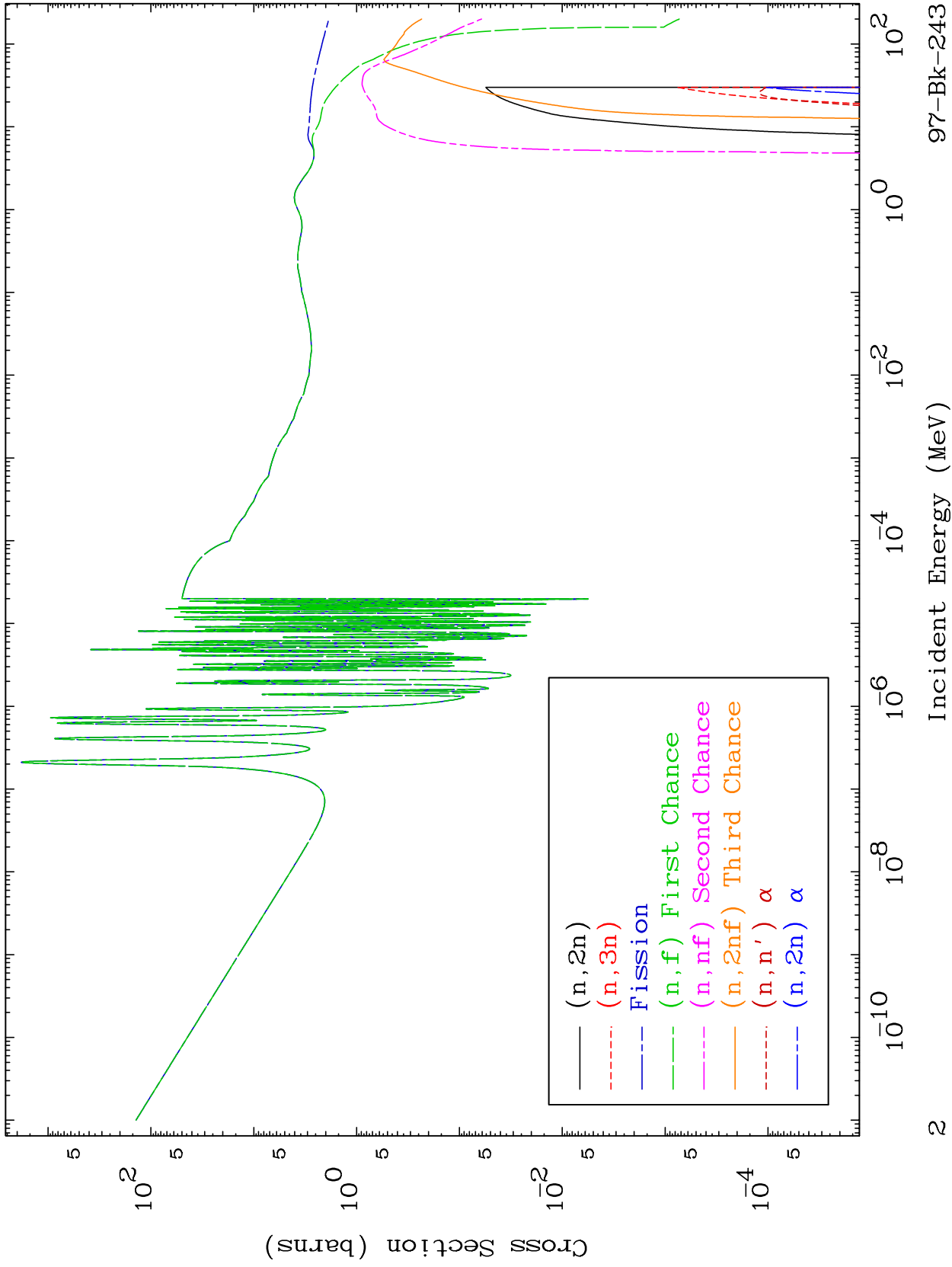
97-Bk-243



MAT 9734

Neutron Absorption
293 Kelvin Cross Sections

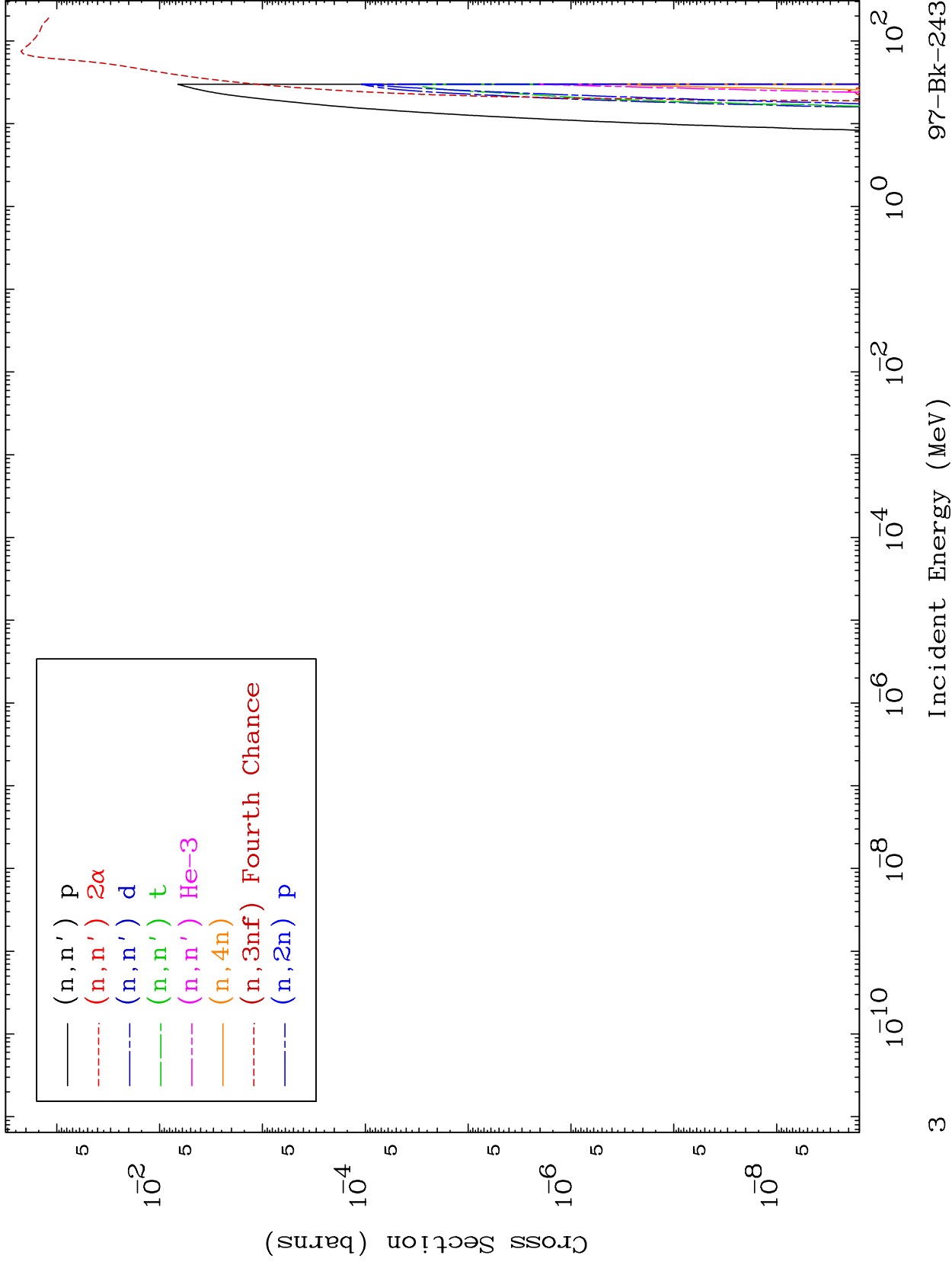
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MAT 9734

Neutron Absorption
293 Kelvin Cross Sections

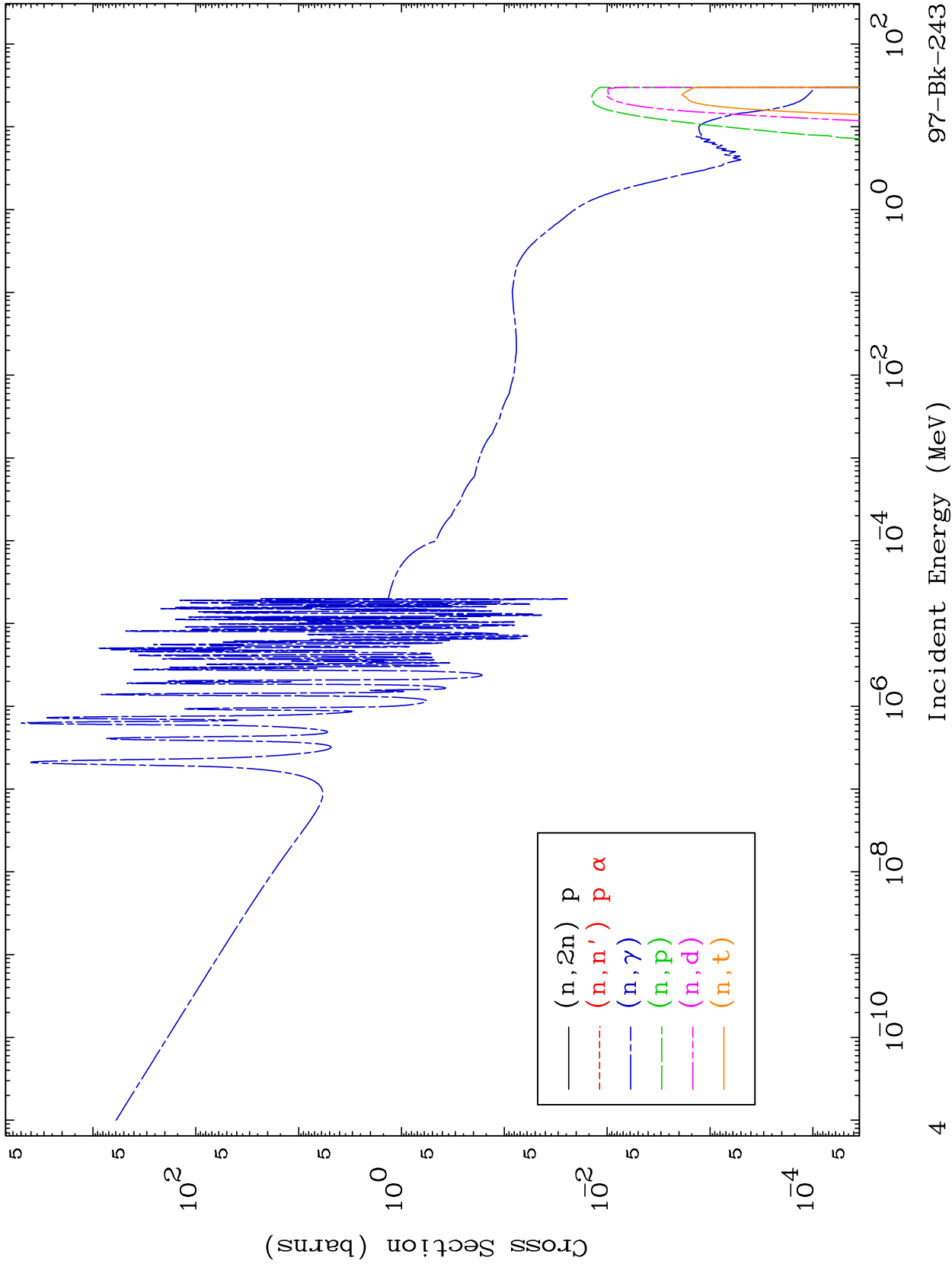
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Neutron Absorption
293 Kelvin Cross Sections

