

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

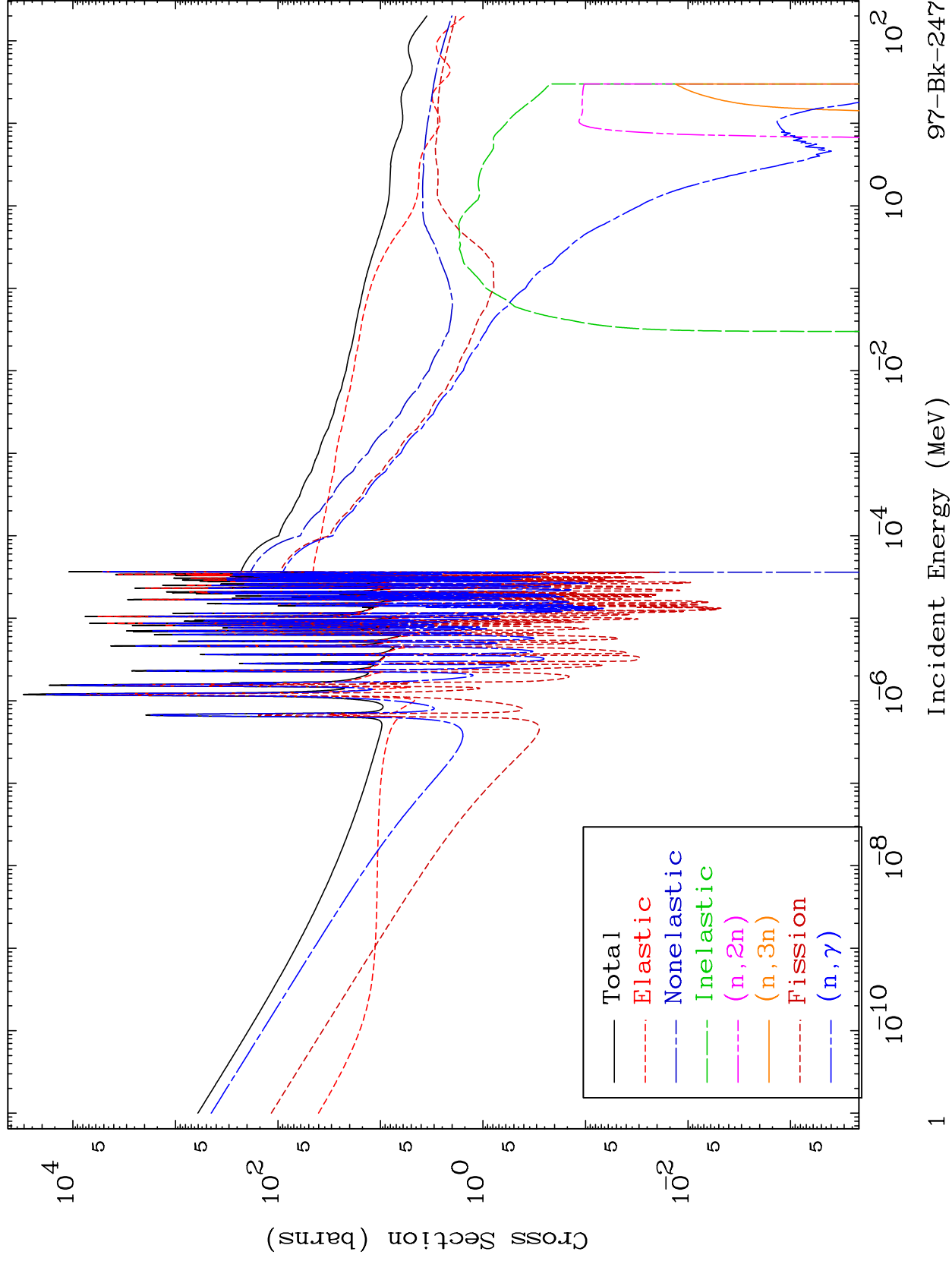
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Web:redcullen1.net/HOMEPAGE.NEW

