

Program EVALPLOT  
(Version 2021-1)

by

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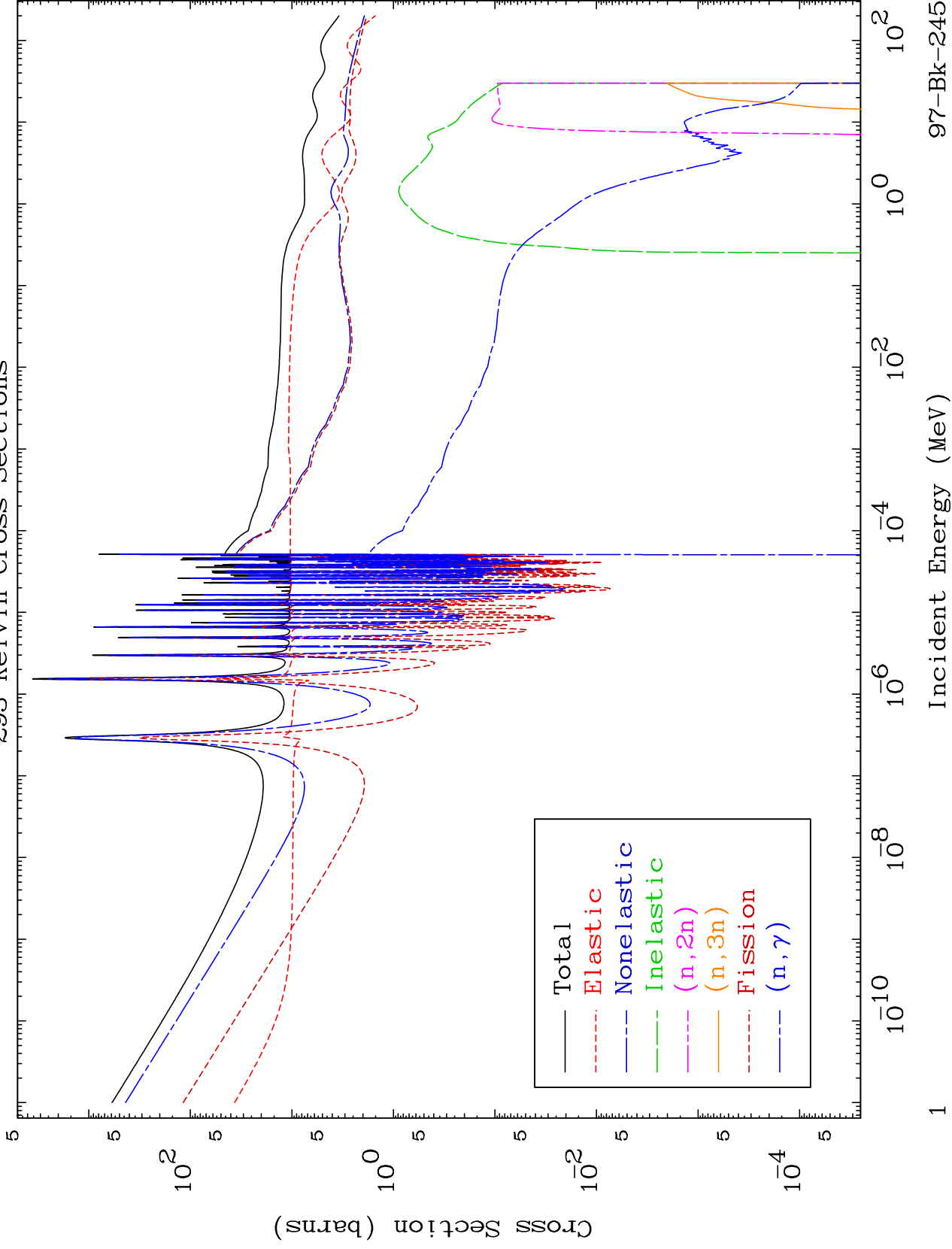
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Press Mouse Button to Start

MAT 9740

Neutron Major  
293 Kelvin Cross Sections

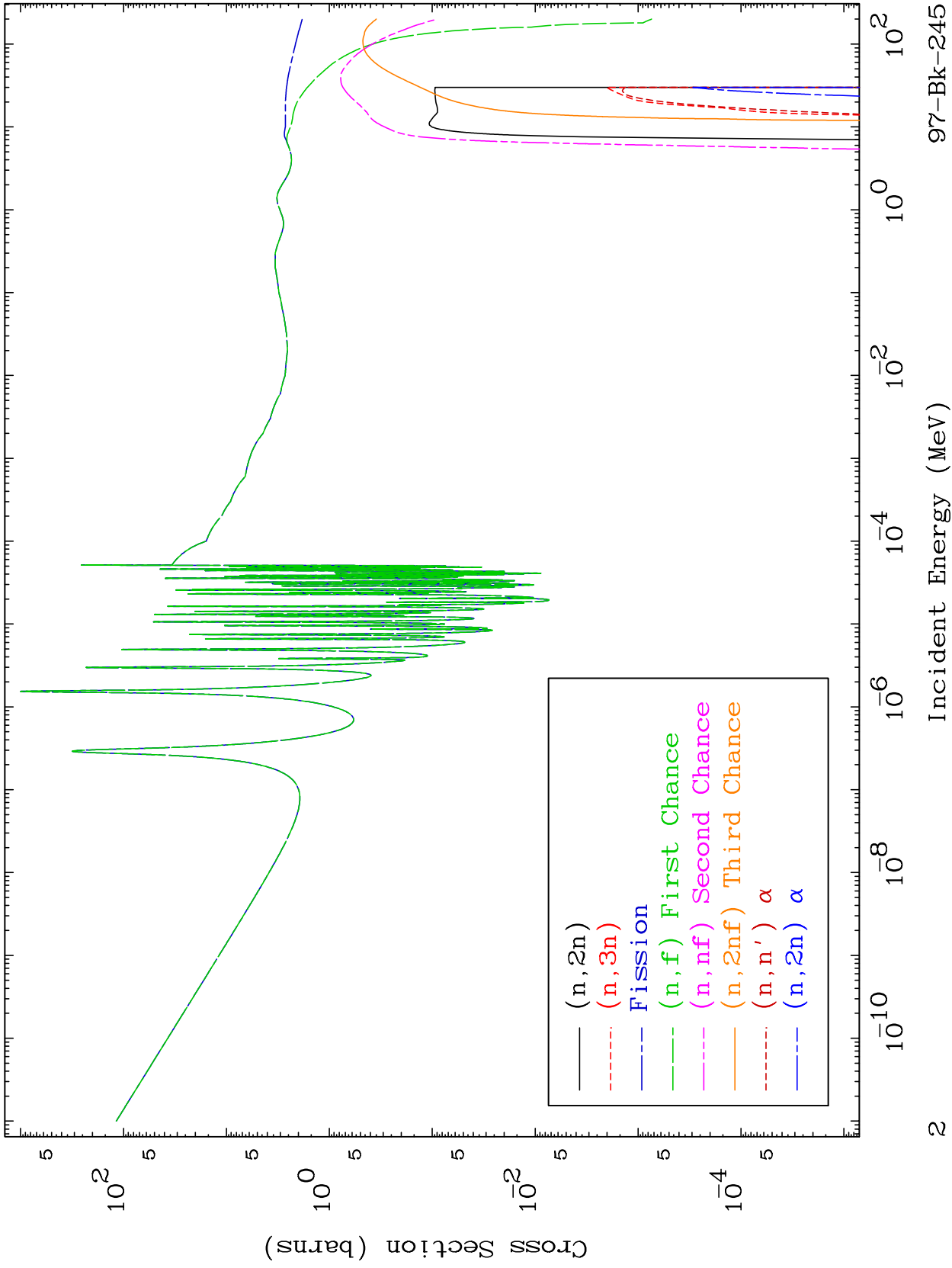
97-Bk-245



MAT 9740

Neutron Absorption  
293 Kelvin Cross Sections

97-Bk-245

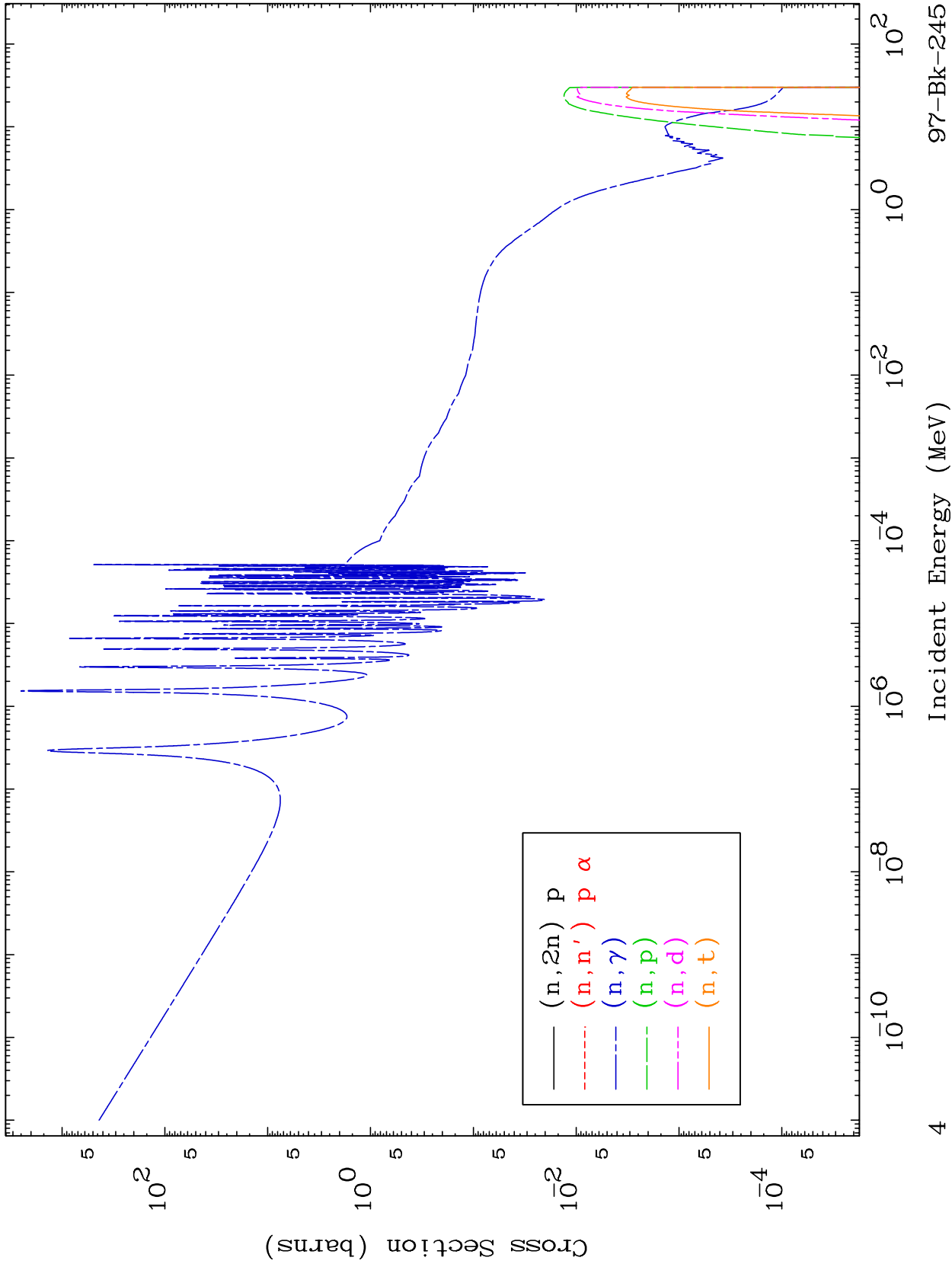




MAT 9740

Neutron Absorption  
293 Kelvin Cross Sections

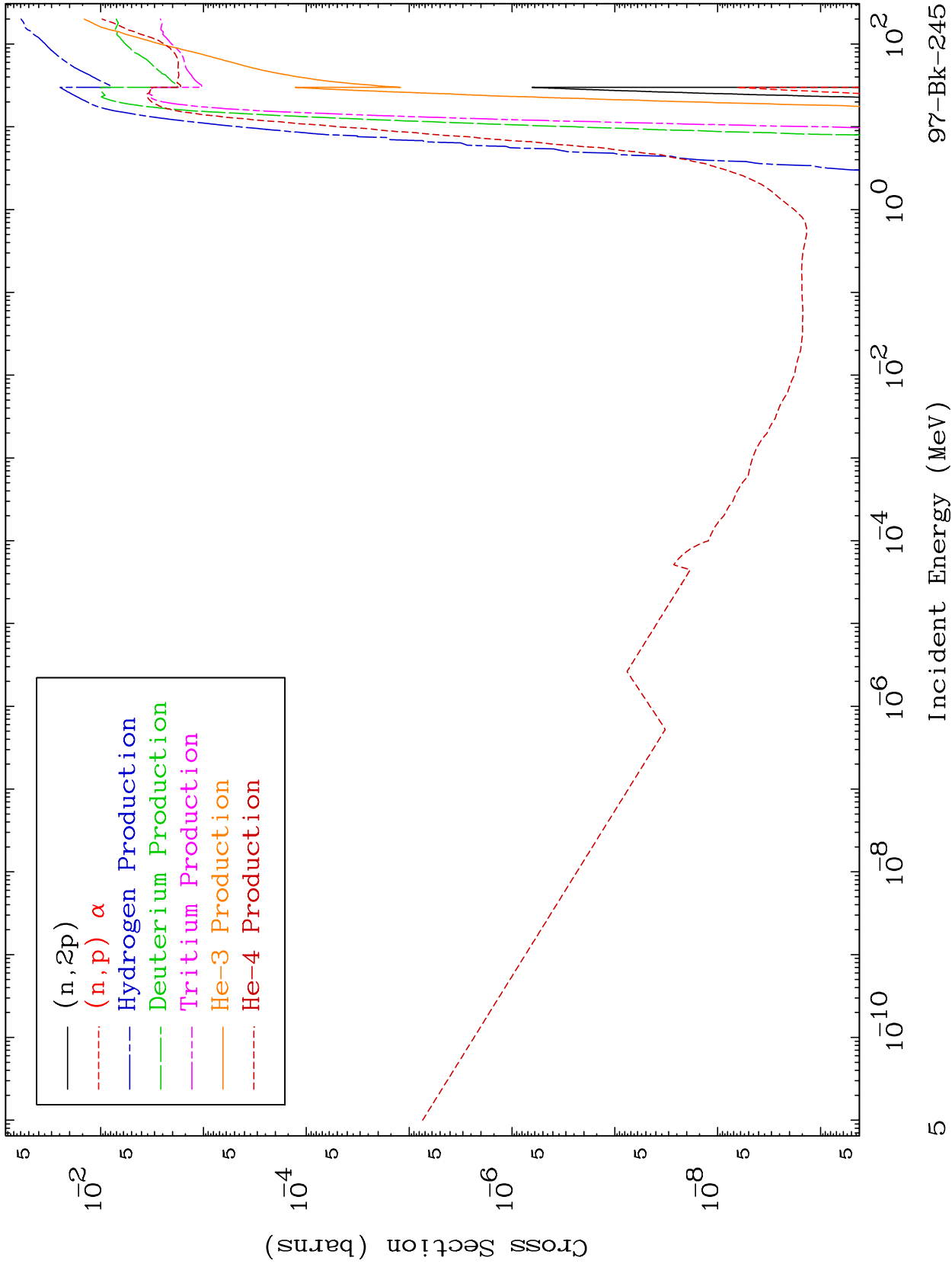
97-Bk-245



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Neutron Absorption  
293 Kelvin Cross Sections