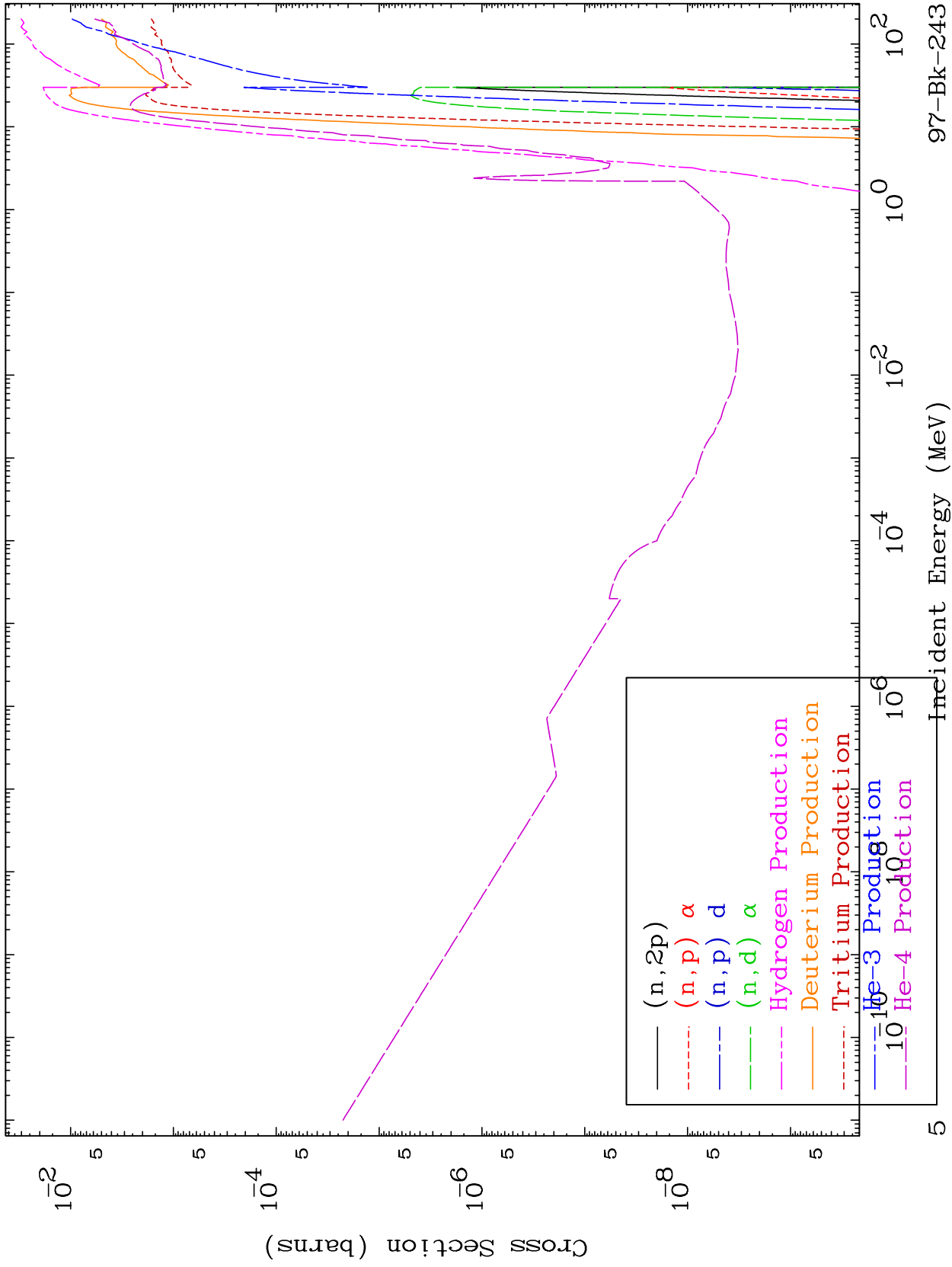
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Neutron Absorption
293 Kelvin Cross Sections

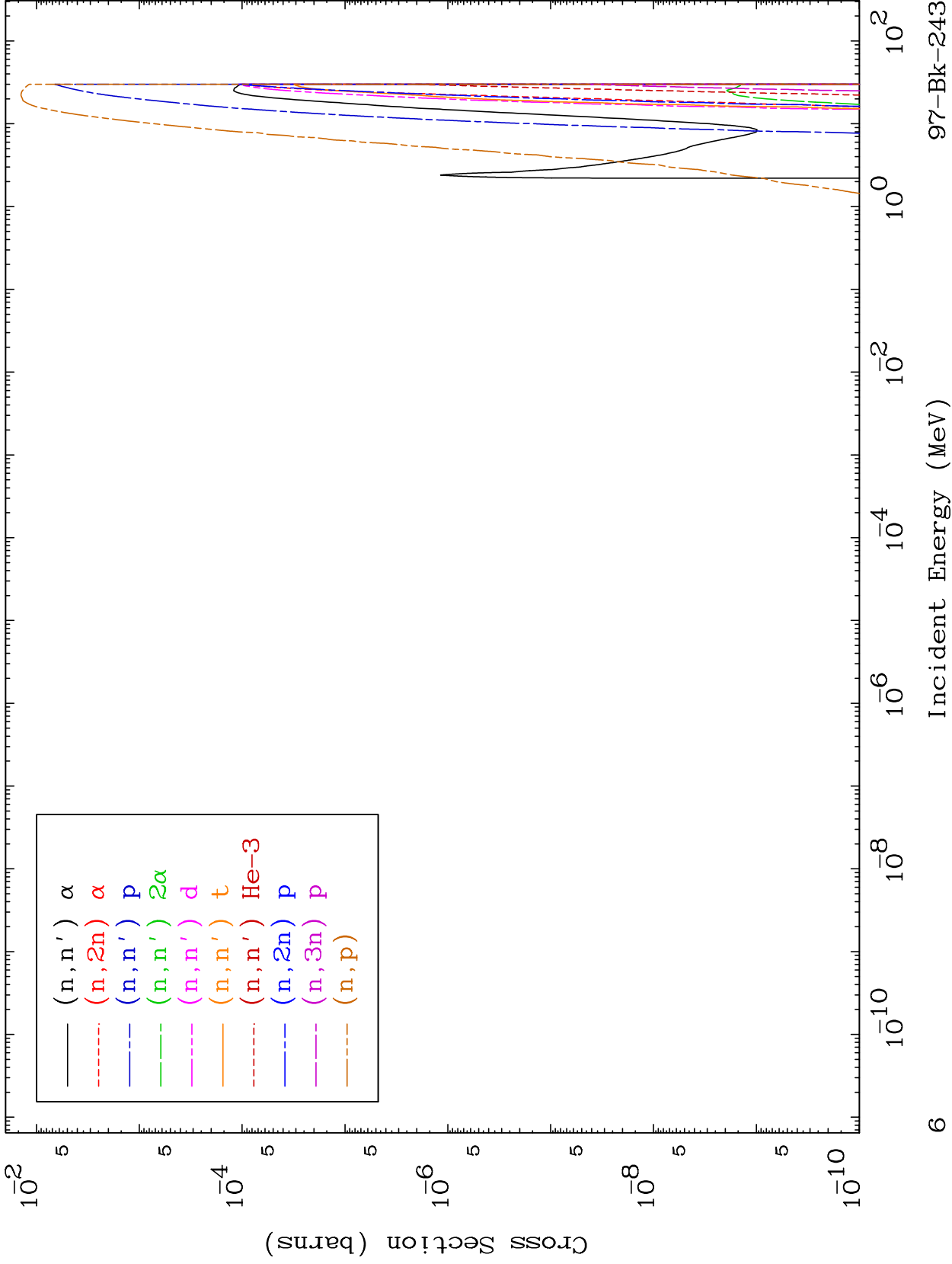
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MAT 9734

Charged Particle
293 Kelvin Cross Sections

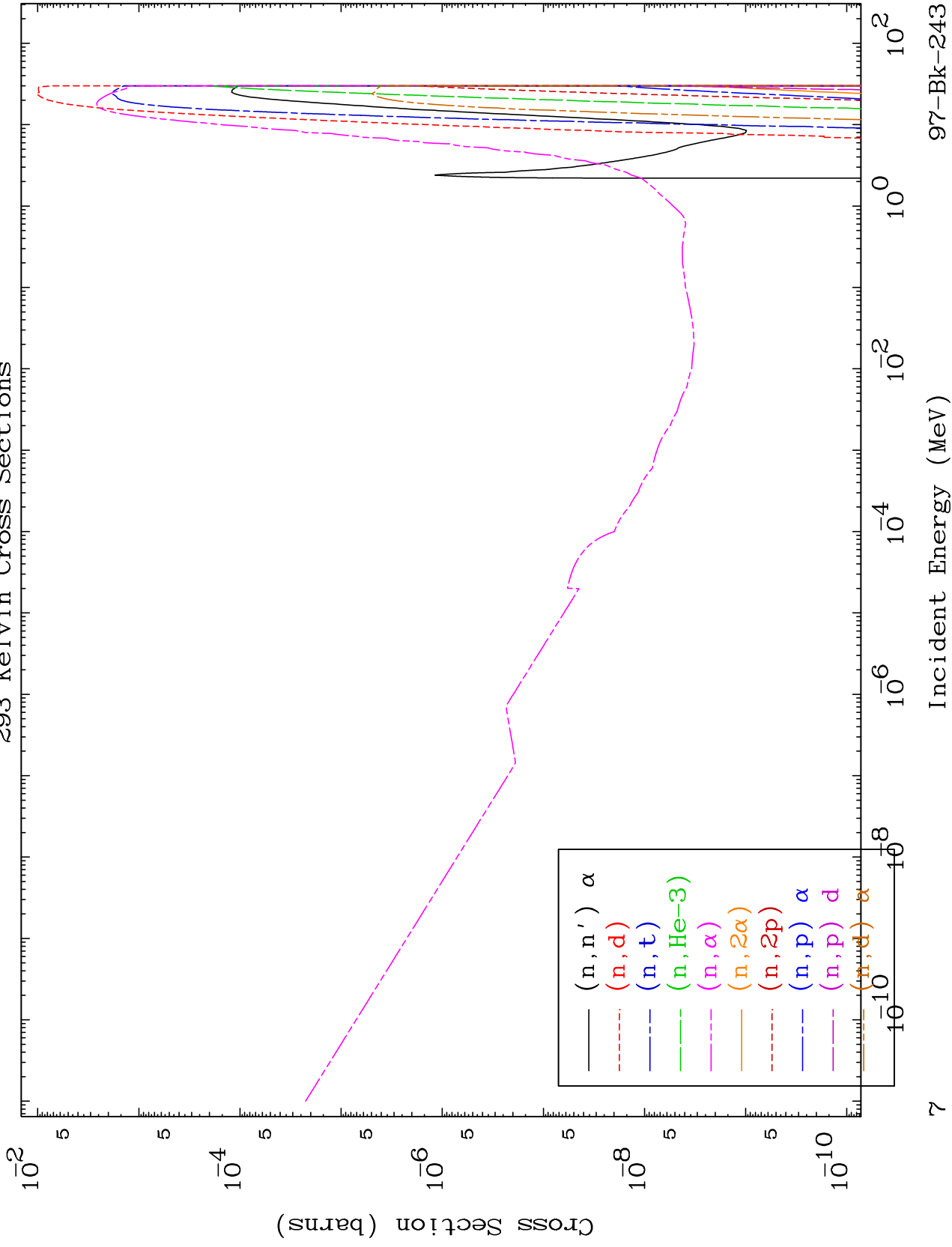
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Charged Particle
293 Kelvin Cross Sections

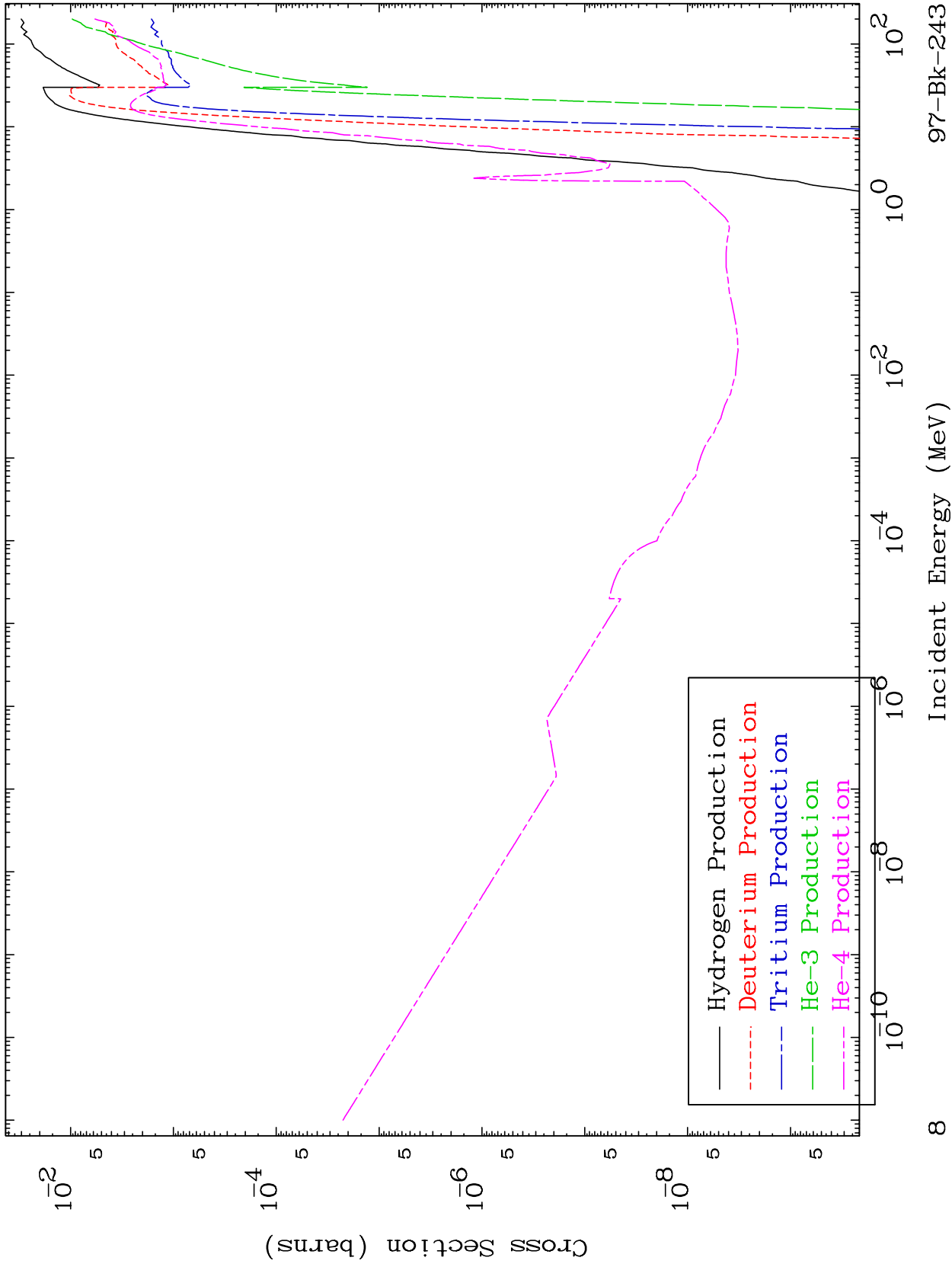
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Particle Production
293 Kelvin Cross Sections