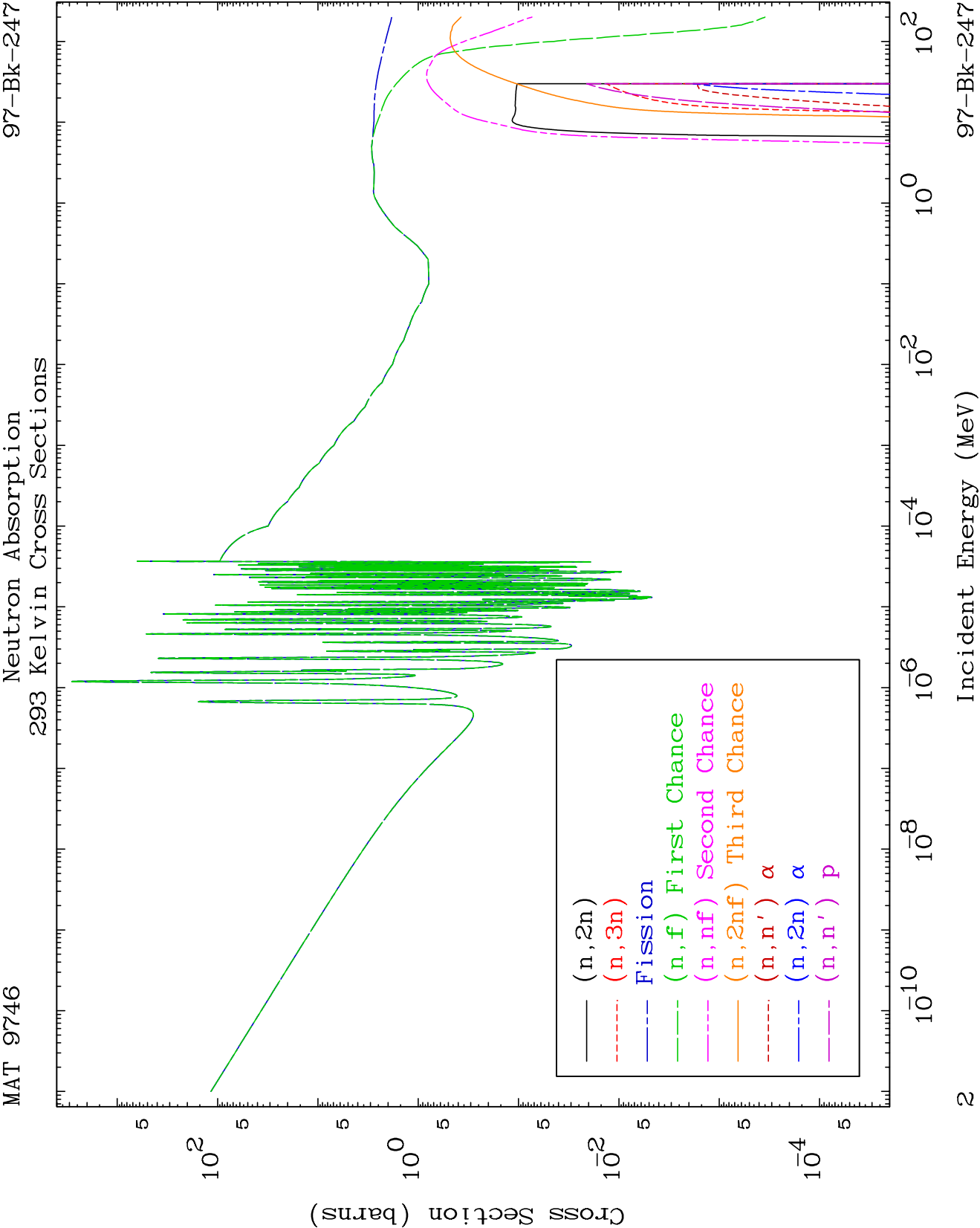
Press Mouse Button to Start

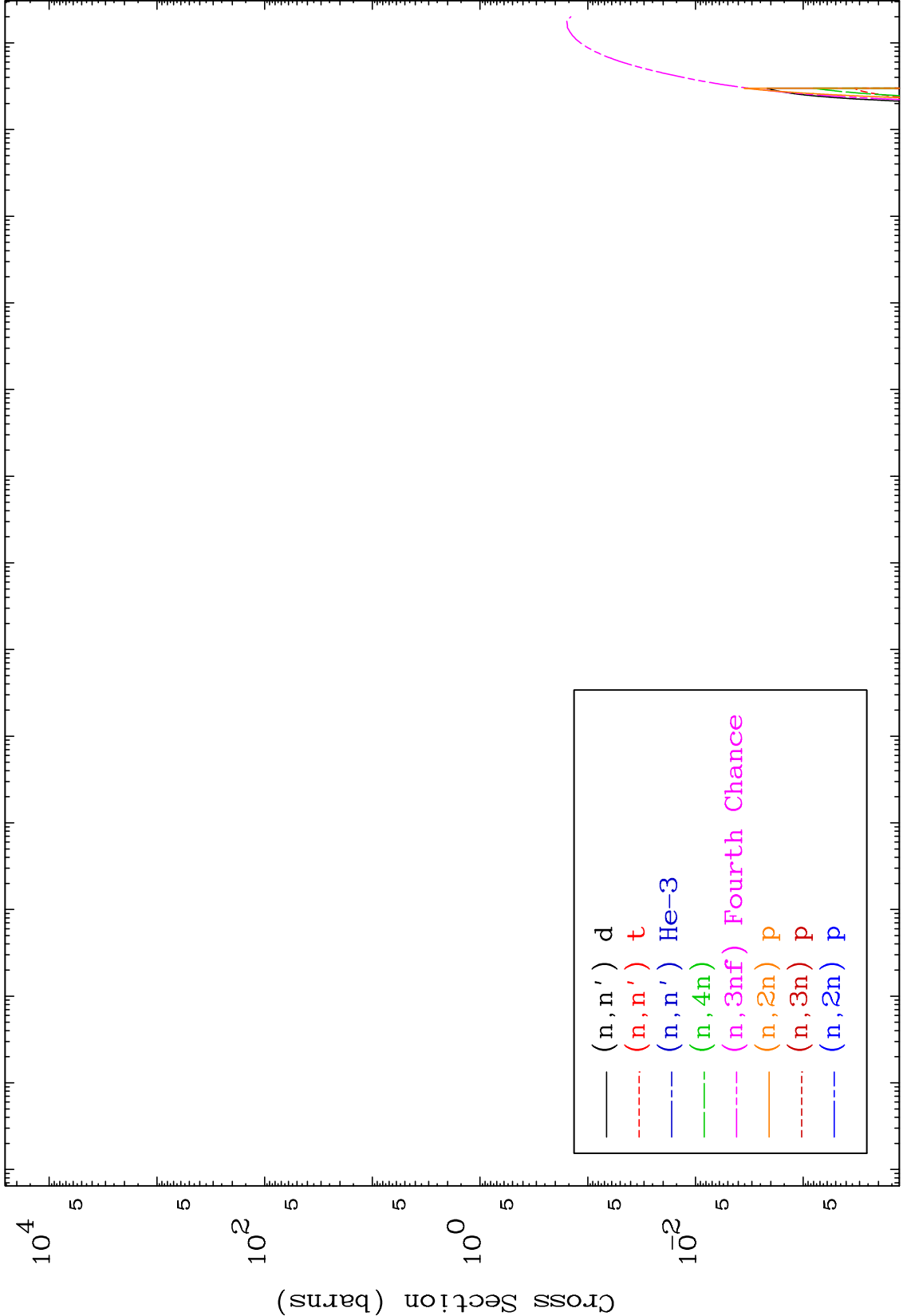
MAT 9746

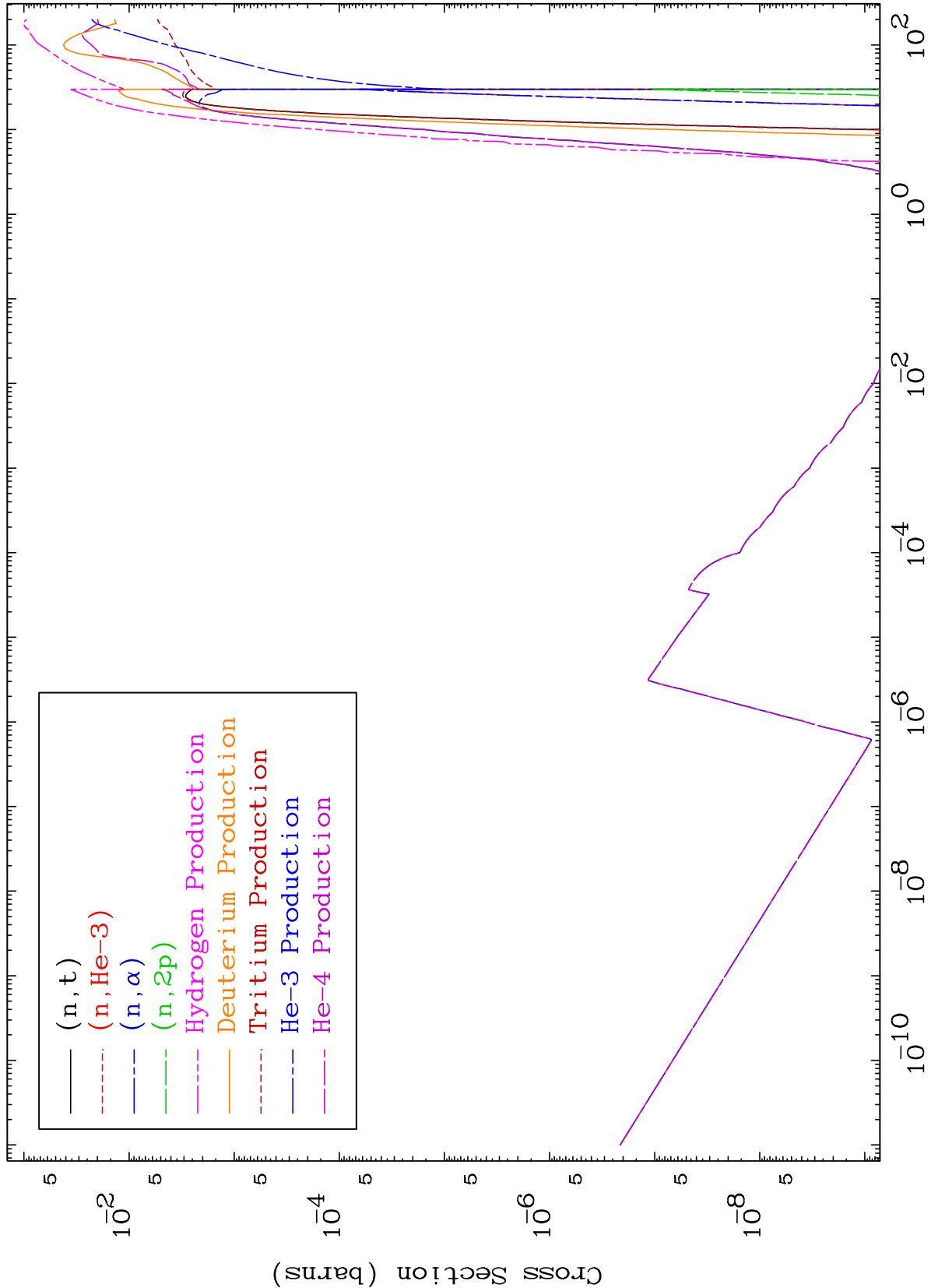
Neutron Major
293 Kelvin Cross Sections

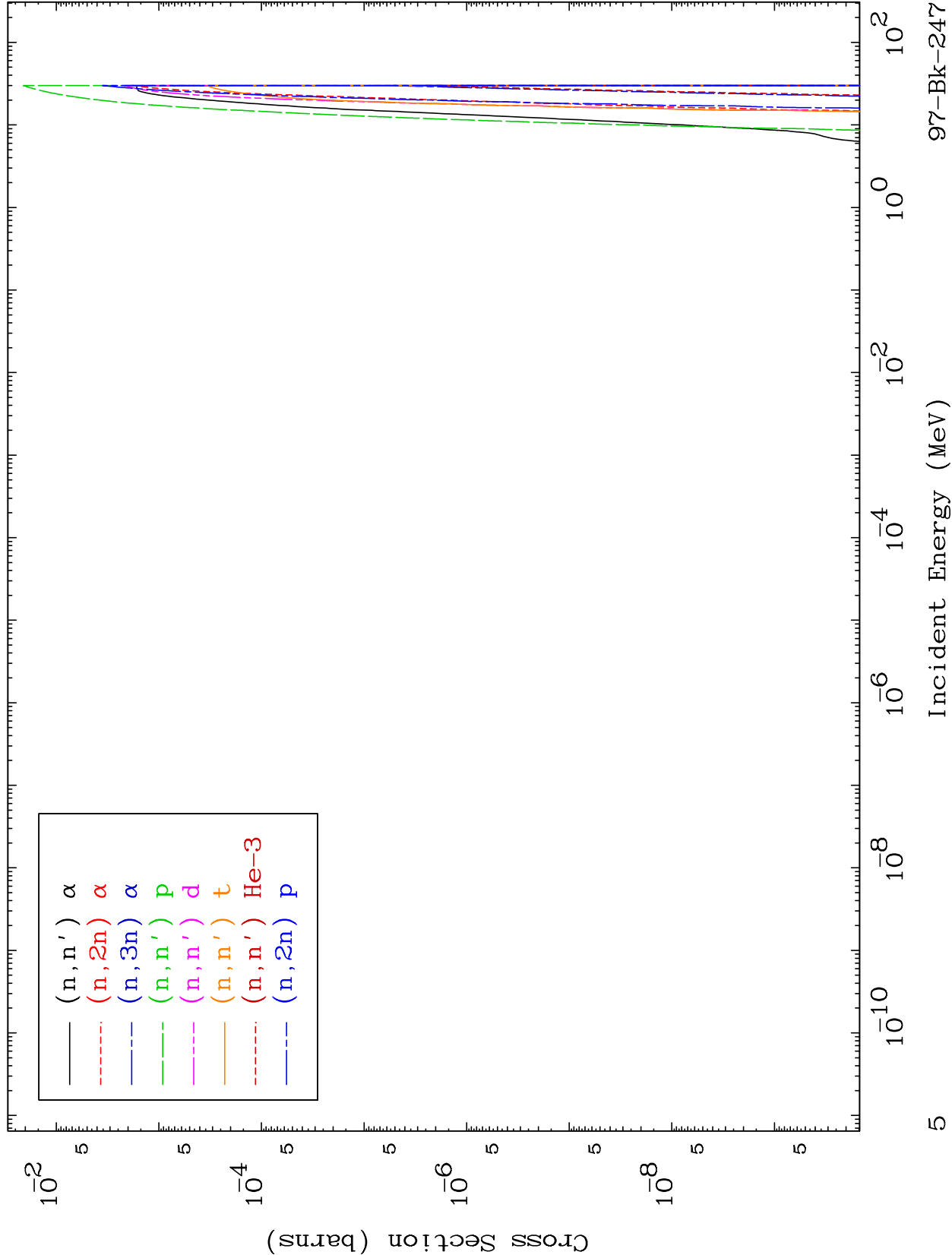
97-Bk-247

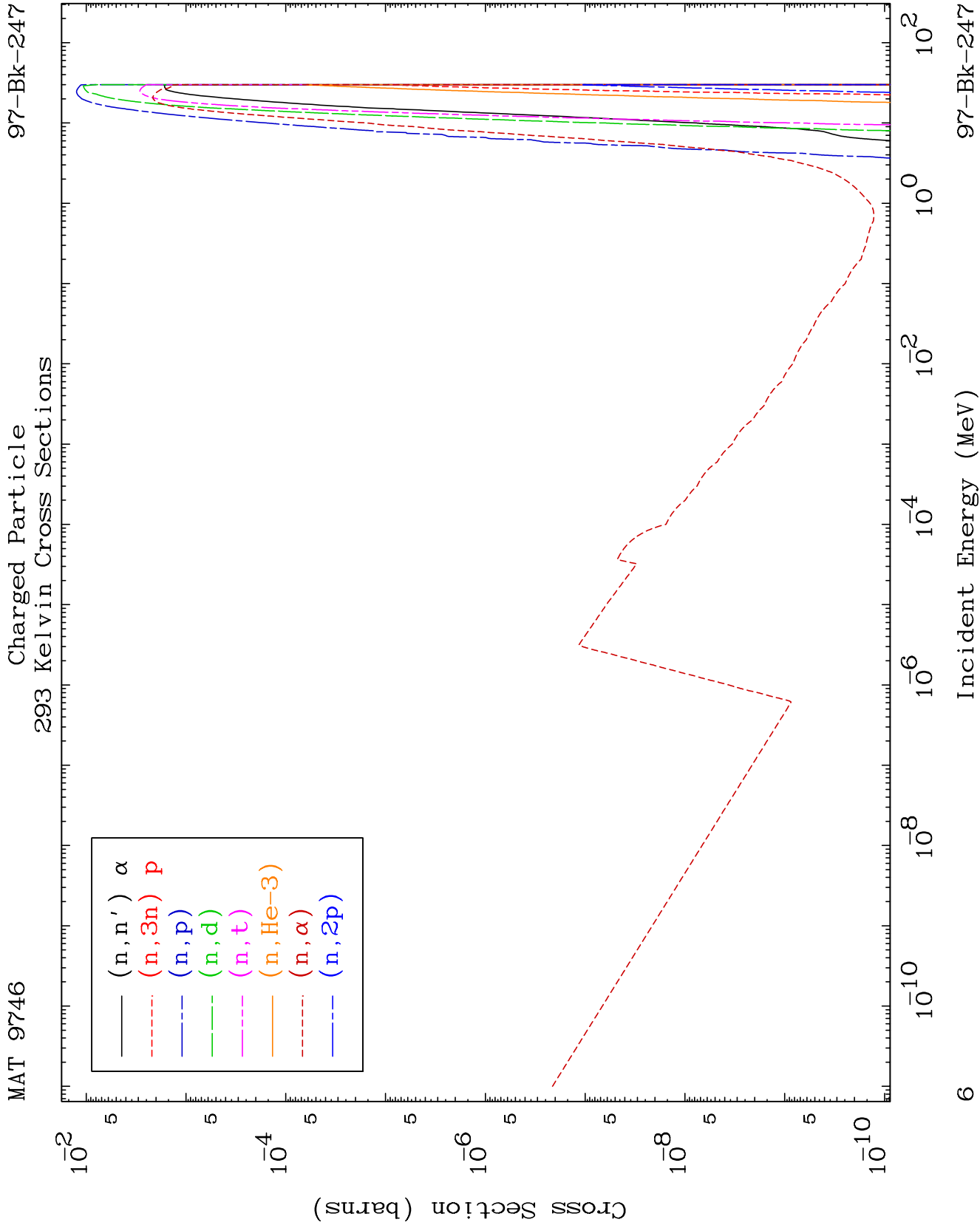








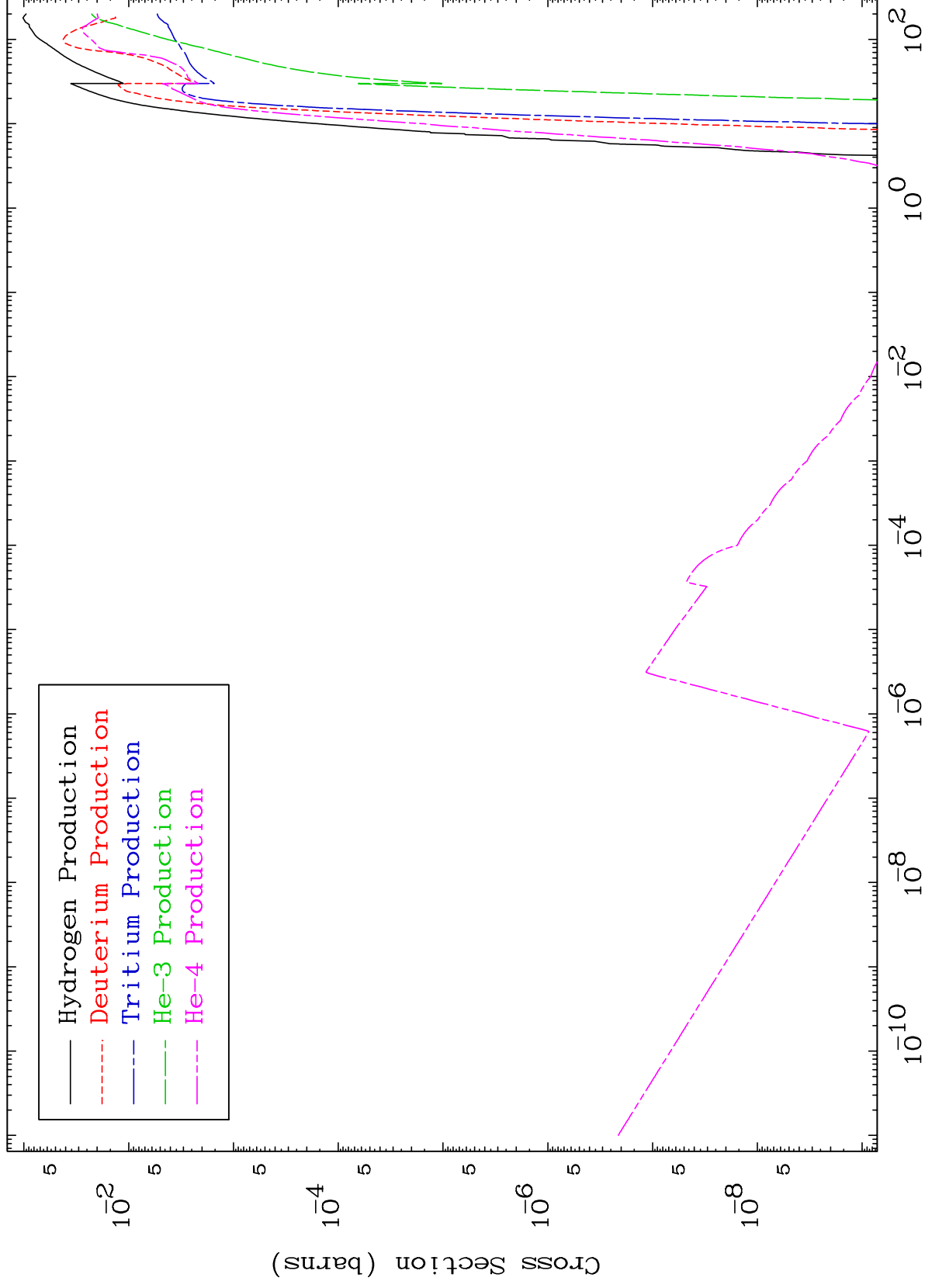