97-Bk-245

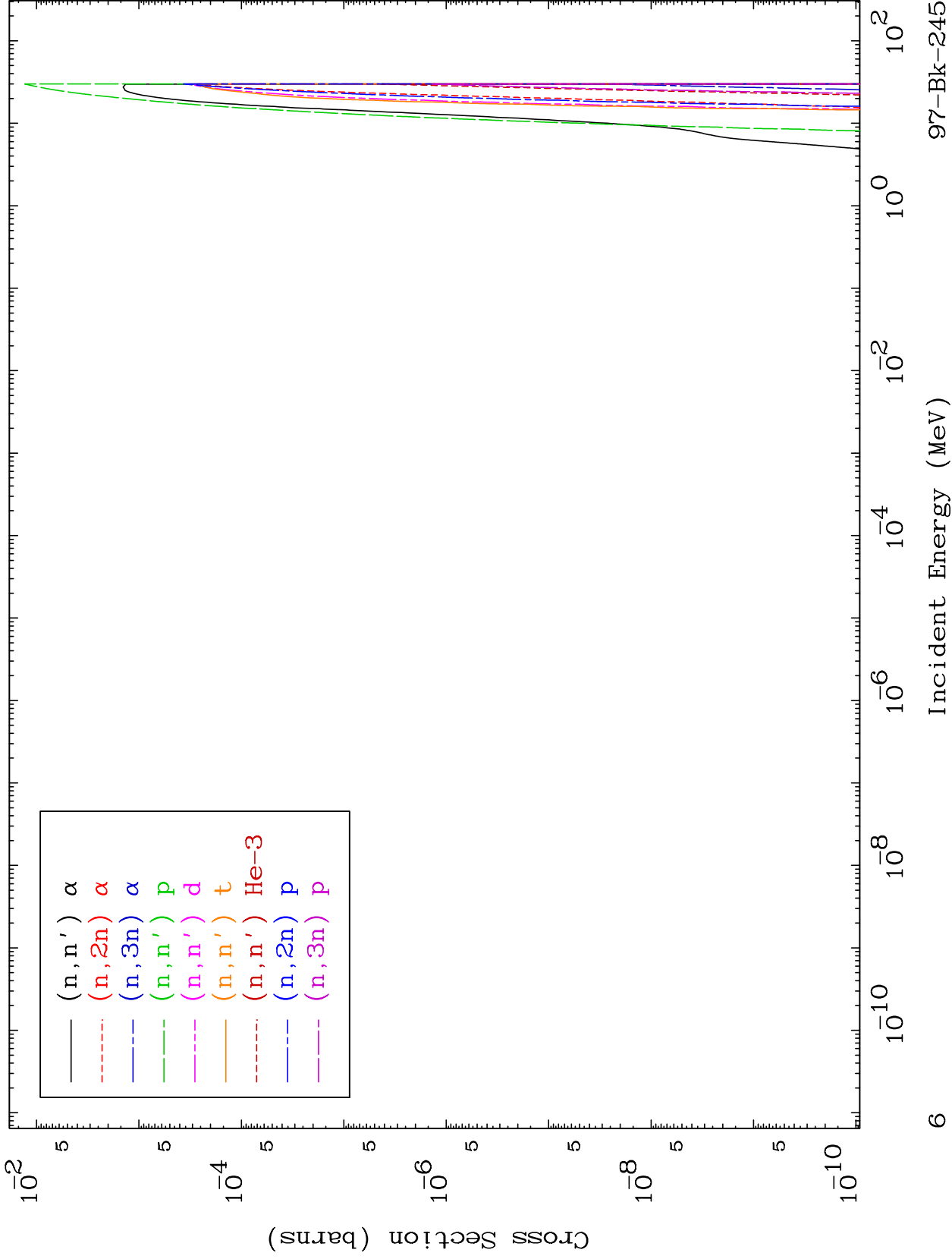


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Charged Particle  
293 Kelvin Cross Sections

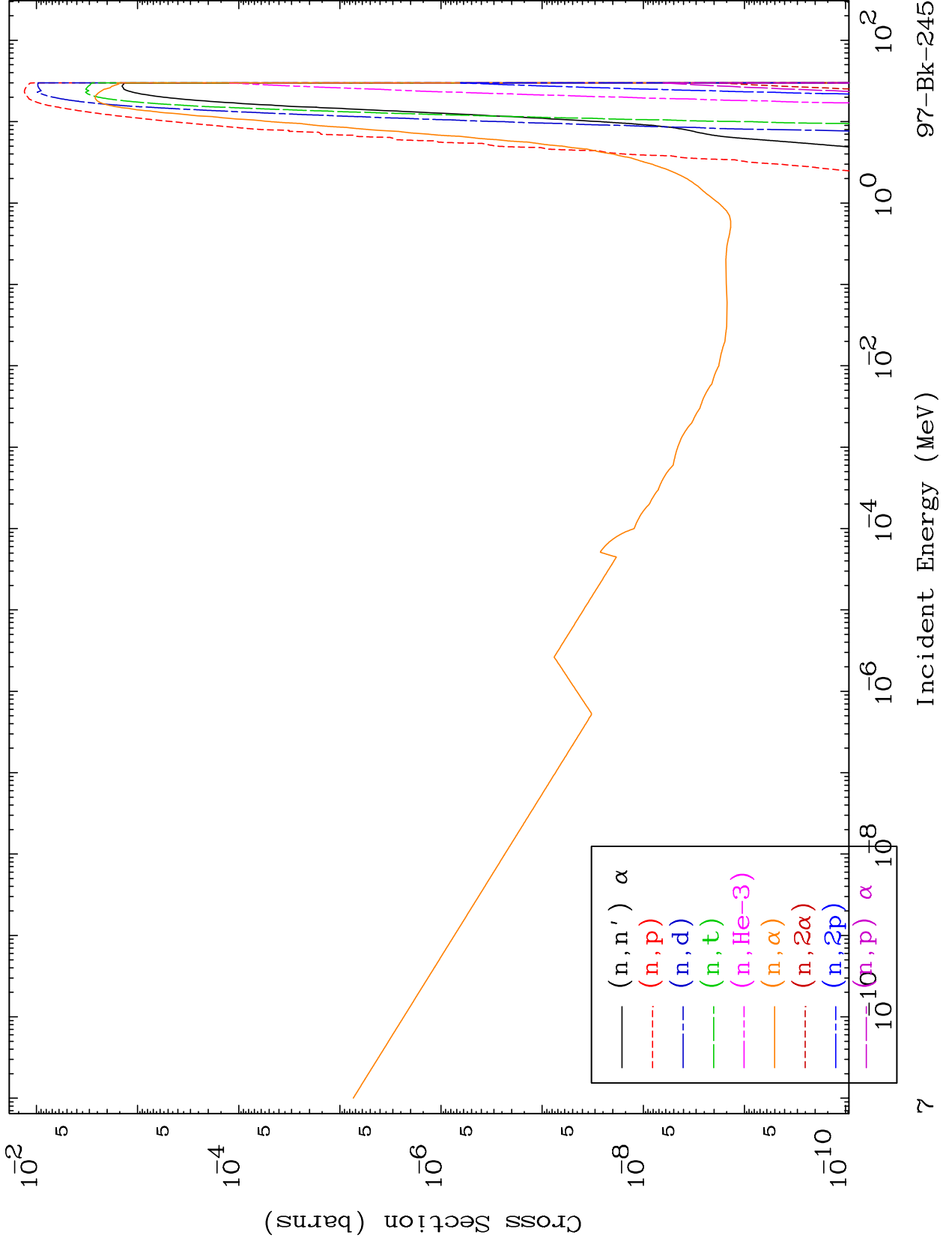
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Charged Particle  
293 Kelvin Cross Sections

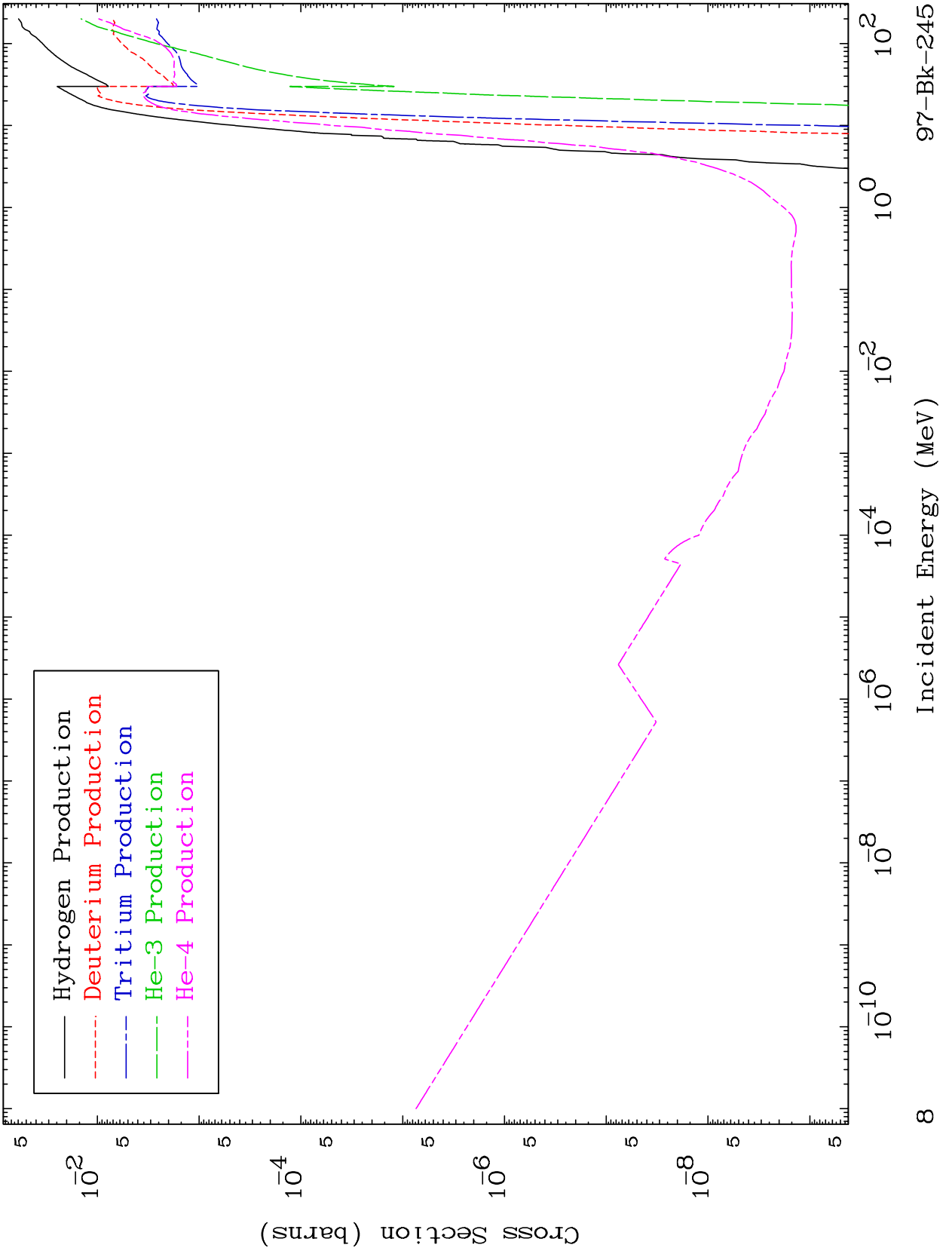
97-Bk-245



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Particle Production  
293 Kelvin Cross Sections