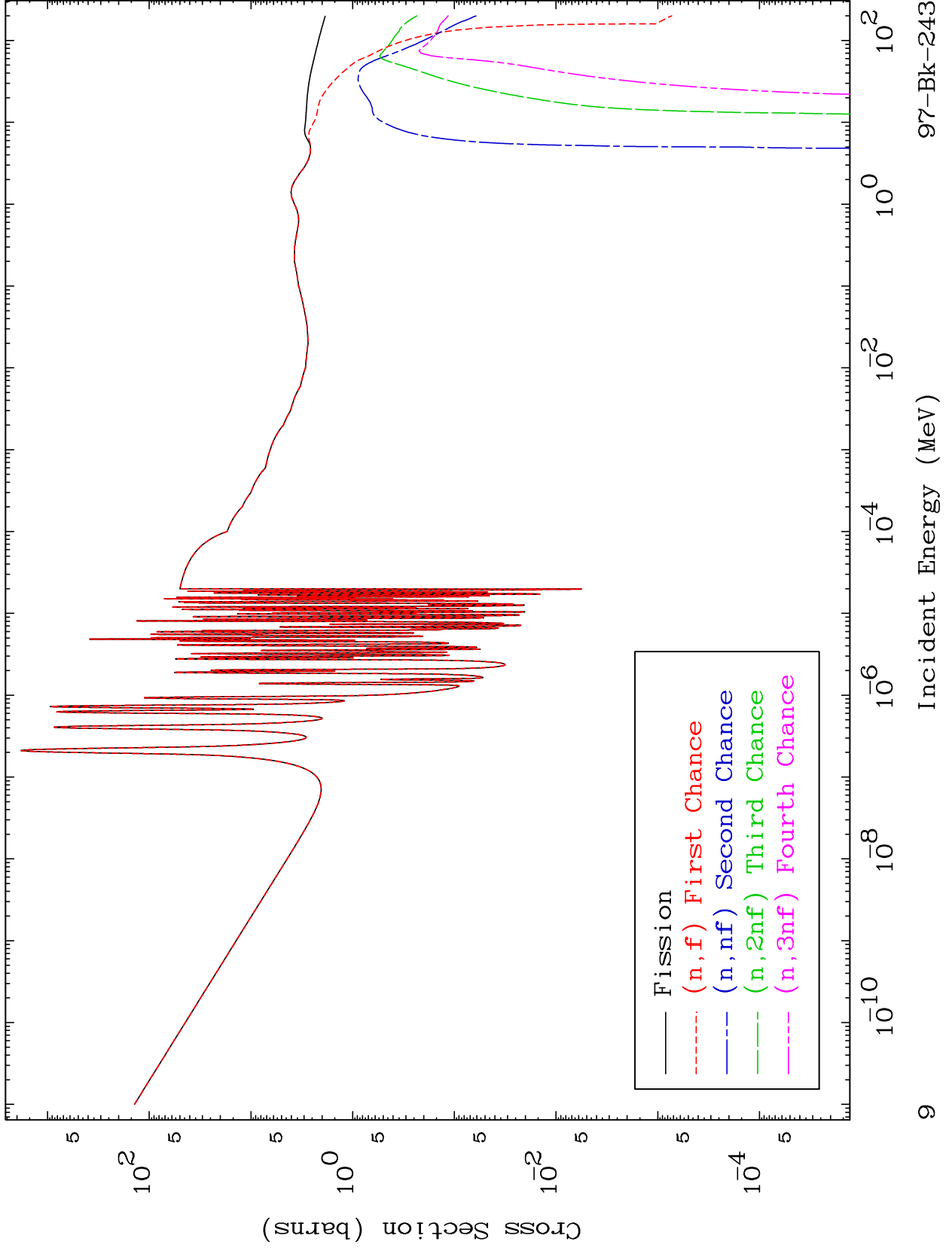
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Fission
293 Kelvin Cross Sections

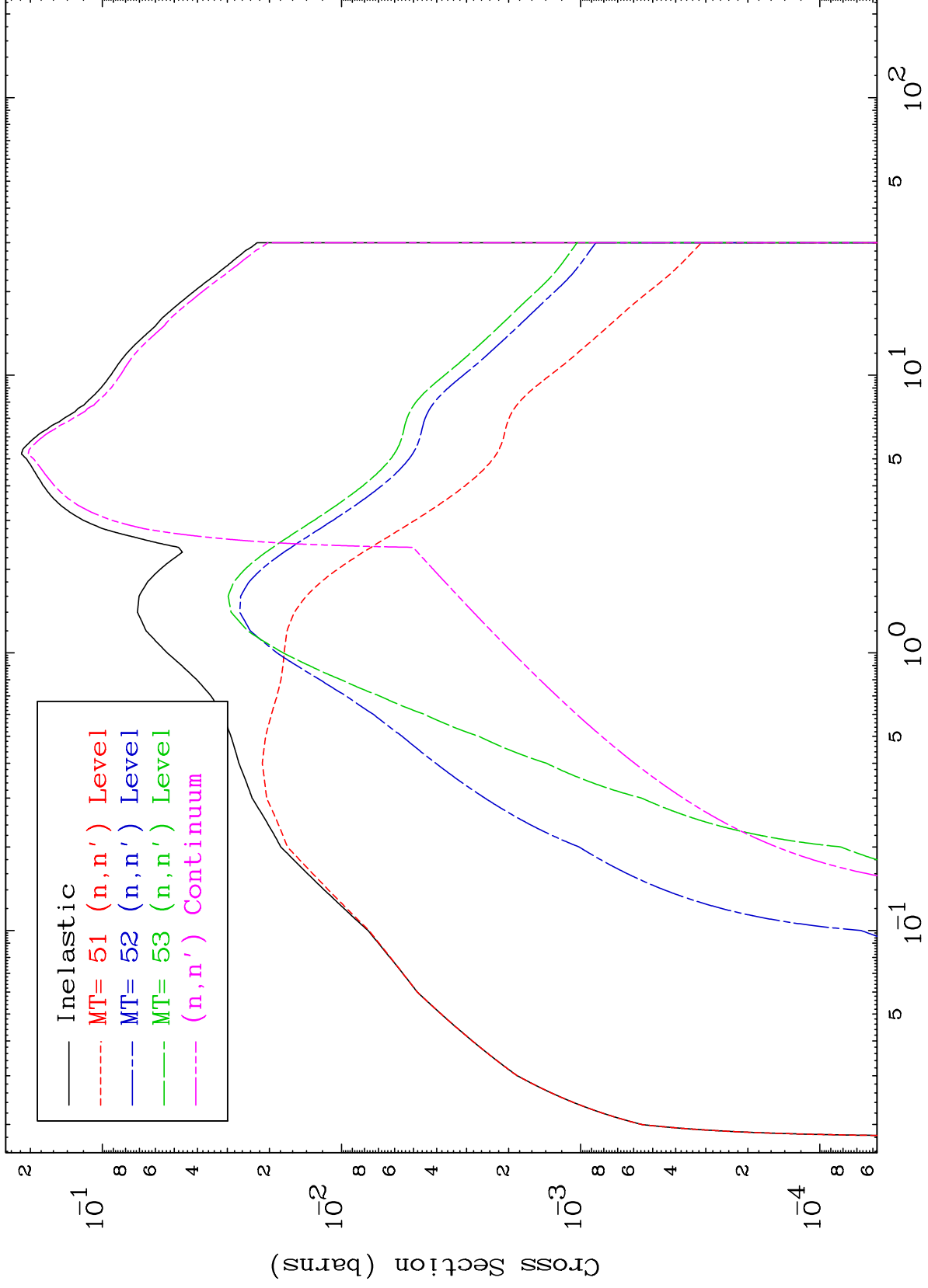
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293 Kelvin Cross Sections
(n,n') Levels

97-Bk-243



10

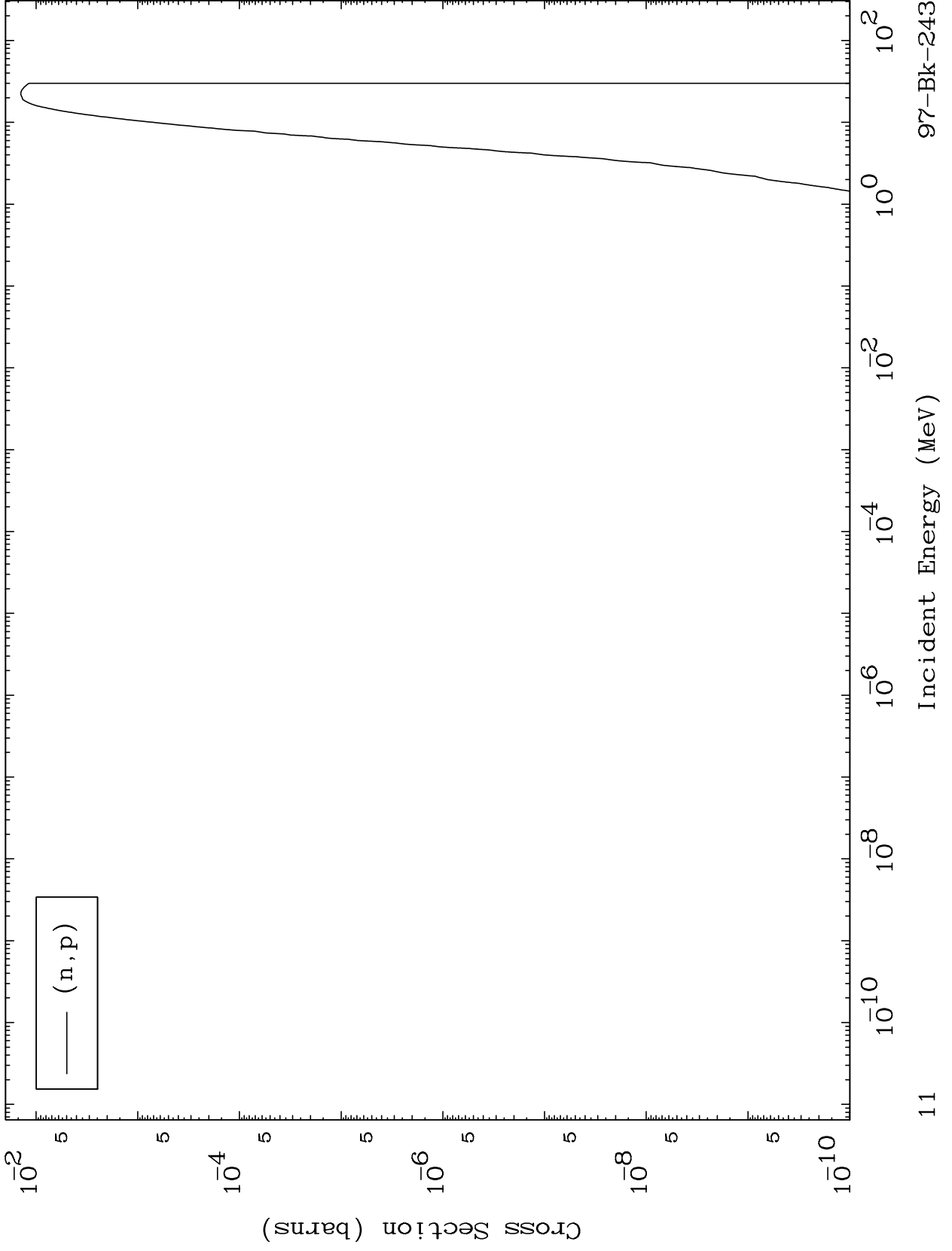
Incident Energy (MeV)