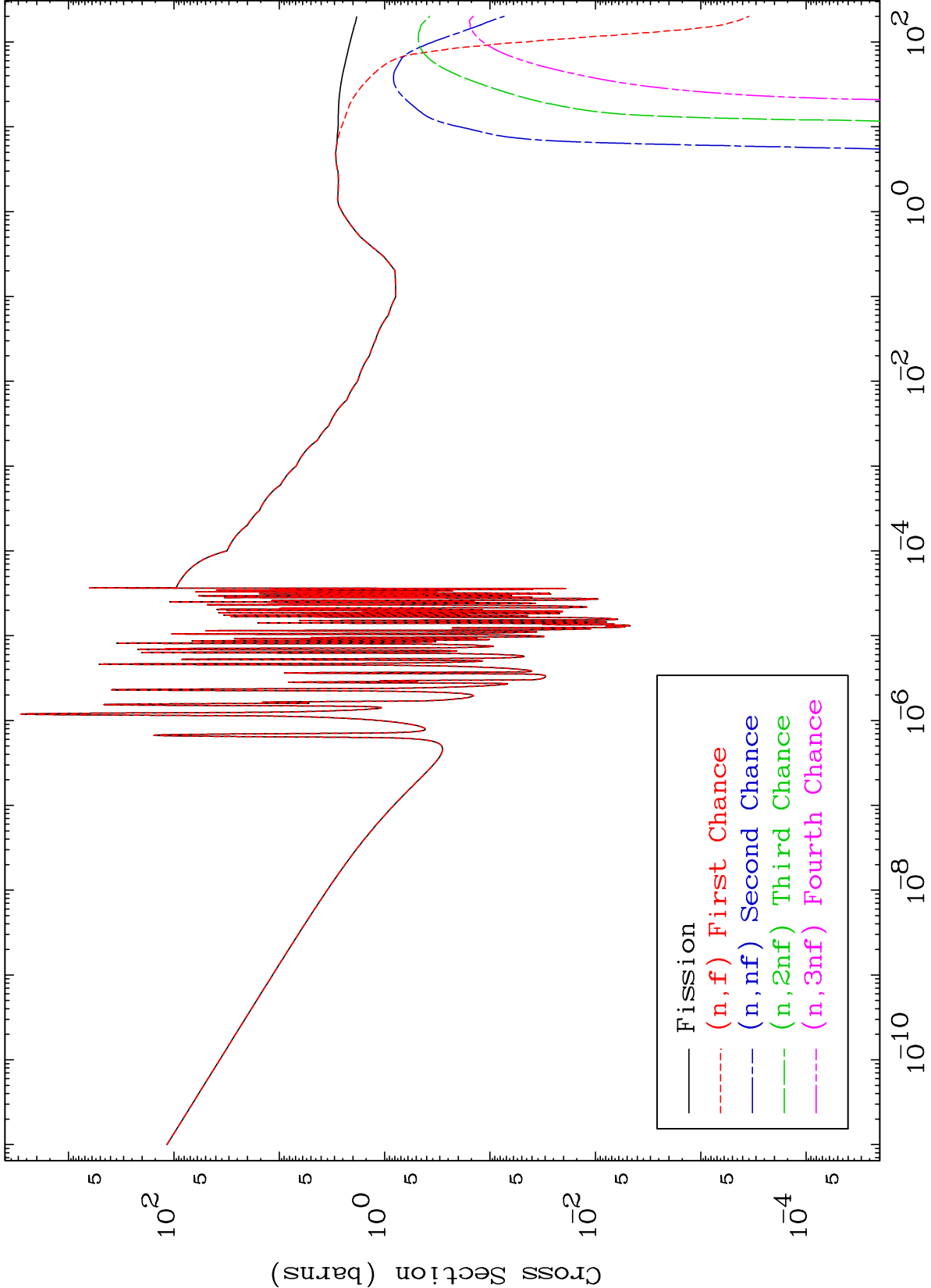


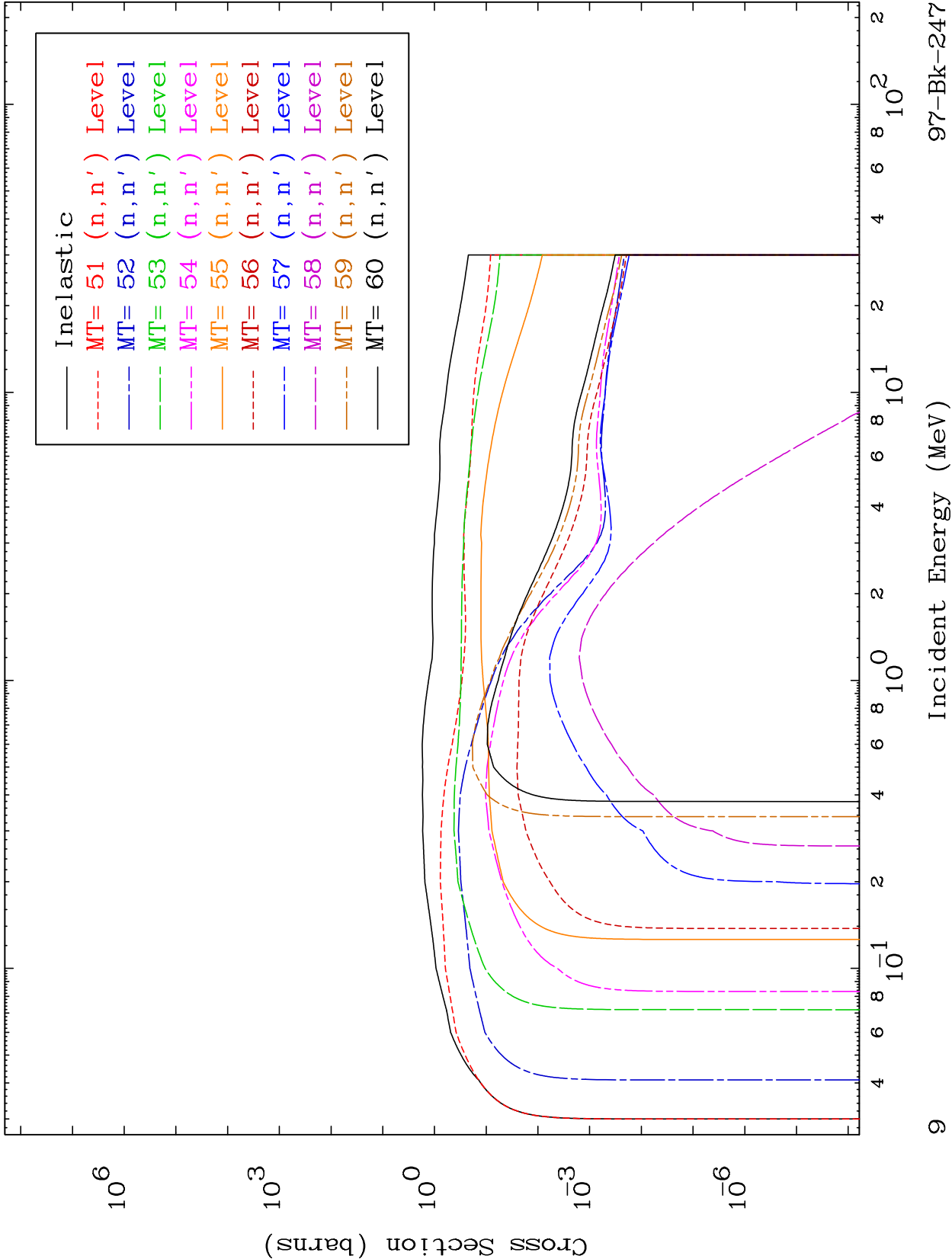
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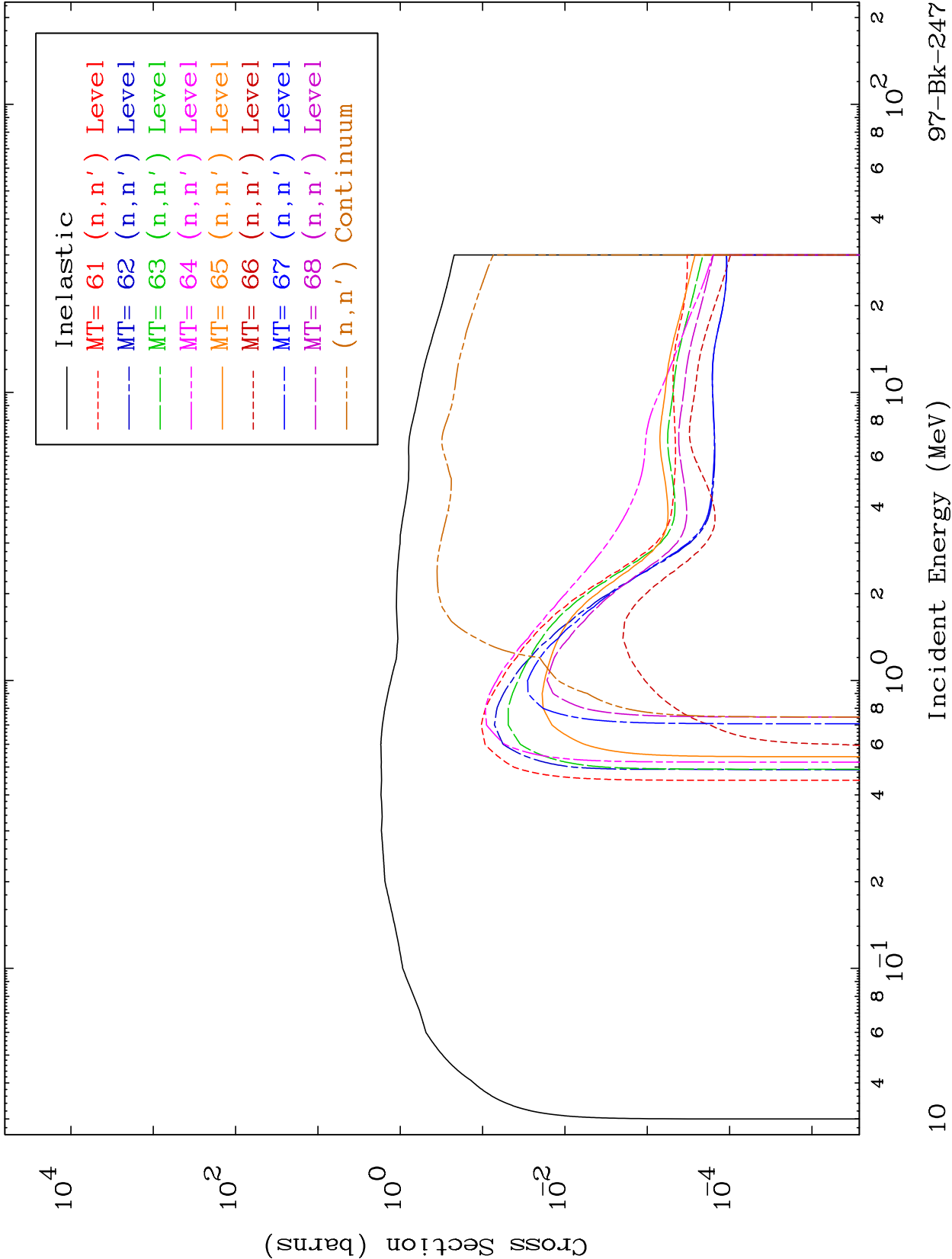
Particle Production
293 Kelvin Cross Sections

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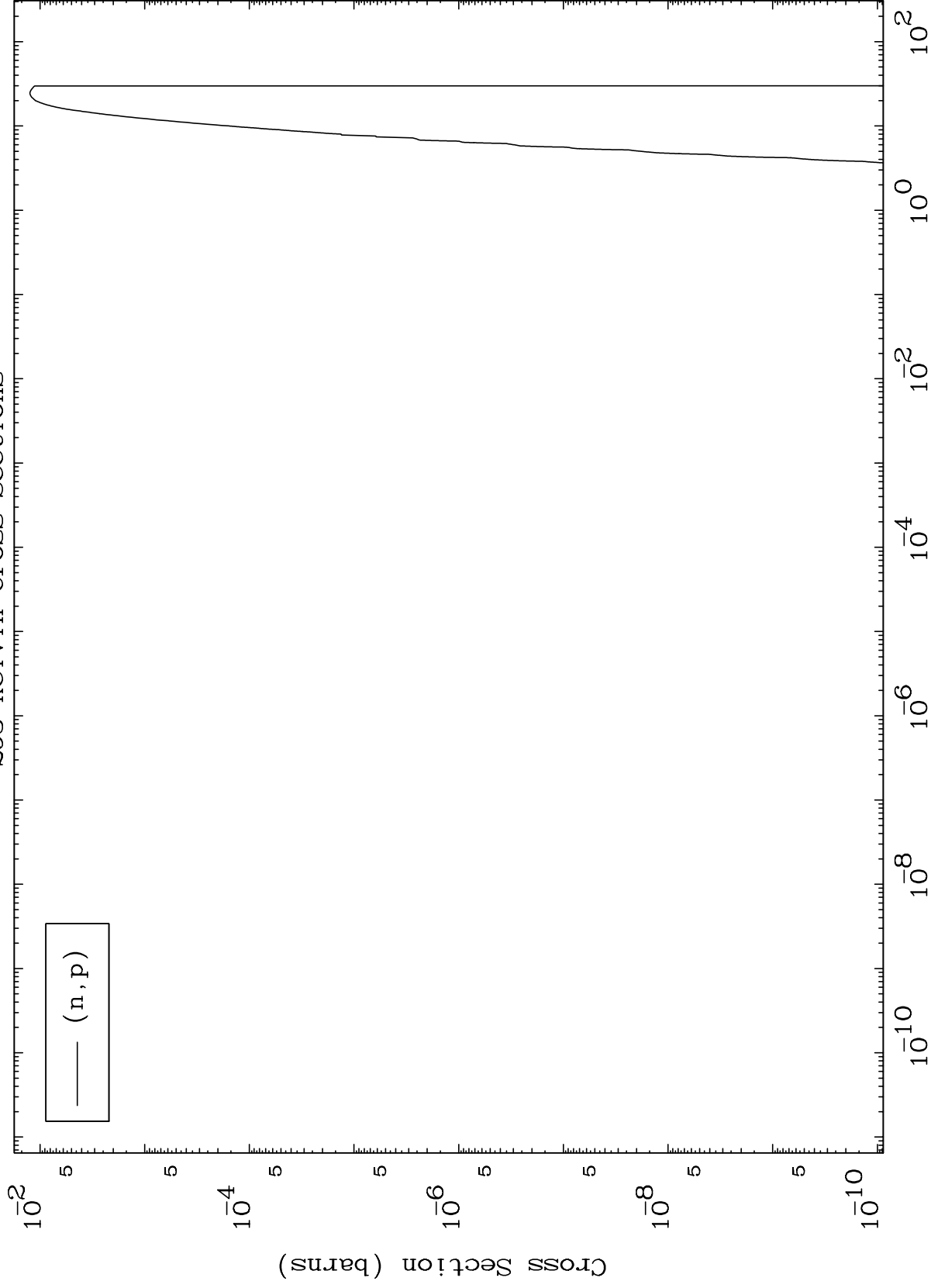