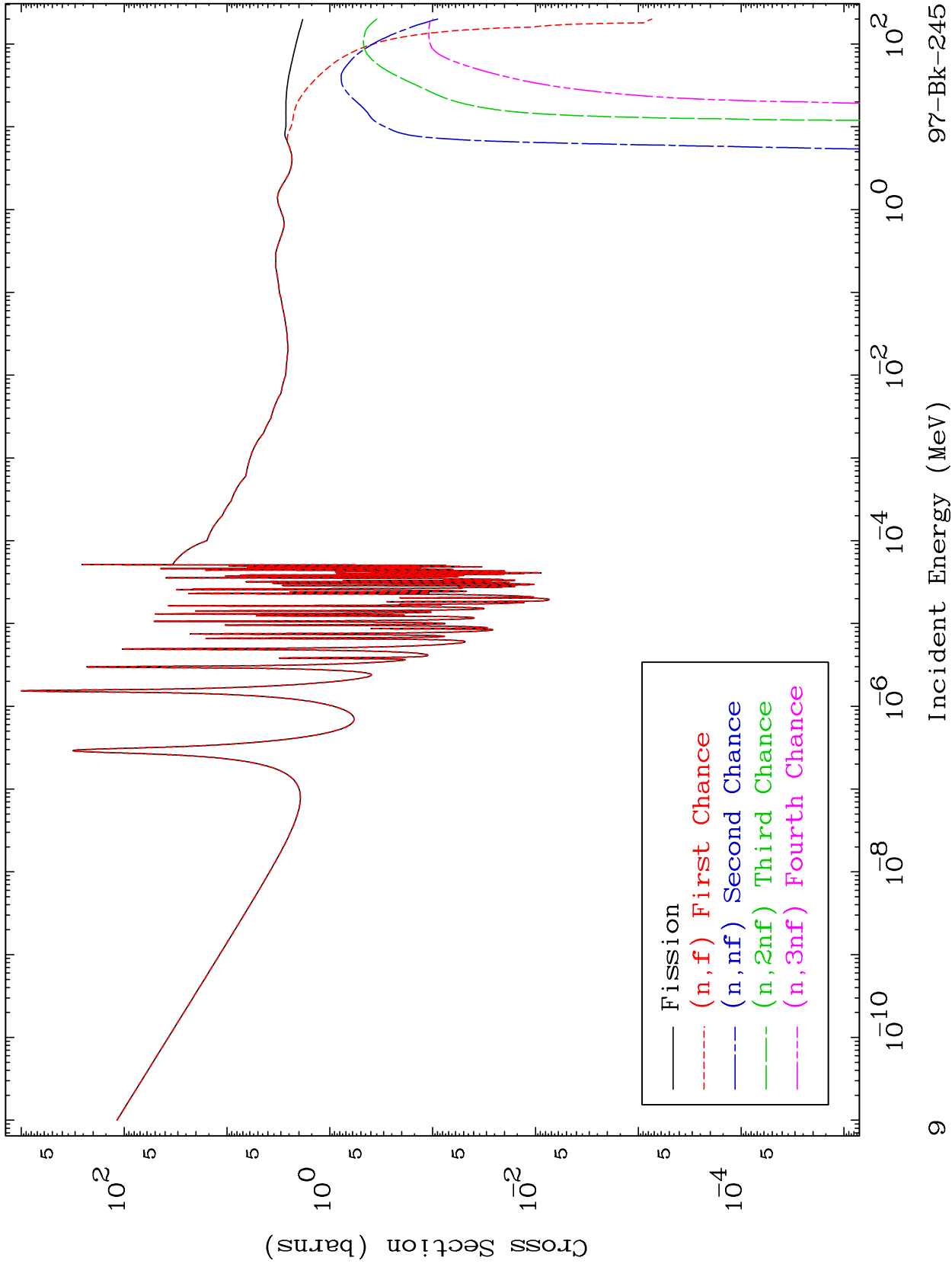
97-Bk-245



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Fission  
293 Kelvin Cross Sections

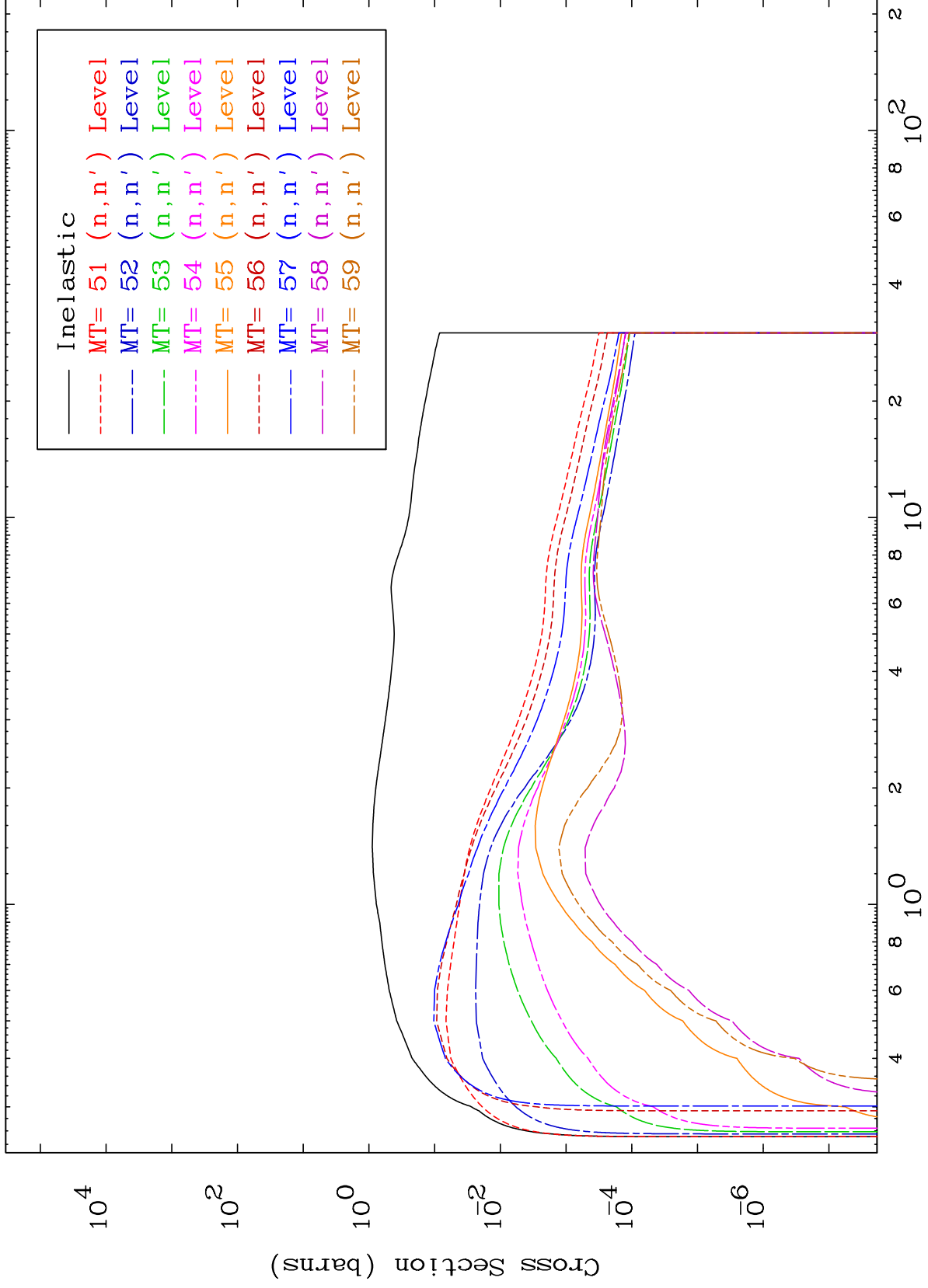
97-Bk-245



MAT 9740

(n,n') Levels  
293 Kelvin Cross Sections

97-Bk-245



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Incident Energy (MeV)

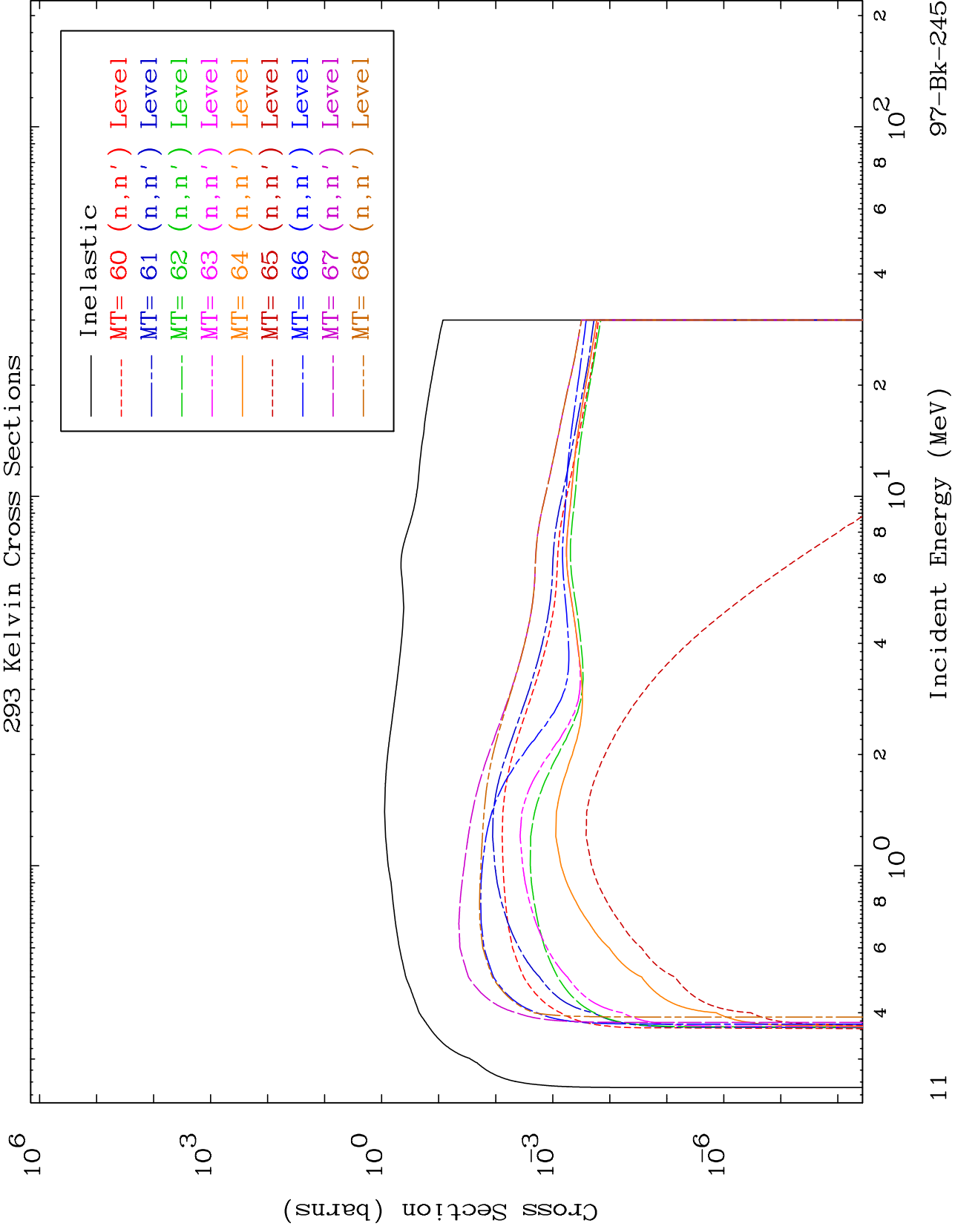
97-Bk-245

MAT 9740

(n,n') Levels

293 Kelvin Cross Sections

97-Bk-245



11

Incident Energy (MeV)