97-Bk-243

MAT 9734

(n,p) Levels
293 Kelvin Cross Sections

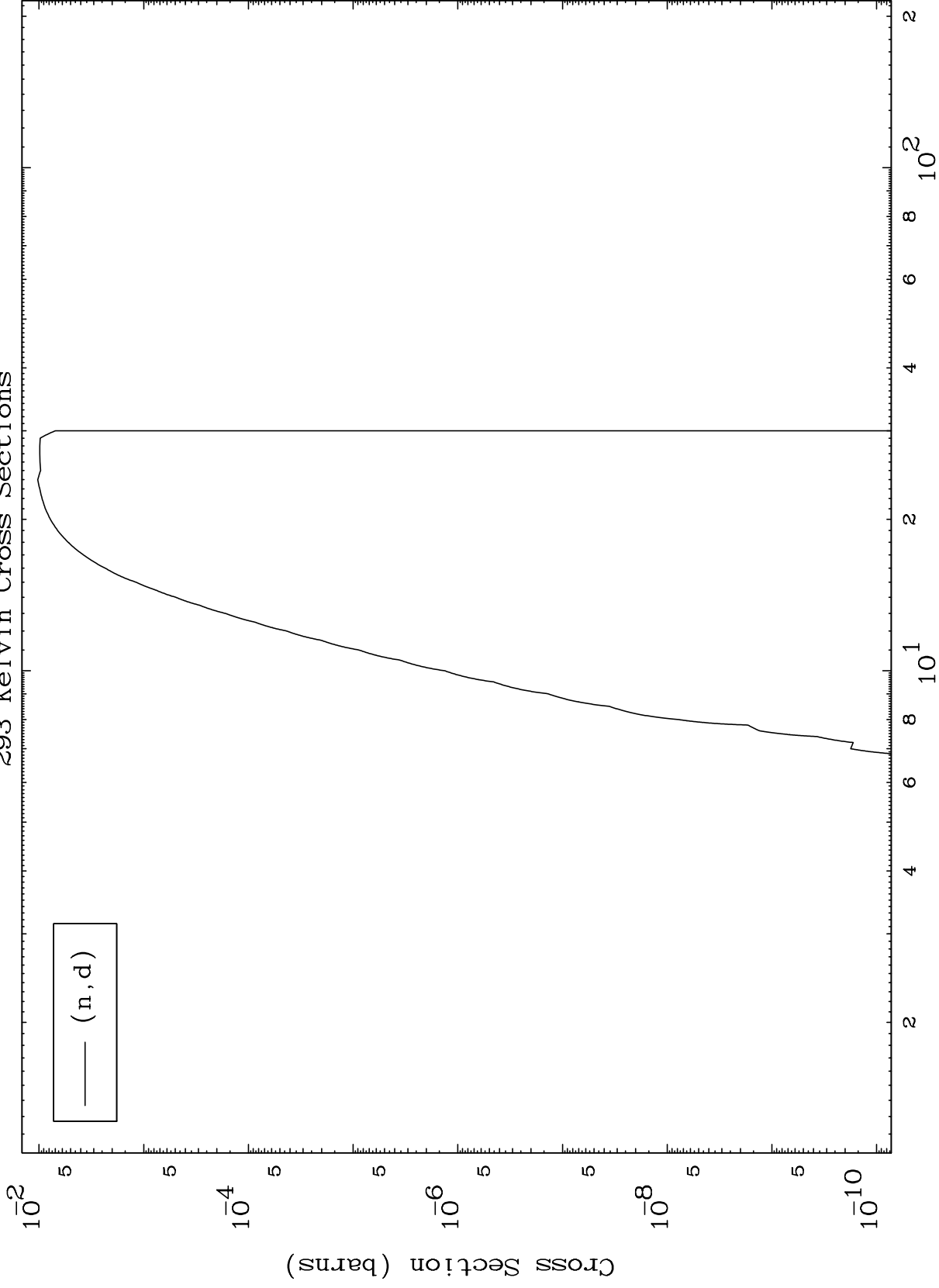
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(n,d) Levels
293 Kelvin Cross Sections

97-Bk-243



12

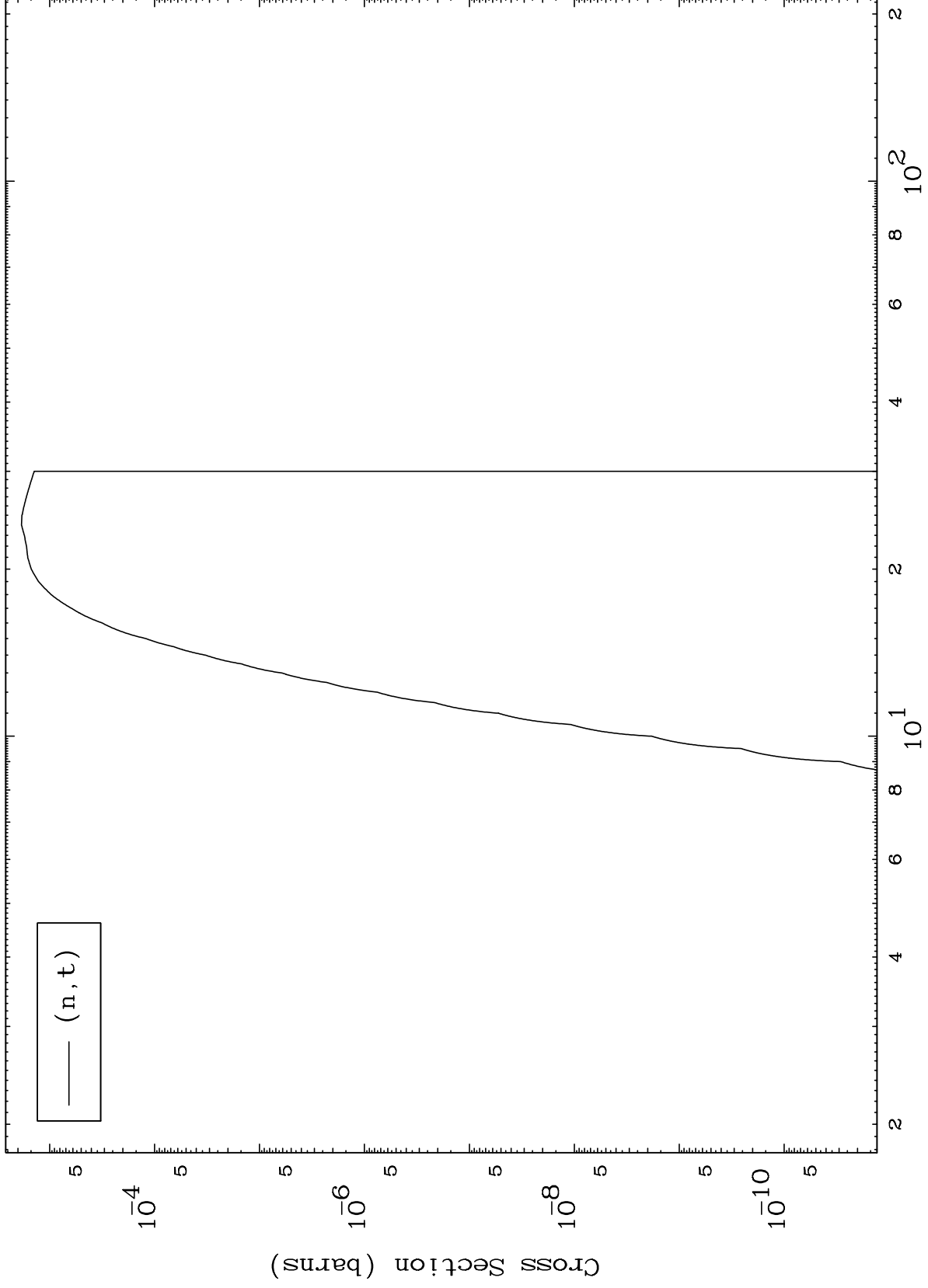
Incident Energy (MeV)

97-Bk-243

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(n,t) Levels
293 Kelvin Cross Sections

97-Bk-243



13

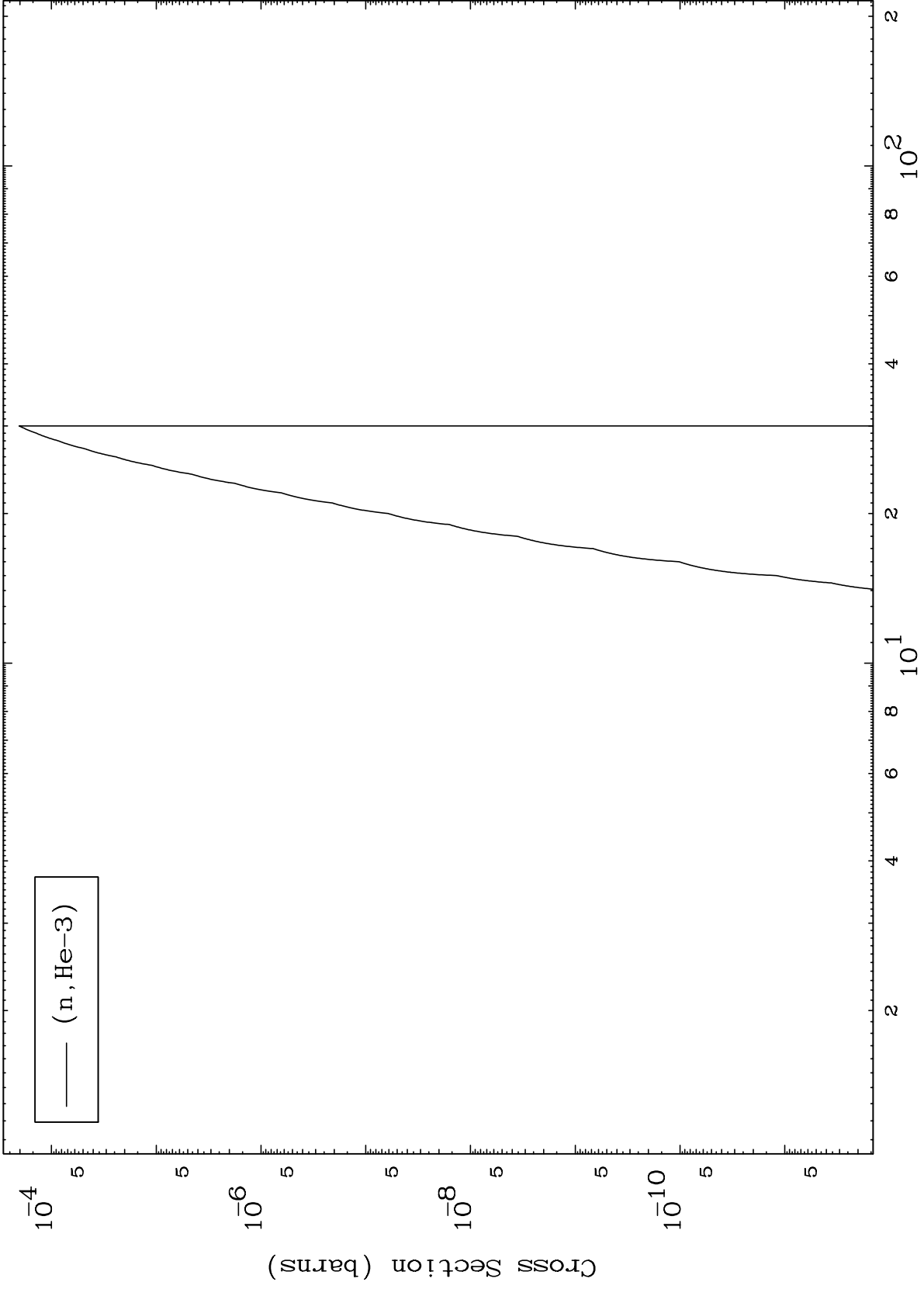
Incident Energy (MeV)

97-Bk-243

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(n,He3) Levels
293 Kelvin Cross Sections