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(n,p) Levels
293 Kelvin Cross Sections

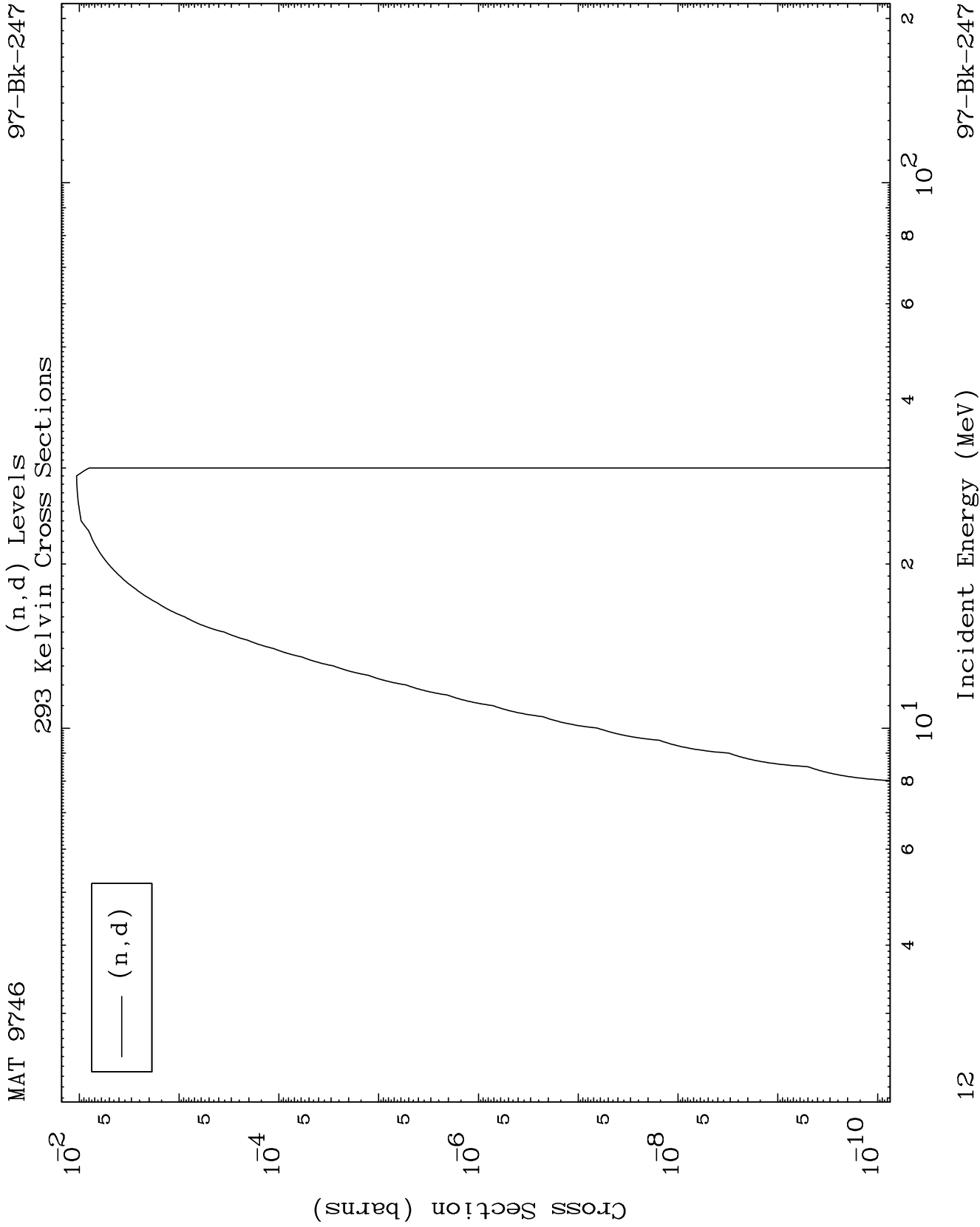
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11

Incident Energy (MeV)

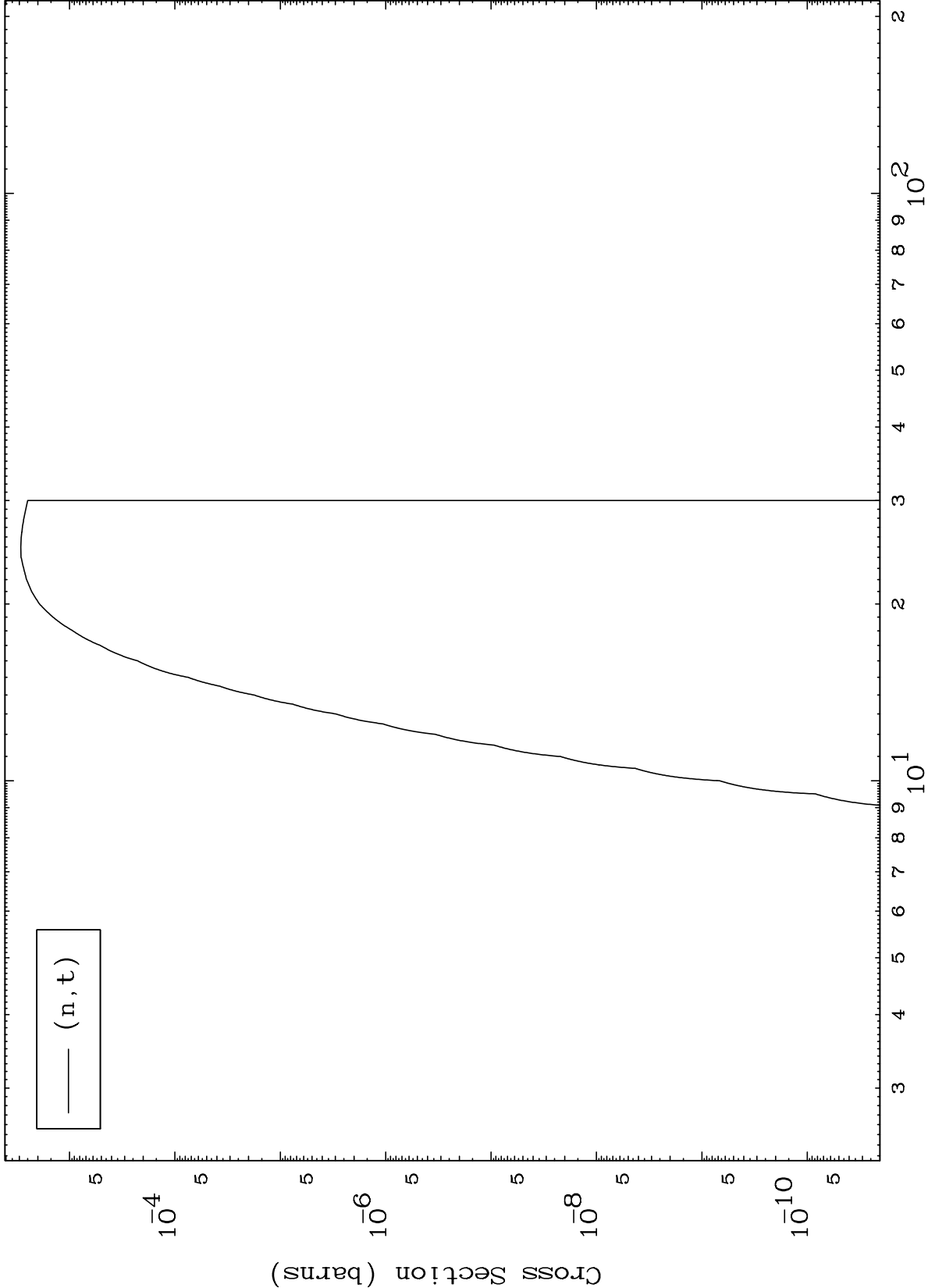
97-Bk-247



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(n,t) Levels
293 Kelvin Cross Sections

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13

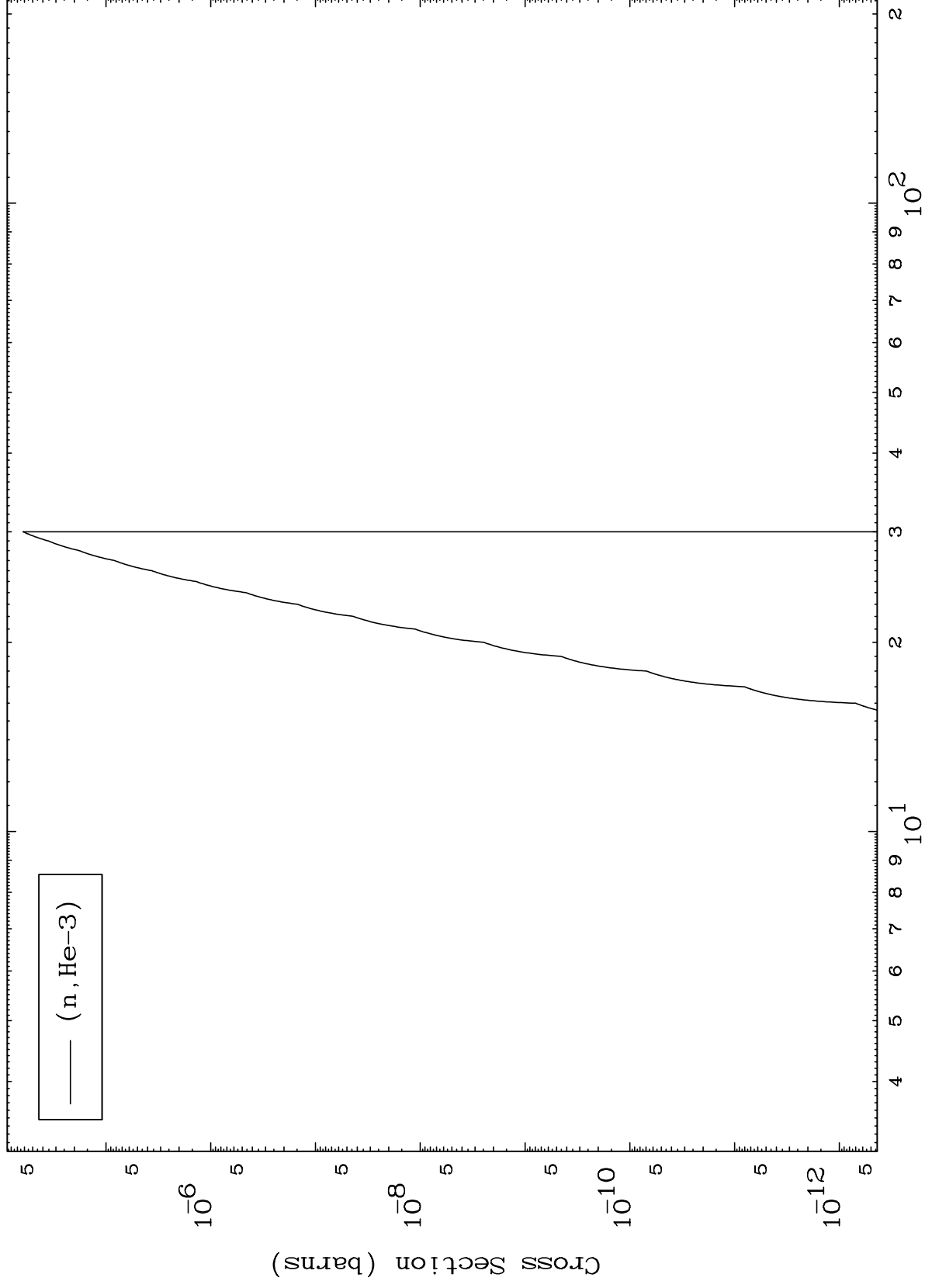
Incident Energy (MeV)