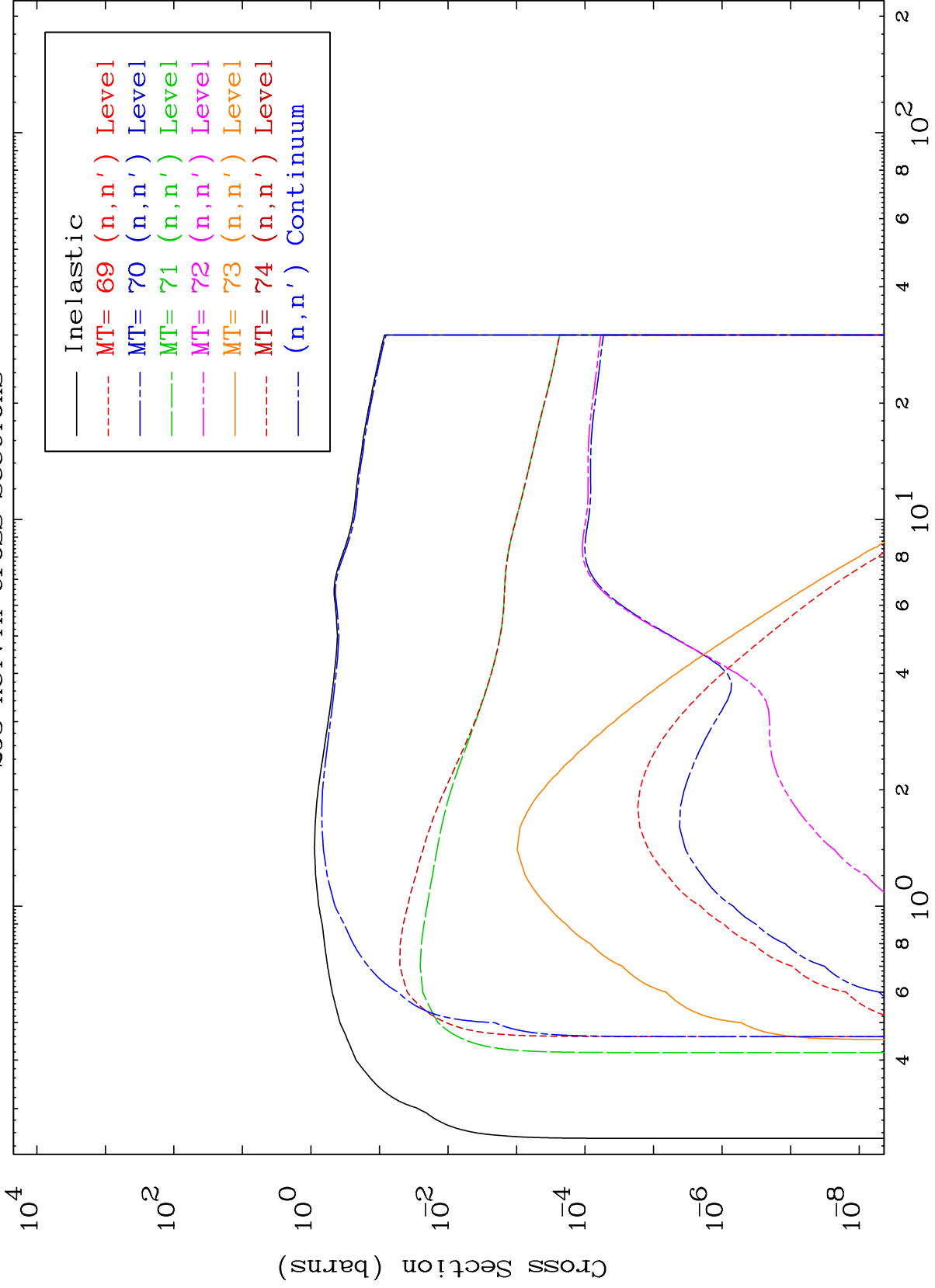
97-Bk-245

MAT 9740

(n,n') Levels

293 Kelvin Cross Sections

97-Bk-245



12

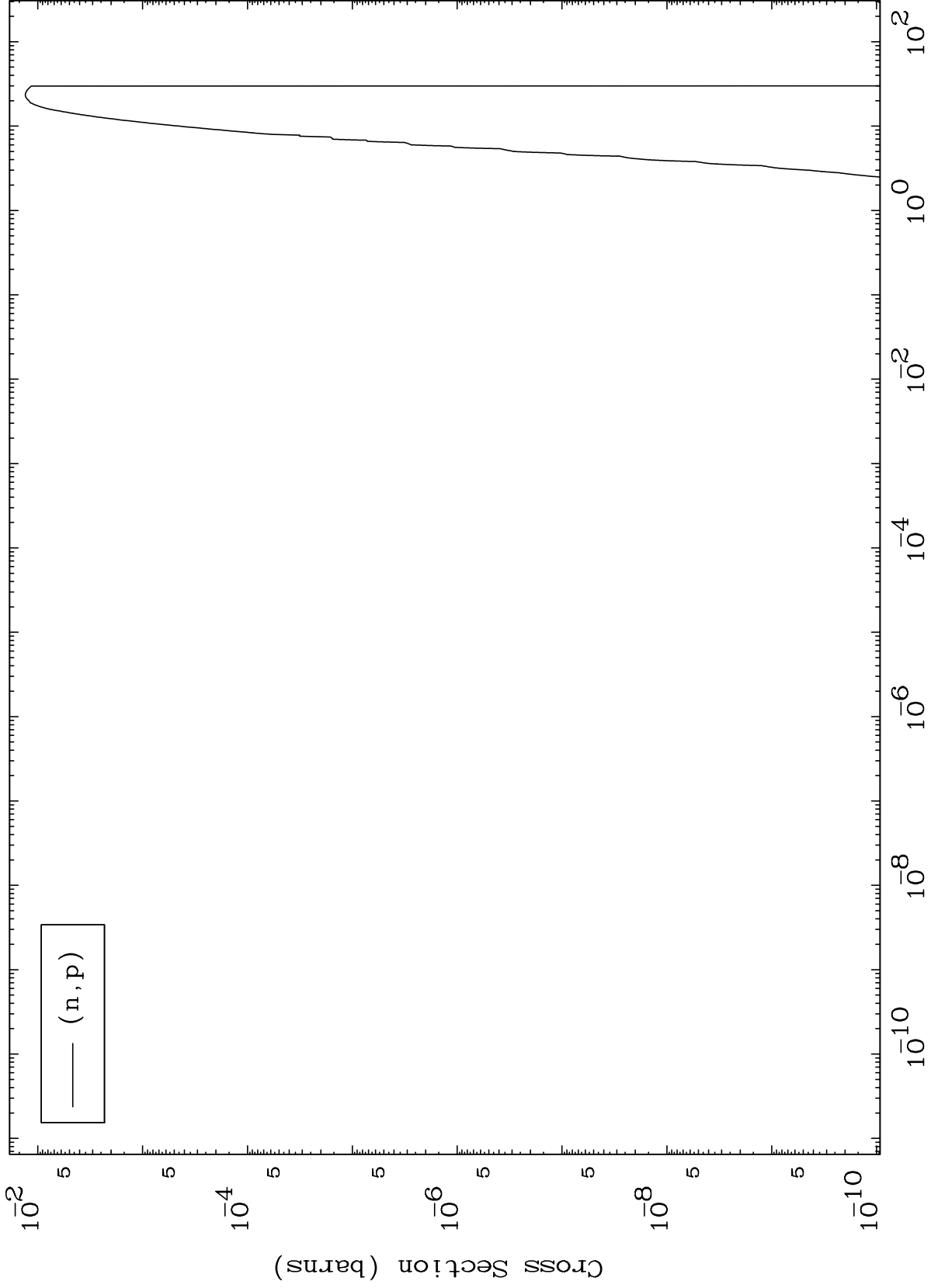
Incident Energy (MeV)

97-Bk-245

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(n,p) Levels  
293 Kelvin Cross Sections

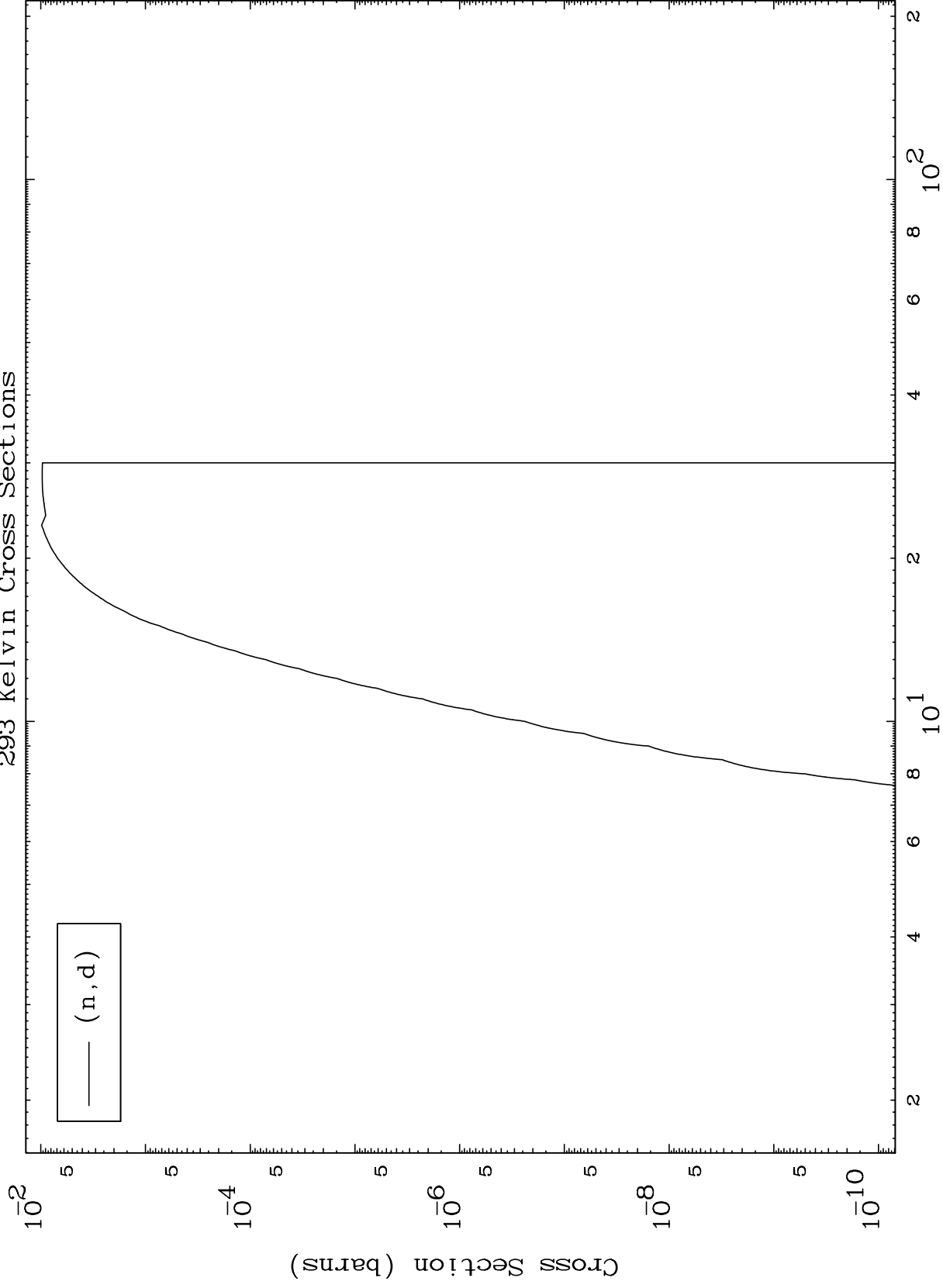
97-Bk-245



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(n,d) Levels  
293 Kelvin Cross Sections

97-Bk-245



14

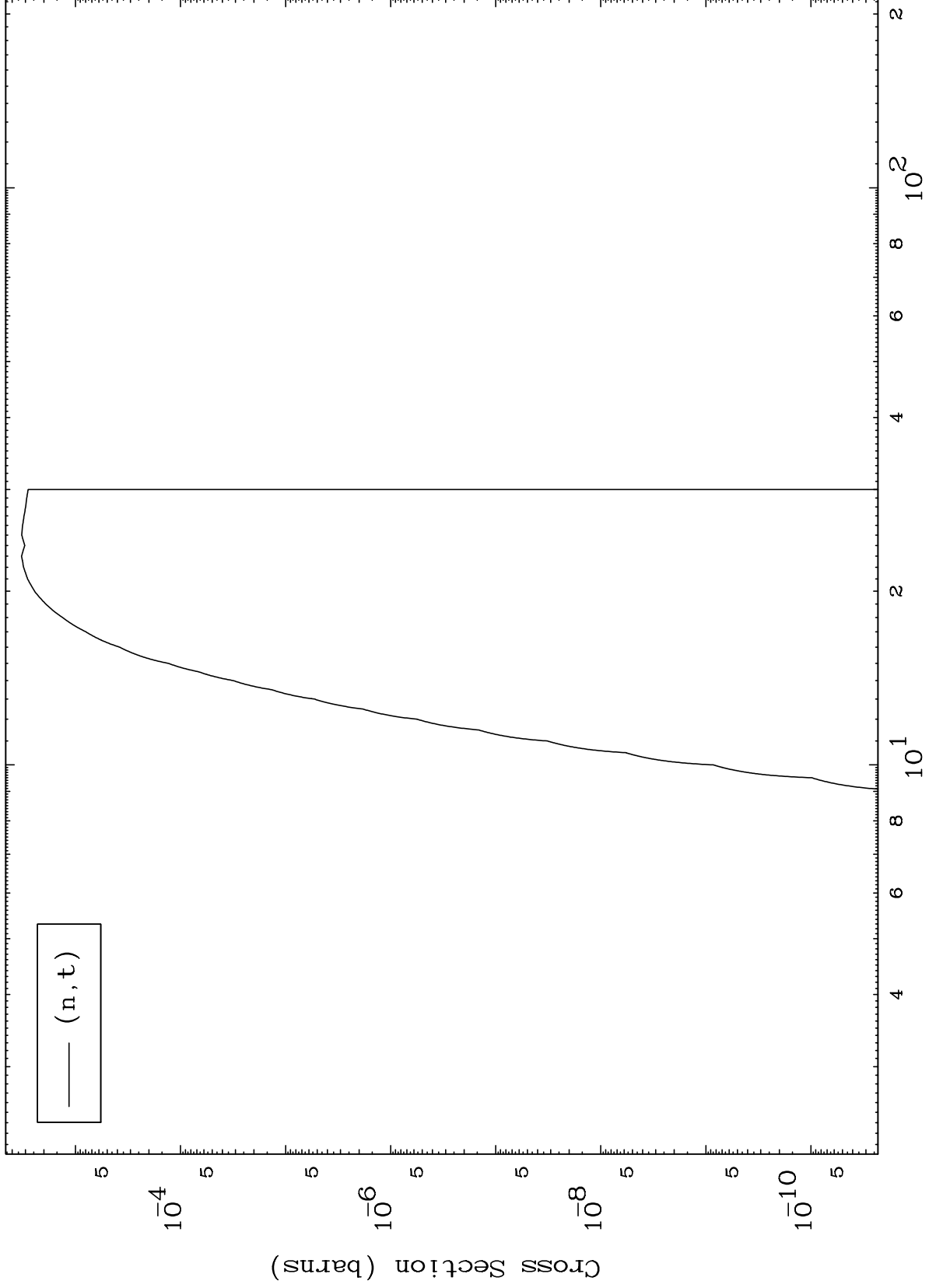
Incident Energy (MeV)

97-Bk-245

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(n,t) Levels  
293 Kelvin Cross Sections

97-Bk-245



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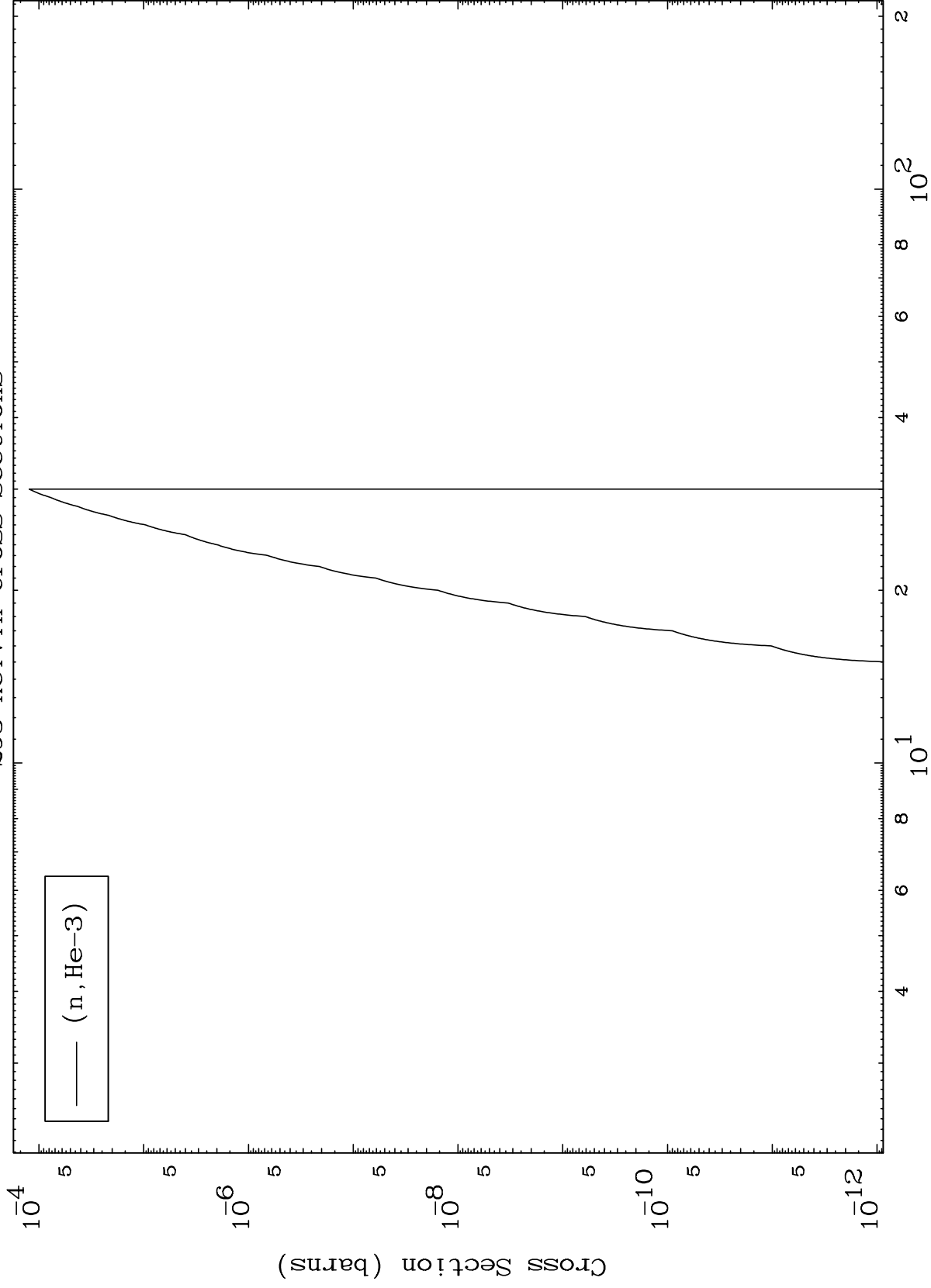
Incident Energy (MeV)