97-Bk-243



14

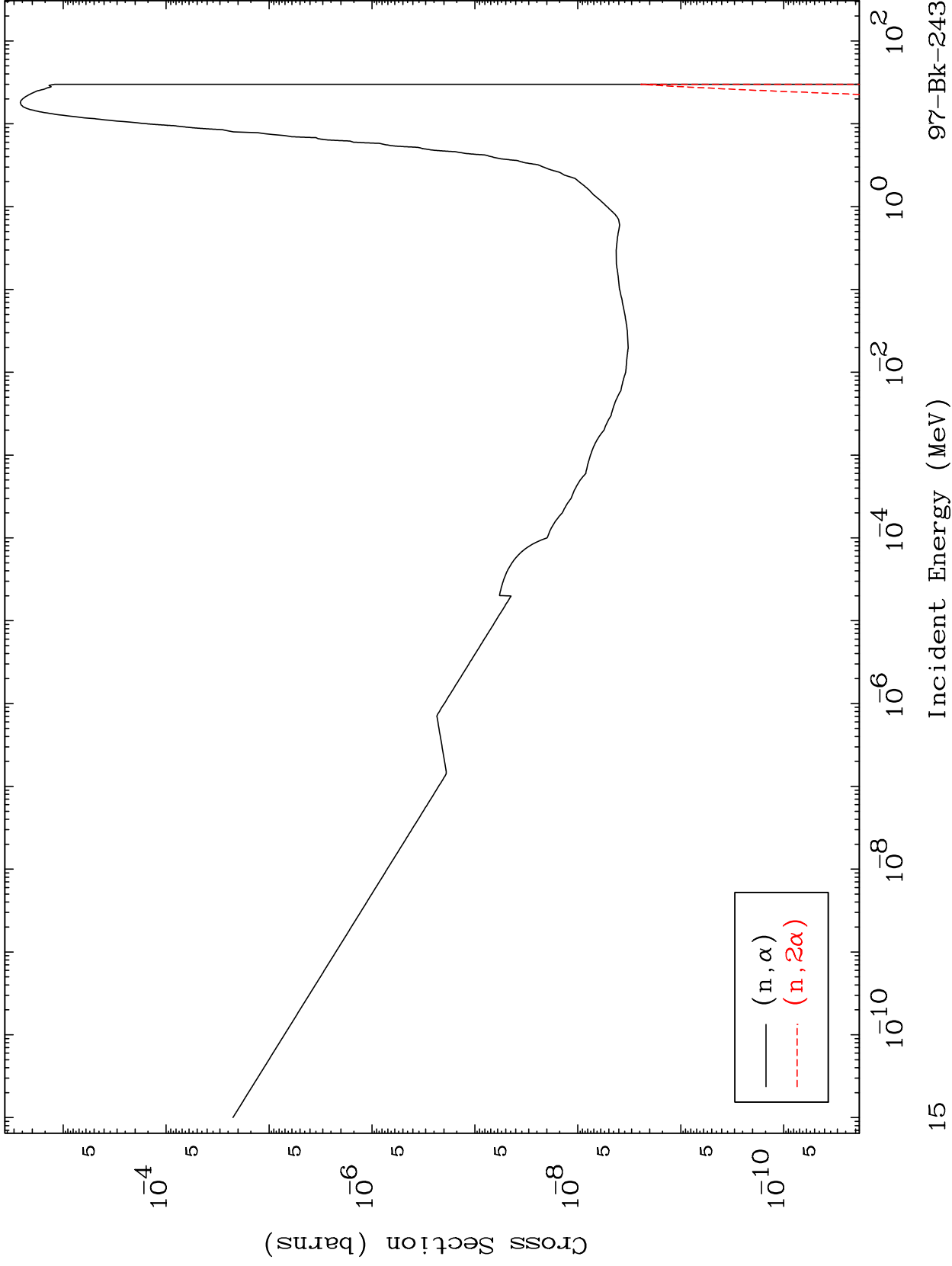
Incident Energy (MeV)

97-Bk-243

MAT 9734

(n, α) Levels
293 Kelvin Cross Sections

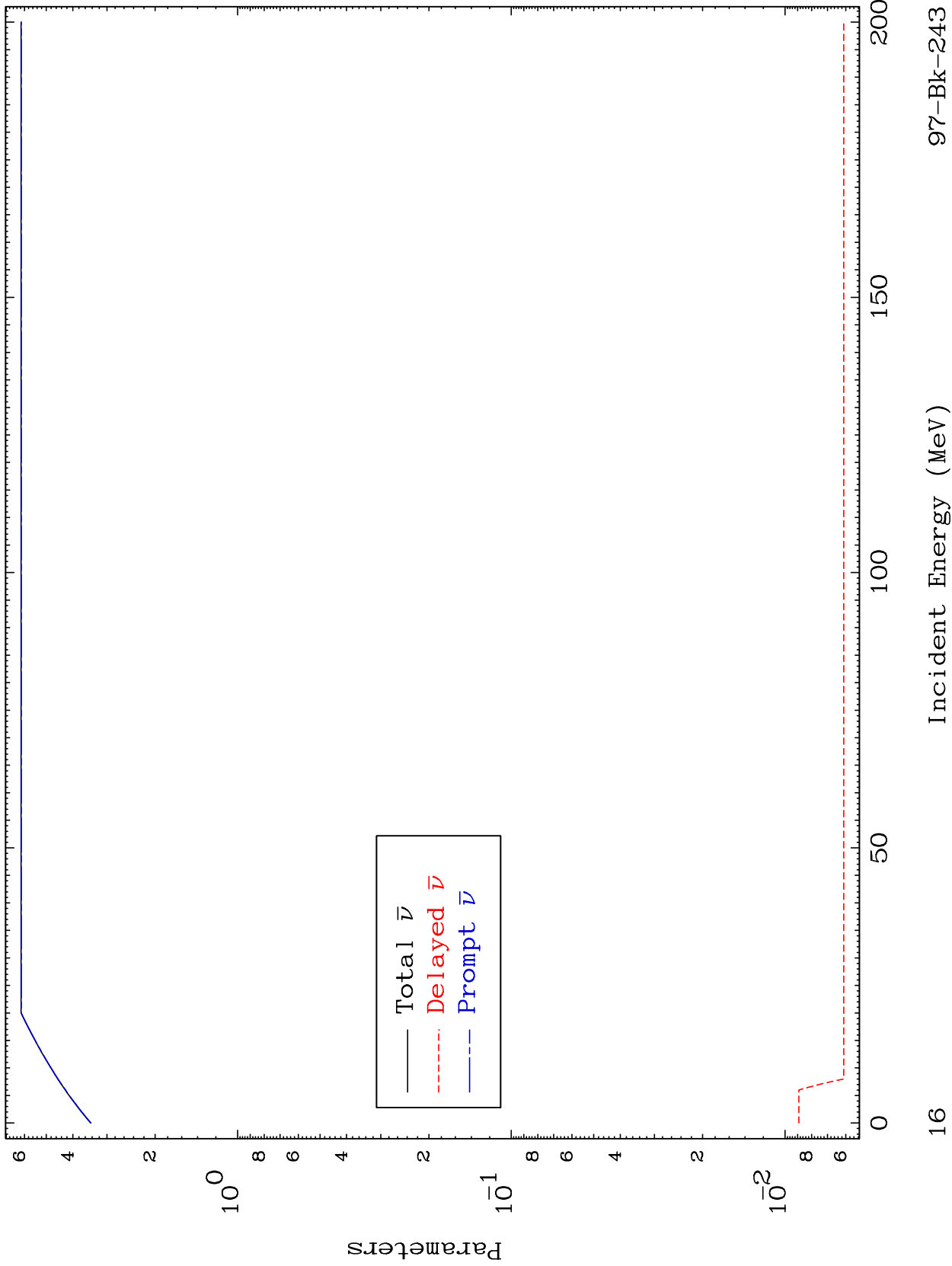
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Energy Release
Parameters

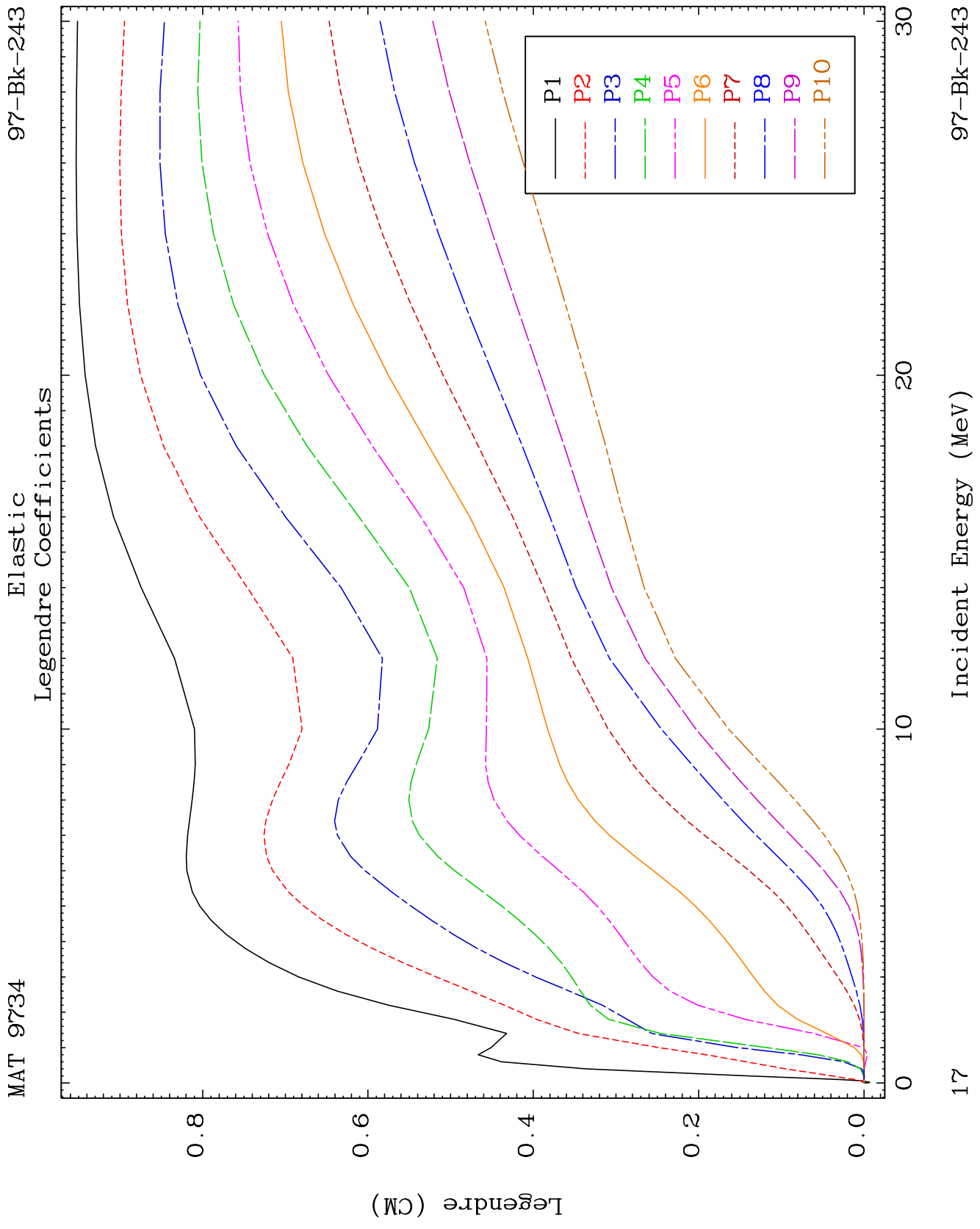
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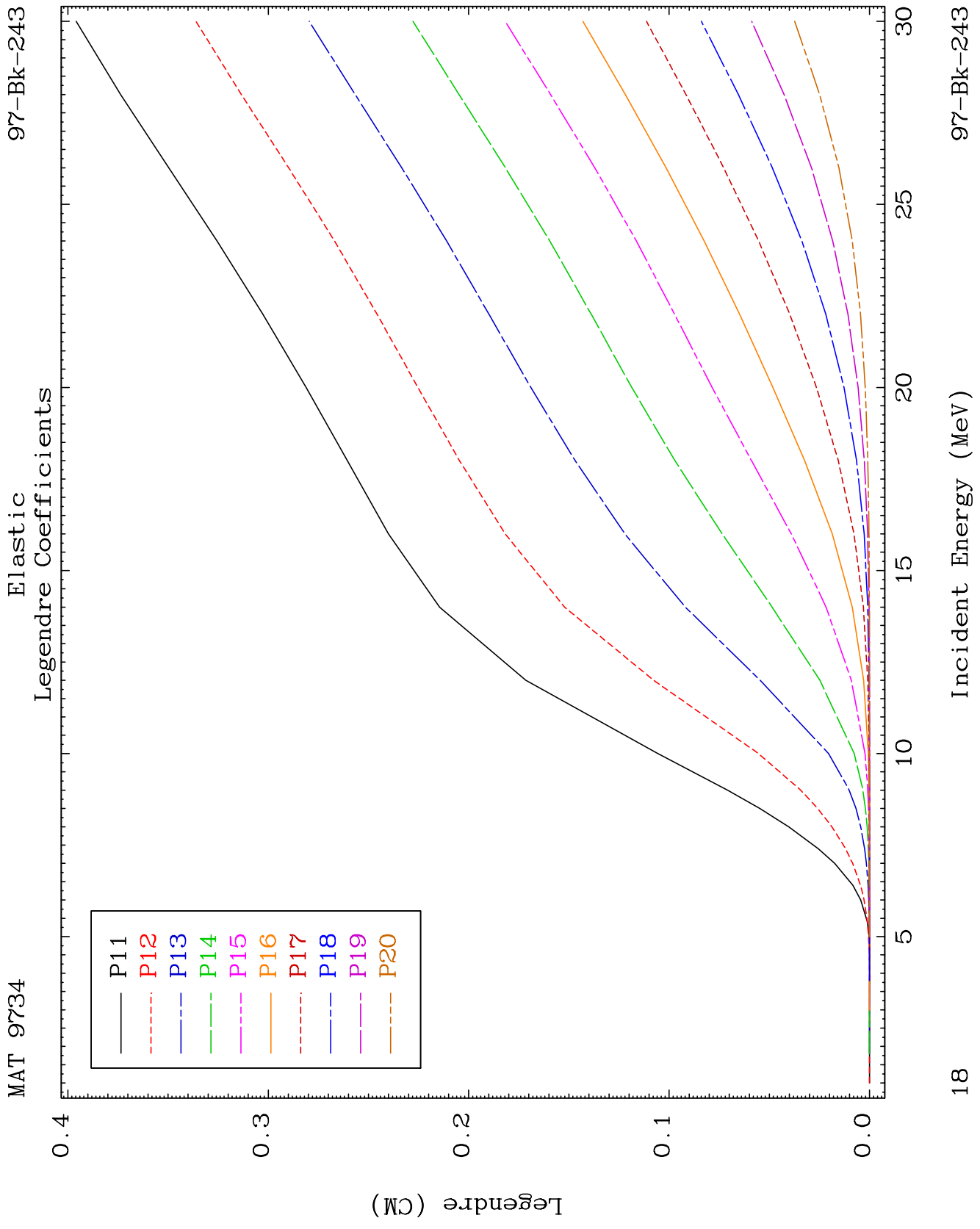