97-Bk-247

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(n,He3) Levels
293 Kelvin Cross Sections

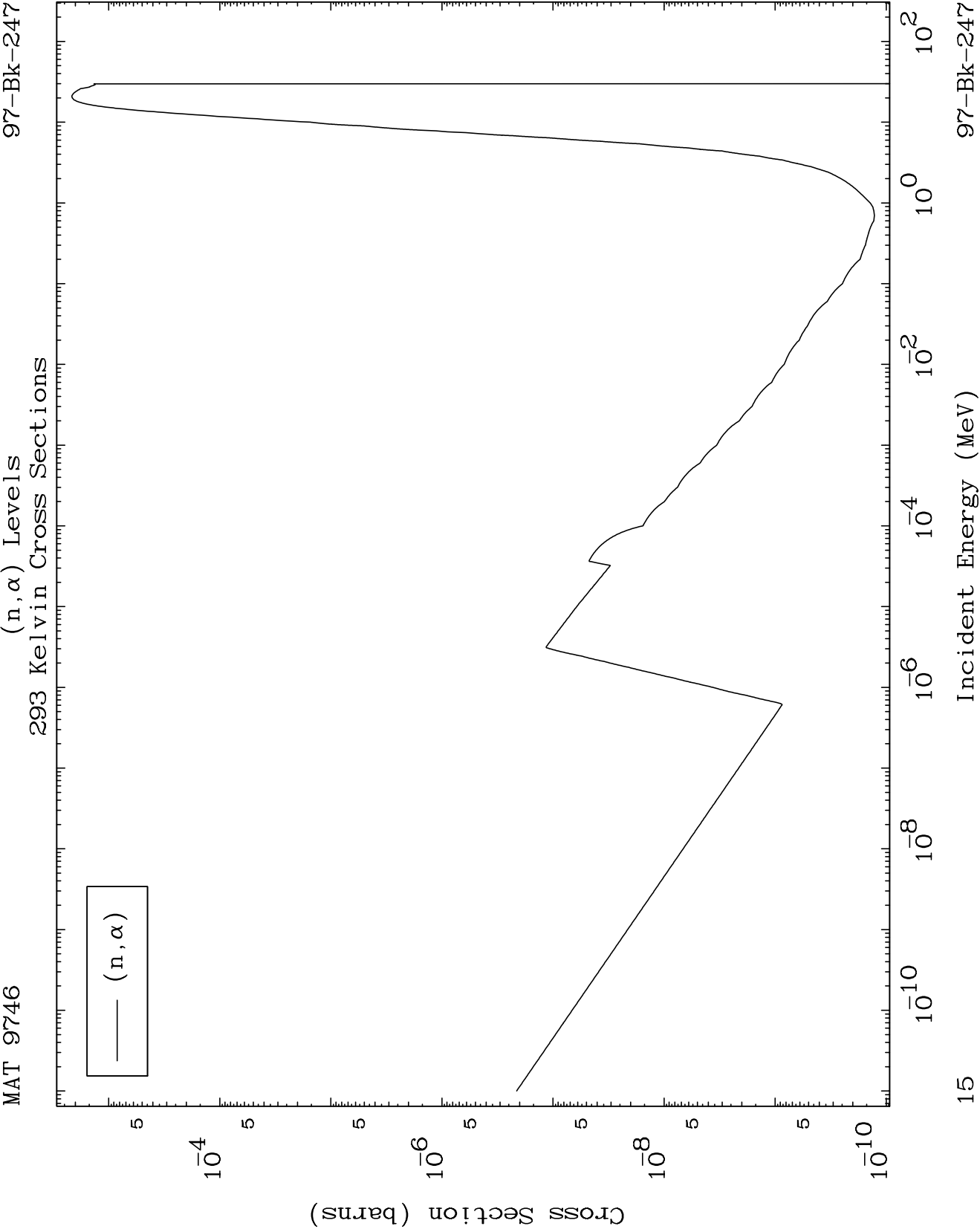
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14

Incident Energy (MeV)

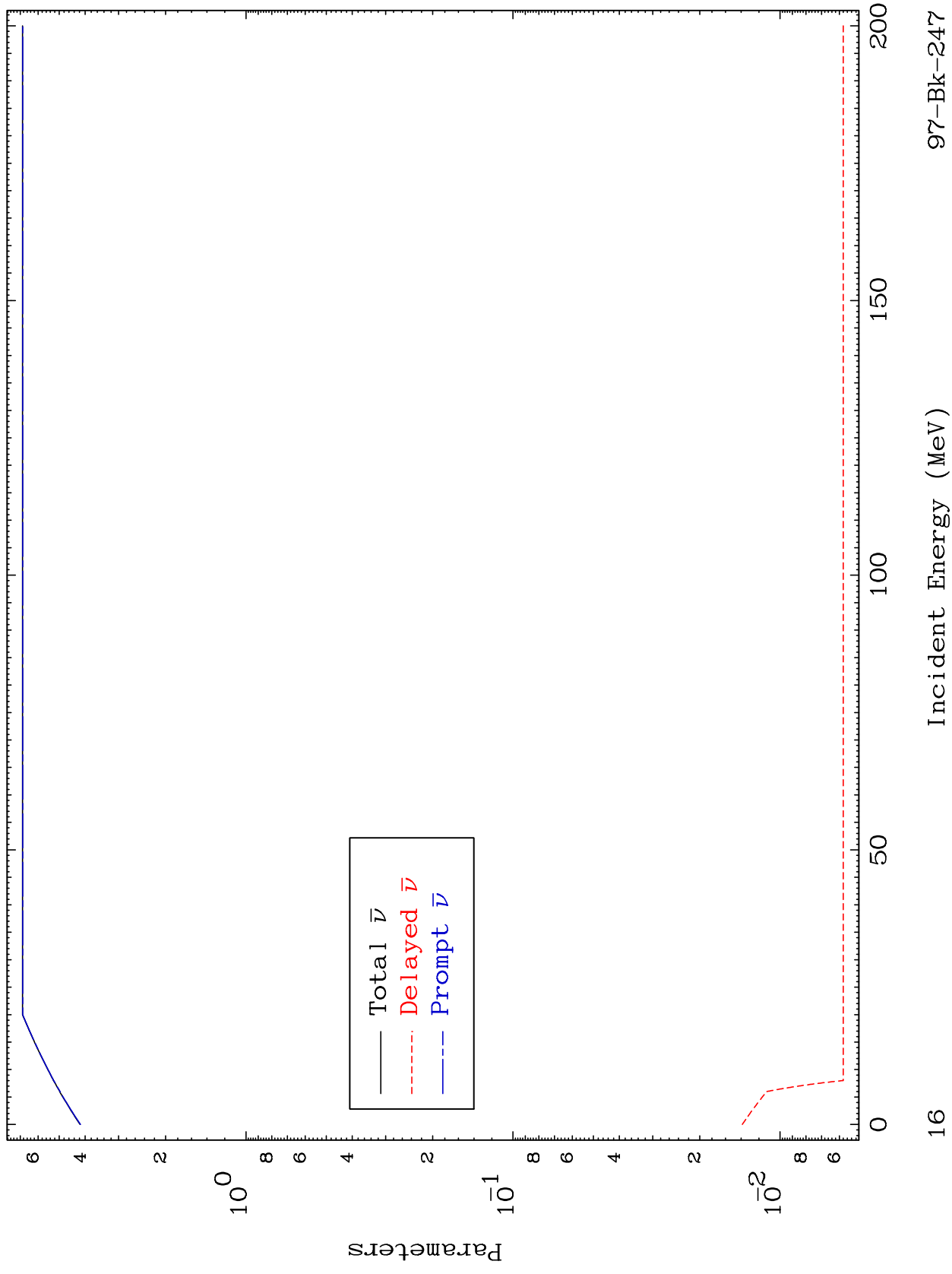
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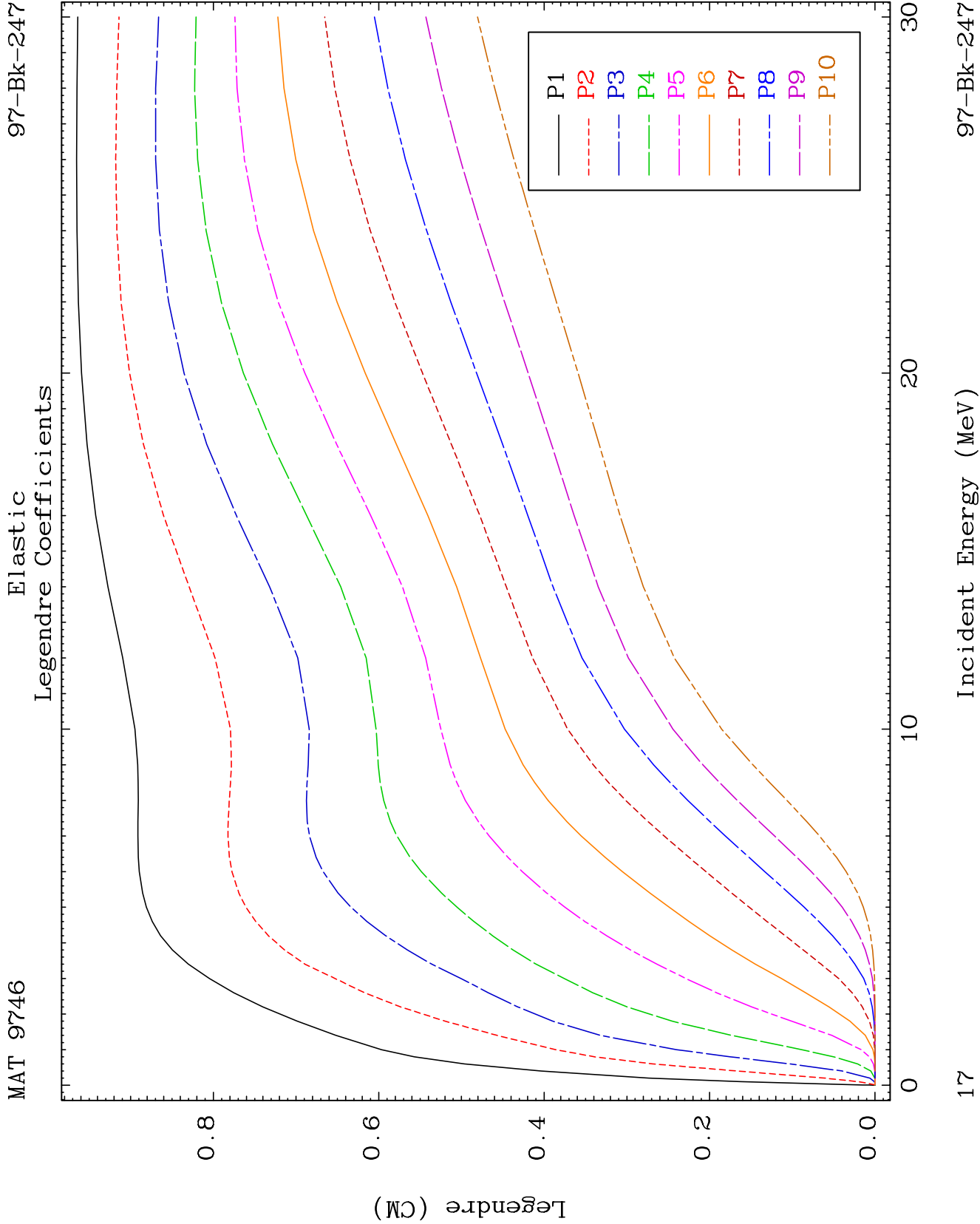