97-Bk-245

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(n,He3) Levels  
293 Kelvin Cross Sections

97-Bk-245



16

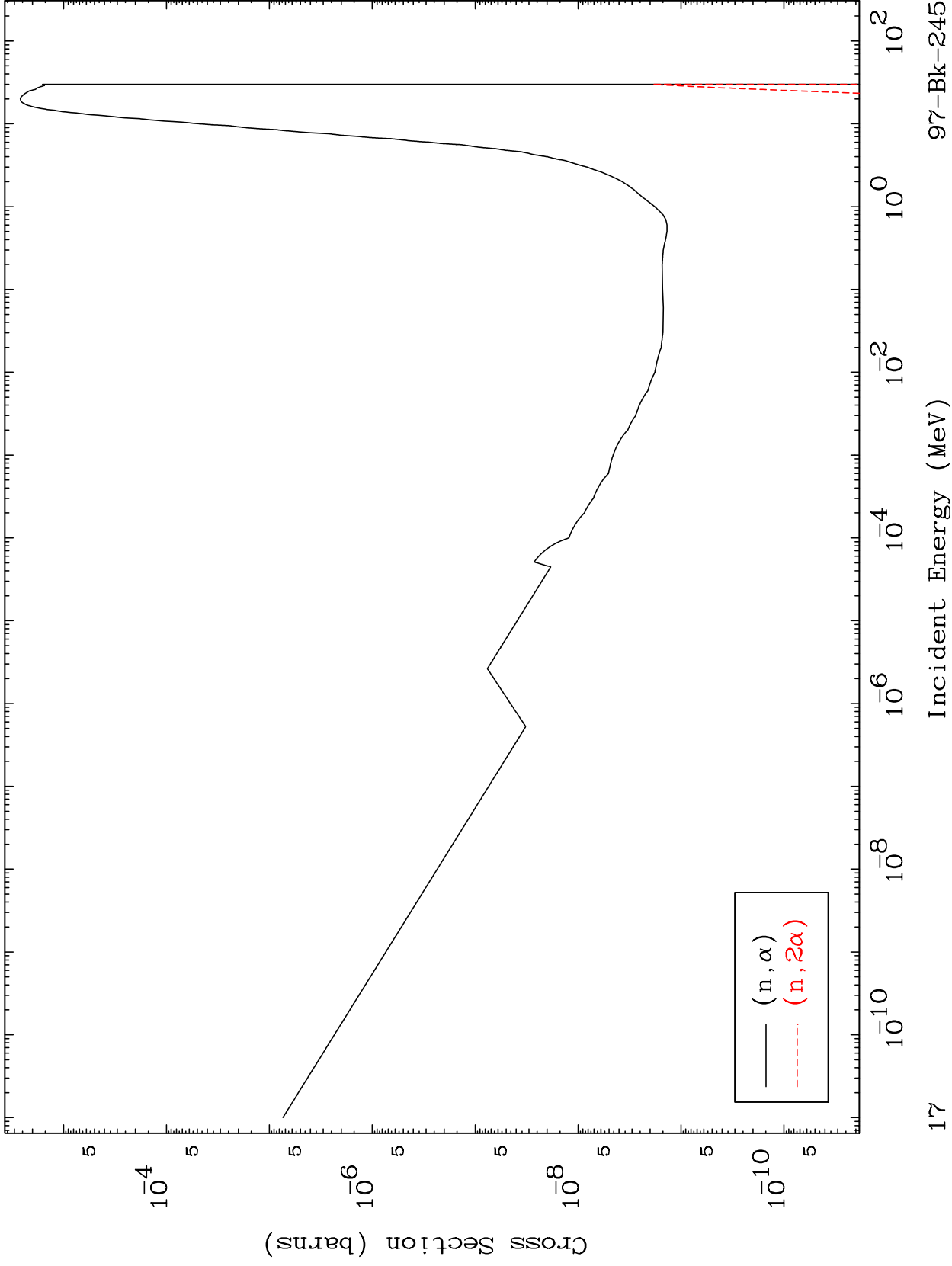
Incident Energy (MeV)

97-Bk-245

MAT 9740

(n,α) Levels  
293 Kelvin Cross Sections

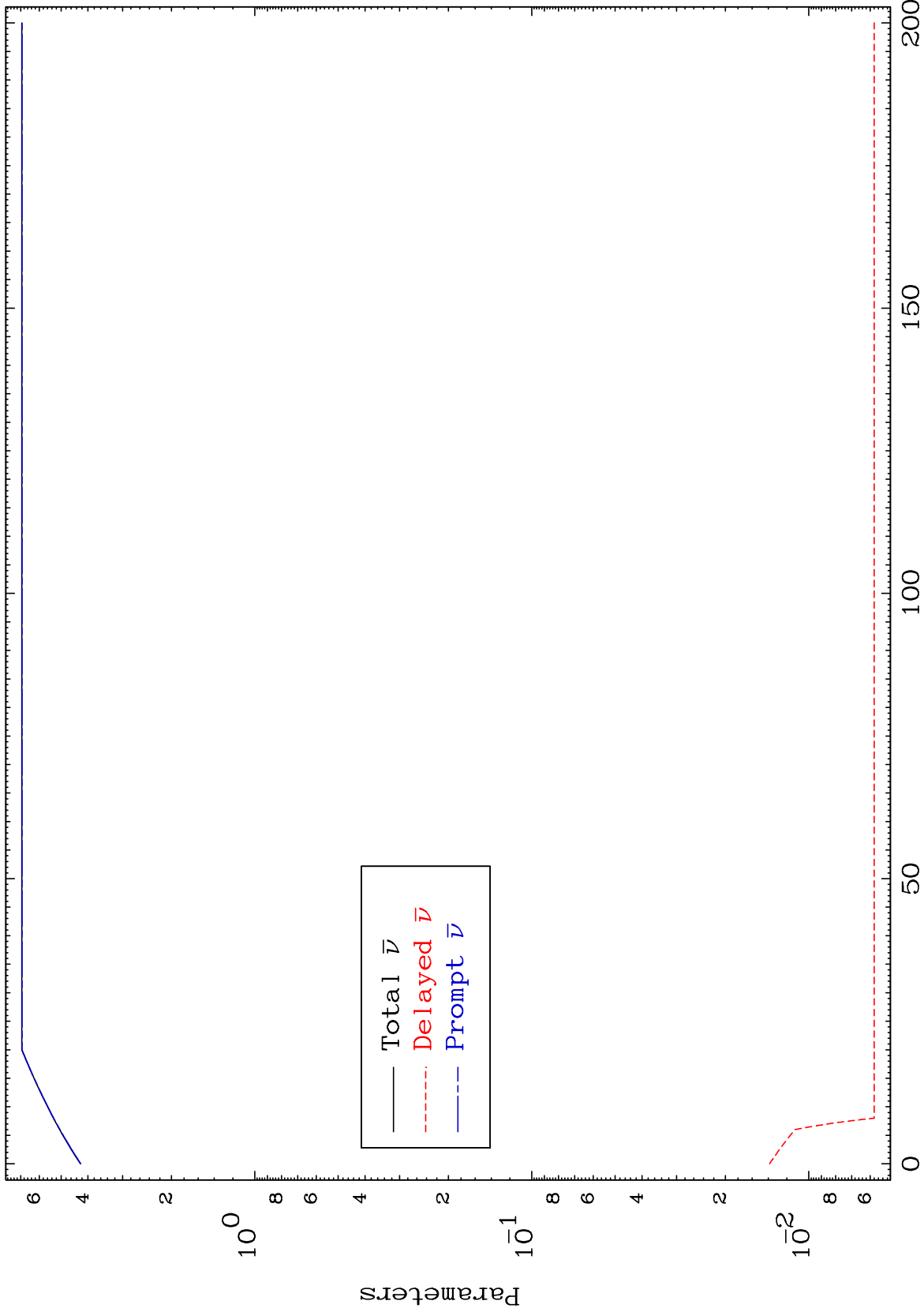
97-Bk-245



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Energy Release  
Parameters

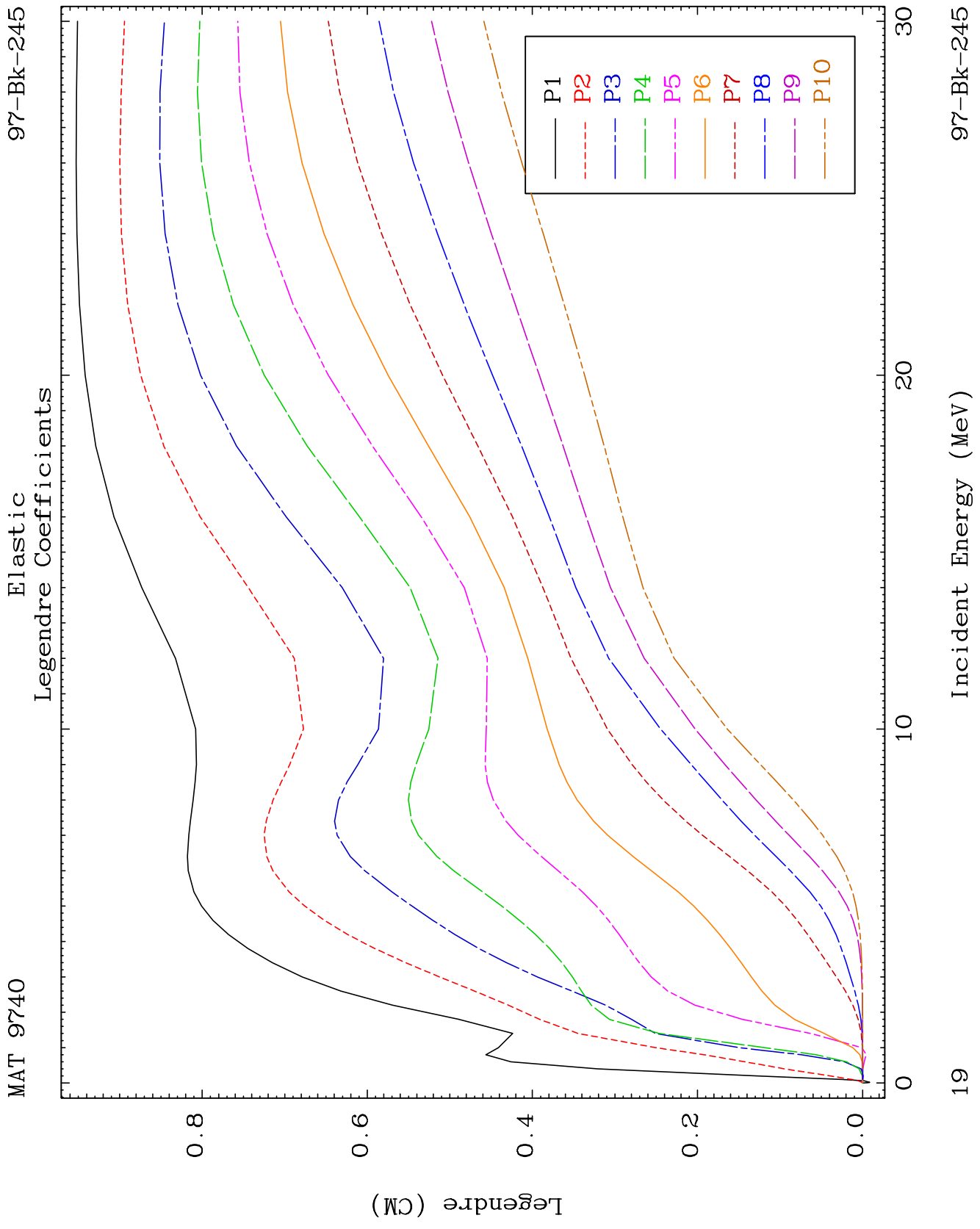
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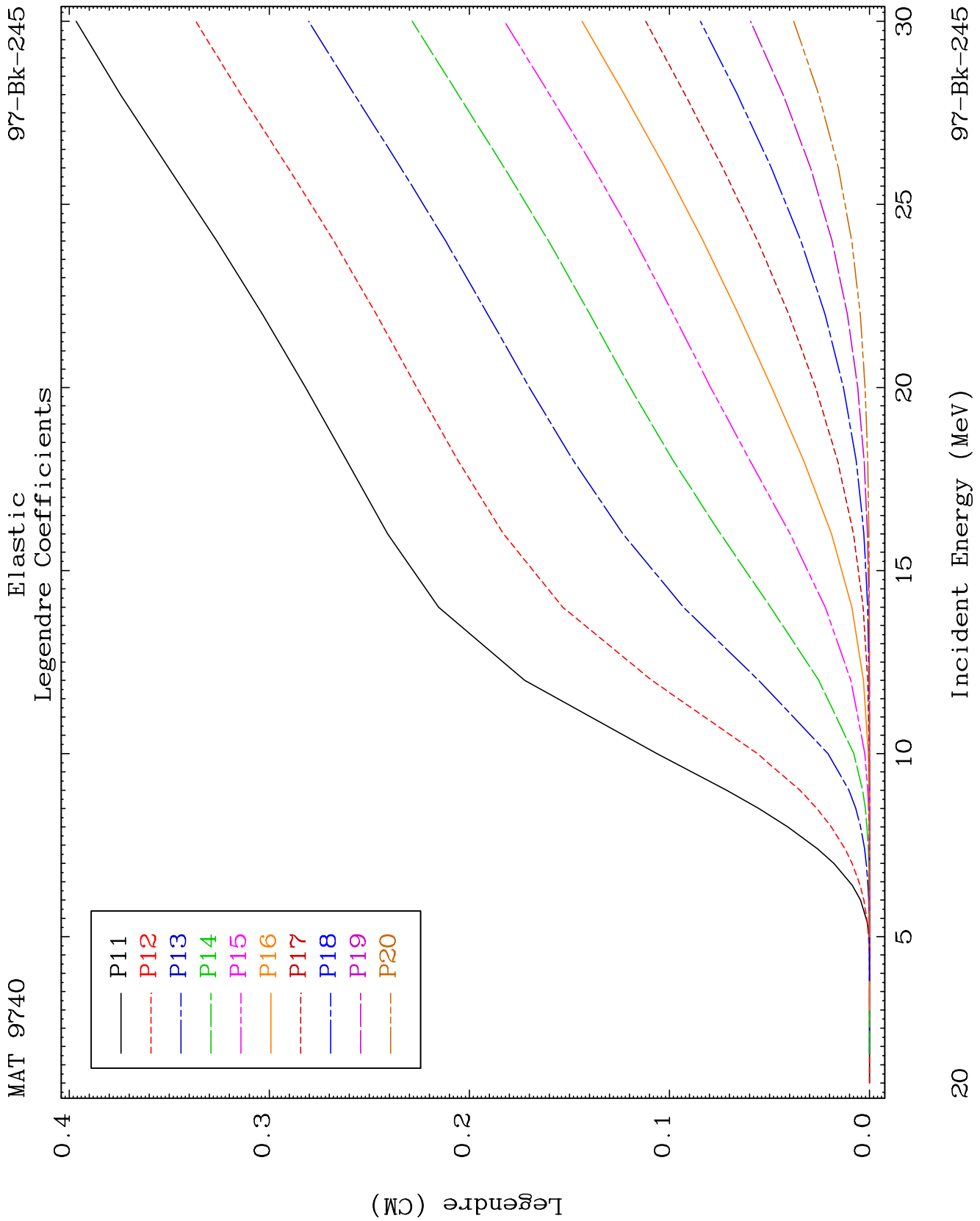