16

Incident Energy (MeV)

97-Bk-243

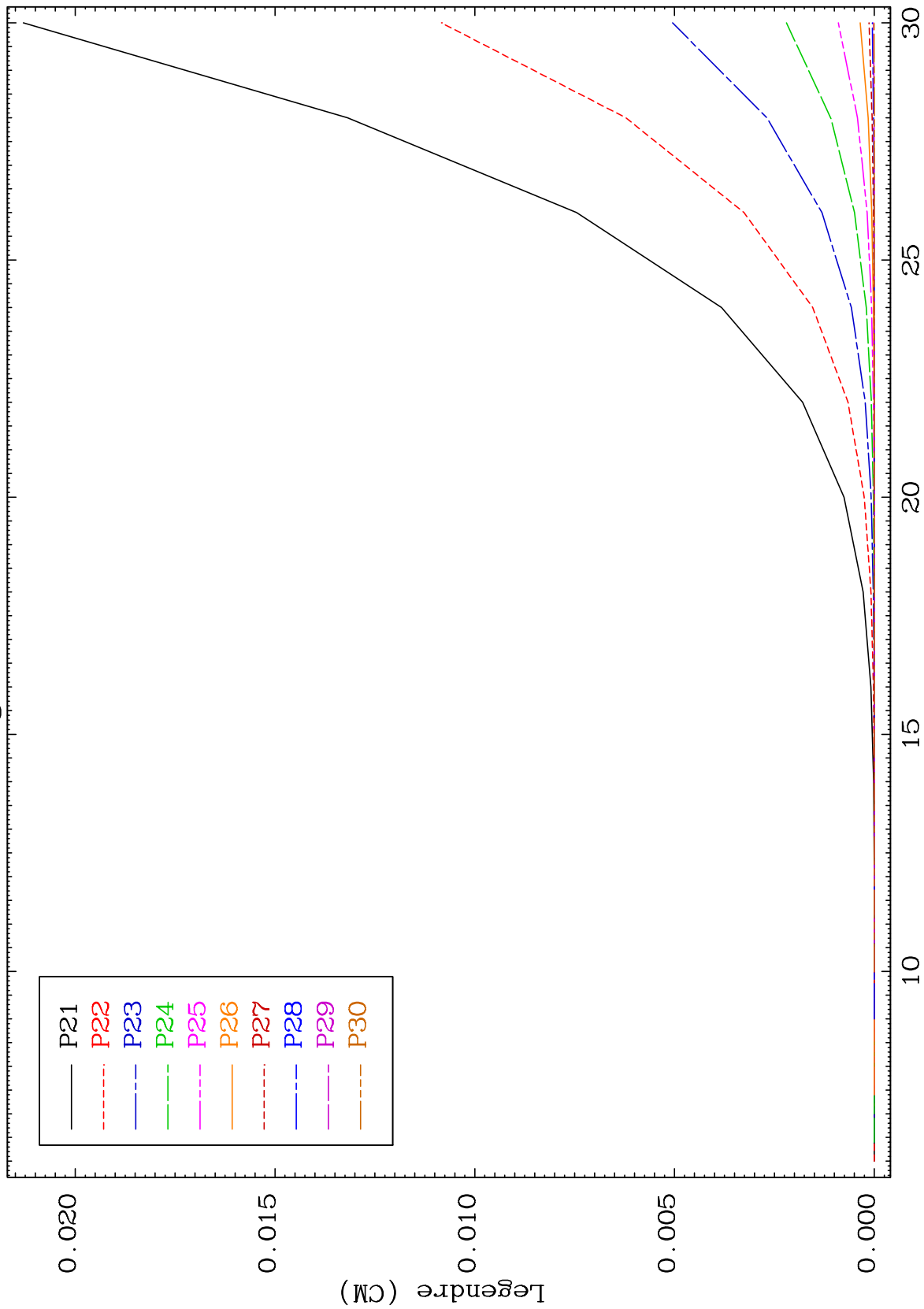




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Elastic Legendre Coefficients

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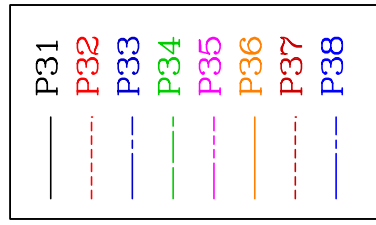
Incident Energy (MeV)

97-Bk-243

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Elastic
Legendre Coefficients

97-Bk-243



$\times 10^{-6}$

Legendre (CM)

1.0

0.5

0.0

15

20

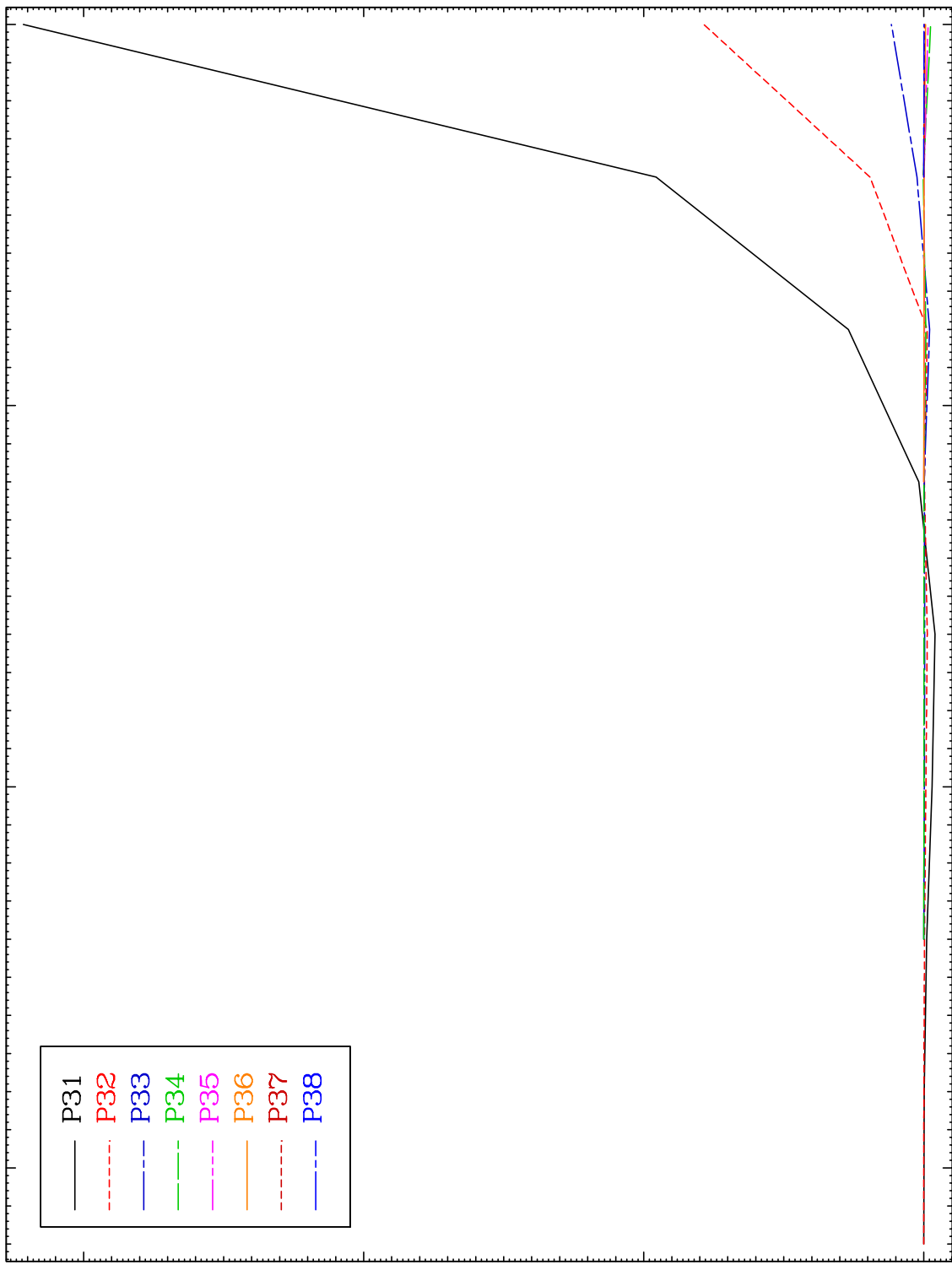
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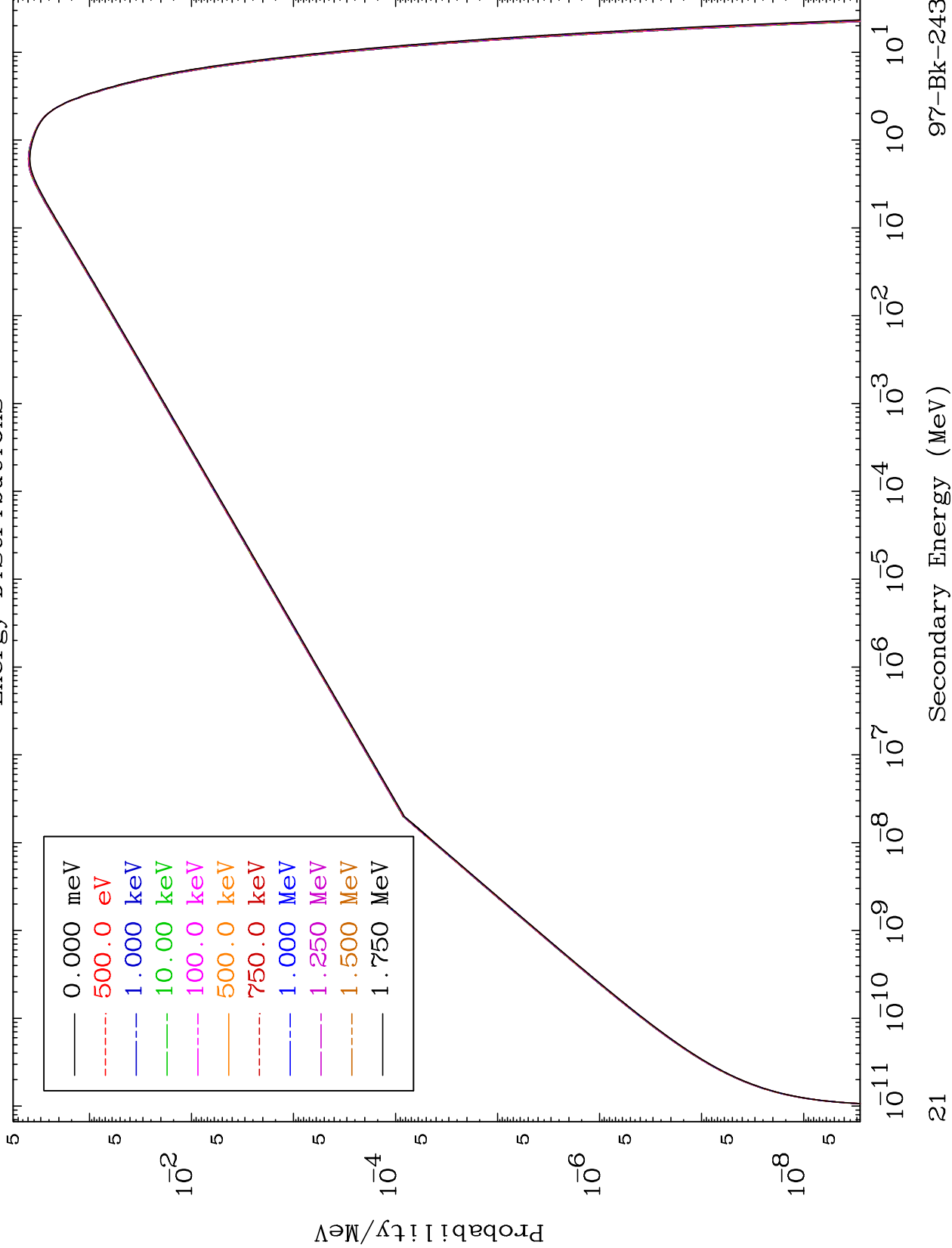
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Incident Energy (MeV)