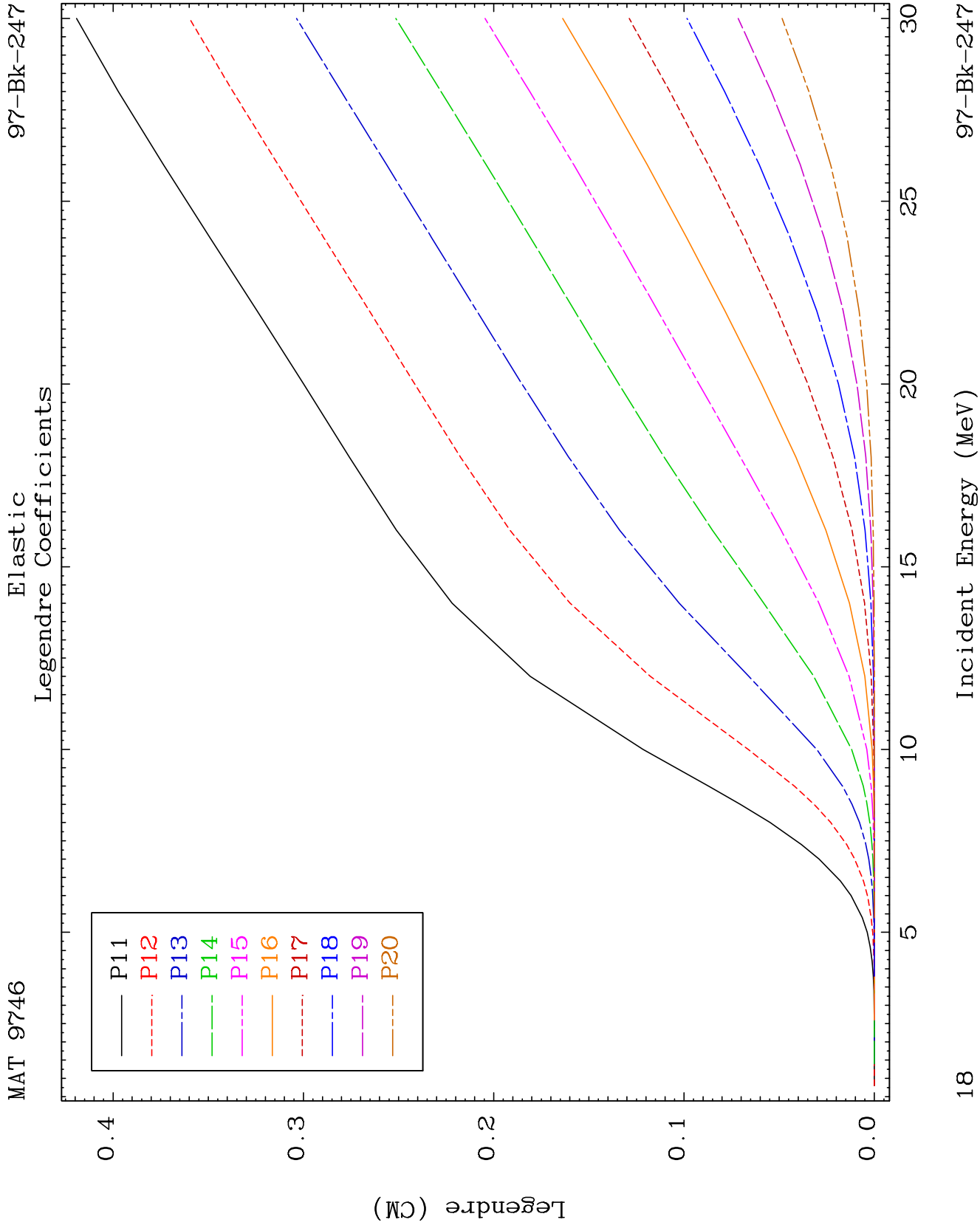
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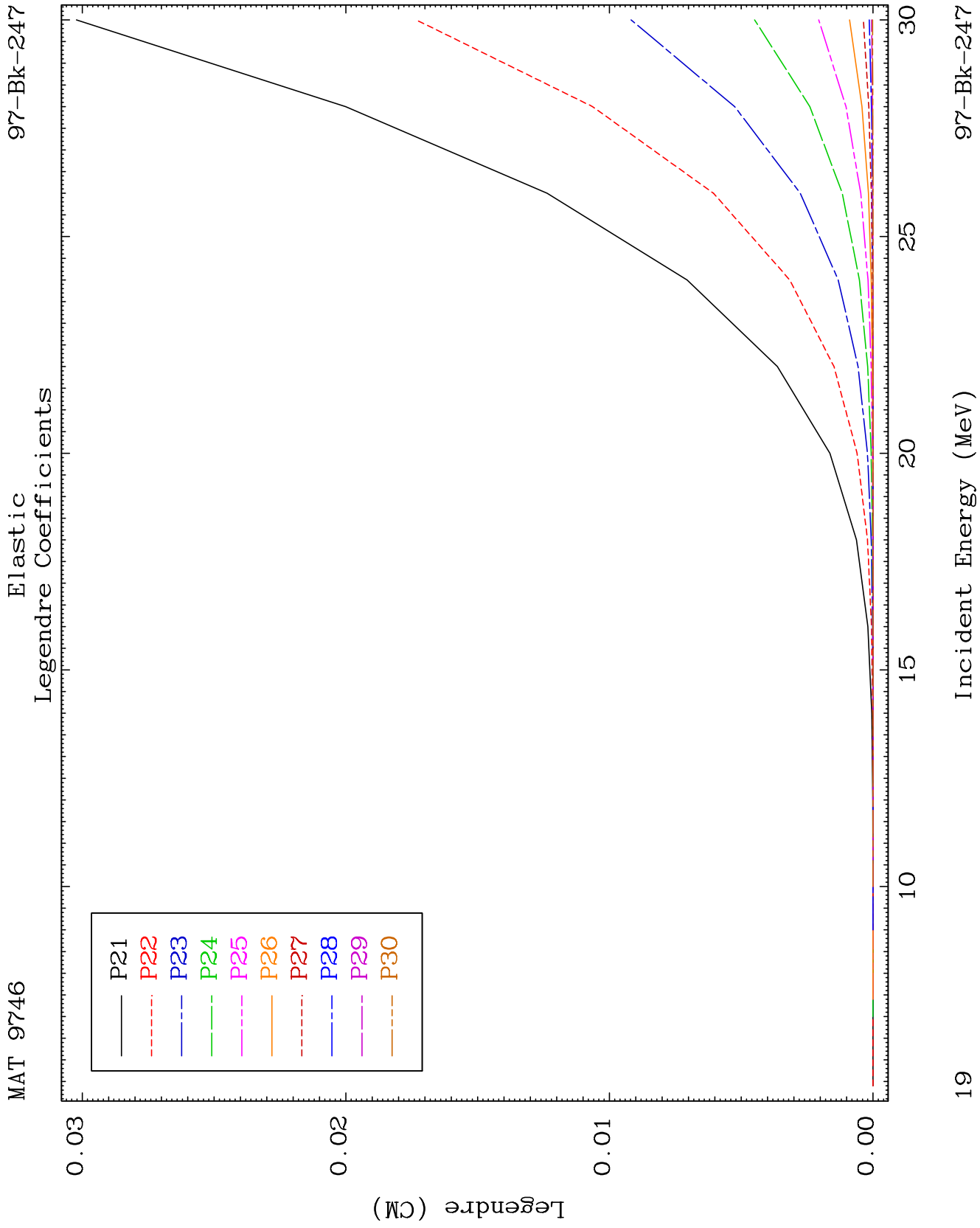
Energy Release Parameters

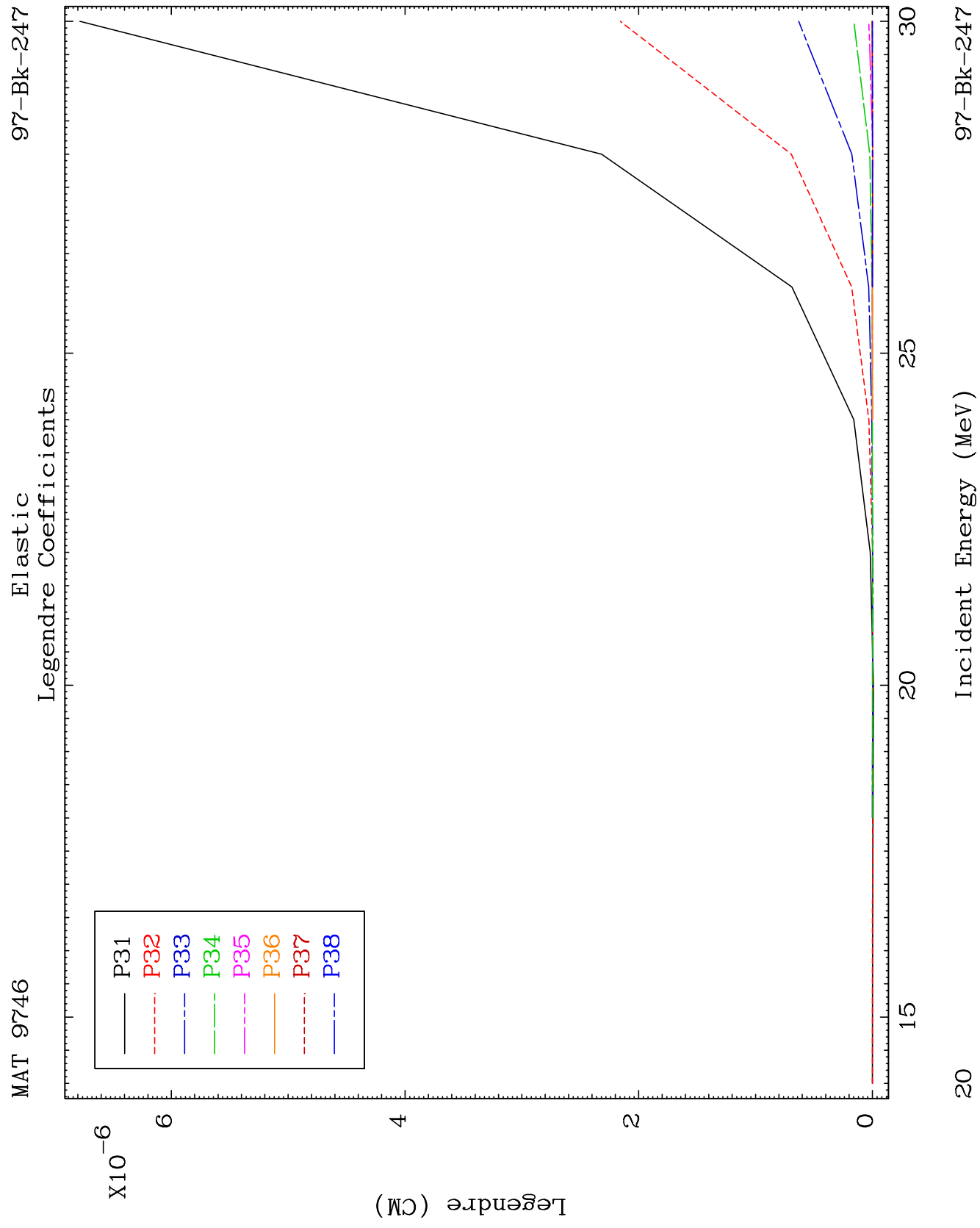
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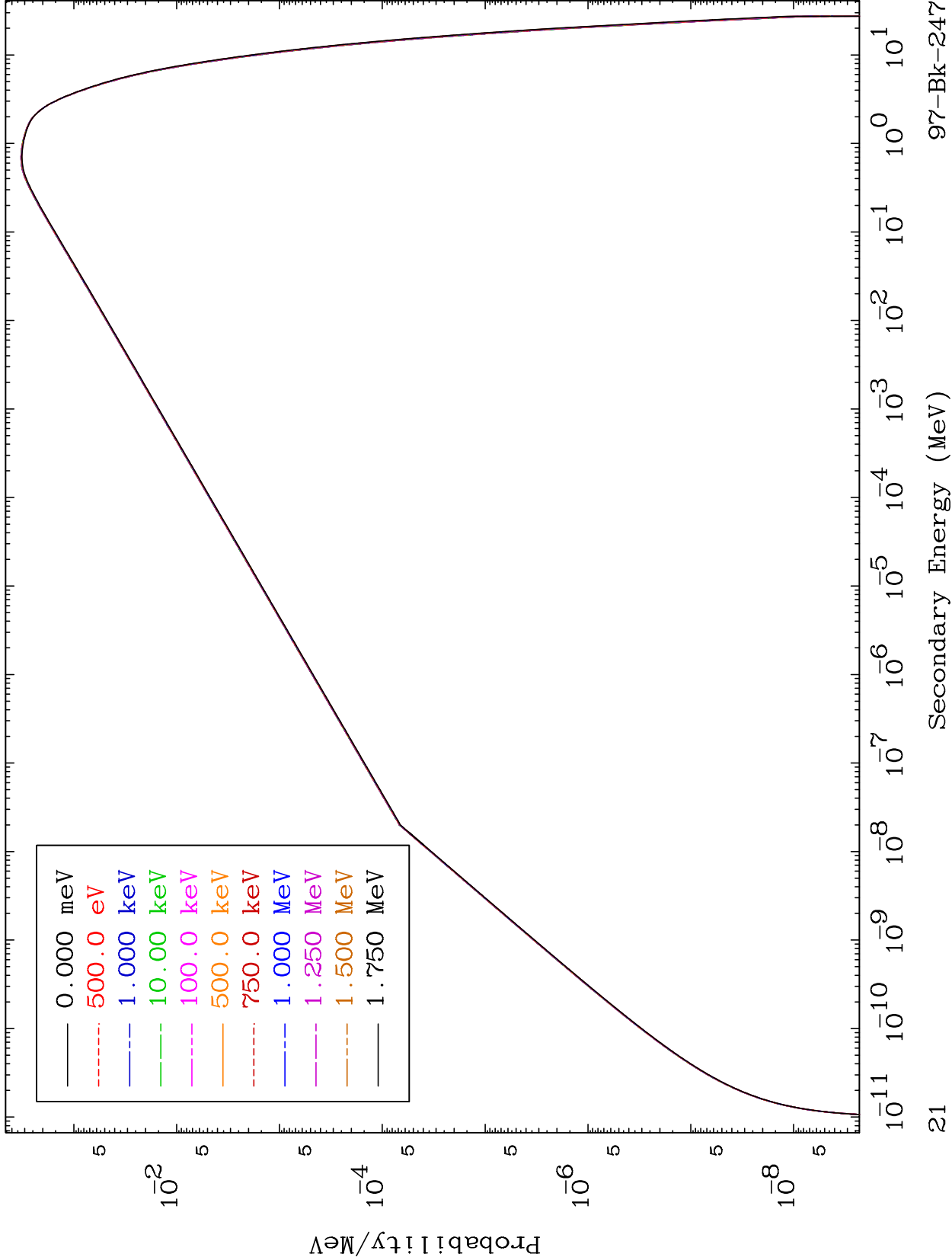