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Incident Energy (MeV)

97-Bk-245

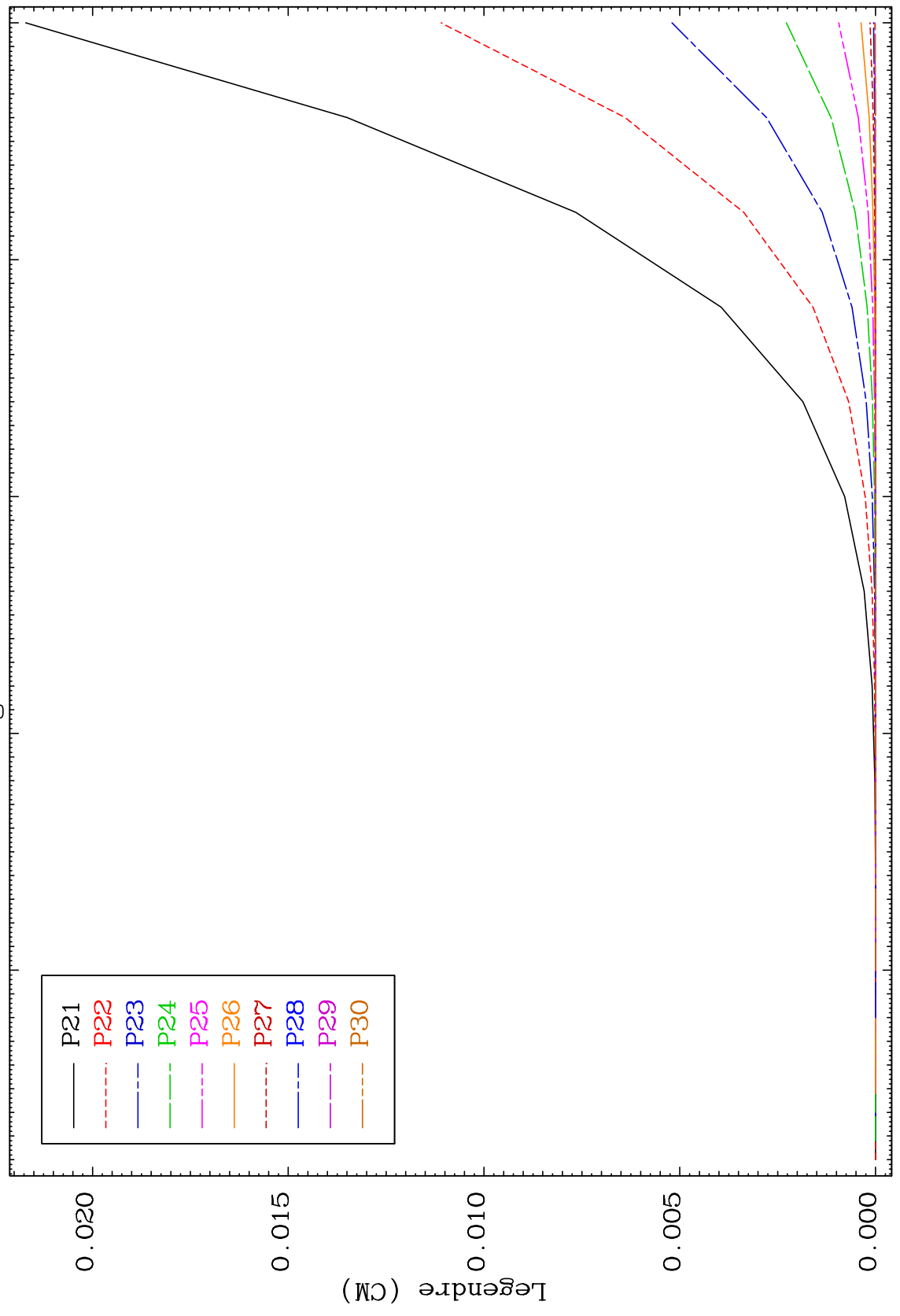




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### Elastic Legendre Coefficients

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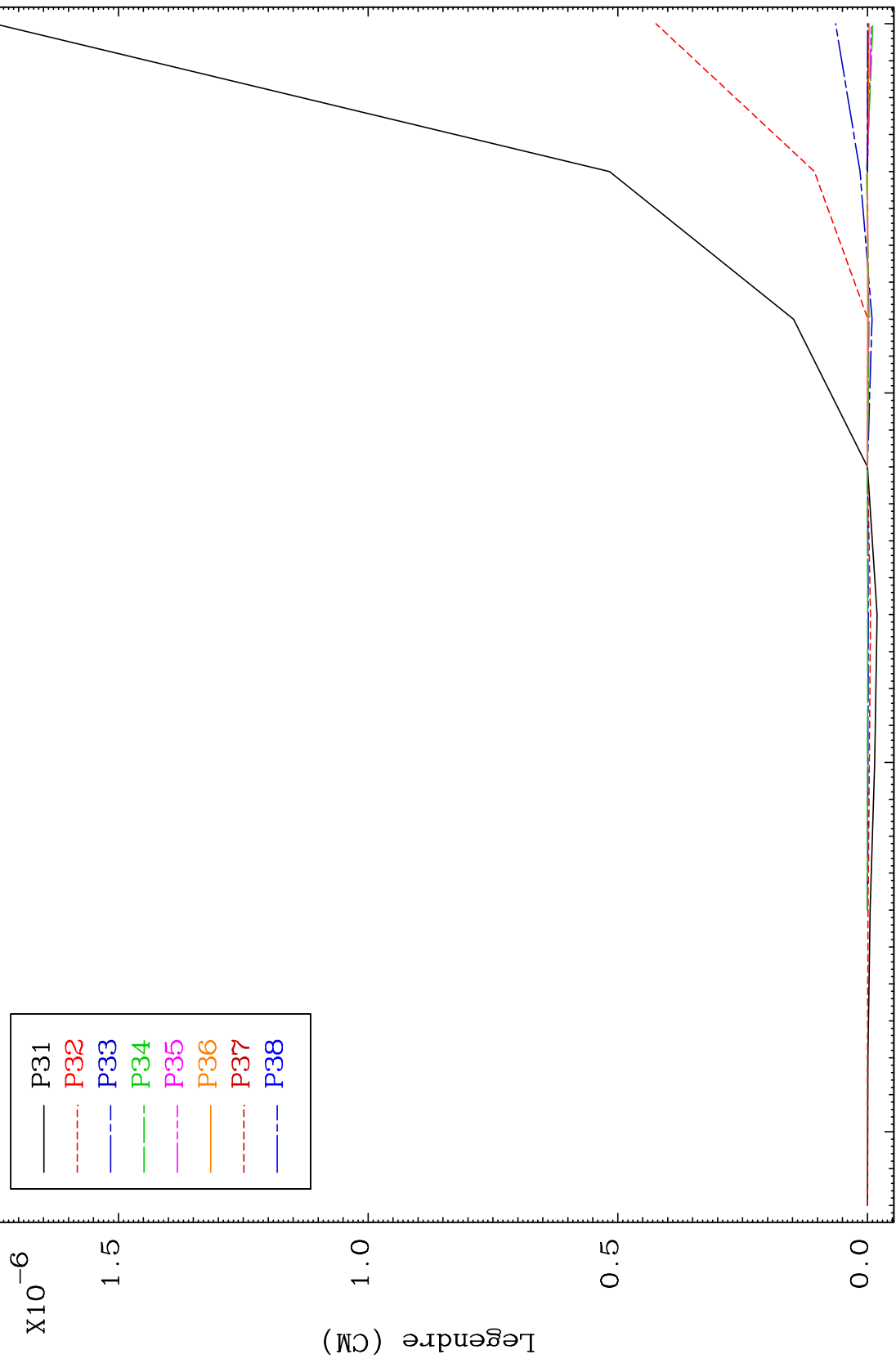
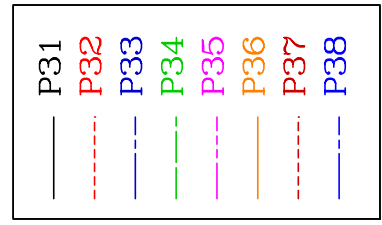
Incident Energy (MeV)

97-Bk-245

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Elastic Legendre Coefficients

97-Bk-245



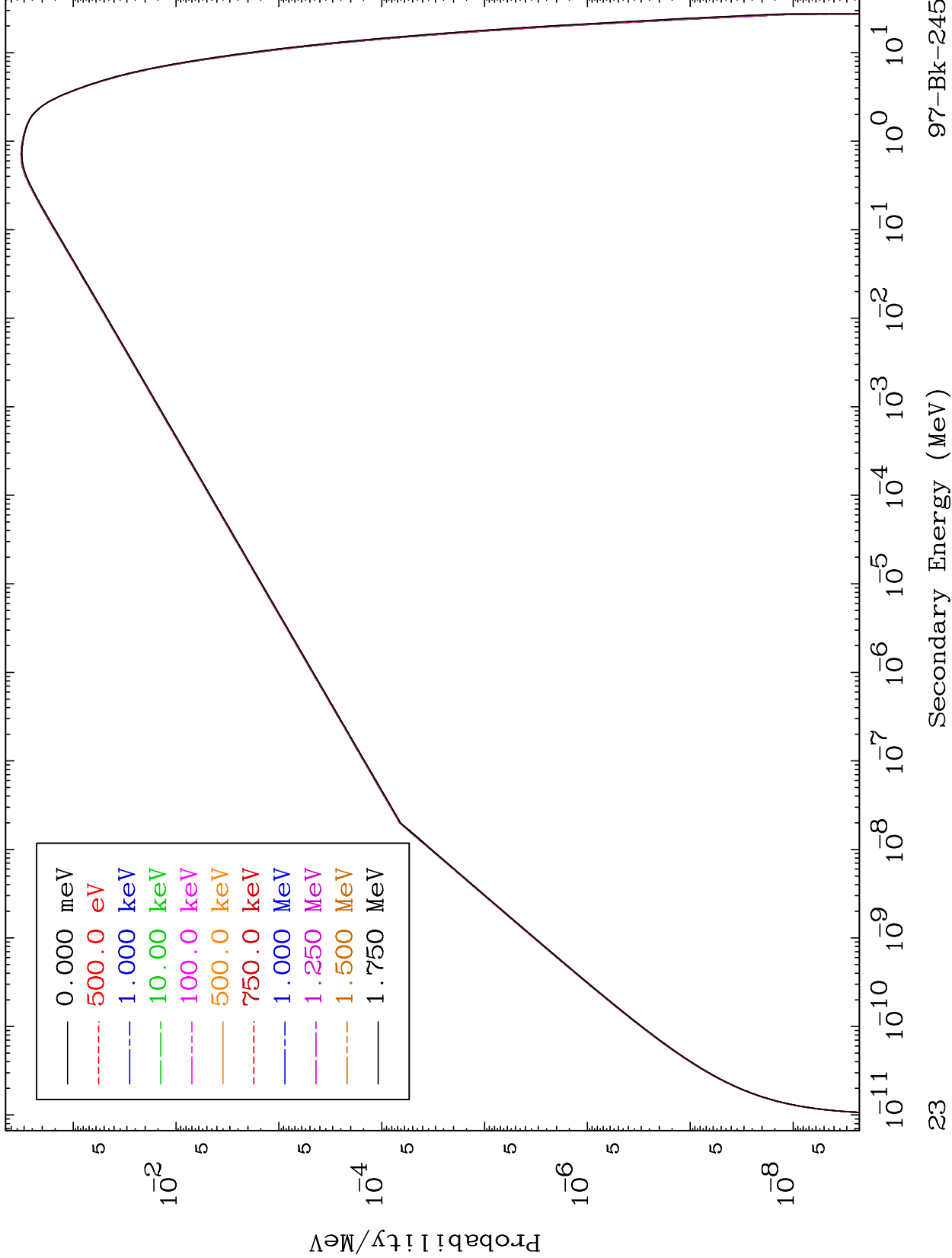
22

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### Fission Energy Distributions