97-Bk-243



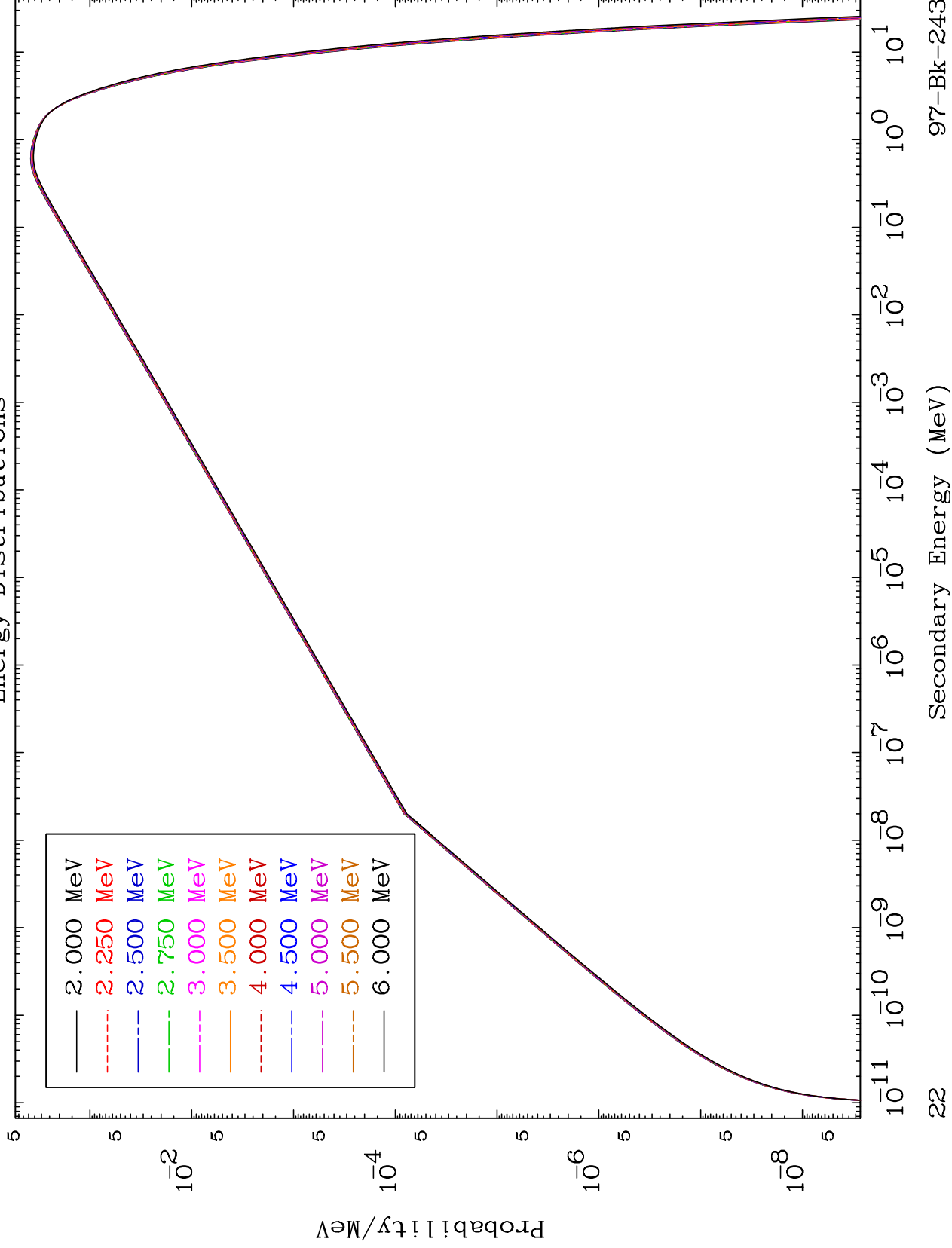
Fission Energy Distributions



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Fission Energy Distributions

97-Bk-243



97-Bk-243

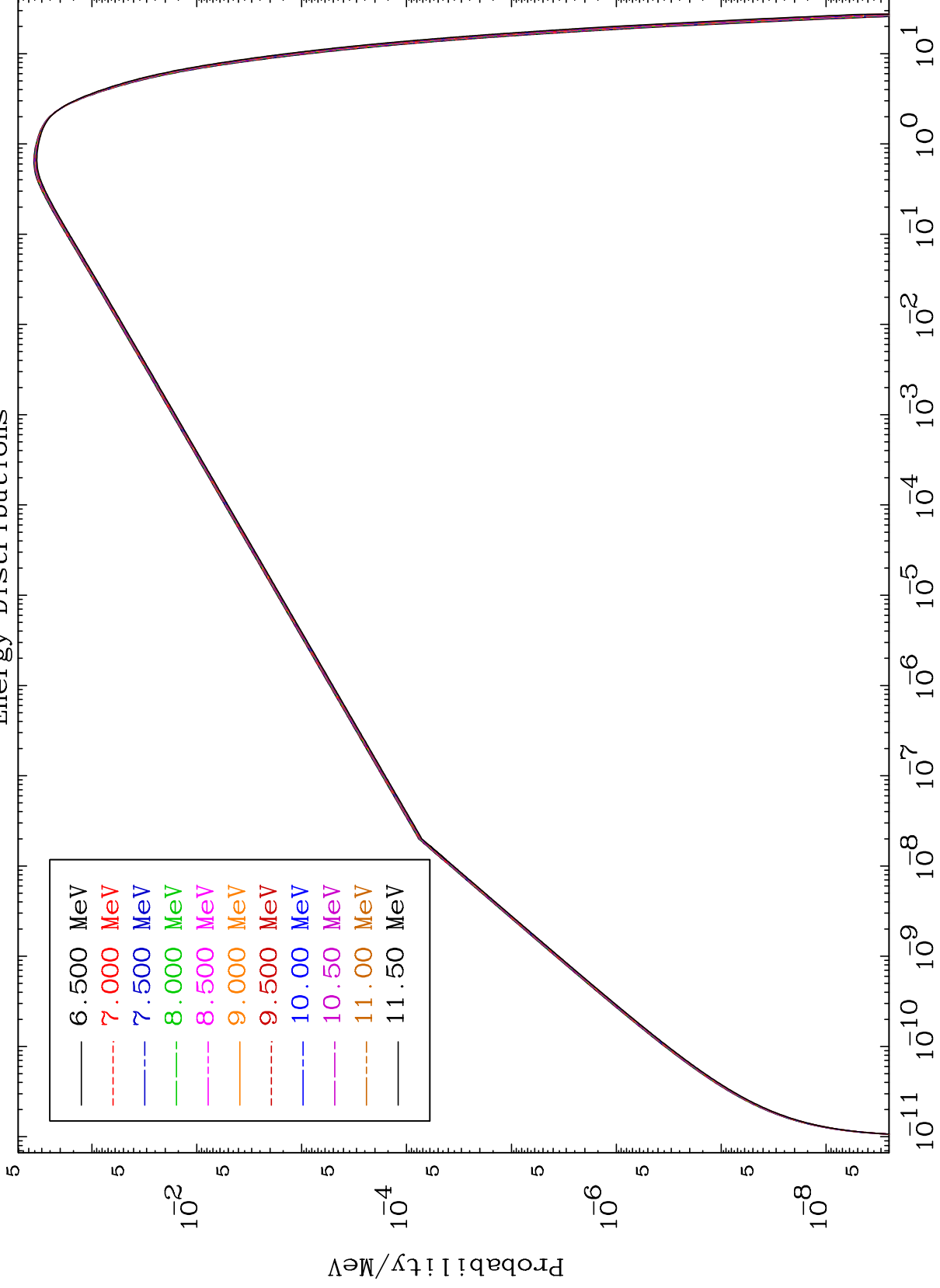
Secondary Energy (MeV)

22

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Fission
Energy Distributions

97-Bk-243



23

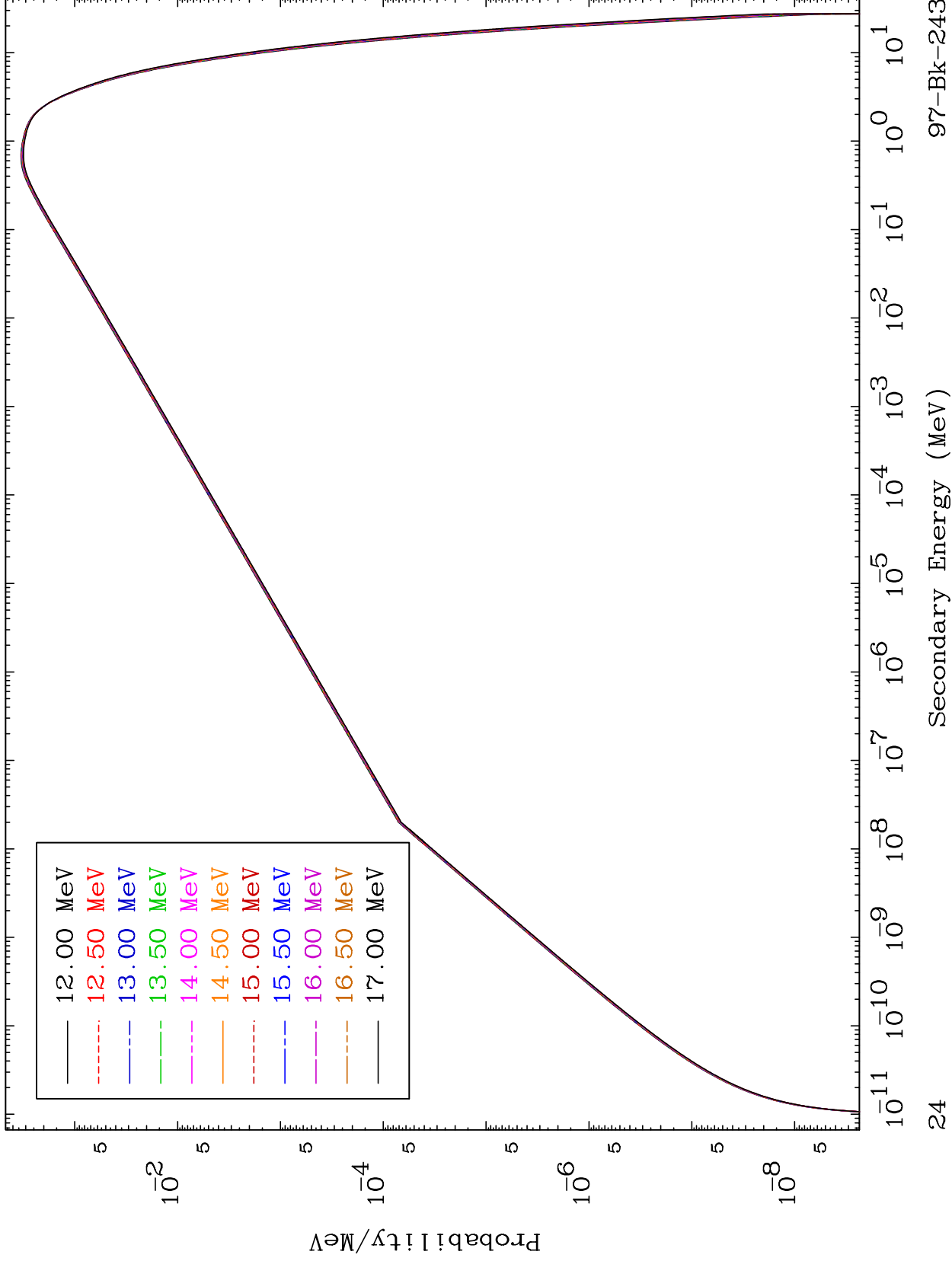
Secondary Energy (MeV)