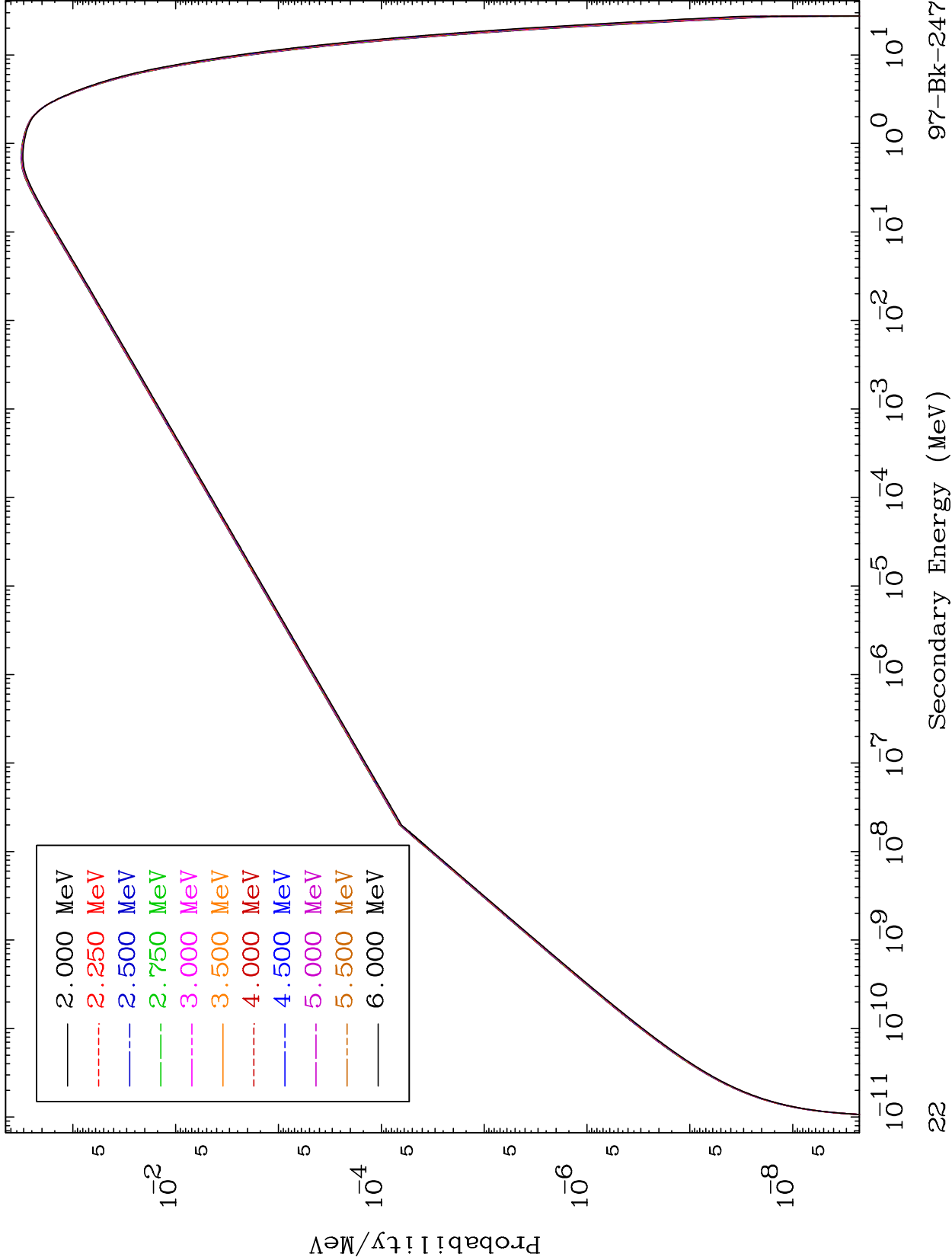




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Fission
Energy Distributions

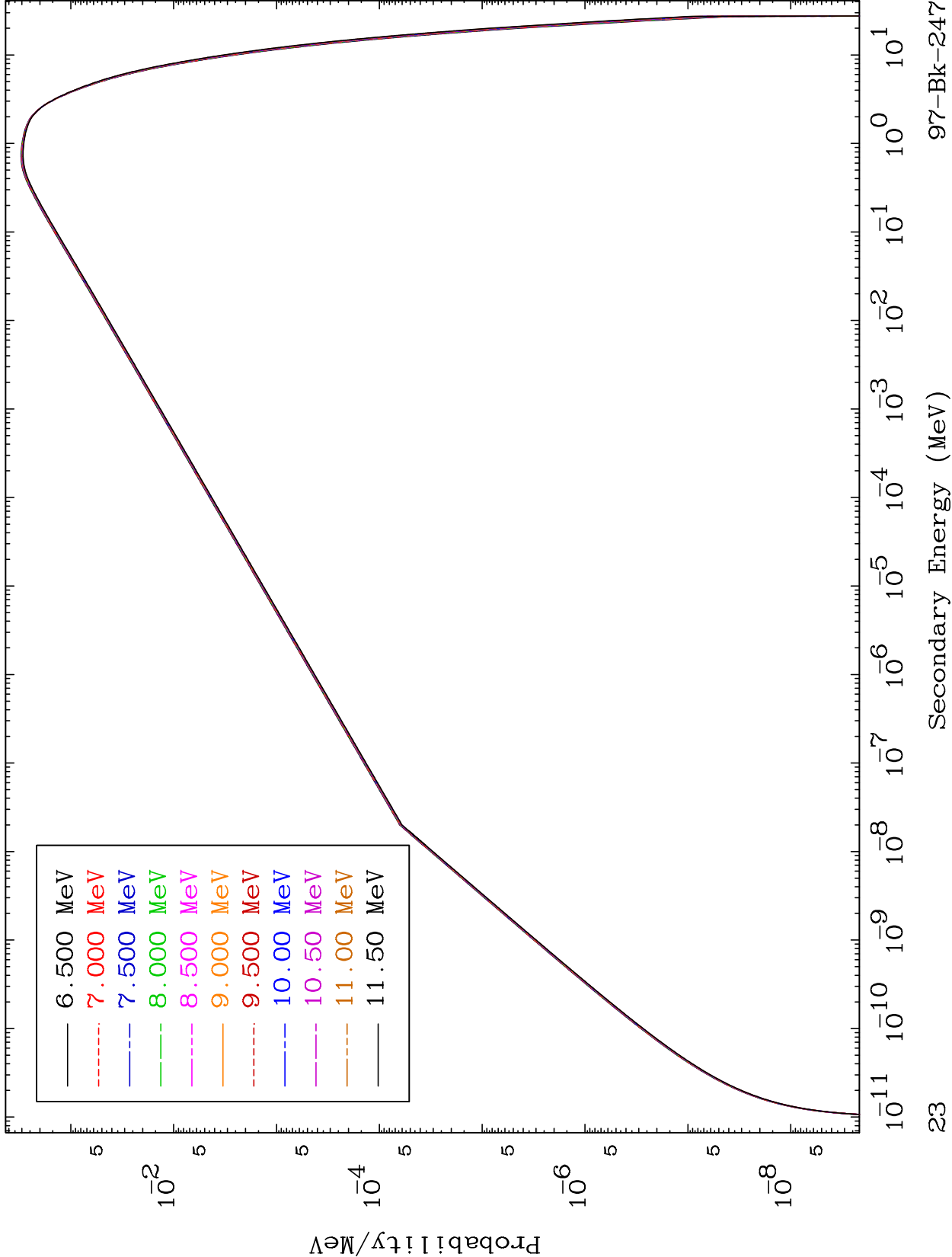
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Fission
Energy Distributions

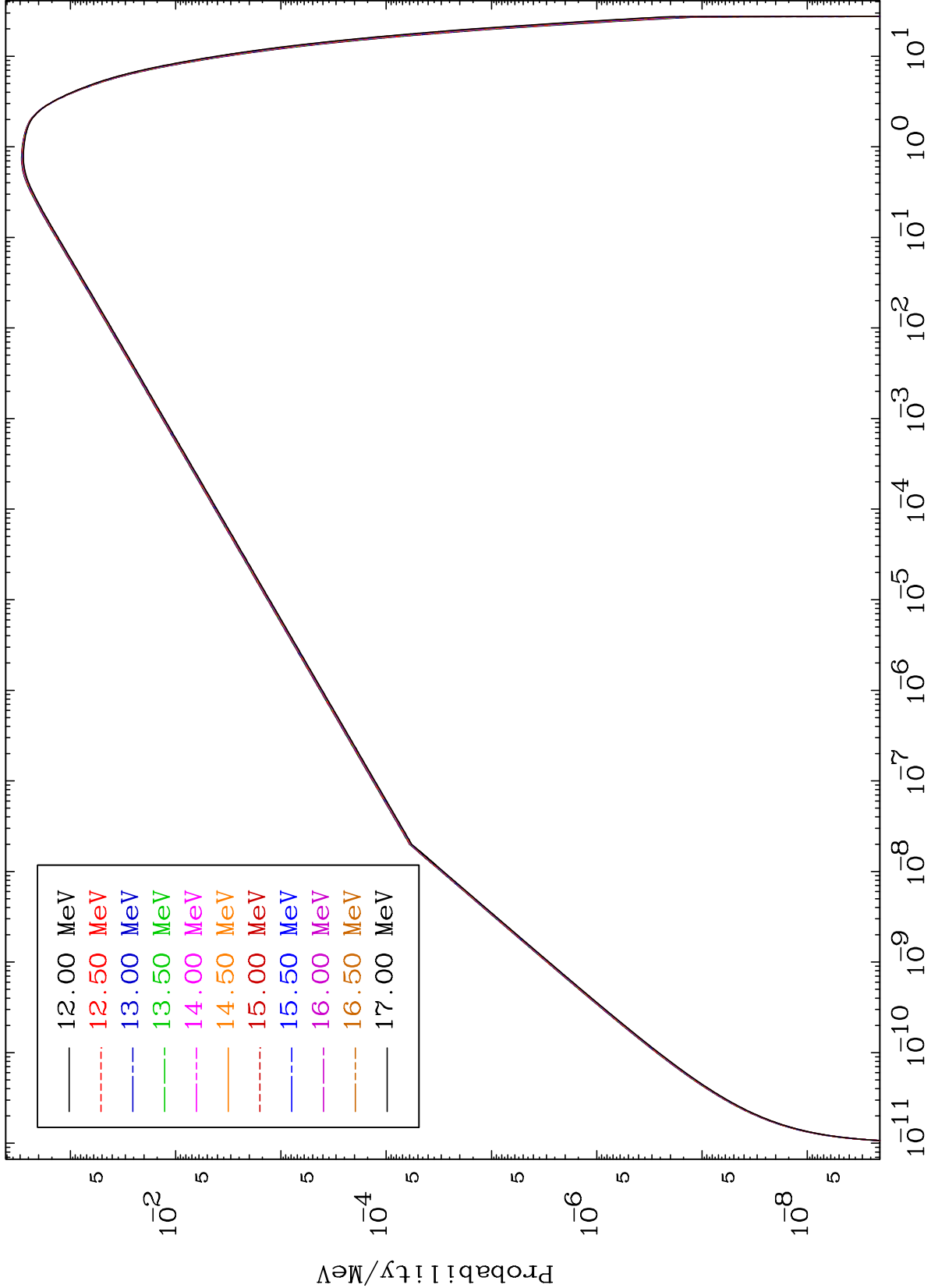
97-Bk-247



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Fission
Energy Distributions

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Secondary Energy (MeV)