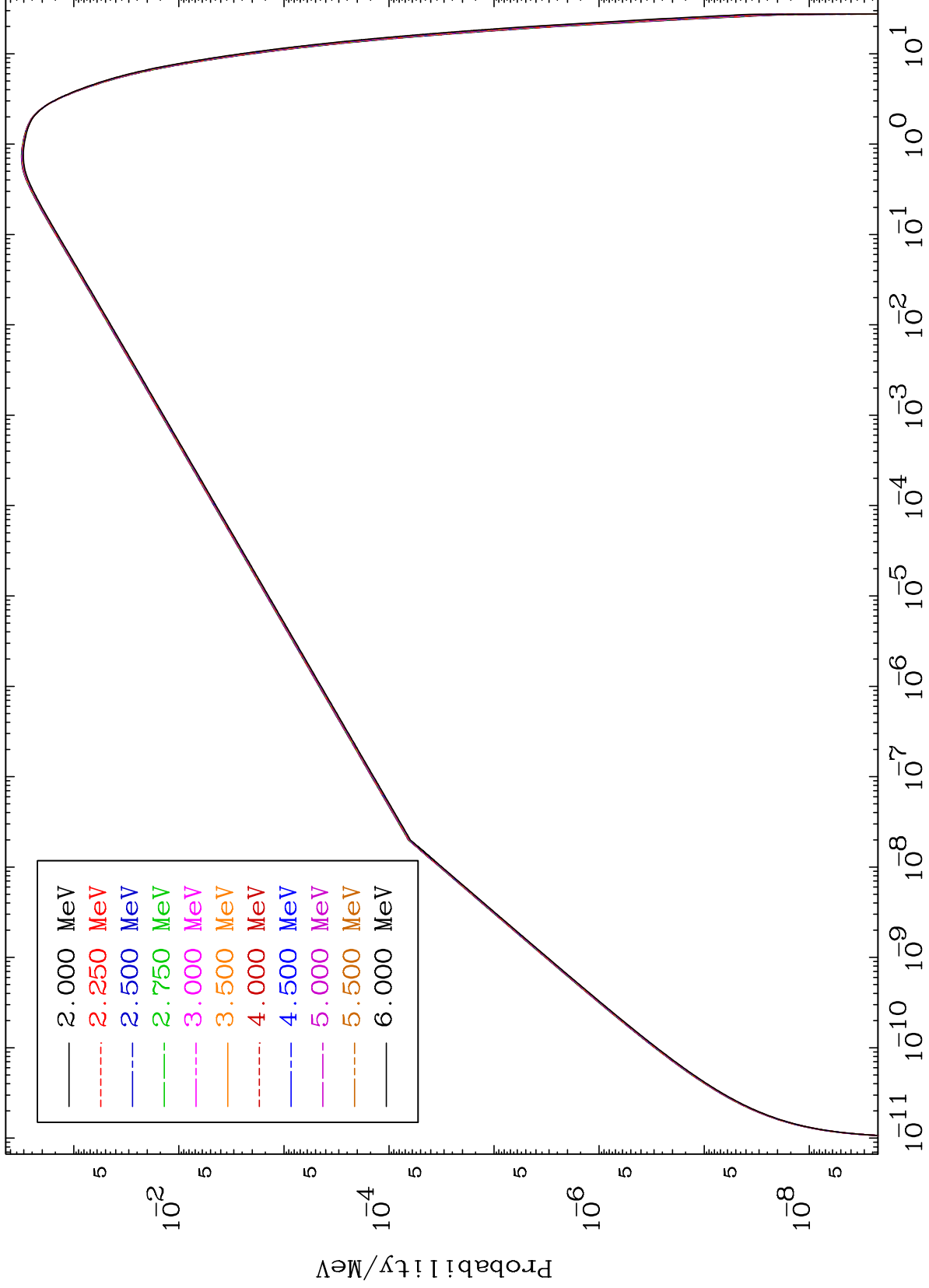
97-Bk-245



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### Fission Energy Distributions

97-Bk-245



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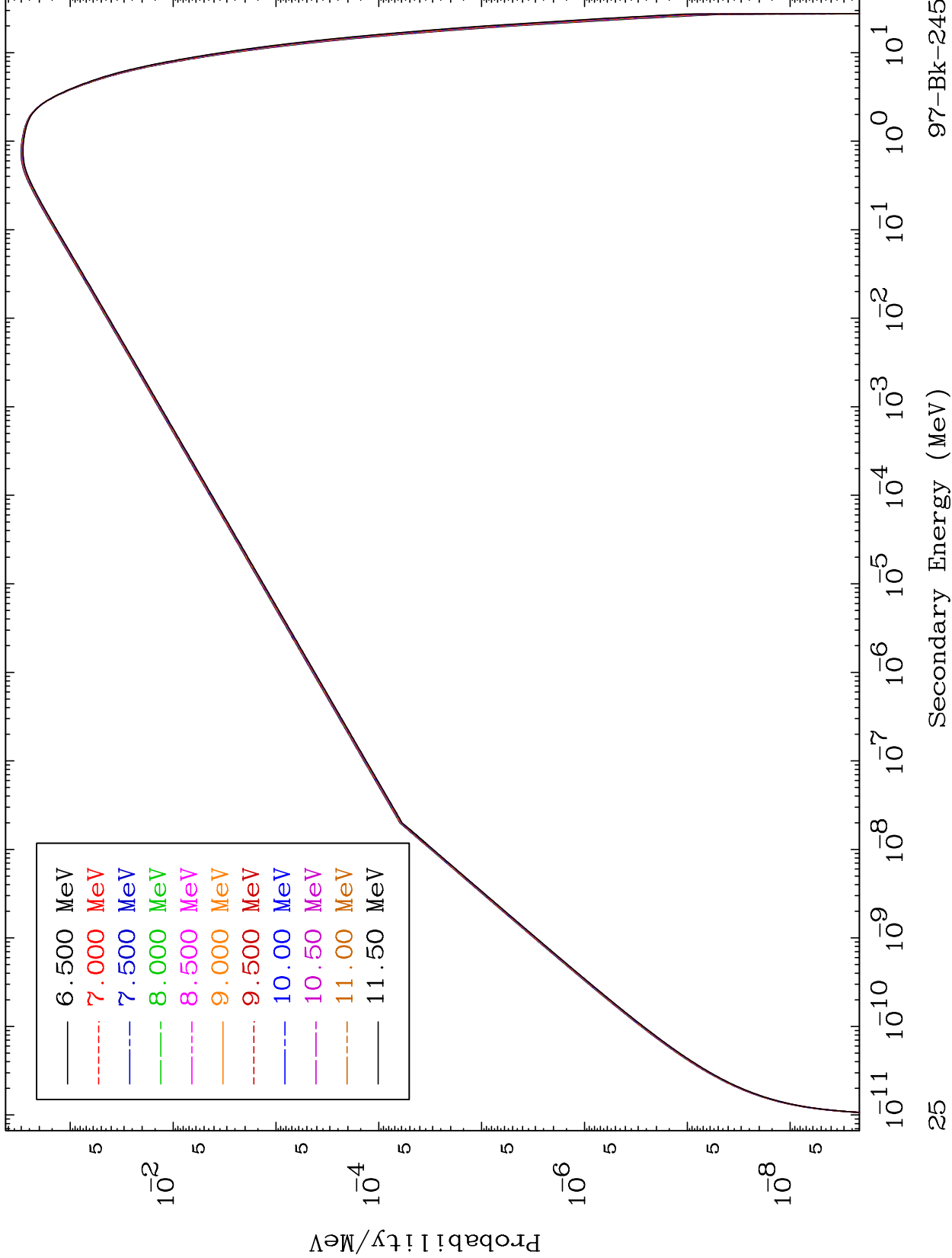
Secondary Energy (MeV)

97-Bk-245

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### Fission Energy Distributions

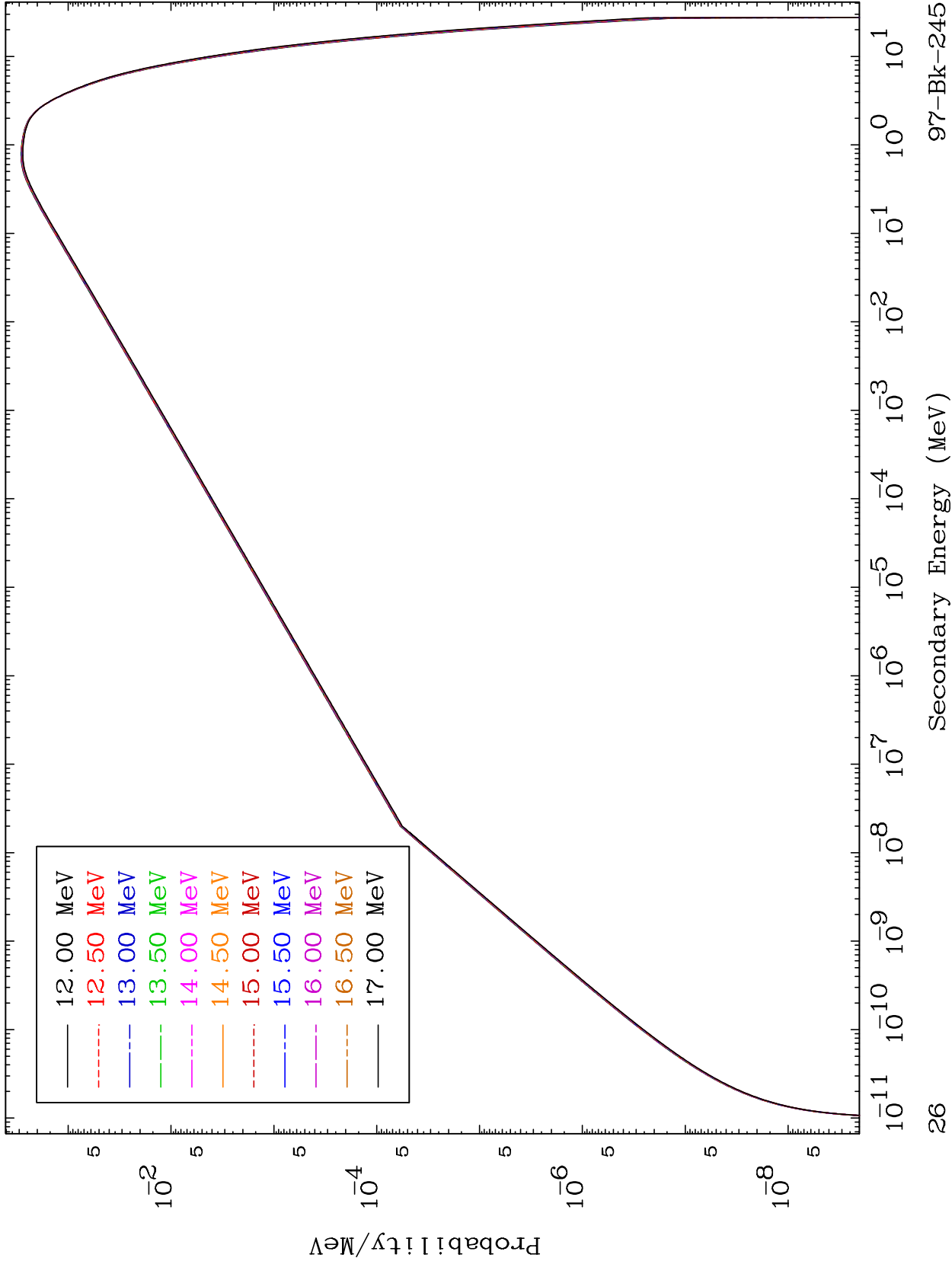
97-Bk-245



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### Fission Energy Distributions