97-Bk-243

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Fission Energy Distributions

97-Bk-243



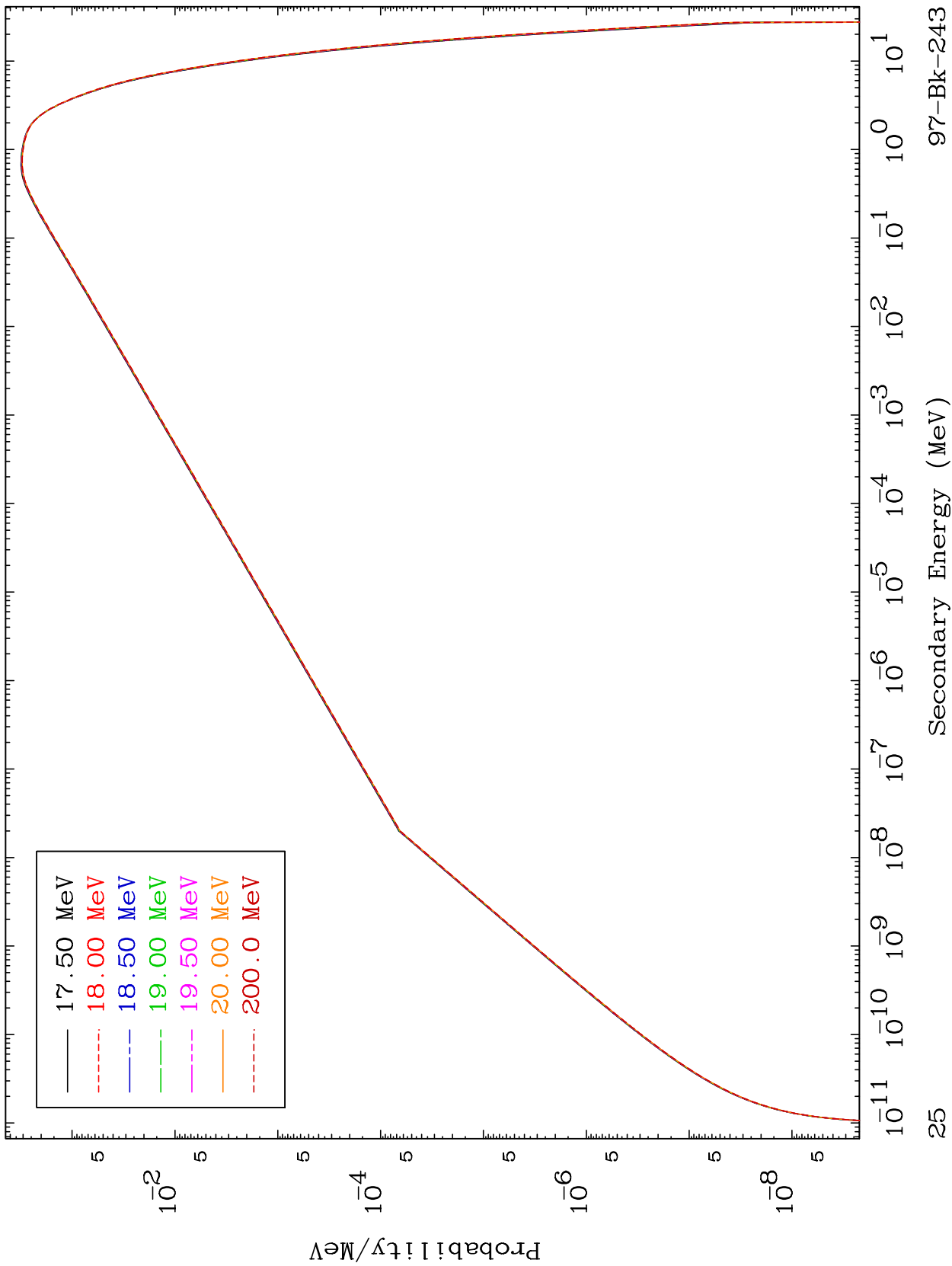
97-Bk-243

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Fission Energy Distributions

97-Bk-243

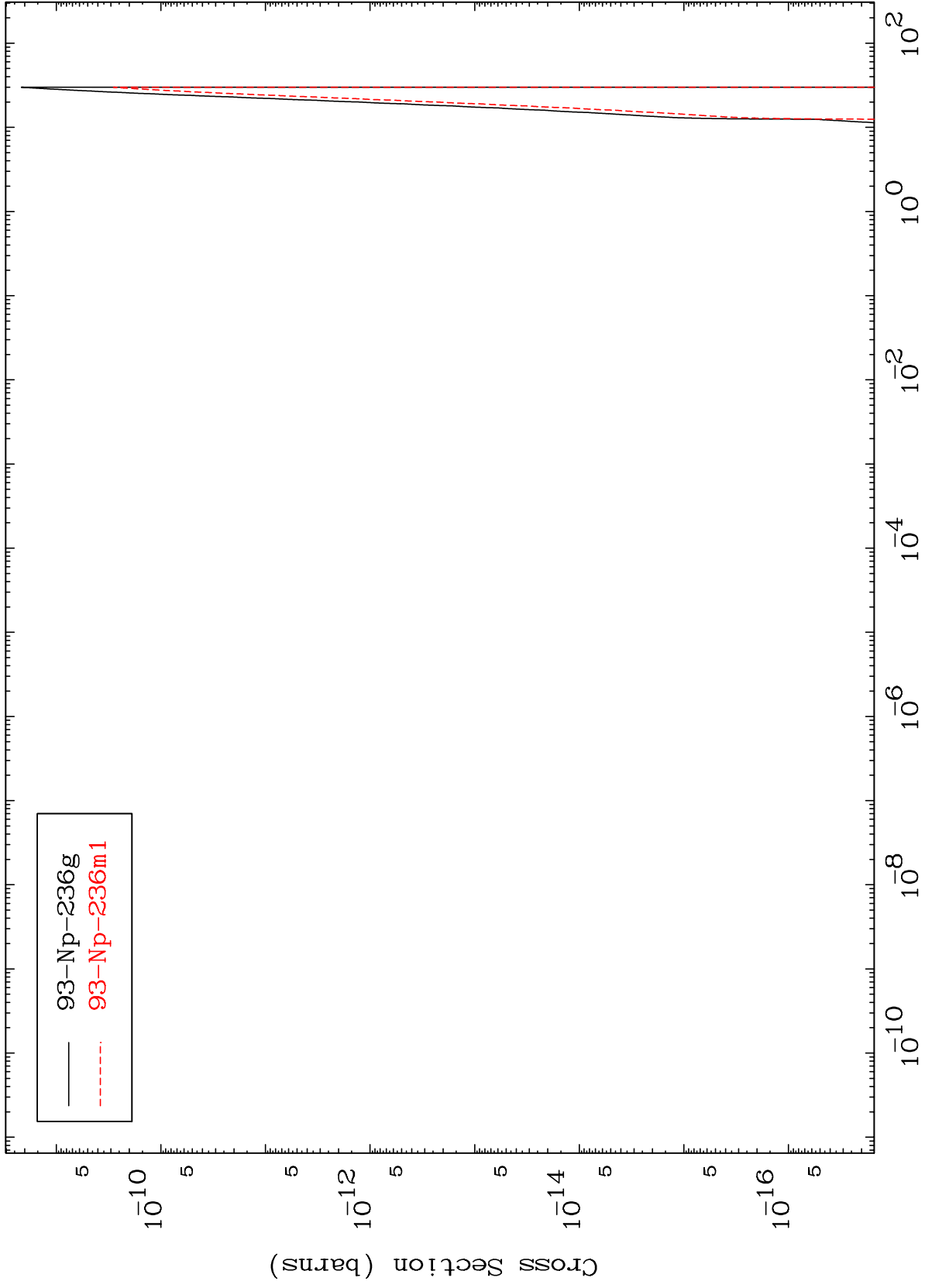


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(n,2α)

97-Bk-243

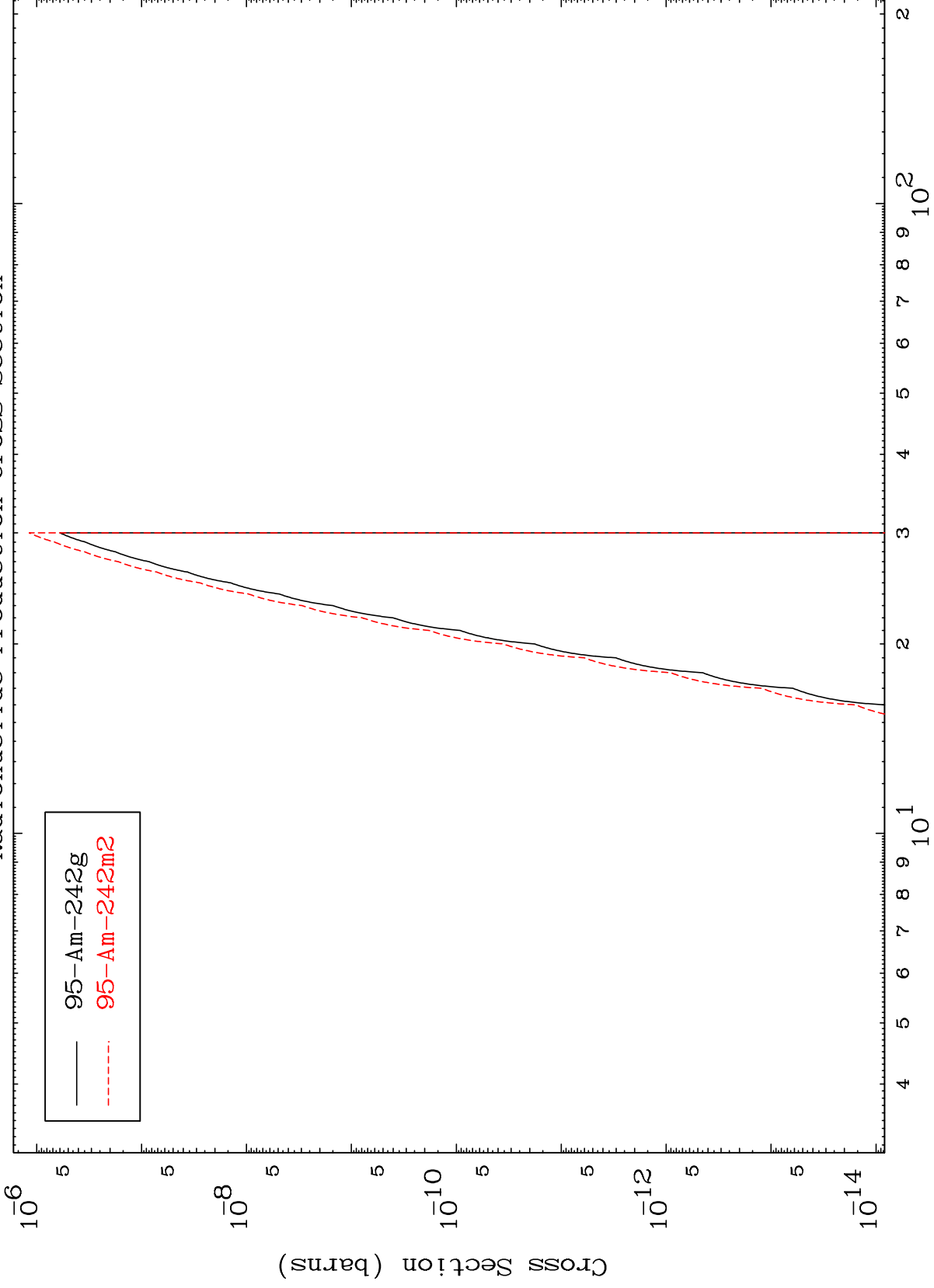
Radionuclide Production Cross Section



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(n,2p)
Radionuclide Production Cross Section



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Incident Energy (MeV)

97-Bk-243