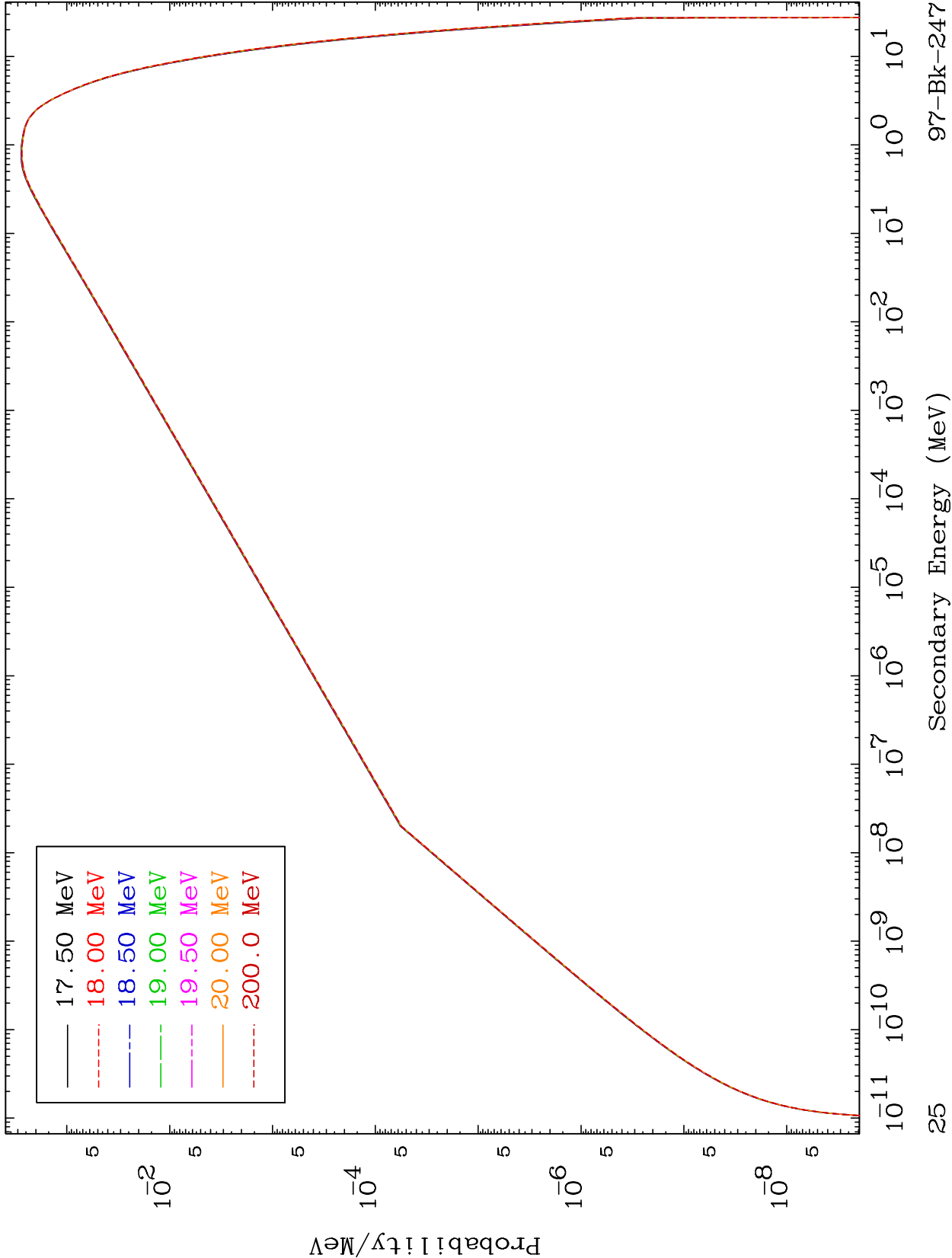
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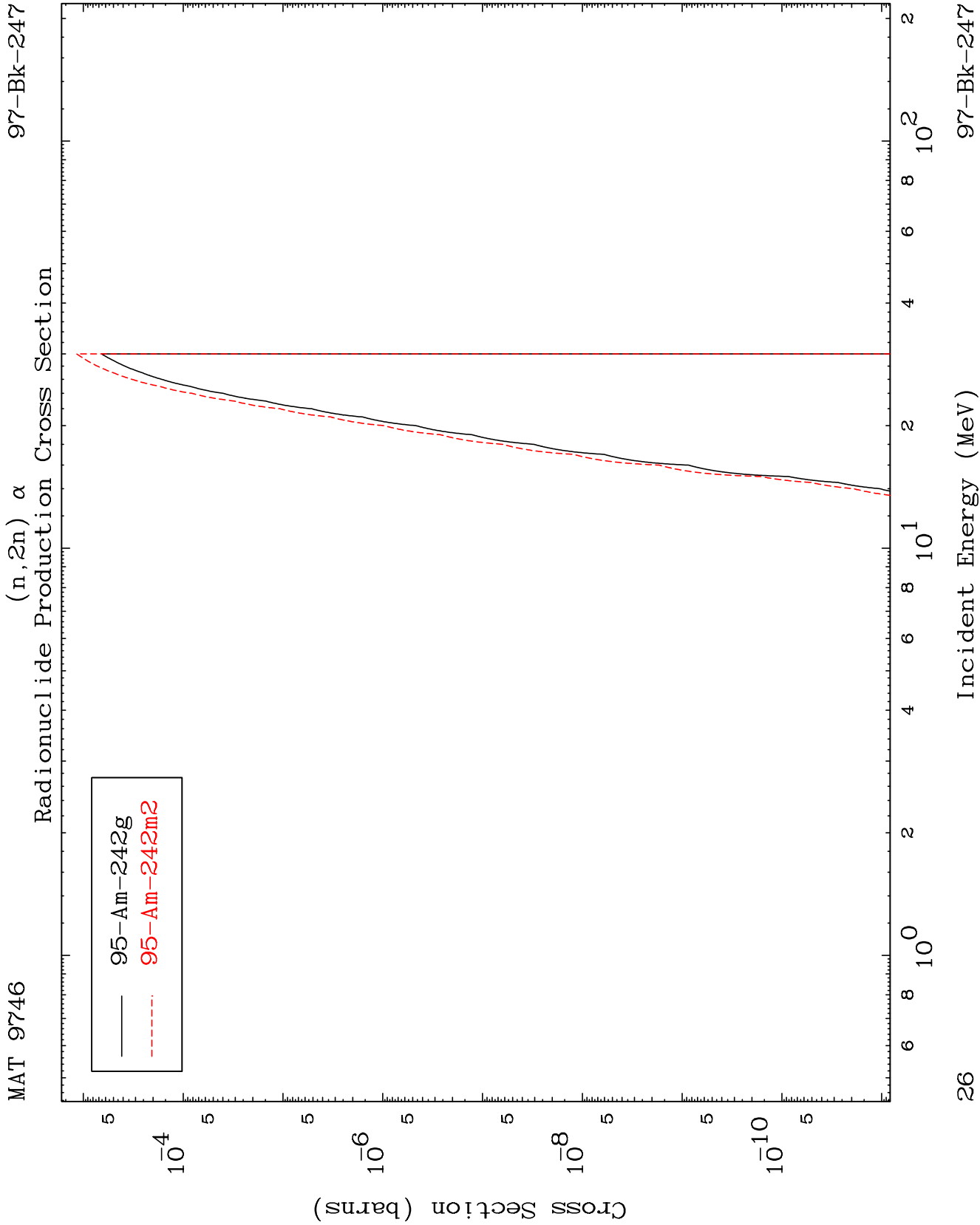
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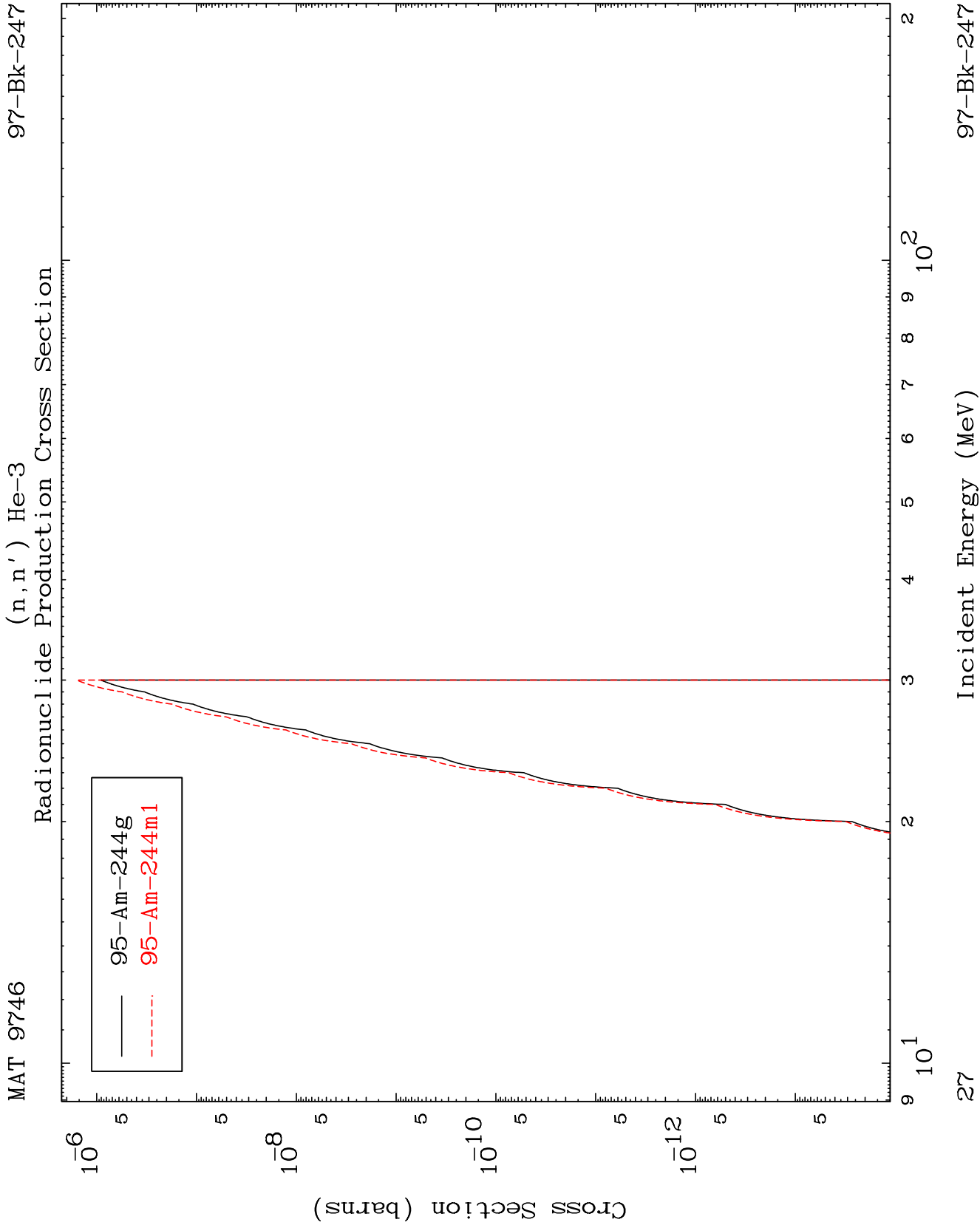
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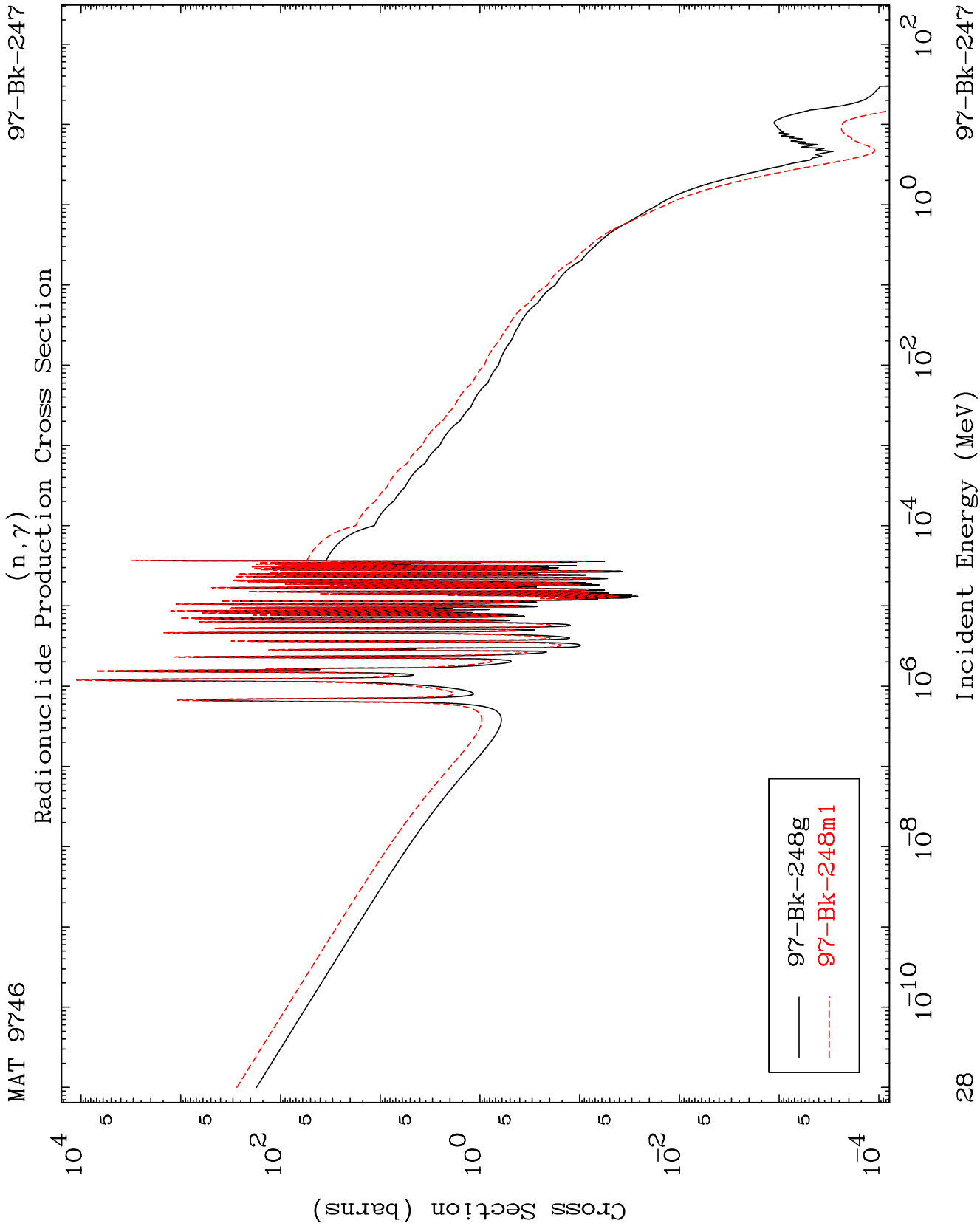
Fission
Energy Distributions

97-Bk-247









Radionuclide Production Cross Section
(n, α)