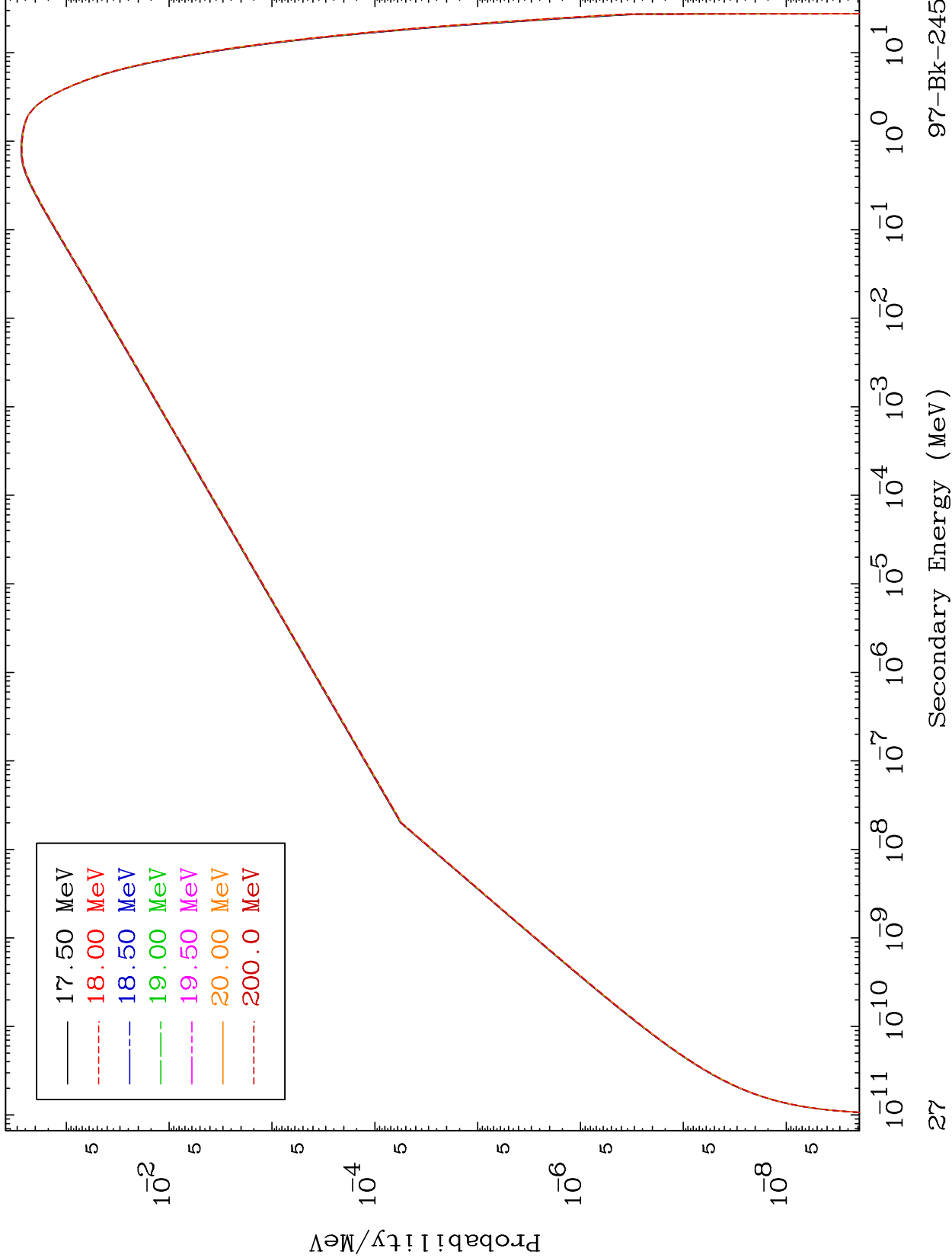
97-Bk-245



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### Fission Energy Distributions

97-Bk-245



97-Bk-245

Secondary Energy (MeV)

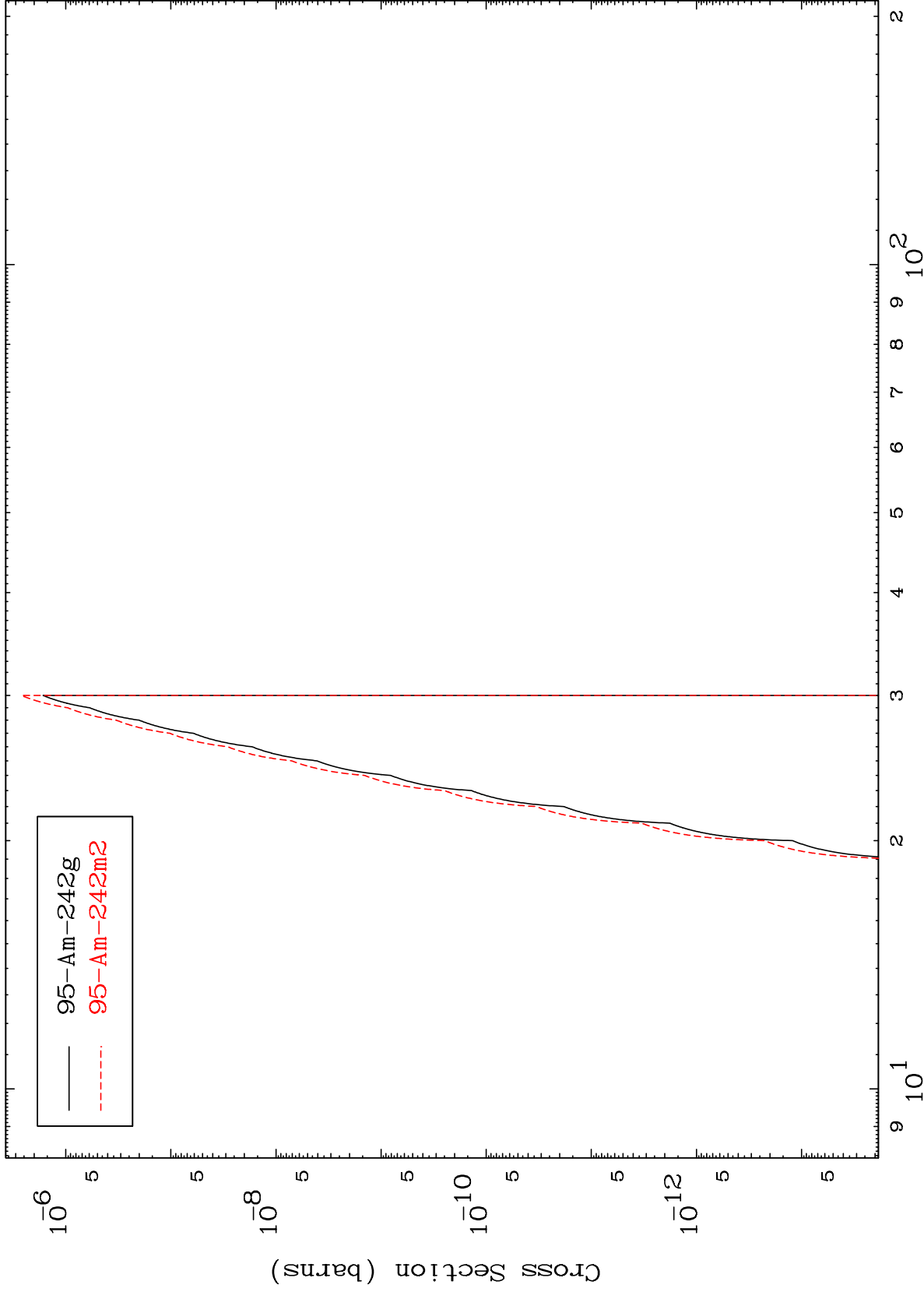
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(n,n') He-3

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Radionuclide Production Cross Section



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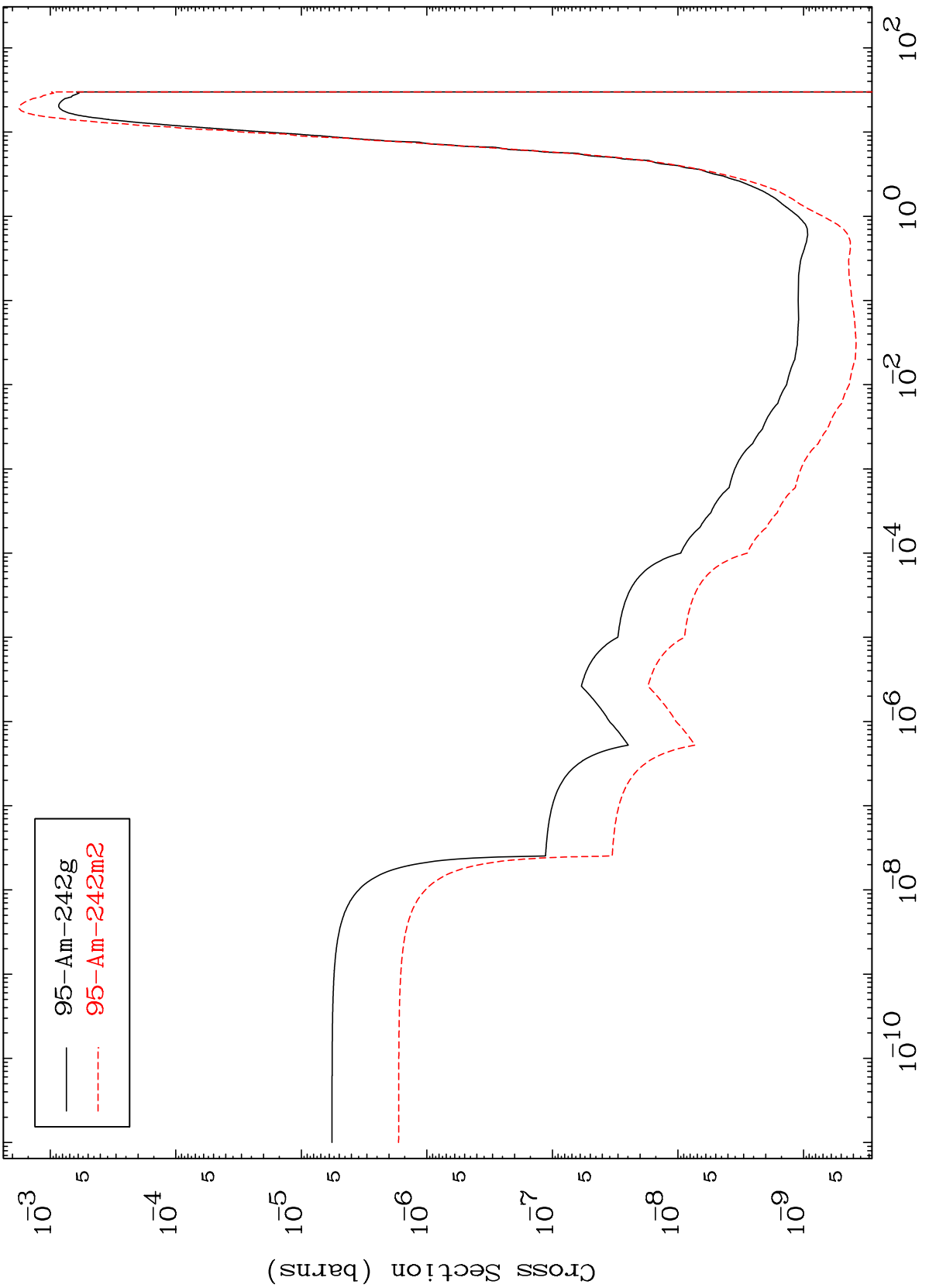
Incident Energy (MeV)

97-Bk-245

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97-Bk-245

Radionuclide Production Cross Section (n,  $\alpha$ )

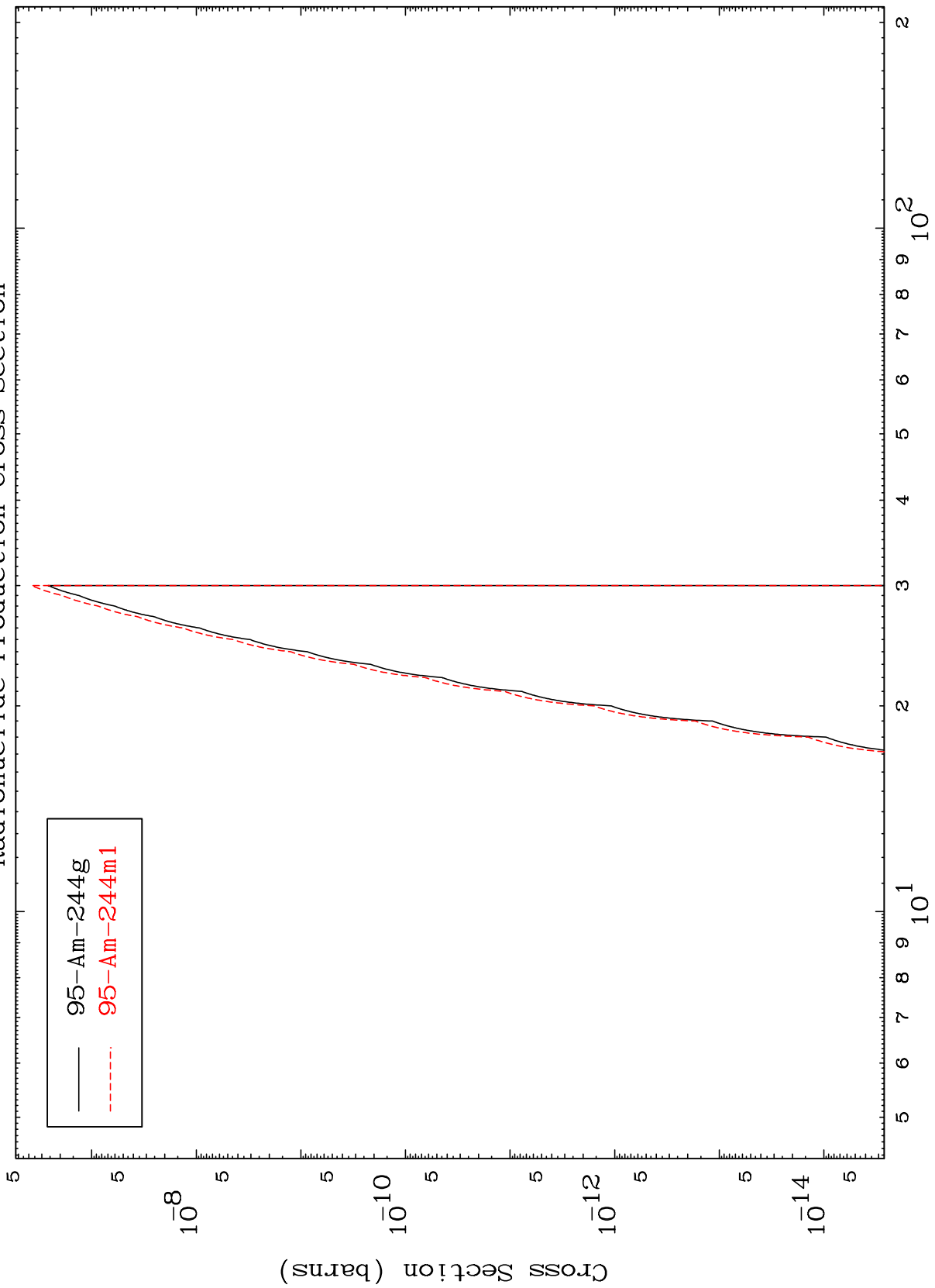


— 95-Am-242g  
- - - 95-Am-242m2

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97-Bk-245

(n,2p)  
Radionuclide Production Cross Section



30

Incident Energy (MeV)

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